

Program EVALPLOT  
(Version 2018-1)

by

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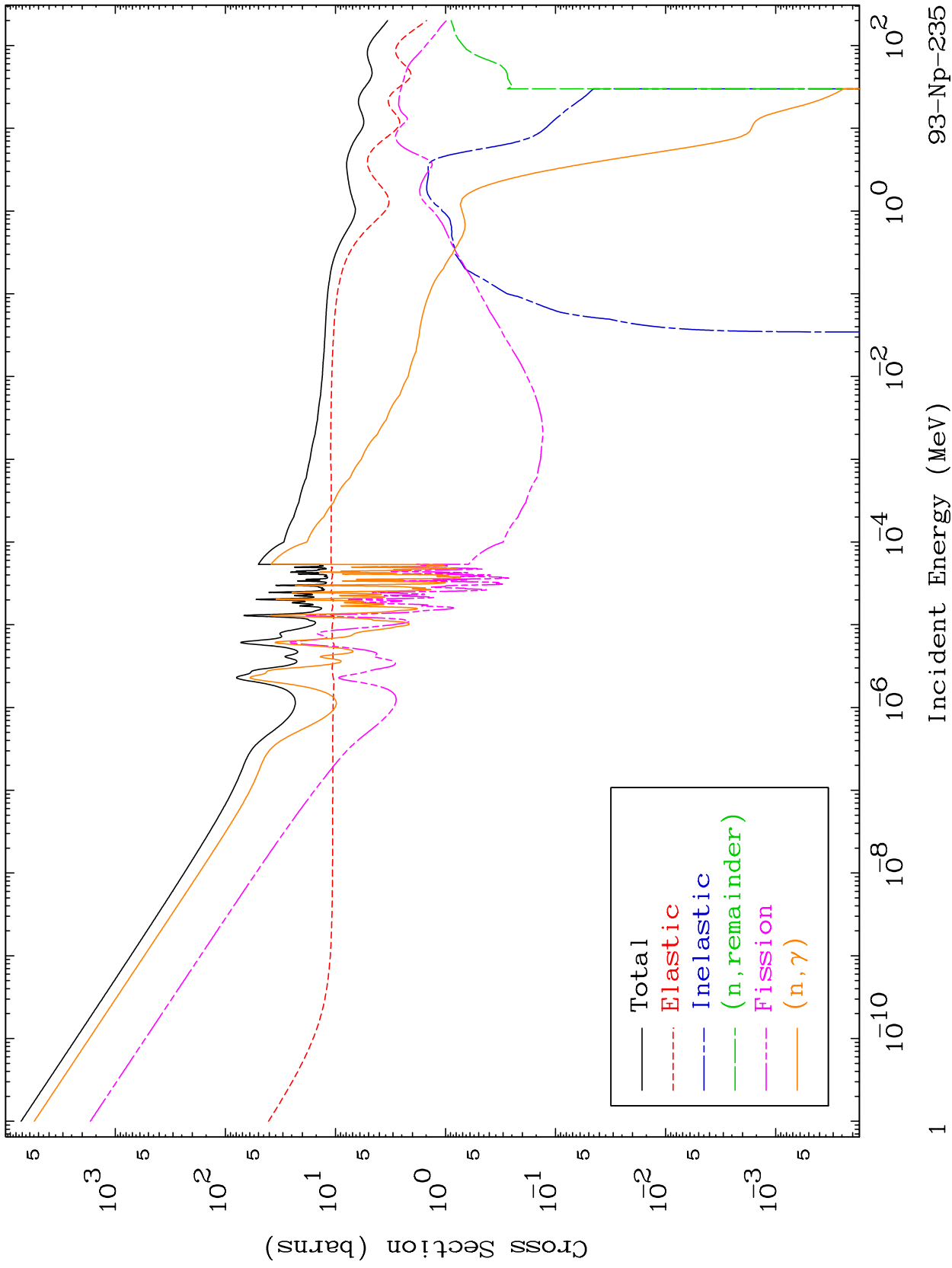
Press Mouse Button to Start

MAT 9340

Major

293 Kelvin Cross Sections

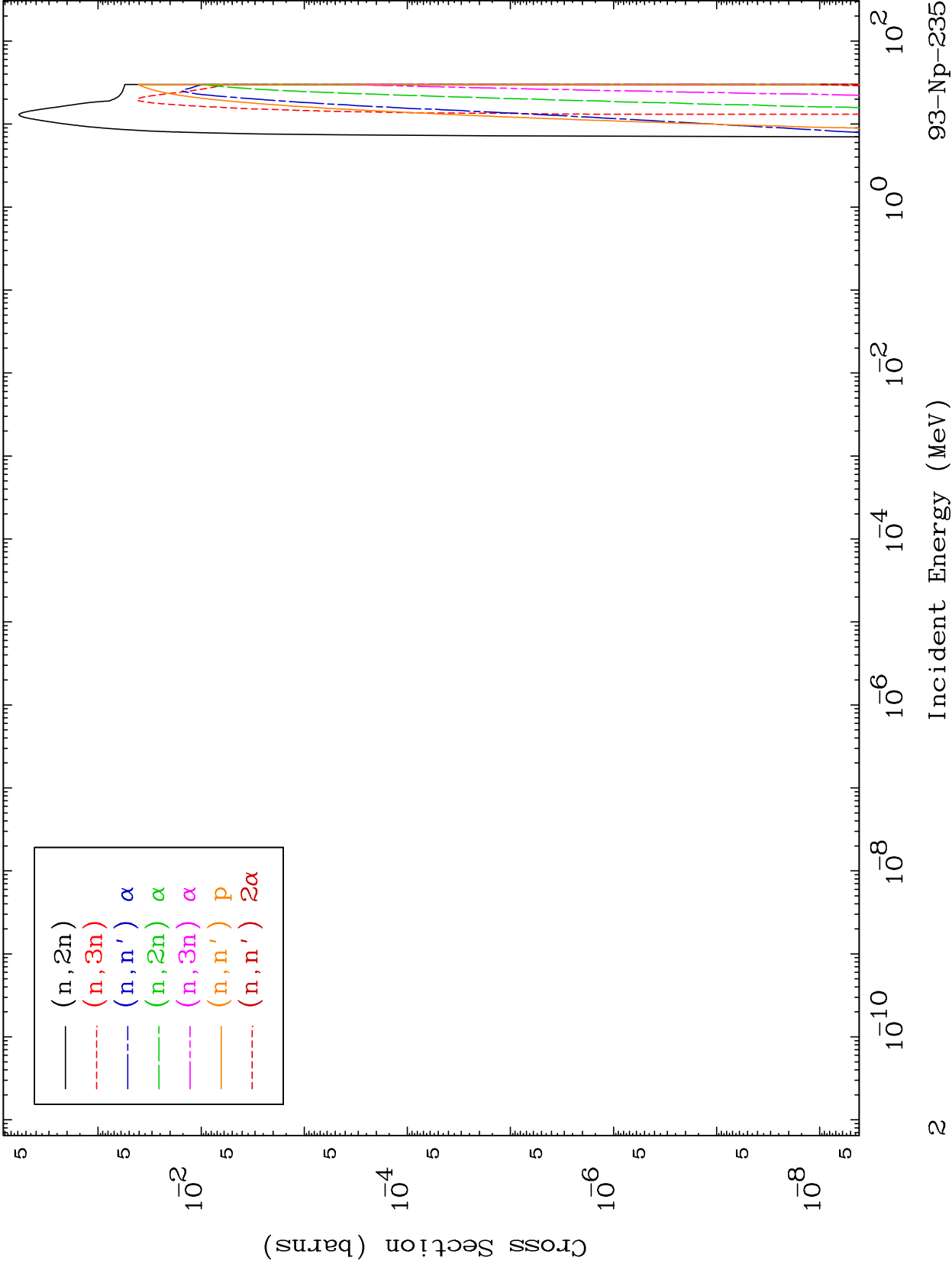
93-Np-235



MAT 9340

Neutron Production  
293 Kelvin Cross Sections

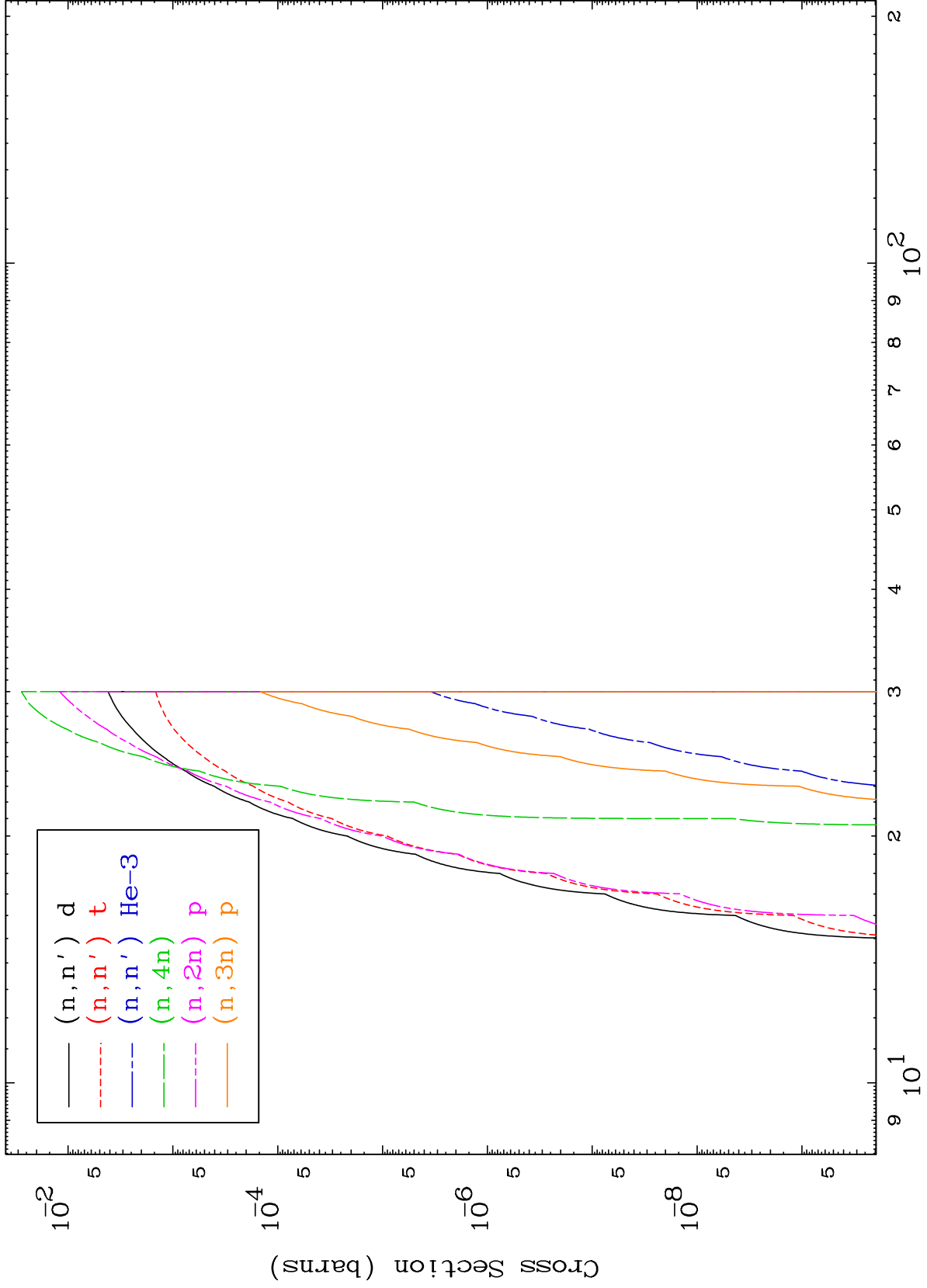
93-Np-235



MAT 9340

Neutron Production  
293 Kelvin Cross Sections

93-Np-235



3

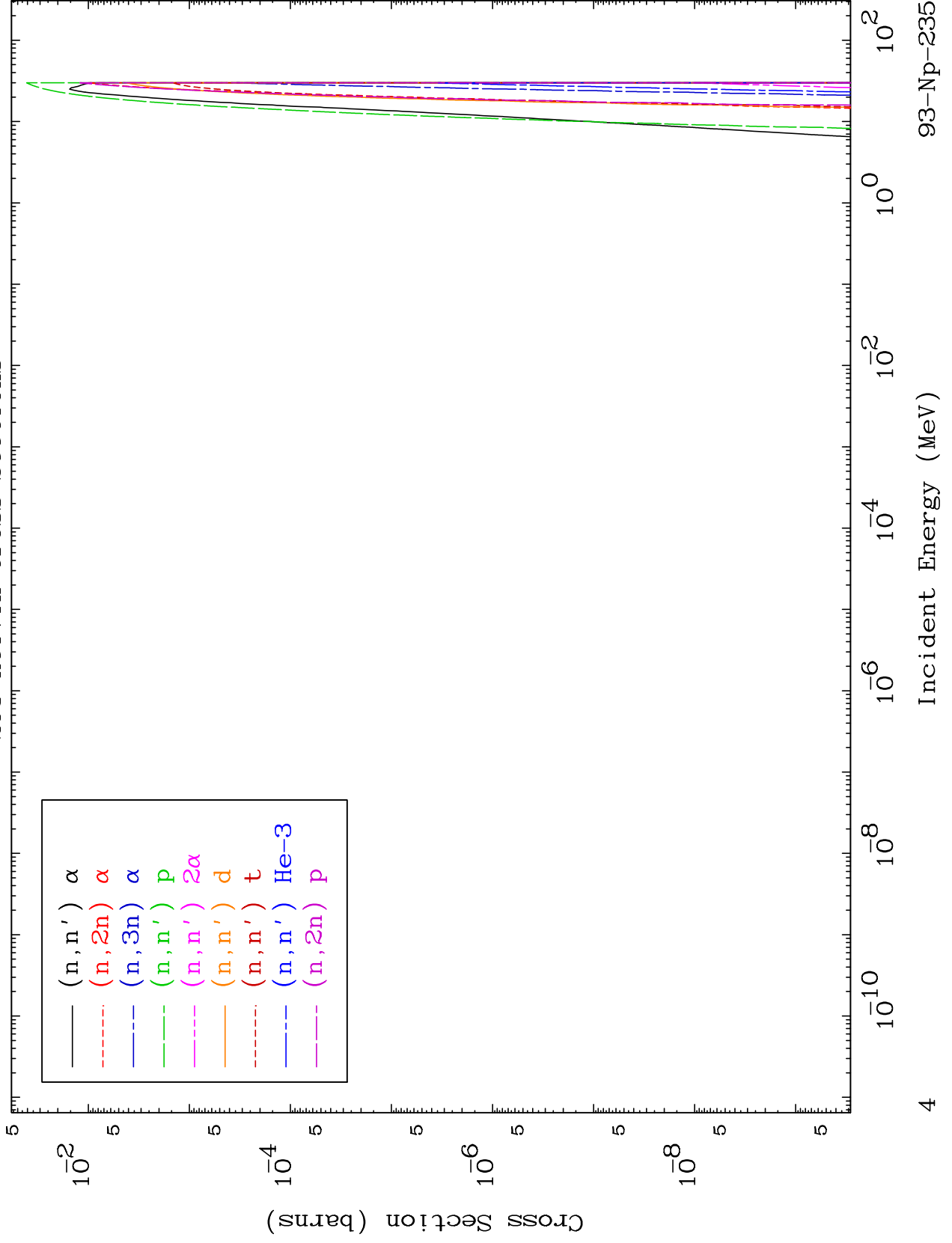
Incident Energy (MeV)

93-Np-235

MAT 9340

Charged Particle  
293 Kelvin Cross Sections

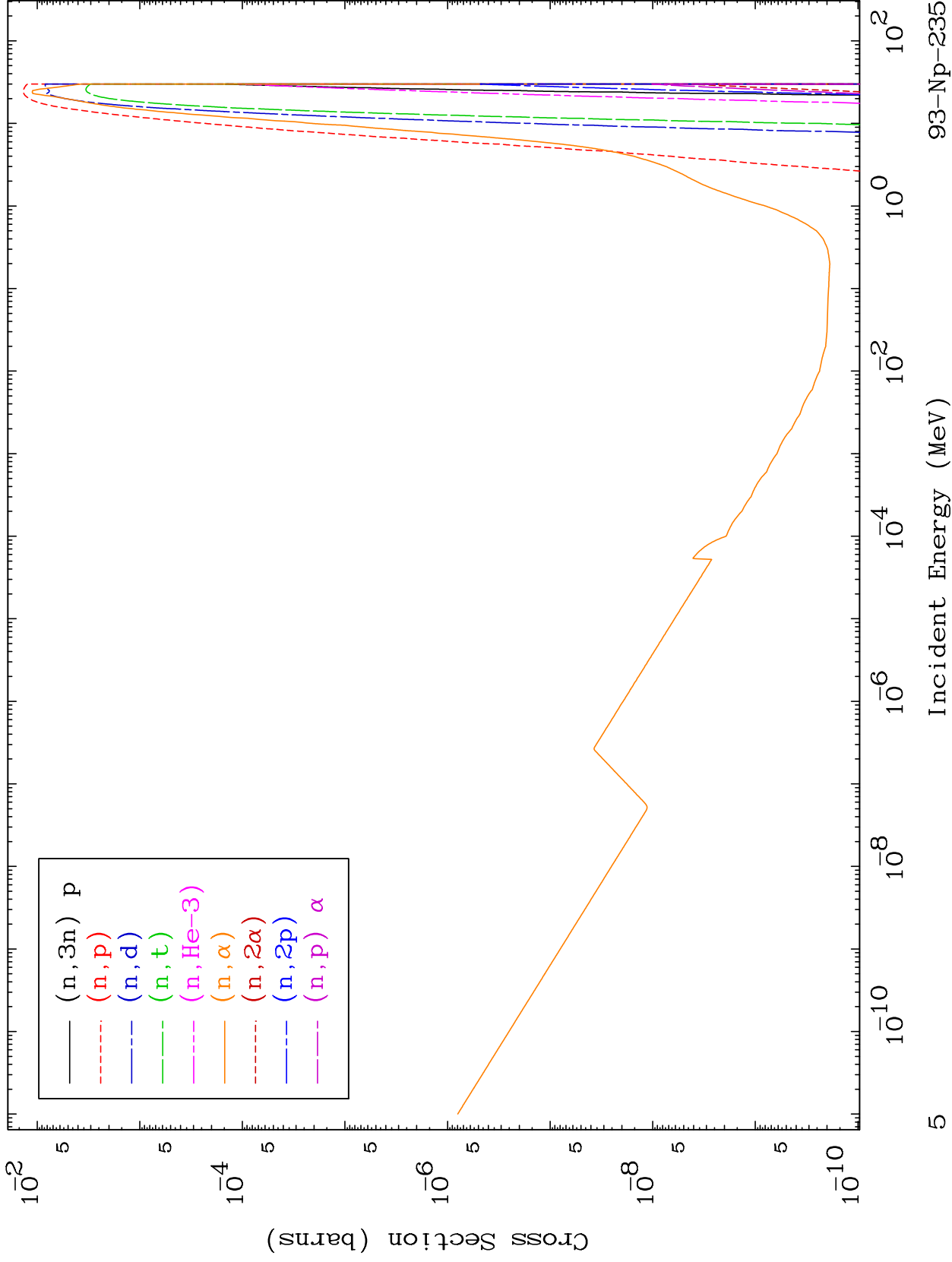
93-Np-235



MAT 9340

Charged Particle  
293 Kelvin Cross Sections

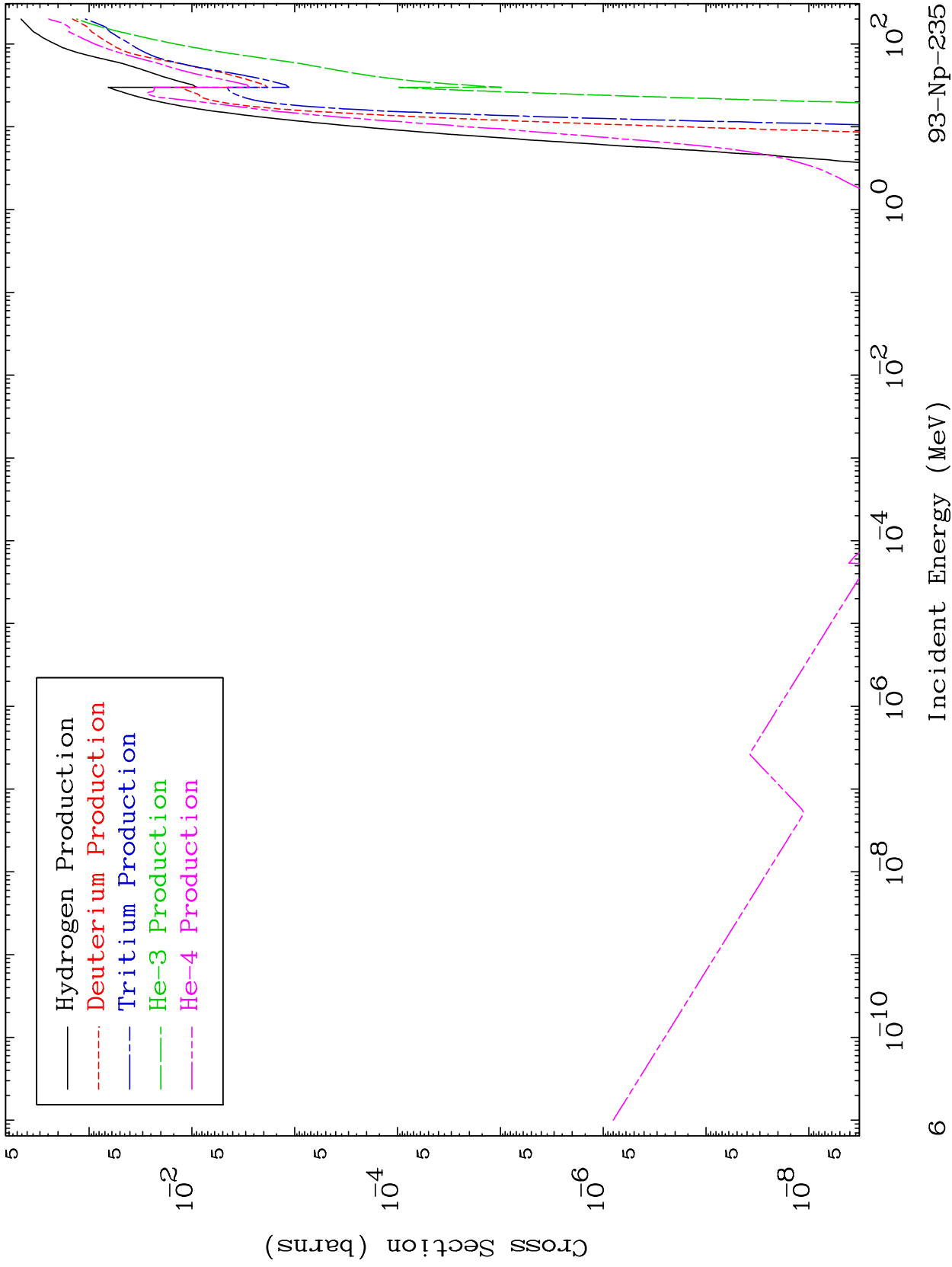
93-Np-235



MAT 9340

Particle Production  
293 Kelvin Cross Sections

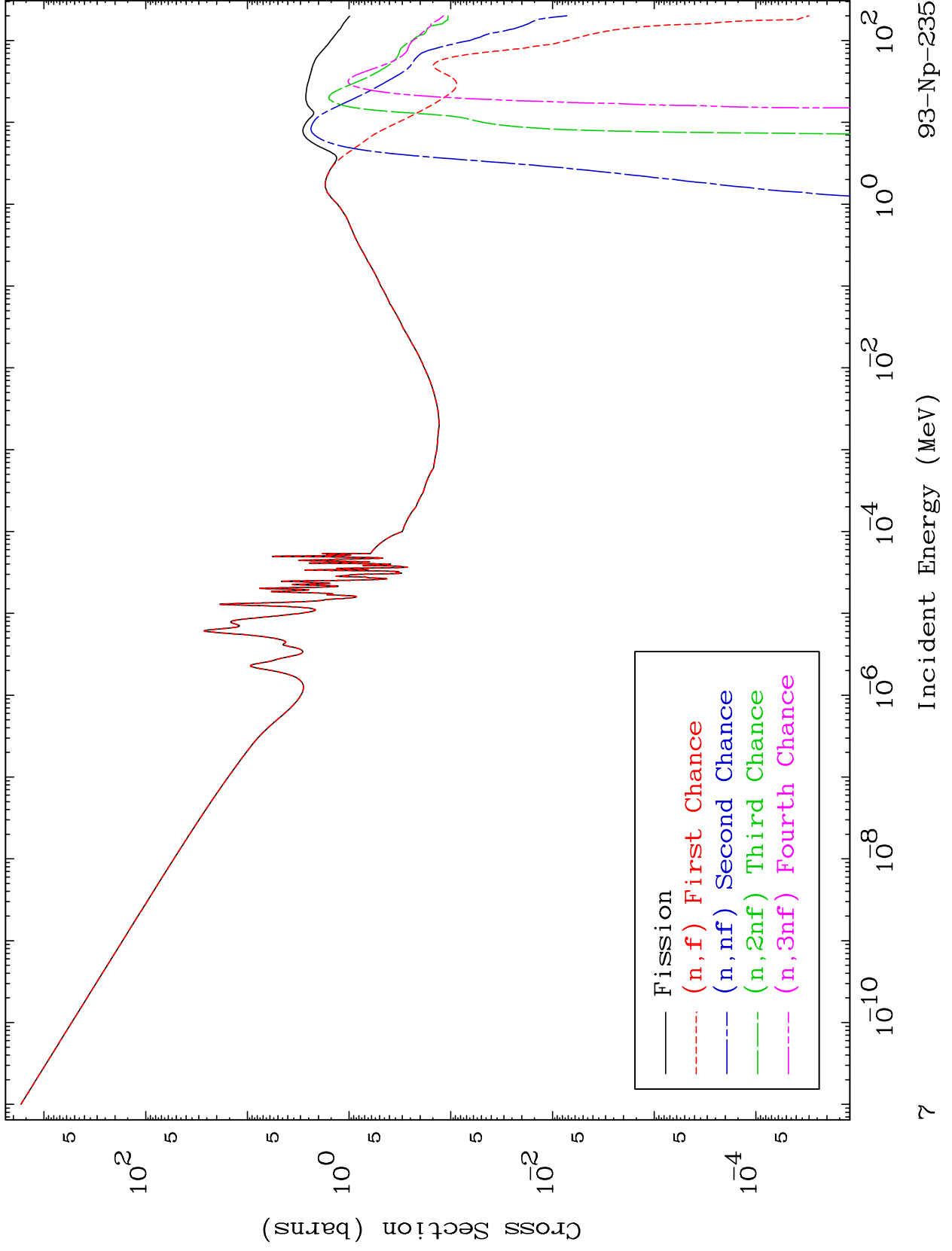
93-Np-235



MAT 9340

Fission  
293 Kelvin Cross Sections

93-Np-235



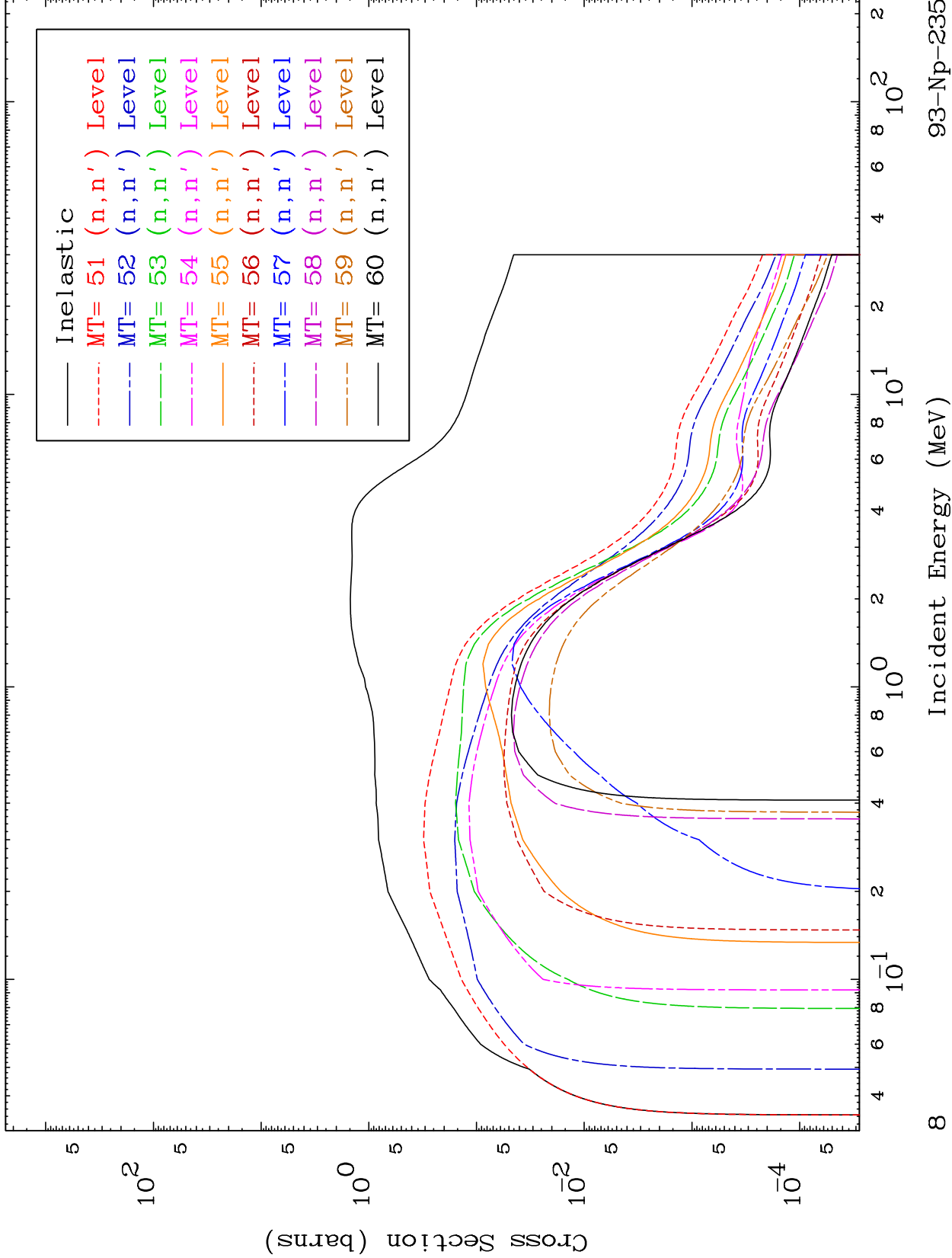


MAT 9340

(n,n') Level

93-Np-235

293 Kelvin Cross Sections

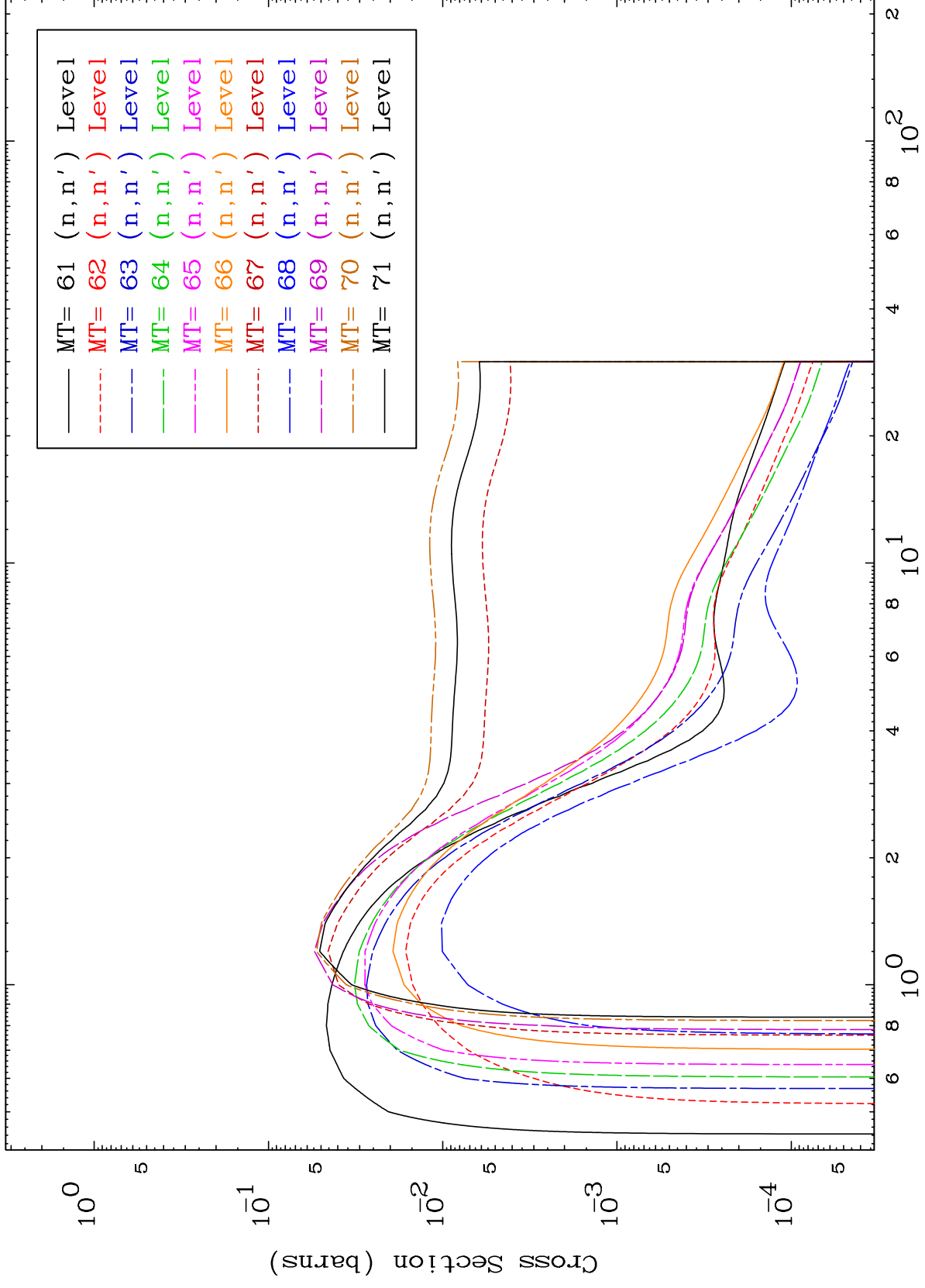


MAT 9340

(n,n') Level

93-Np-235

293 Kelvin Cross Sections

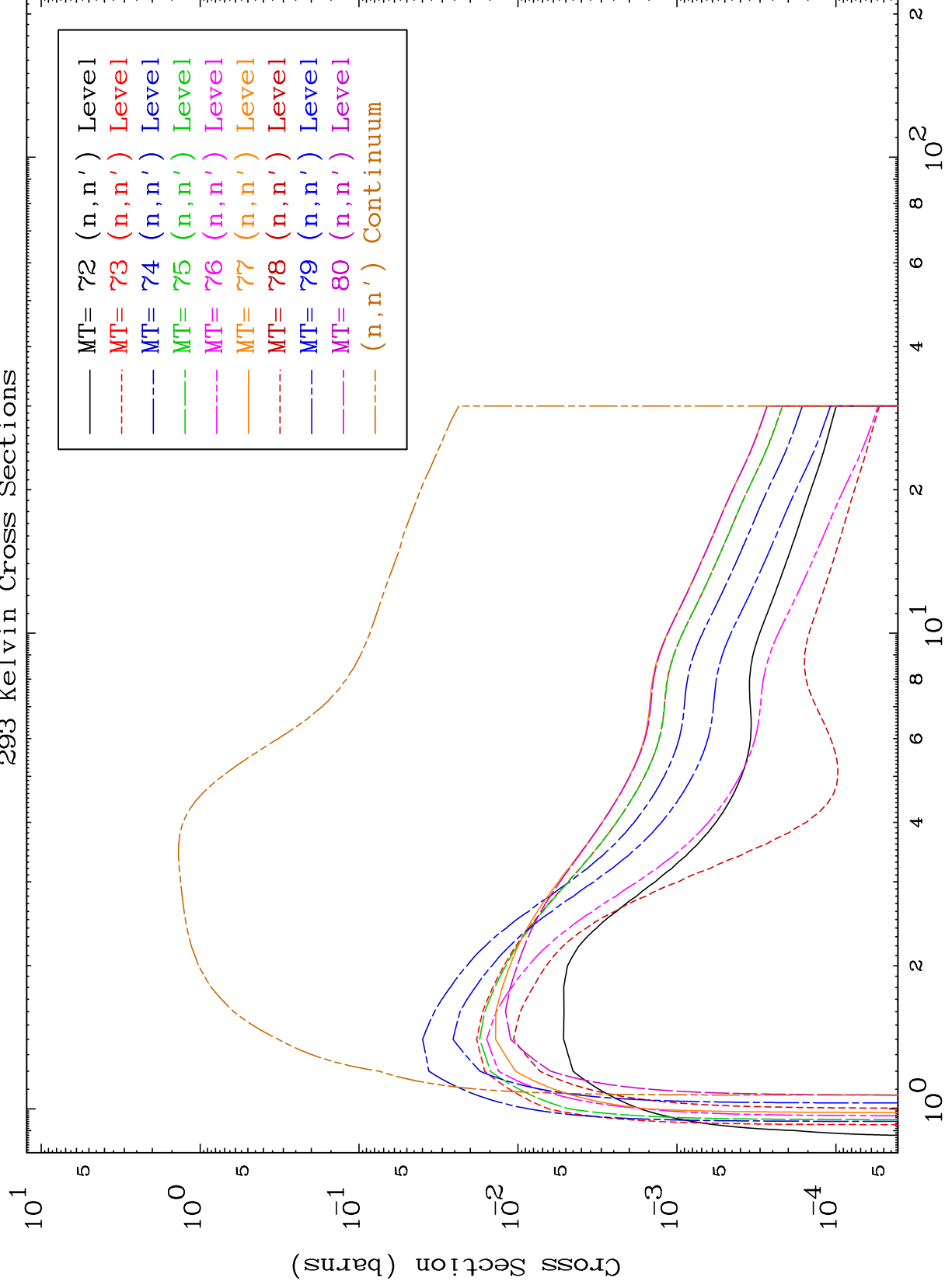


MAT 9340

(n,n') Level

93-Np-235

293 Kelvin Cross Sections



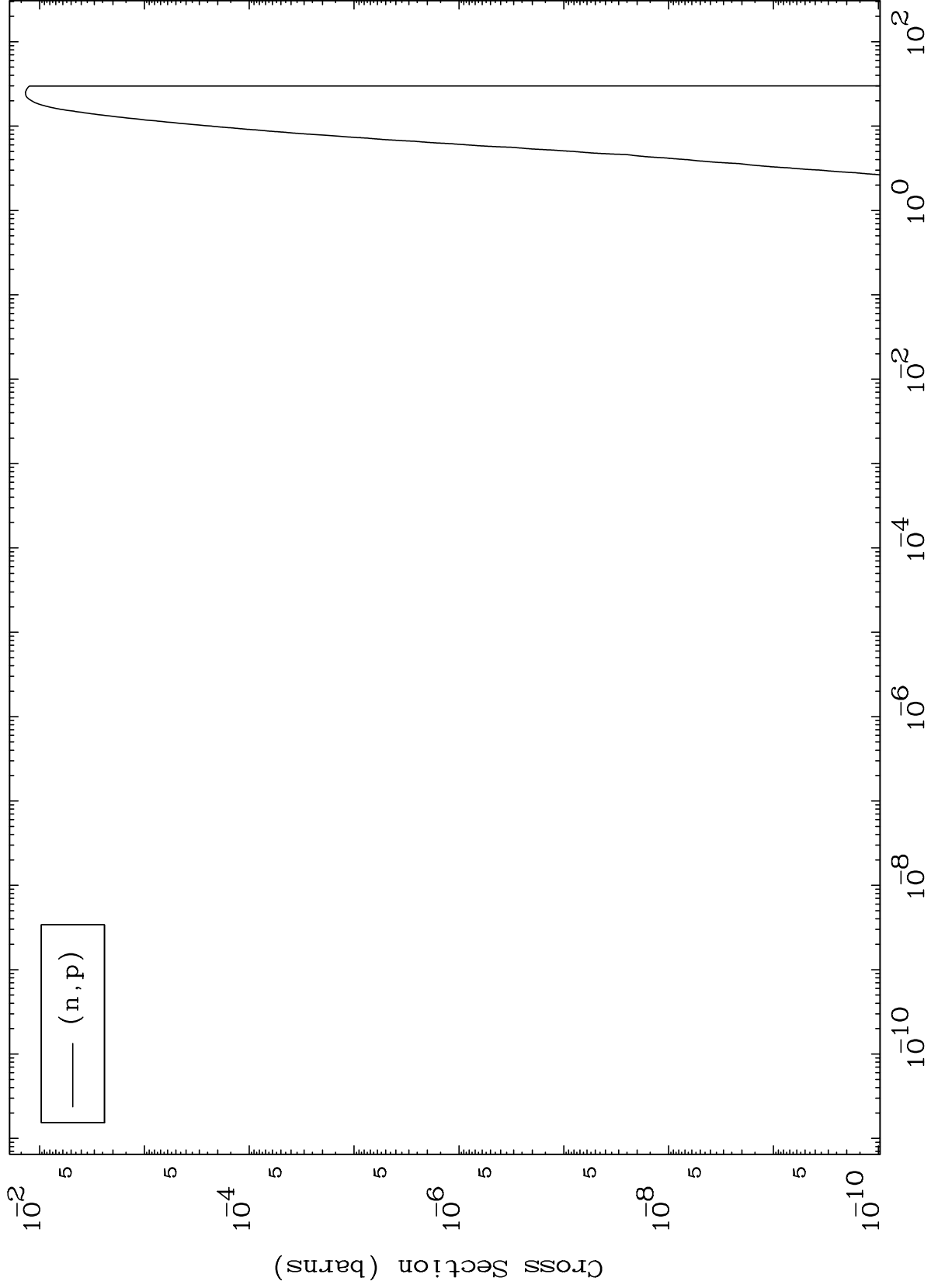
Incident Energy (MeV)

93-Np-235

MAT 9340

(n,p) Levels  
293 Kelvin Cross Sections

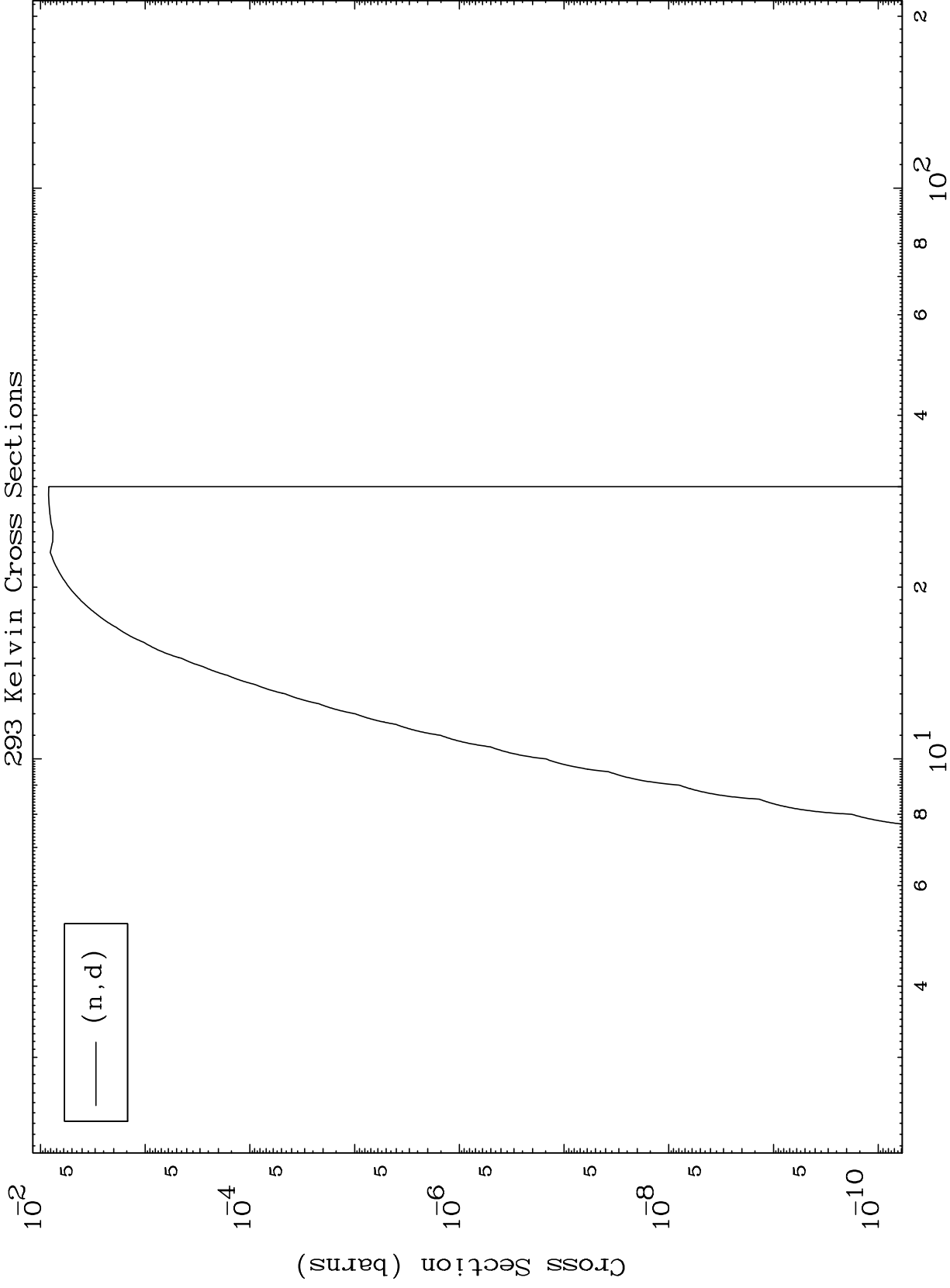
93-Np-235



MAT 9340

(n,d) Levels  
293 Kelvin Cross Sections

93-Np-235



12

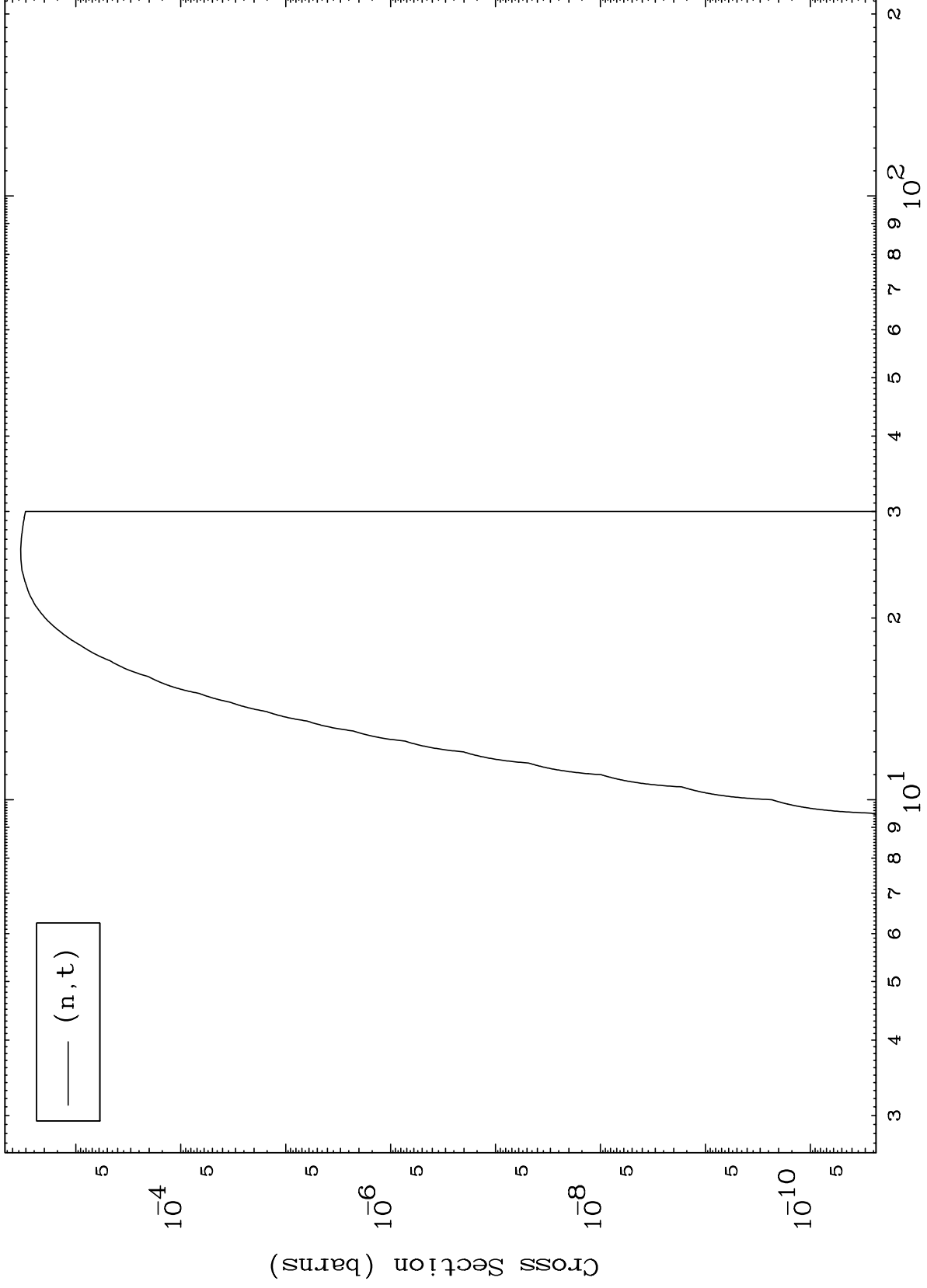
Incident Energy (MeV)

93-Np-235

MAT 9340

(n,t) Levels  
293 Kelvin Cross Sections

93-Np-235



13

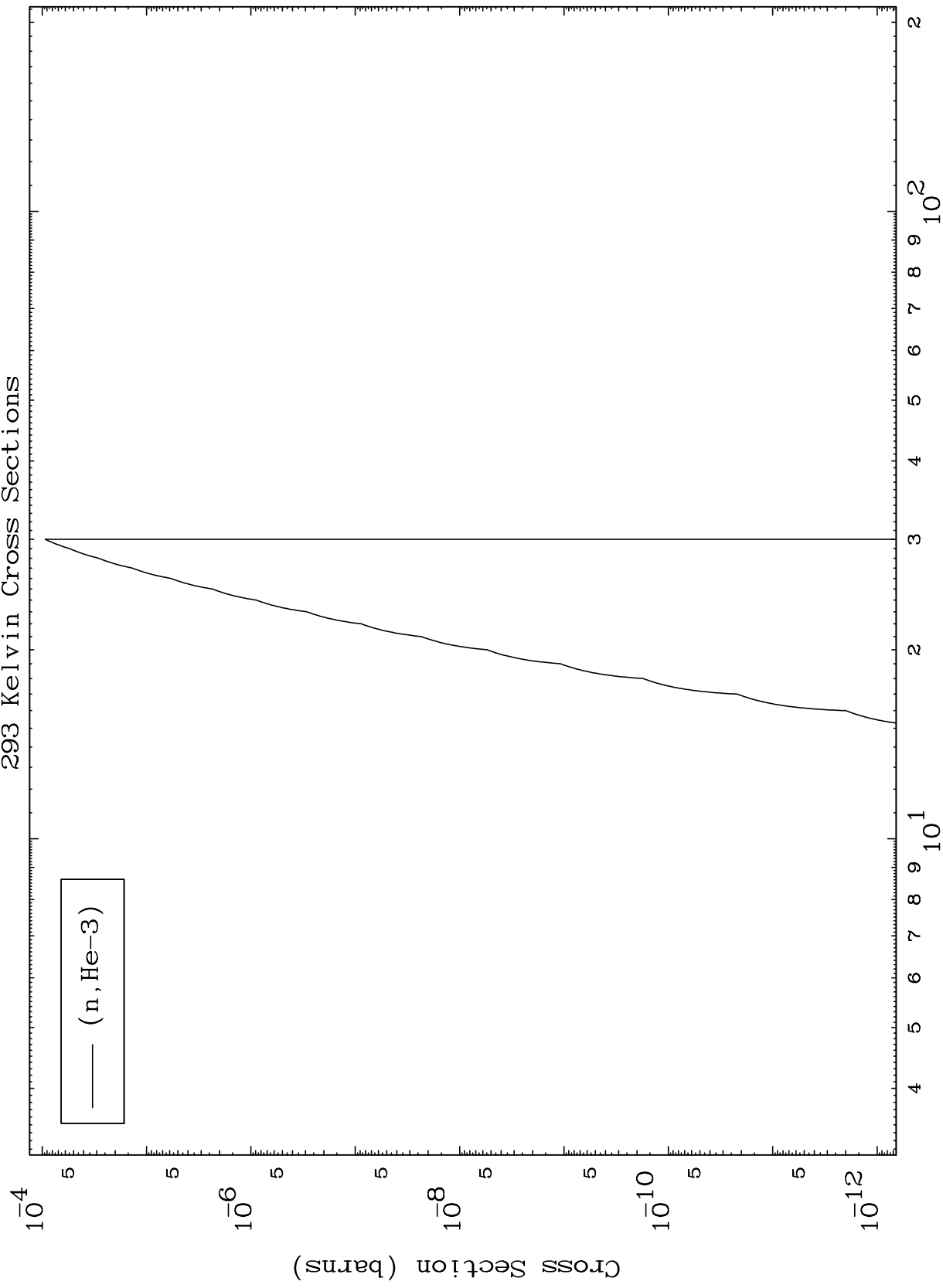
Incident Energy (MeV)

93-Np-235

MAT 9340

93-Np-235

(n,He3) Levels  
293 Kelvin Cross Sections



14

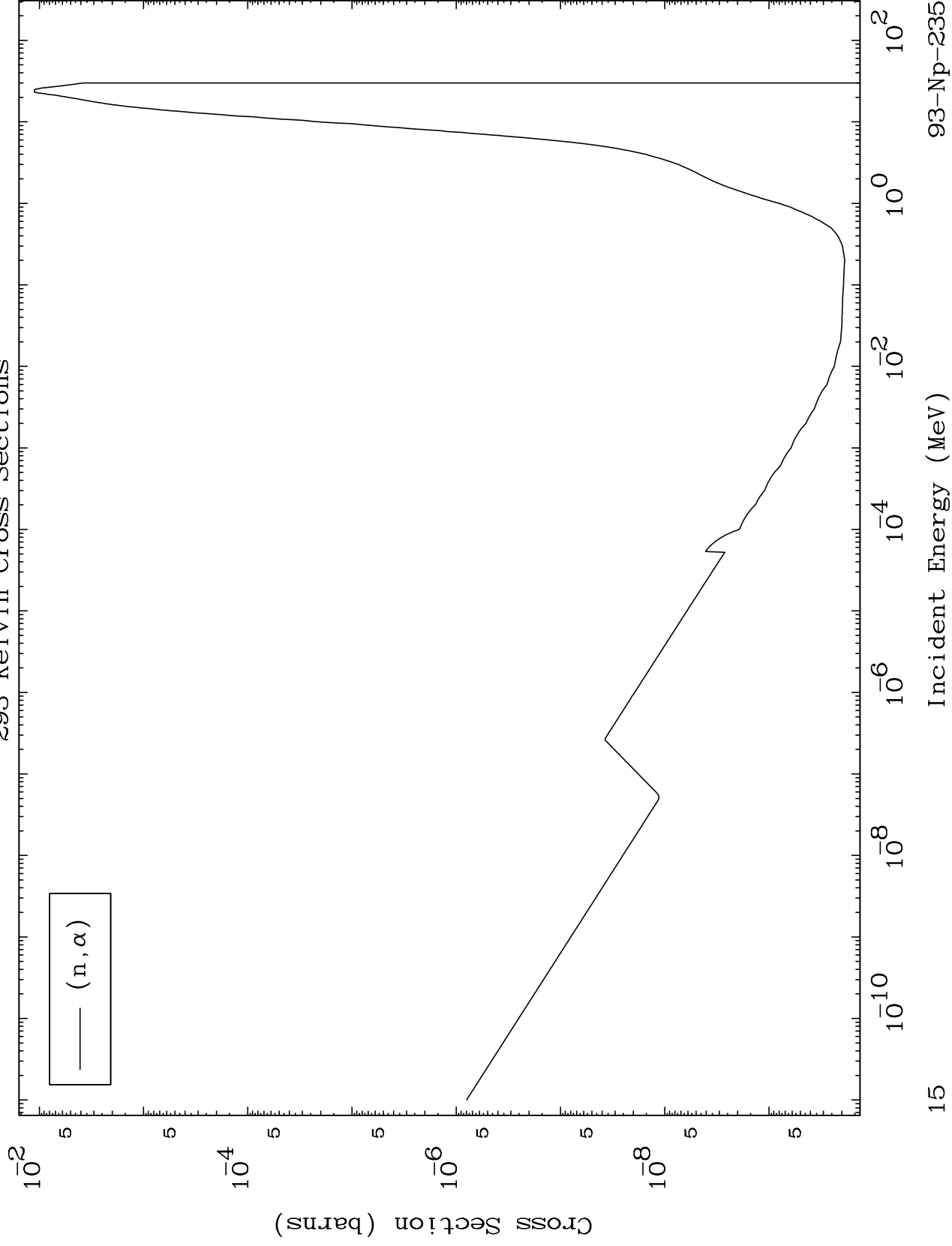
Incident Energy (MeV)

93-Np-235

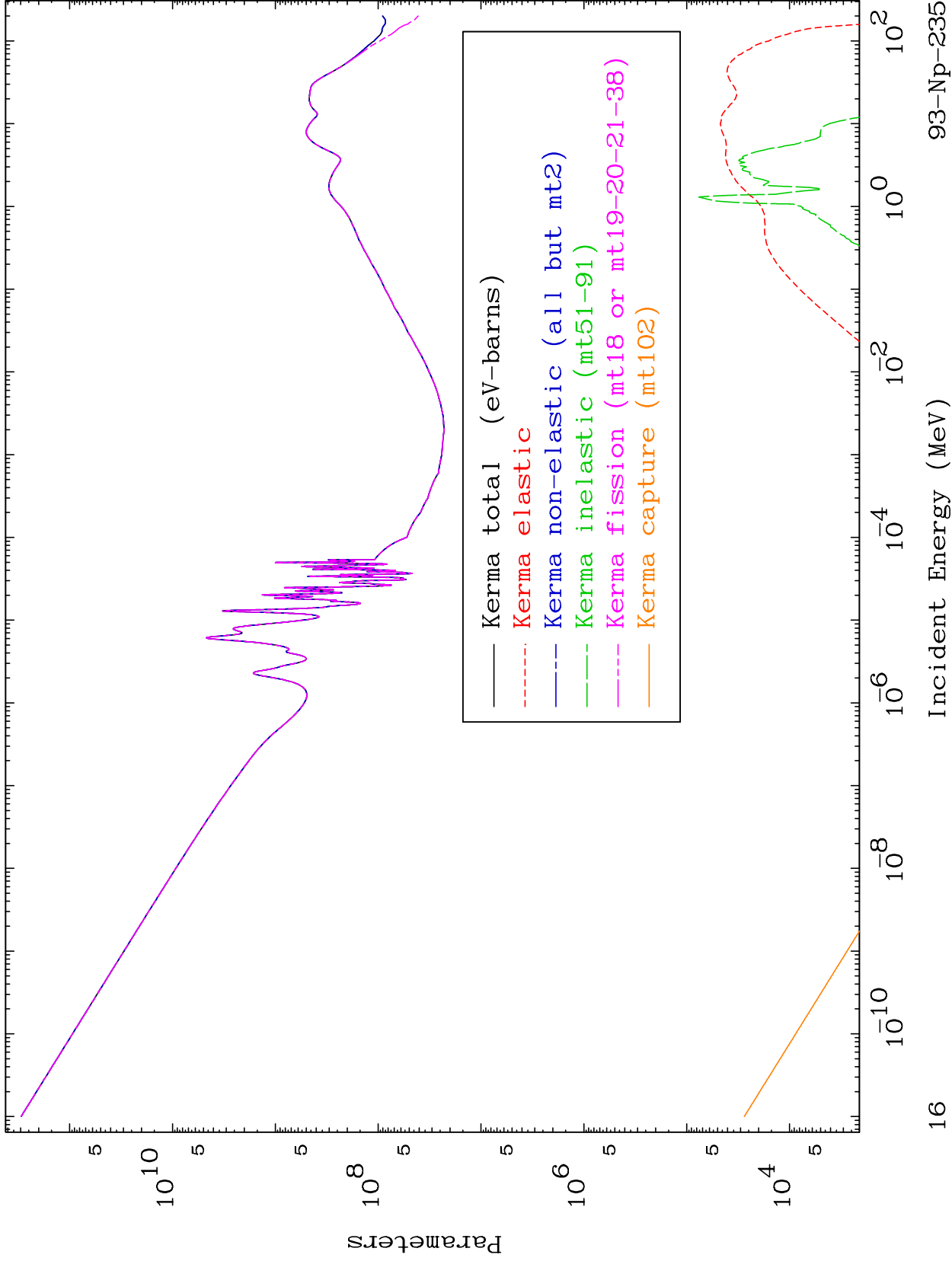
MAT 9340

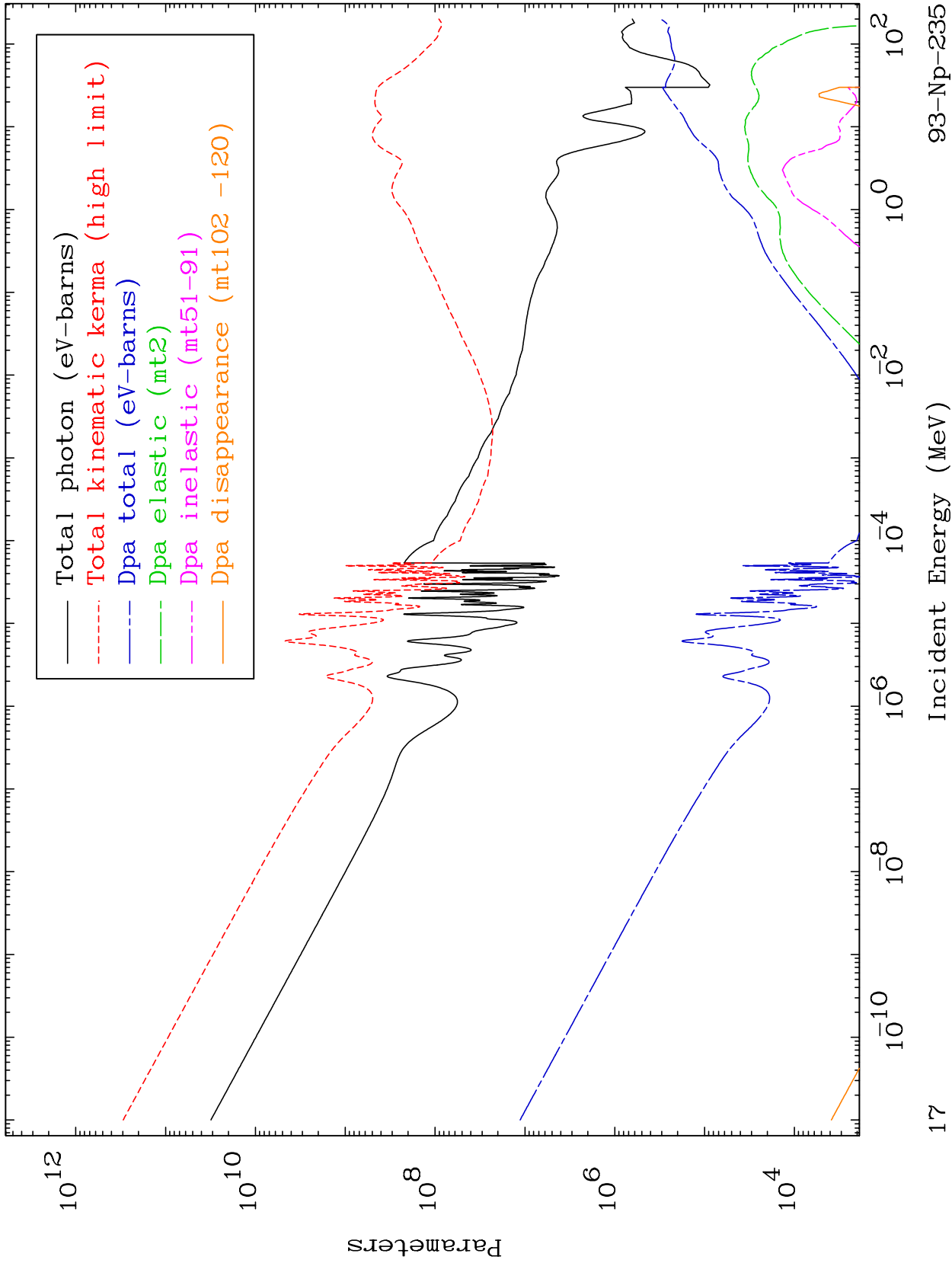
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

93-Np-235





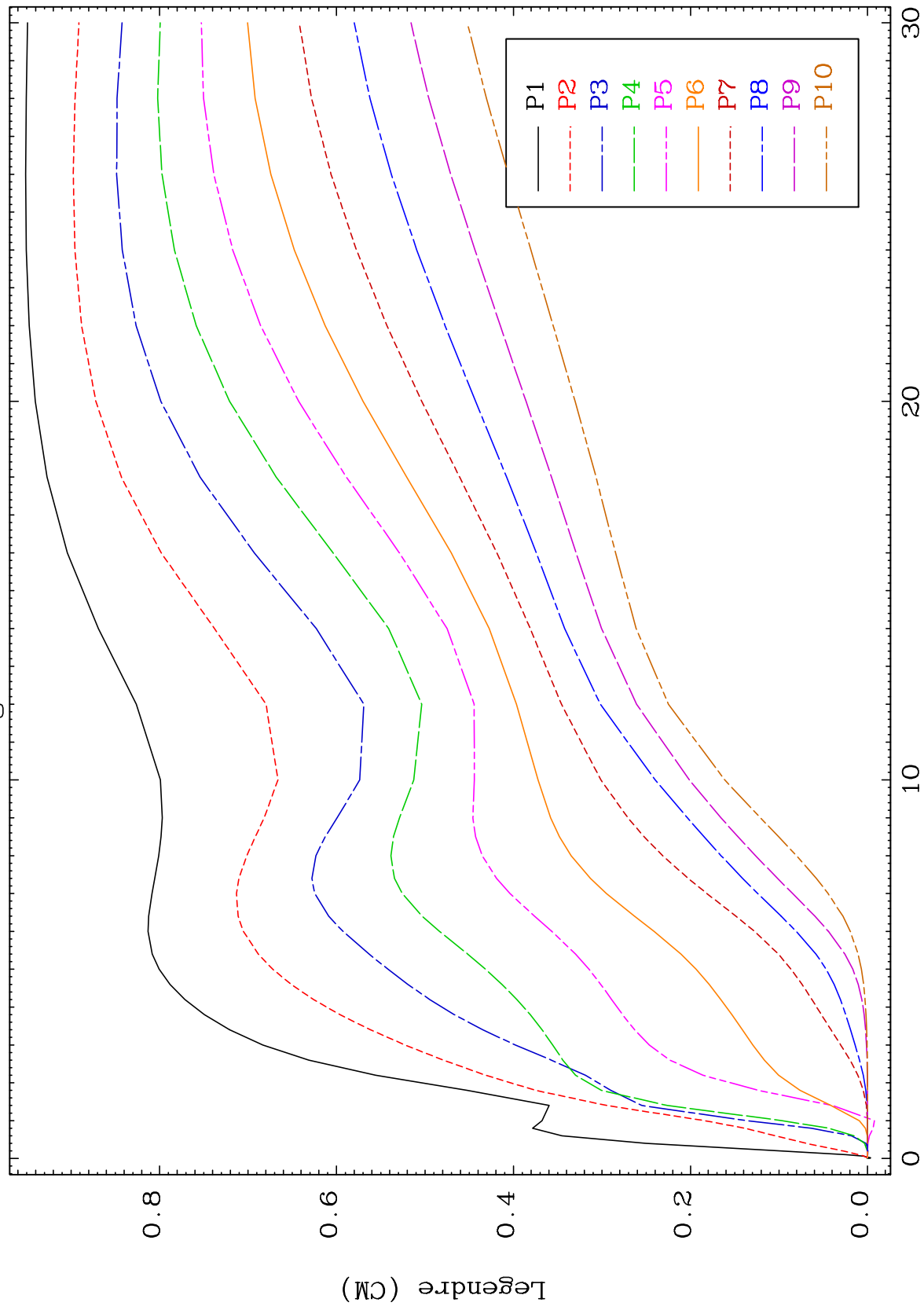




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Elastic Legendre Coefficients

<sup>93</sup>Np-235



18

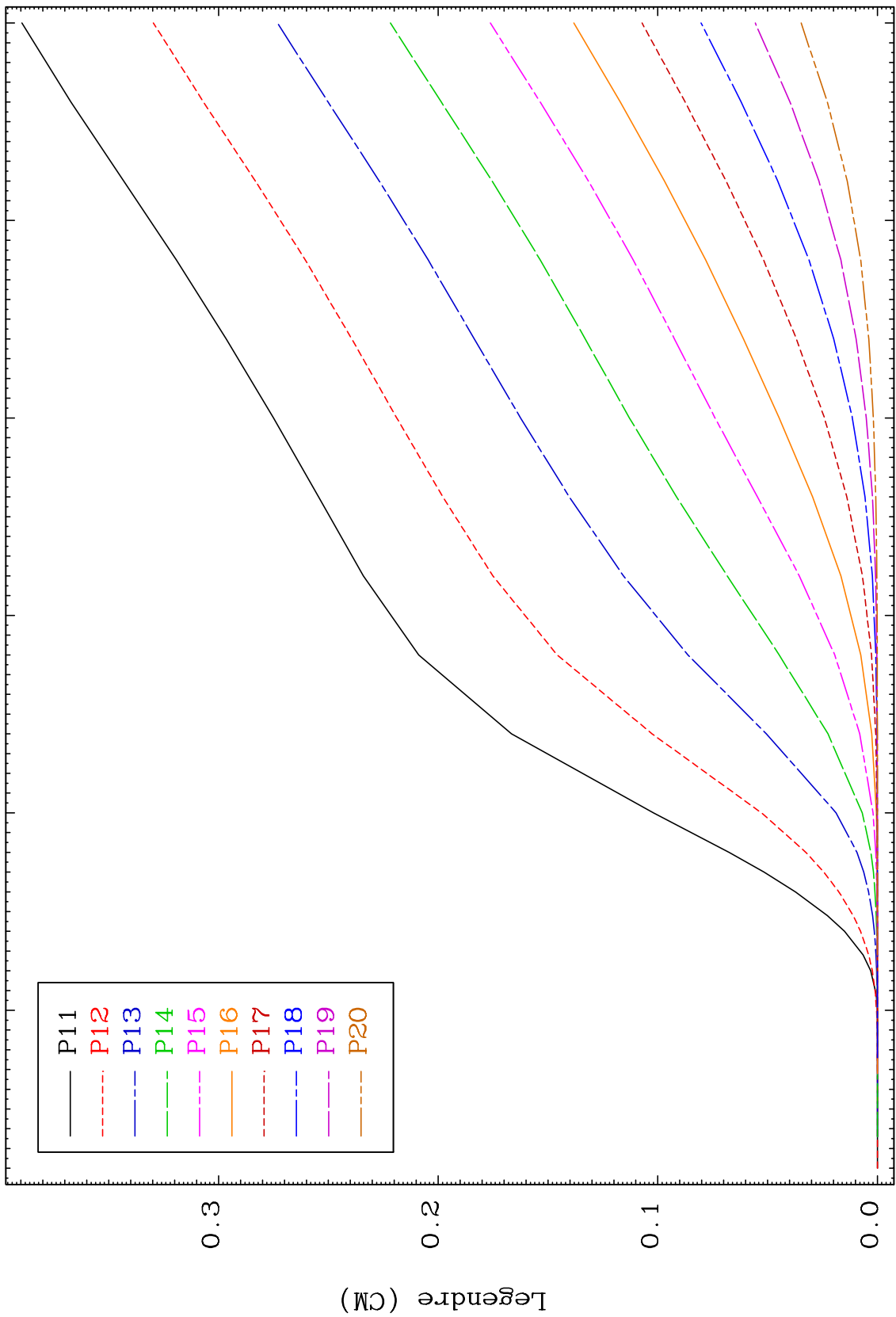
Incident Energy (MeV)

<sup>93</sup>Np-235

MAT 9340

Elastic Legendre Coefficients

<sup>93</sup>Np-235



19

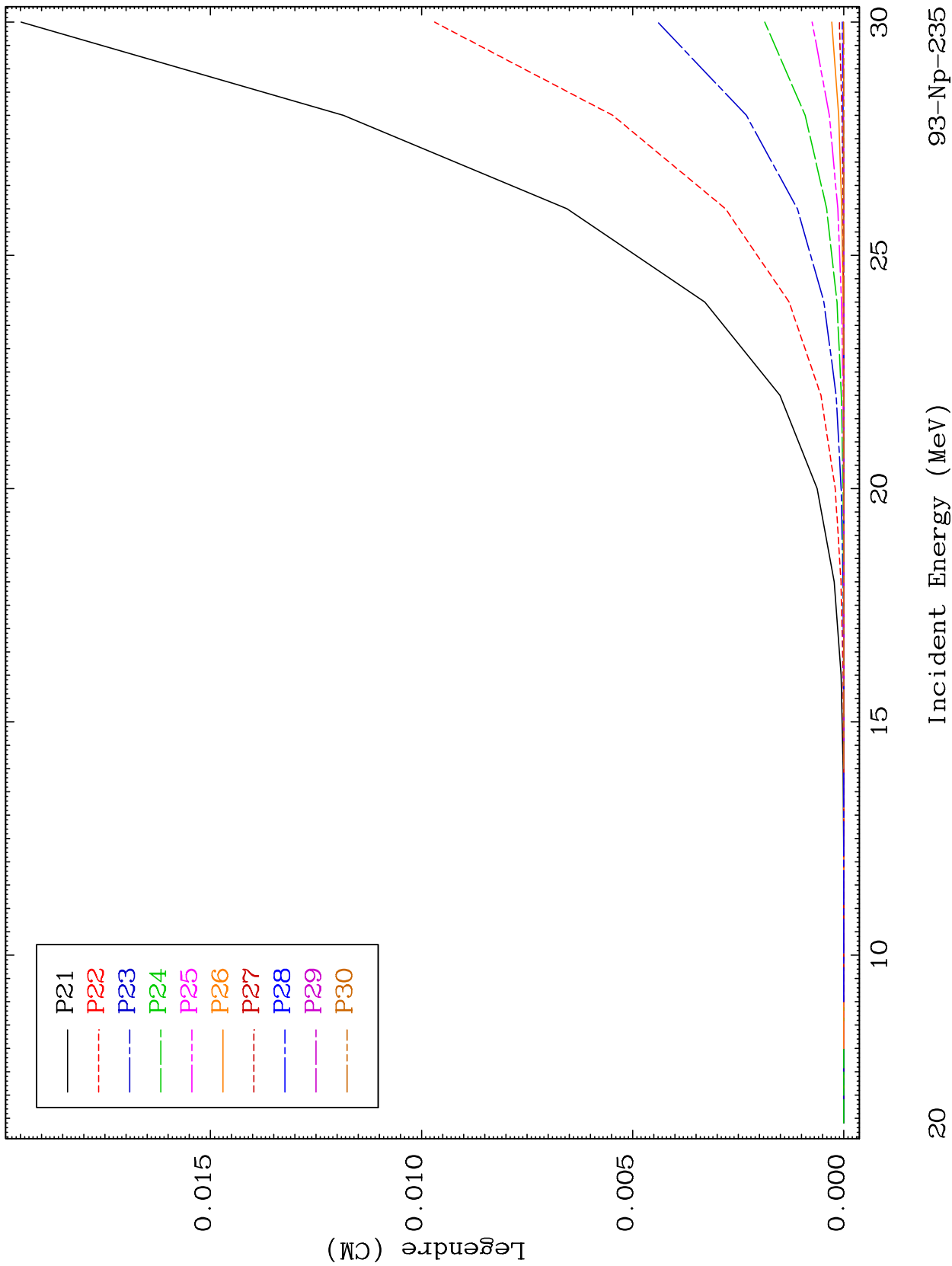
Incident Energy (MeV)

<sup>93</sup>Np-235

MAT 9340

Elastic Legendre Coefficients

93-Np-235



93-Np-235

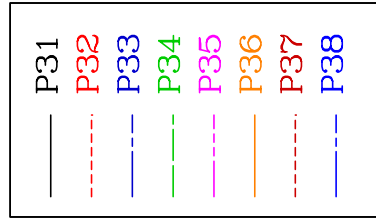
Incident Energy (MeV)

20

MAT 9340

Elastic Legendre Coefficients

93-Np-235



$\times 10^{-6}$   
Legendre (CM)

1.0

0.8

0.6

0.4

0.2

0.0

18

20

22

24

26

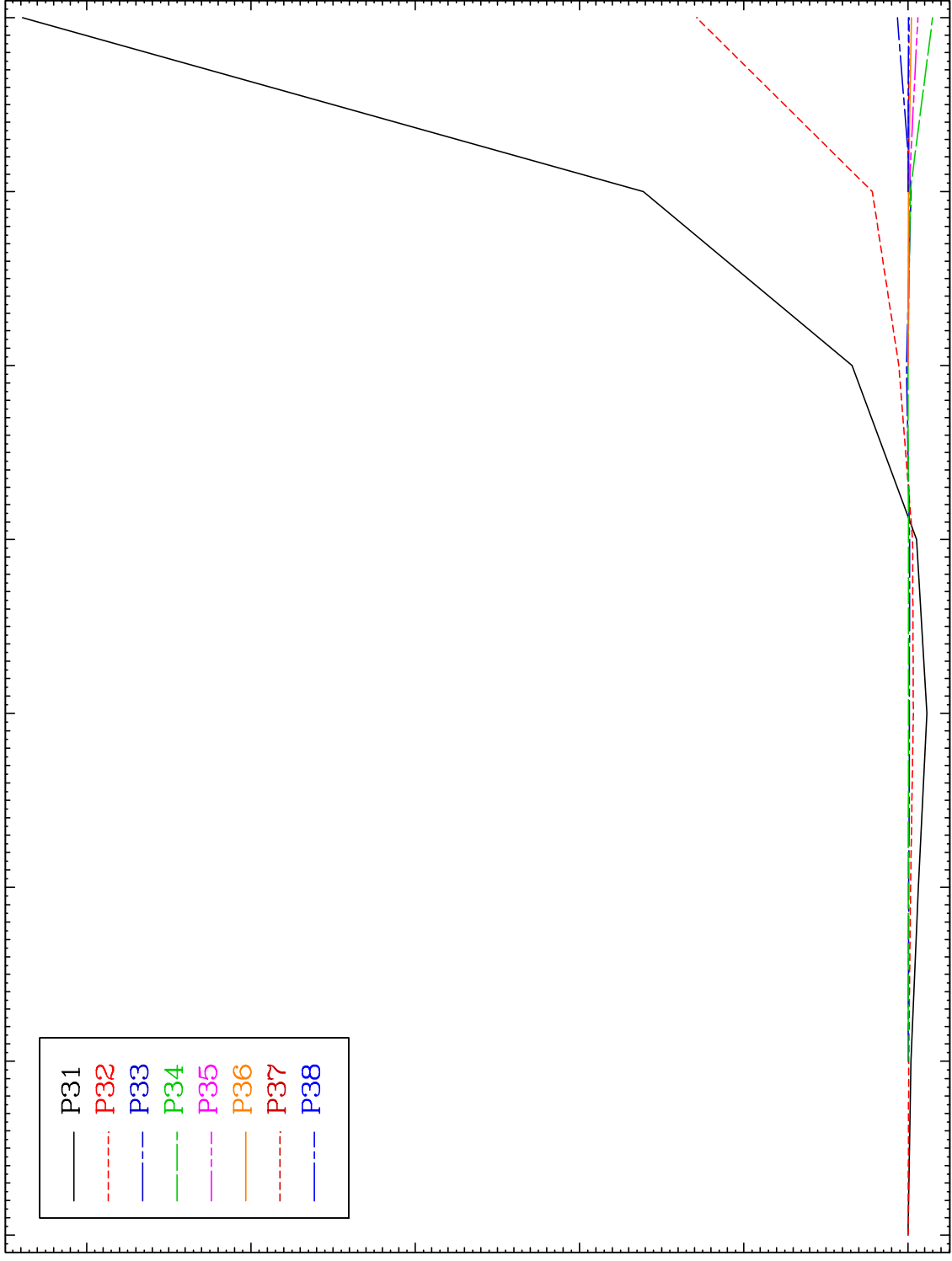
28

30

21

Incident Energy (MeV)

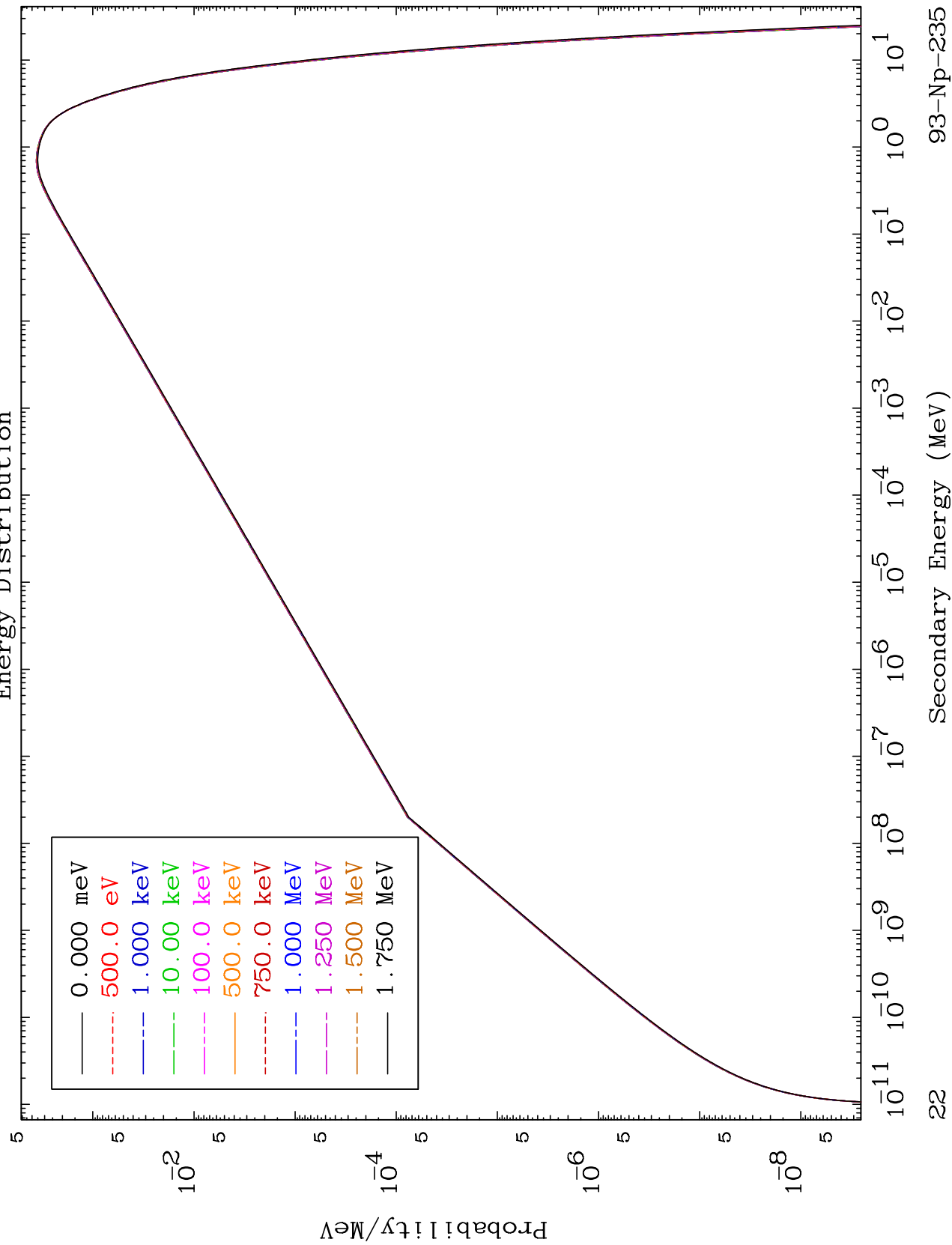
93-Np-235



MAT 9340

### Fission Energy Distribution

93-Np-235



93-Np-235

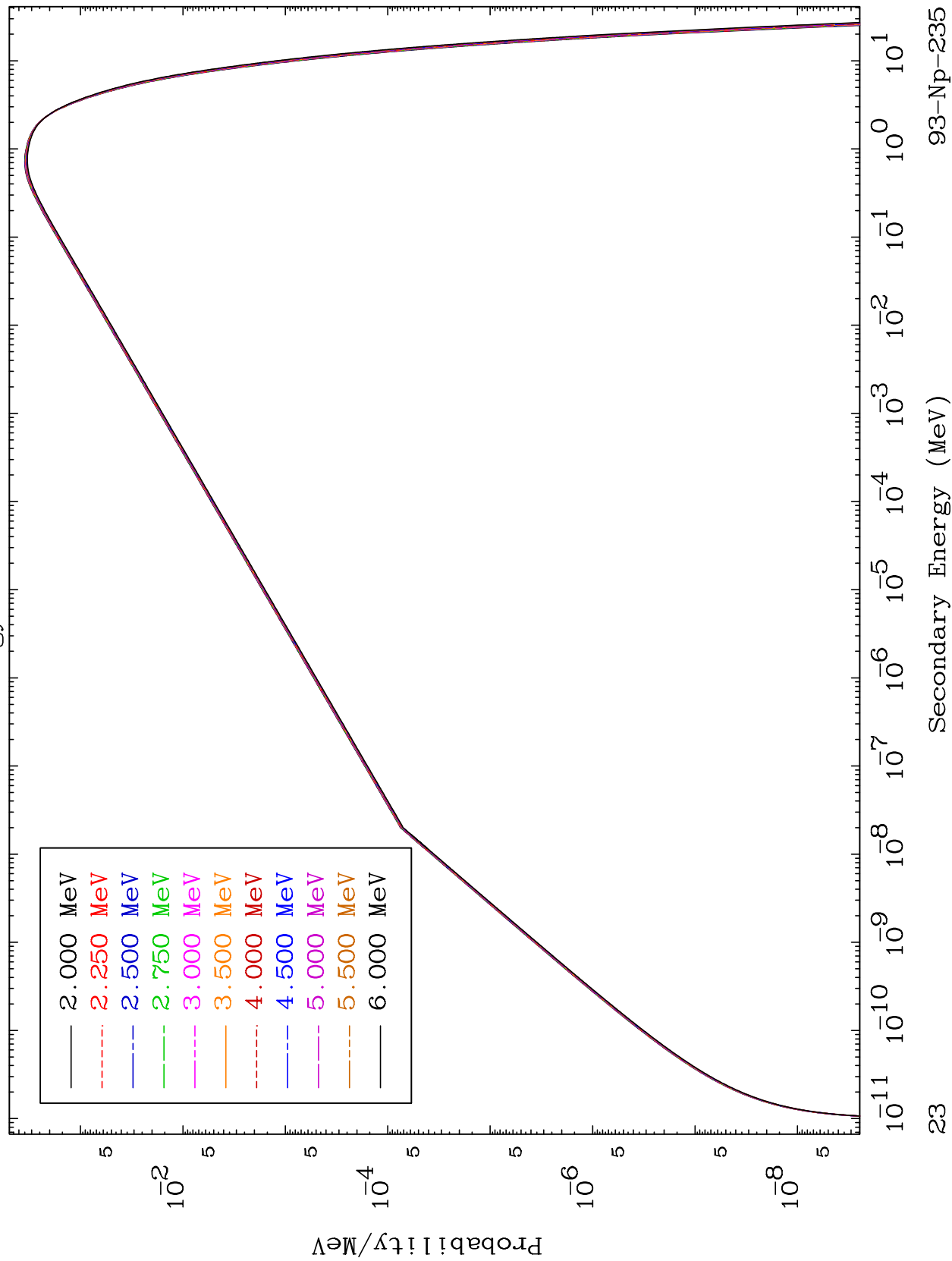
Secondary Energy (MeV)

22

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### Fission Energy Distribution

93-Np-235



93-Np-235

Secondary Energy (MeV)

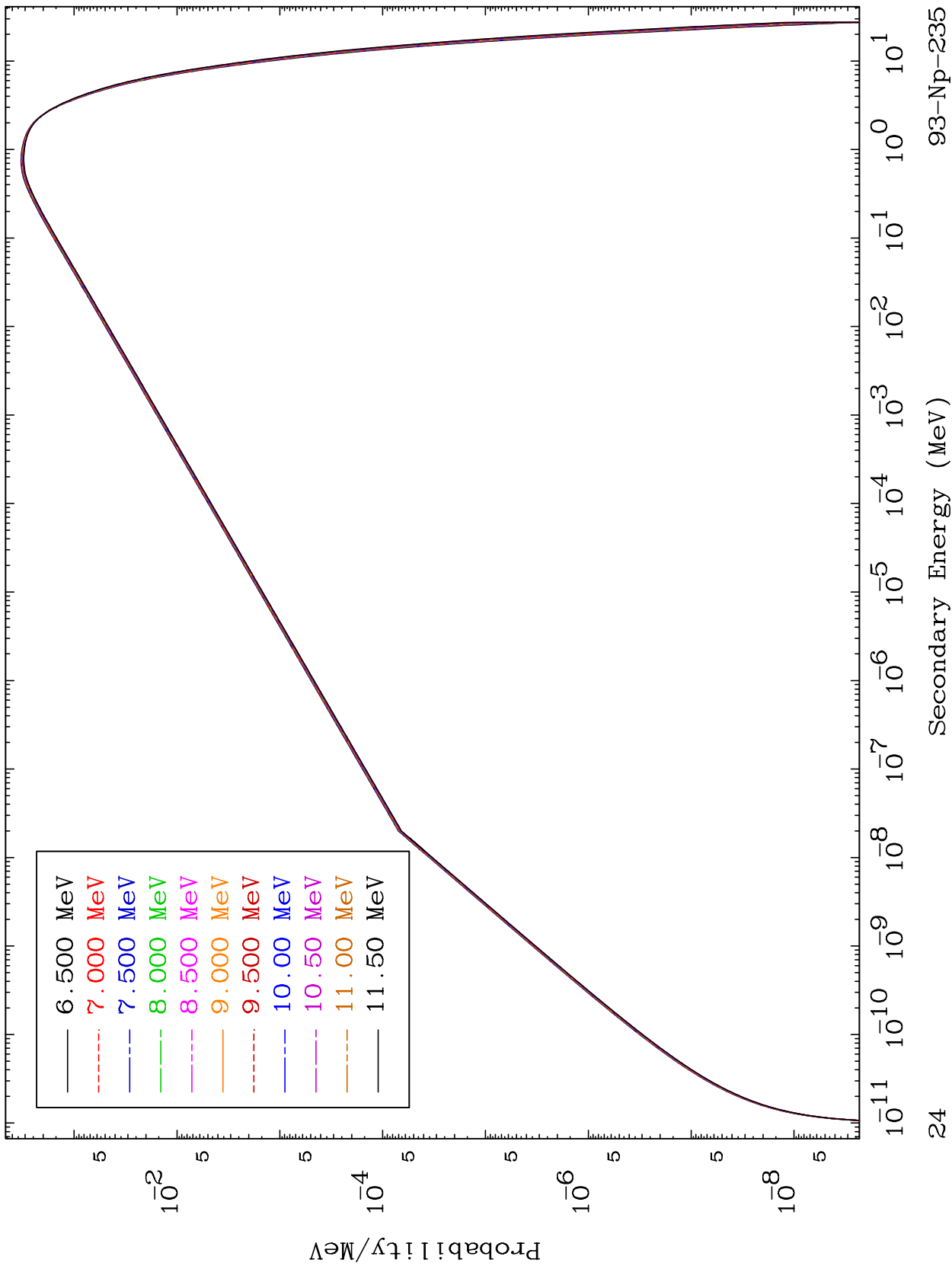
23



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### Fission Energy Distribution

93-Np-235



24

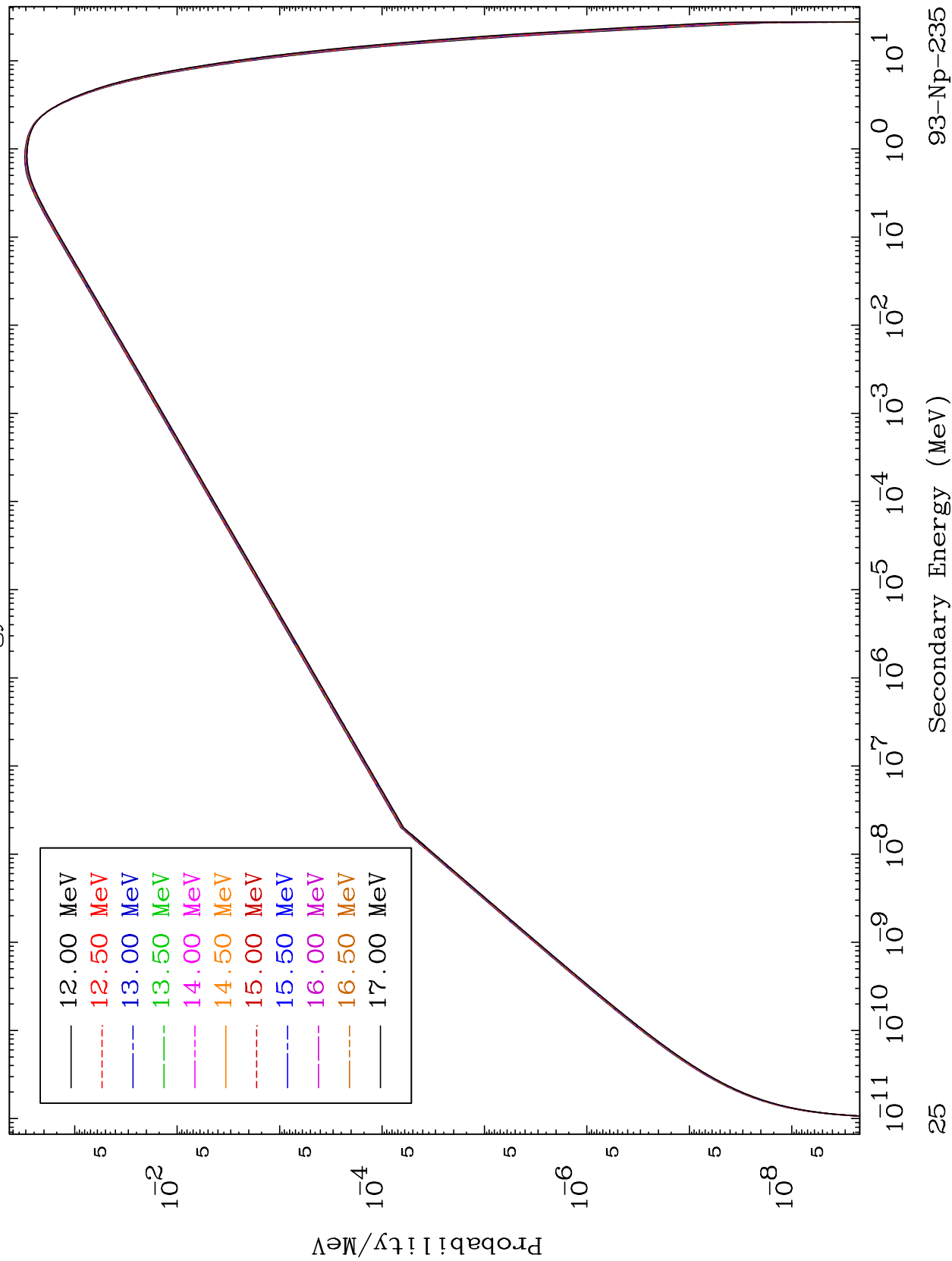
Secondary Energy (MeV)

93-Np-235

MAT 9340

### Fission Energy Distribution

93-Np-235



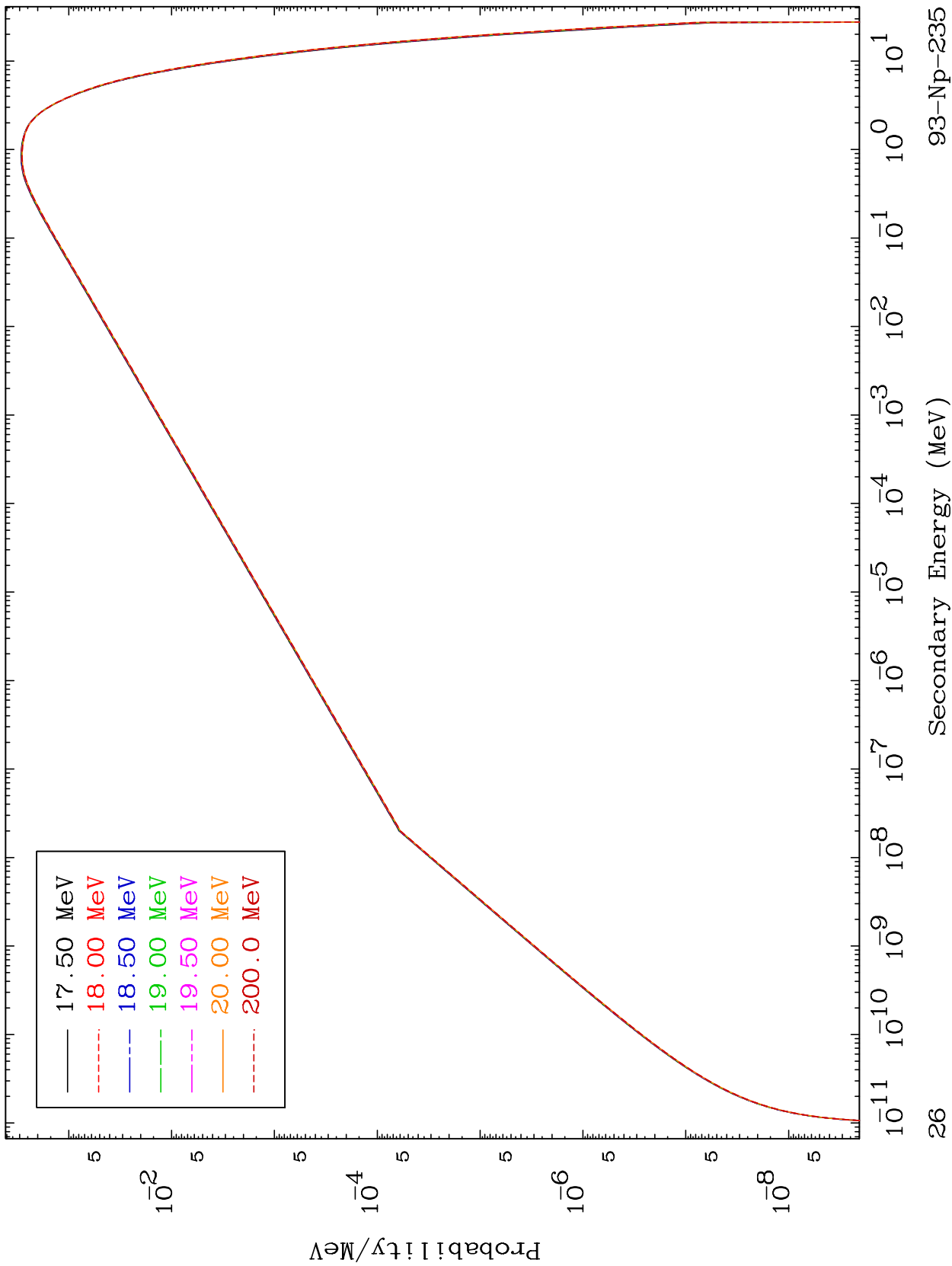
25

93-Np-235

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### Fission Energy Distribution

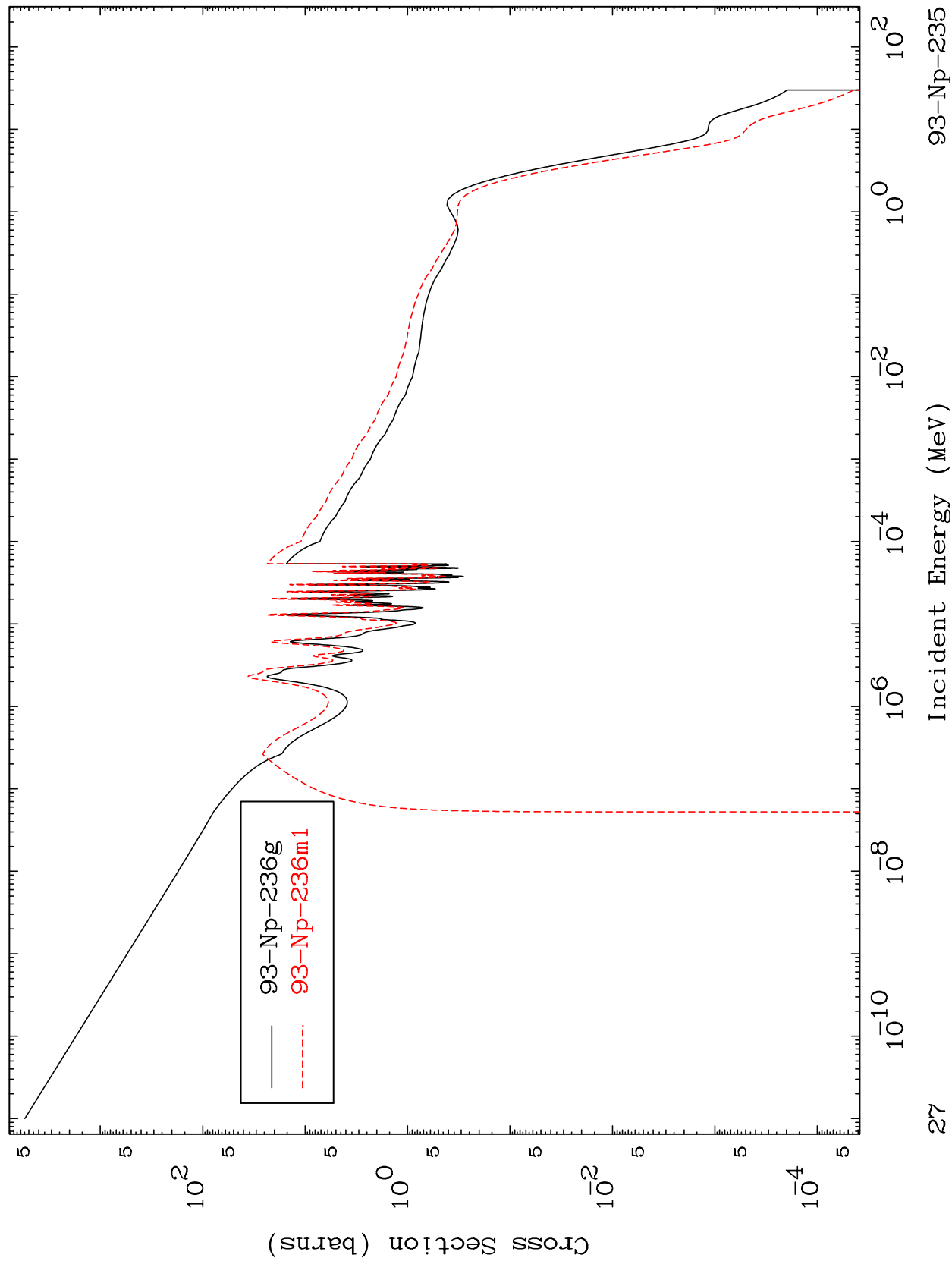
93-Np-235



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<sup>93</sup>Np-235

(n,γ)  
Radionuclide Production Cross Section

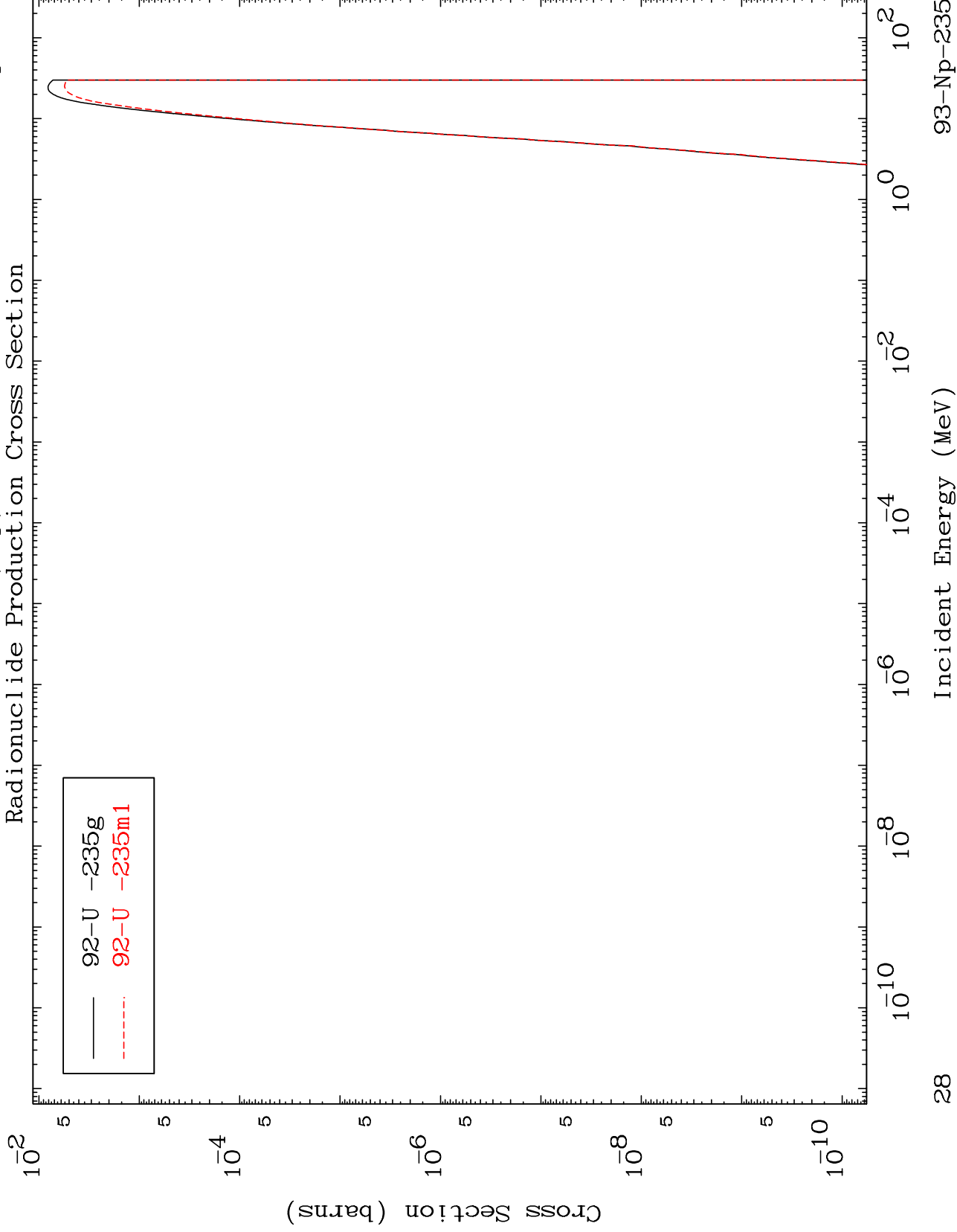


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(n,p)

93-Np-235

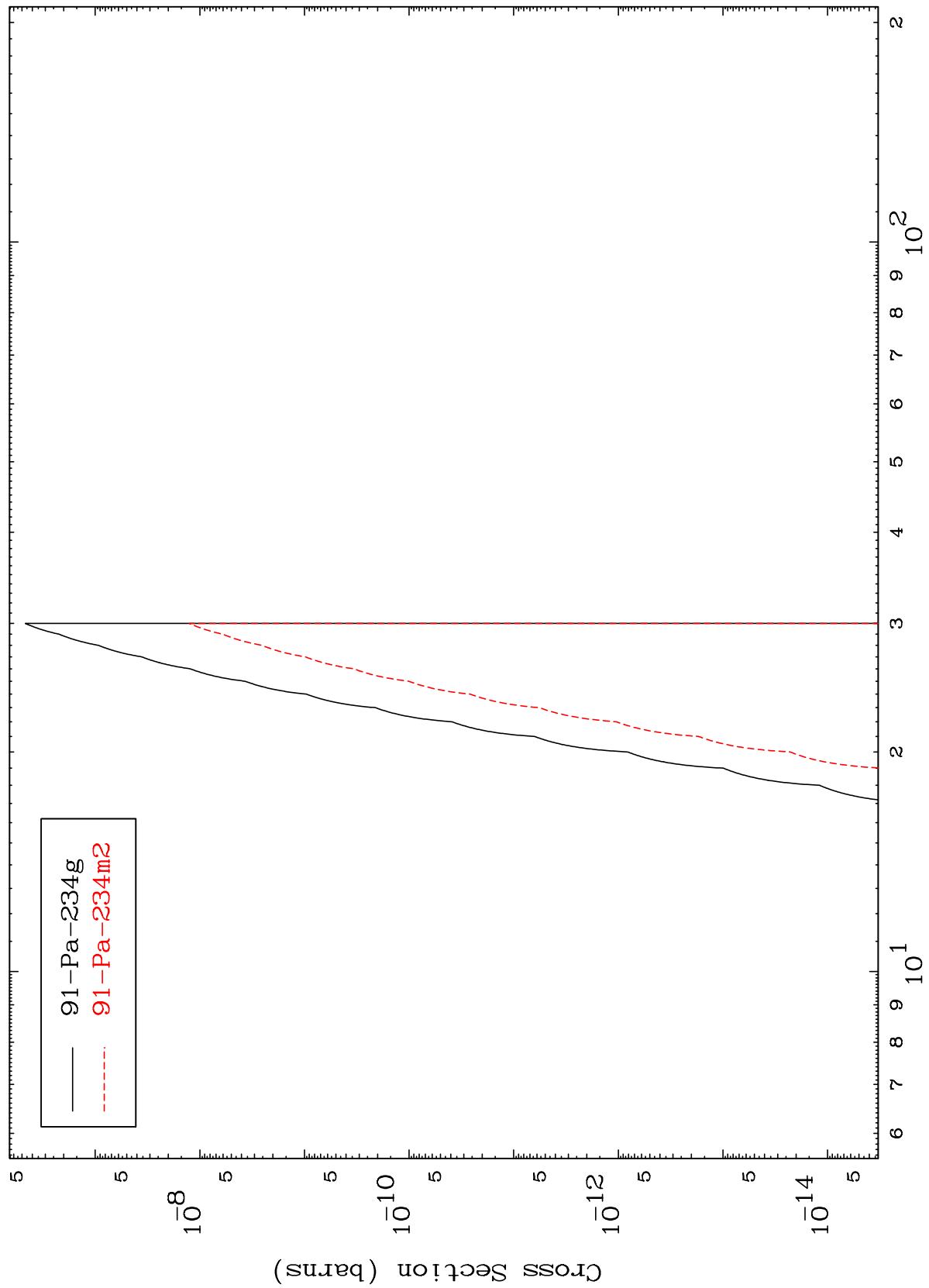
Radionuclide Production Cross Section



MAT 9340

93-Np-235

(n,2p)  
Radionuclide Production Cross Section



— 91-Pa-234g  
- - - 91-Pa-234m2

29

Incident Energy (MeV)

93-Np-235