

Program EVALPLOT  
(Version 2018-1)

by

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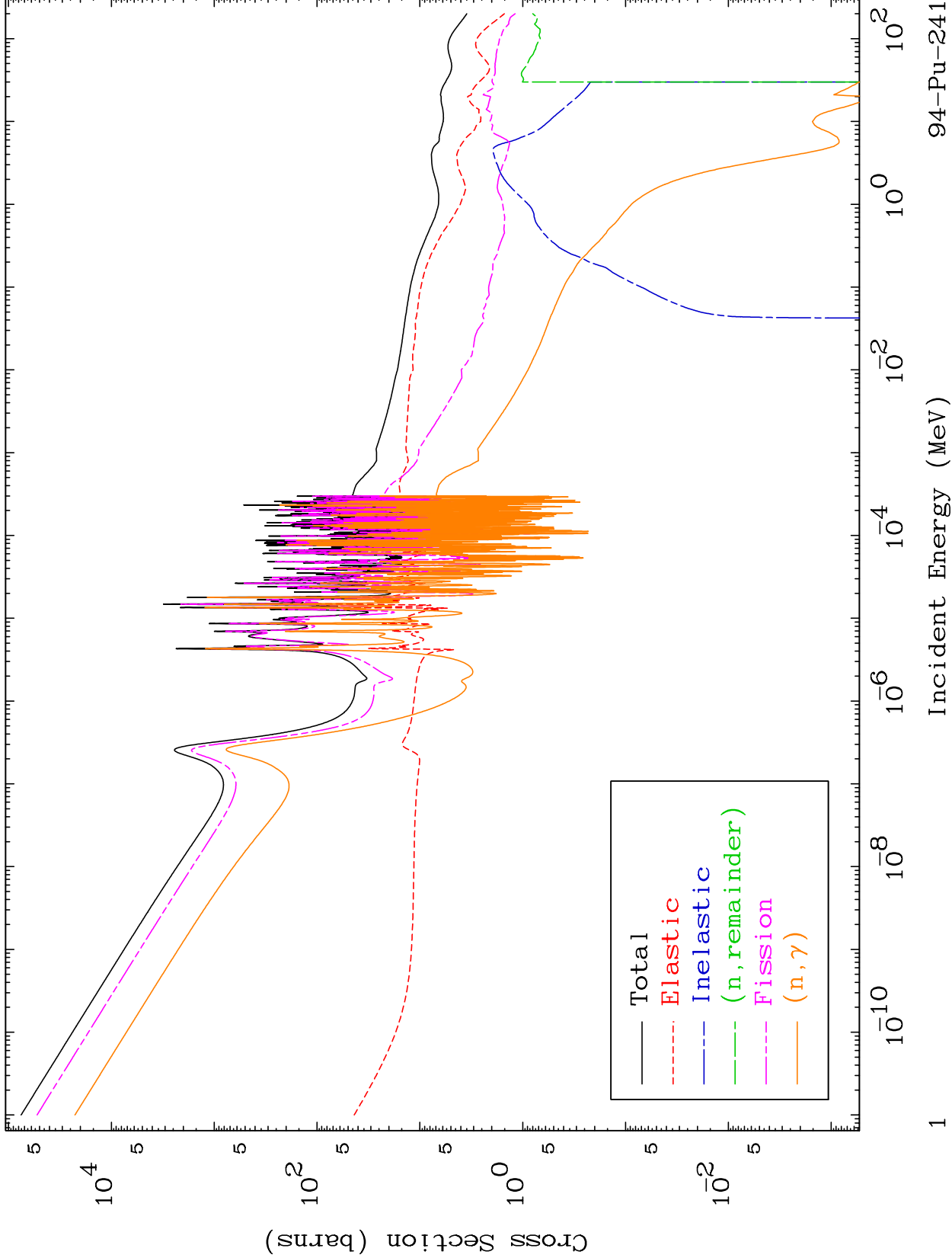
Press Mouse Button to Start

MAT 9443

Major

293 Kelvin Cross Sections

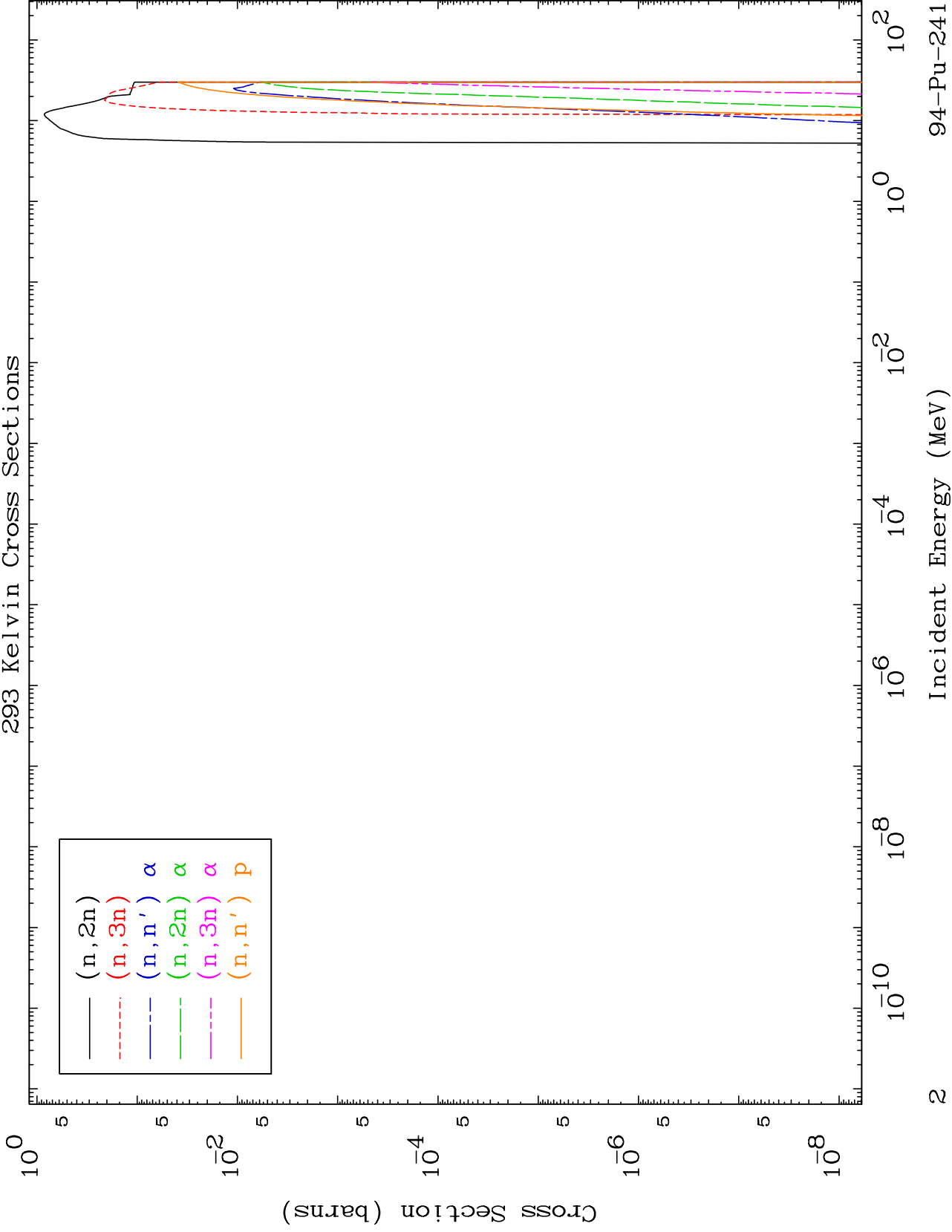
94-Pu-241



MAT 9443

Neutron Production  
293 Kelvin Cross Sections

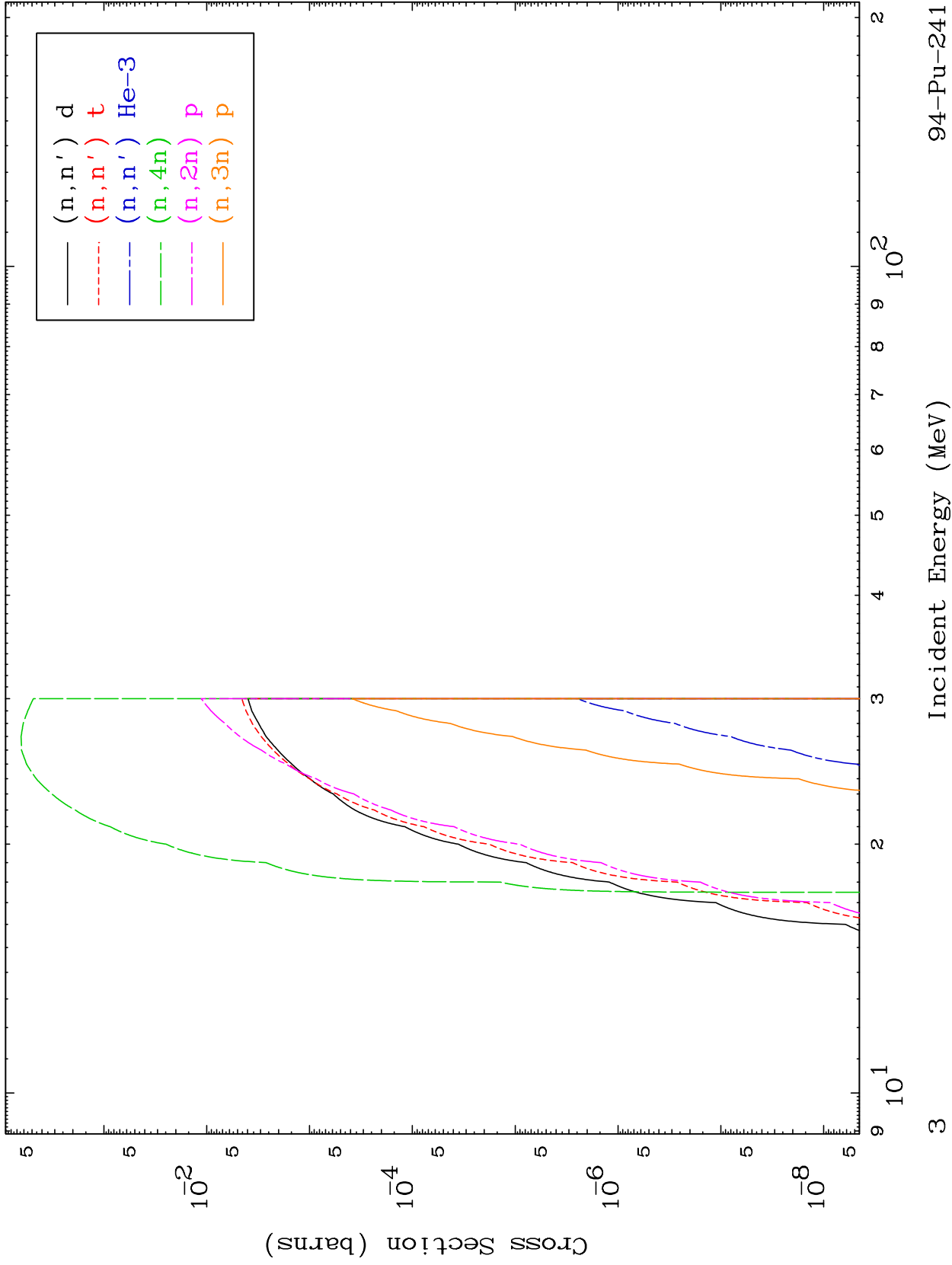
94-Pu-241



MAT 9443

Neutron Production  
293 Kelvin Cross Sections

94-Pu-241



94-Pu-241

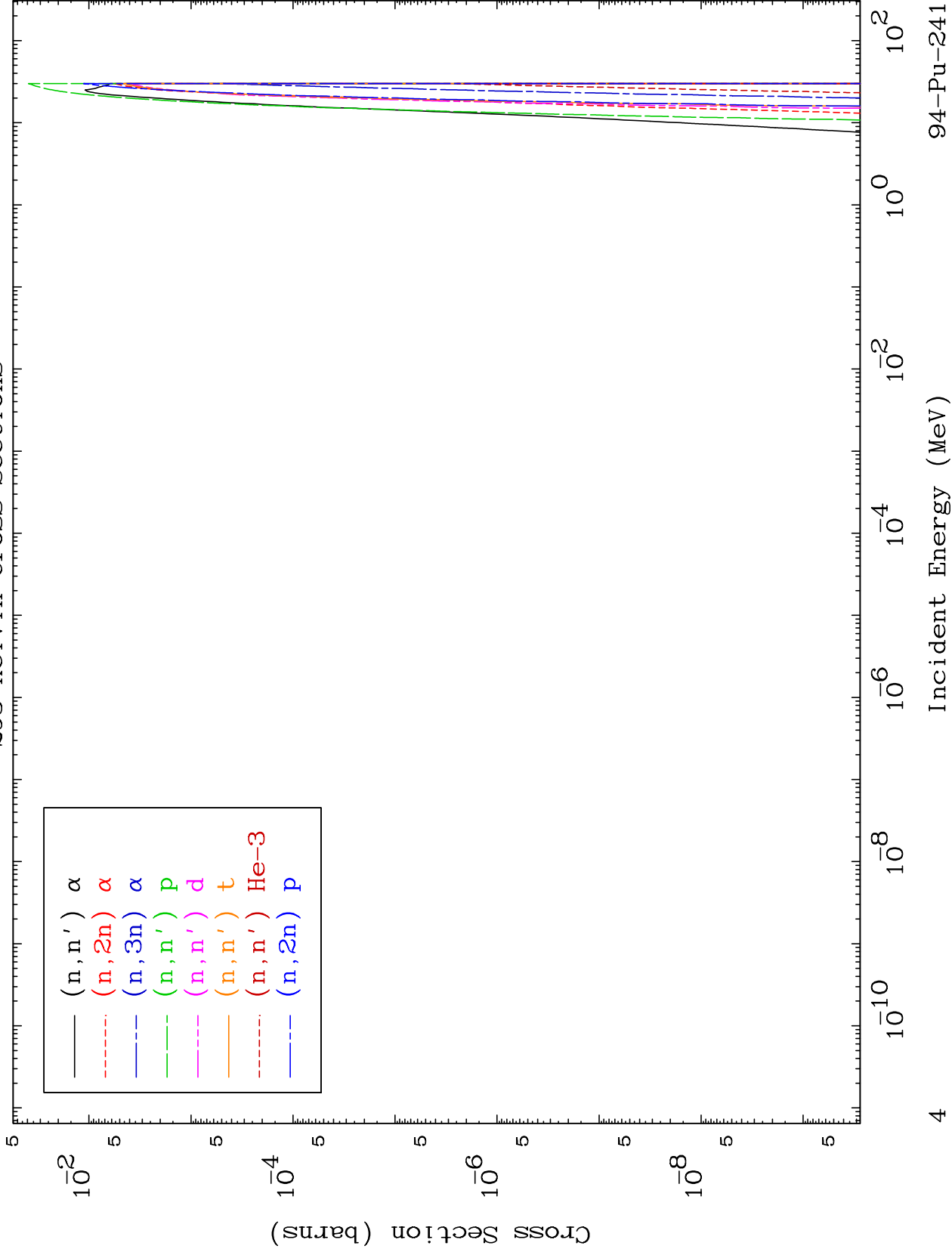
Incident Energy (MeV)

3

MAT 9443

Charged Particle  
293 Kelvin Cross Sections

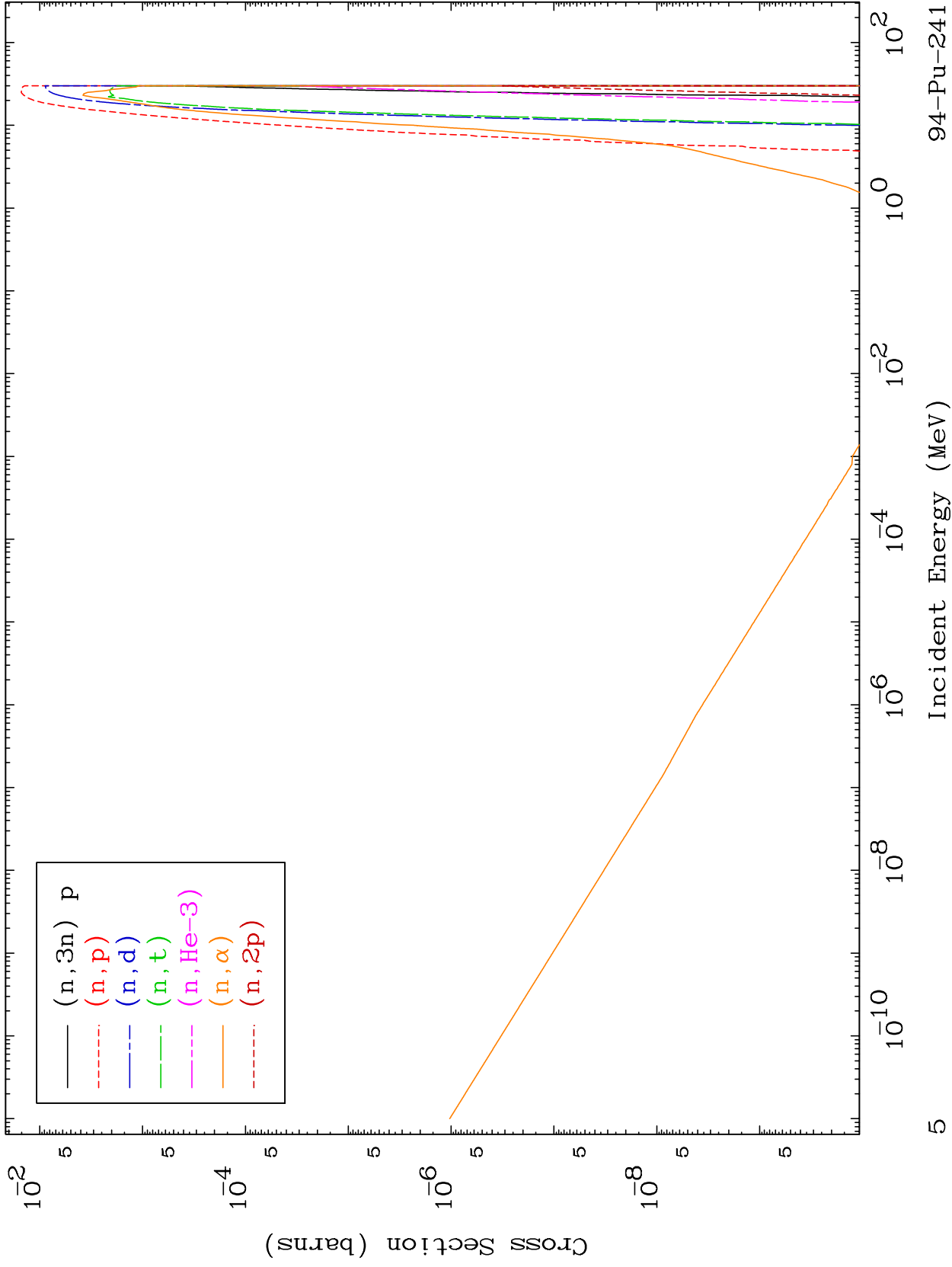
94-Pu-241



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Charged Particle  
293 Kelvin Cross Sections

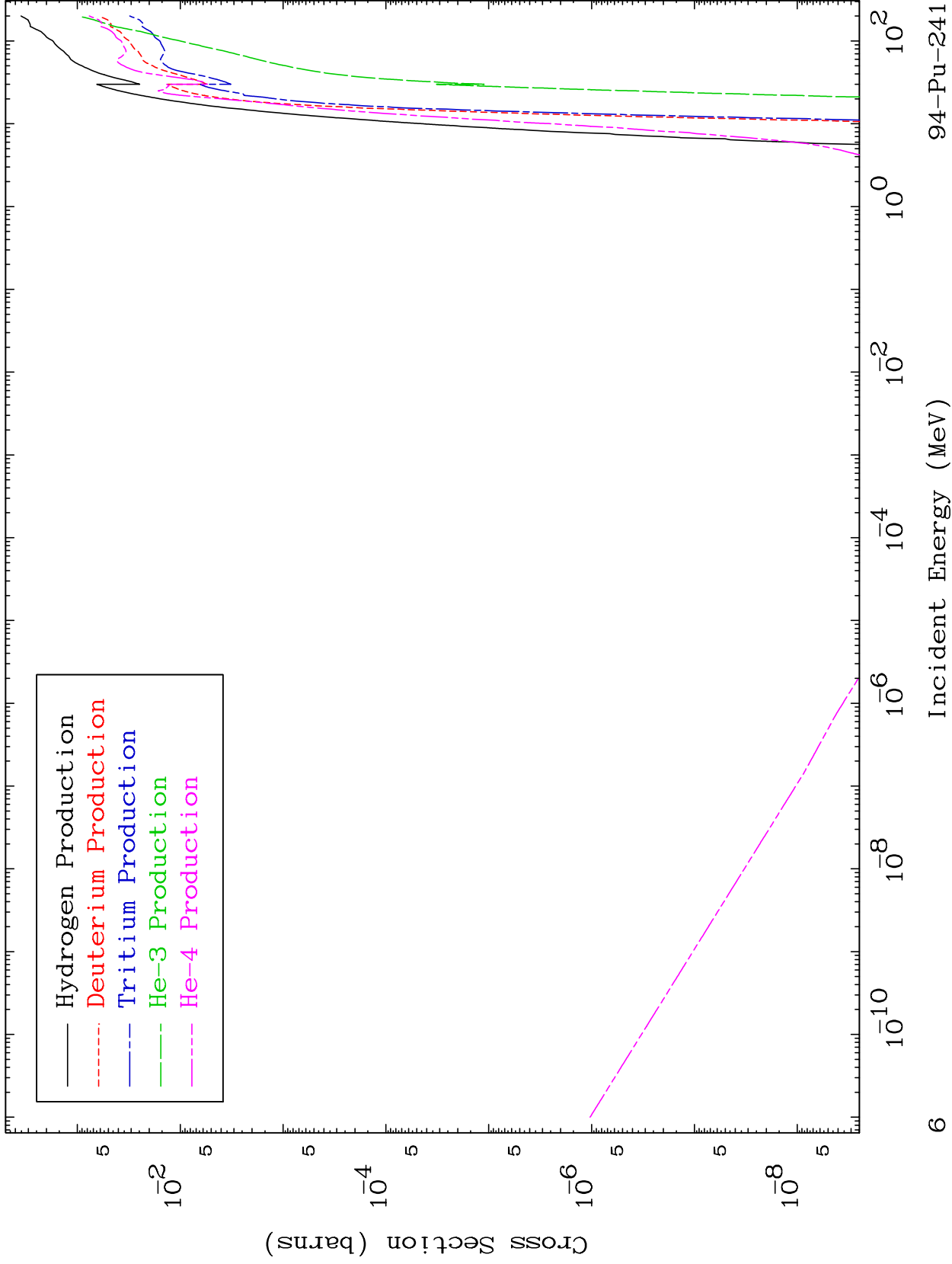
94-Pu-241



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Particle Production  
293 Kelvin Cross Sections

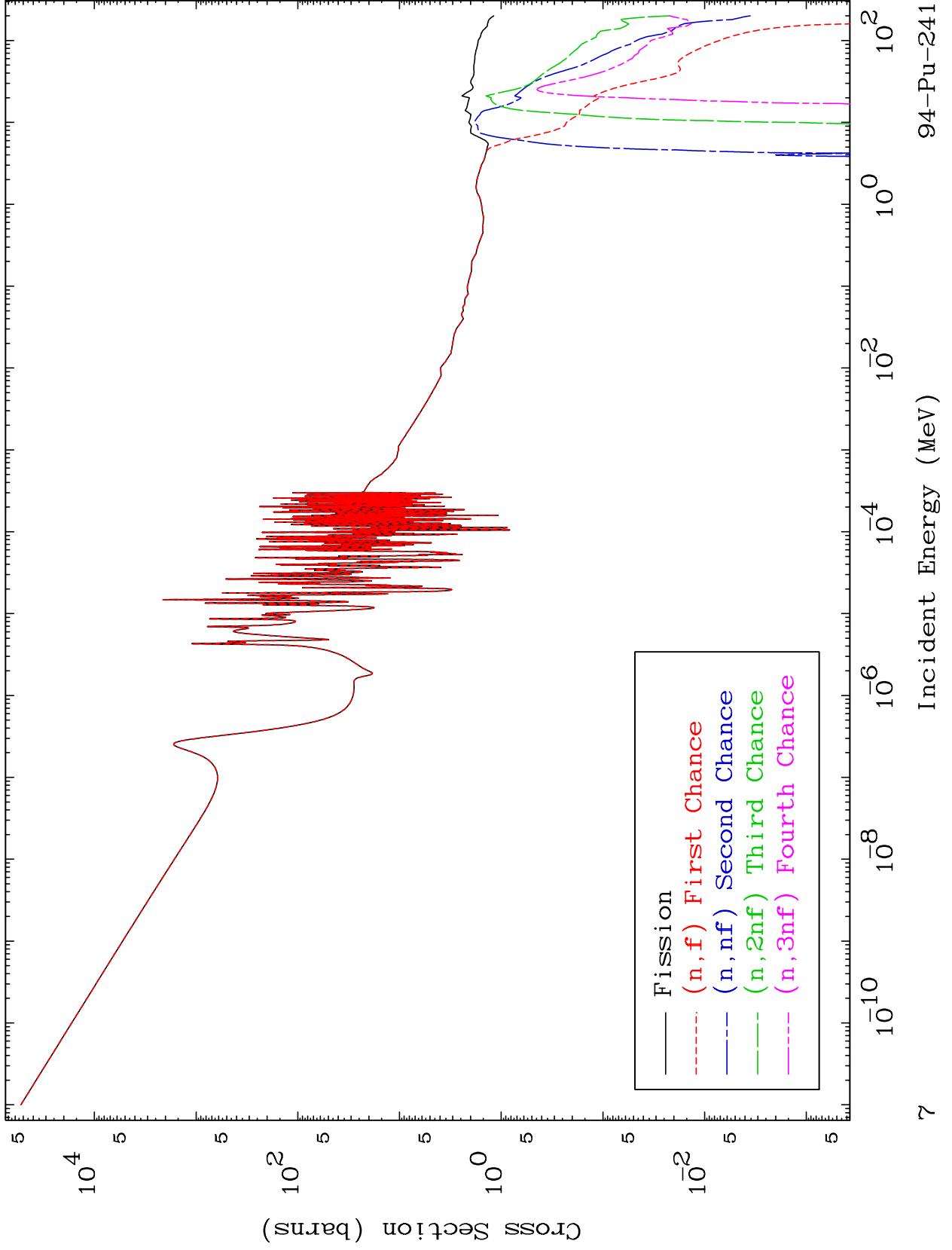
94-Pu-241



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Fission  
293 Kelvin Cross Sections

94-Pu-241



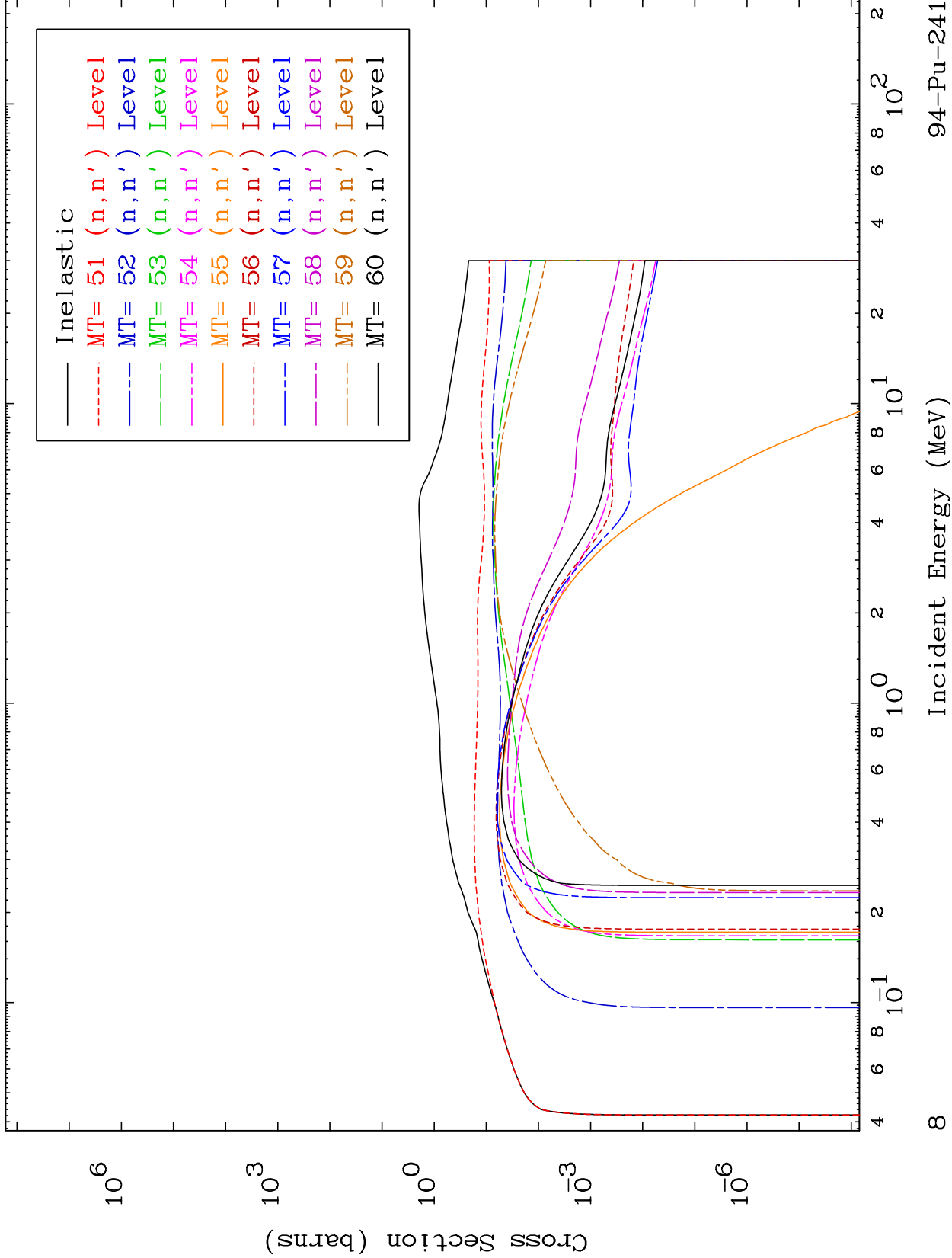


MAT 9443

(n,n') Level

94-Pu-241

293 Kelvin Cross Sections

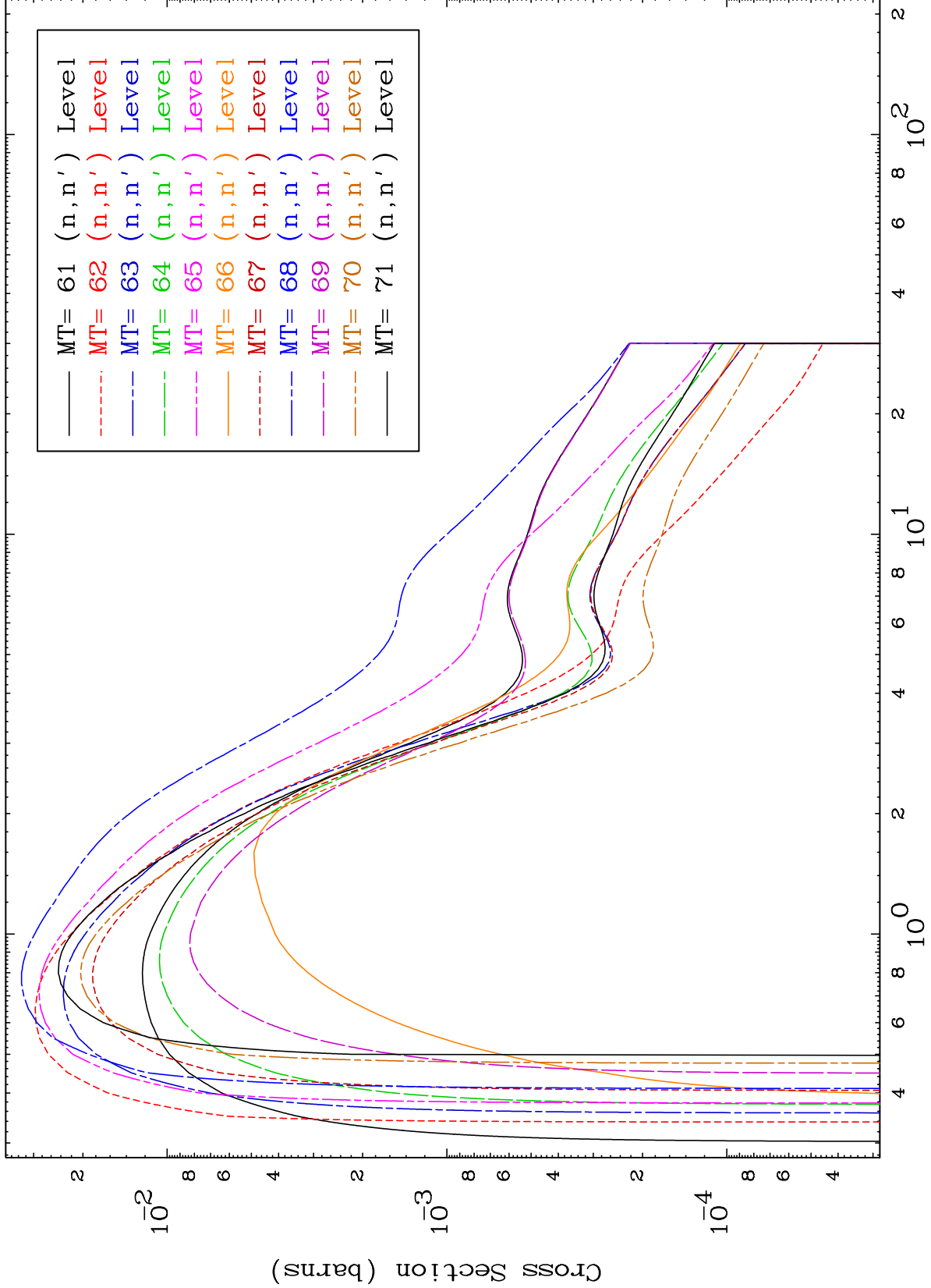


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(n,n') Level

94-Pu-241

293 Kelvin Cross Sections



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Incident Energy (MeV)

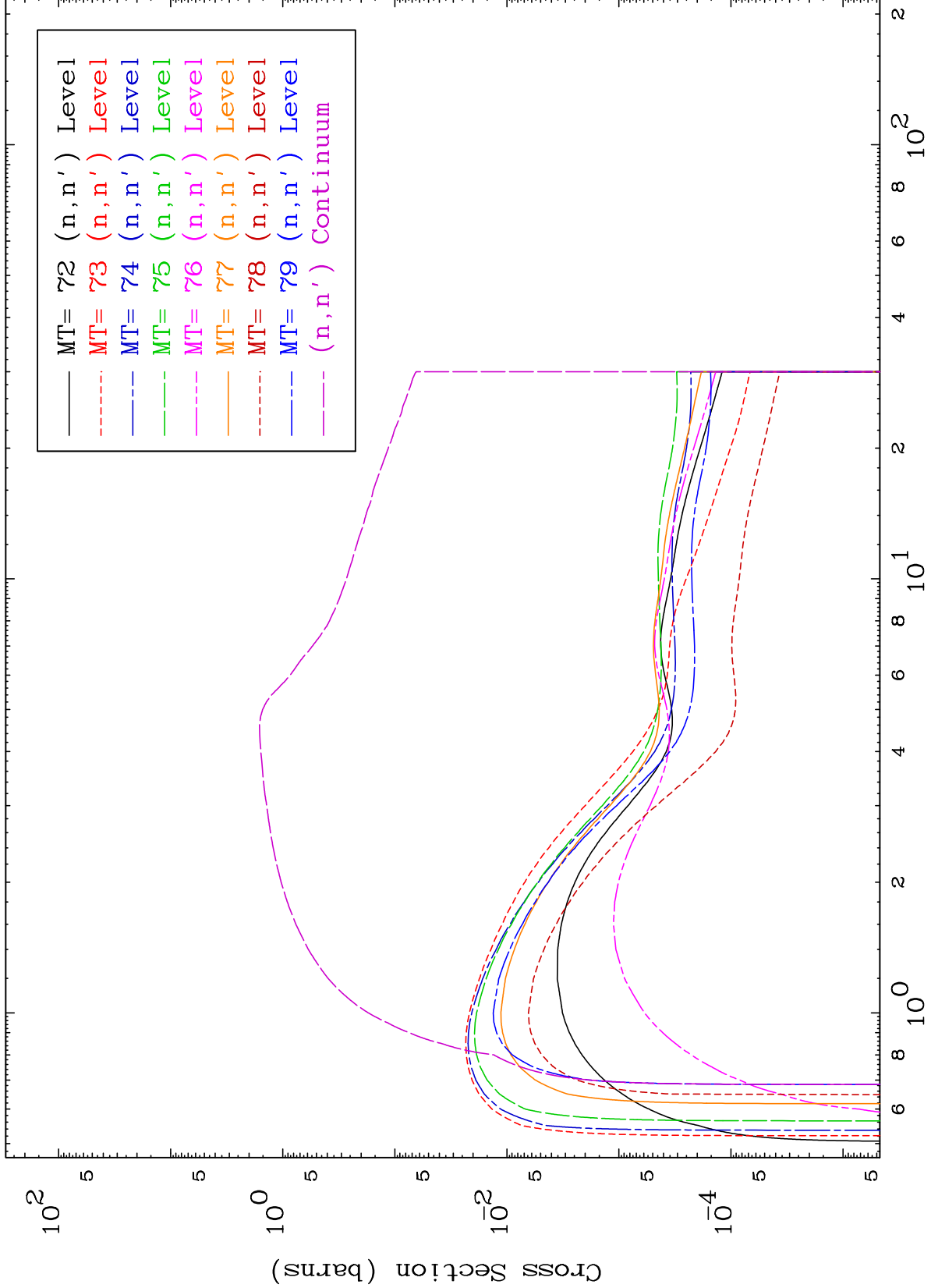
94-Pu-241

MAT 9443

(n,n') Level

94-Pu-241

293 Kelvin Cross Sections



10

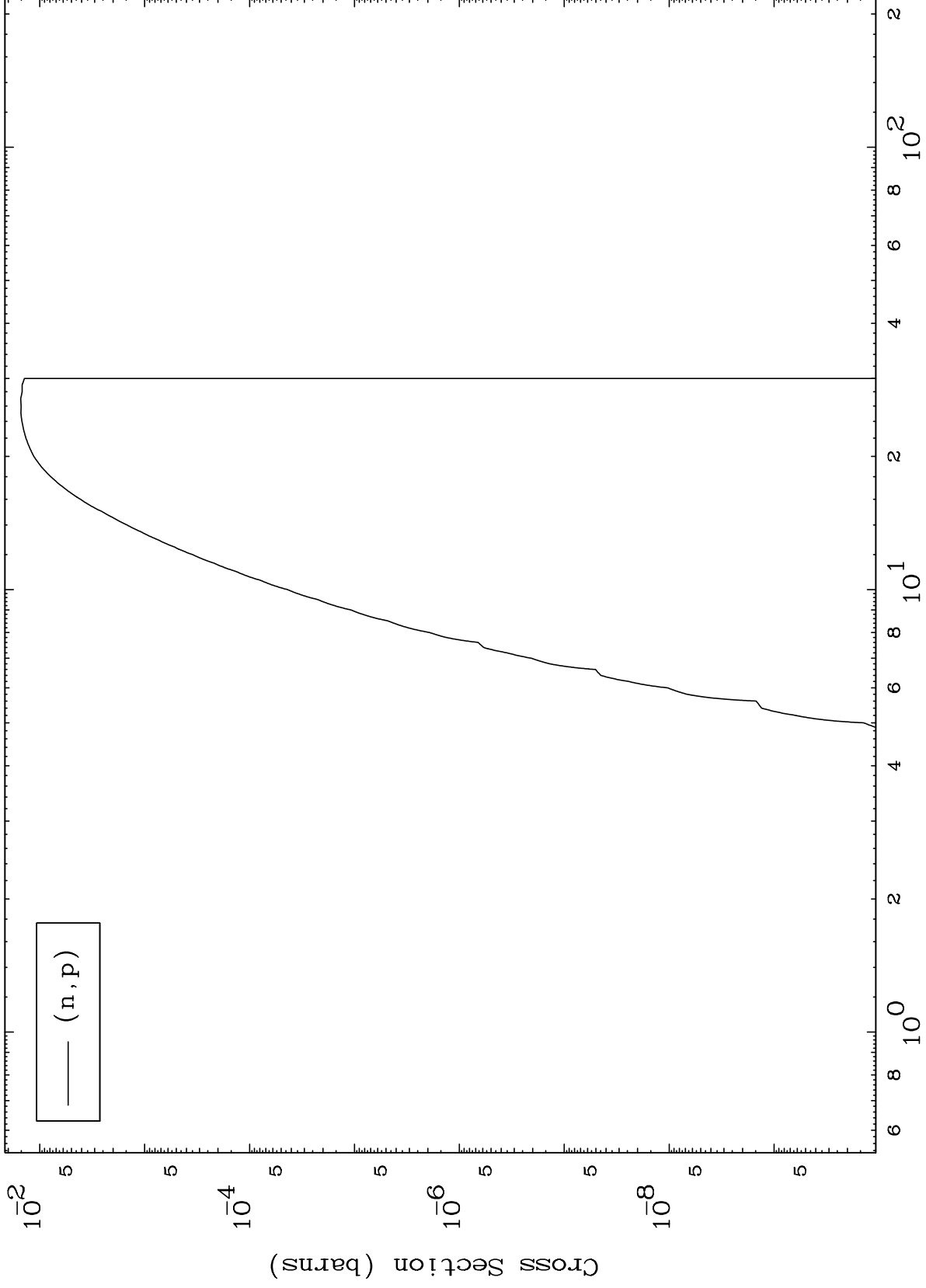
Incident Energy (MeV)

94-Pu-241

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(n,p) Levels  
293 Kelvin Cross Sections

94-Pu-241



11

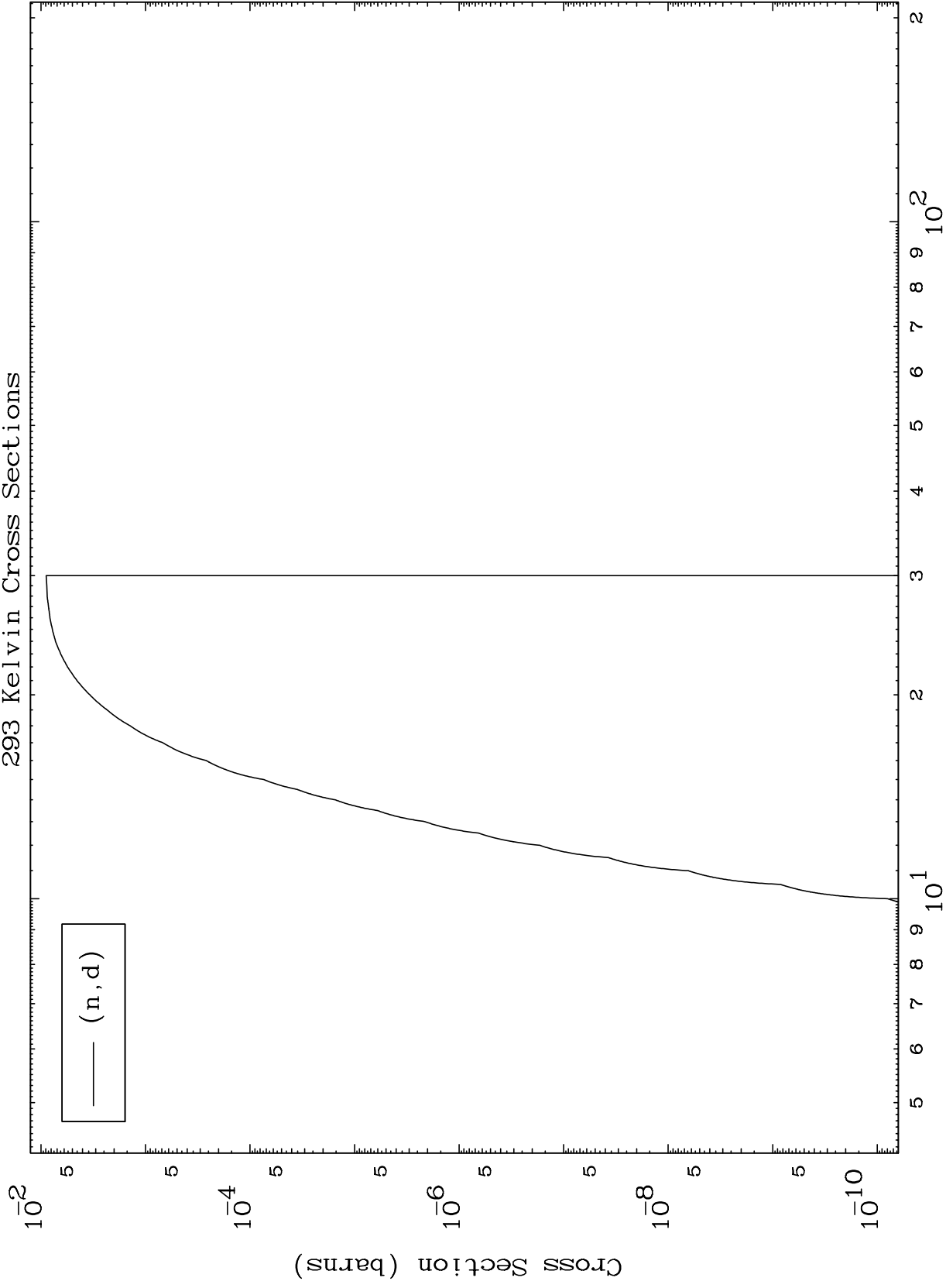
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,d) Levels  
293 Kelvin Cross Sections

94-Pu-241



12

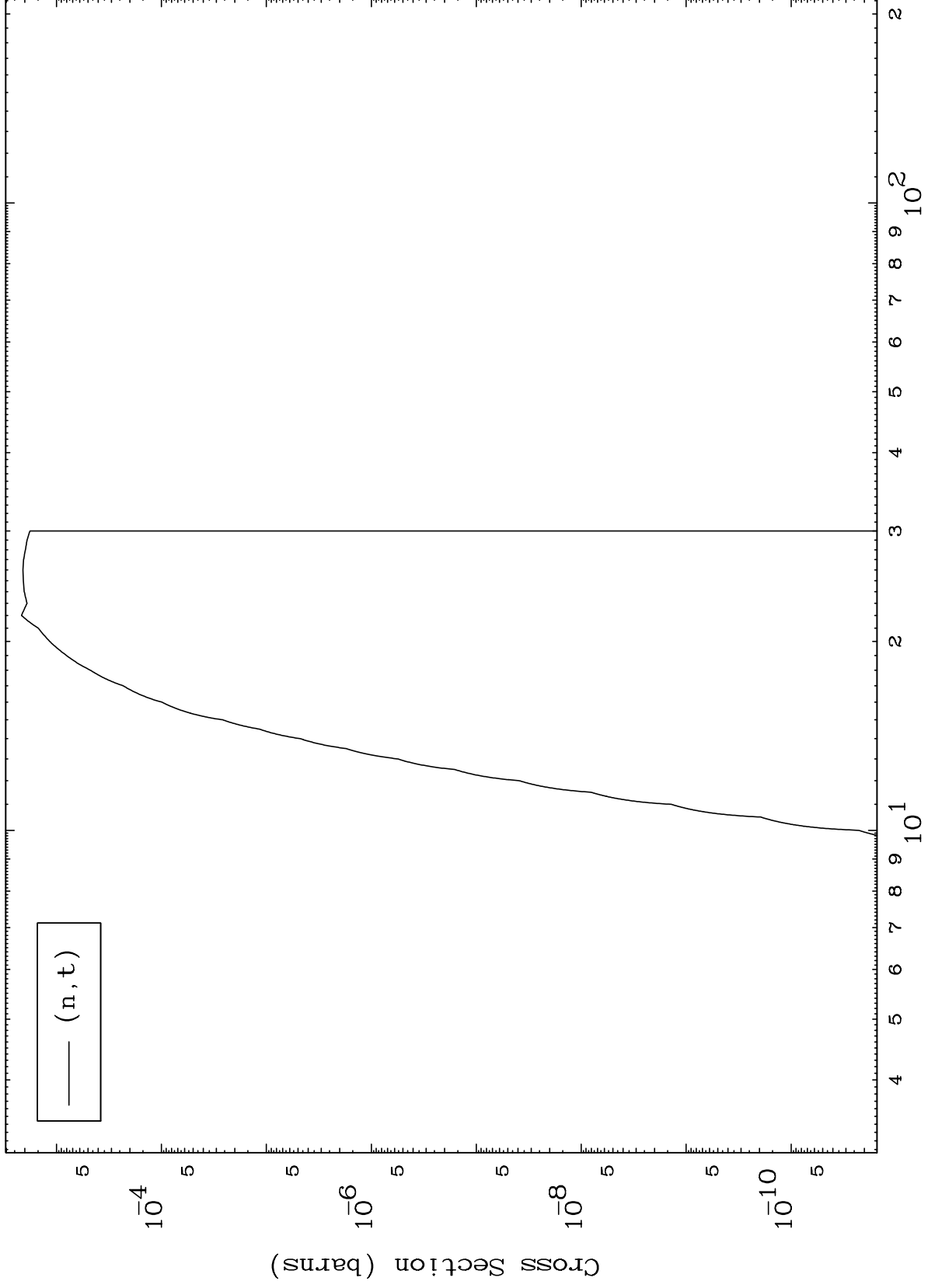
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,t) Levels  
293 Kelvin Cross Sections

94-Pu-241



13

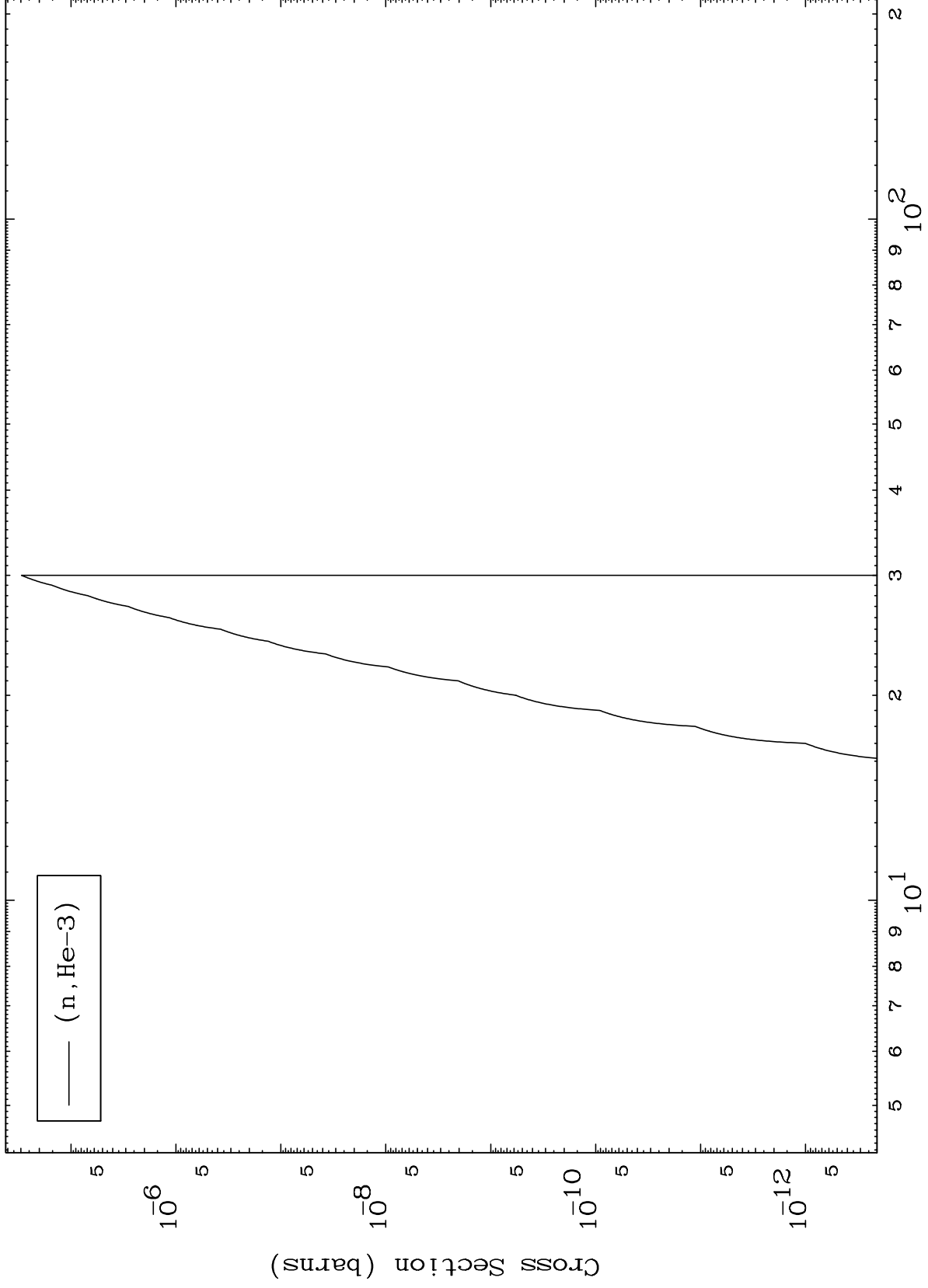
Incident Energy (MeV)

94-Pu-241

MAT 9443

(n,He3) Levels  
293 Kelvin Cross Sections

94-Pu-241



14

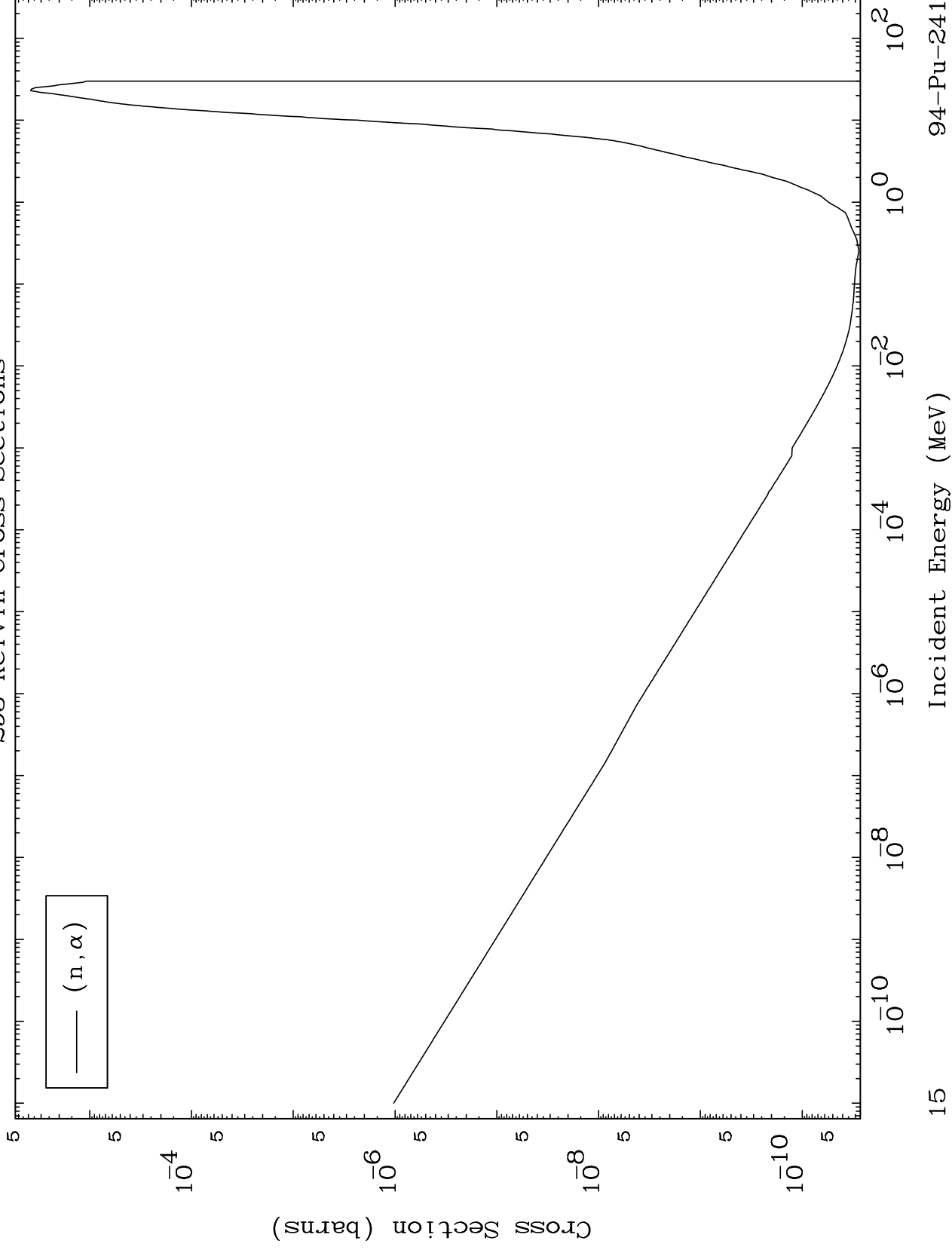
Incident Energy (MeV)

94-Pu-241

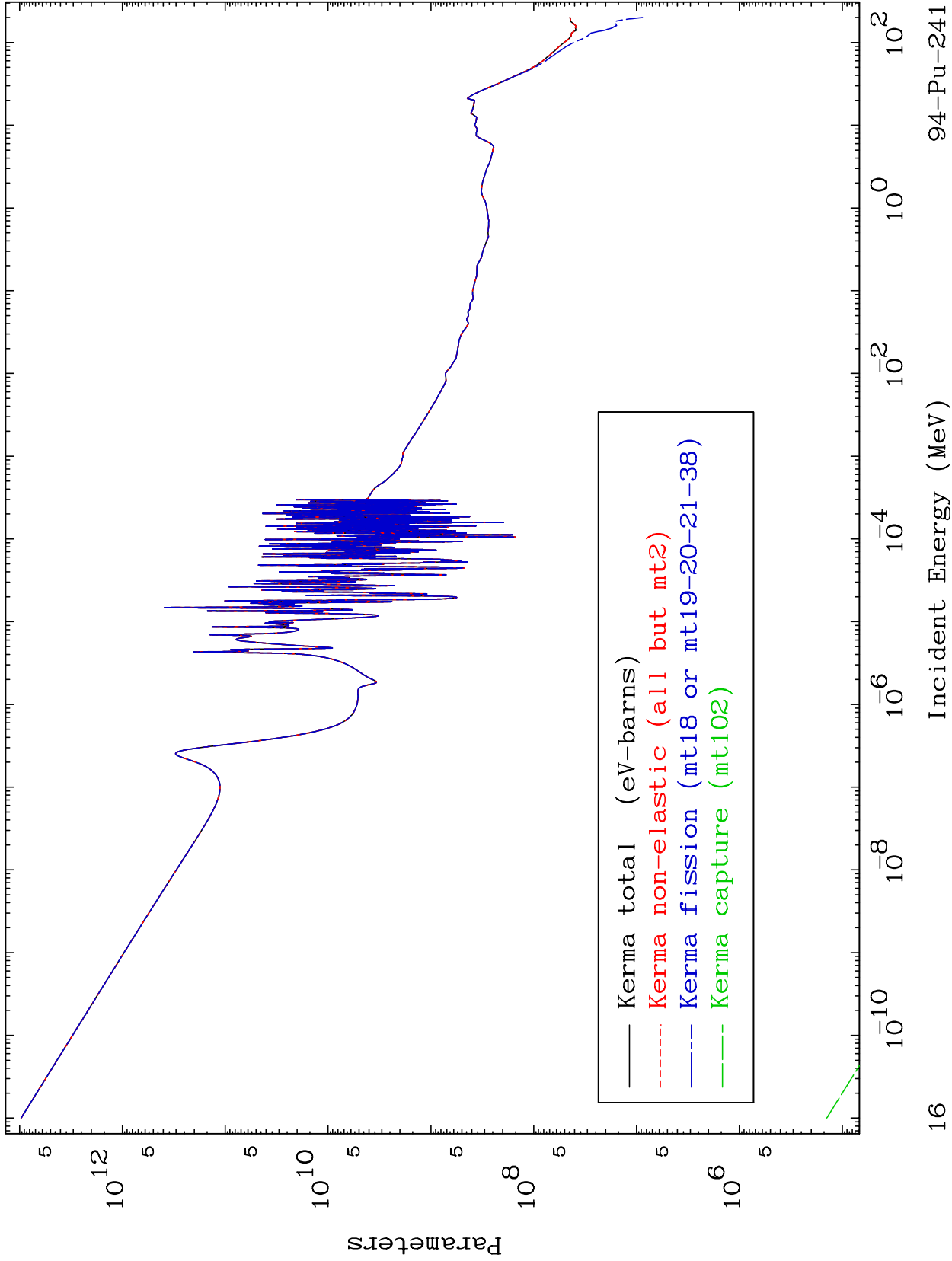
MAT 9443

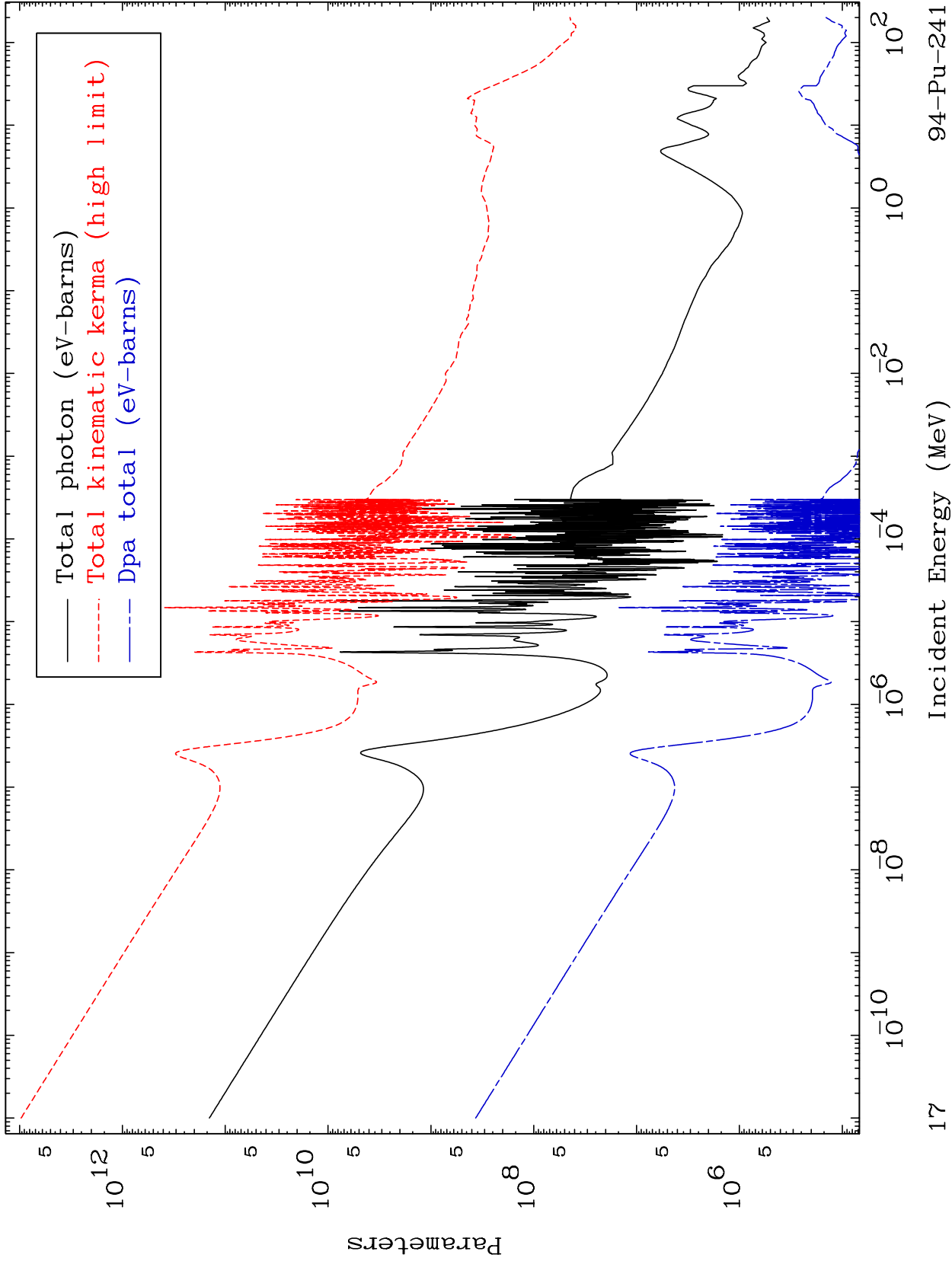
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

94-Pu-241





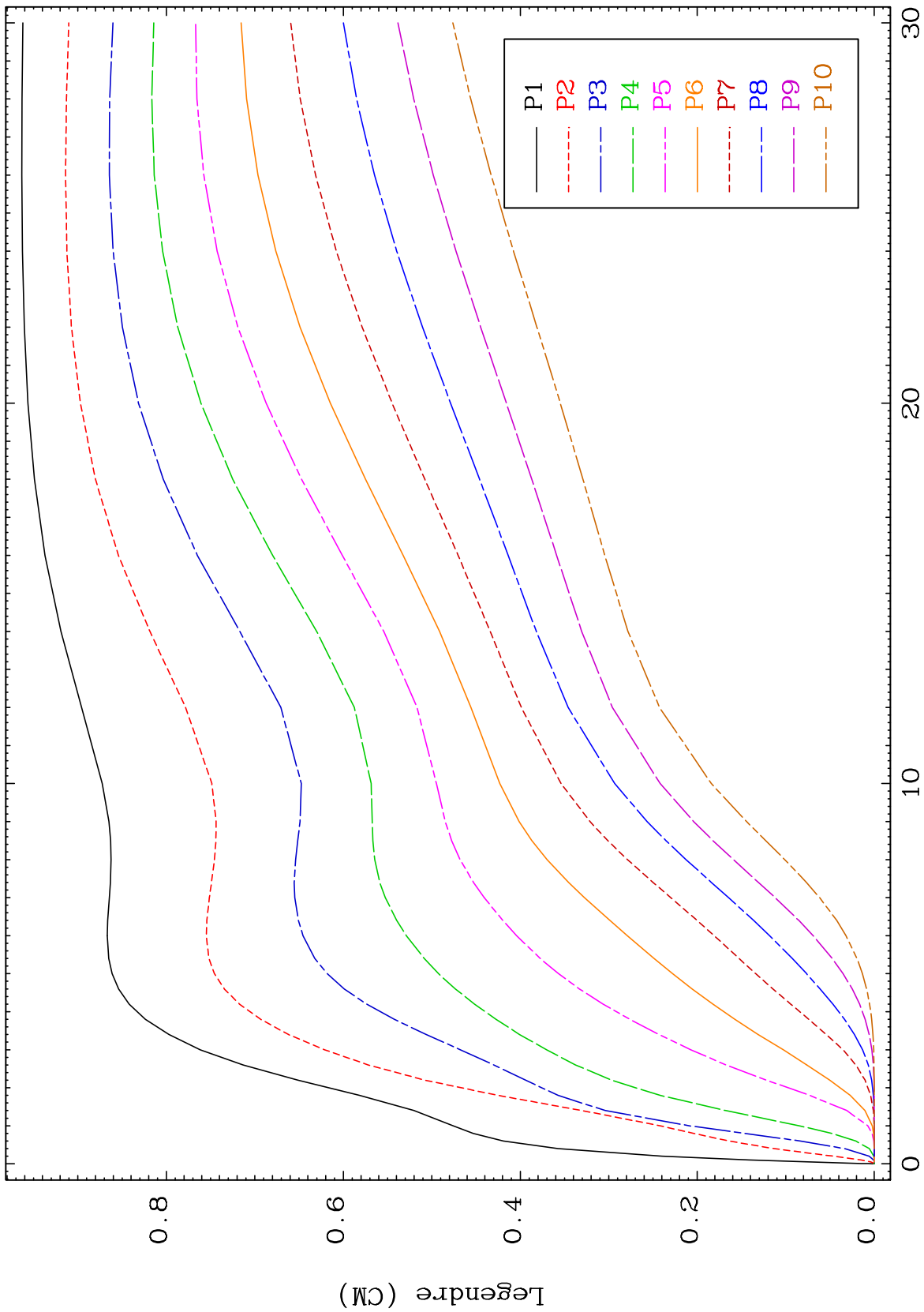




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Elastic Legendre Coefficients



94-Pu-241

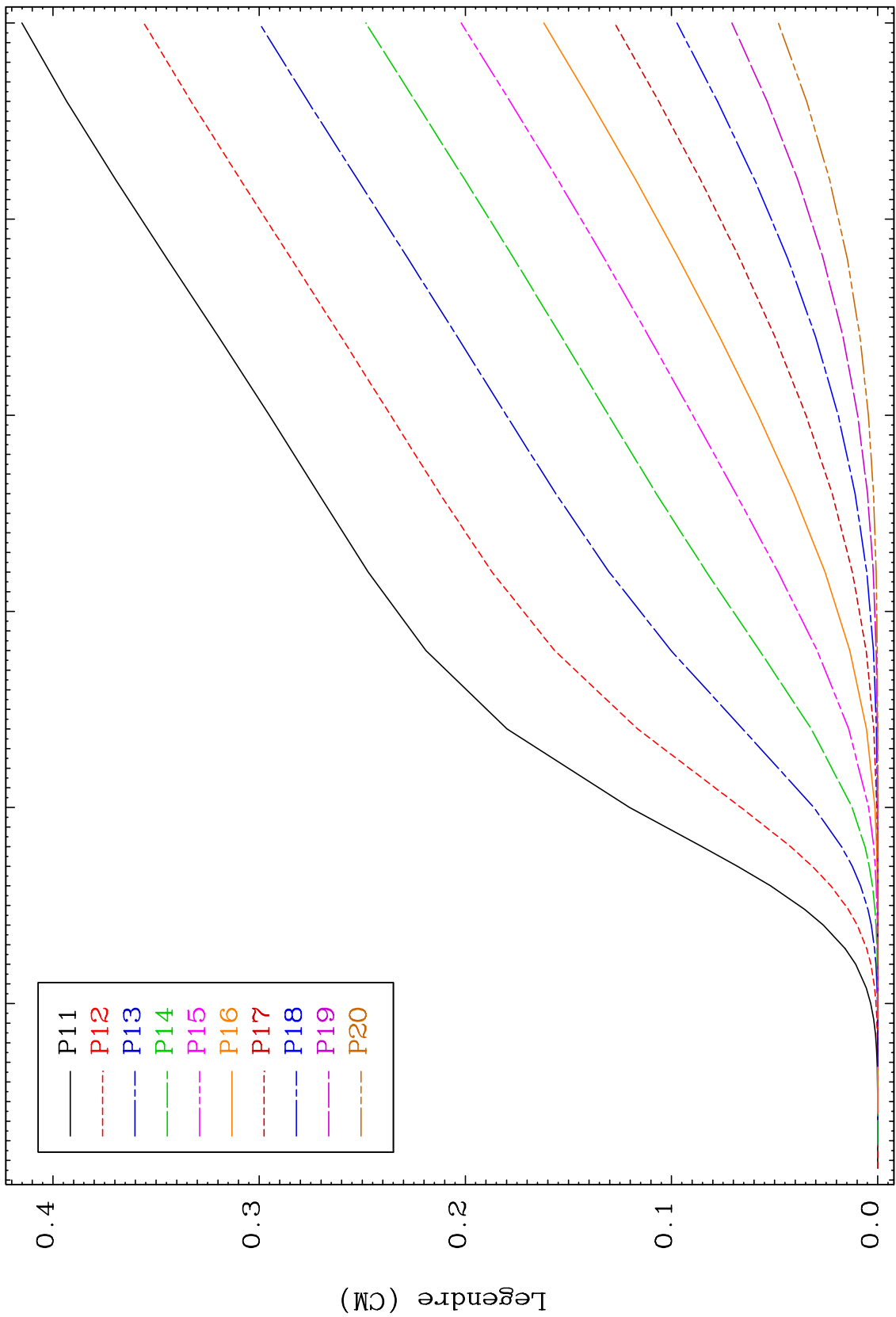
Incident Energy (MeV)

18

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Elastic Legendre Coefficients

94-Pu-241



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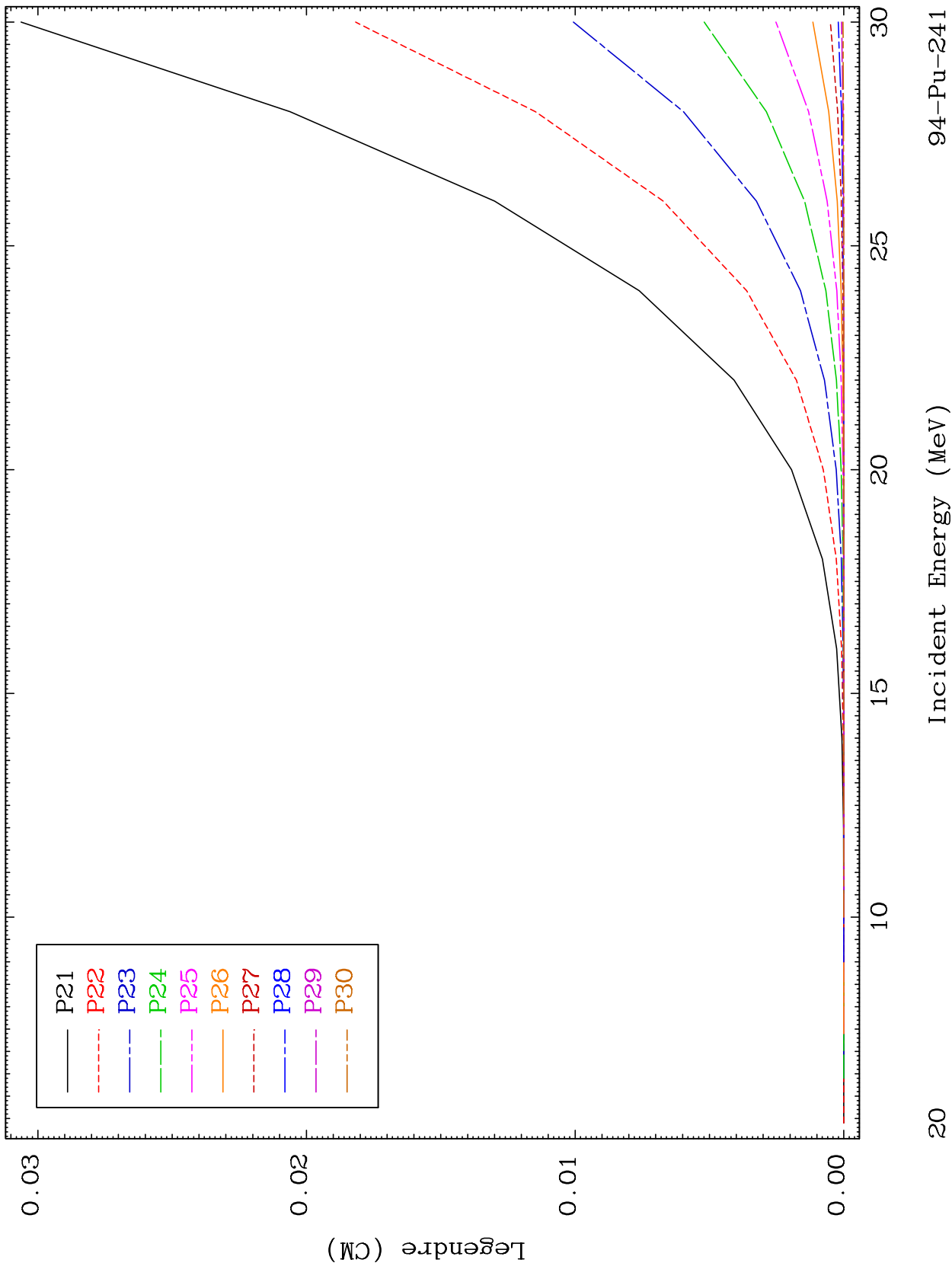
Incident Energy (MeV)

94-Pu-241

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



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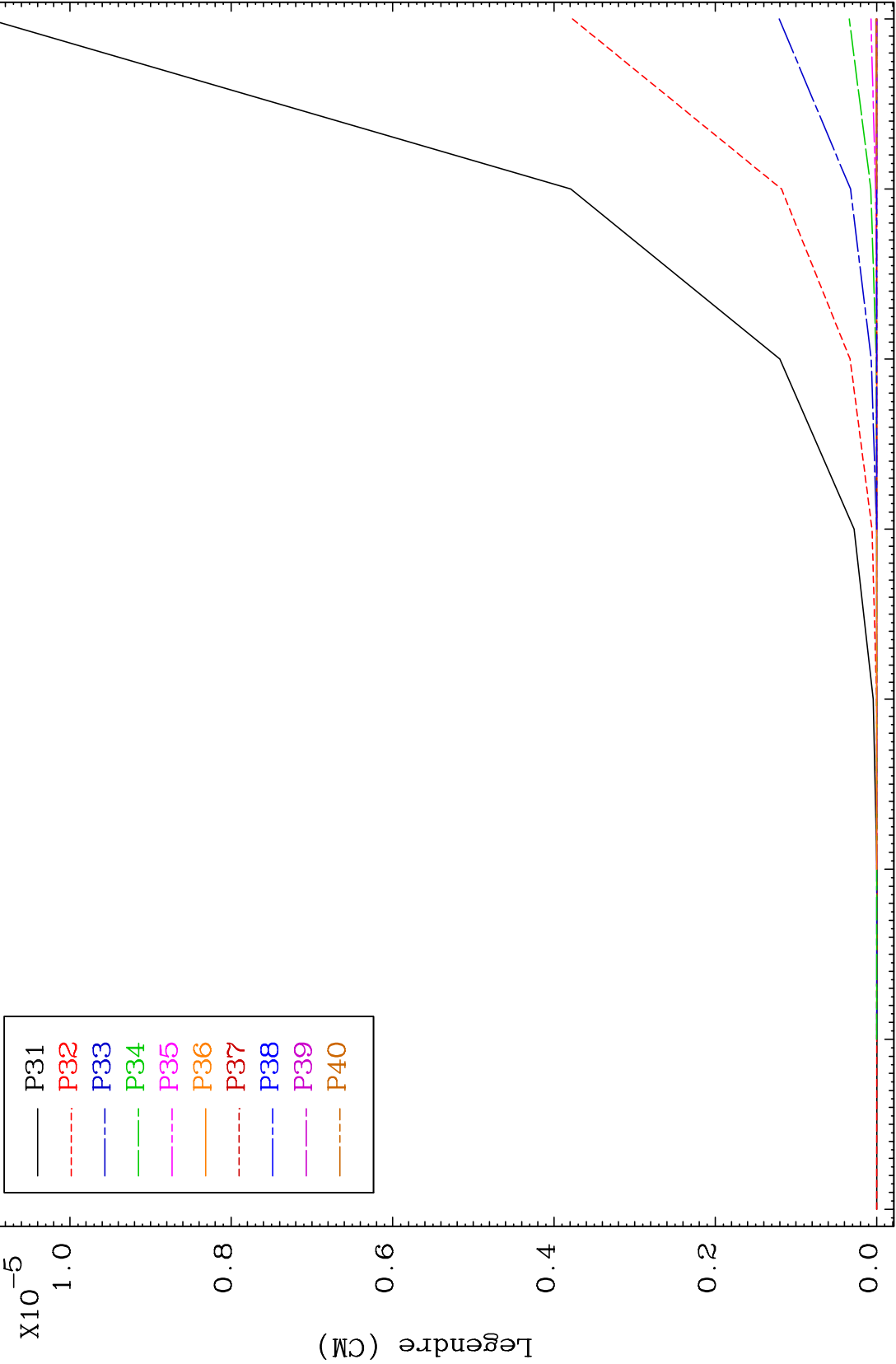
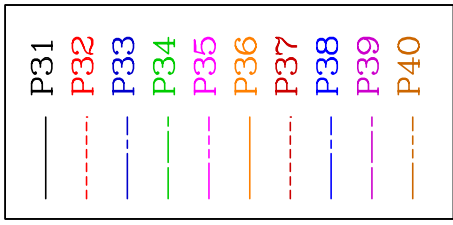
Incident Energy (MeV)

94-Pu-241

MAT 9443

Elastic Legendre Coefficients

94-Pu-241



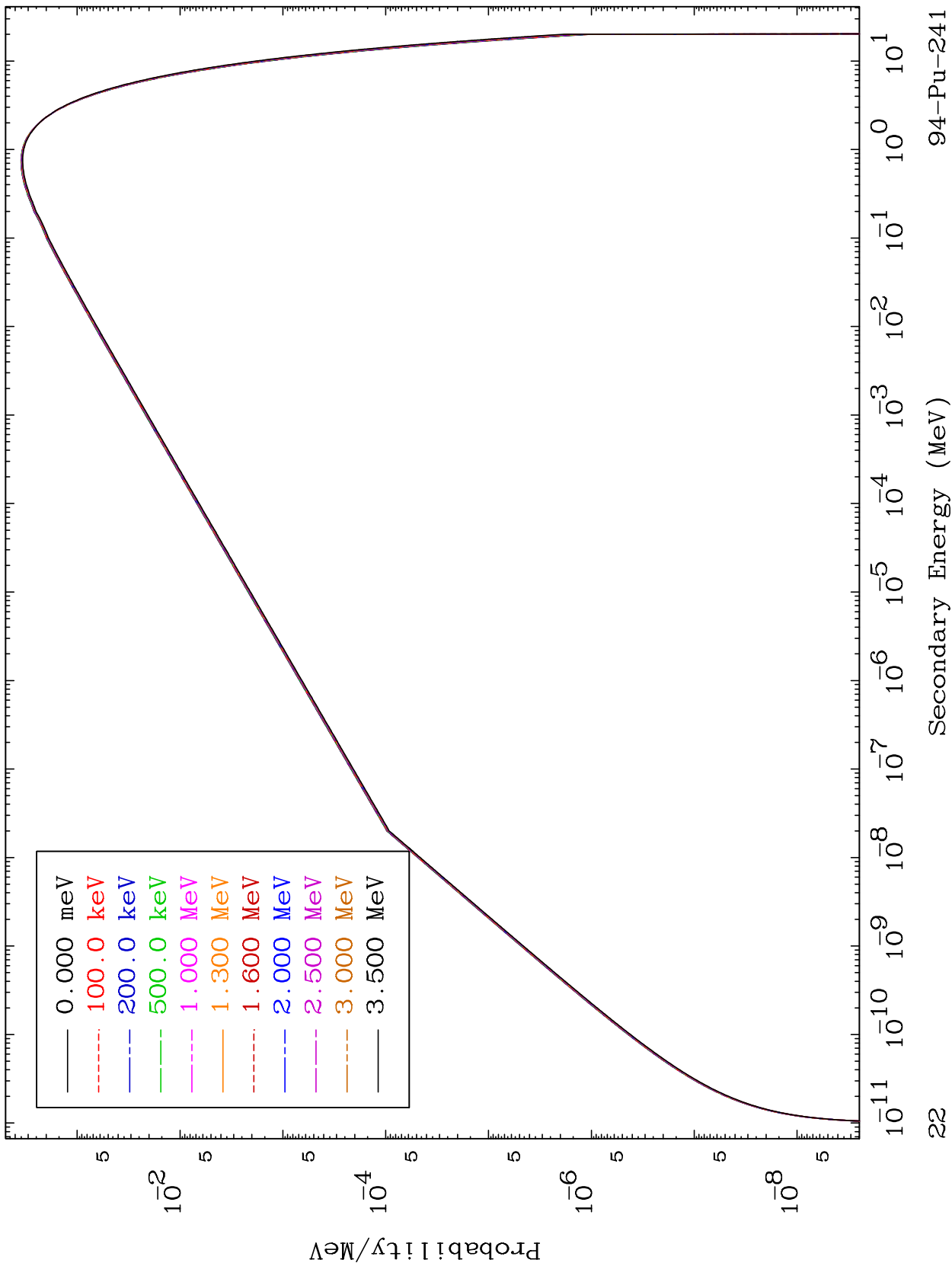
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94-Pu-241

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### Fission Energy Distribution

94-Pu-241



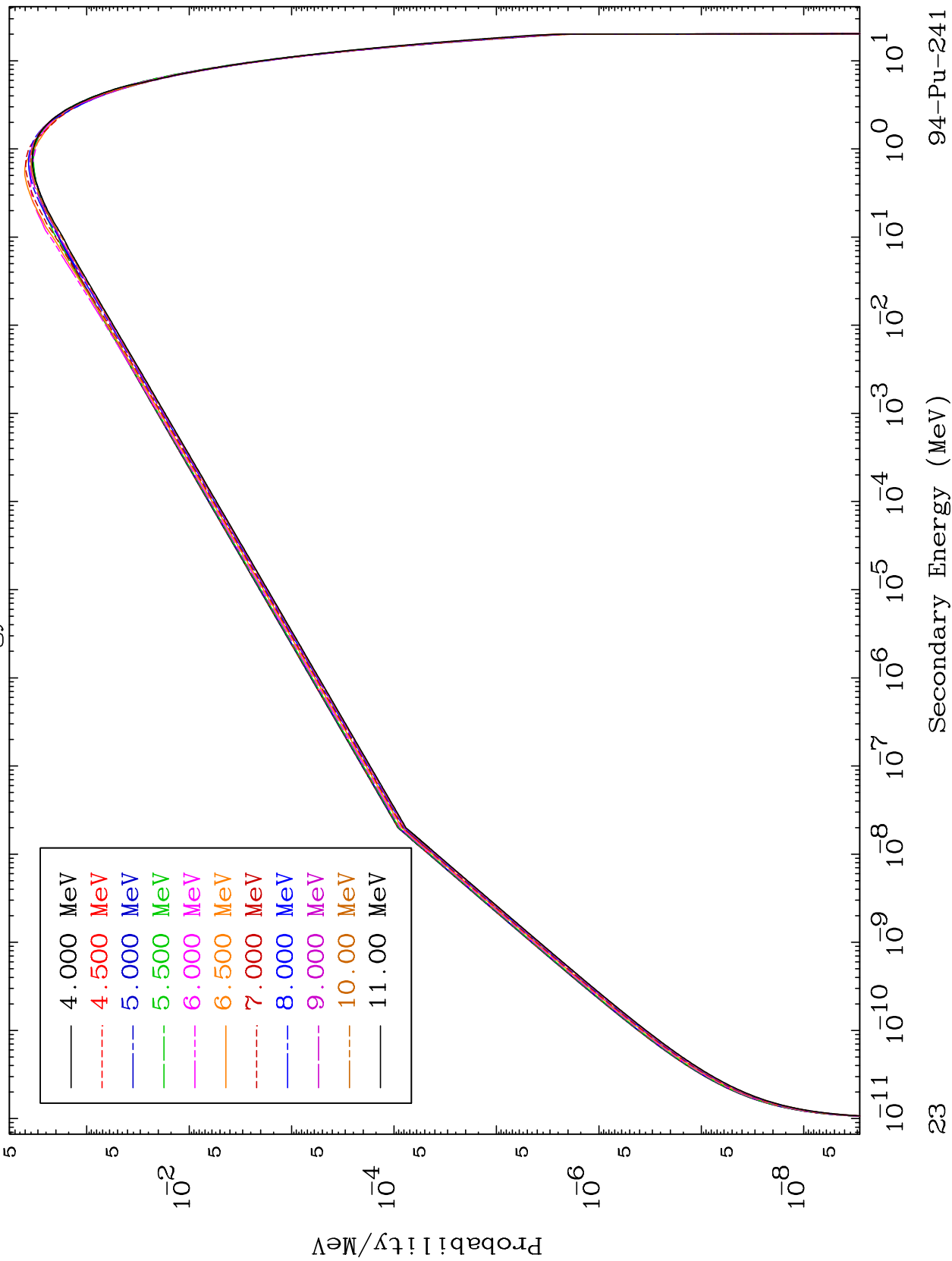
94-Pu-241

Secondary Energy (MeV)

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### Fission Energy Distribution

94-Pu-241



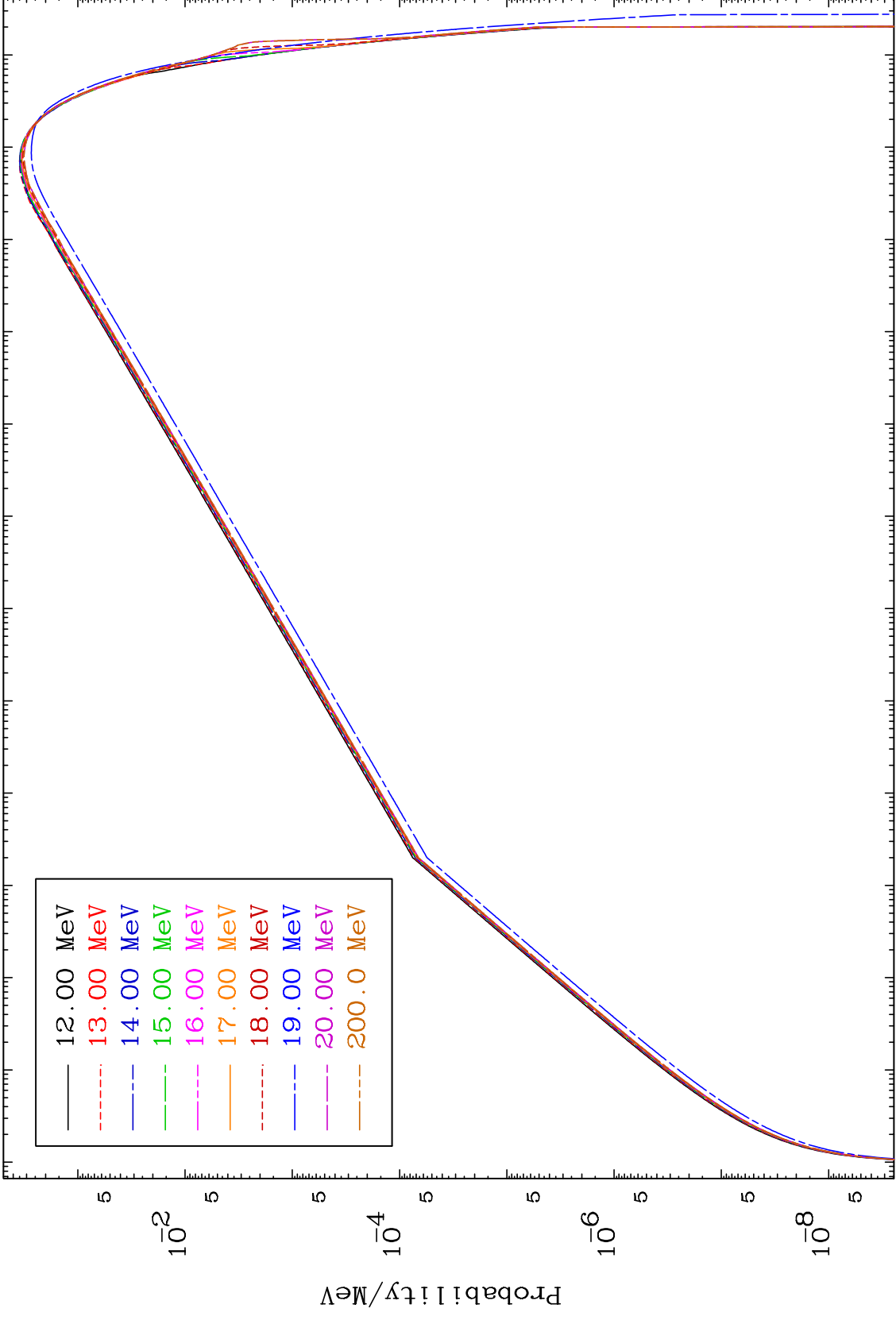


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Fission

94-Pu-241

Energy Distribution



10<sup>-11</sup> 10<sup>-10</sup> 10<sup>-9</sup> 10<sup>-8</sup> 10<sup>-7</sup> 10<sup>-6</sup> 10<sup>-5</sup> 10<sup>-4</sup> 10<sup>-3</sup> 10<sup>-2</sup> 10<sup>-1</sup> 10<sup>0</sup> 10<sup>1</sup>

Secondary Energy (MeV)

94-Pu-241

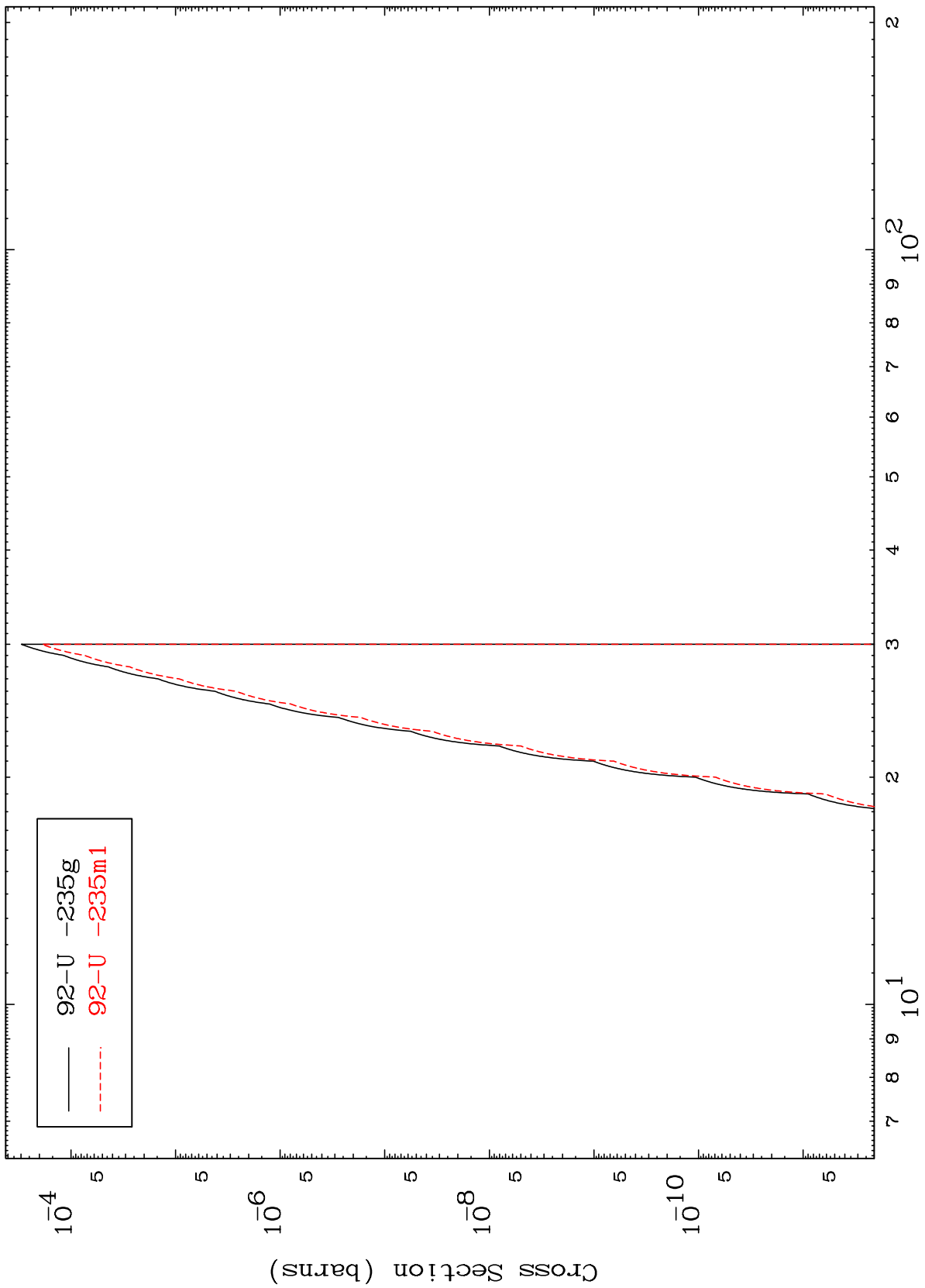
24

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(n,3n)  $\alpha$

94-Pu-241

Radionuclide Production Cross Section



25

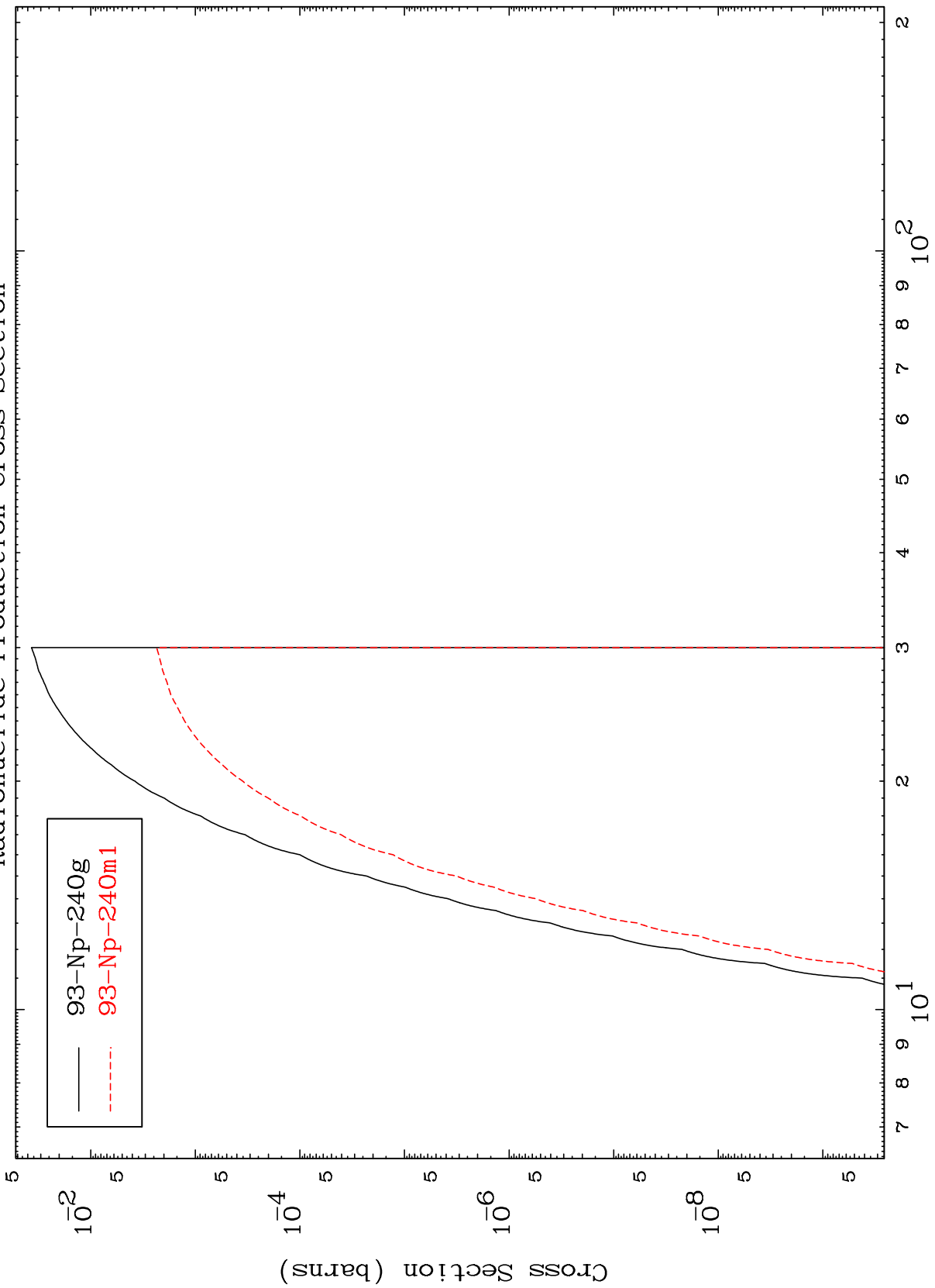
Incident Energy (MeV)

94-Pu-241

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94-Pu-241

(n,n') p  
Radionuclide Production Cross Section



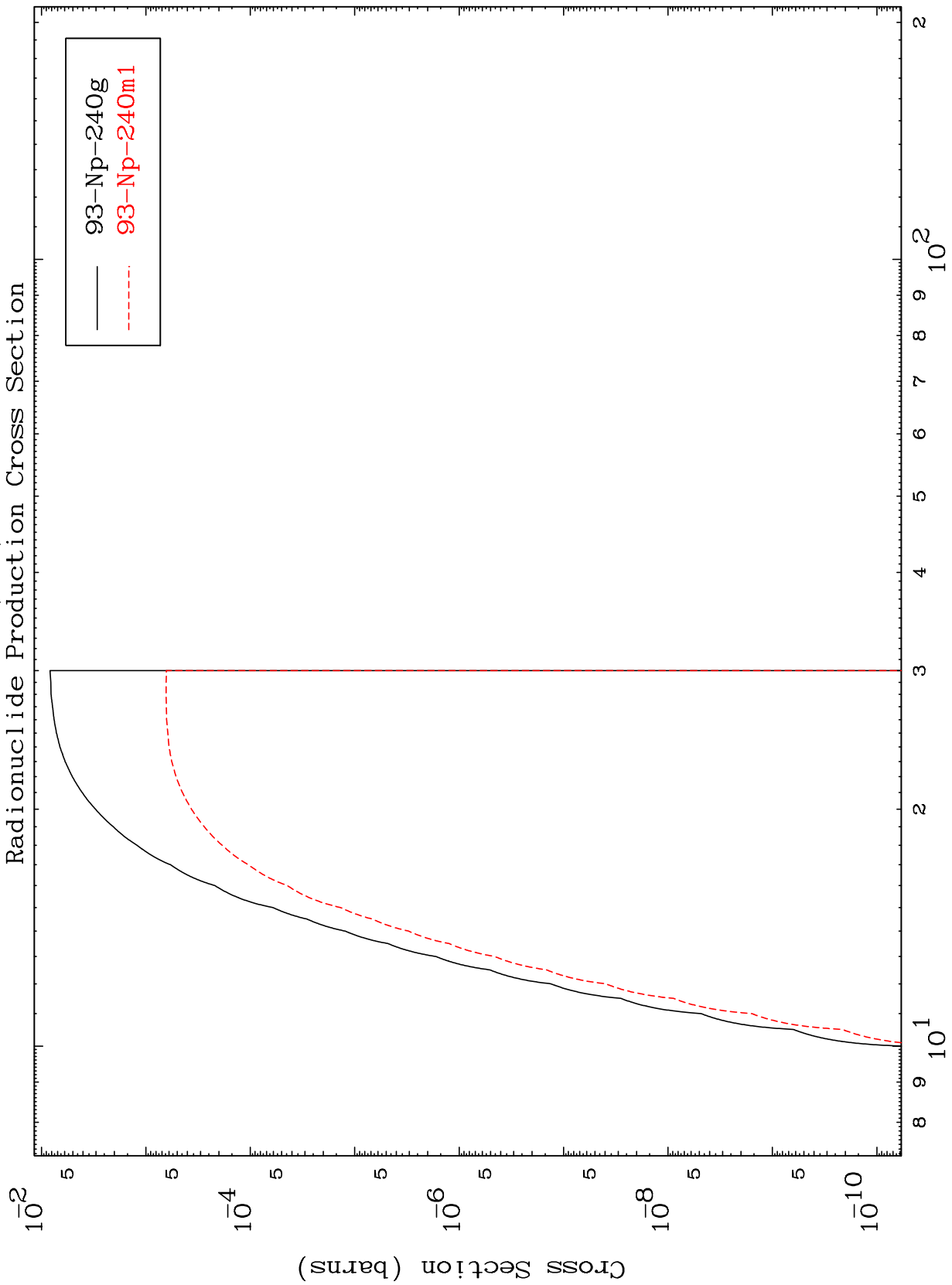
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94-Pu-241

(n,d)  
Radionuclide Production Cross Section



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Incident Energy (MeV)