

Program EVALPLOT  
(Version 2018-1)

by

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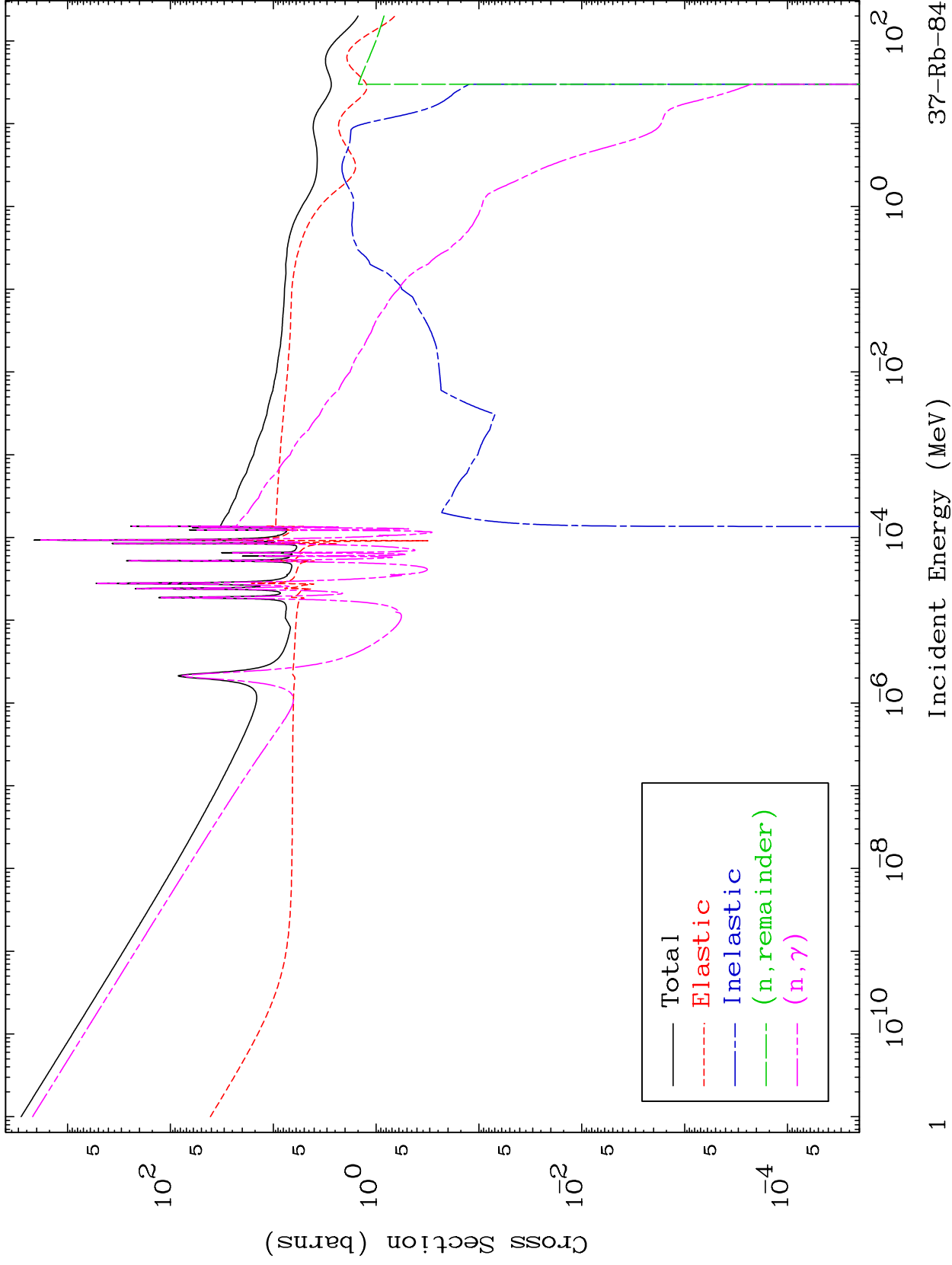
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 3723

Major  
293 Kelvin Cross Sections

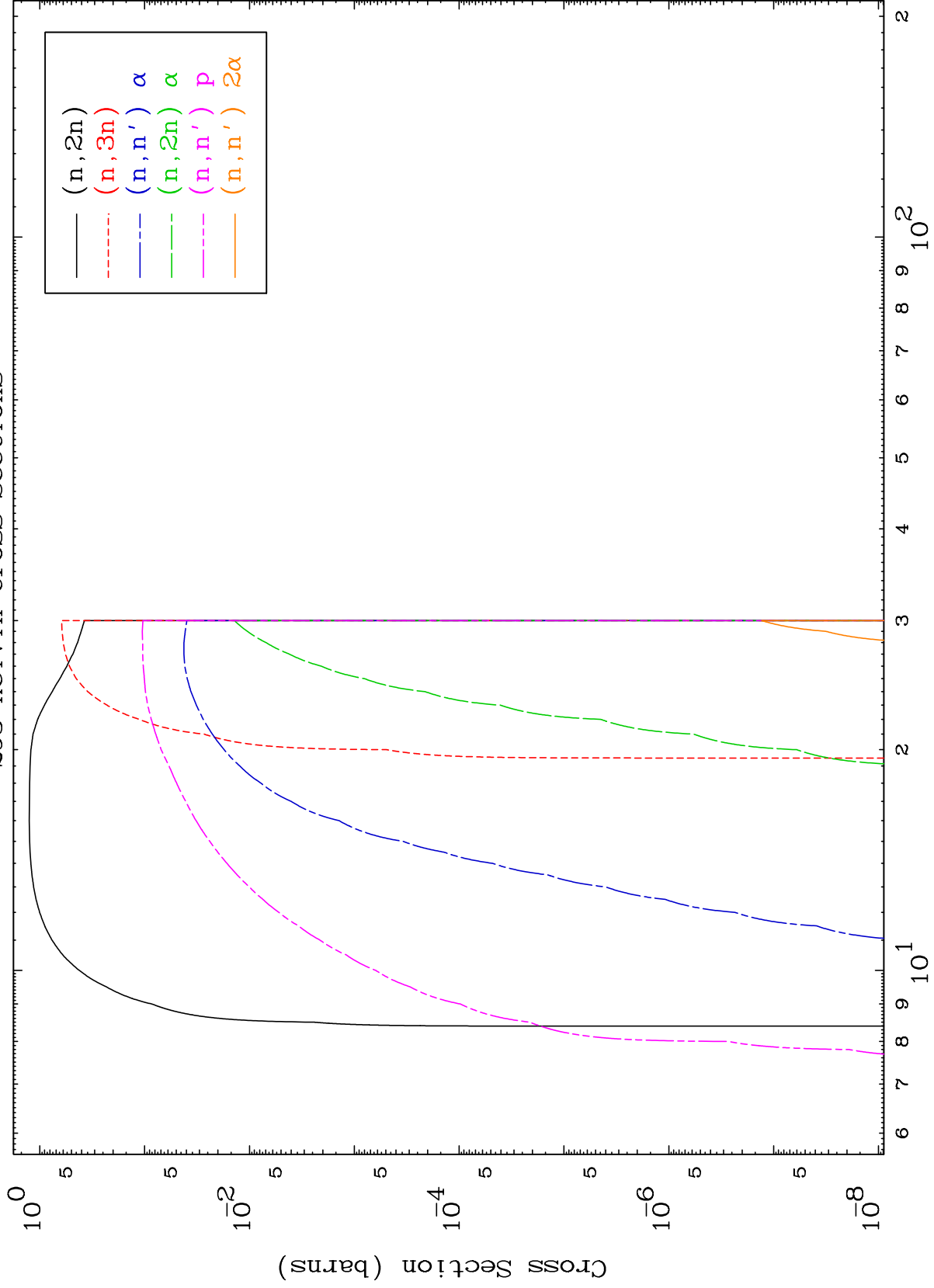
37-Rb-84



MAT 3723

Neutron Production  
293 Kelvin Cross Sections

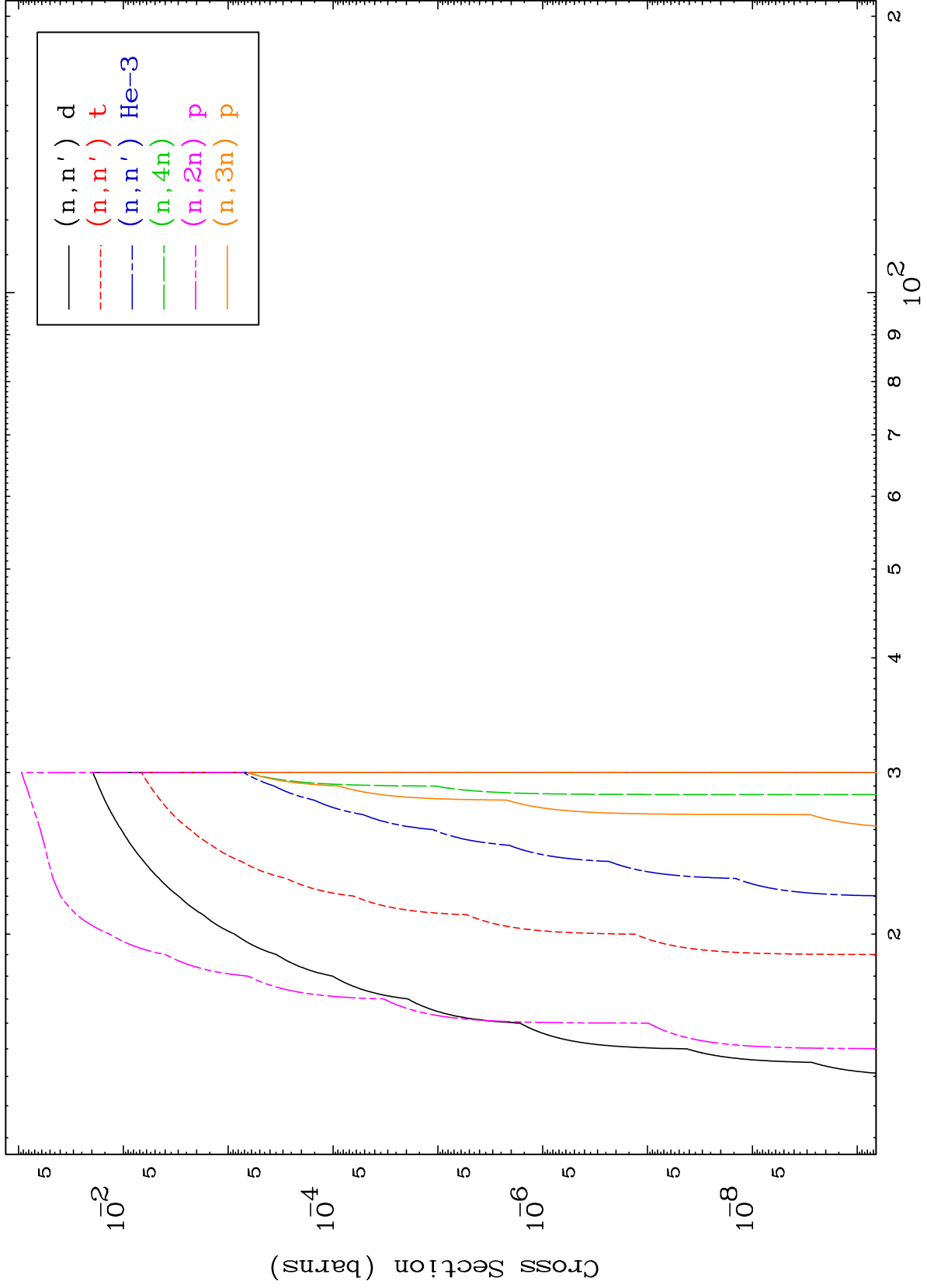
37-Rb-84



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Incident Energy (MeV)

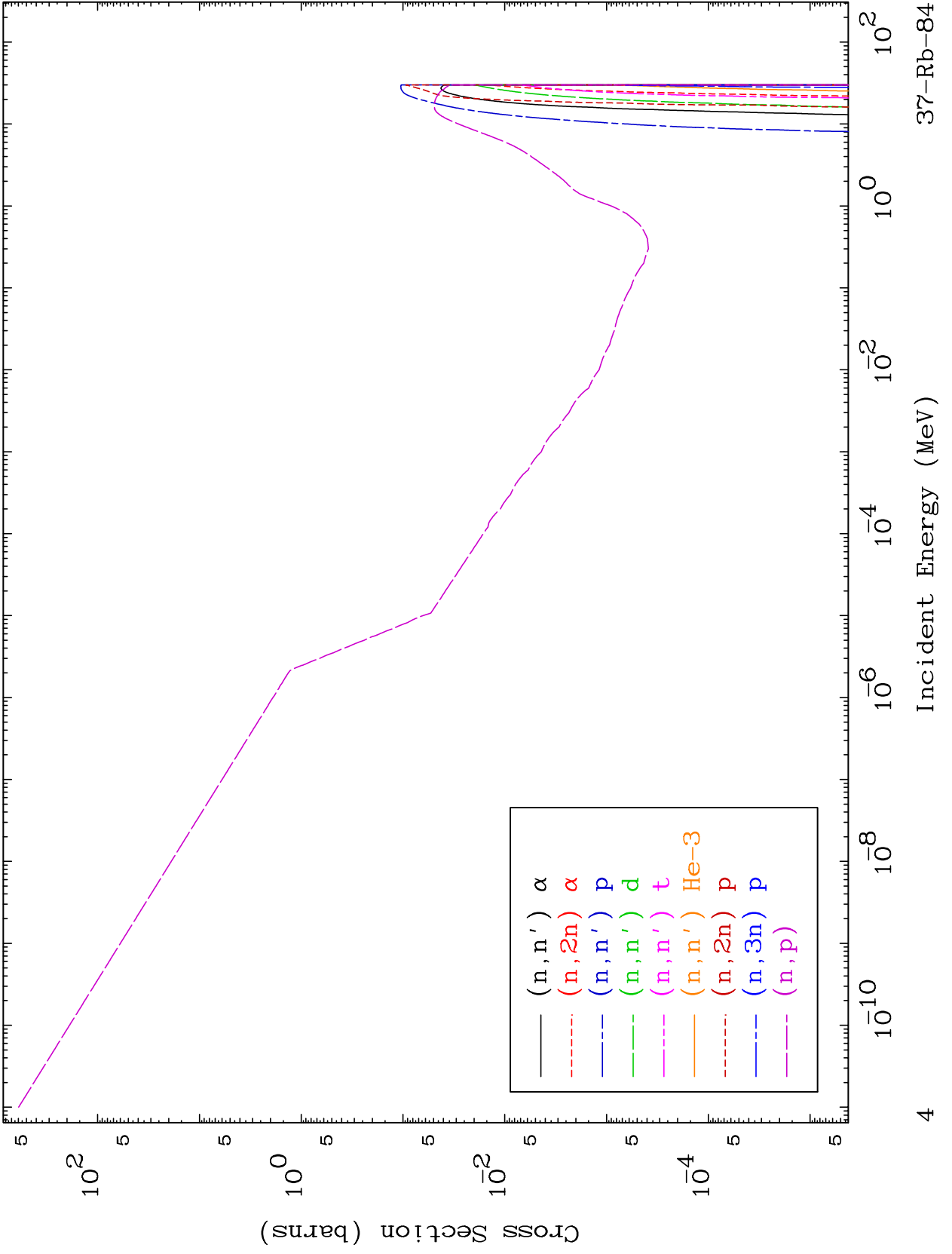
37-Rb-84



MAT 3723

Charged Particle  
293 Kelvin Cross Sections

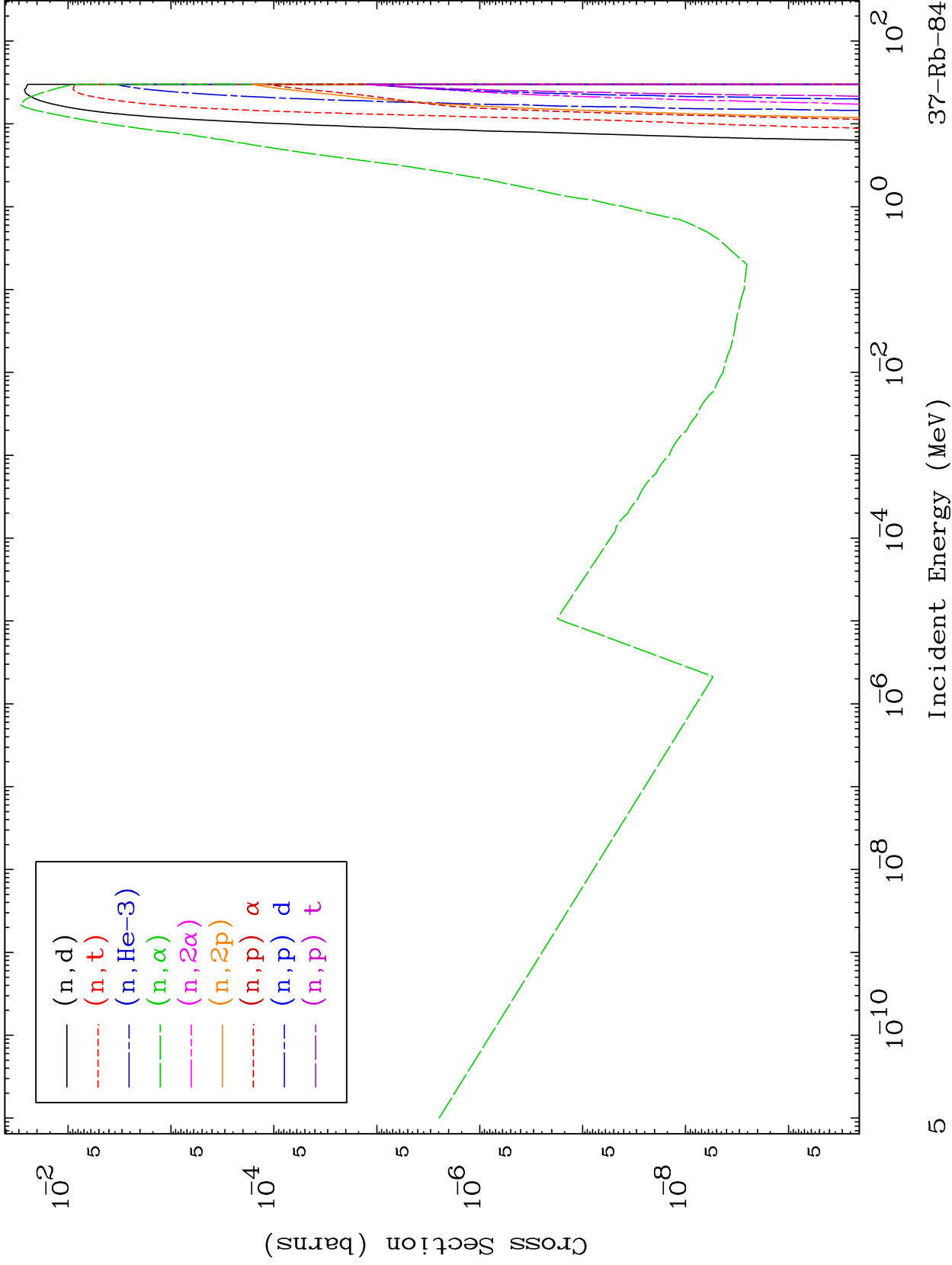
37-Rb-84



MAT 3723

Charged Particle  
293 Kelvin Cross Sections

37-Rb-84



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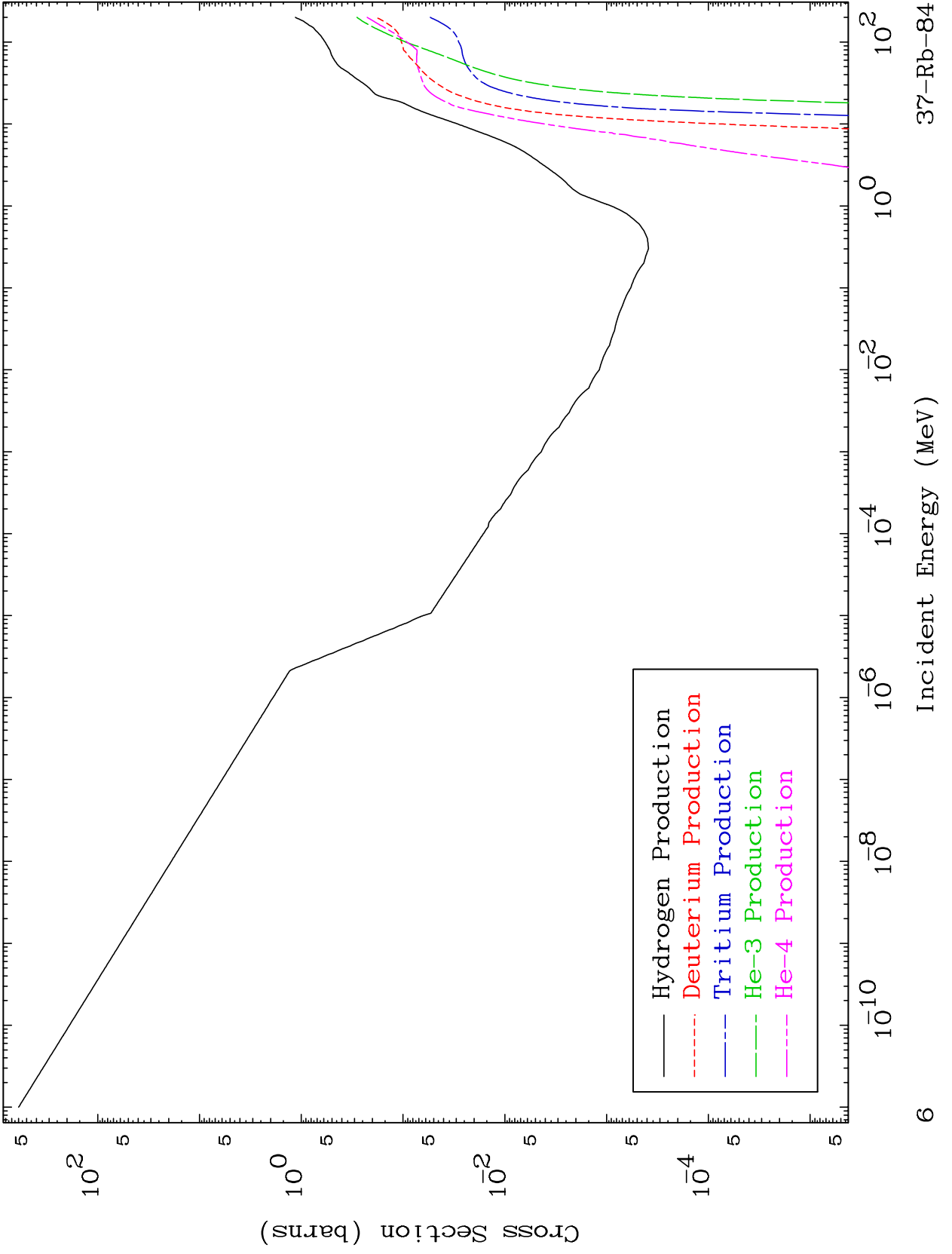
Incident Energy (MeV)

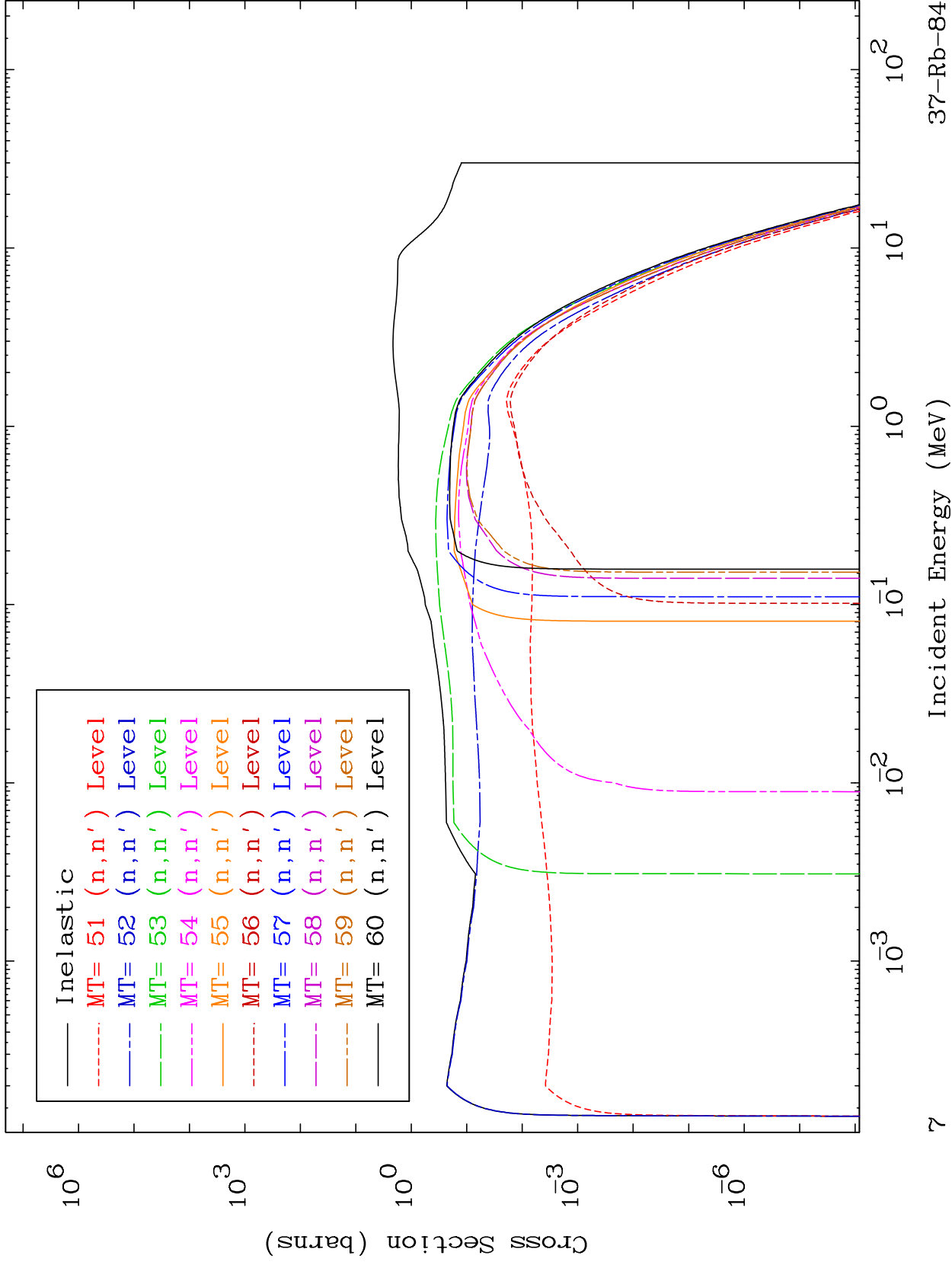
37-Rb-84

MAT 3723

Particle Production  
293 Kelvin Cross Sections

37-Rb-84





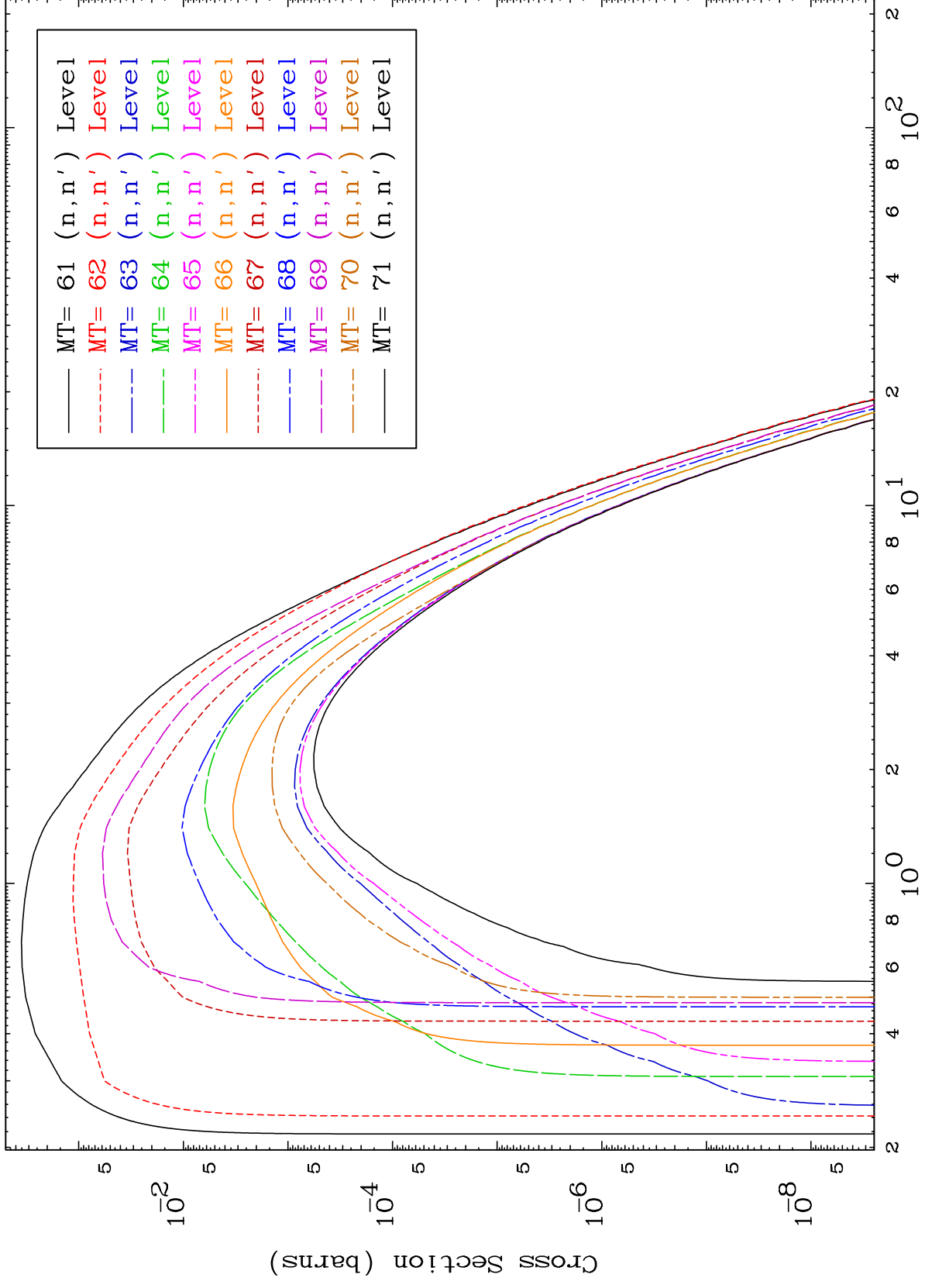


MAT 3723

(n,n') Level

37-Rb-84

293 Kelvin Cross Sections



Incident Energy (MeV)

37-Rb-84

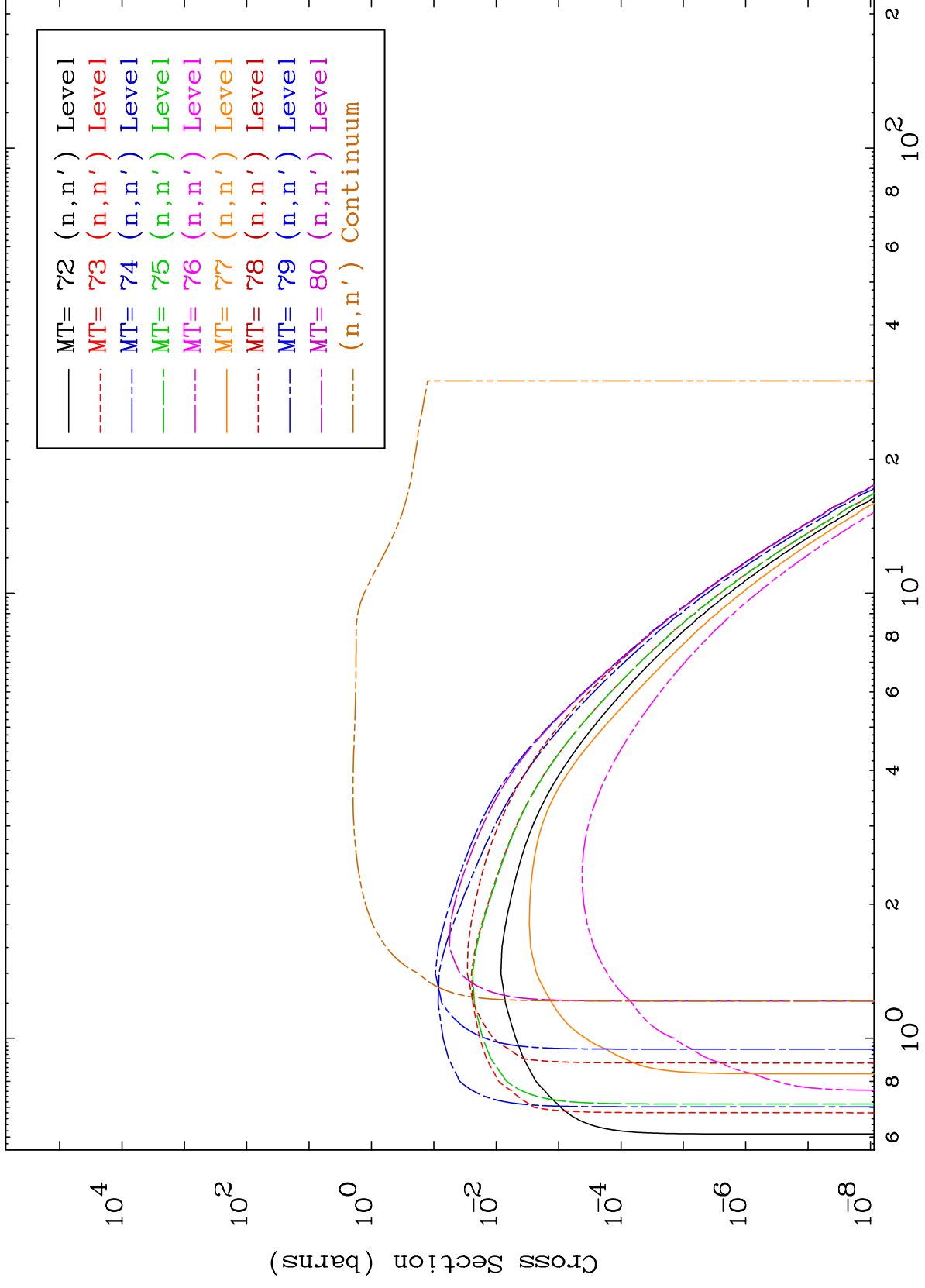
8

MAT 3723

(n,n') Level

37-Rb-84

293 Kelvin Cross Sections



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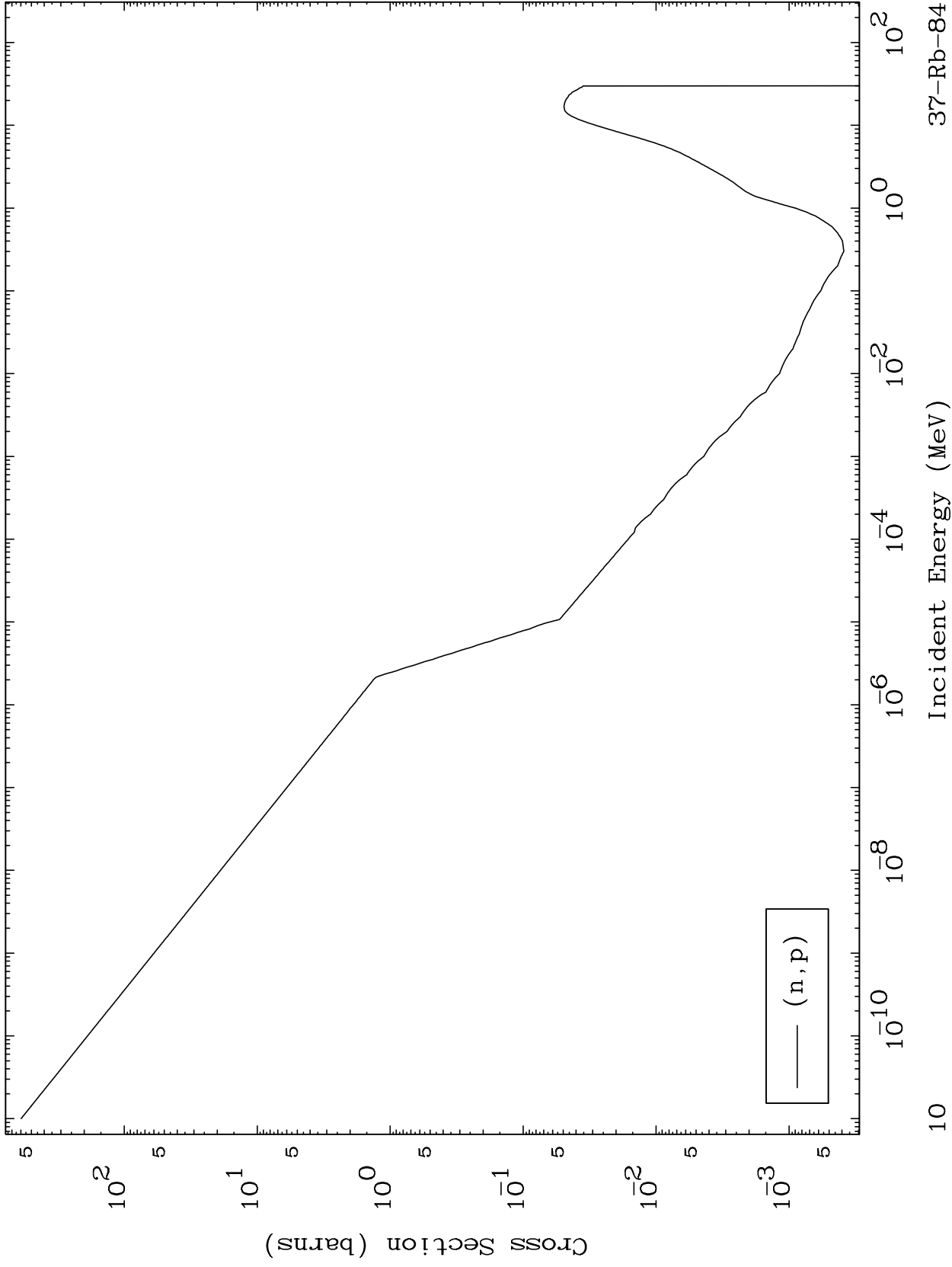
Incident Energy (MeV)

37-Rb-84

MAT 3723

(n,p) Levels  
293 Kelvin Cross Sections

37-Rb-84



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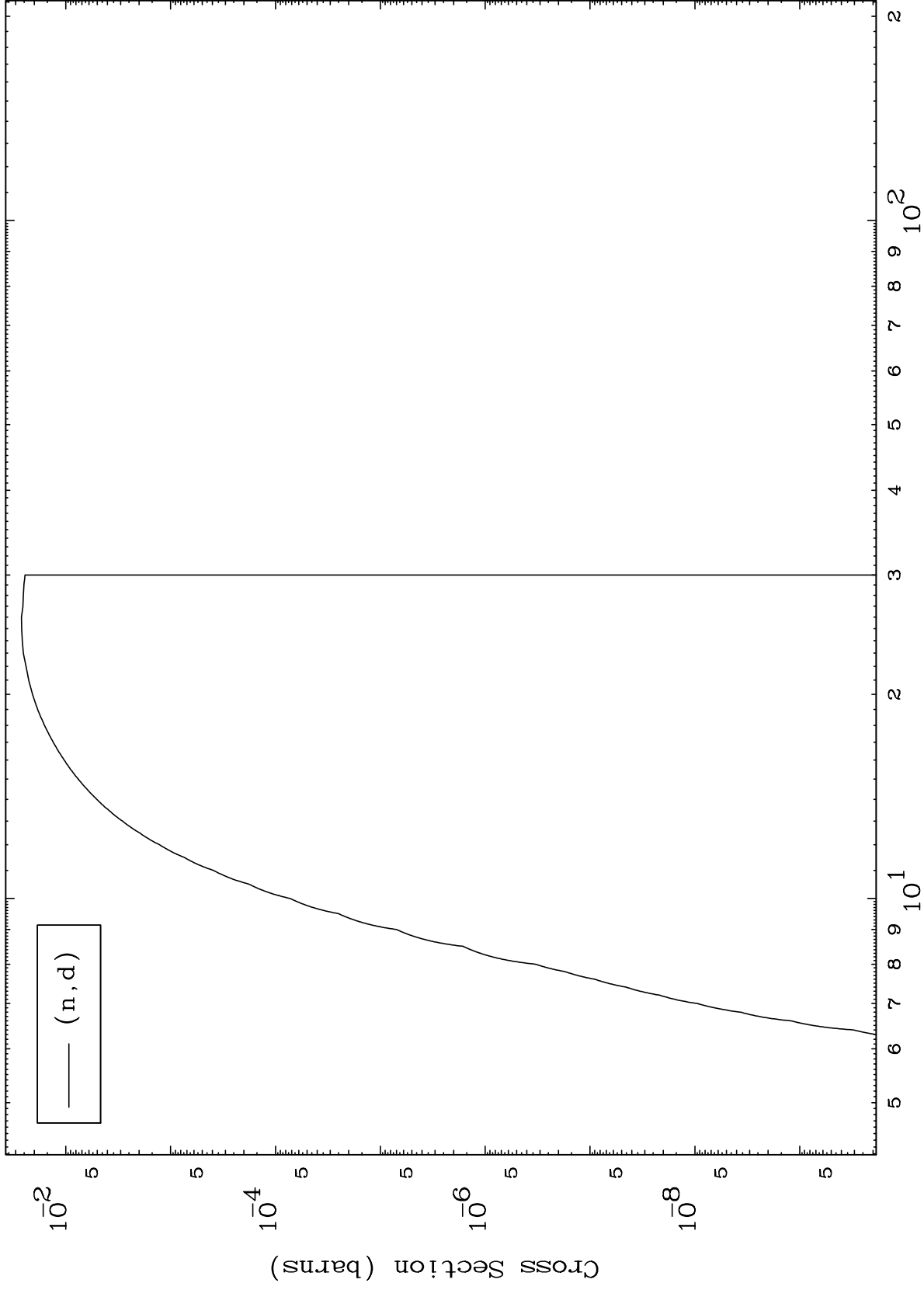
Incident Energy (MeV)

37-Rb-84

MAT 3723

(n,d) Levels  
293 Kelvin Cross Sections

37-Rb-84



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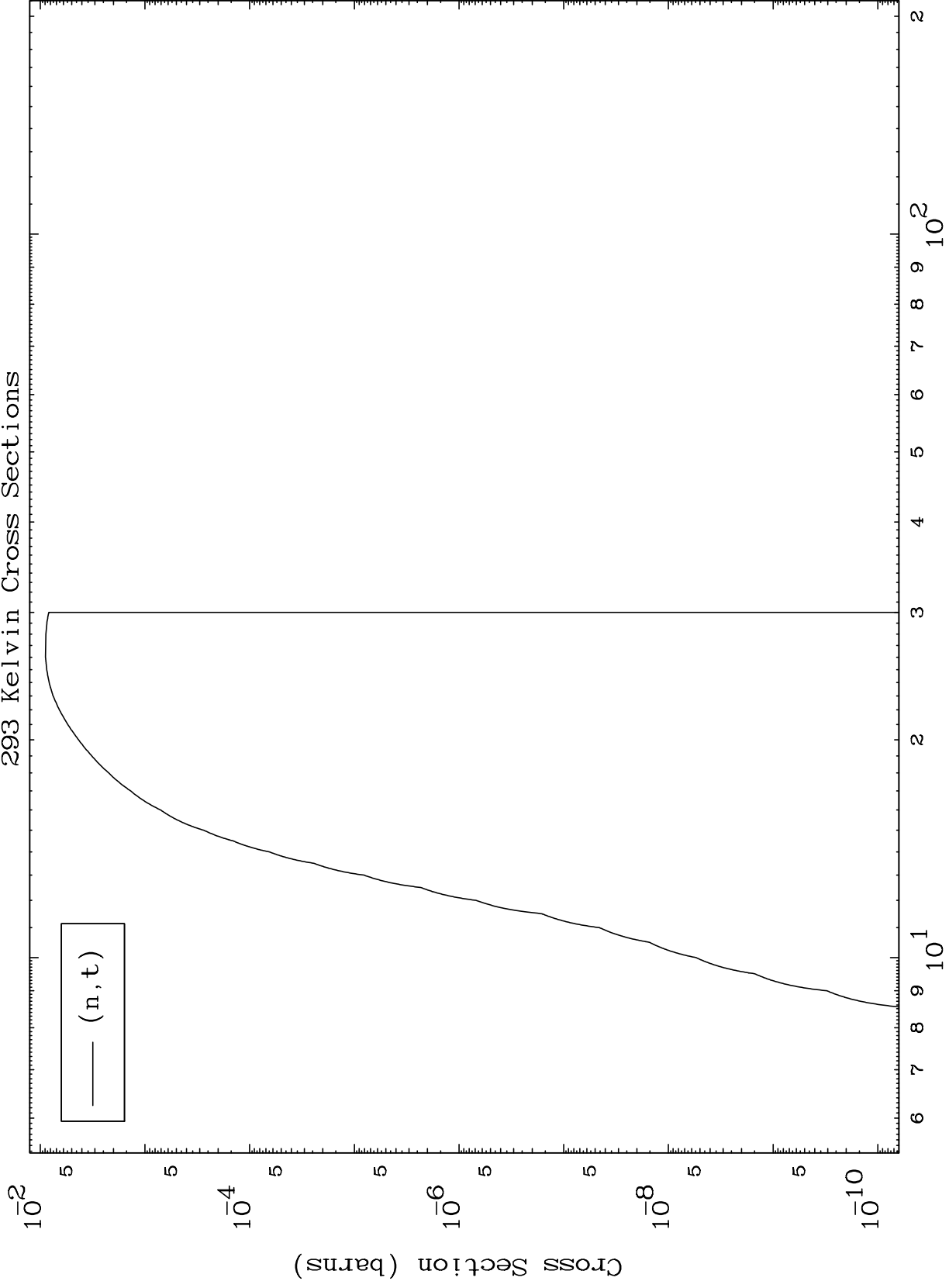
Incident Energy (MeV)

37-Rb-84

MAT 3723

(n,t) Levels  
293 Kelvin Cross Sections

37-Rb-84



12

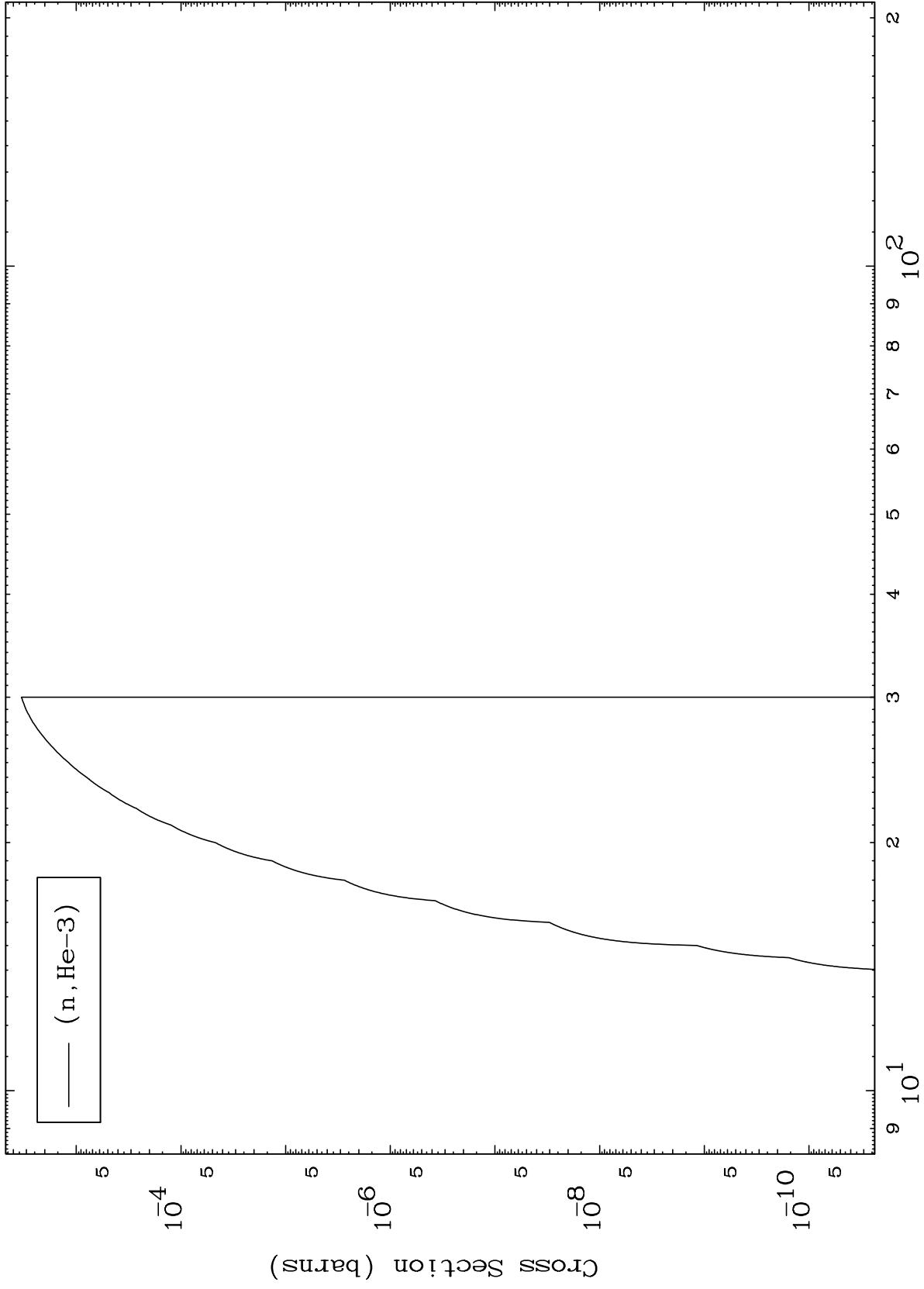
Incident Energy (MeV)

37-Rb-84

MAT 3723

(n,He3) Levels  
293 Kelvin Cross Sections

37-Rb-84



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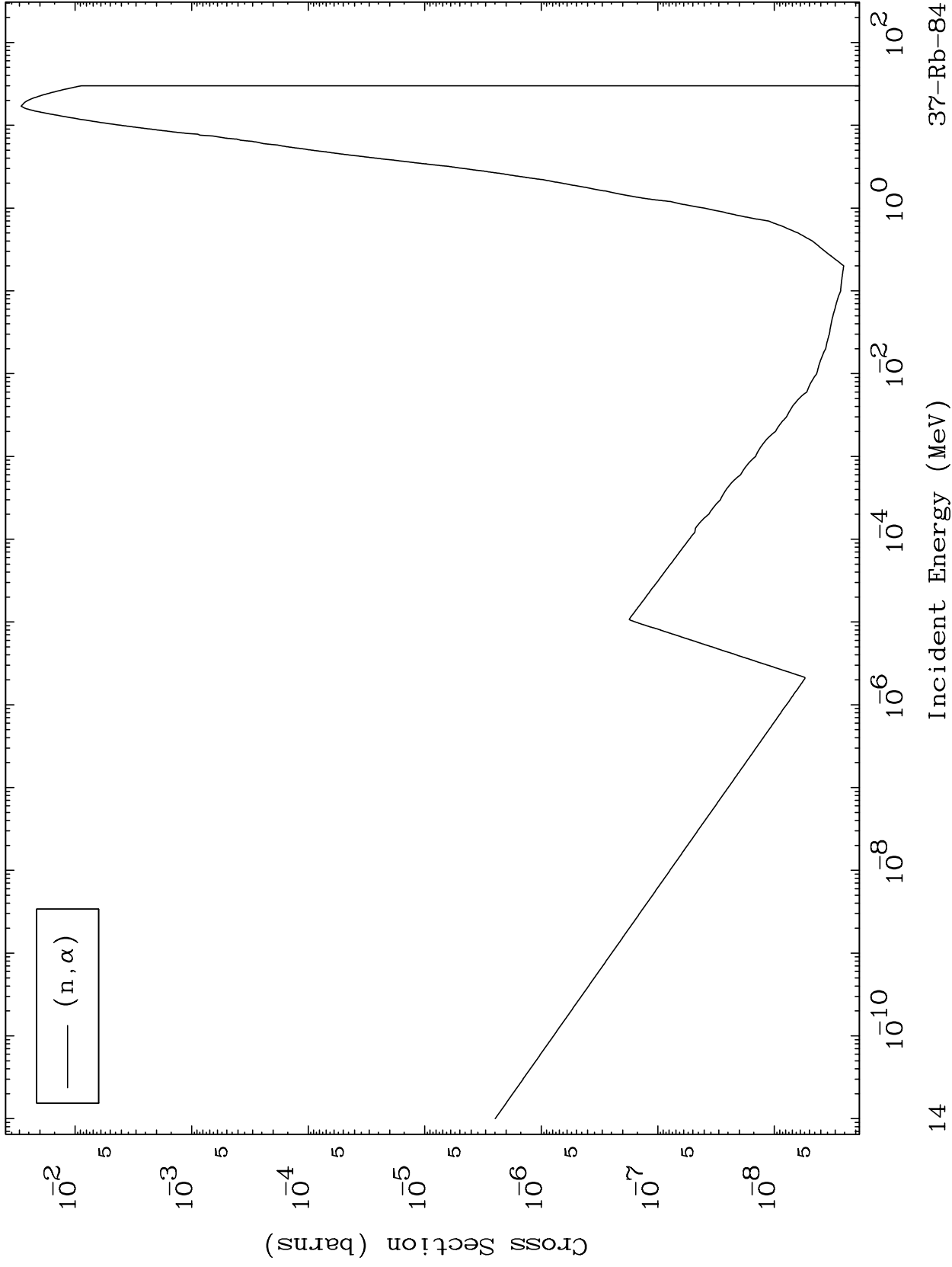
Incident Energy (MeV)

37-Rb-84

MAT 3723

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

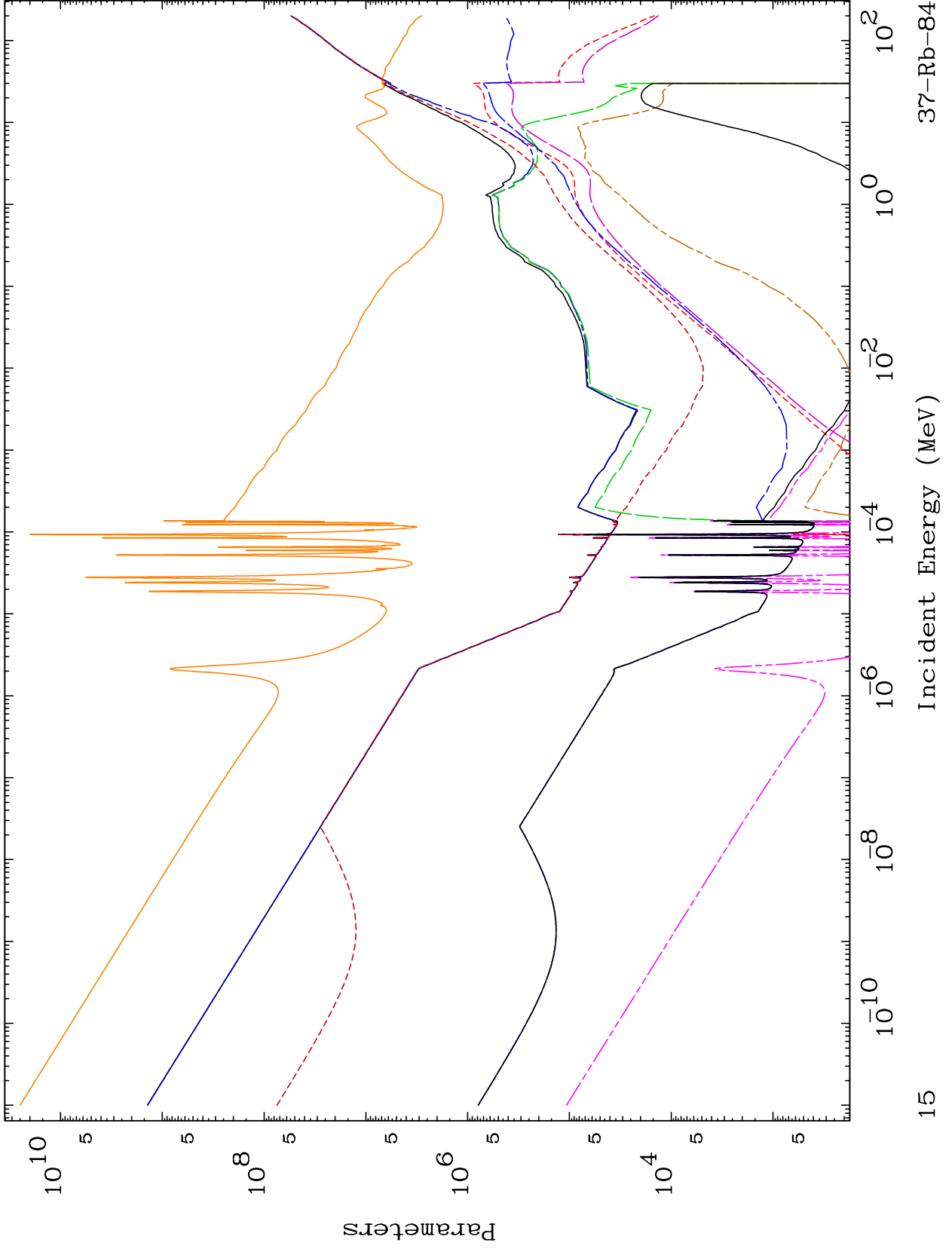
37-Rb-84



MAT 3723

Energy Release  
Parameters

37-Rb-84



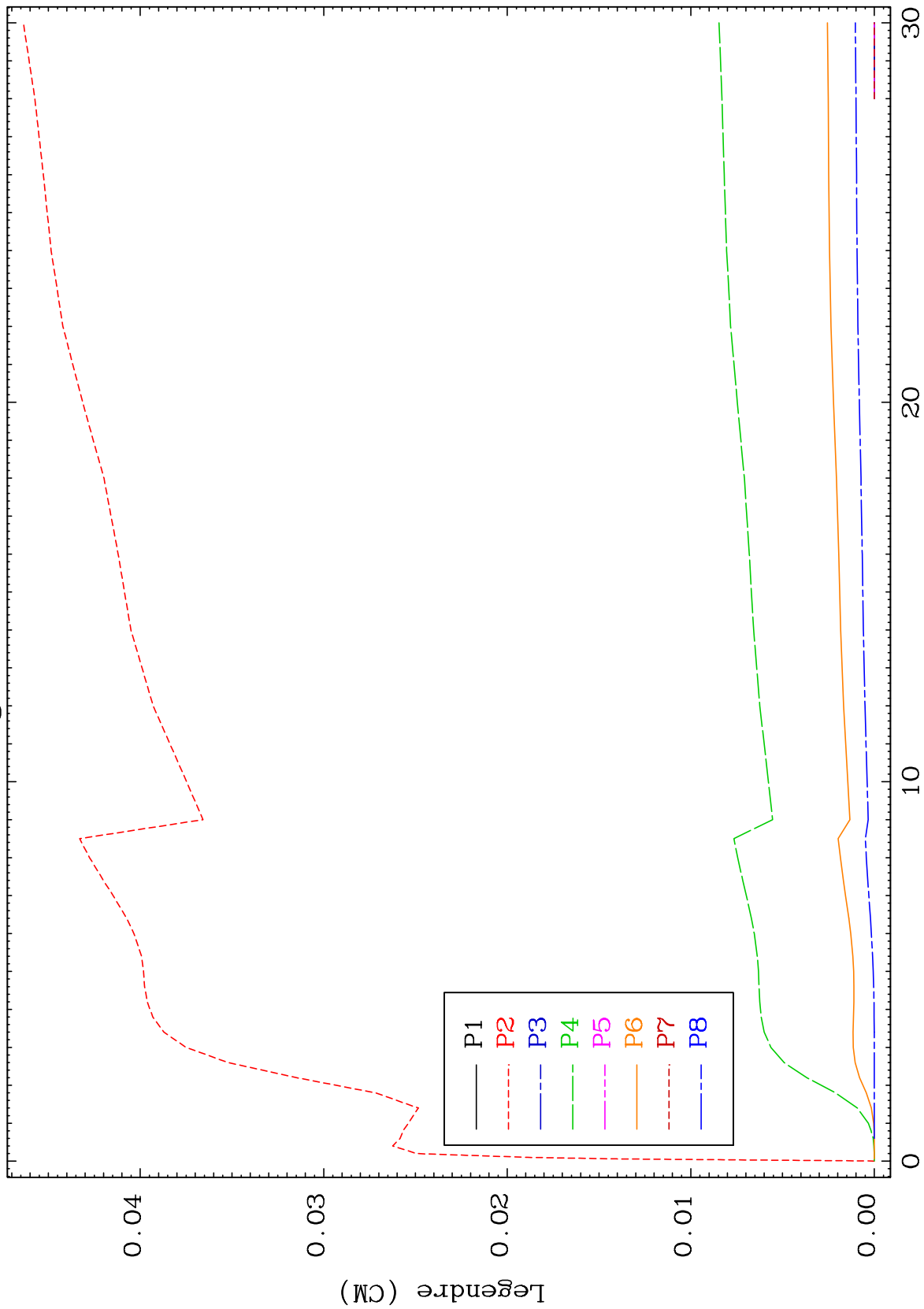
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Elastic Legendre Coefficients

37-Rb-84



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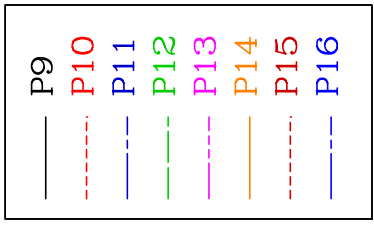
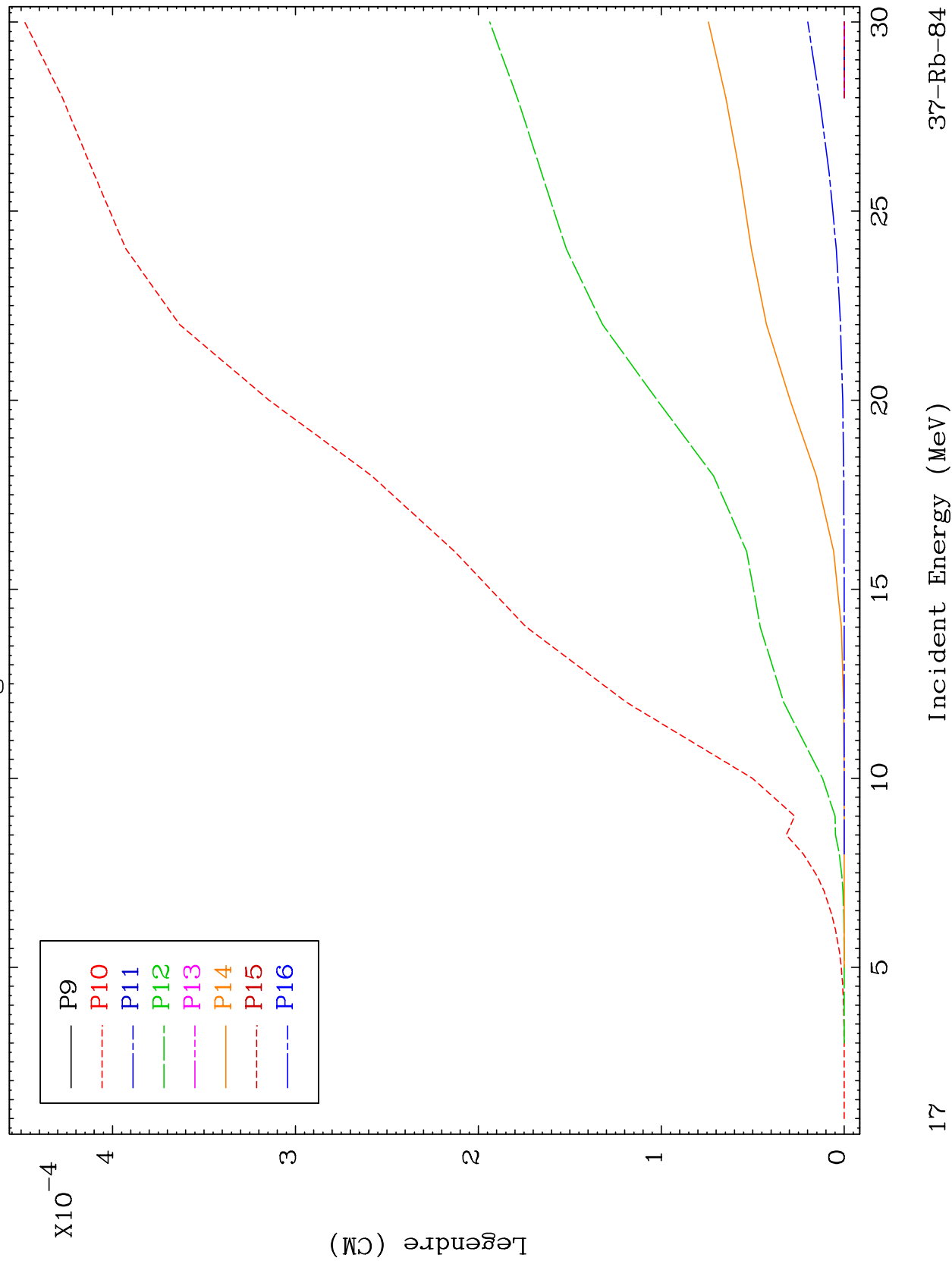
Incident Energy (MeV)

37-Rb-84

MAT 3723

Elastic Legendre Coefficients

37-Rb-84

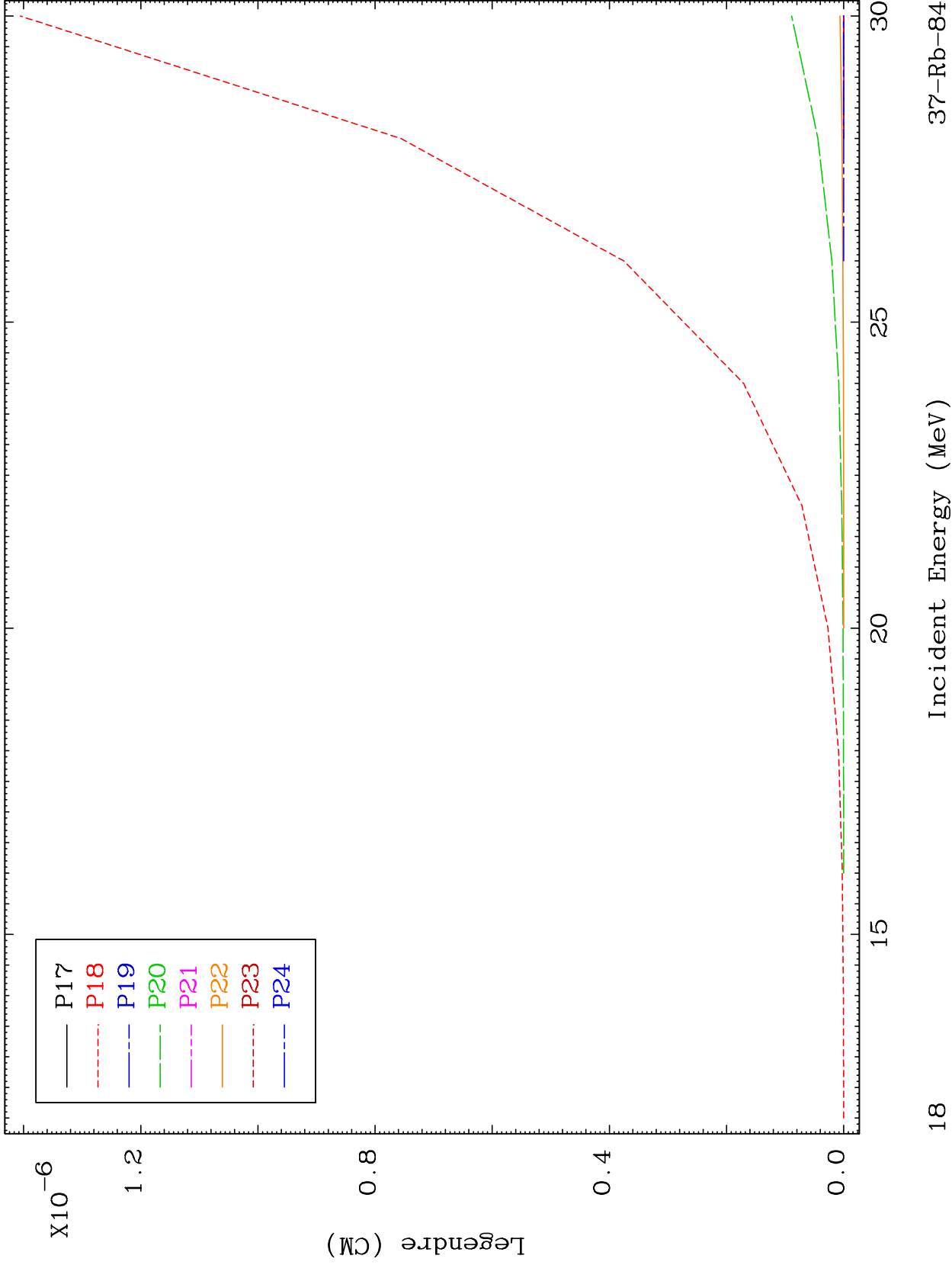


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Elastic Legendre Coefficients

37-Rb-84



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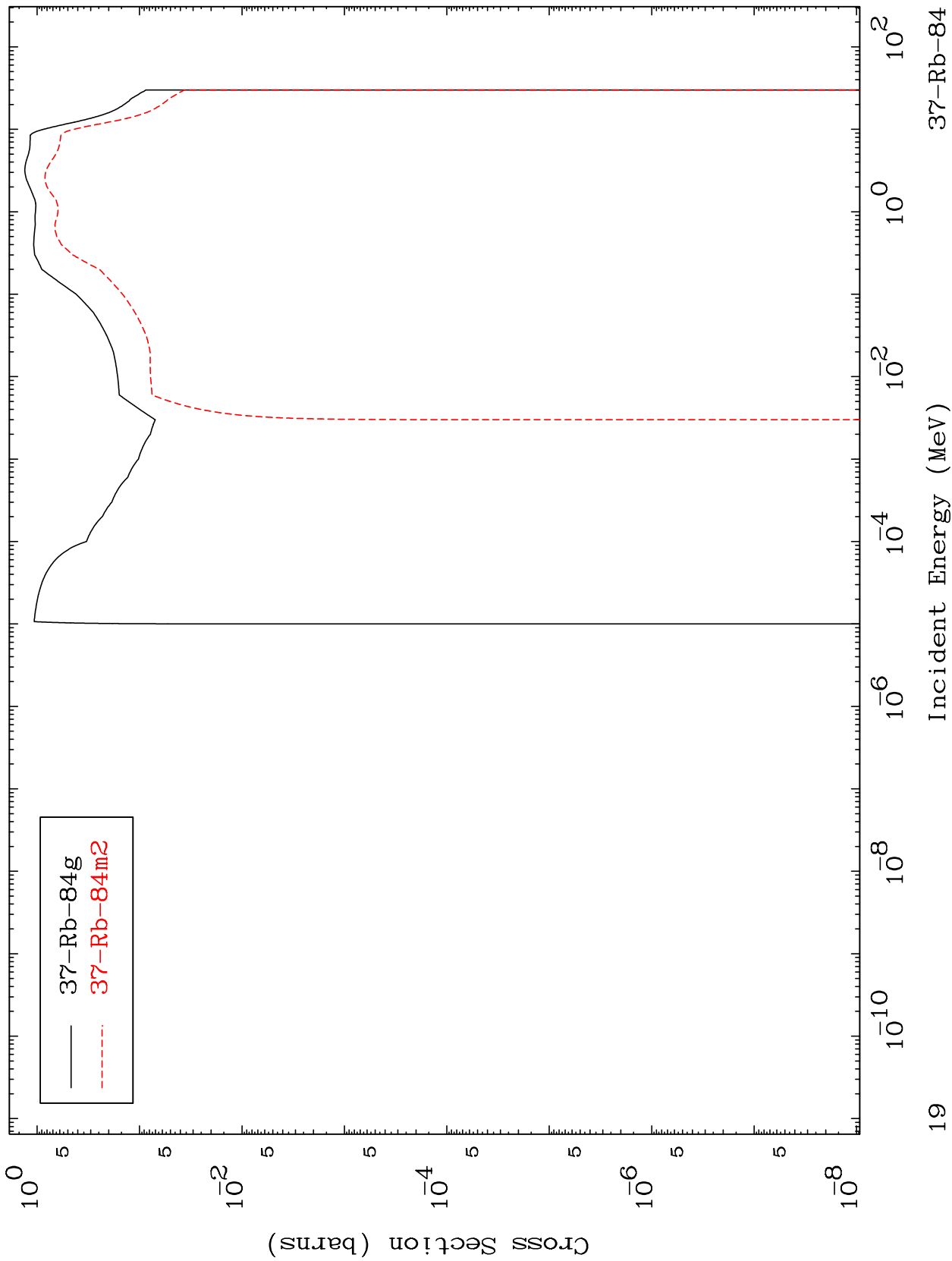
Incident Energy (MeV)

37-Rb-84

MAT 3723

37-Rb-84

Inelastic  
Radionuclide Production Cross Section

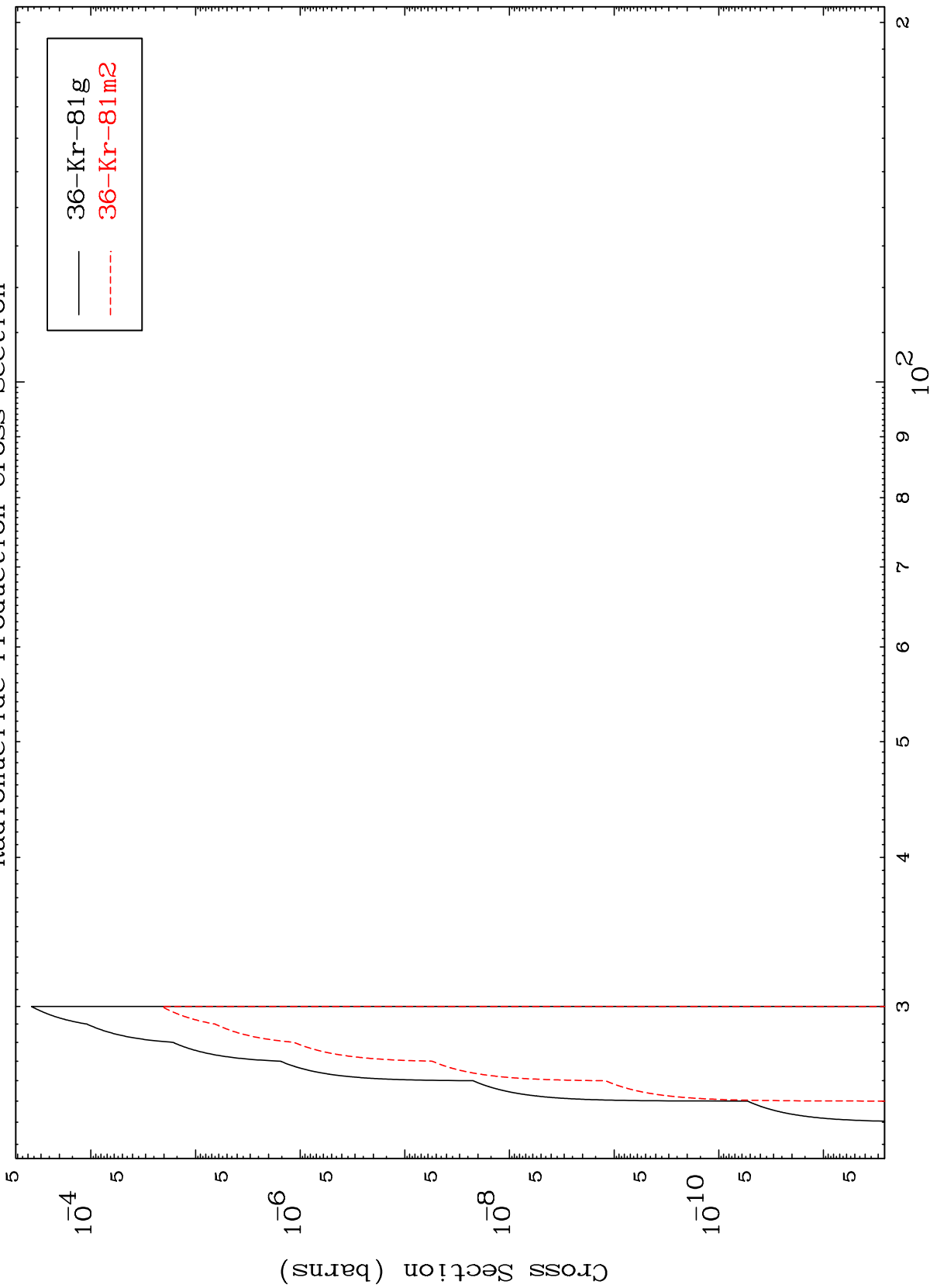


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(n,2n) d

37-Rb-84

Radionuclide Production Cross Section



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Incident Energy (MeV)

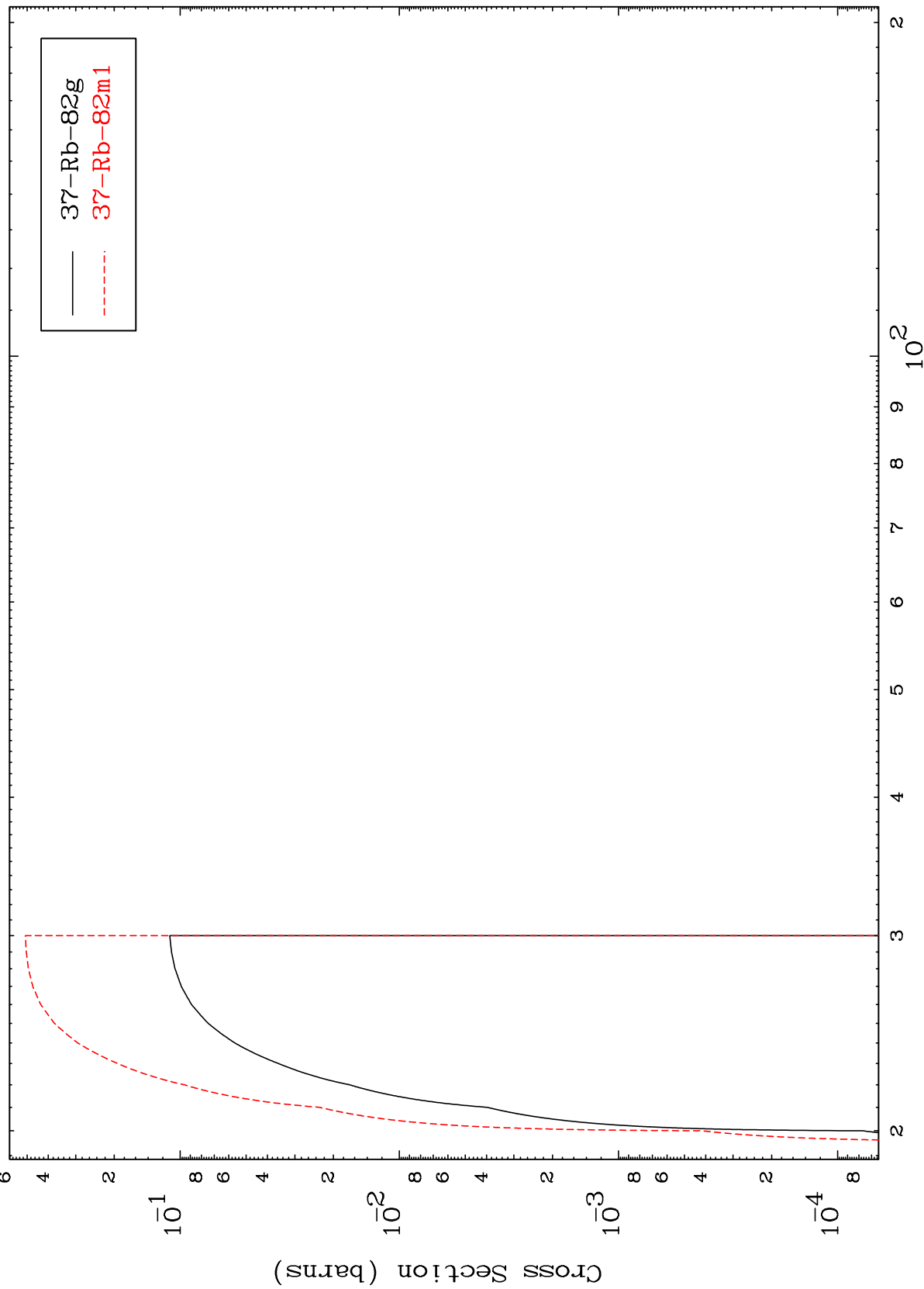
37-Rb-84

MAT 3723

<sup>37</sup>Rb-84

(n,3n)

Radionuclide Production Cross Section



21

Incident Energy (MeV)

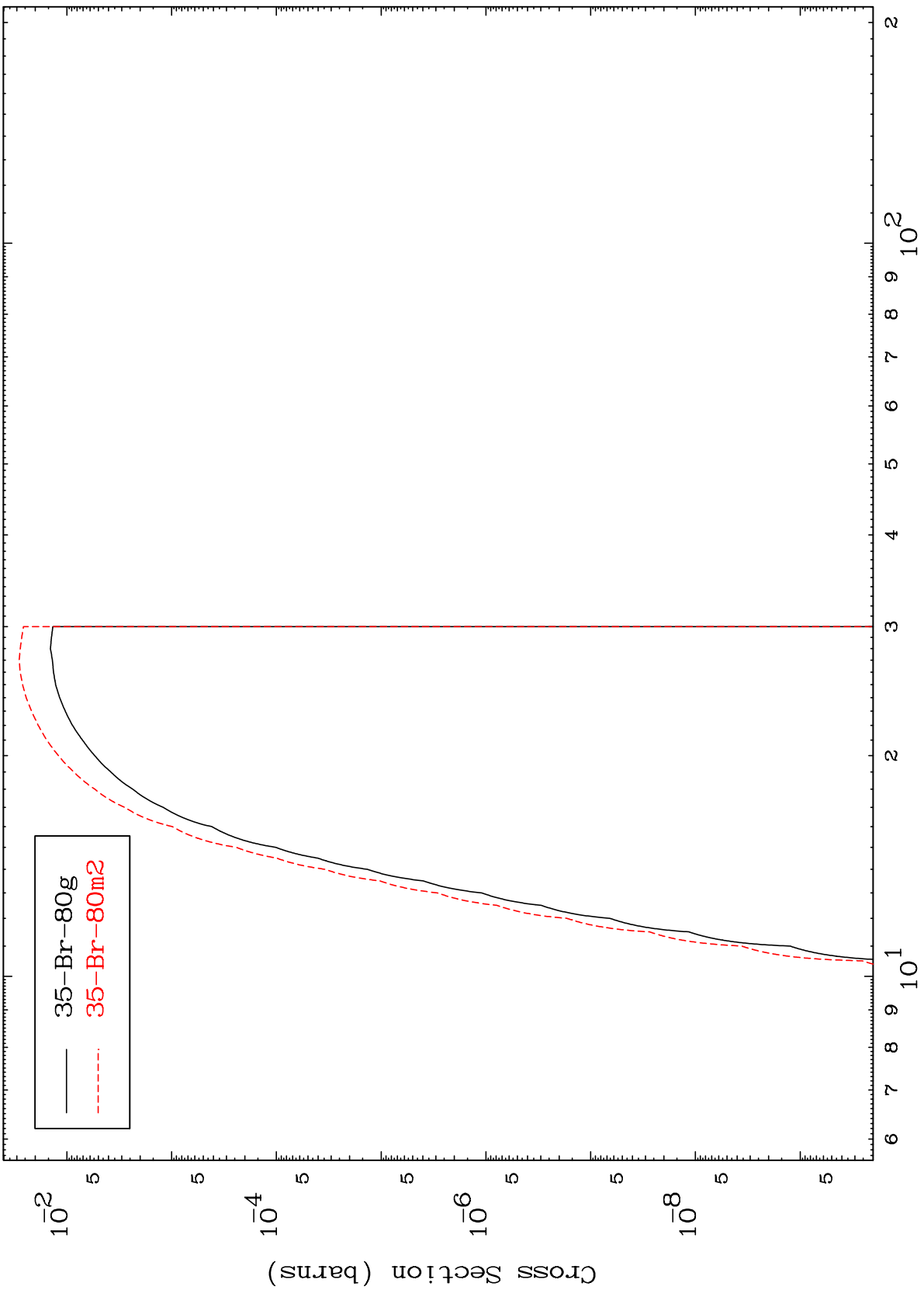
<sup>37</sup>Rb-84

MAT 3723

37-Rb-84

(n,n')  $\alpha$

Radionuclide Production Cross Section



22

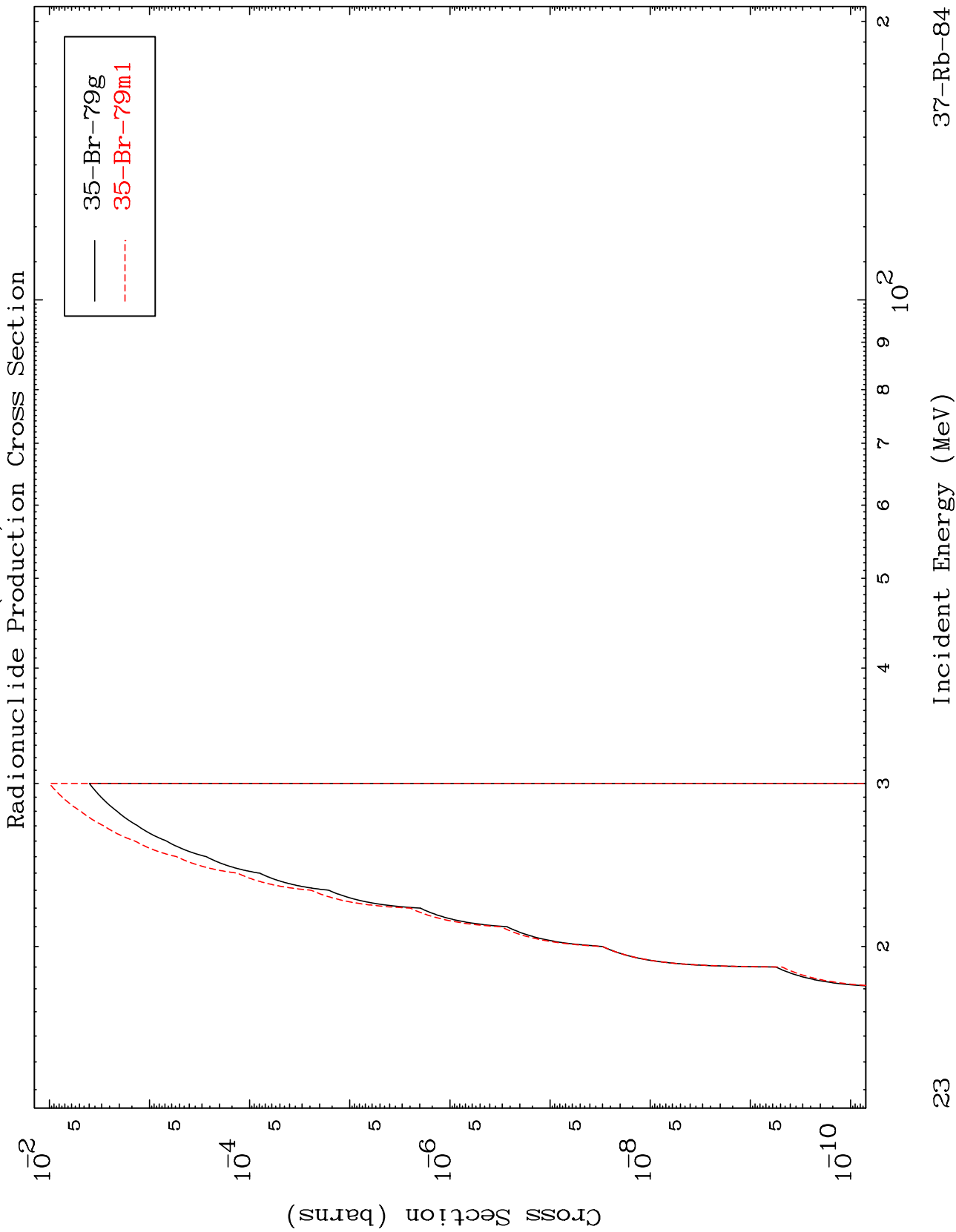
Incident Energy (MeV)

37-Rb-84

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$(n,2n) \alpha$

37-Rb-84



23

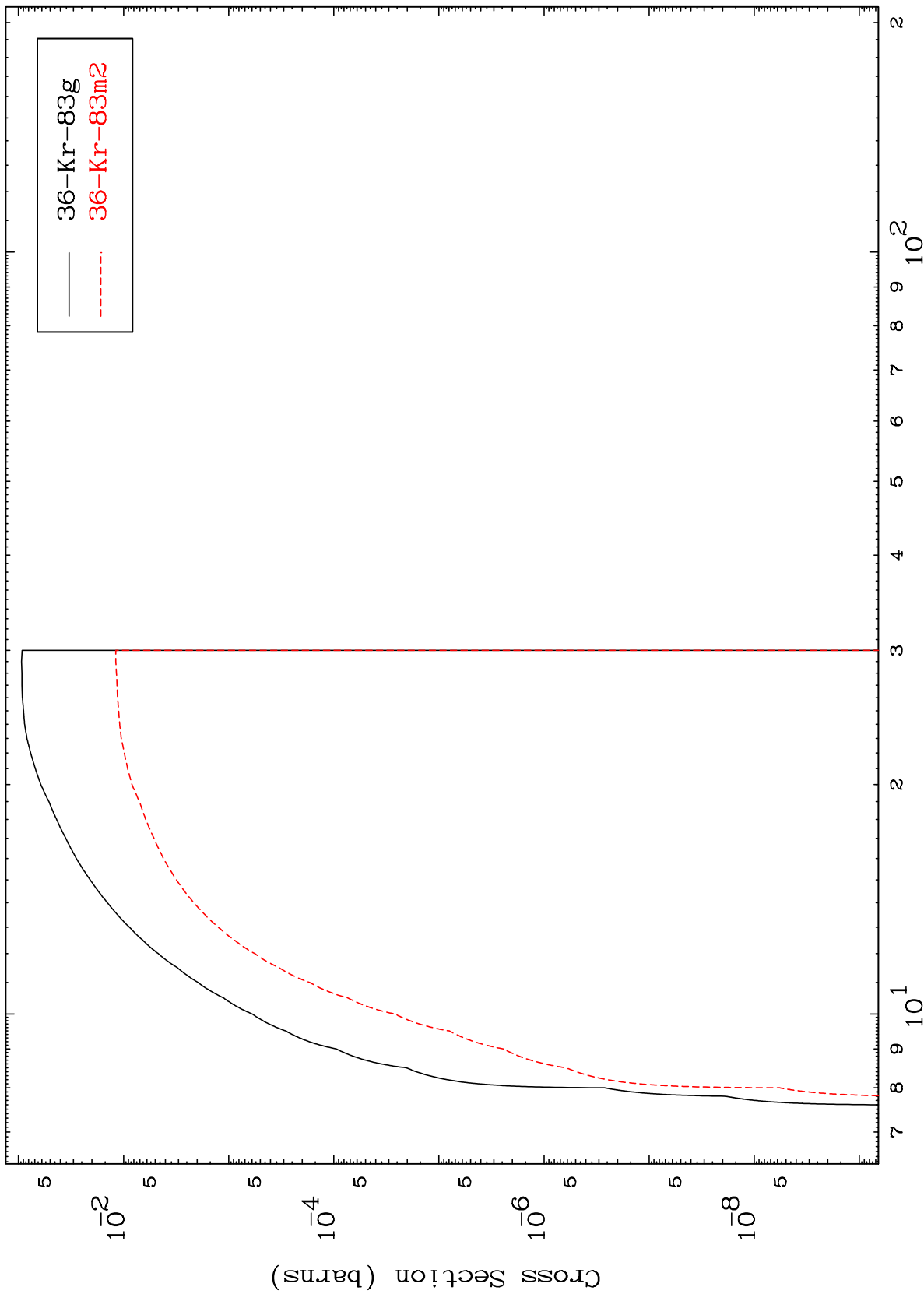
37-Rb-84



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37-Rb-84

$(n, n')$  p  
Radionuclide Production Cross Section

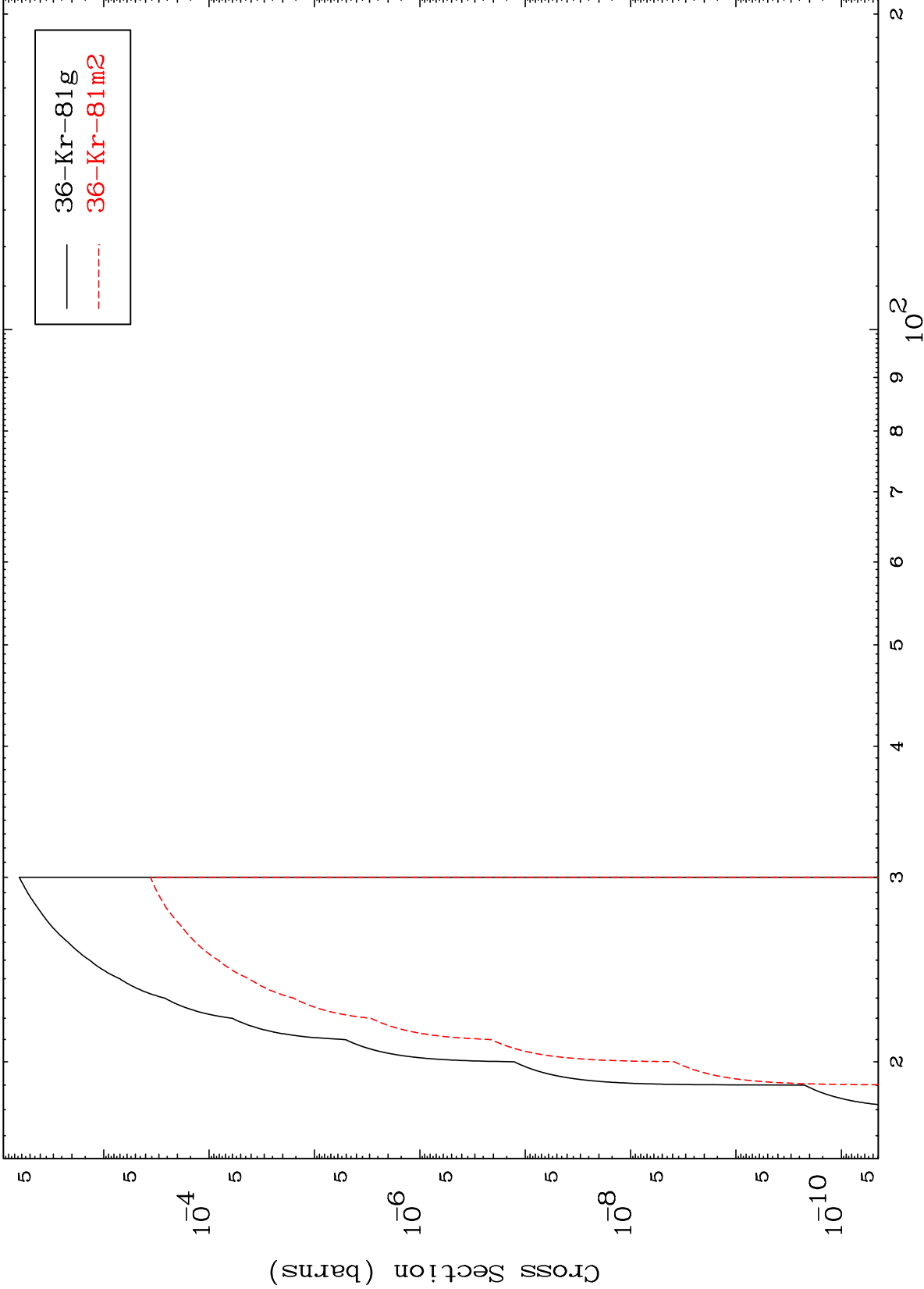


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Incident Energy (MeV)

37-Rb-84

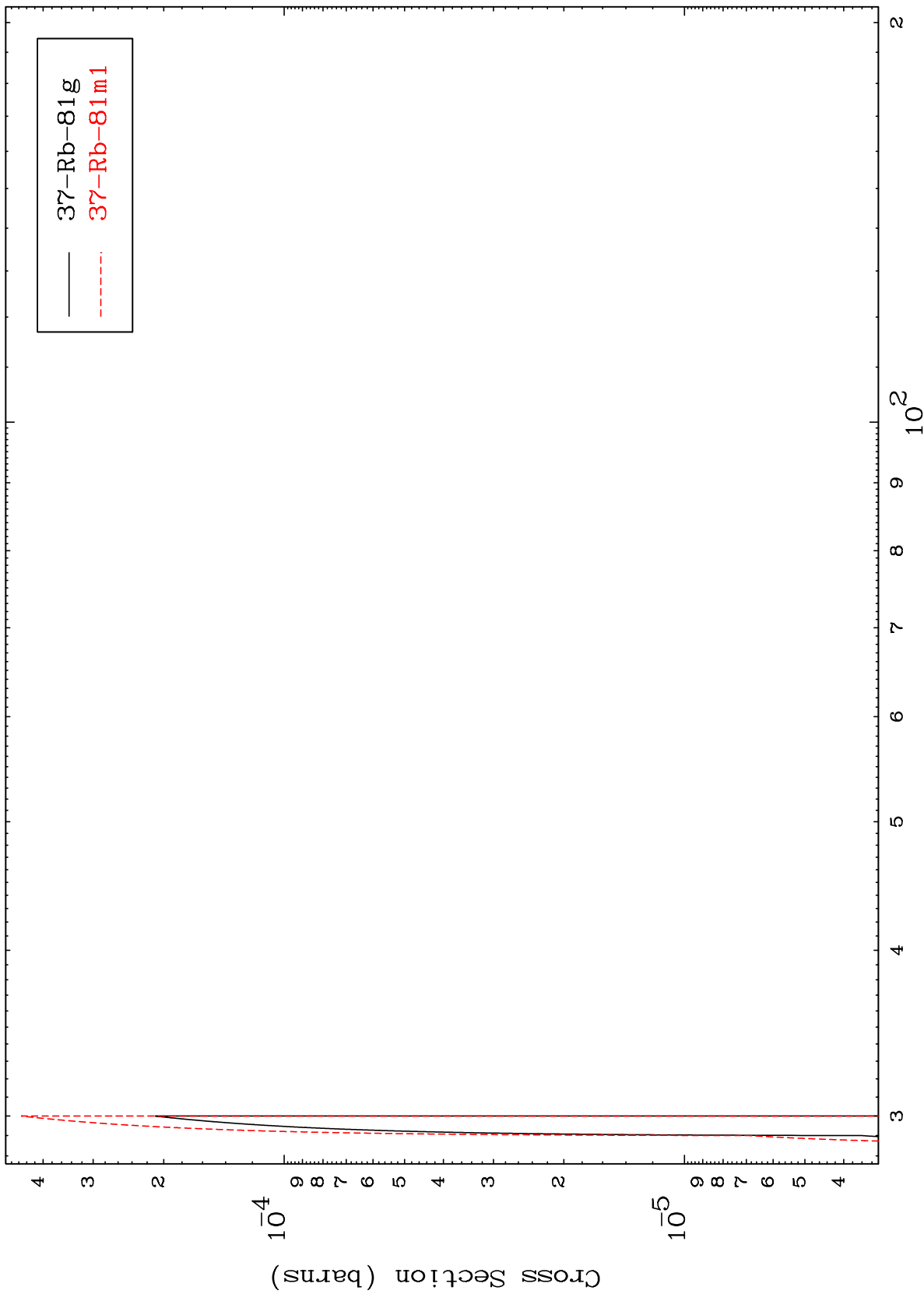
Radionuclide Production Cross Section



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<sup>37</sup>Rb-84

(n,4n)  
Radionuclide Production Cross Section



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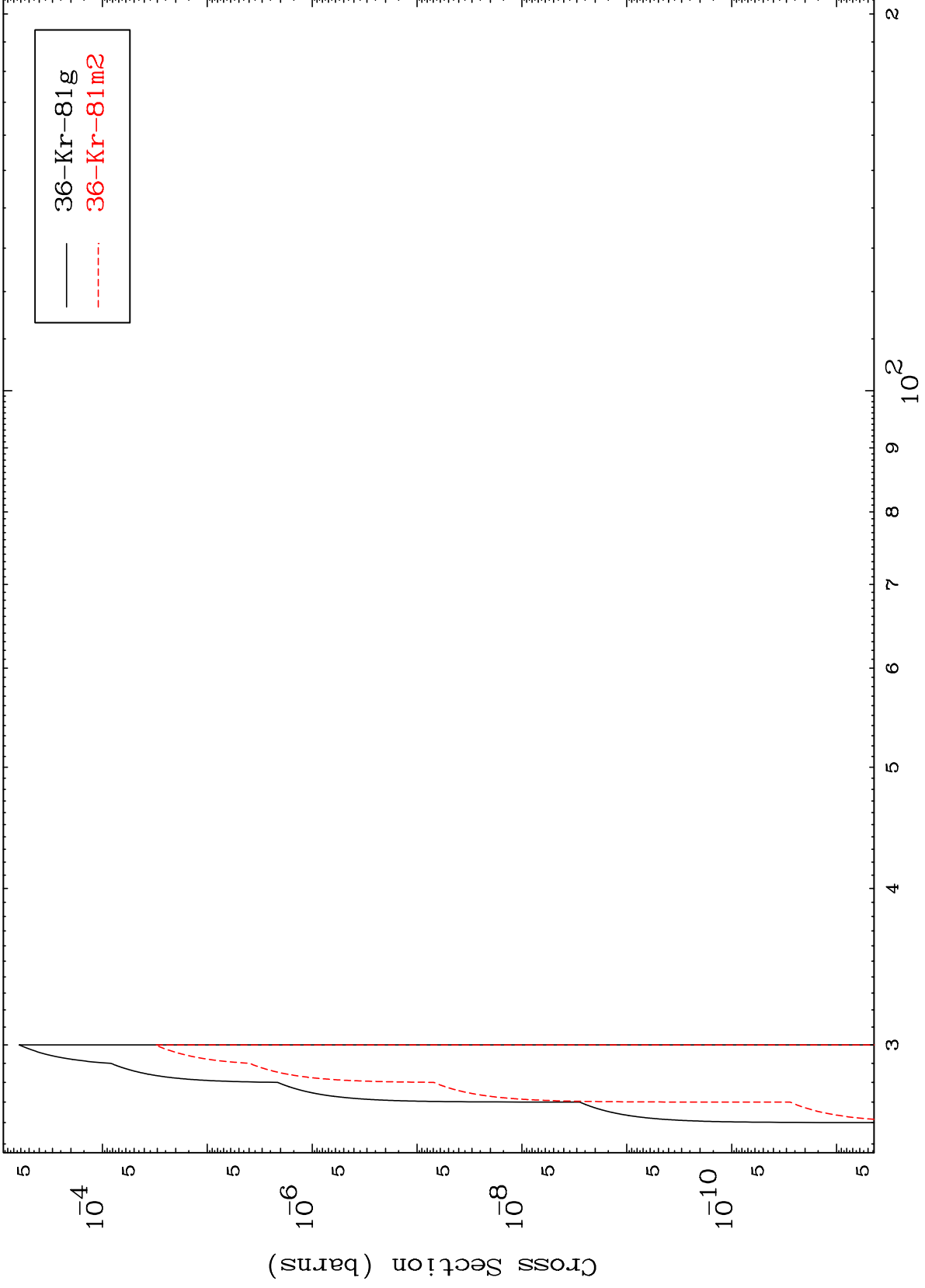
<sup>37</sup>Rb-84

MAT 3723

(n,3n) p

37-Rb-84

Radionuclide Production Cross Section

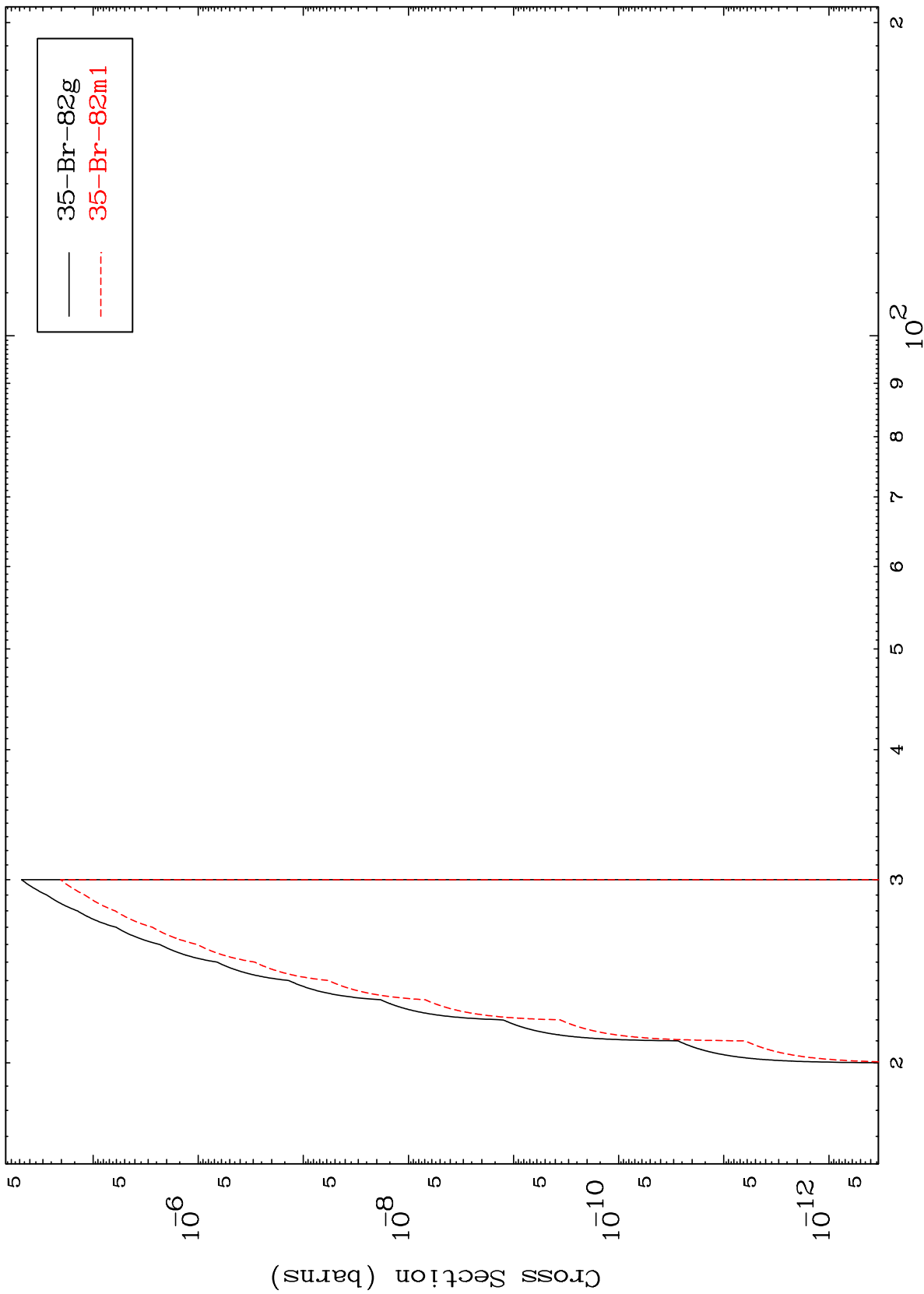


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Incident Energy (MeV)

37-Rb-84

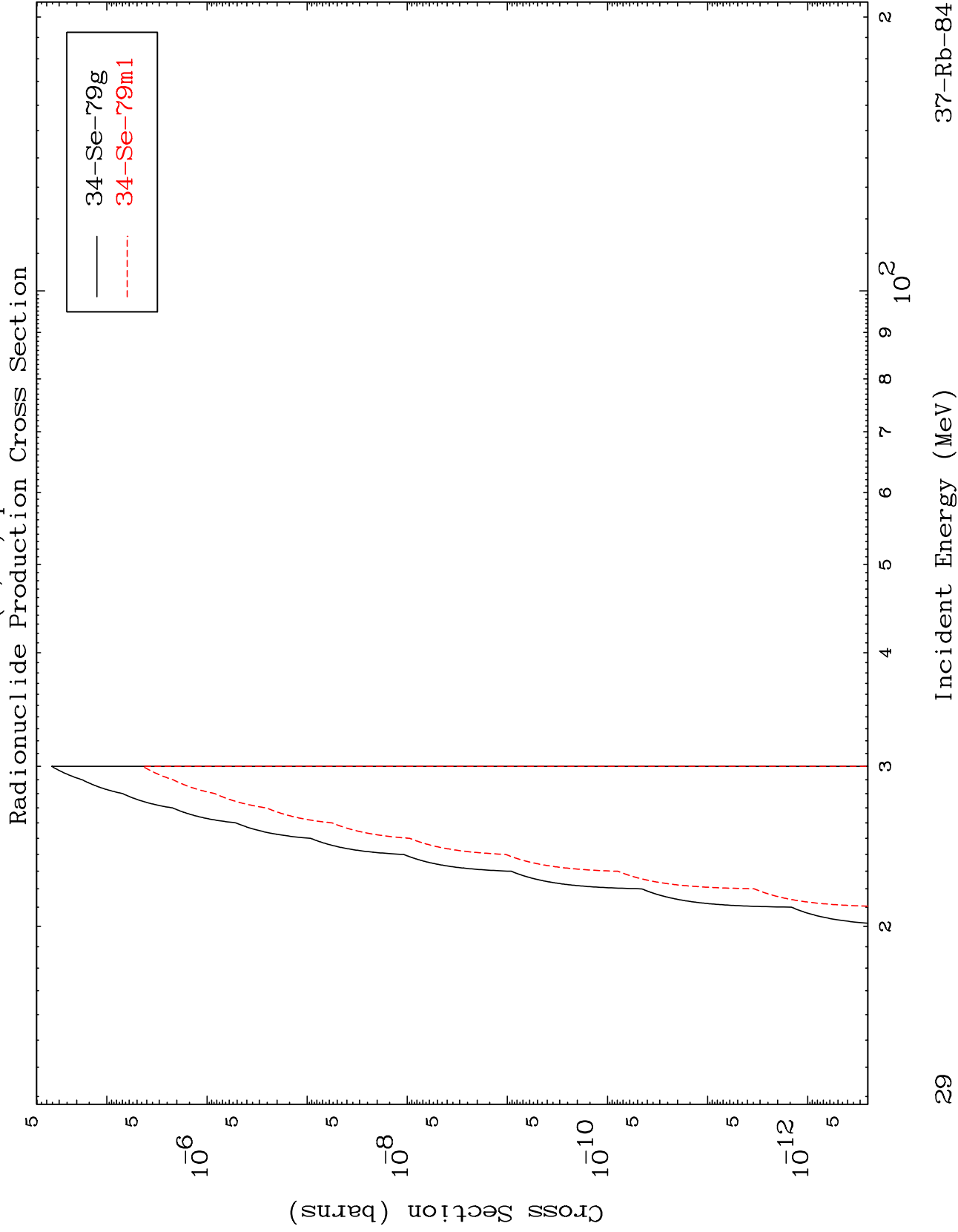
(n,2n) p  
Radionuclide Production Cross Section



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(n,n') p  $\alpha$

37-Rb-84



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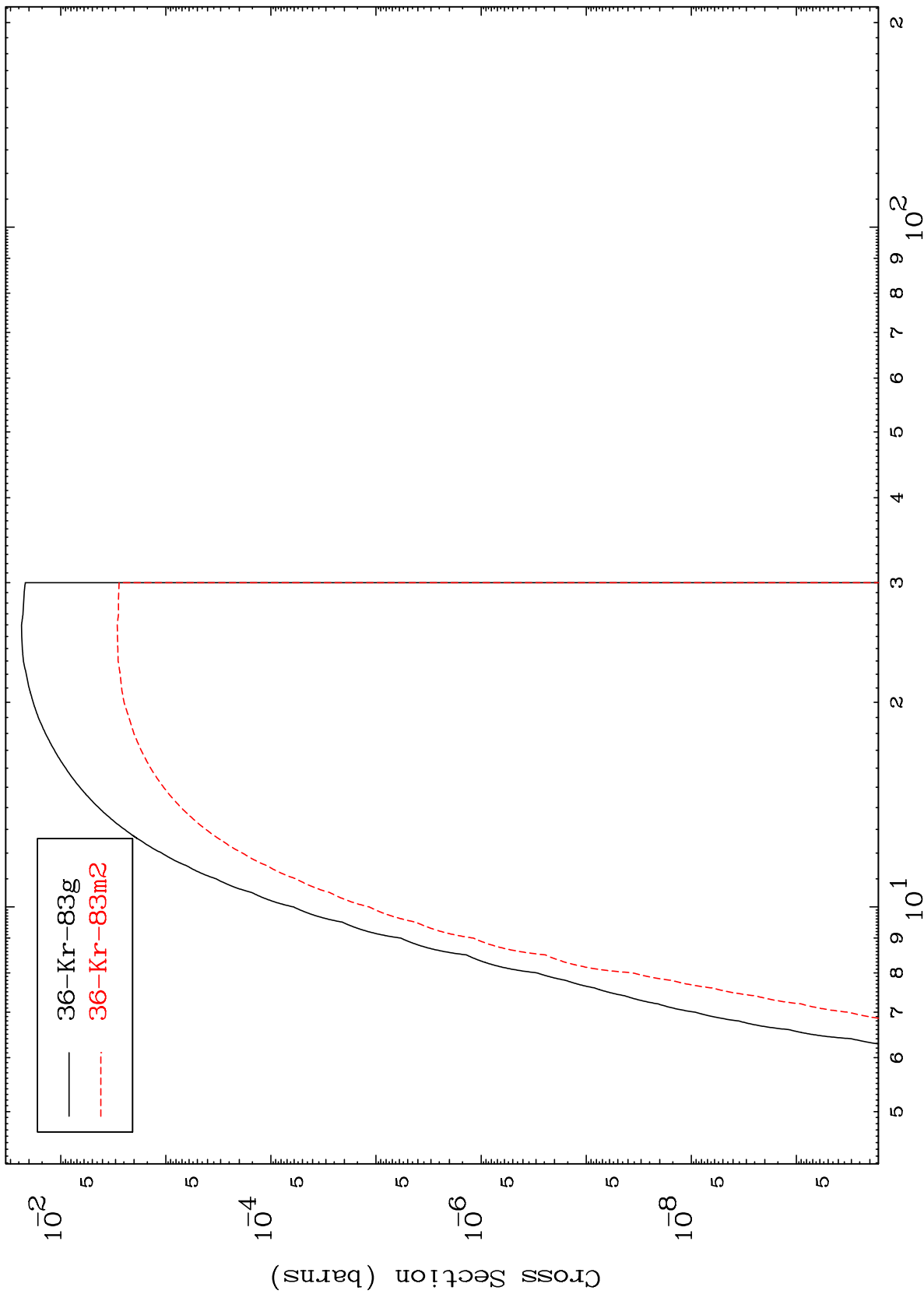
Incident Energy (MeV)

37-Rb-84

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37-Rb-84

(n,d)  
Radionuclide Production Cross Section

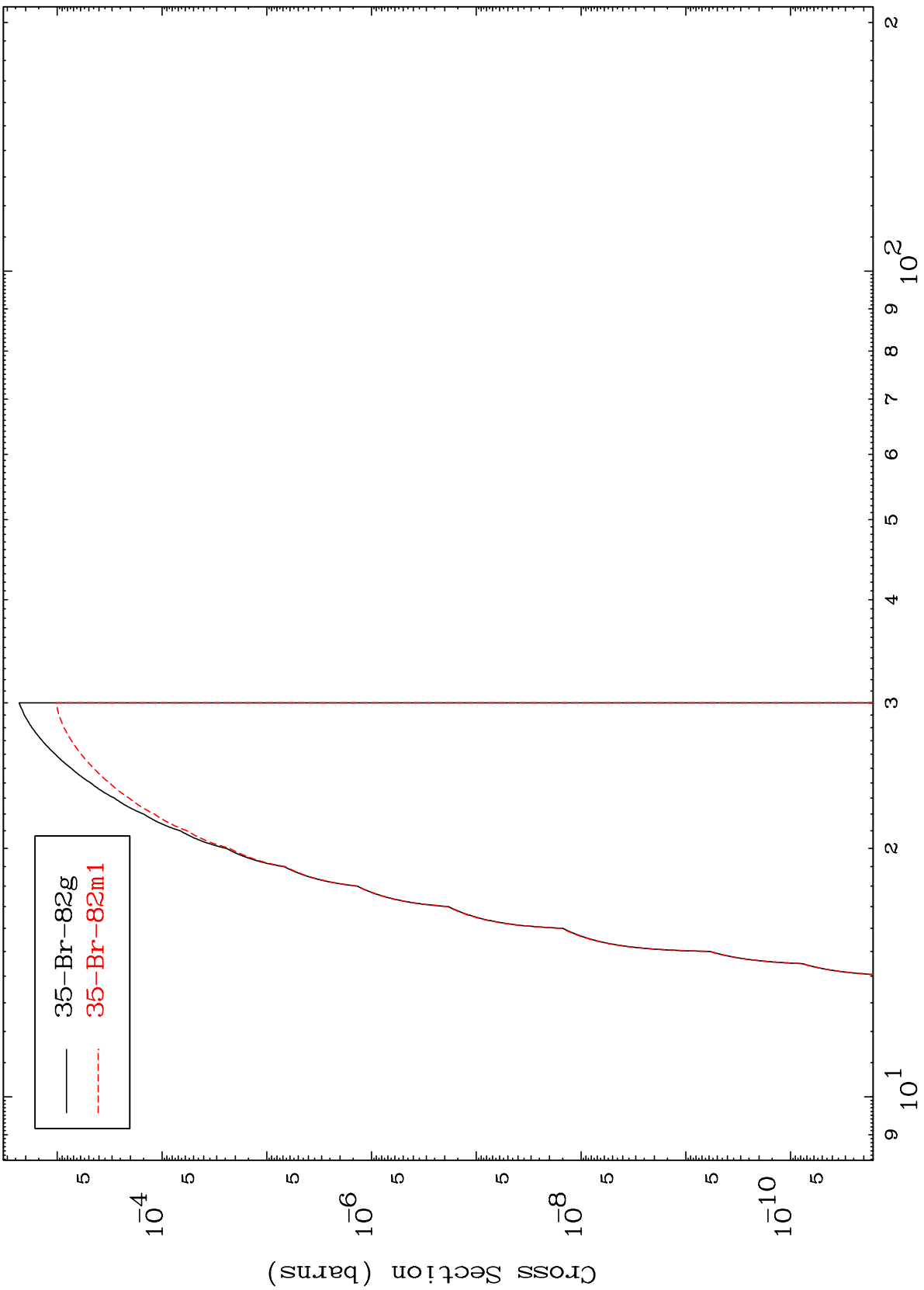


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Incident Energy (MeV)

37-Rb-84

Radionuclide Production Cross Section  
(n,He-3)



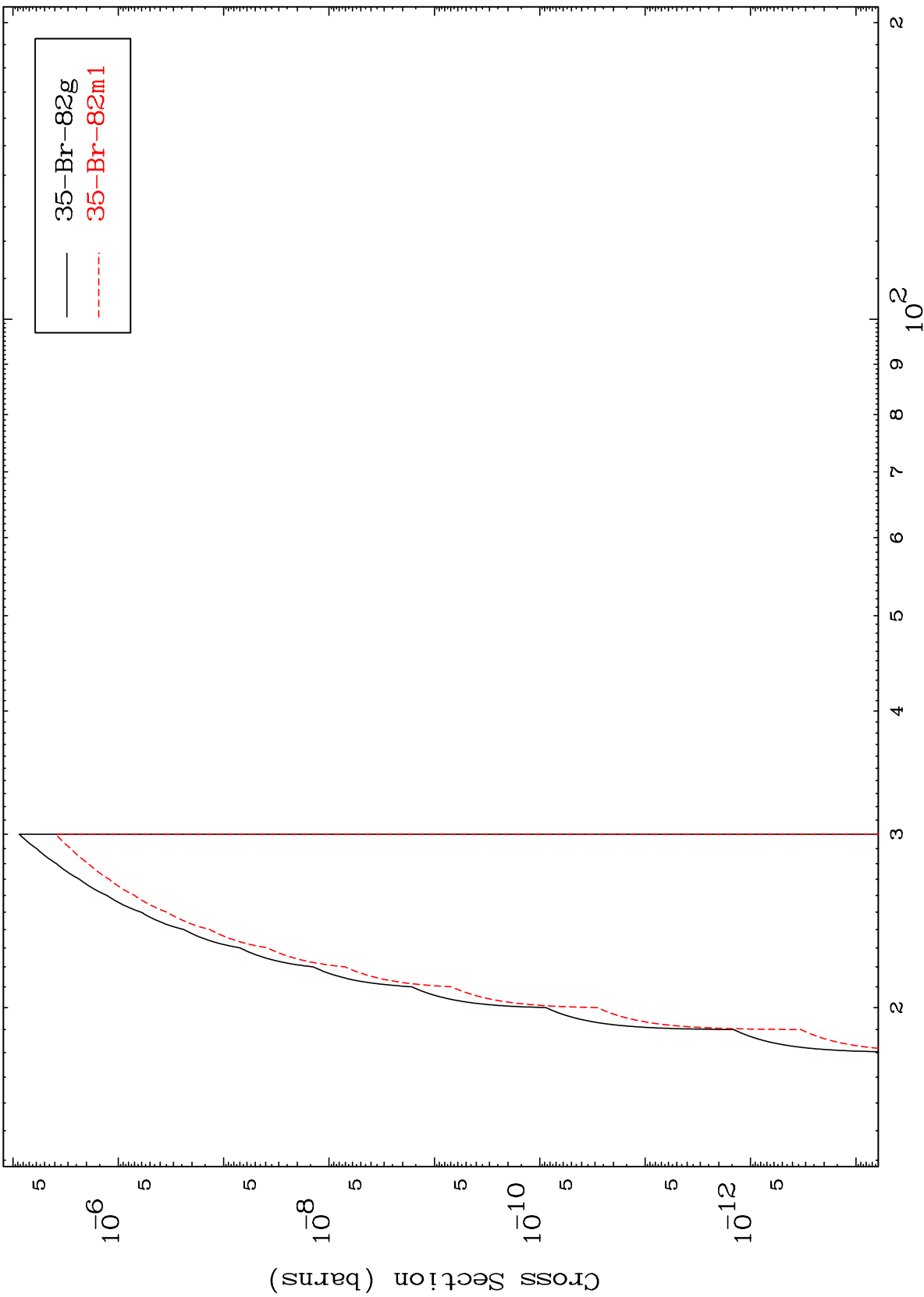


MAT 3723

(n,p) d

37-Rb-84

Radionuclide Production Cross Section



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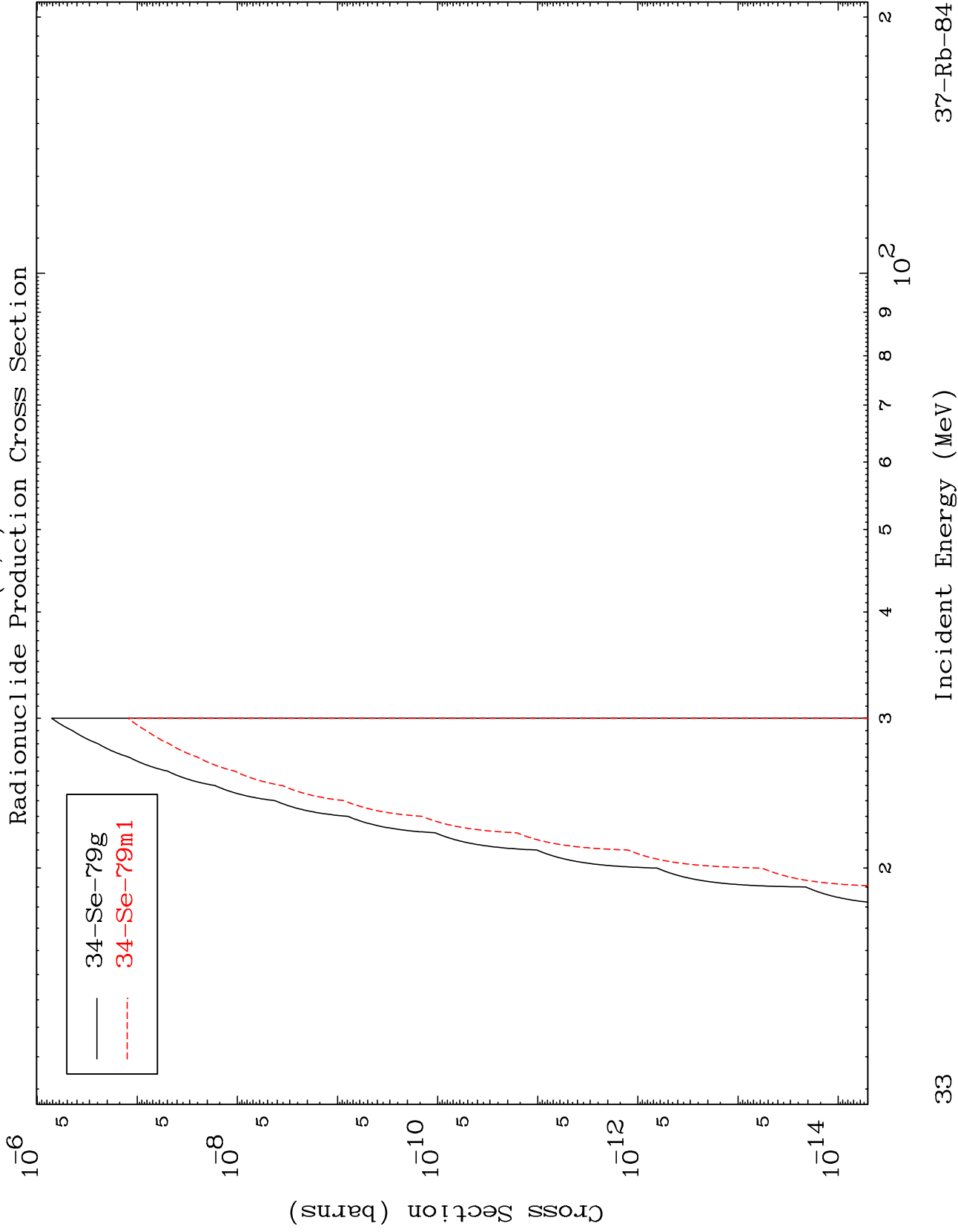
Incident Energy (MeV)

37-Rb-84

MAT 3723

(n,d)  $\alpha$

37-Rb-84



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Incident Energy (MeV)

37-Rb-84