

Program EVALPLOT
(Version 2018-1)

by

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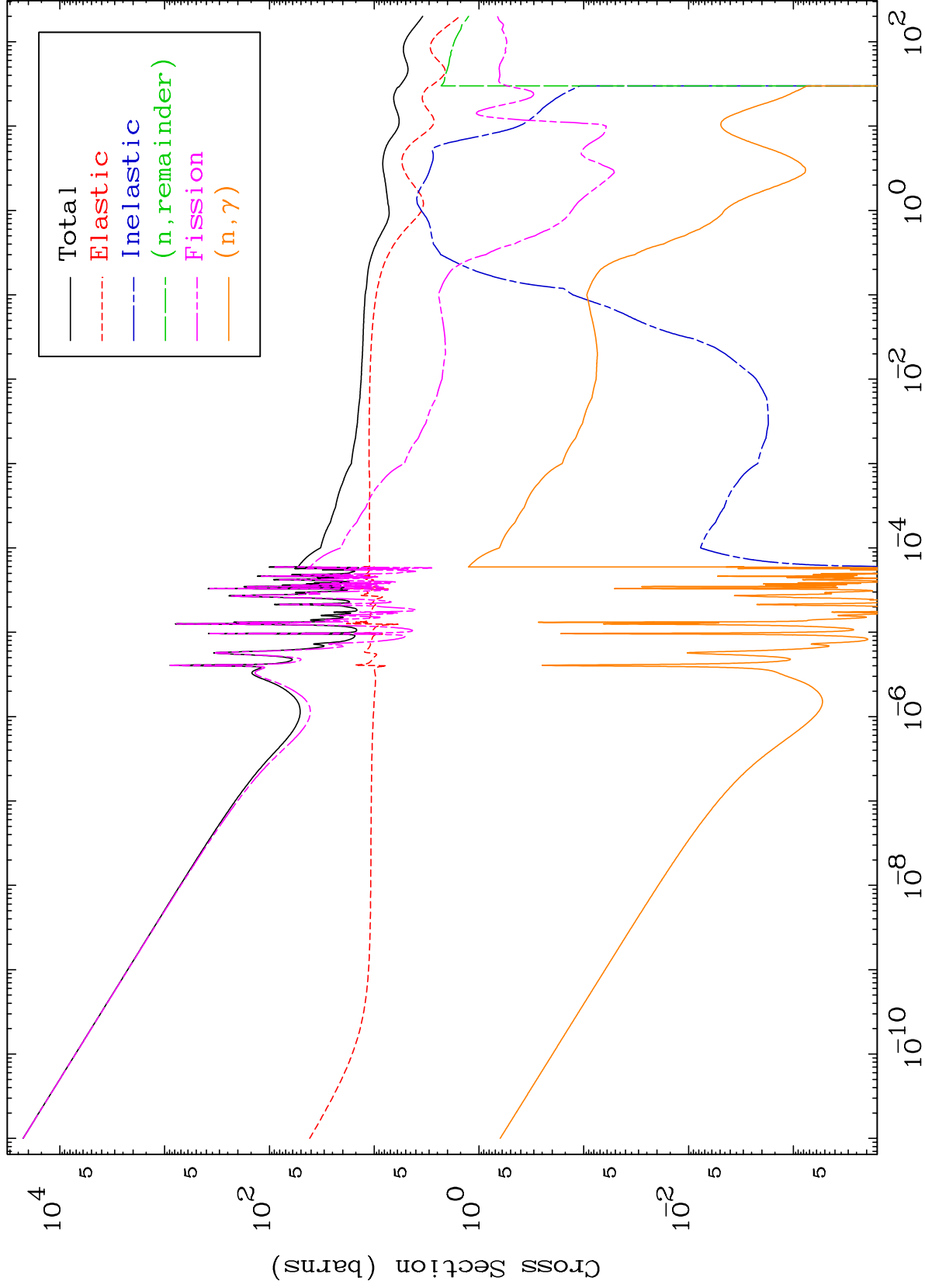
Press Mouse Button to Start

MAT 9141

Major

293 Kelvin Cross Sections

91-Pa-234



1

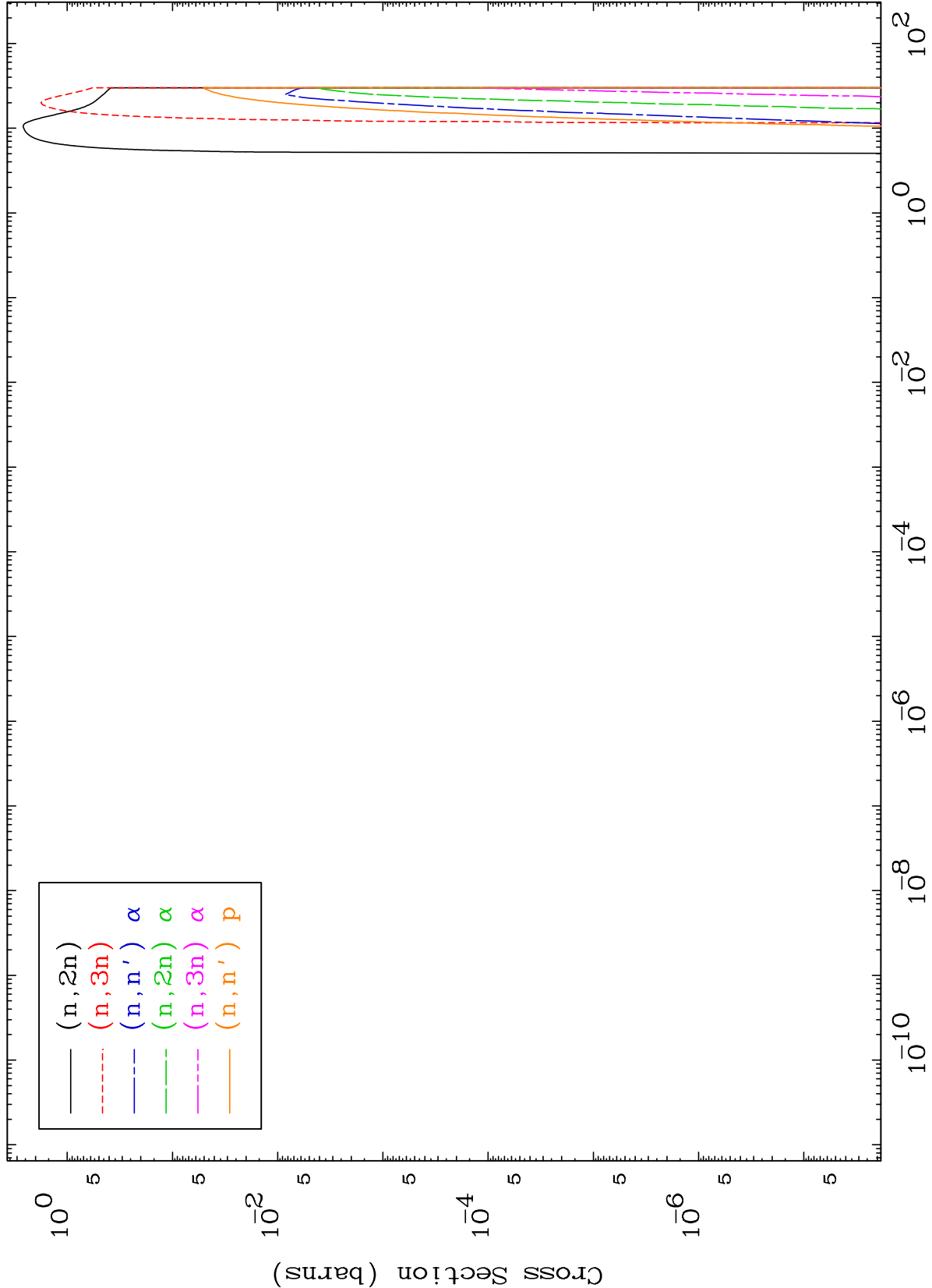
Incident Energy (MeV)

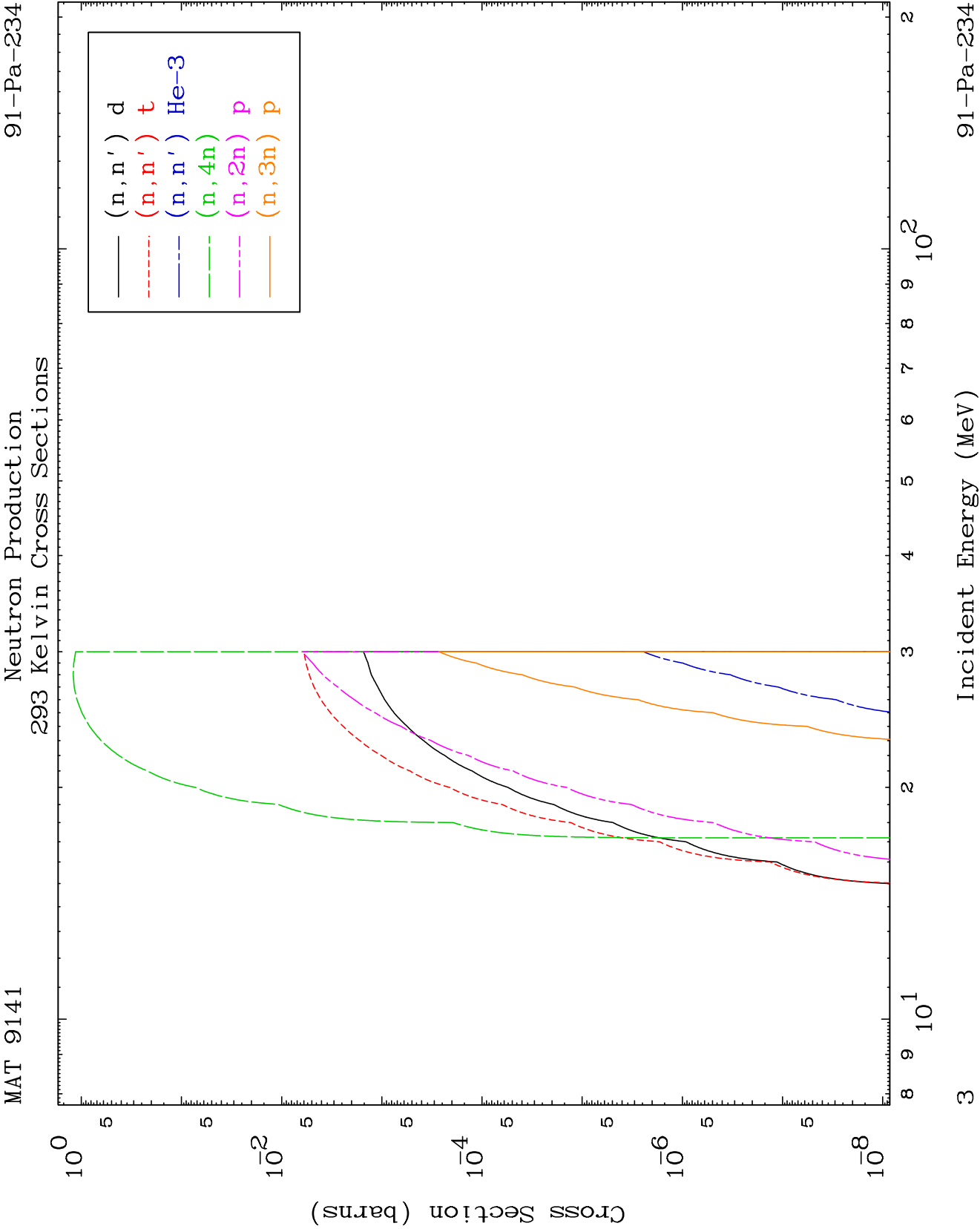
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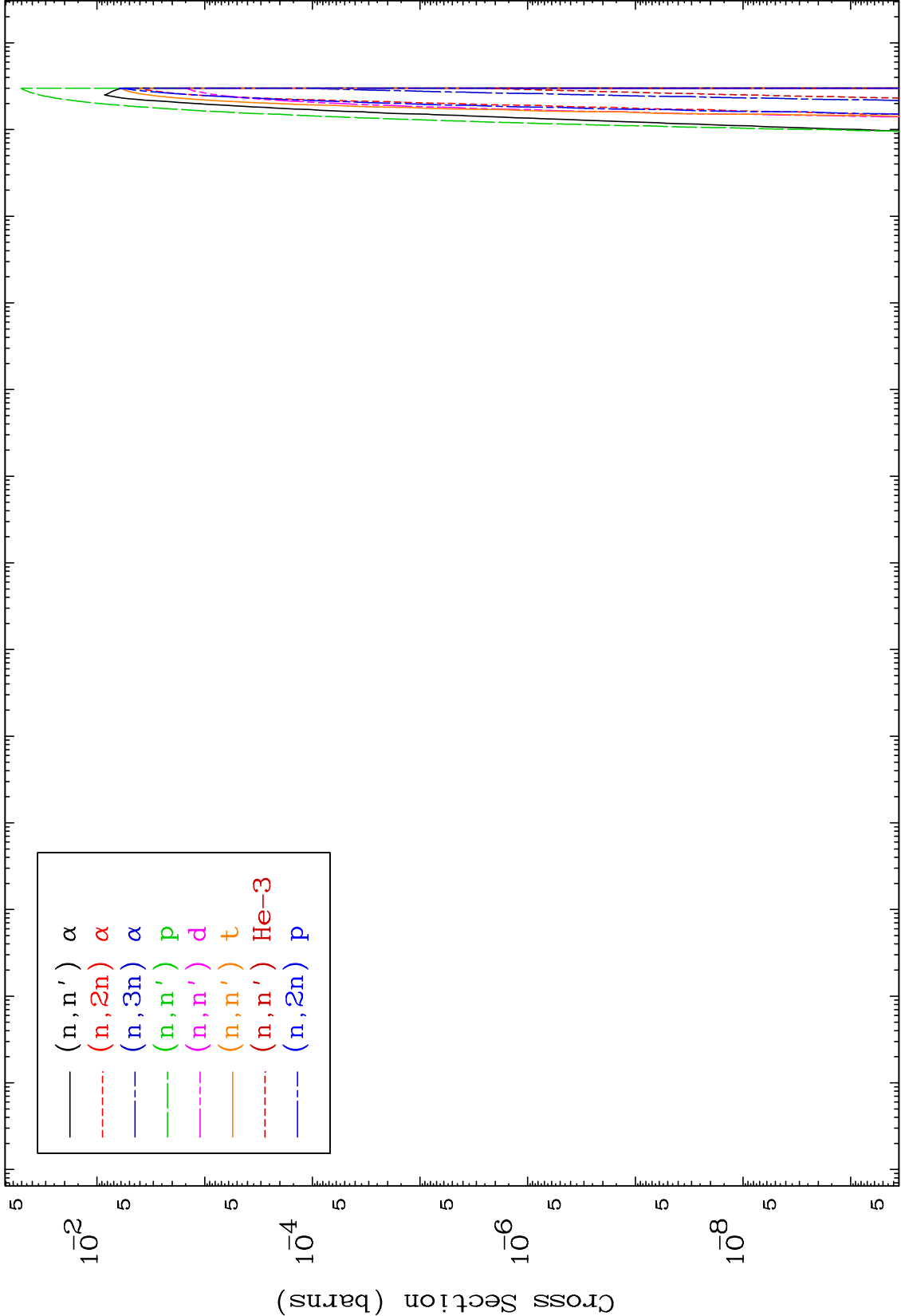
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Neutron Production
293 Kelvin Cross Sections

91-Pa-234



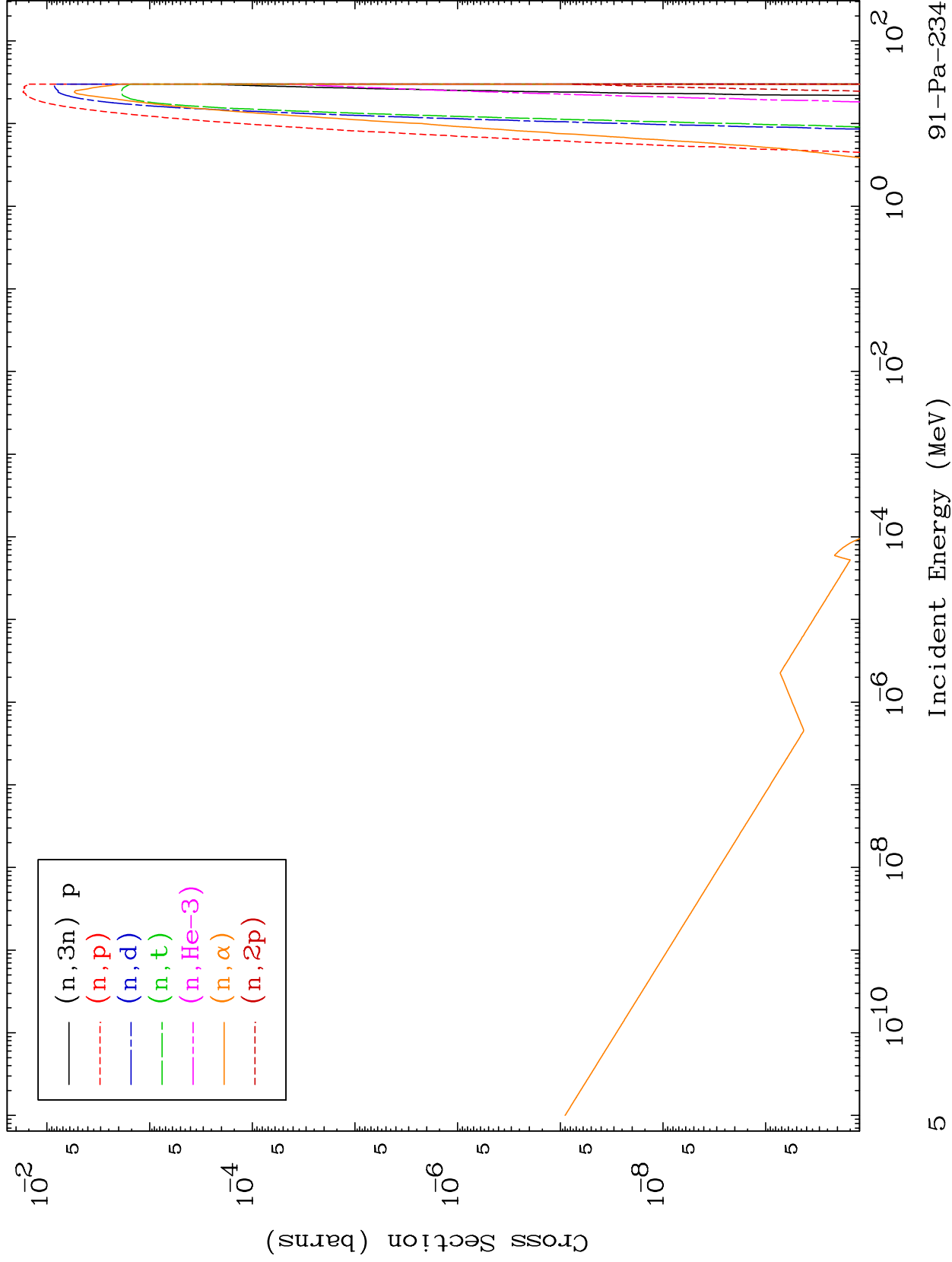


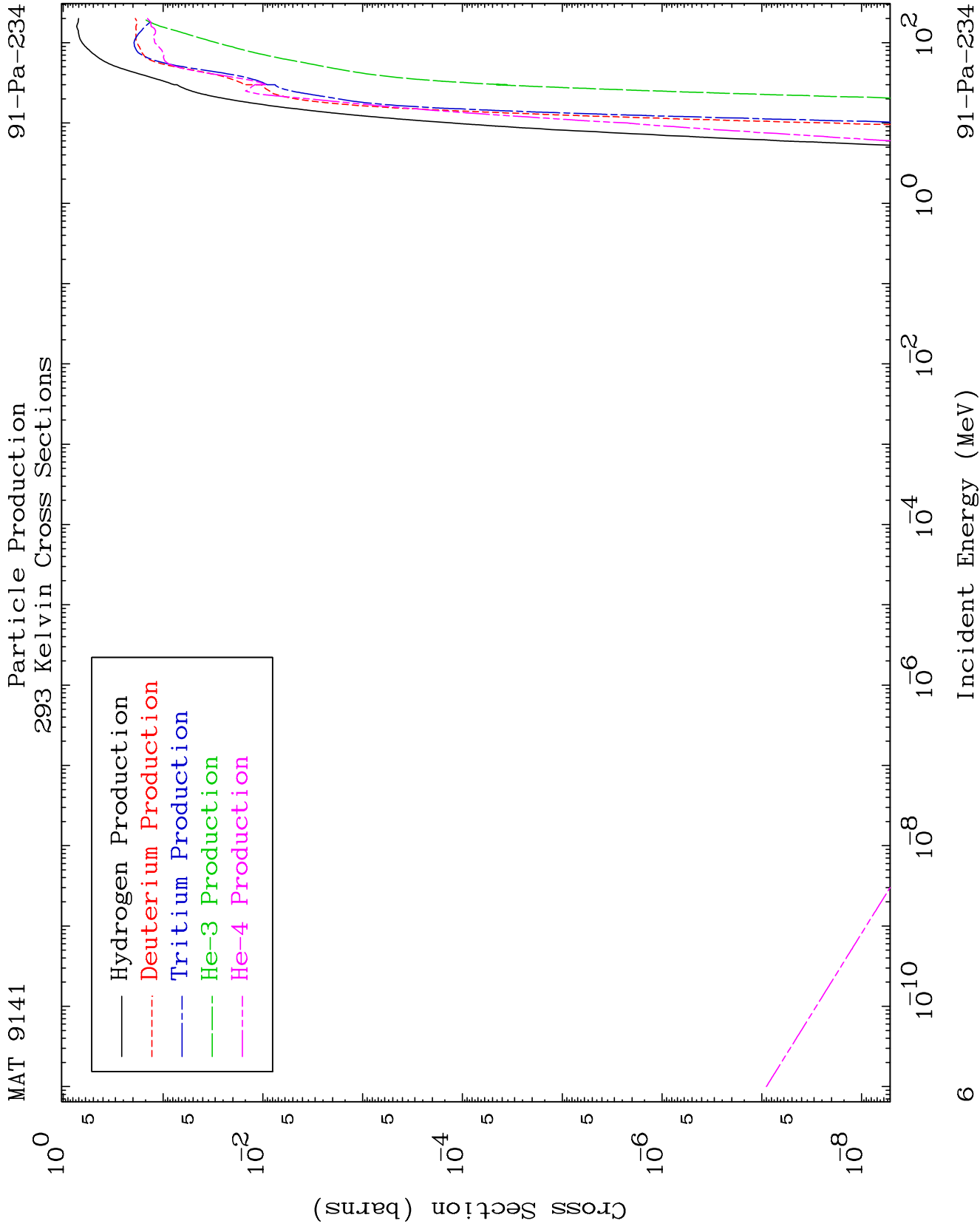


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Charged Particle
293 Kelvin Cross Sections

91-Pa-234



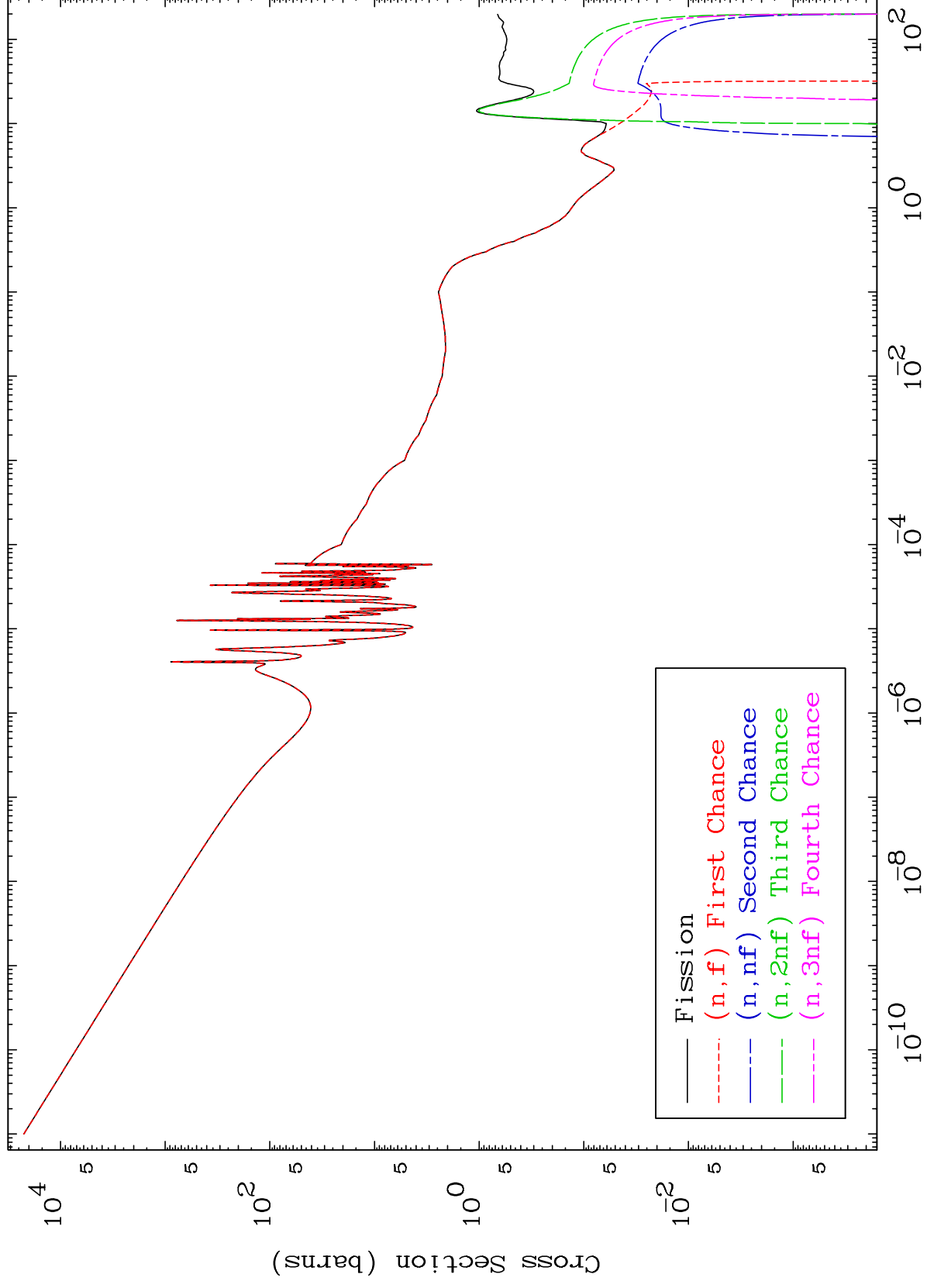


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Fission

91-Pa-234

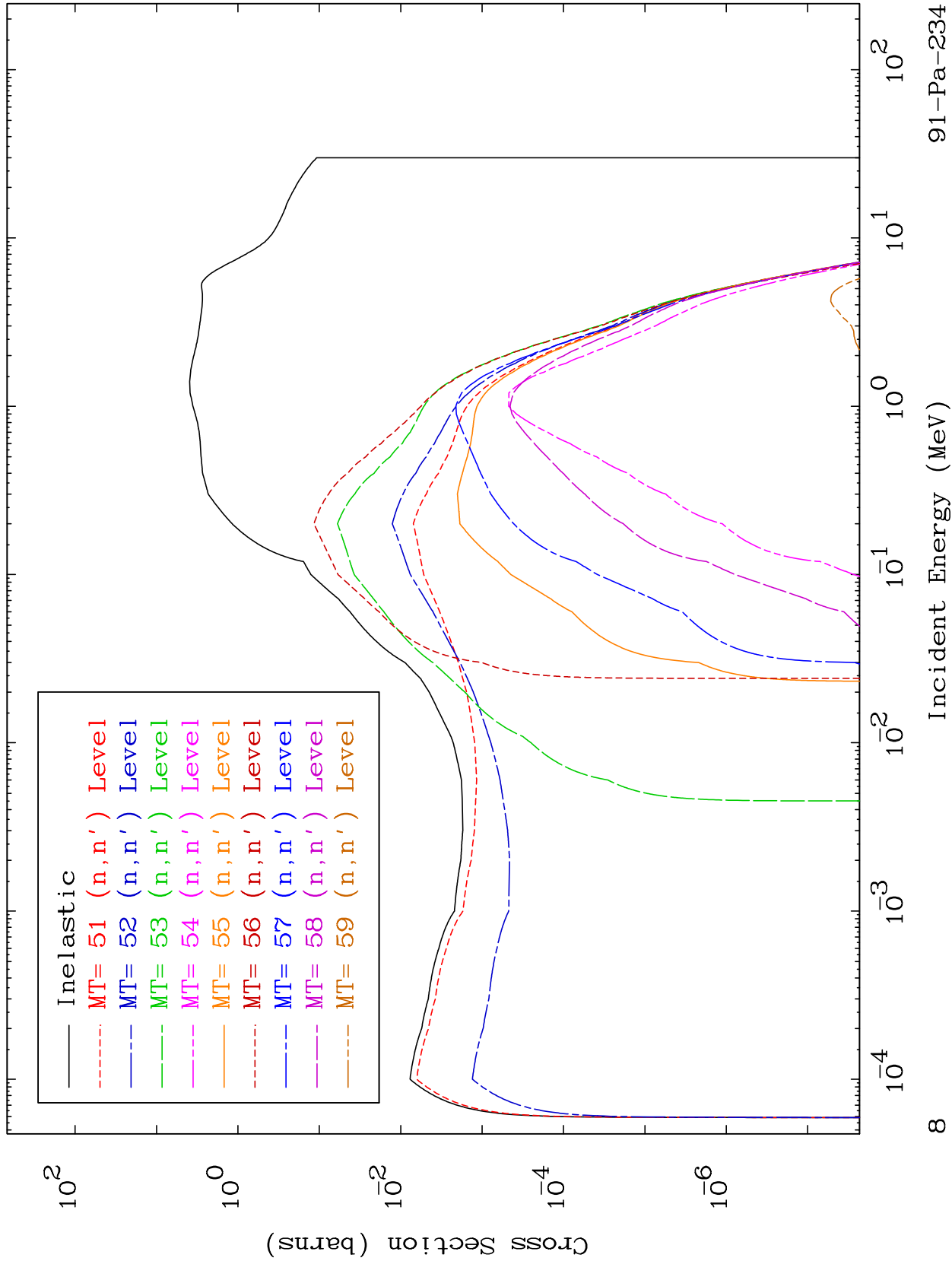
293 Kelvin Cross Sections

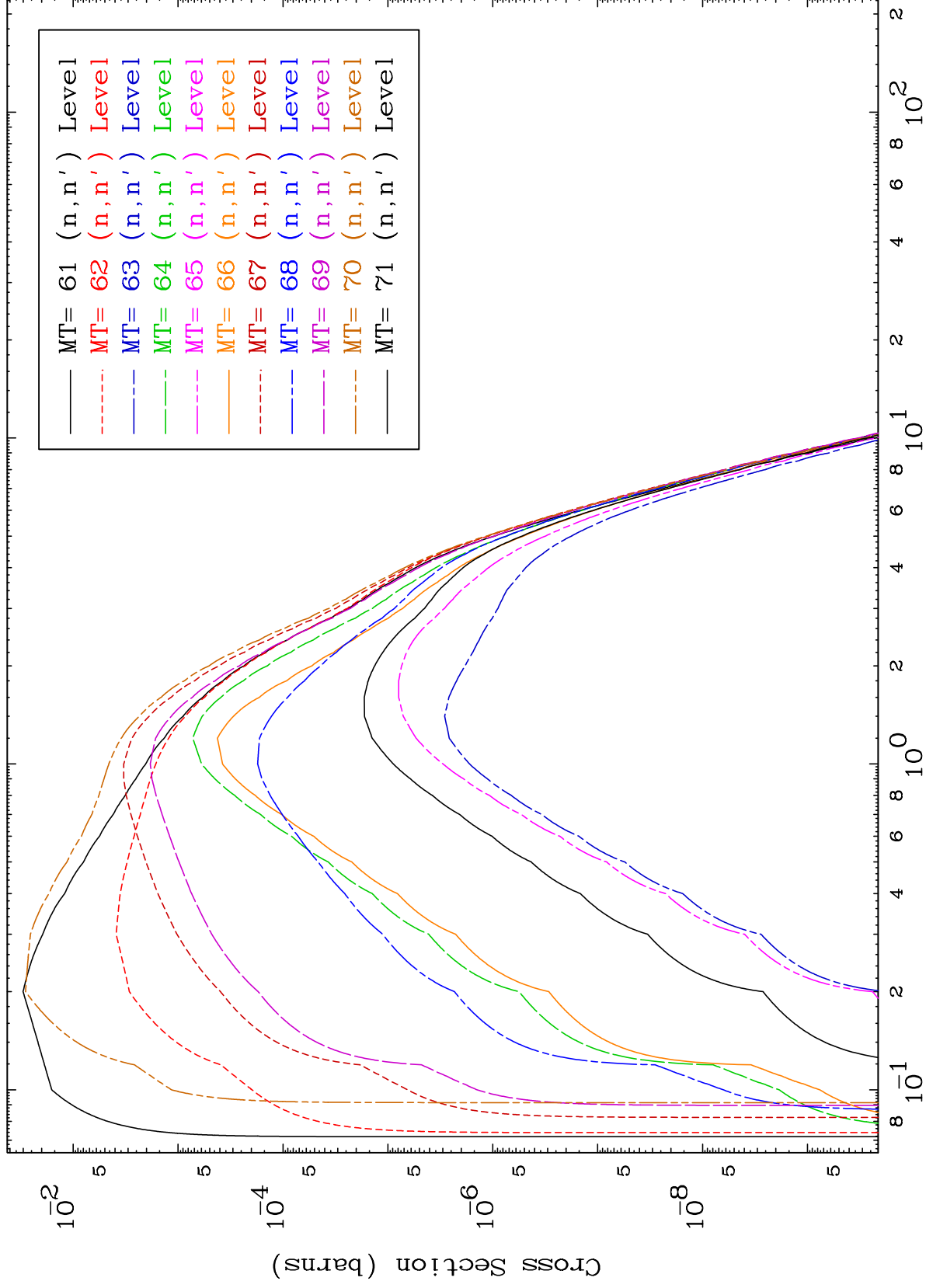


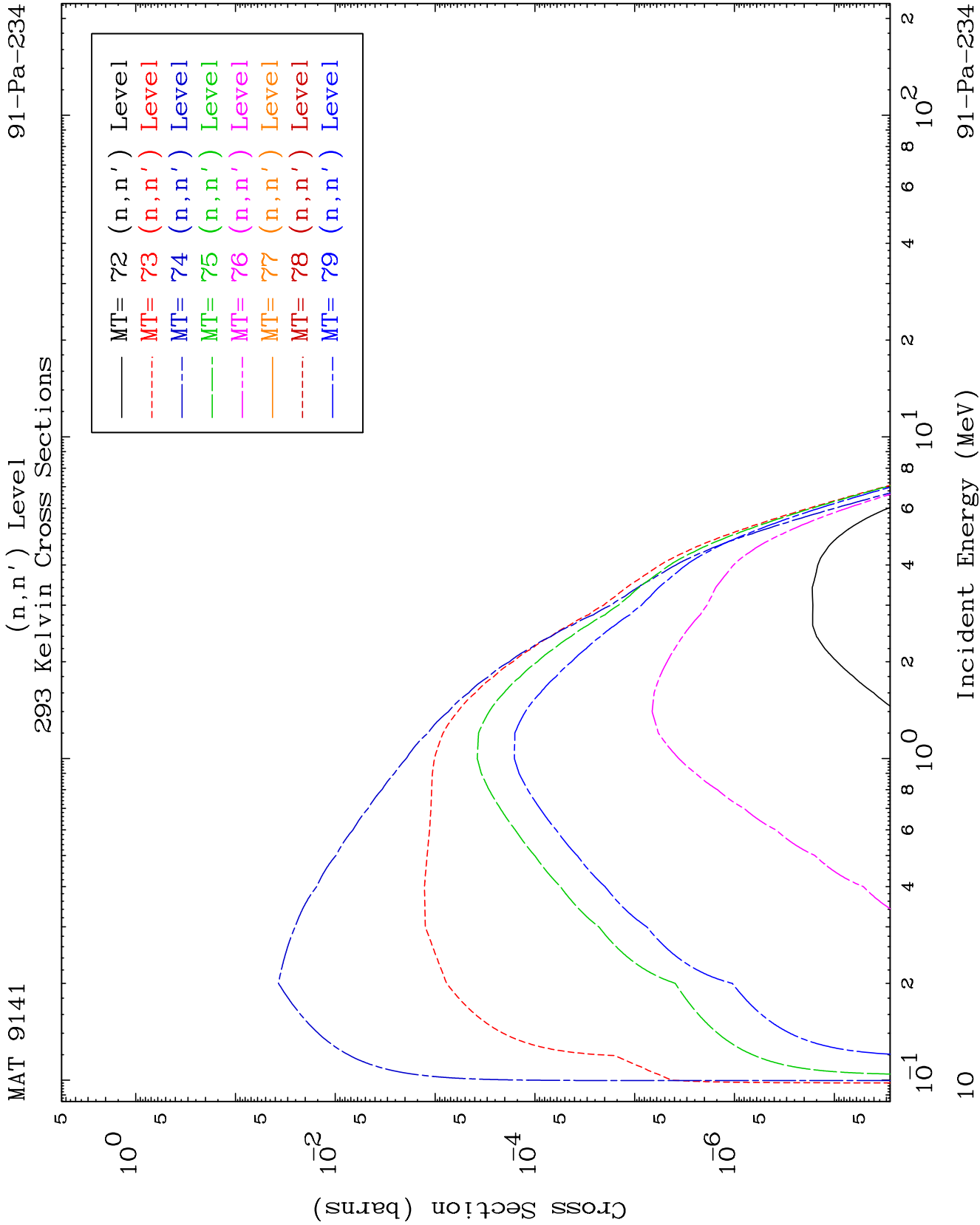
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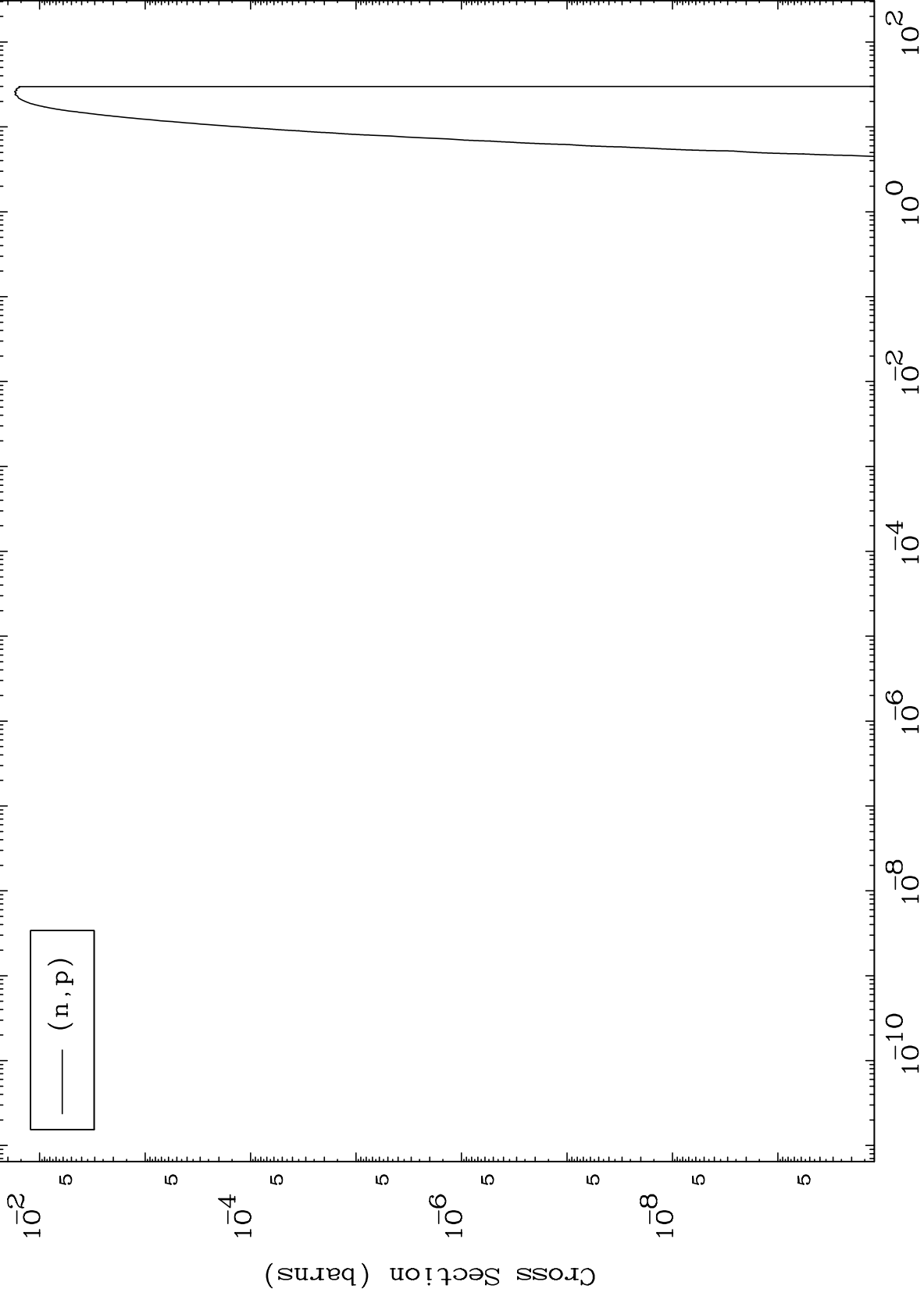
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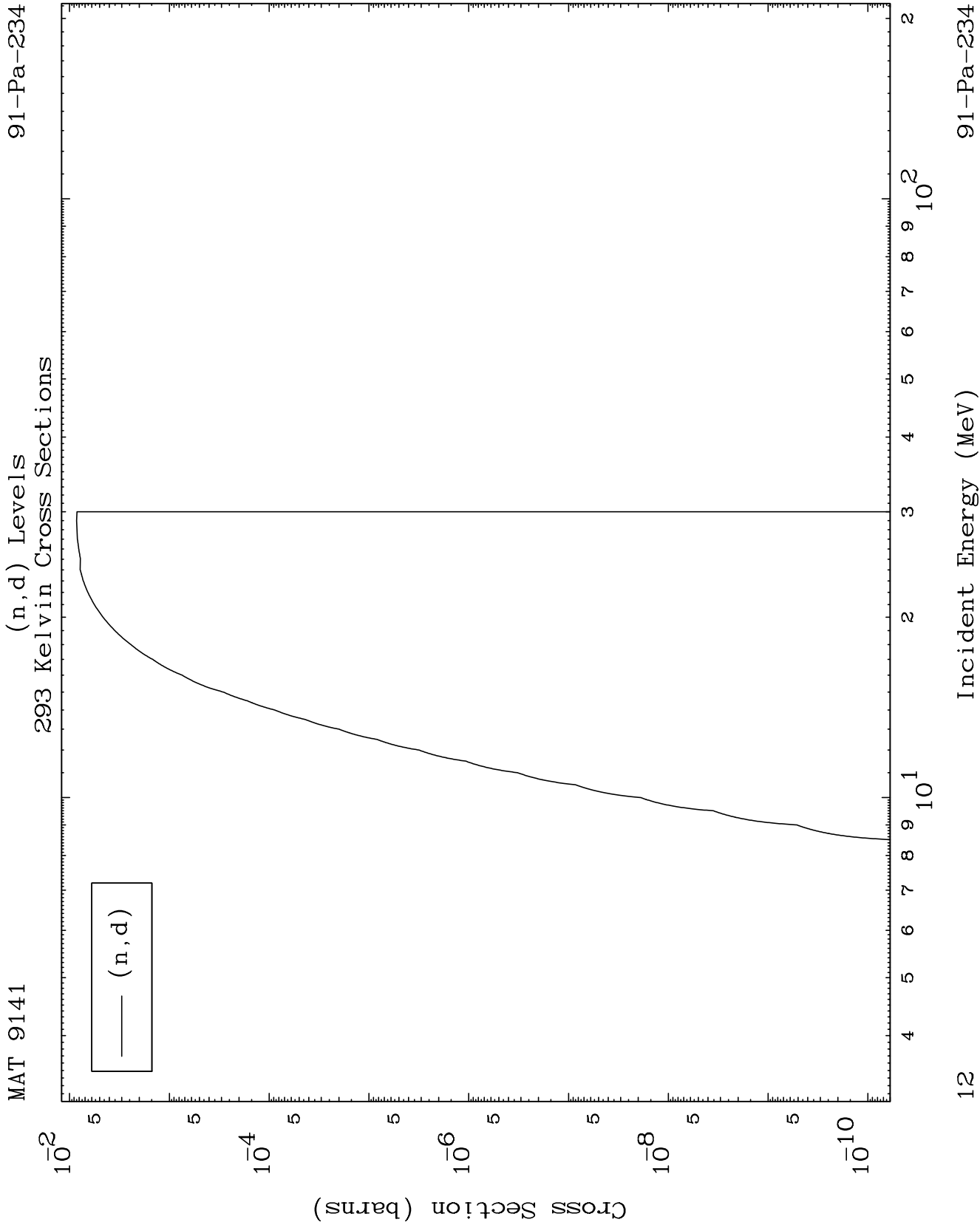
(n,n') Level
293 Kelvin Cross Sections







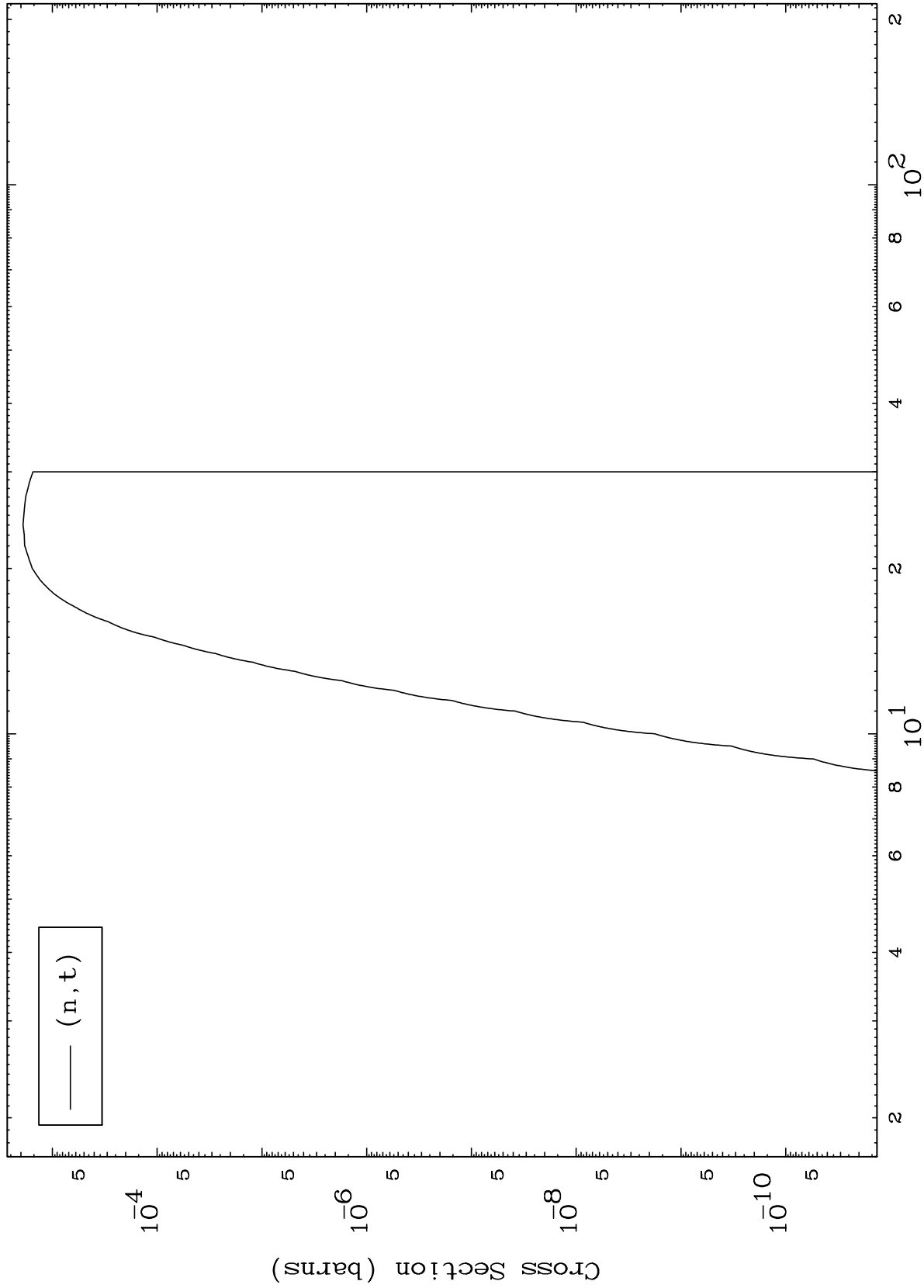




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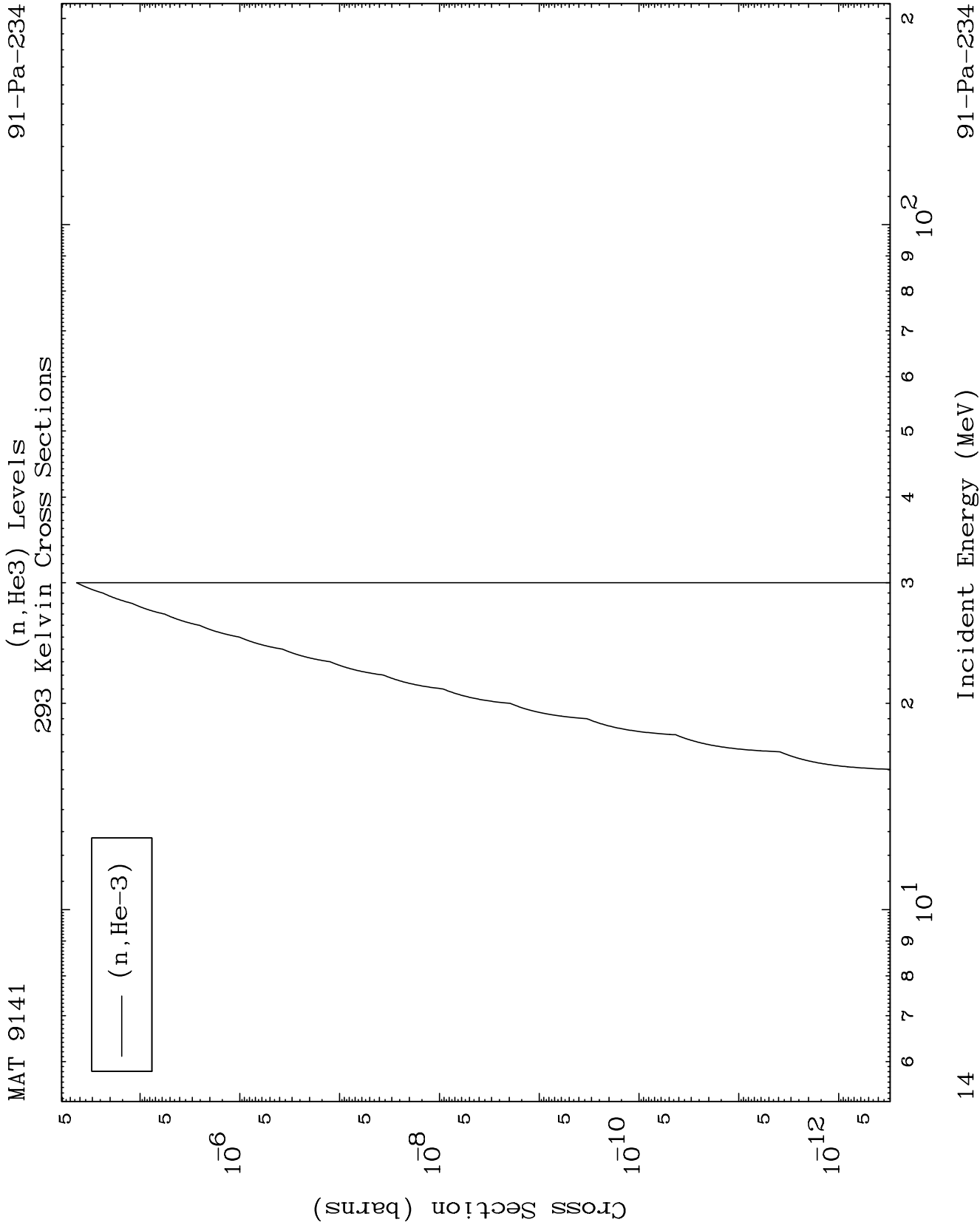
(n,t) Levels
293 Kelvin Cross Sections

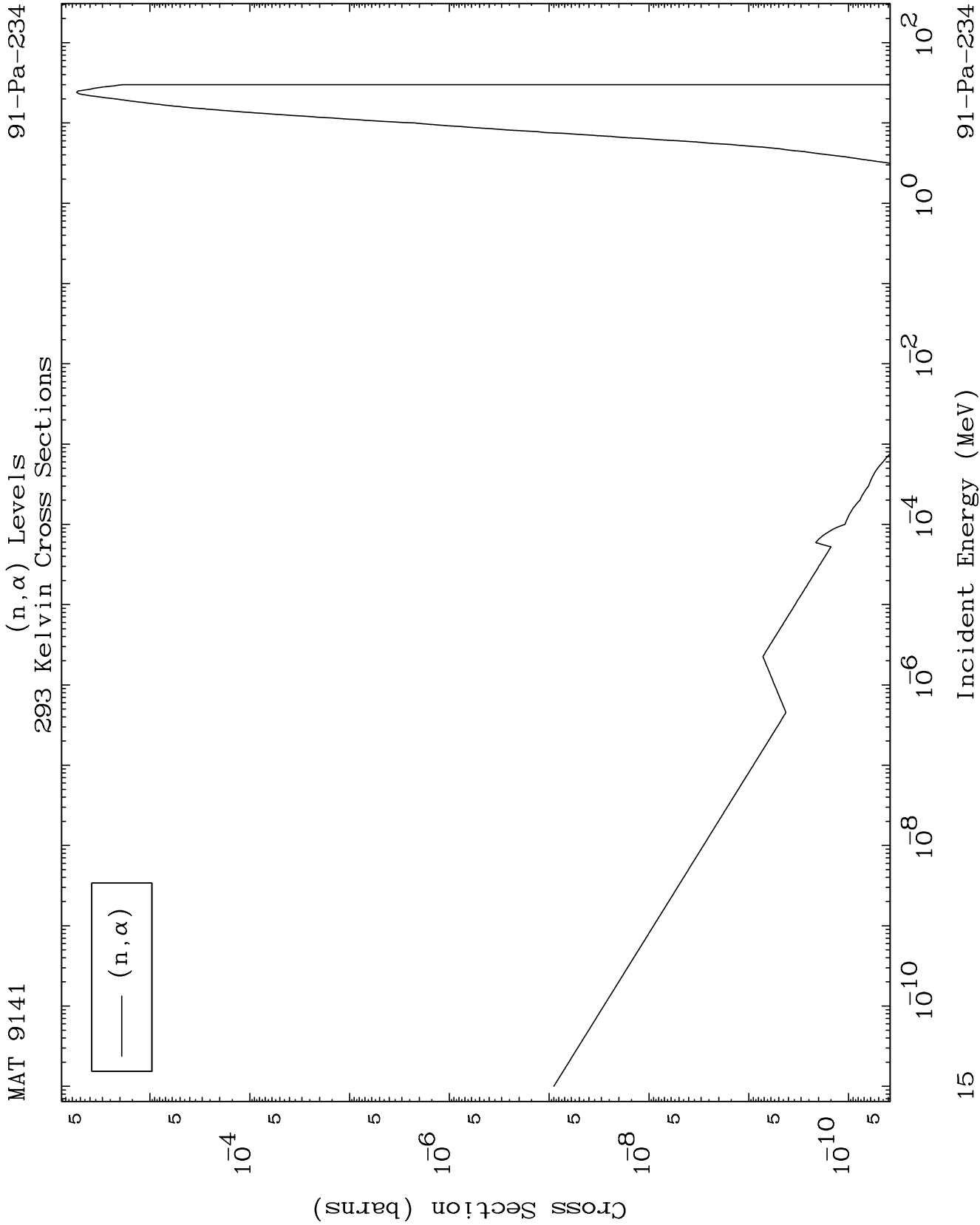


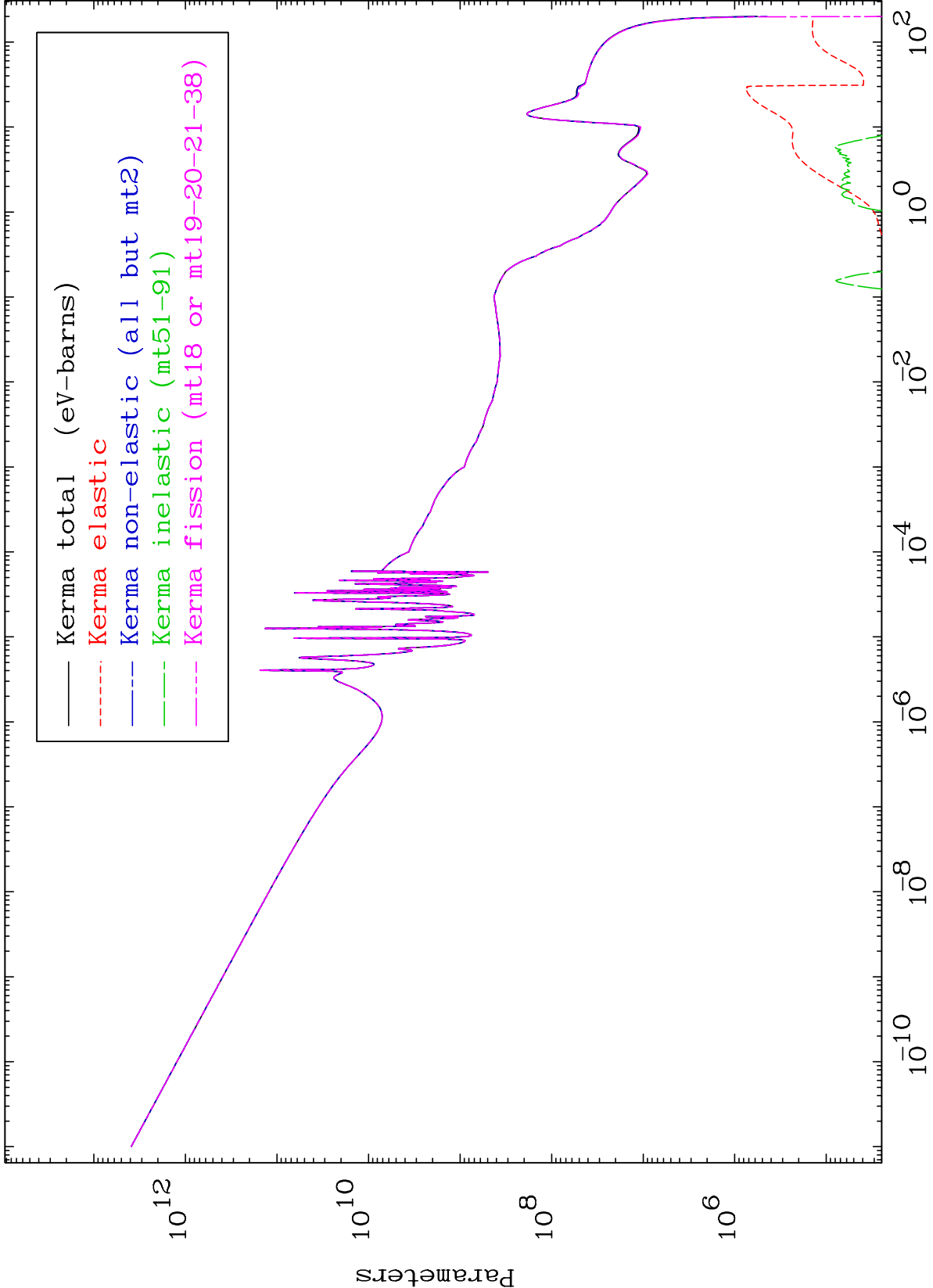
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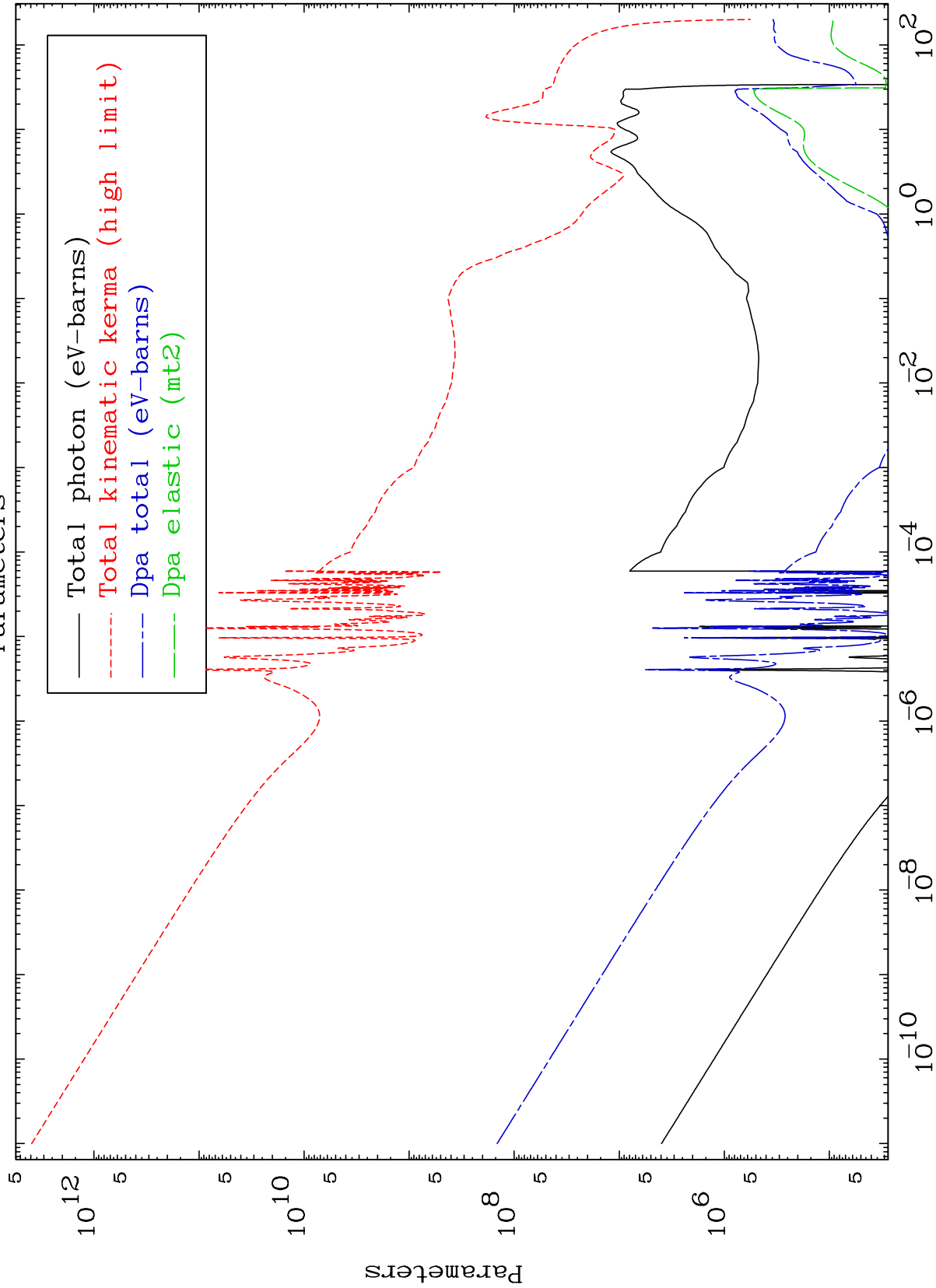
Incident Energy (MeV)

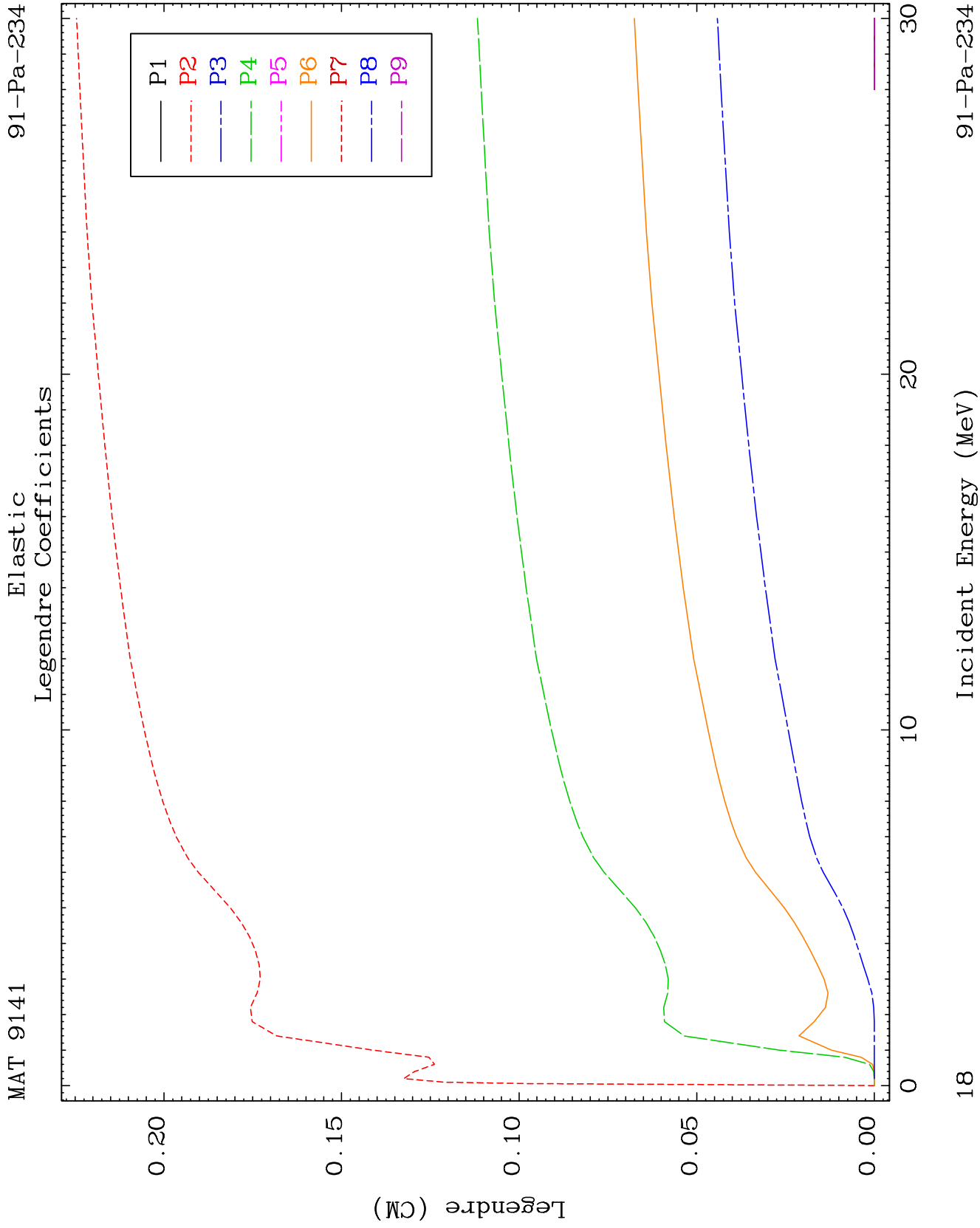
13

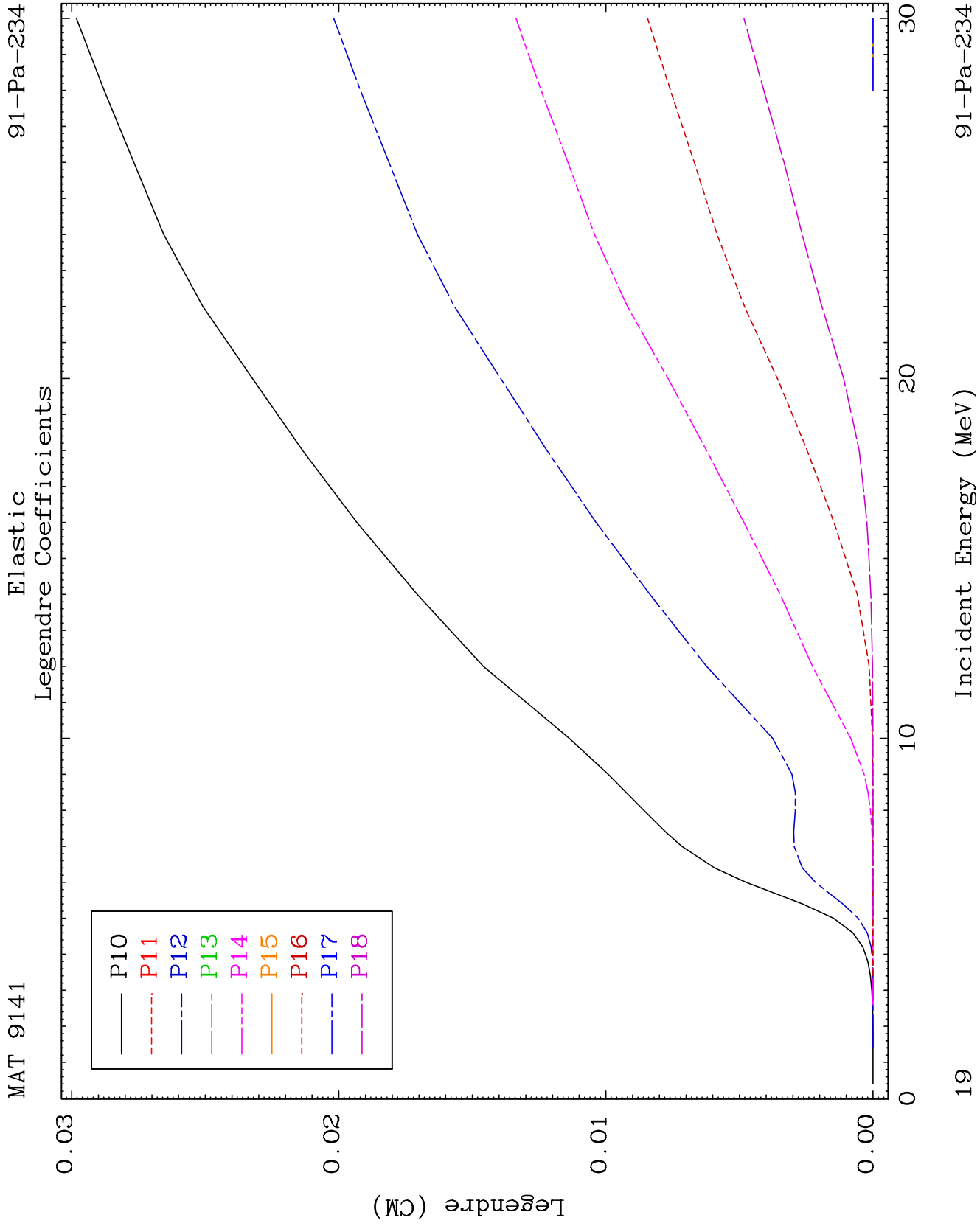


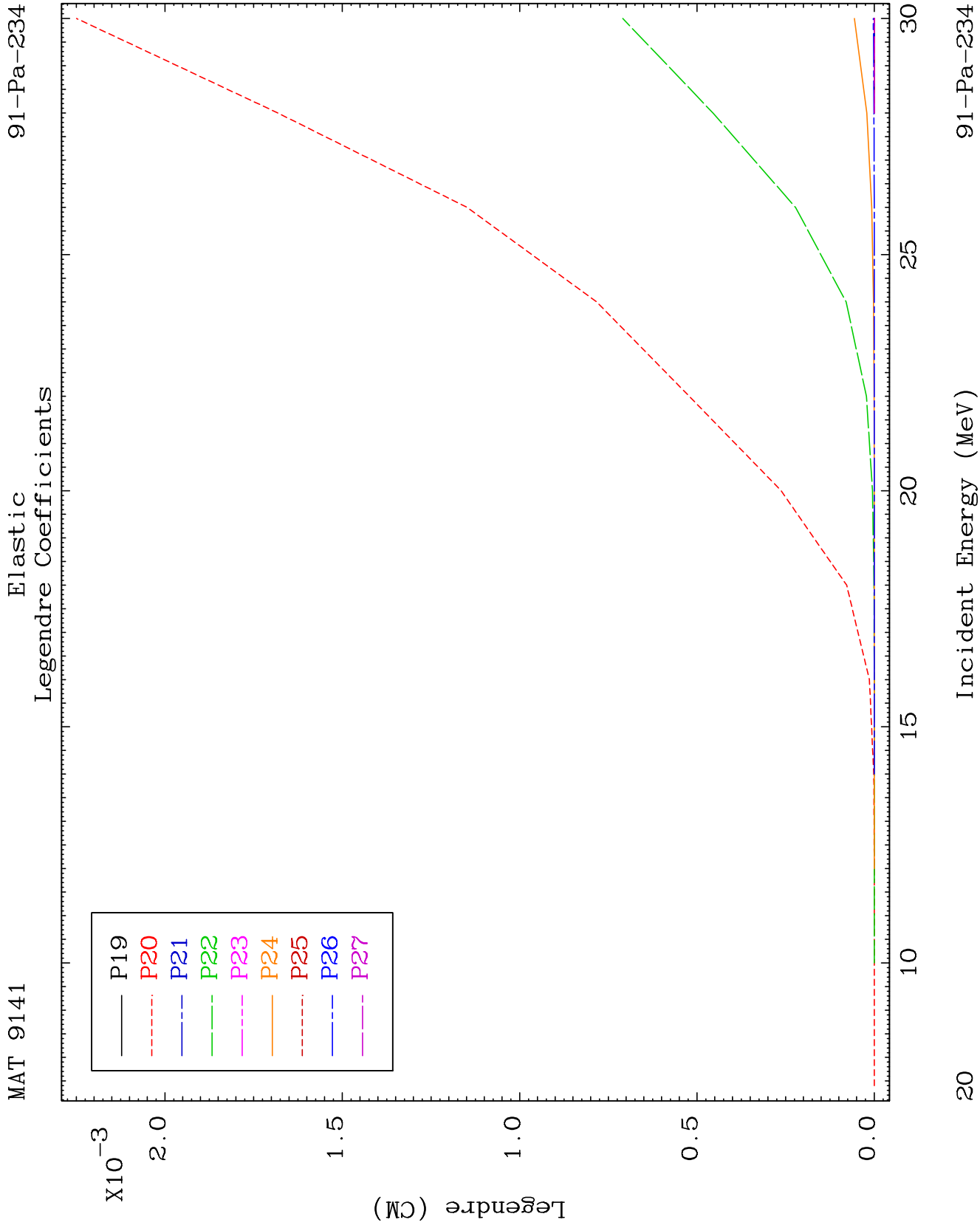


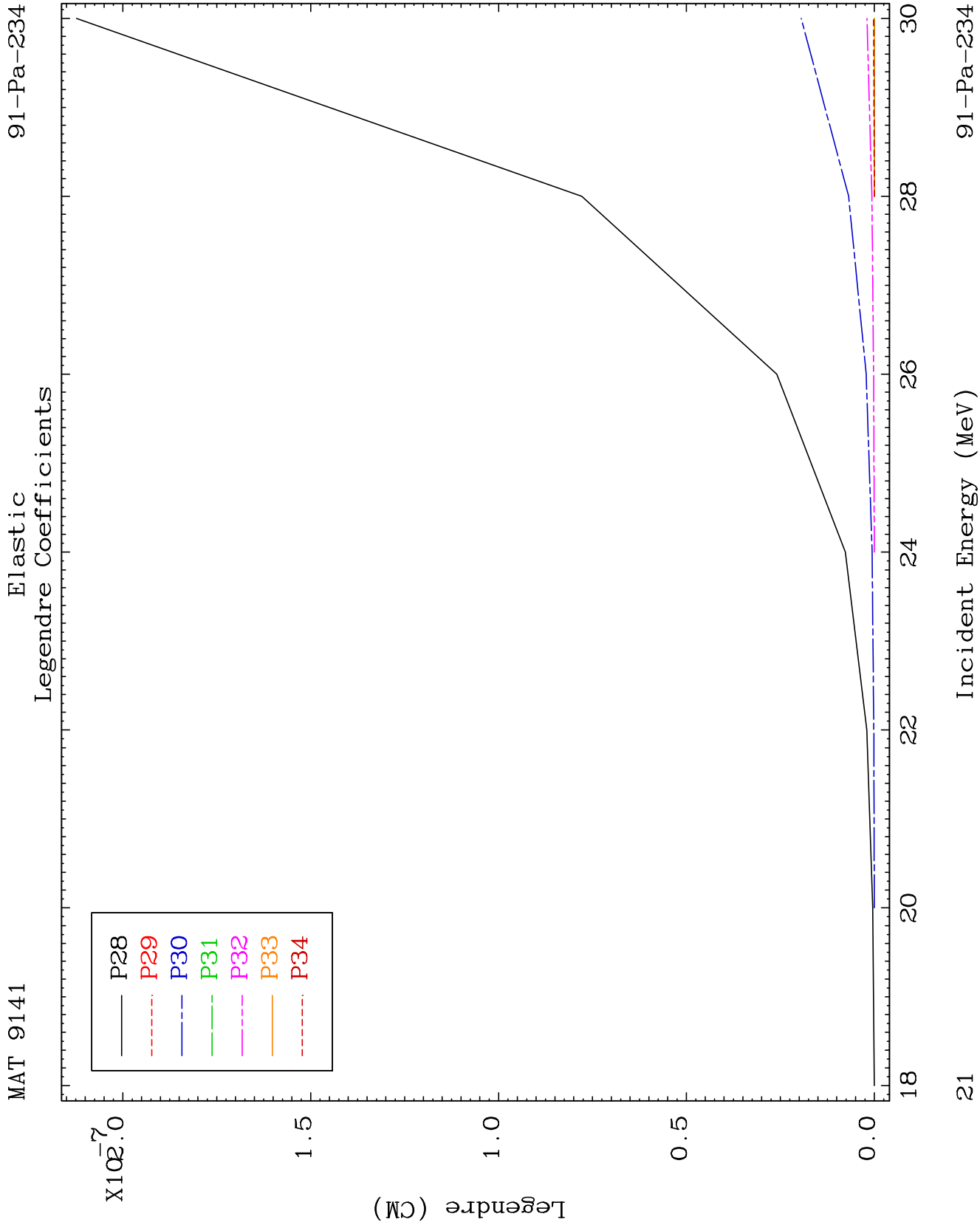










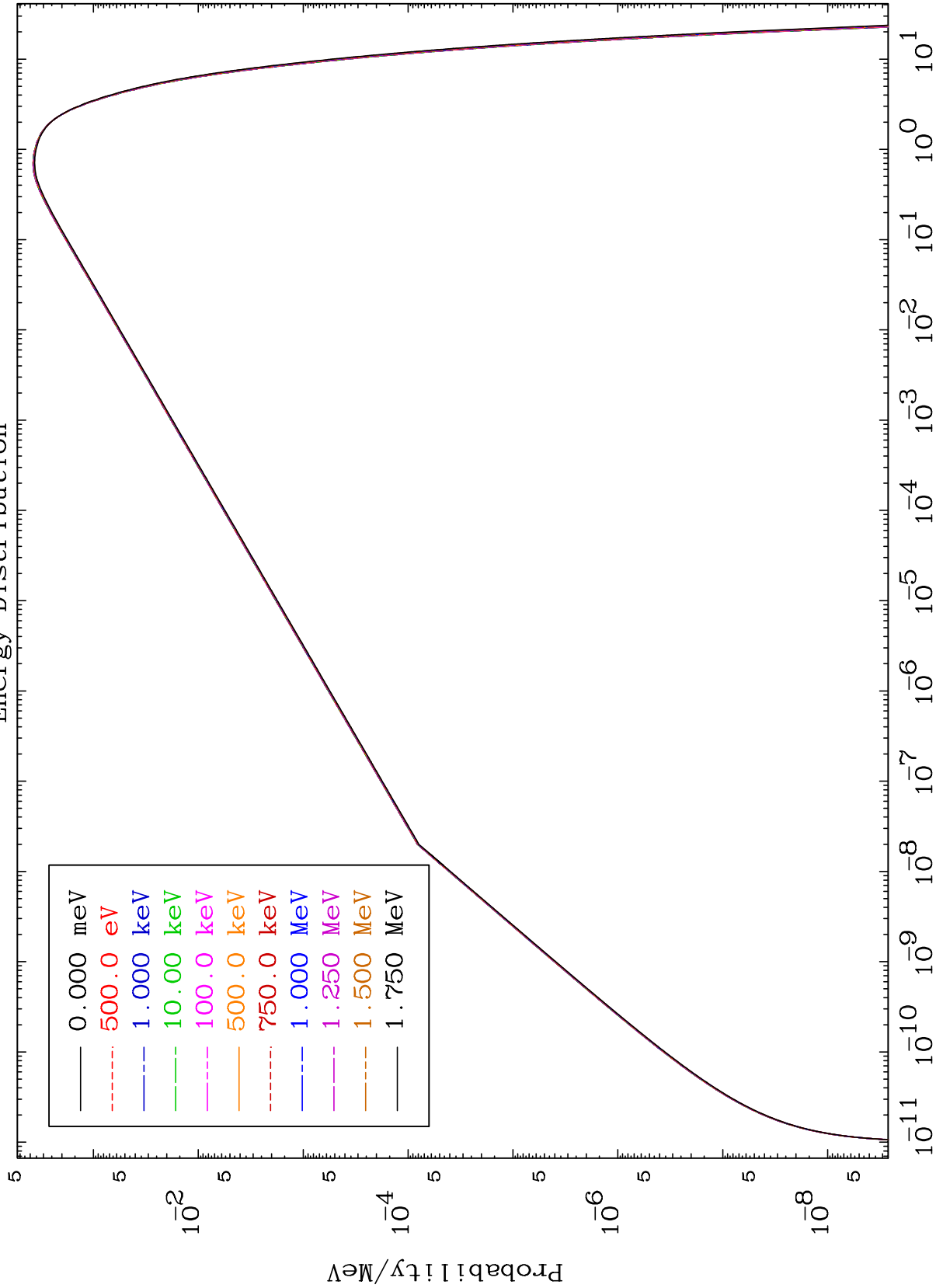


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Fission

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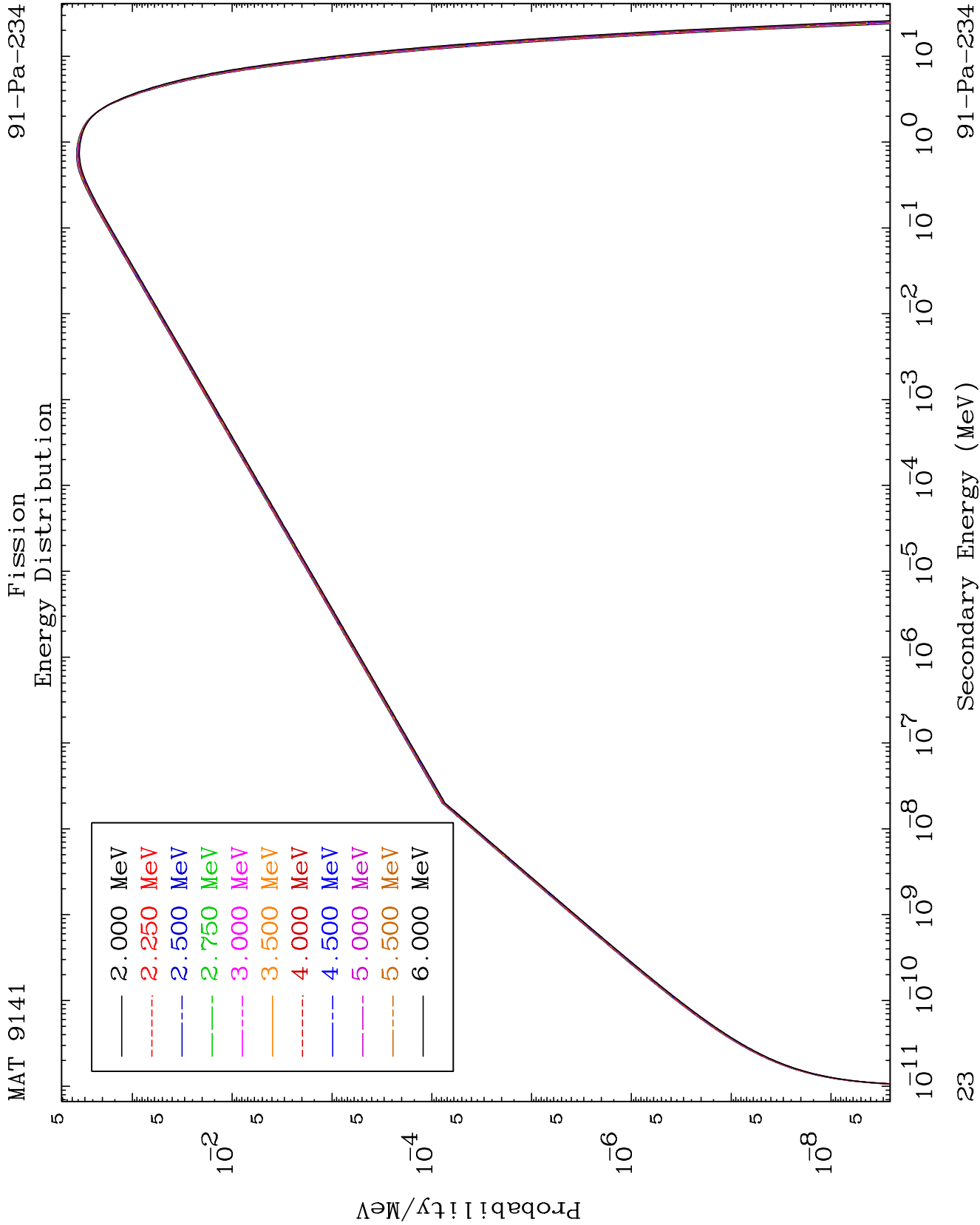
Energy Distribution



Secondary Energy (MeV)

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22

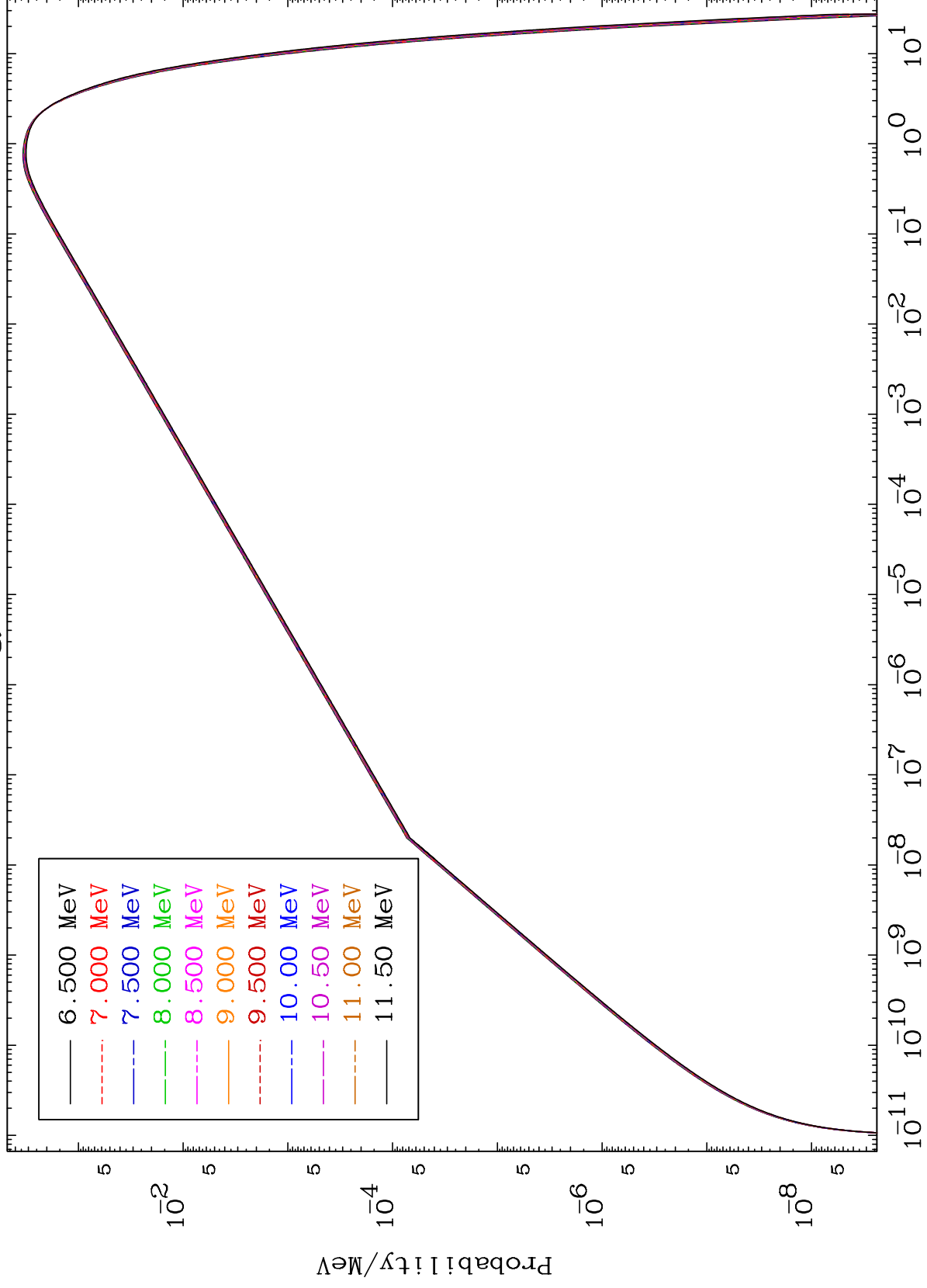


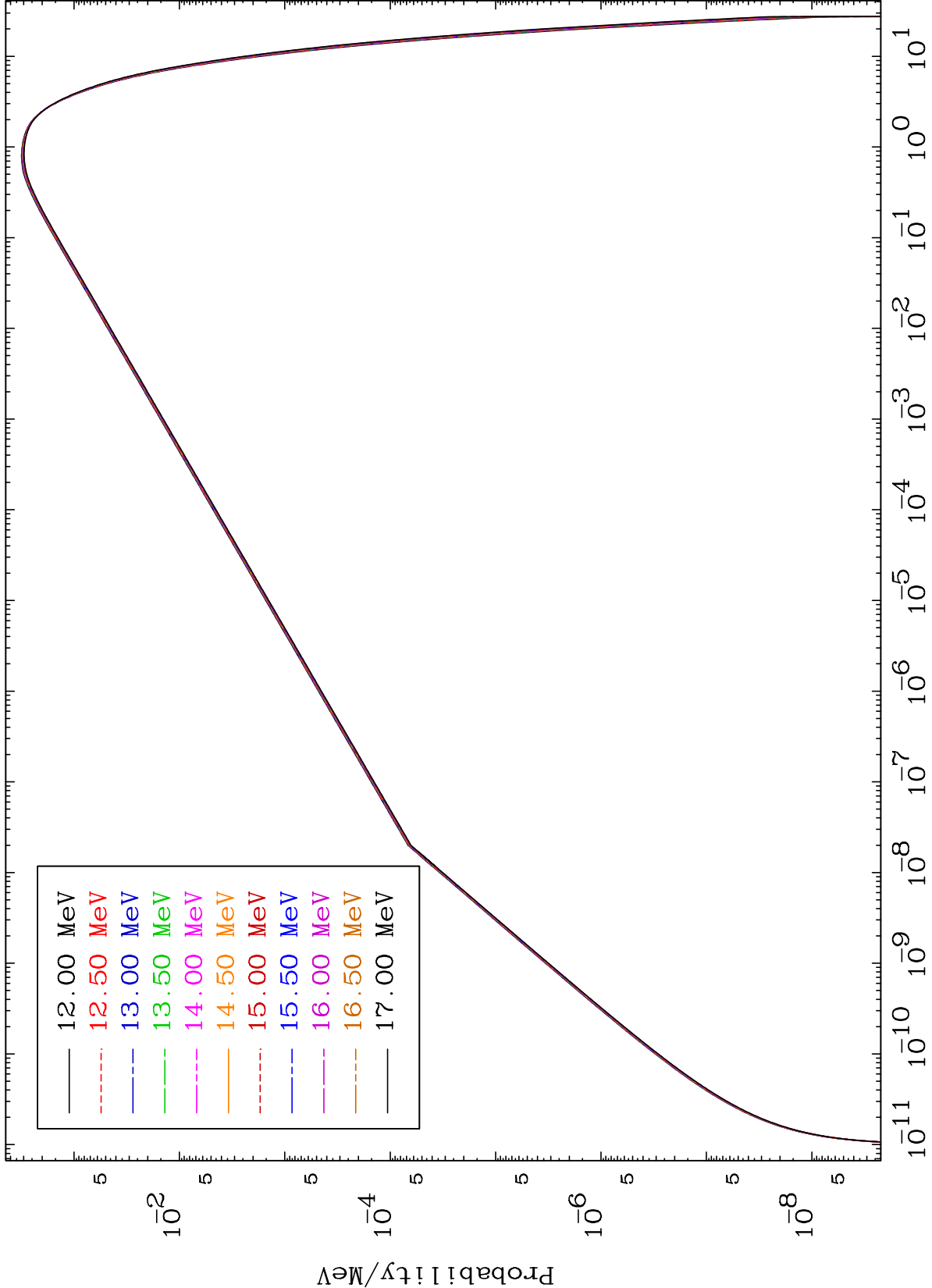
MAT 9141

Fission

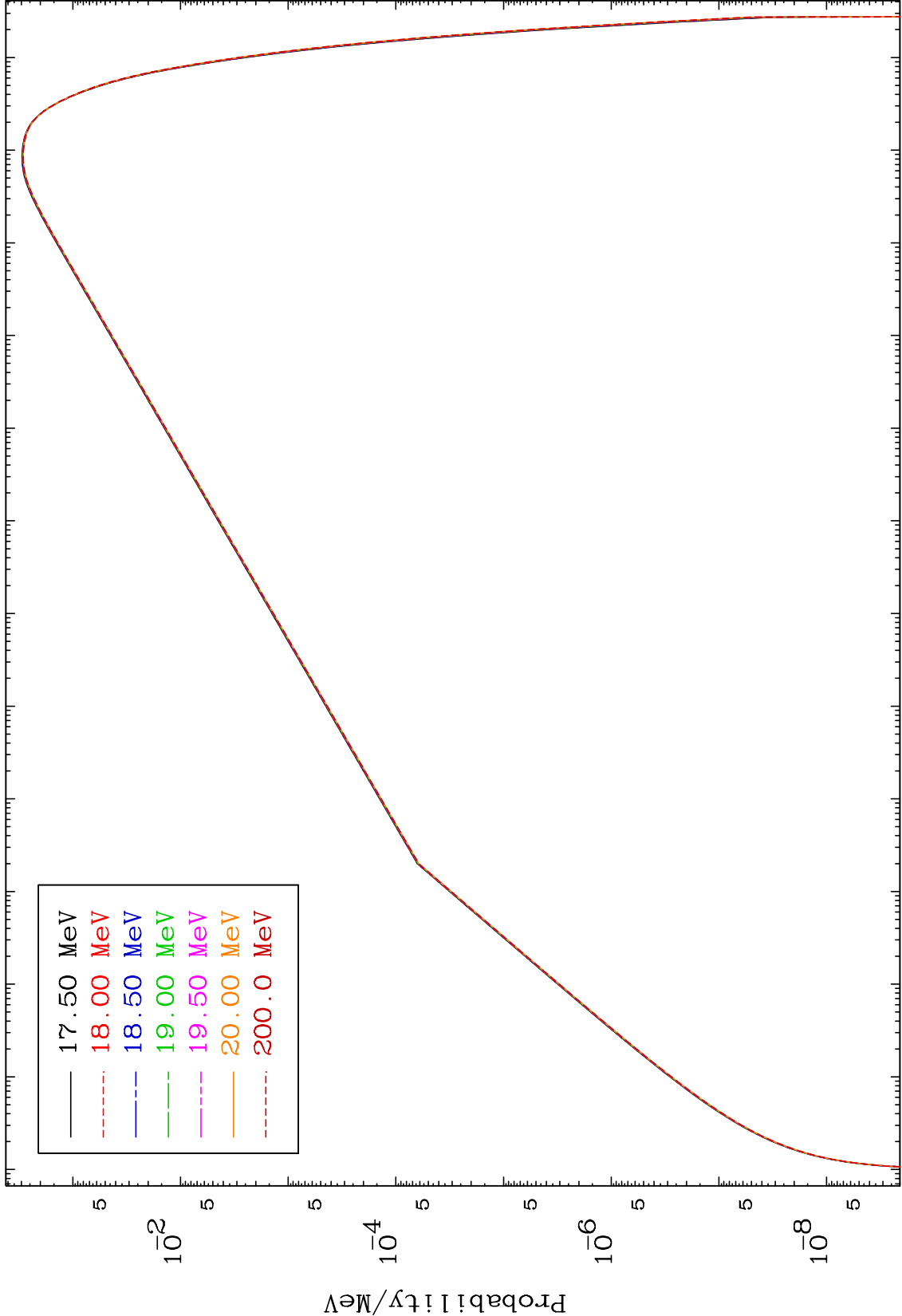
91-Pa-234

Energy Distribution





Fission
Energy Distribution



Radionuclide Production Cross Section

Inelastic

