

Program EVALPLOT
(Version 2018-1)

by

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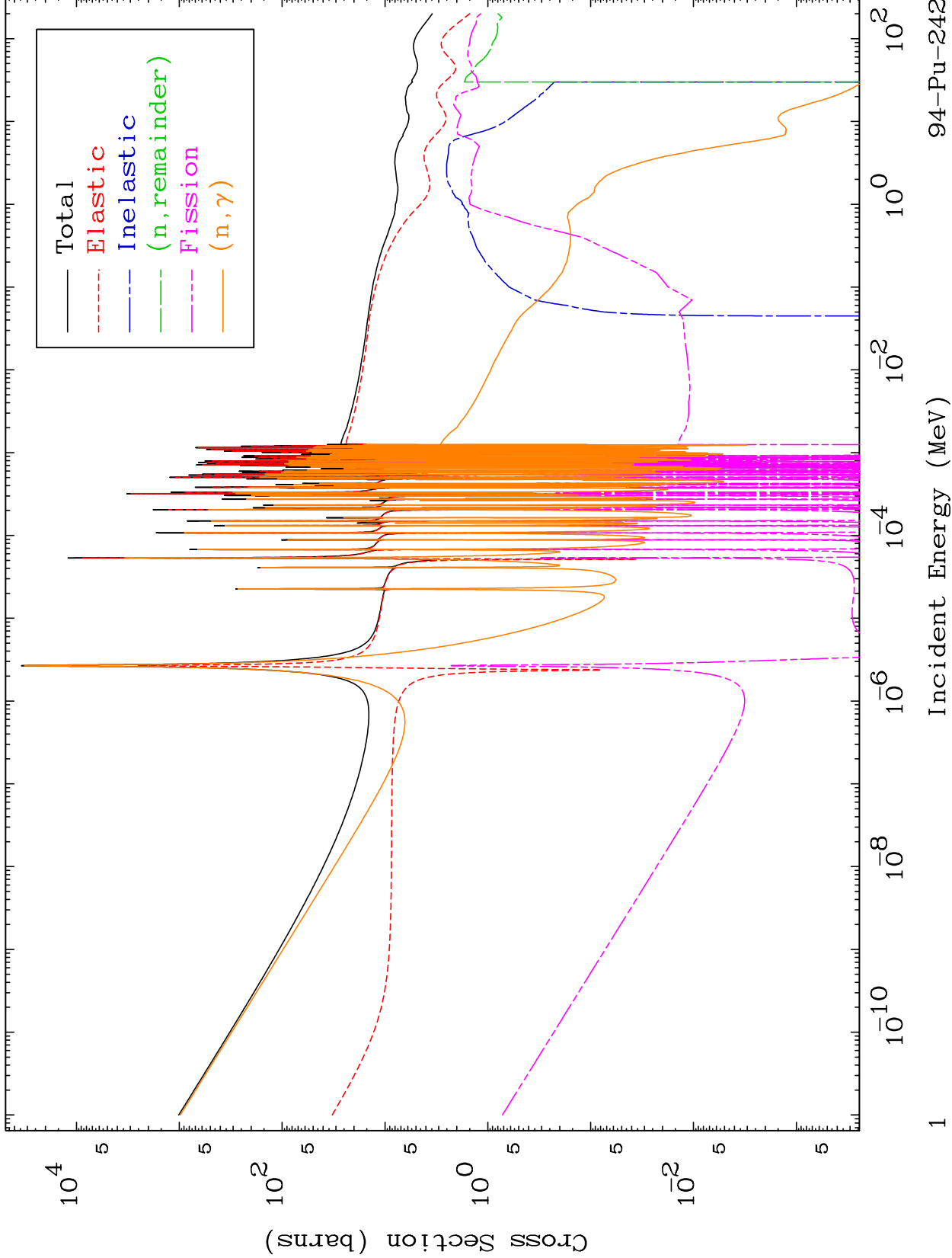
Press Mouse Button to Start

MAT 9446

Major

293 Kelvin Cross Sections

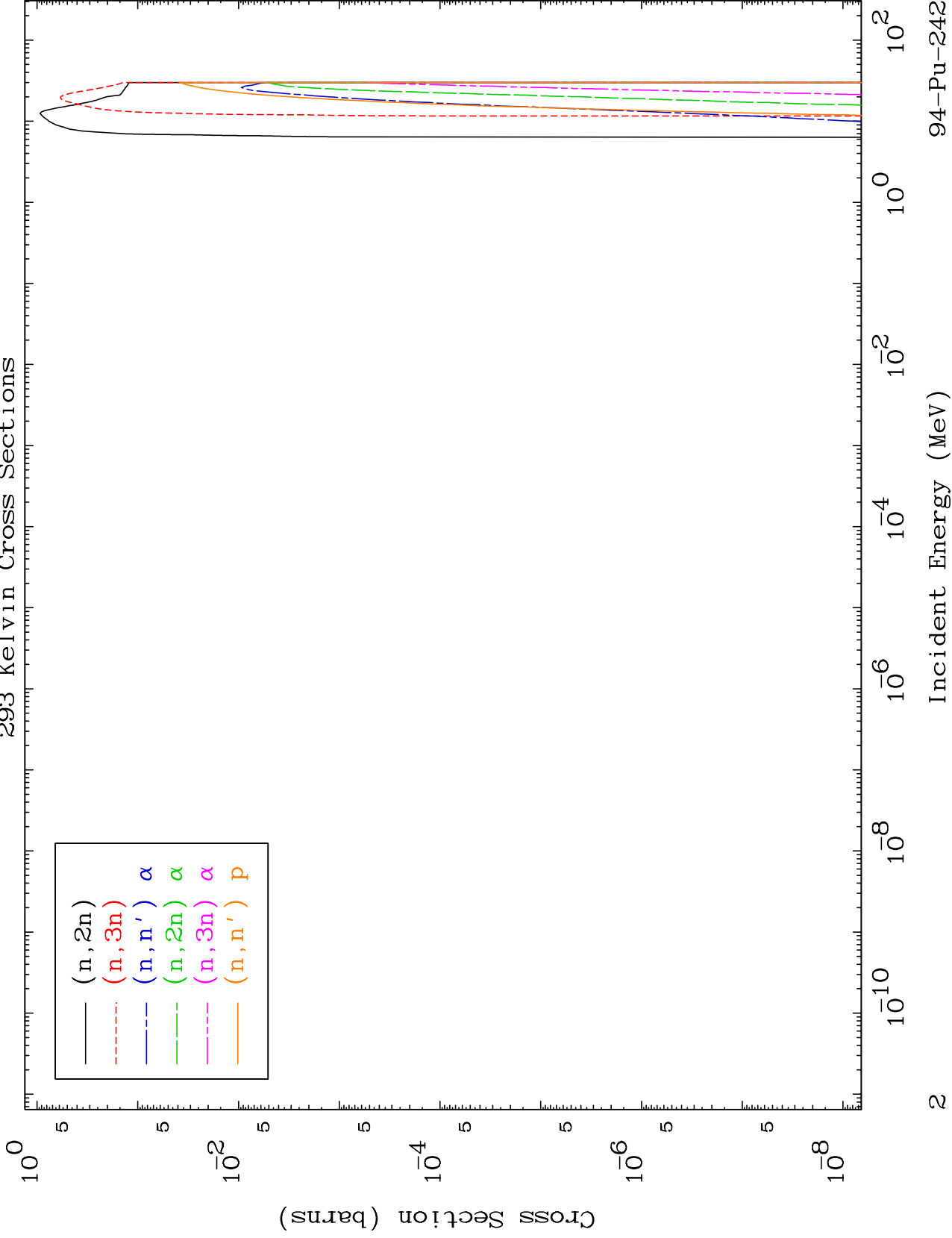
94-Pu-242



MAT 9446

Neutron Production
293 Kelvin Cross Sections

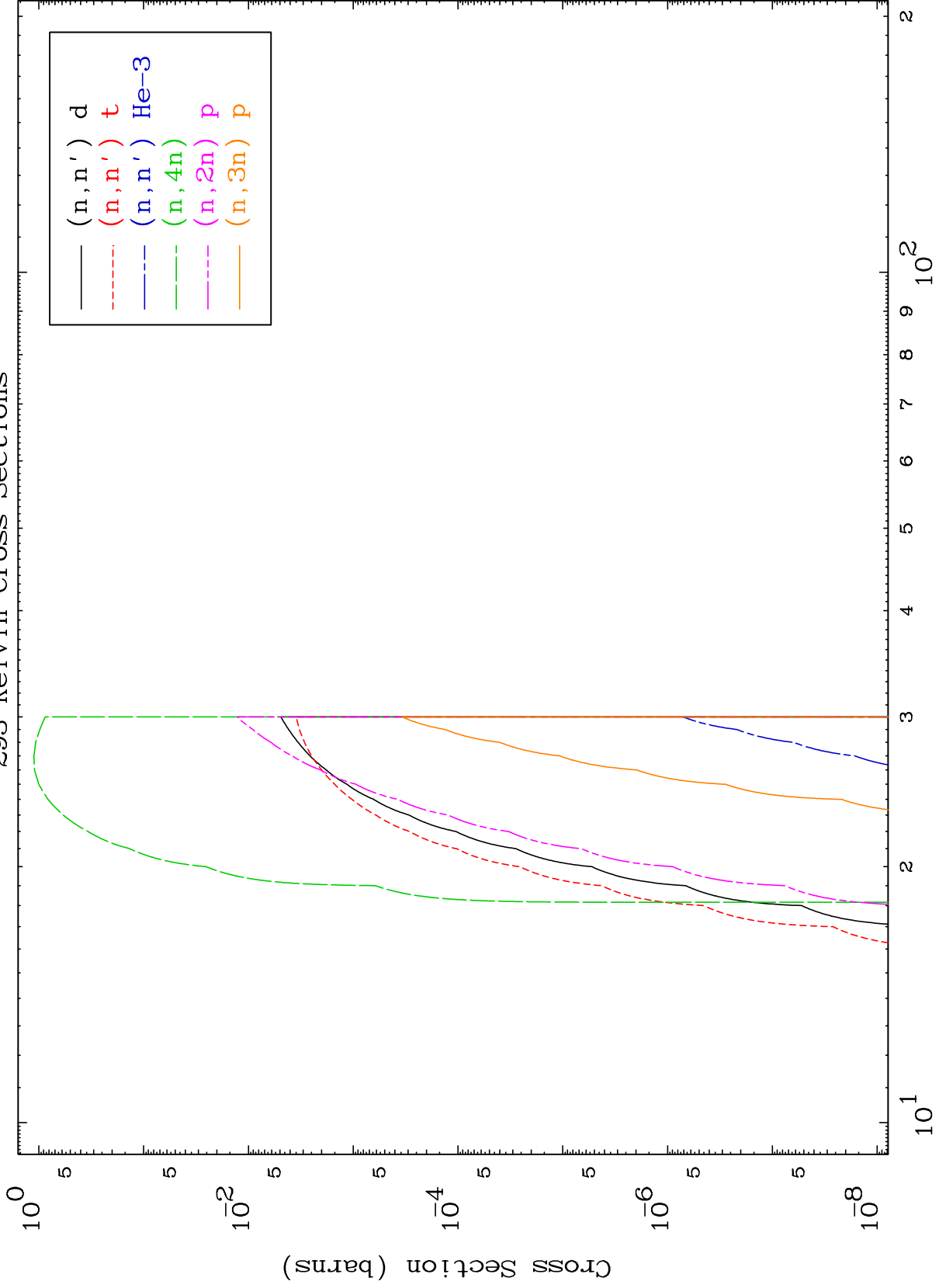
94-Pu-242



MAT 9446

Neutron Production
293 Kelvin Cross Sections

94-Pu-242



Incident Energy (MeV)

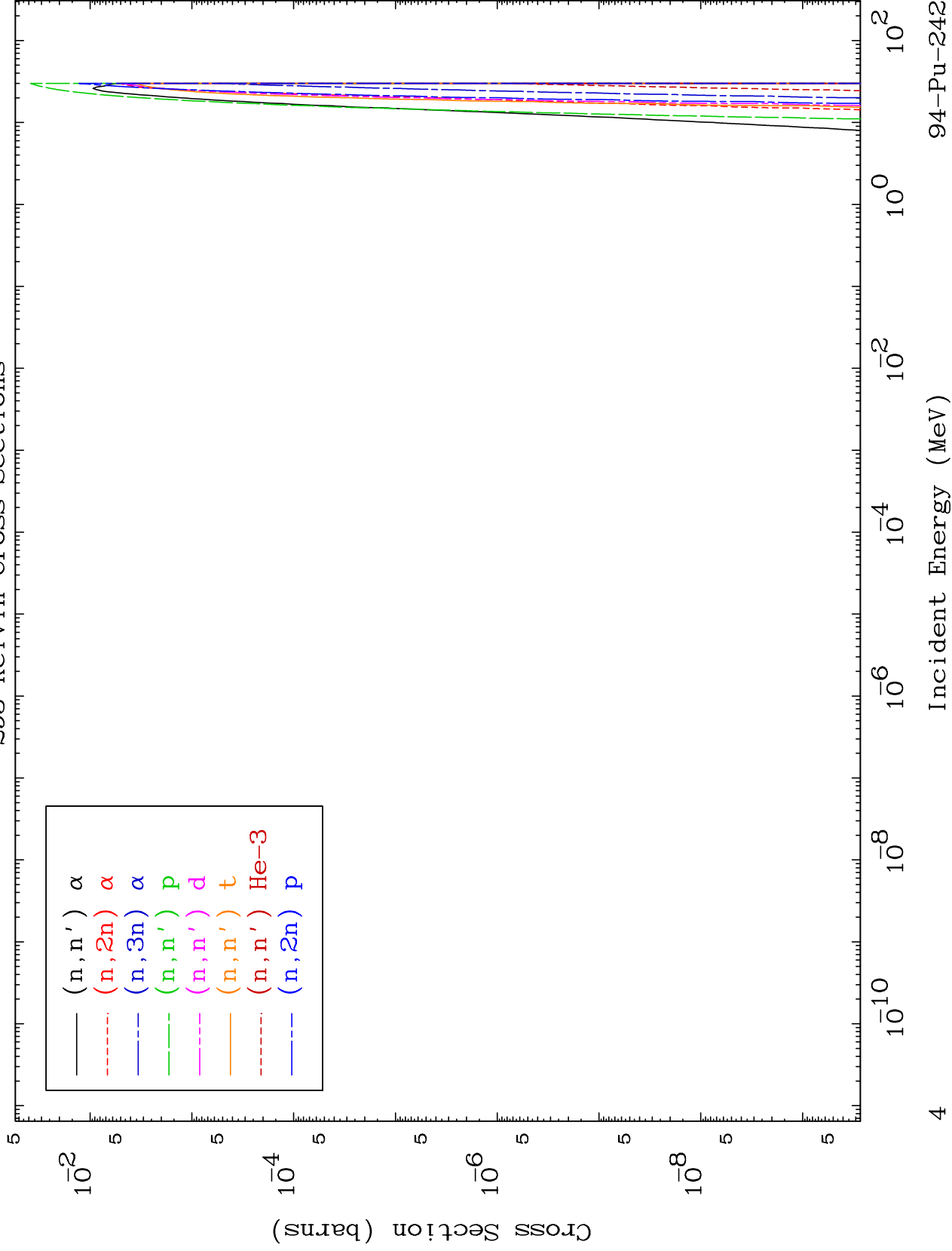
94-Pu-242

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MAT 9446

Charged Particle
293 Kelvin Cross Sections

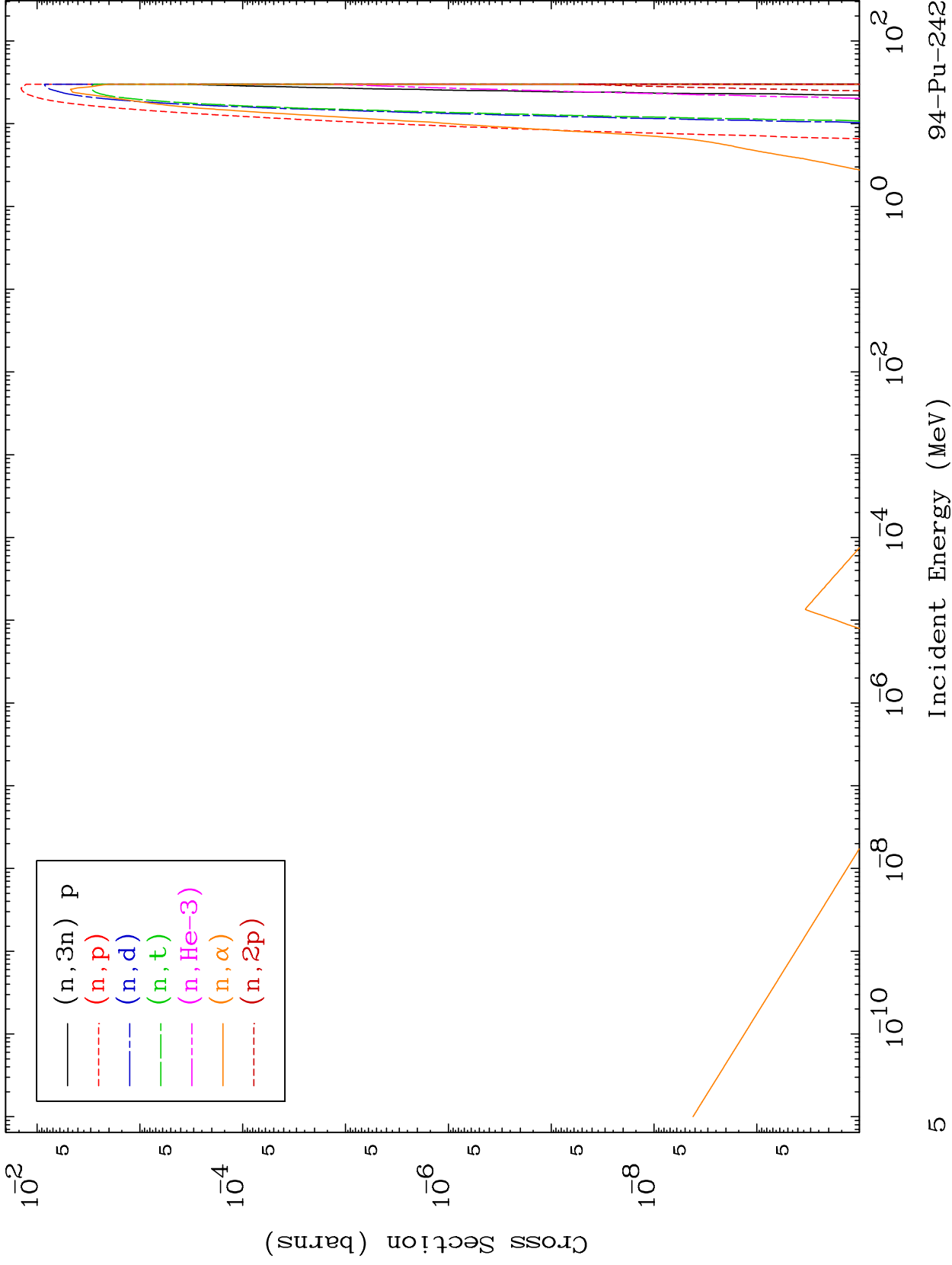
94-Pu-242



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Charged Particle
293 Kelvin Cross Sections

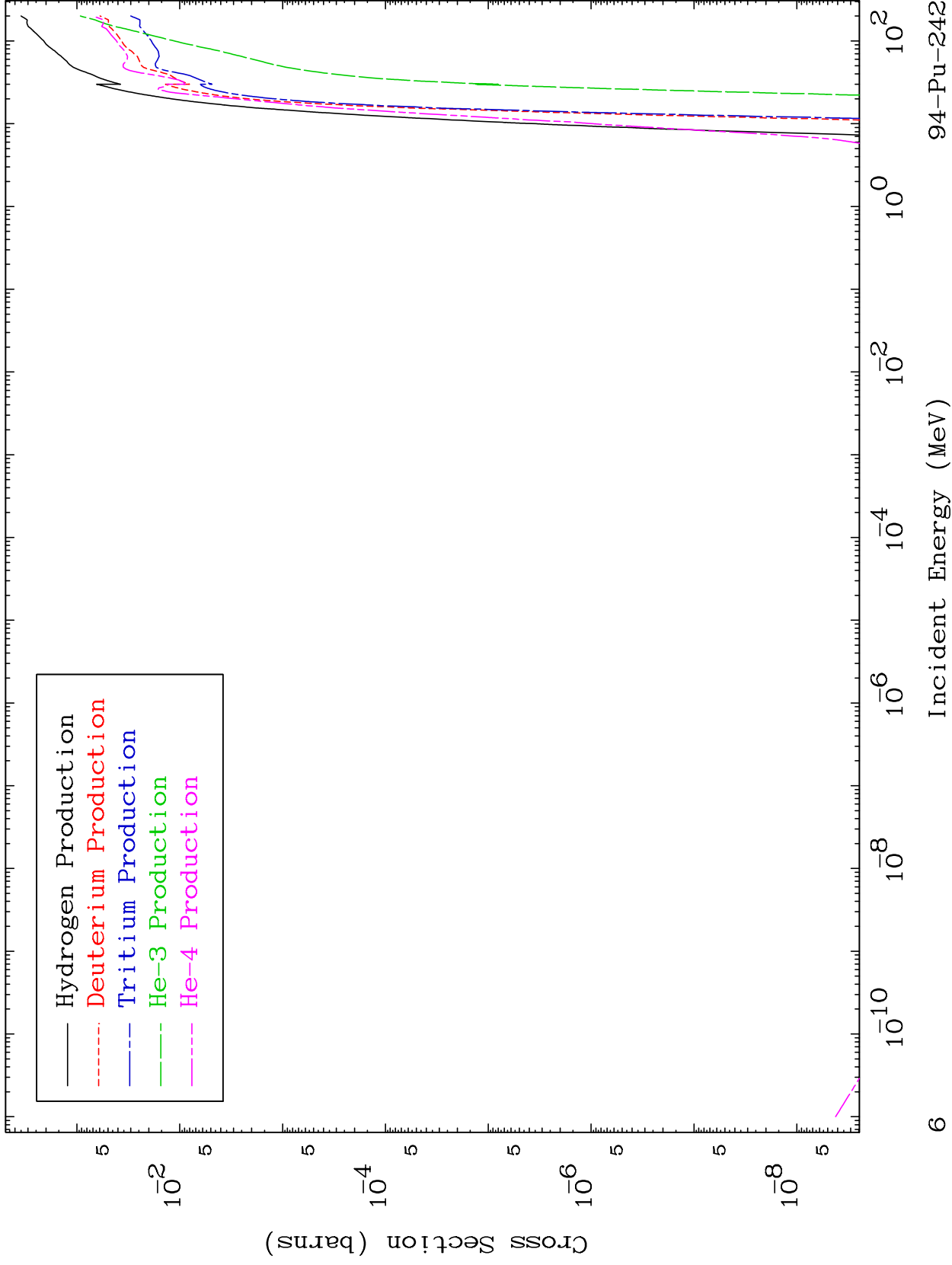
94-Pu-242



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Particle Production
293 Kelvin Cross Sections

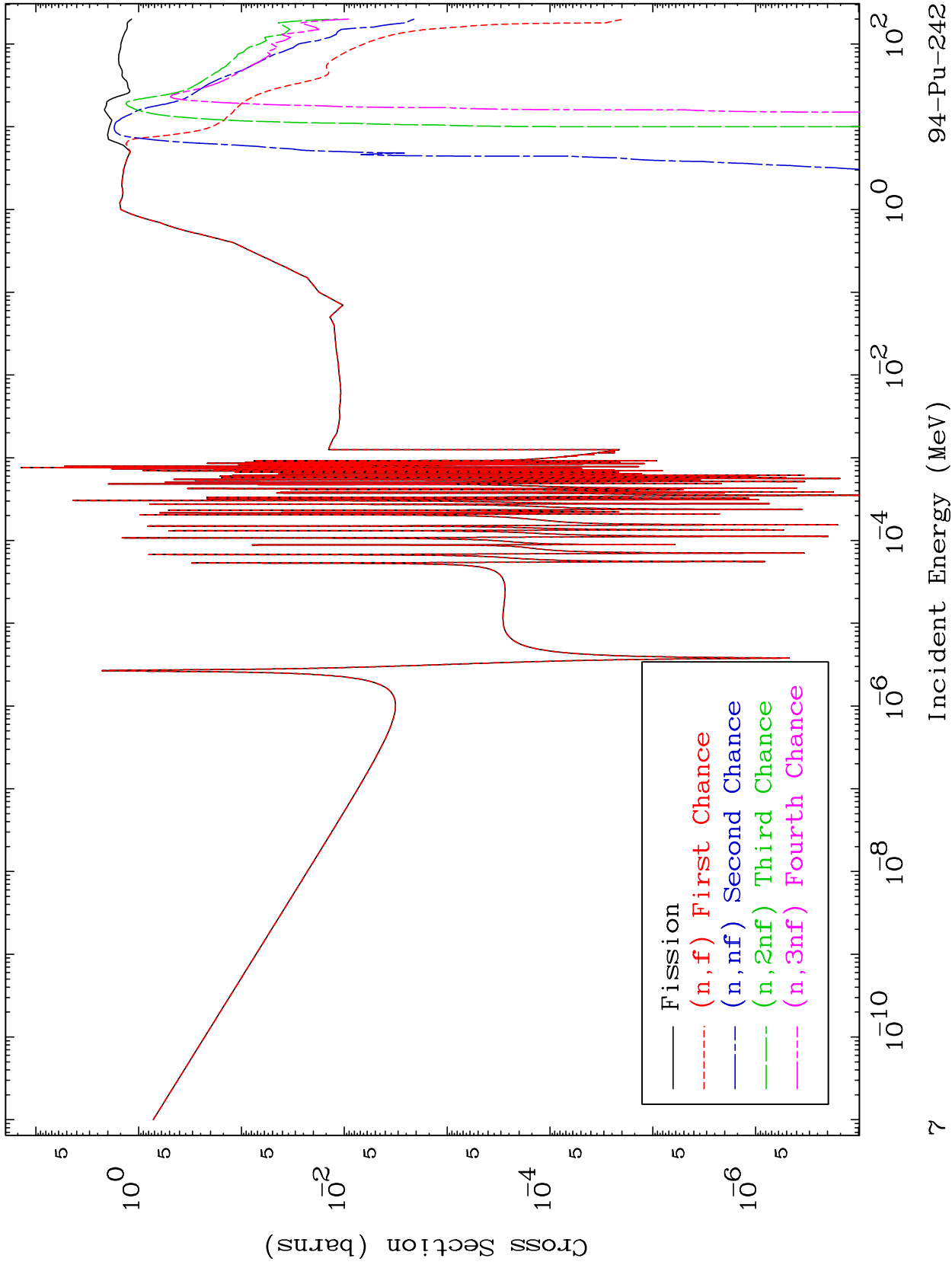
94-Pu-242



MAT 9446

Fission
293 Kelvin Cross Sections

94-Pu-242

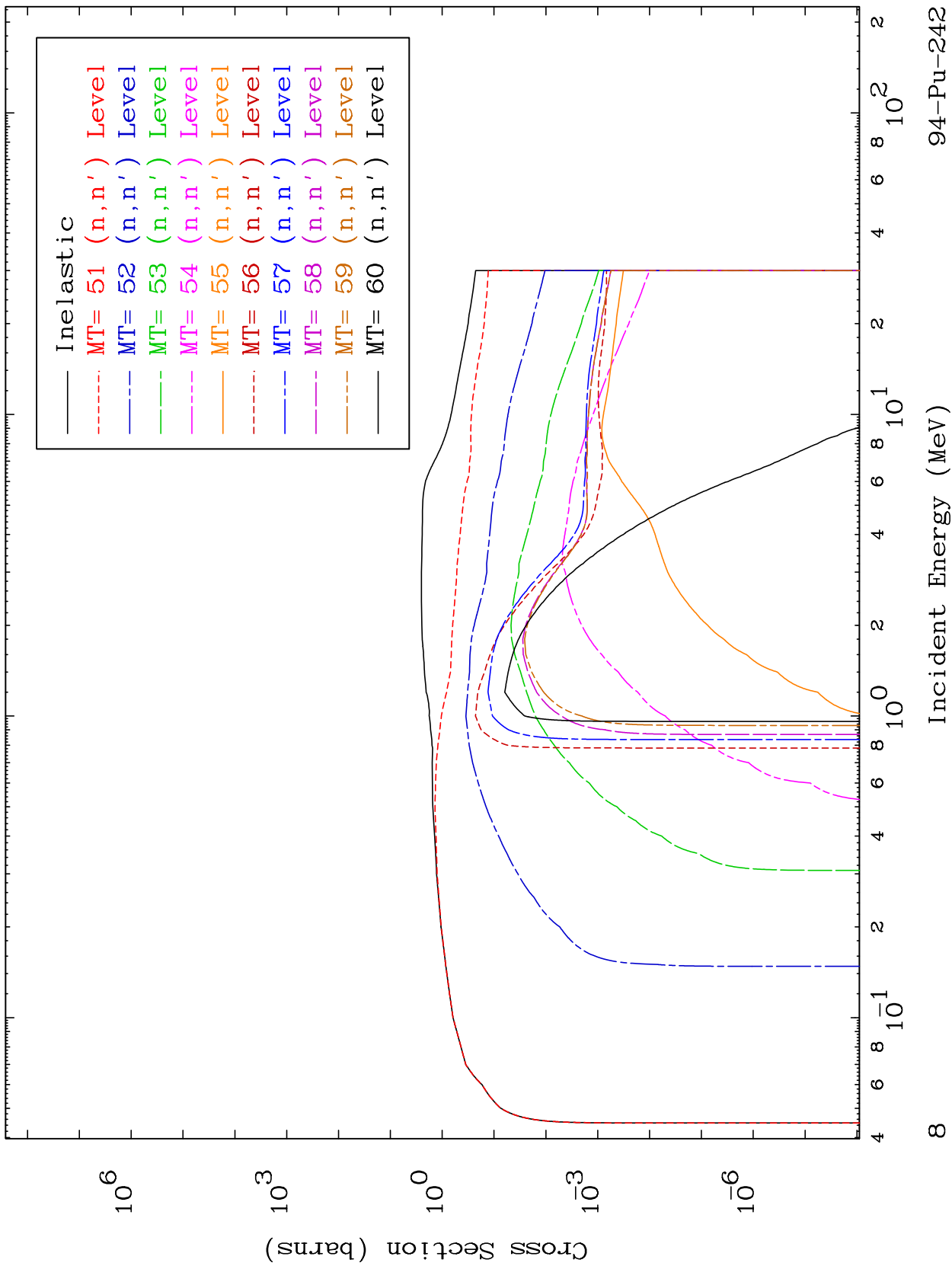


MAT 9446

(n,n') Level

94-Pu-242

293 Kelvin Cross Sections

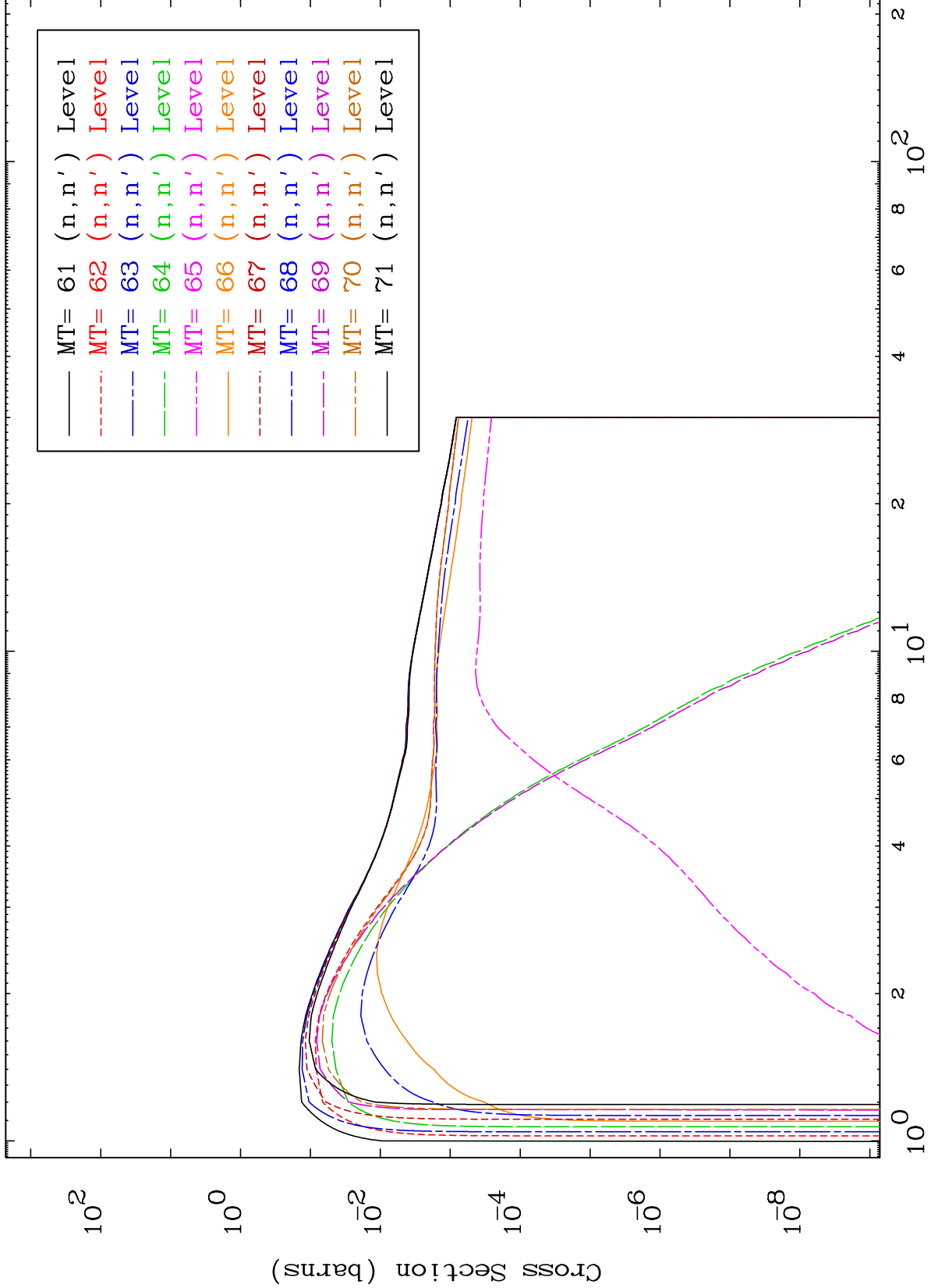


MAT 9446

(n,n') Level

94-Pu-242

293 Kelvin Cross Sections



Incident Energy (MeV)

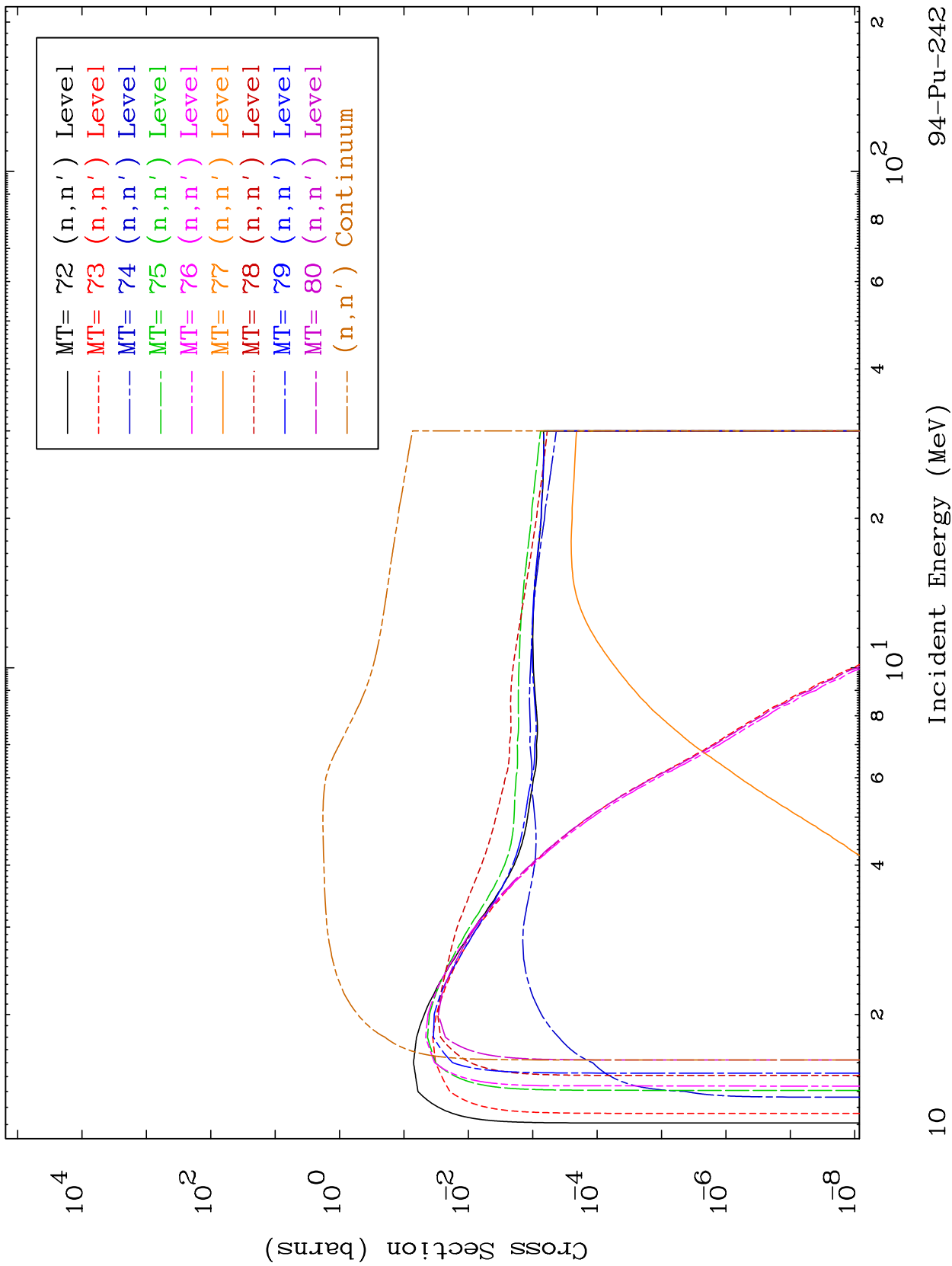
94-Pu-242

MAT 9446

(n,n') Level

94-Pu-242

293 Kelvin Cross Sections



10

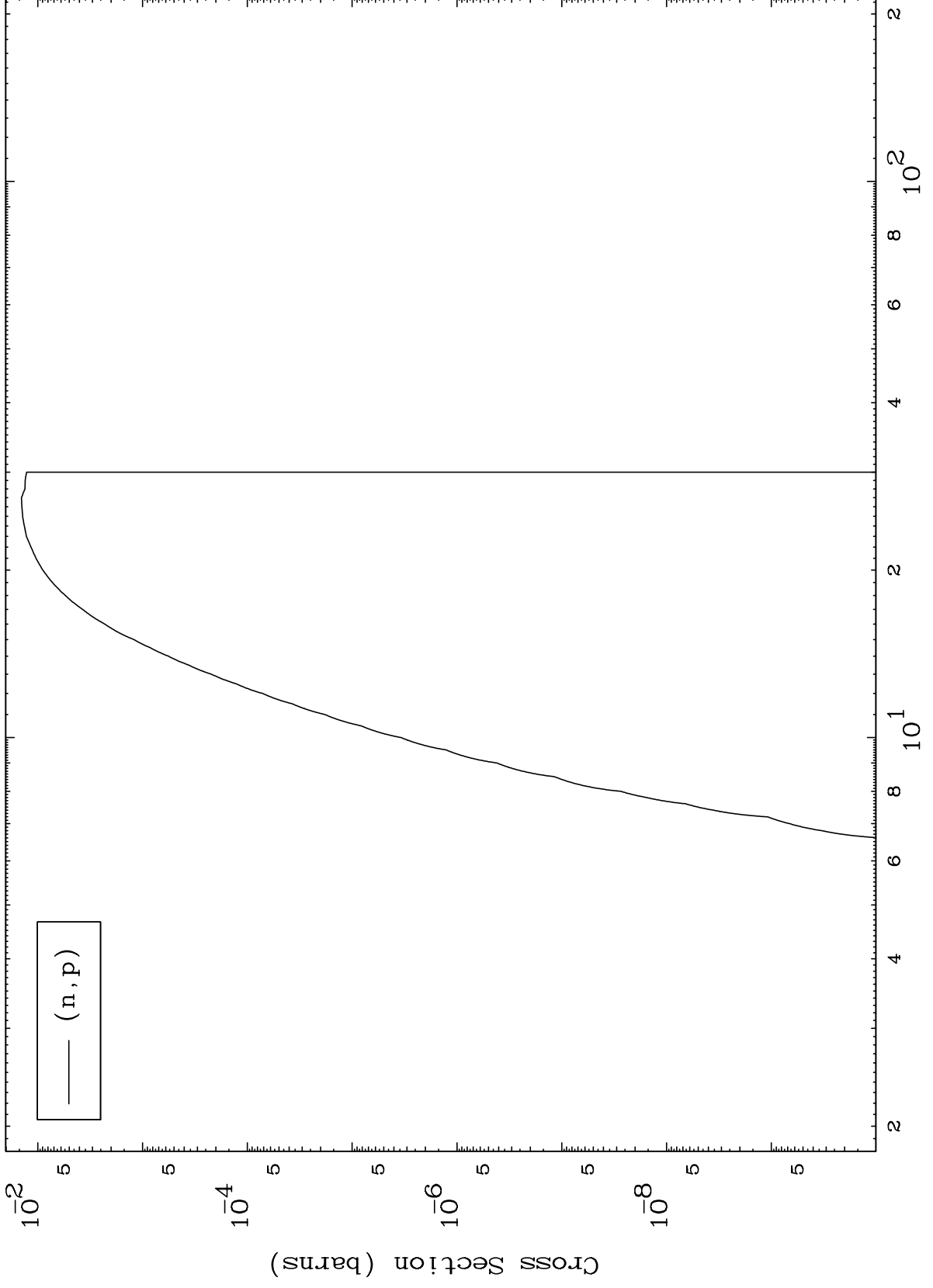
Incident Energy (MeV)

94-Pu-242

MAT 9446

(n,p) Levels
293 Kelvin Cross Sections

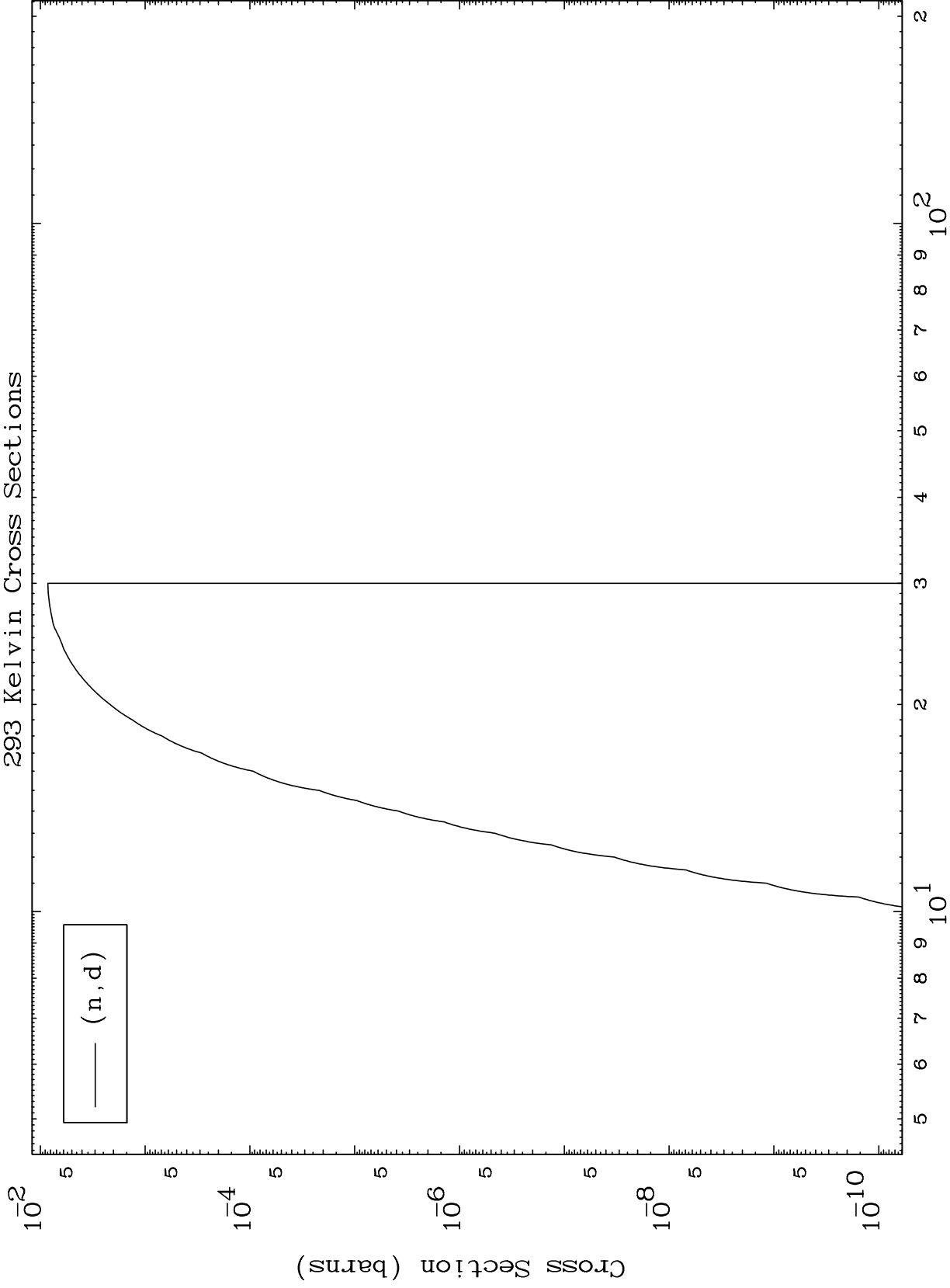
94-Pu-242



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(n,d) Levels
293 Kelvin Cross Sections

94-Pu-242



12

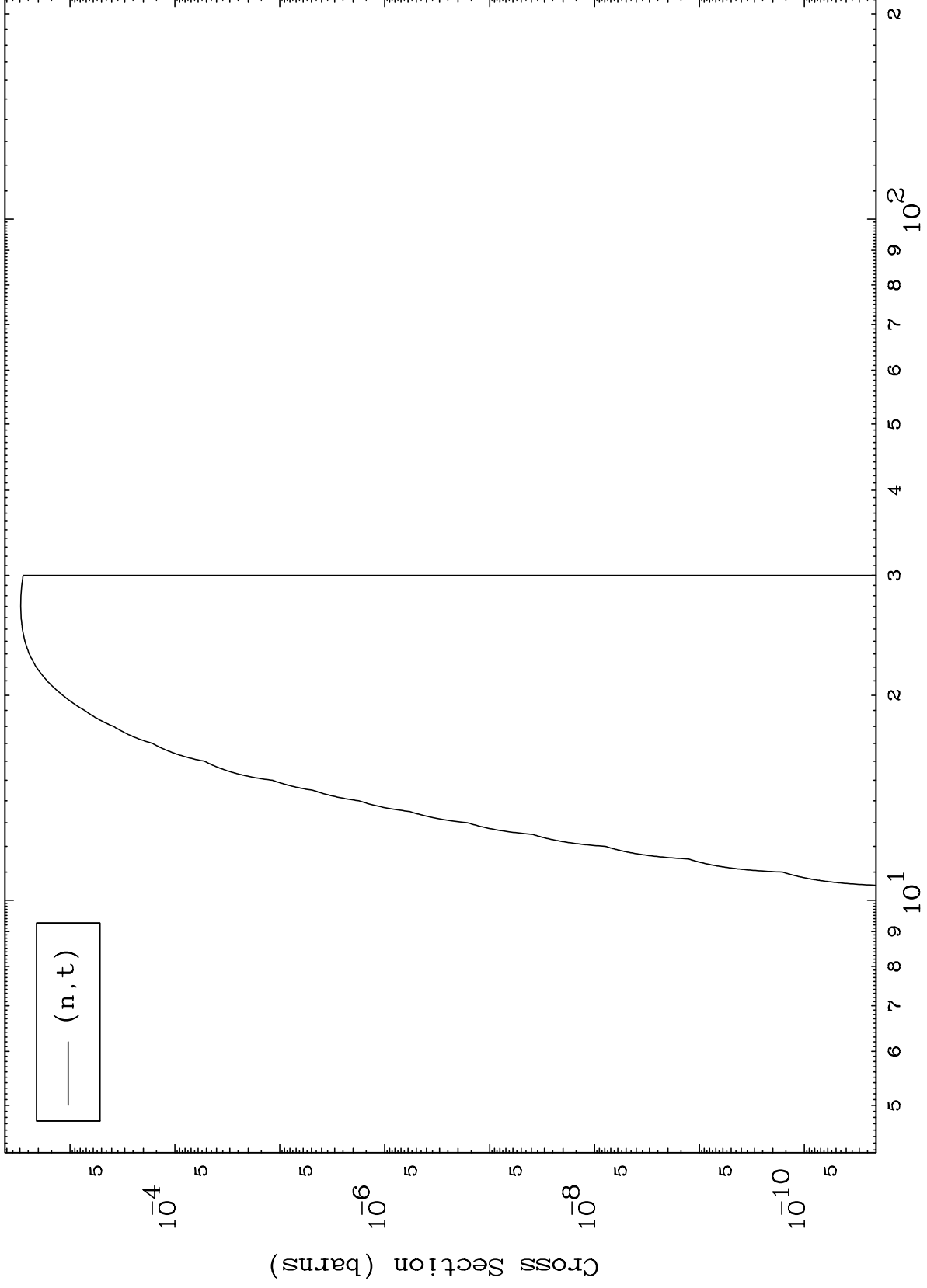
Incident Energy (MeV)

94-Pu-242

MAT 9446

(n,t) Levels
293 Kelvin Cross Sections

94-Pu-242



13

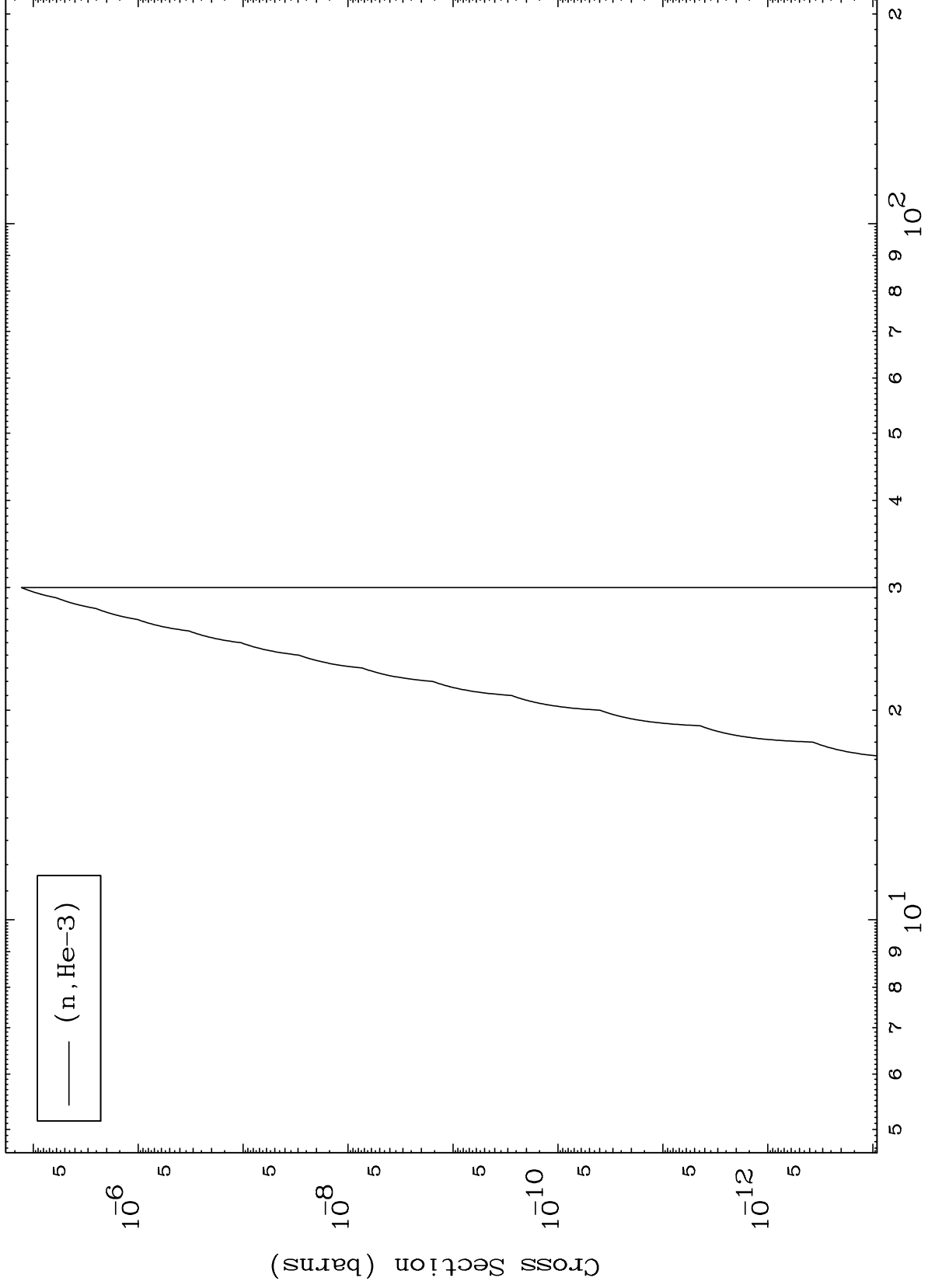
Incident Energy (MeV)

94-Pu-242

MAT 9446

(n,He3) Levels
293 Kelvin Cross Sections

94-Pu-242



14

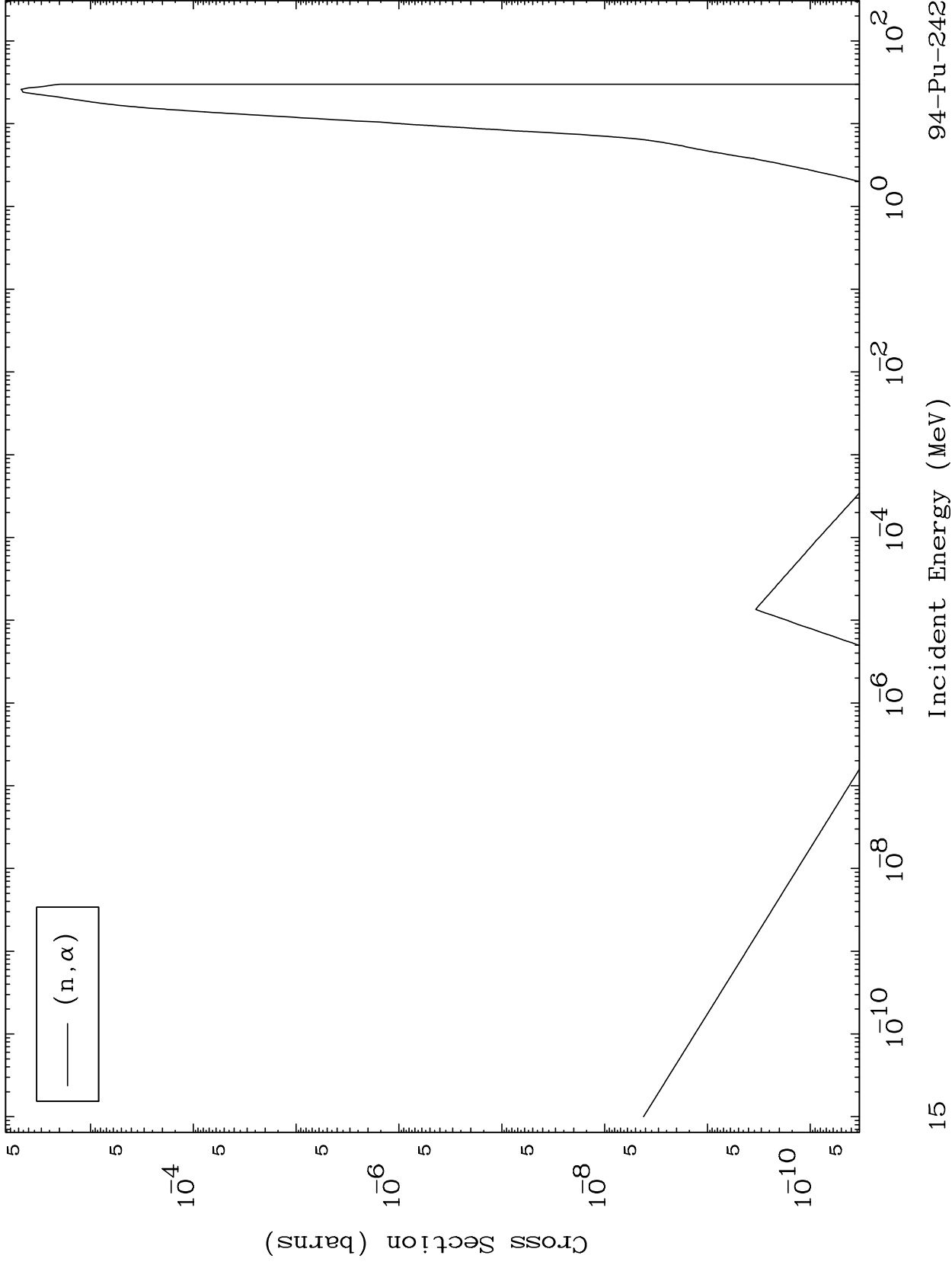
Incident Energy (MeV)

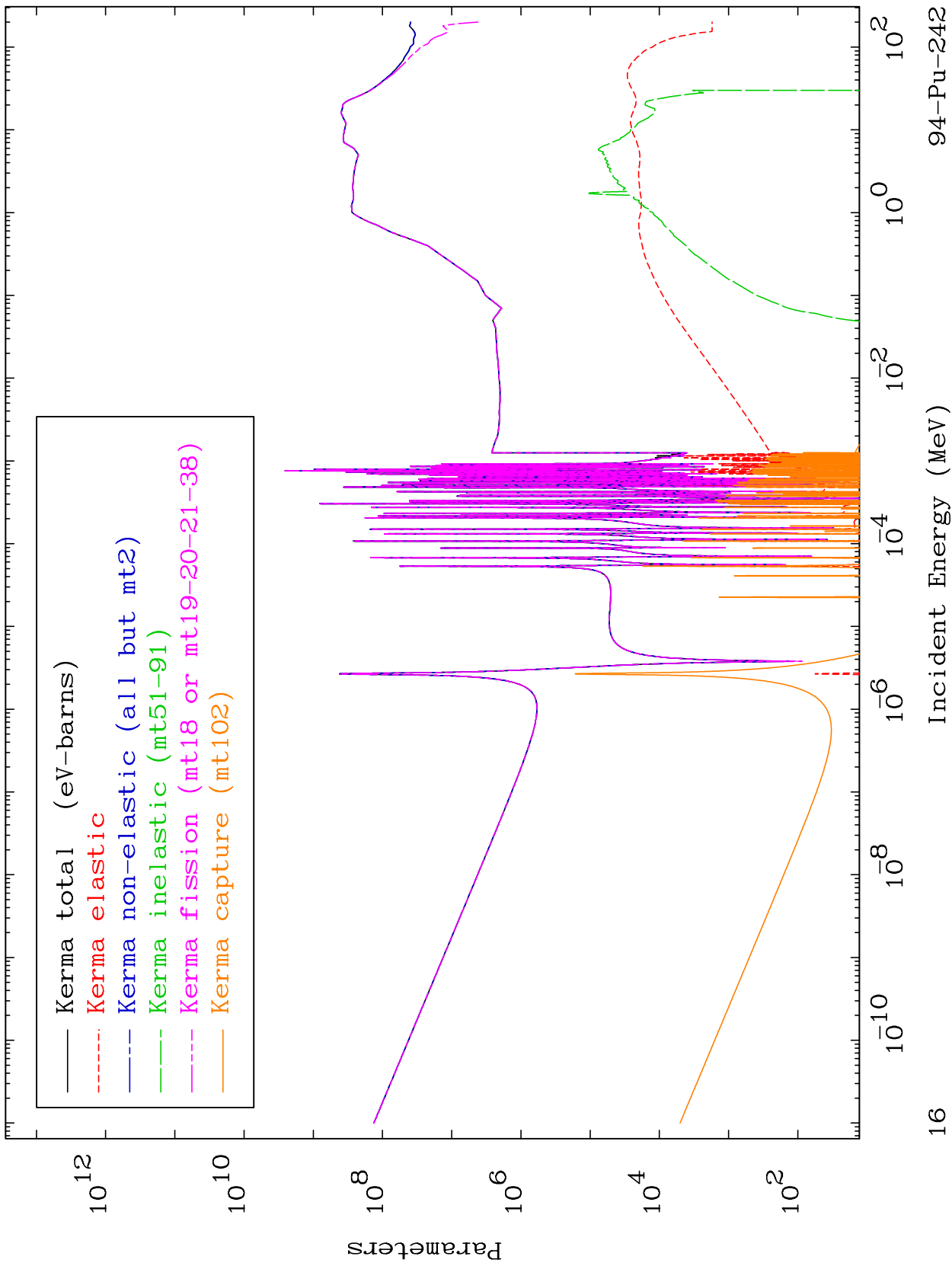
94-Pu-242

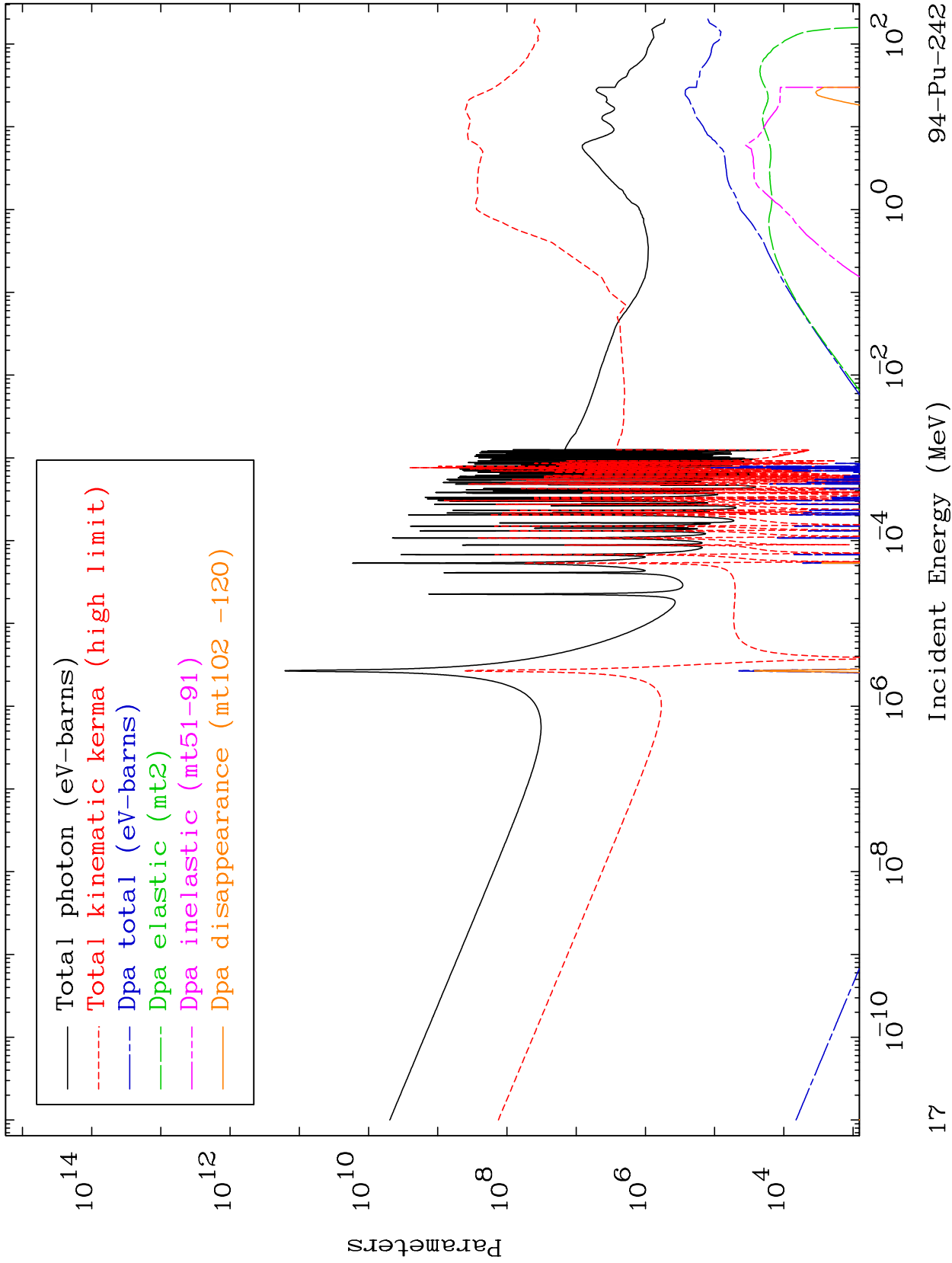
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(n, α) Levels
293 Kelvin Cross Sections

94-Pu-242



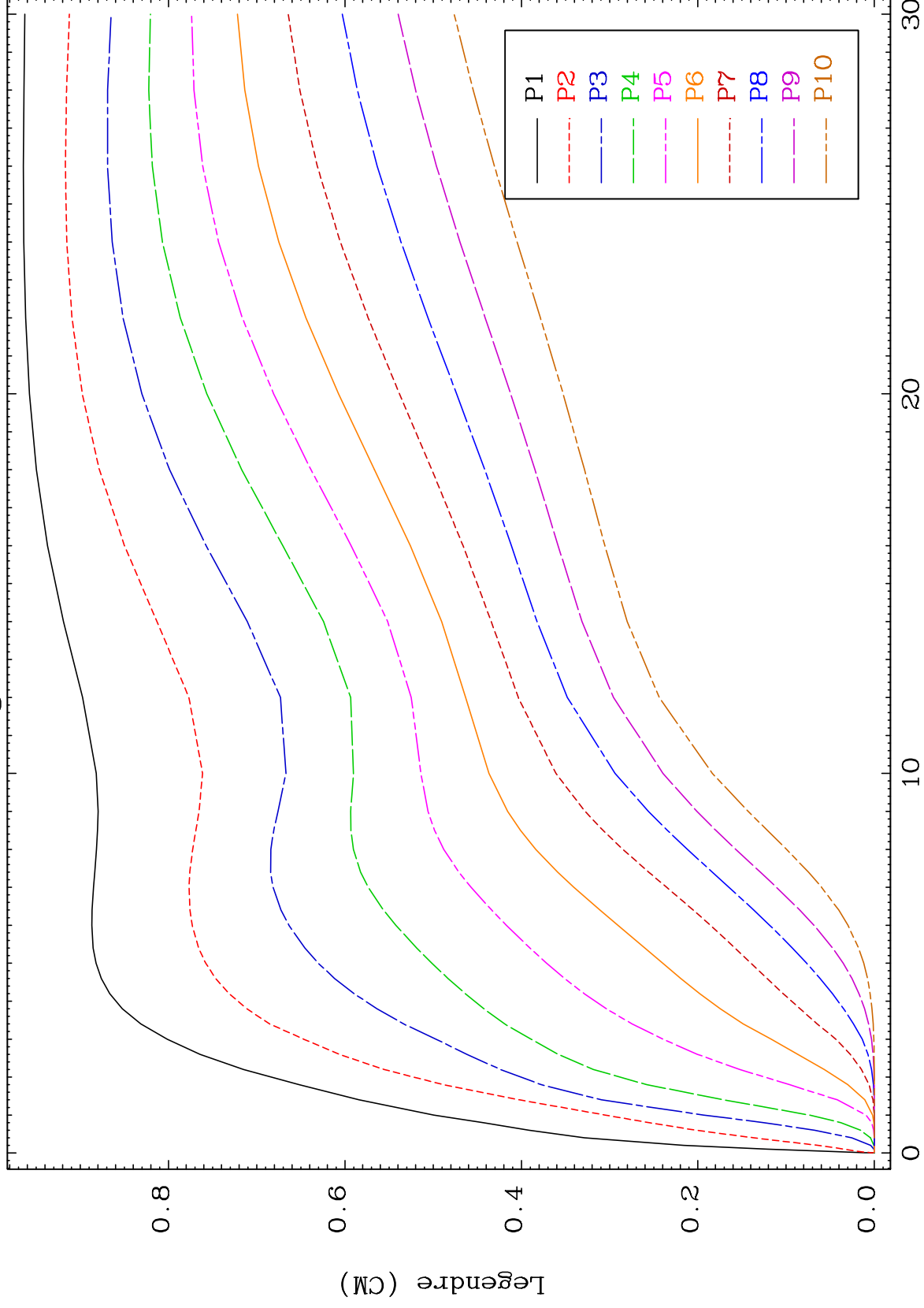




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Elastic Legendre Coefficients

94-Pu-242



18

Incident Energy (MeV)

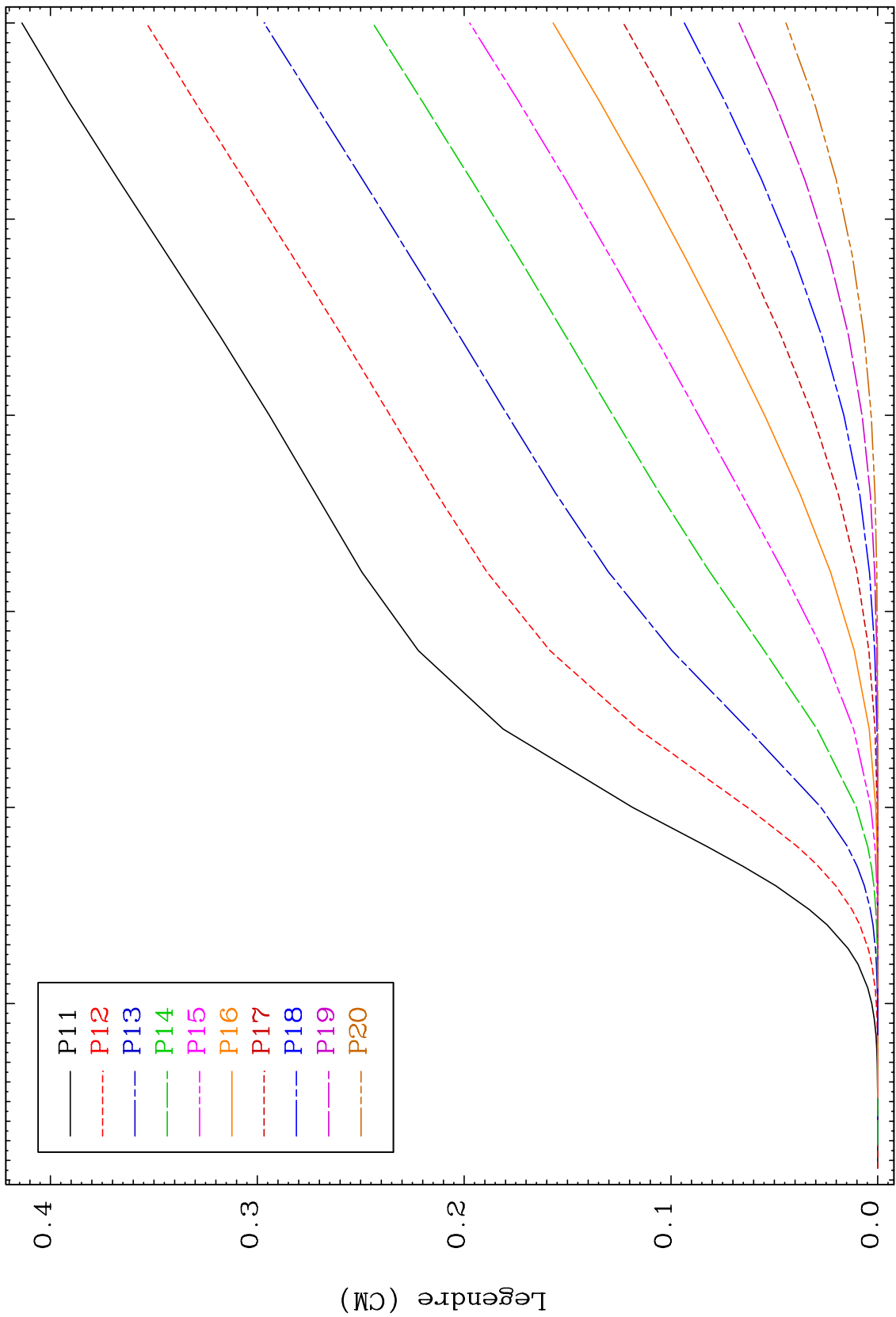
94-Pu-242

30

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Elastic Legendre Coefficients

94-Pu-242



19

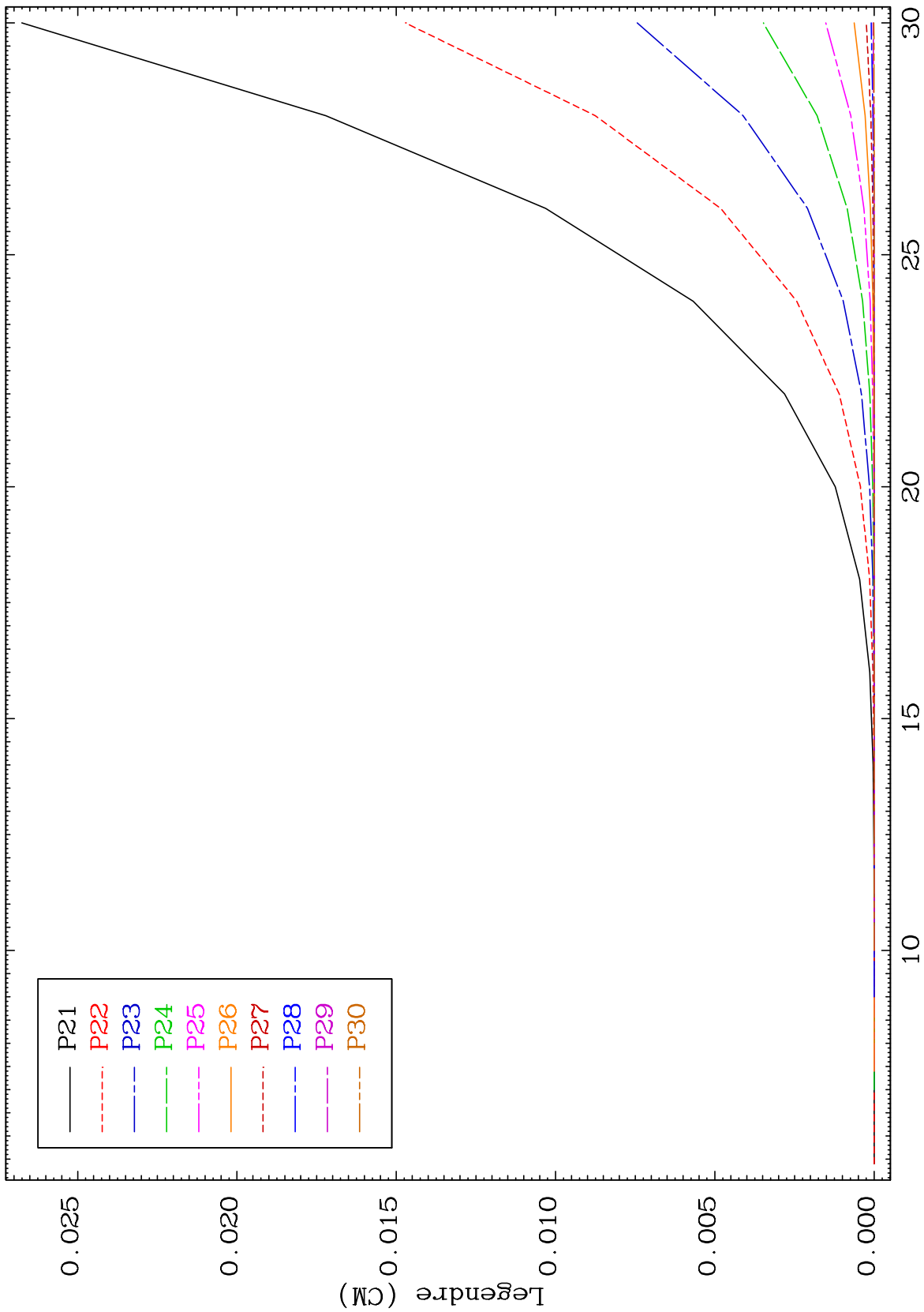
94-Pu-242

Incident Energy (MeV)

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Elastic Legendre Coefficients

94-Pu-242



20

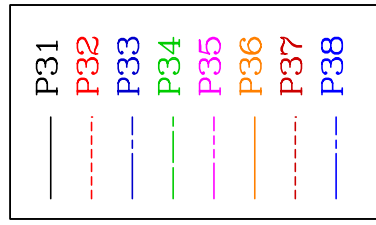
Incident Energy (MeV)

94-Pu-242

MAT 9446

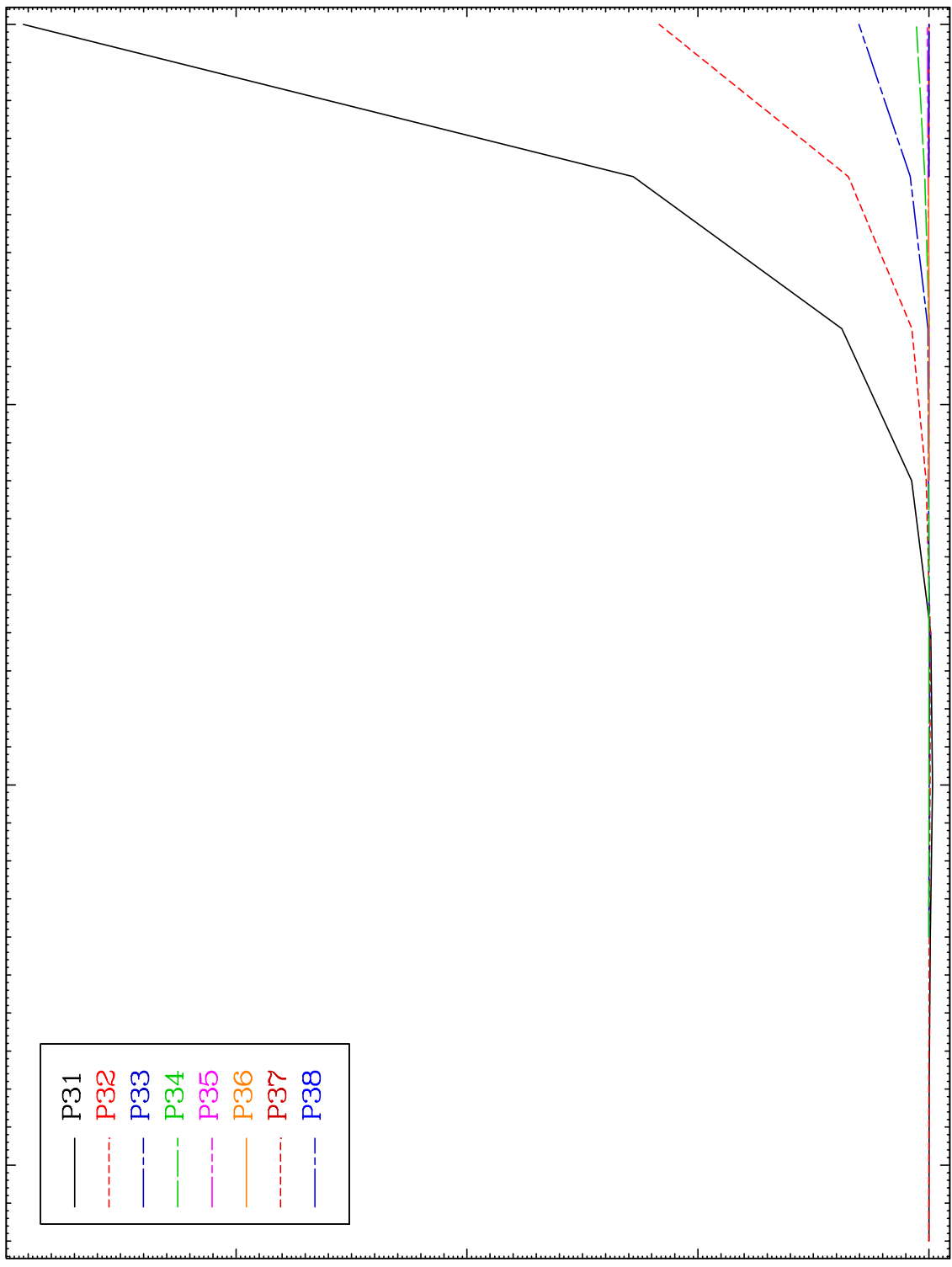
Elastic Legendre Coefficients

94-Pu-242



$\times 10^{-6}$

Legendre (CM)



15

20

25

30

21

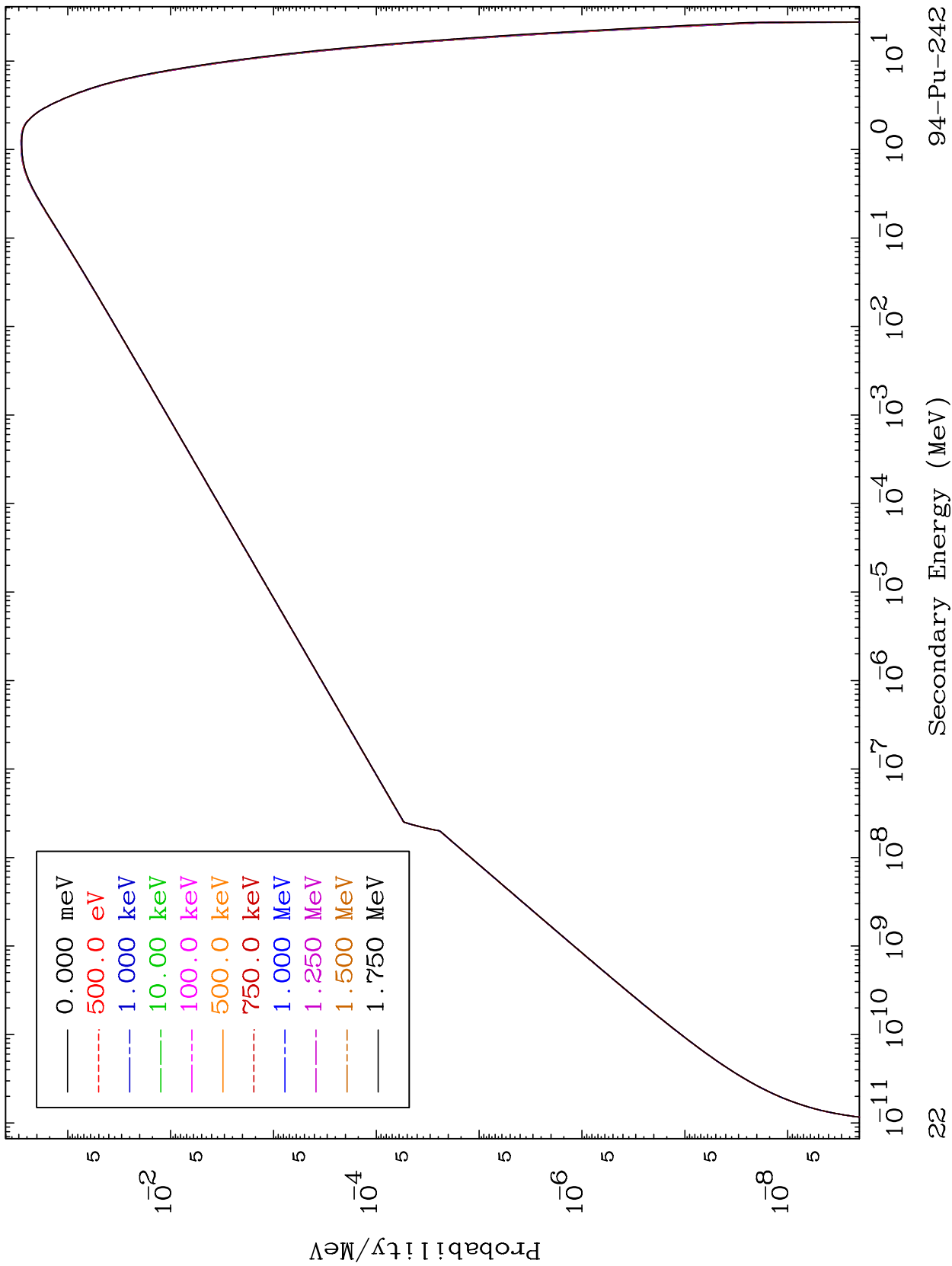
Incident Energy (MeV)

94-Pu-242

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Fission Energy Distribution

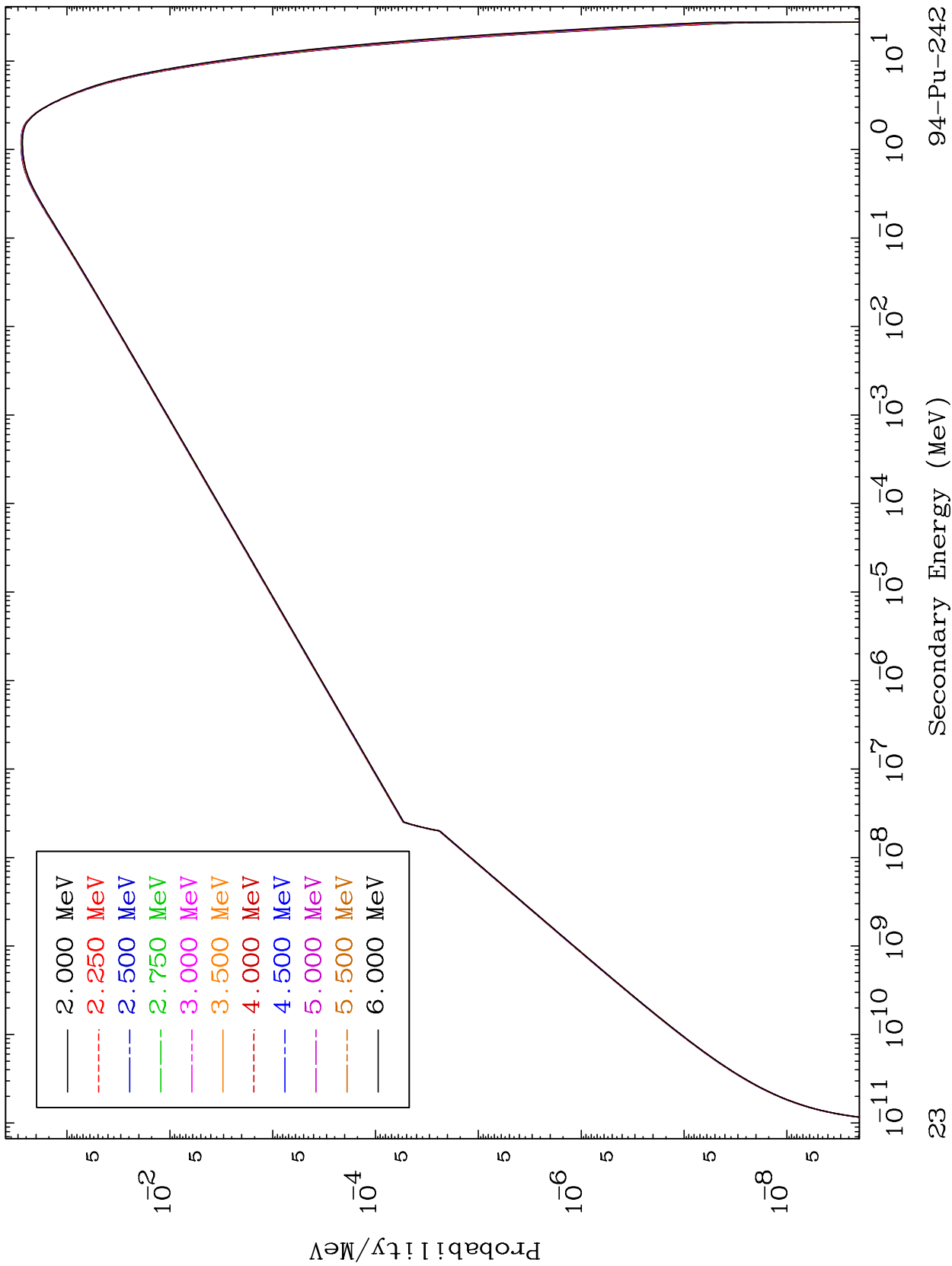
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Fission Energy Distribution

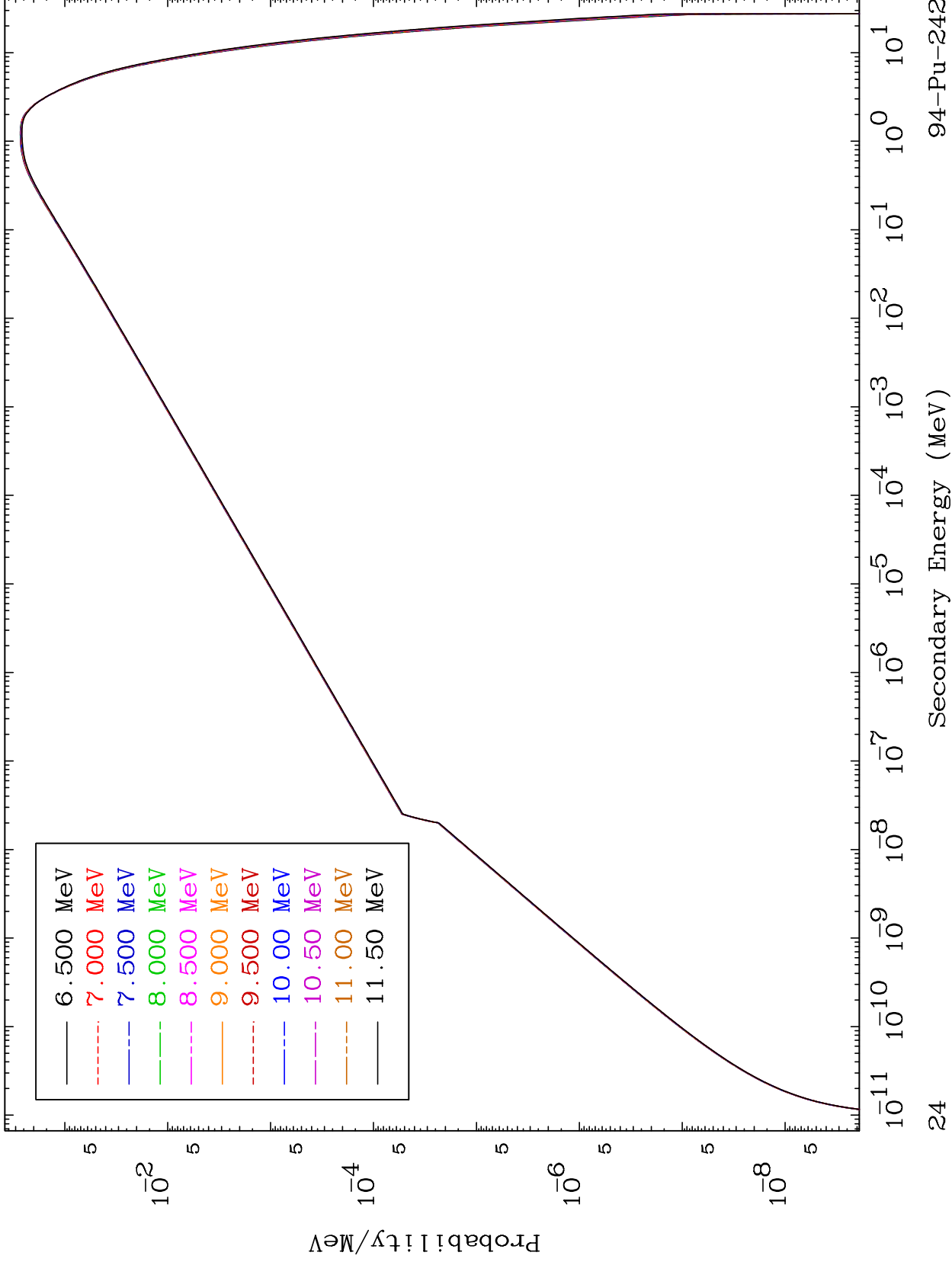
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Fission Energy Distribution

94-Pu-242



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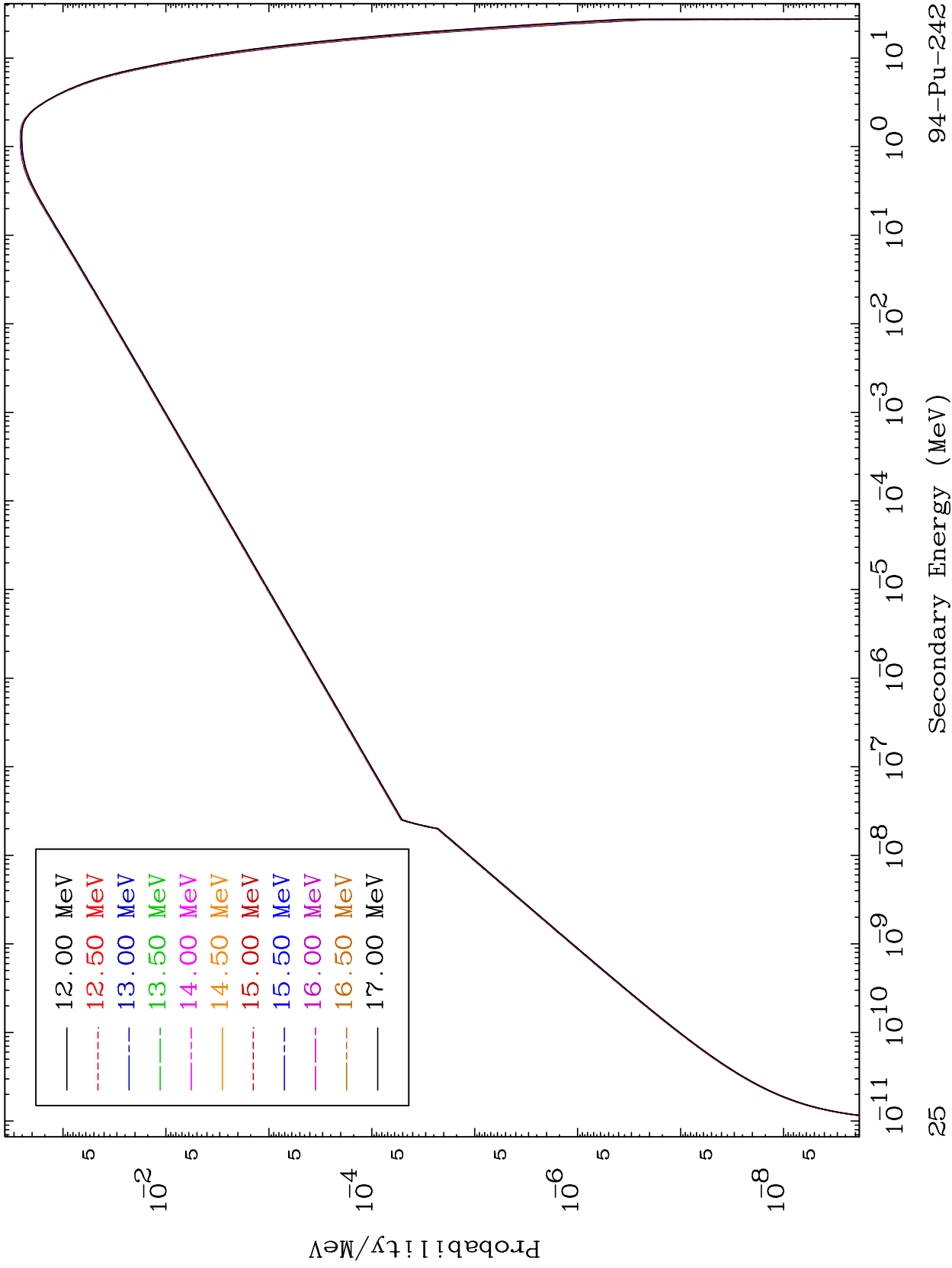
Secondary Energy (MeV)

94-Pu-242

MAT 9446

Fission Energy Distribution

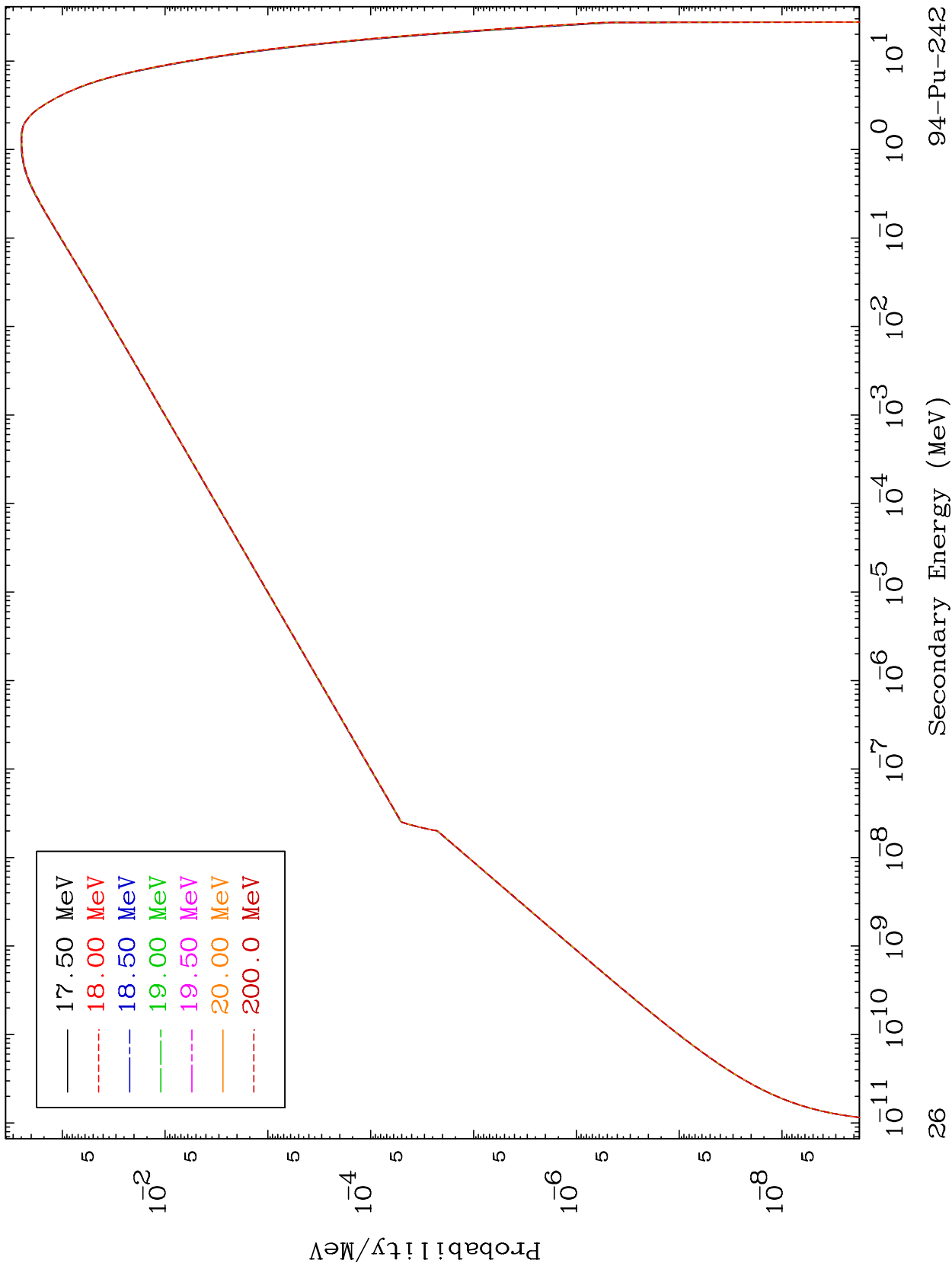
94-Pu-242



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Fission Energy Distribution

94-Pu-242

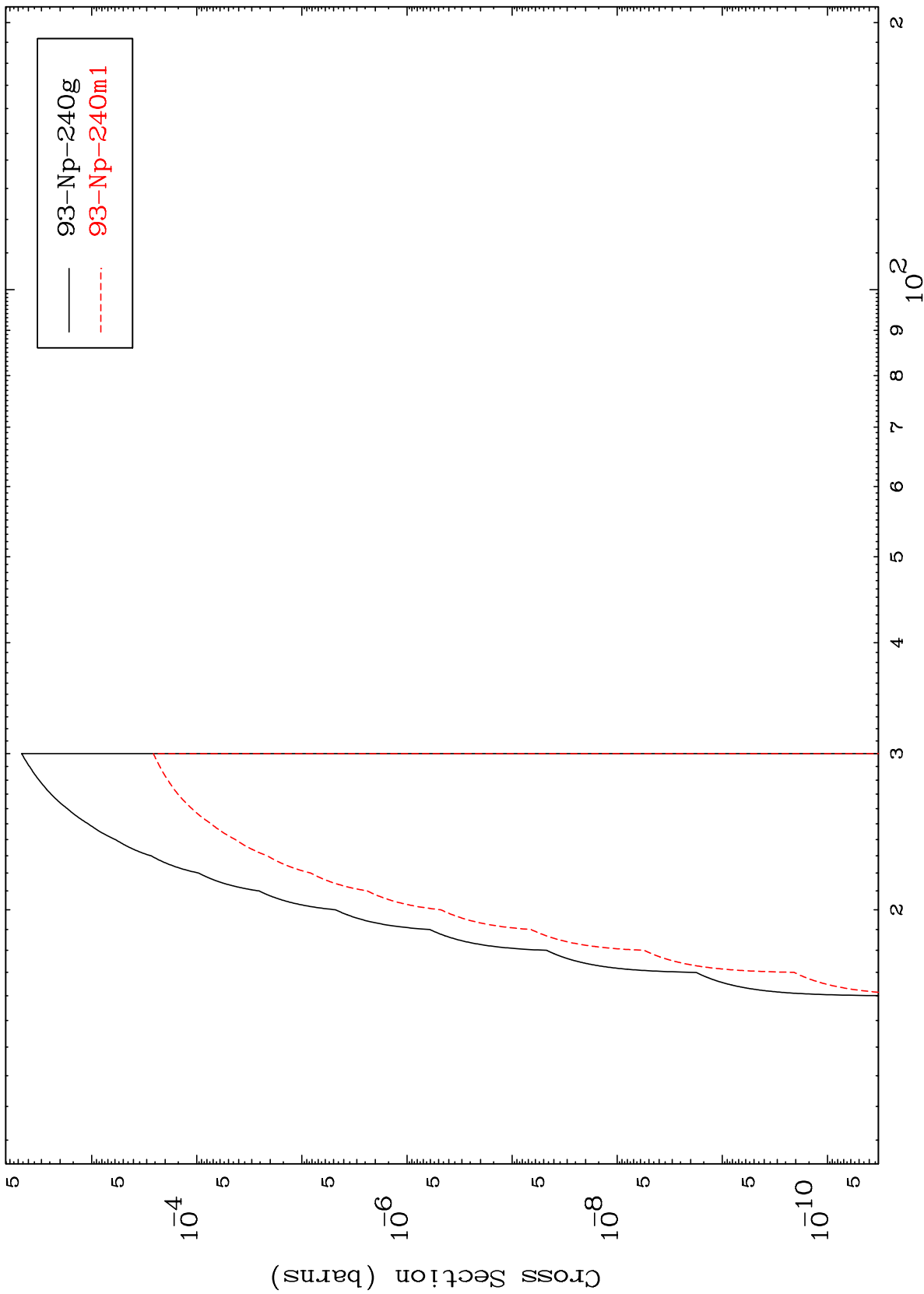


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(n,n') d

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Radionuclide Production Cross Section

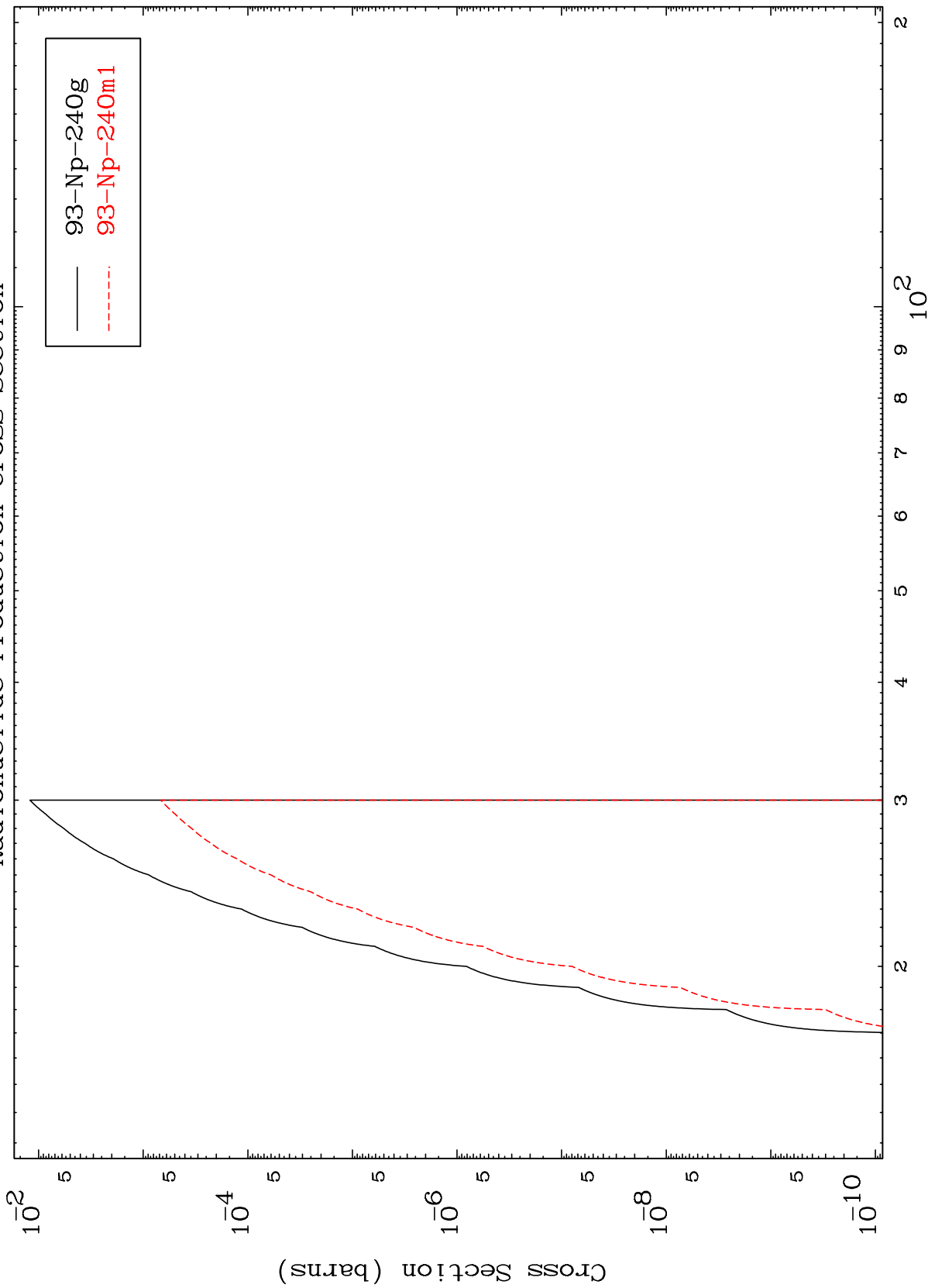


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(n,2n) p

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Radionuclide Production Cross Section



28

Incident Energy (MeV)

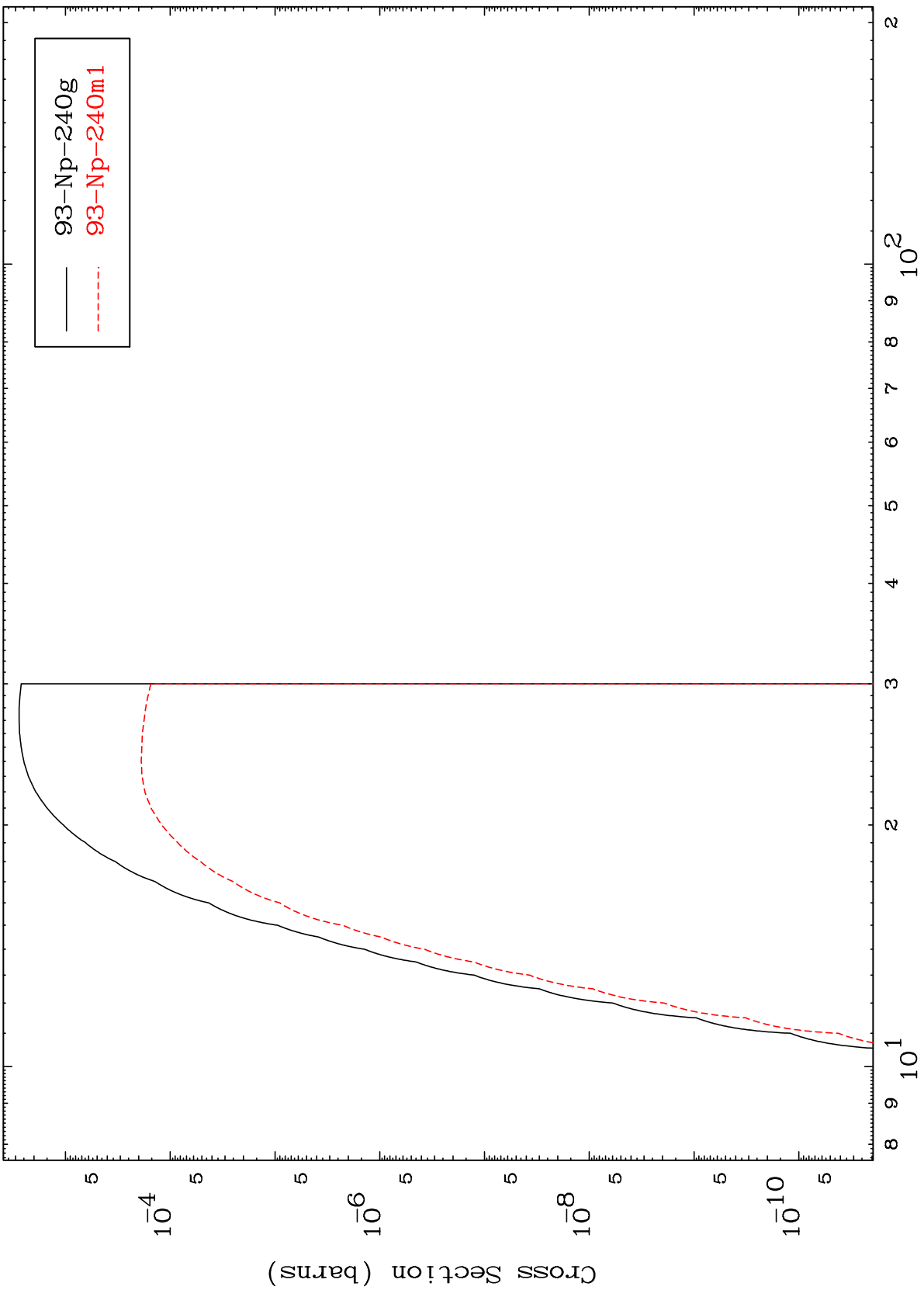
94-Pu-242

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(n, t)

94-Pu-242

Radionuclide Production Cross Section



30

Incident Energy (MeV)

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