

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

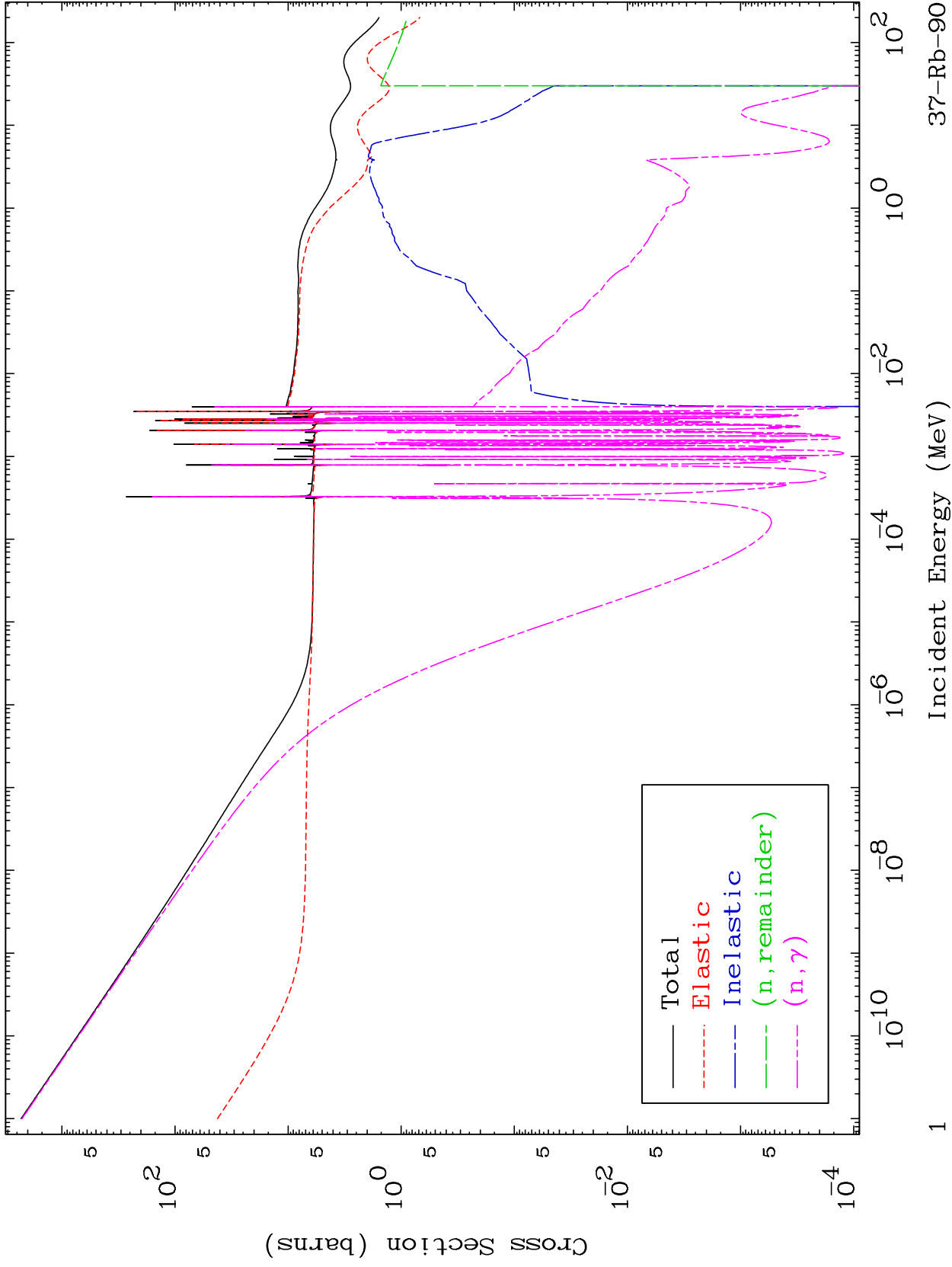
Press Mouse Button to Start

MAT 3741

Major

293 Kelvin Cross Sections

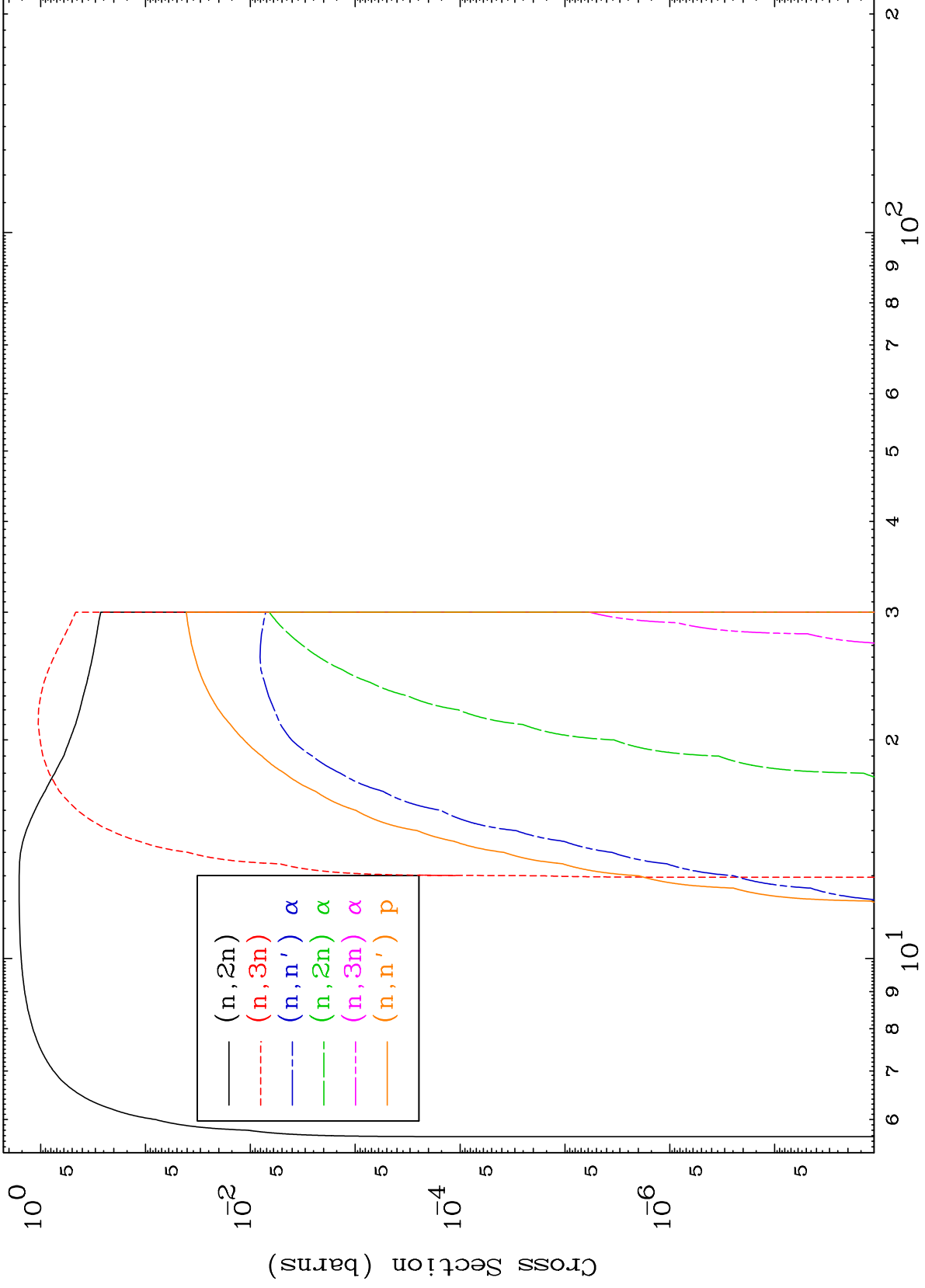
37-Rb-90



MAT 3741

Neutron Production  
293 Kelvin Cross Sections

37-Rb-90



2

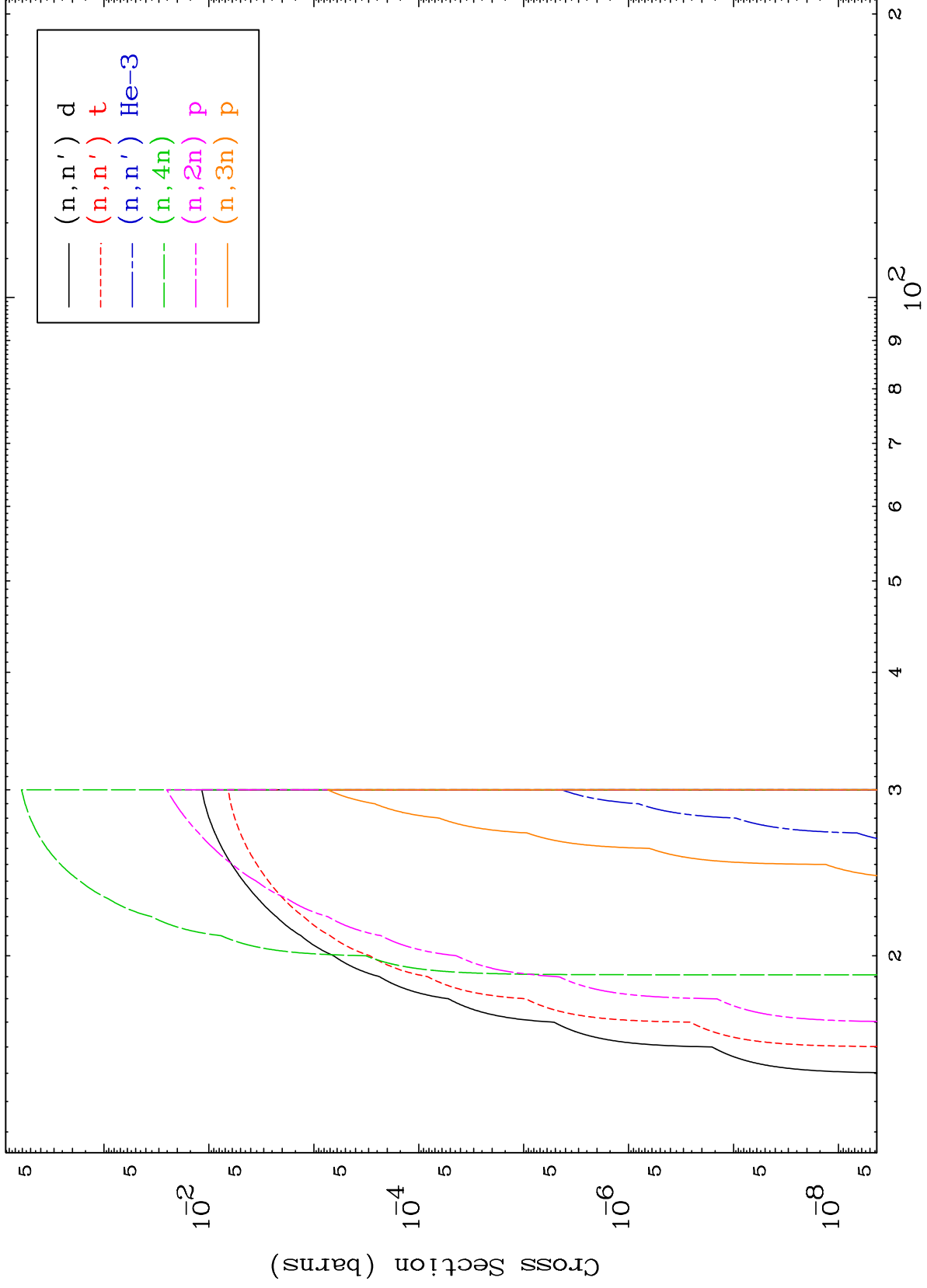
Incident Energy (MeV)

37-Rb-90

MAT 3741

Neutron Production  
293 Kelvin Cross Sections

37-Rb-90



3

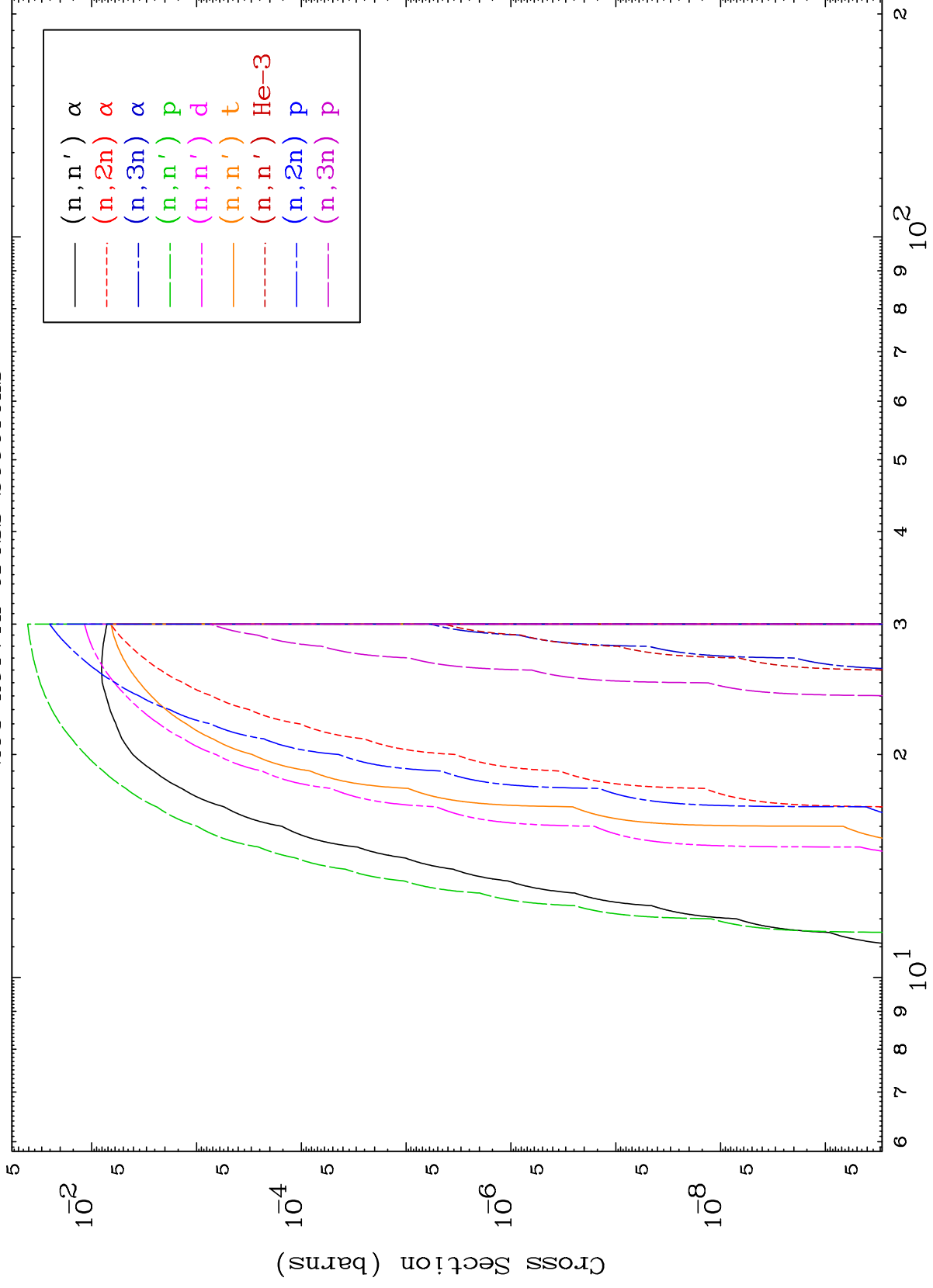
Incident Energy (MeV)

37-Rb-90

MAT 3741

Charged Particle  
293 Kelvin Cross Sections

37-Rb-90



4

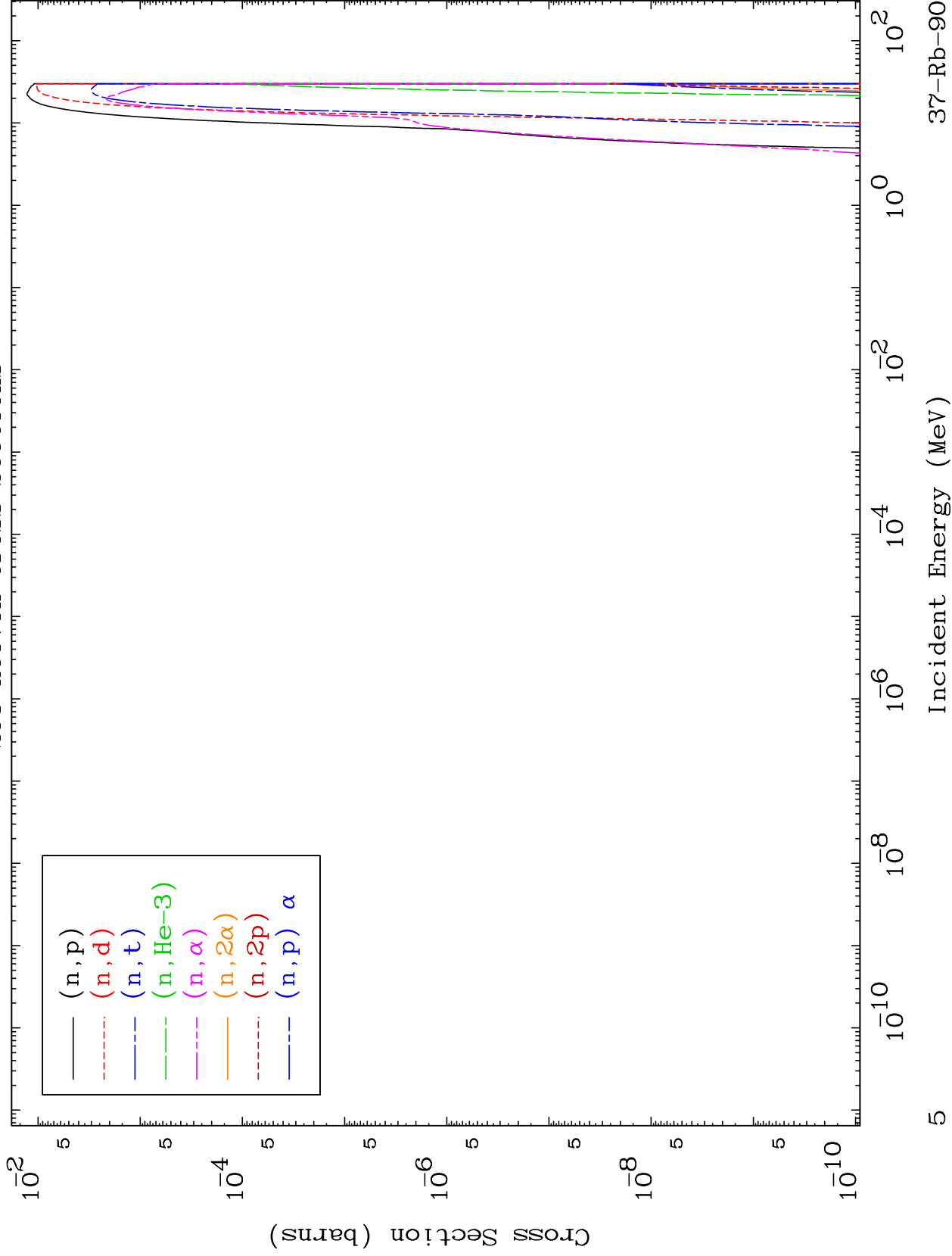
Incident Energy (MeV)

37-Rb-90

MAT 3741

Charged Particle  
293 Kelvin Cross Sections

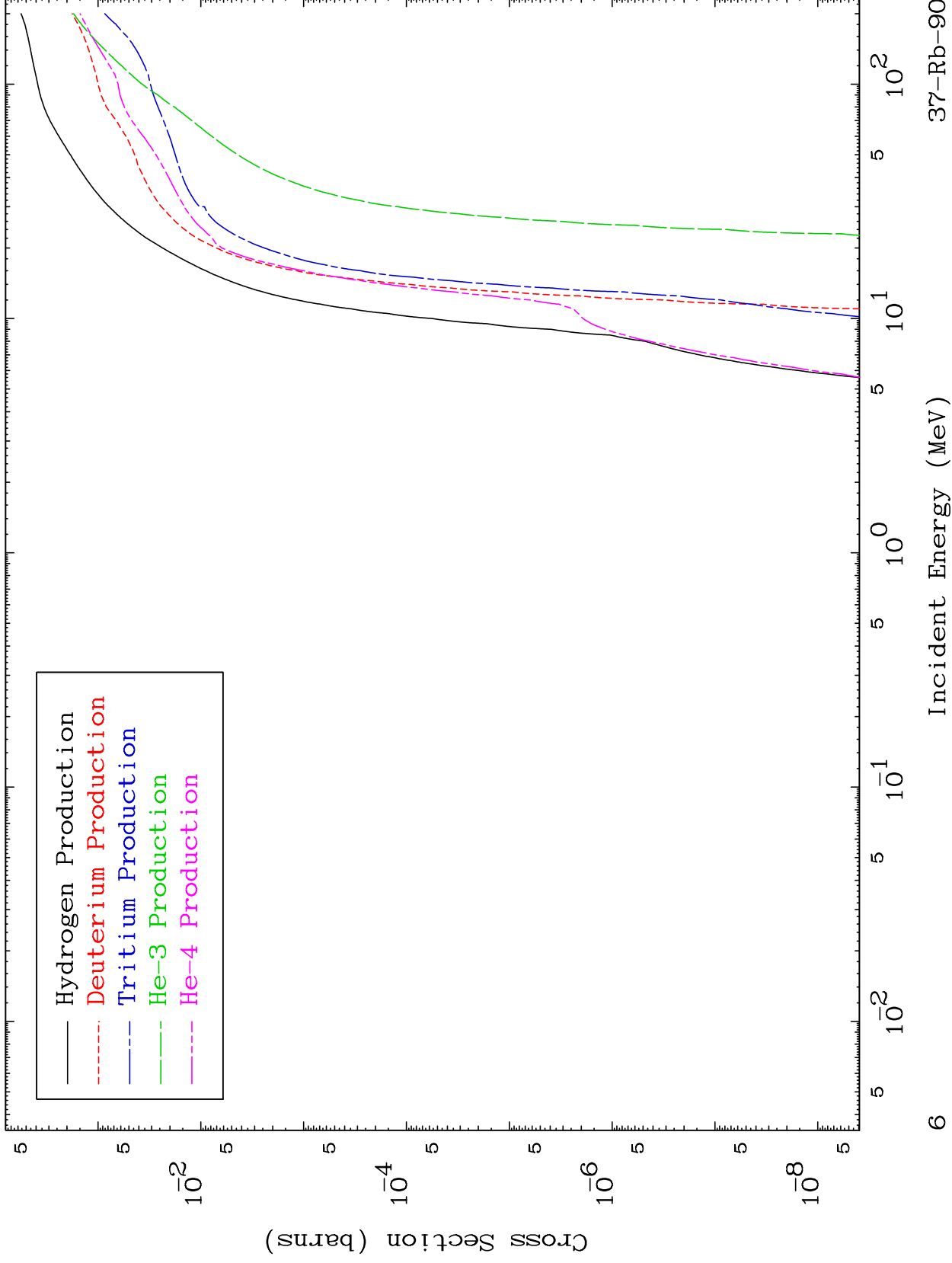
37-Rb-90



MAT 3741

Particle Production  
293 Kelvin Cross Sections

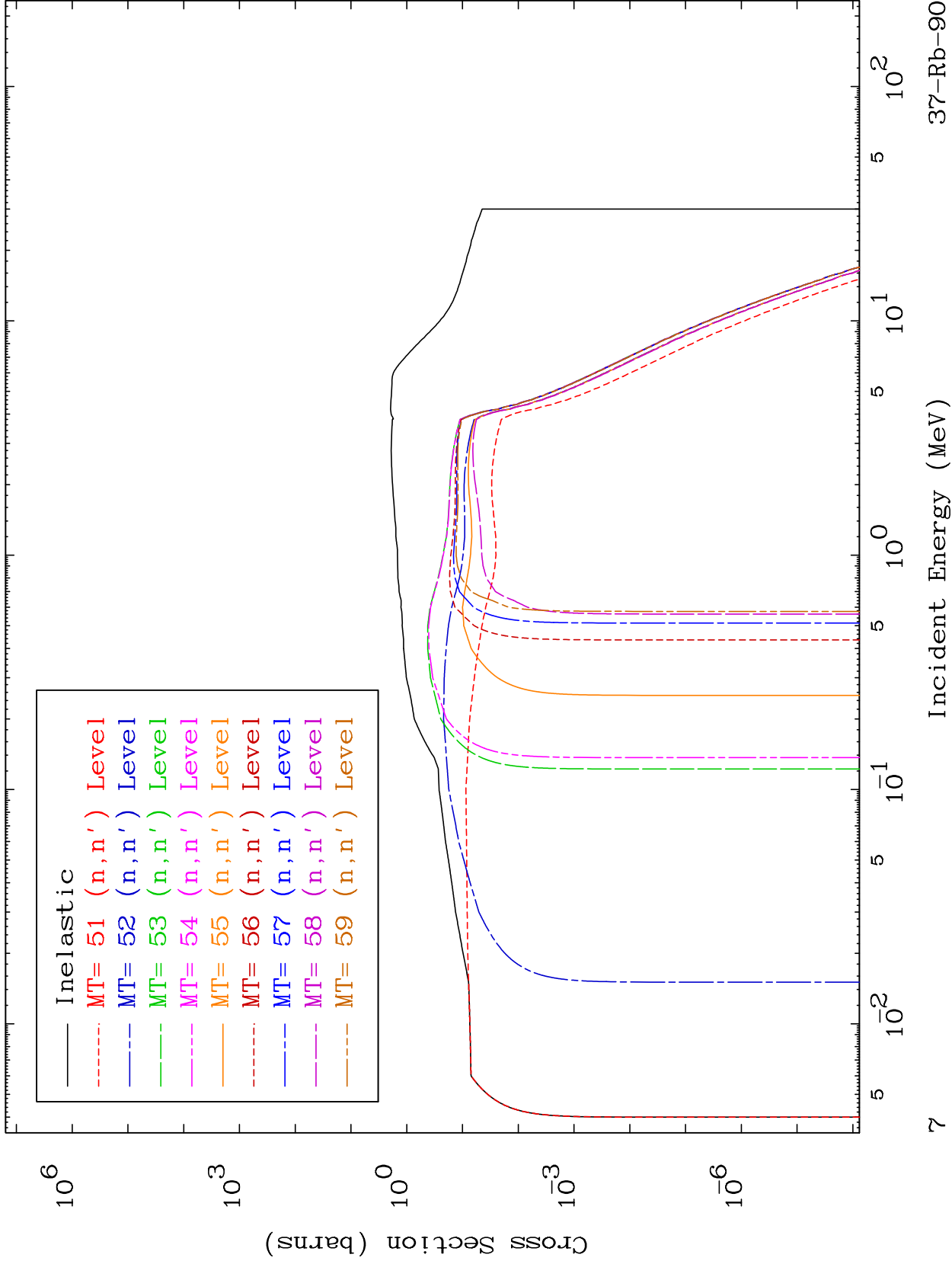
37-Rb-90



MAT 3741

(n,n') Level  
293 Kelvin Cross Sections

37-Rb-90



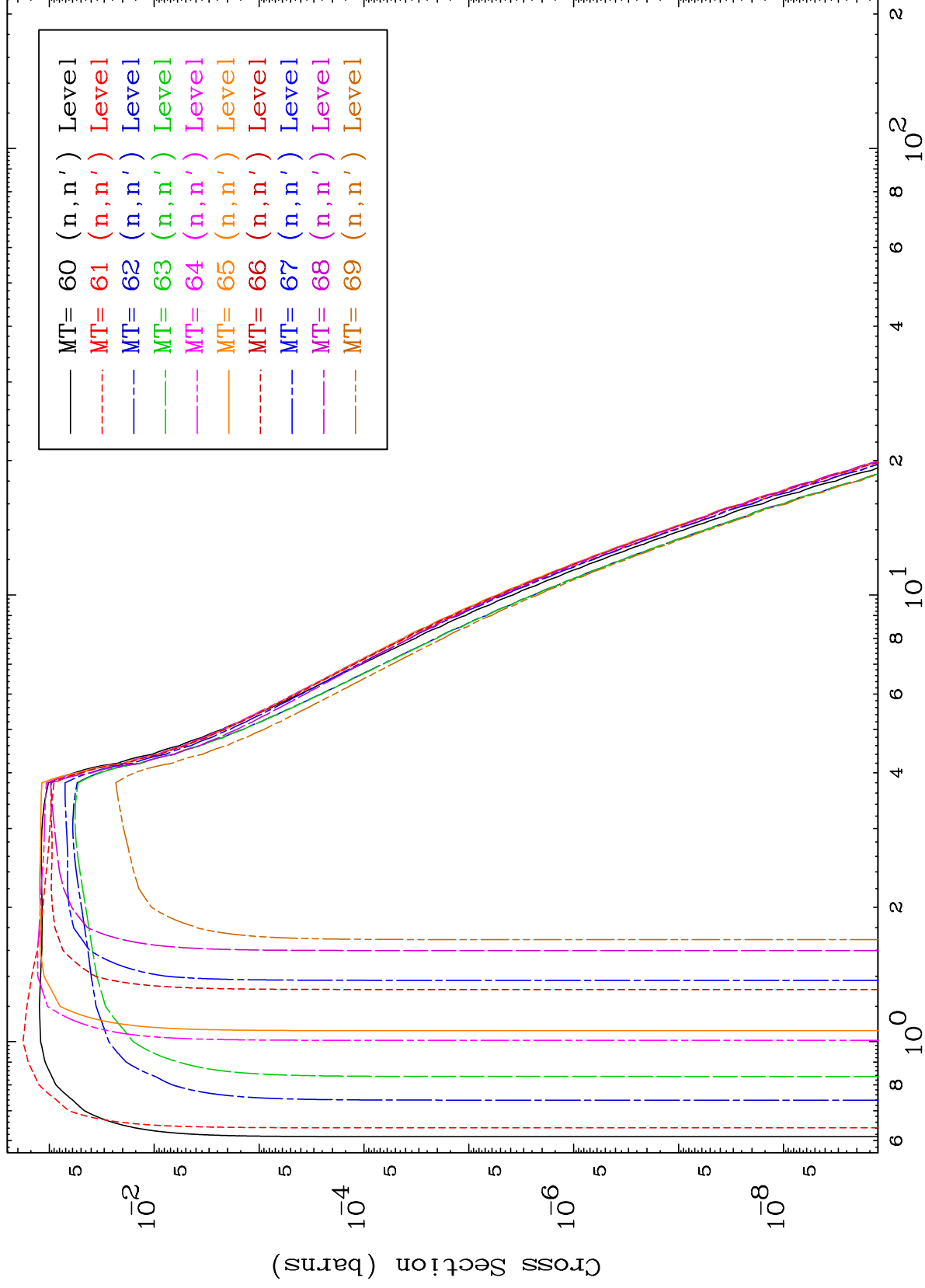


MAT 3741

(n,n') Level

37-Rb-90

293 Kelvin Cross Sections



8

Incident Energy (MeV)

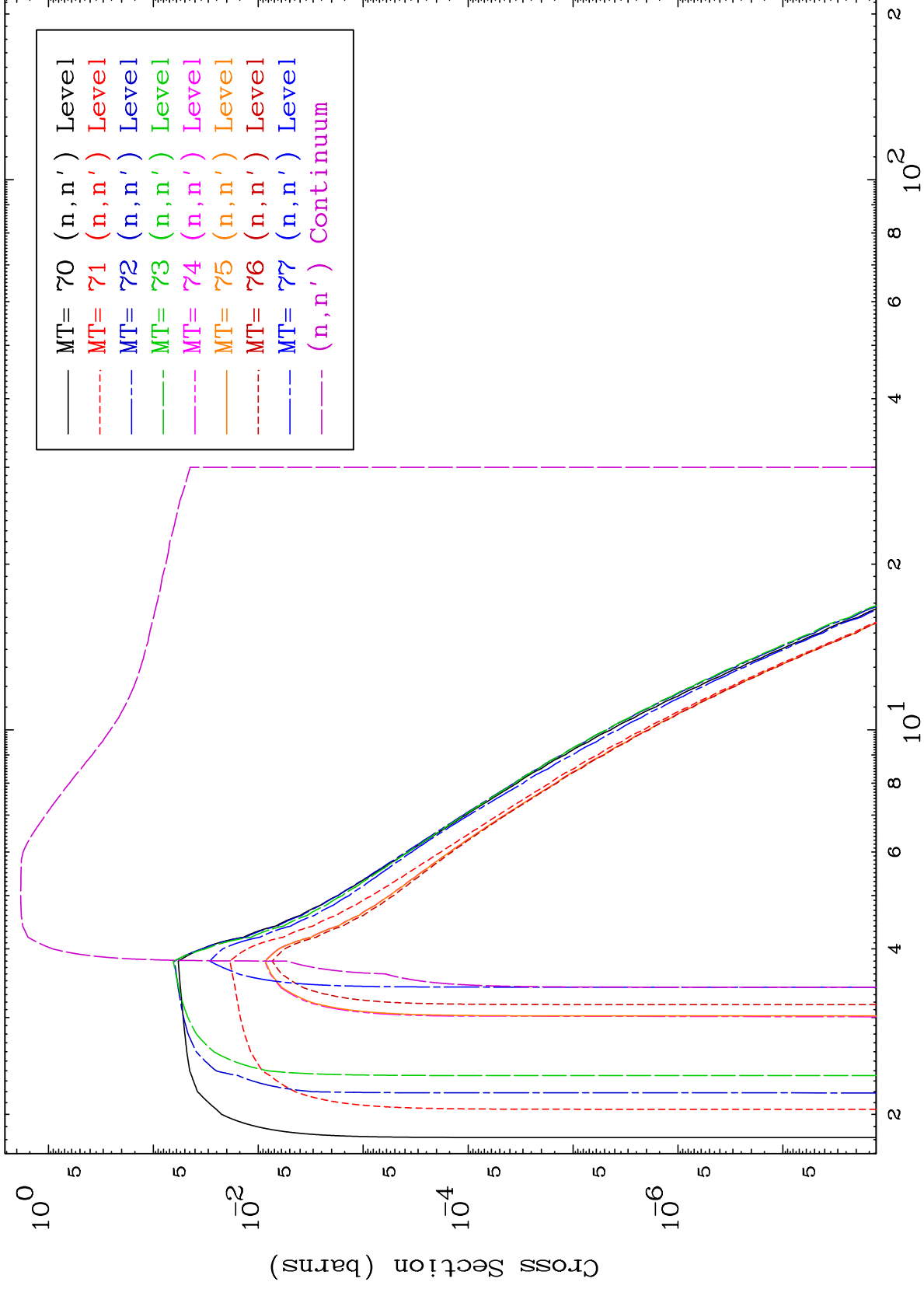
37-Rb-90

MAT 3741

(n,n') Level

37-Rb-90

293 Kelvin Cross Sections



9

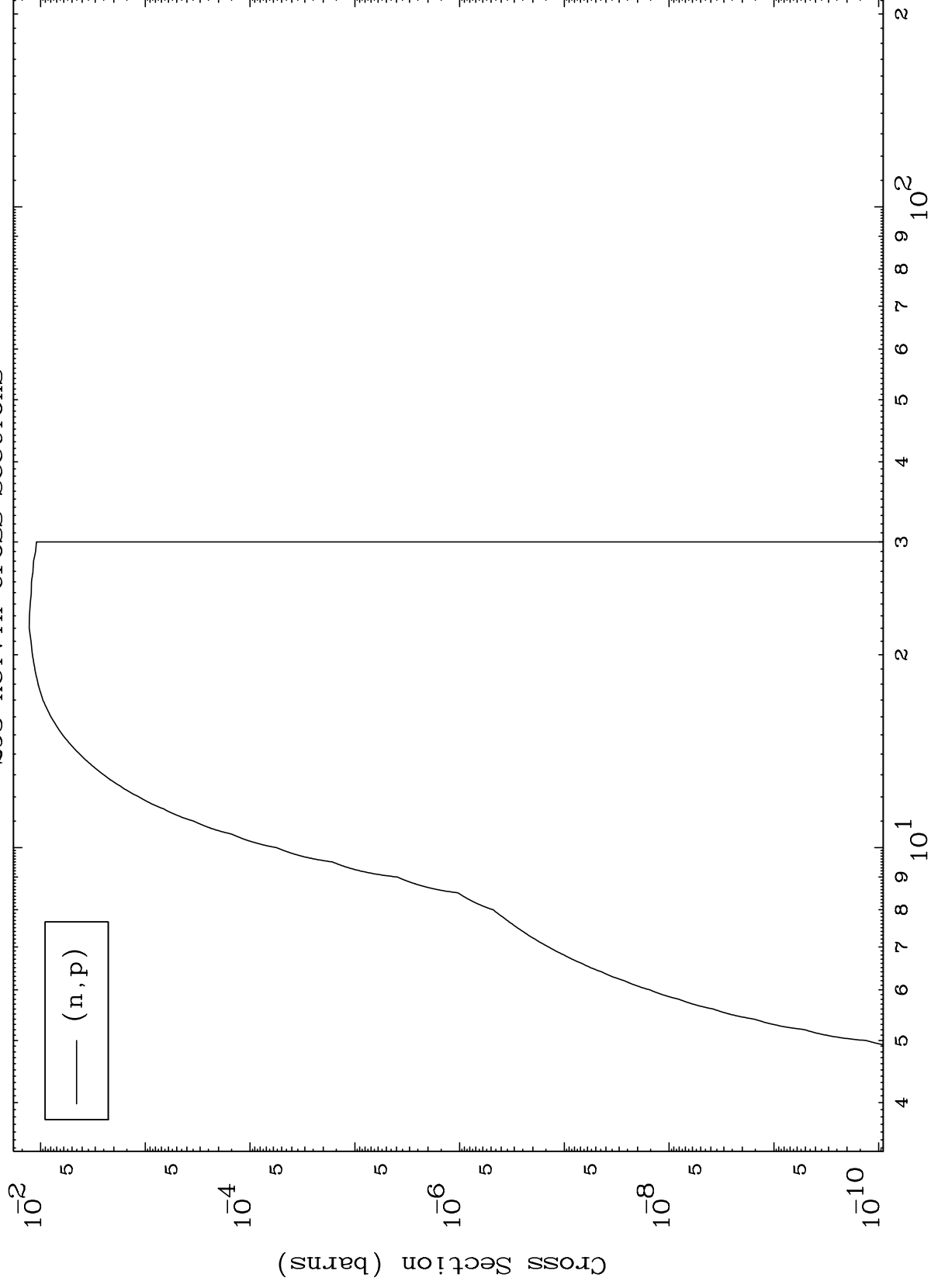
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,p) Levels  
293 Kelvin Cross Sections

37-Rb-90



10

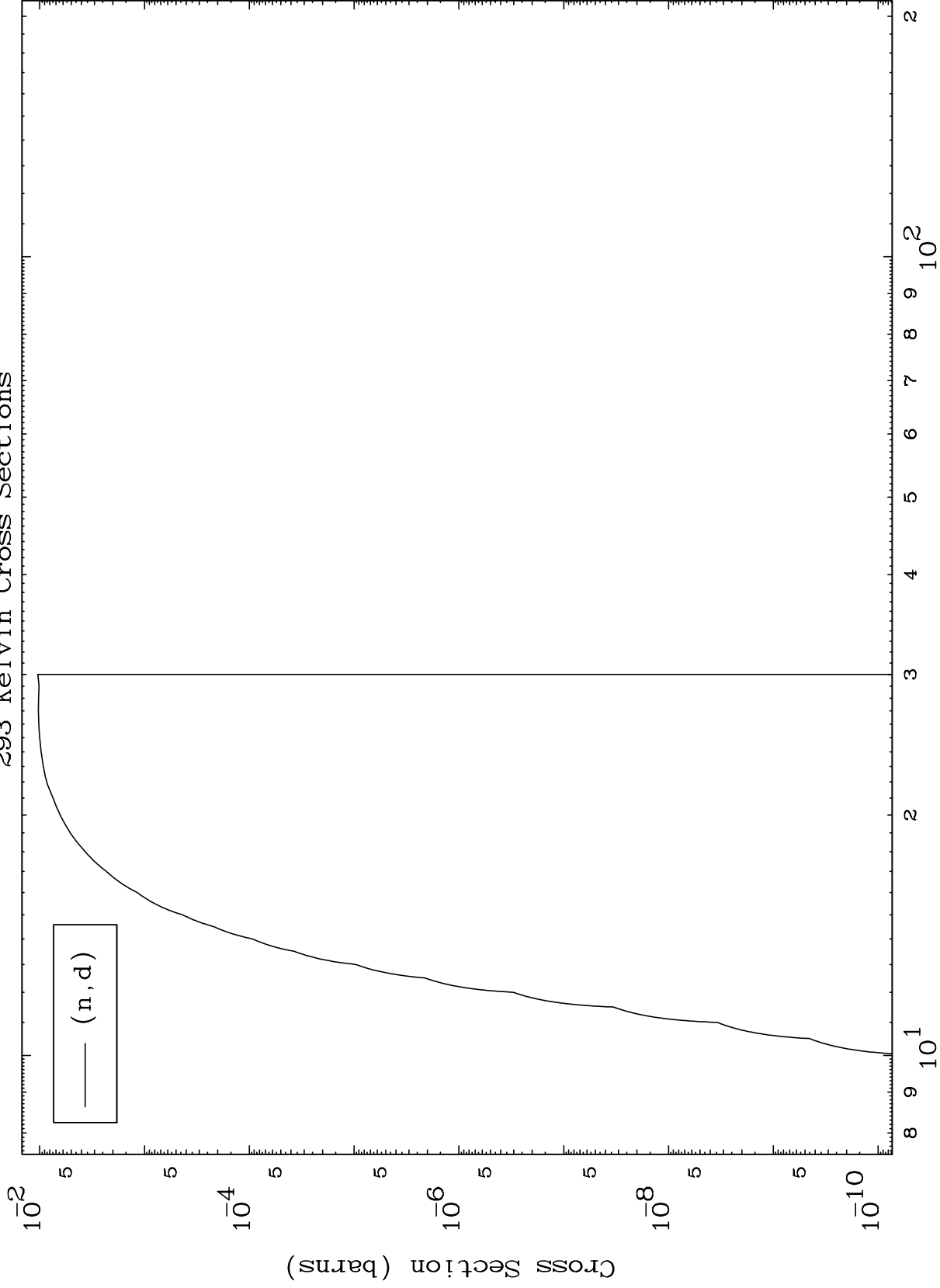
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,d) Levels  
293 Kelvin Cross Sections

37-Rb-90



11

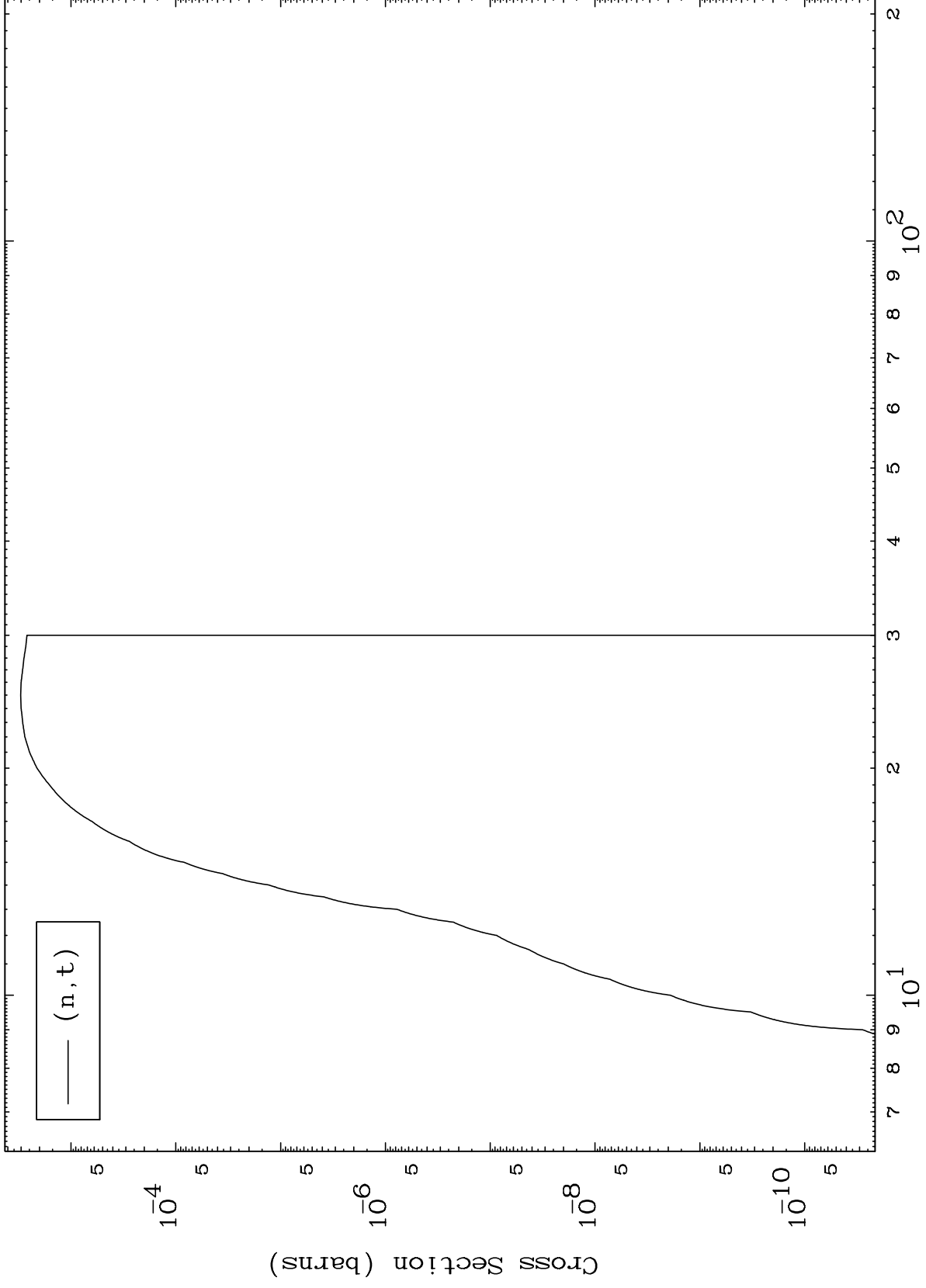
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,t) Levels  
293 Kelvin Cross Sections

37-Rb-90



12

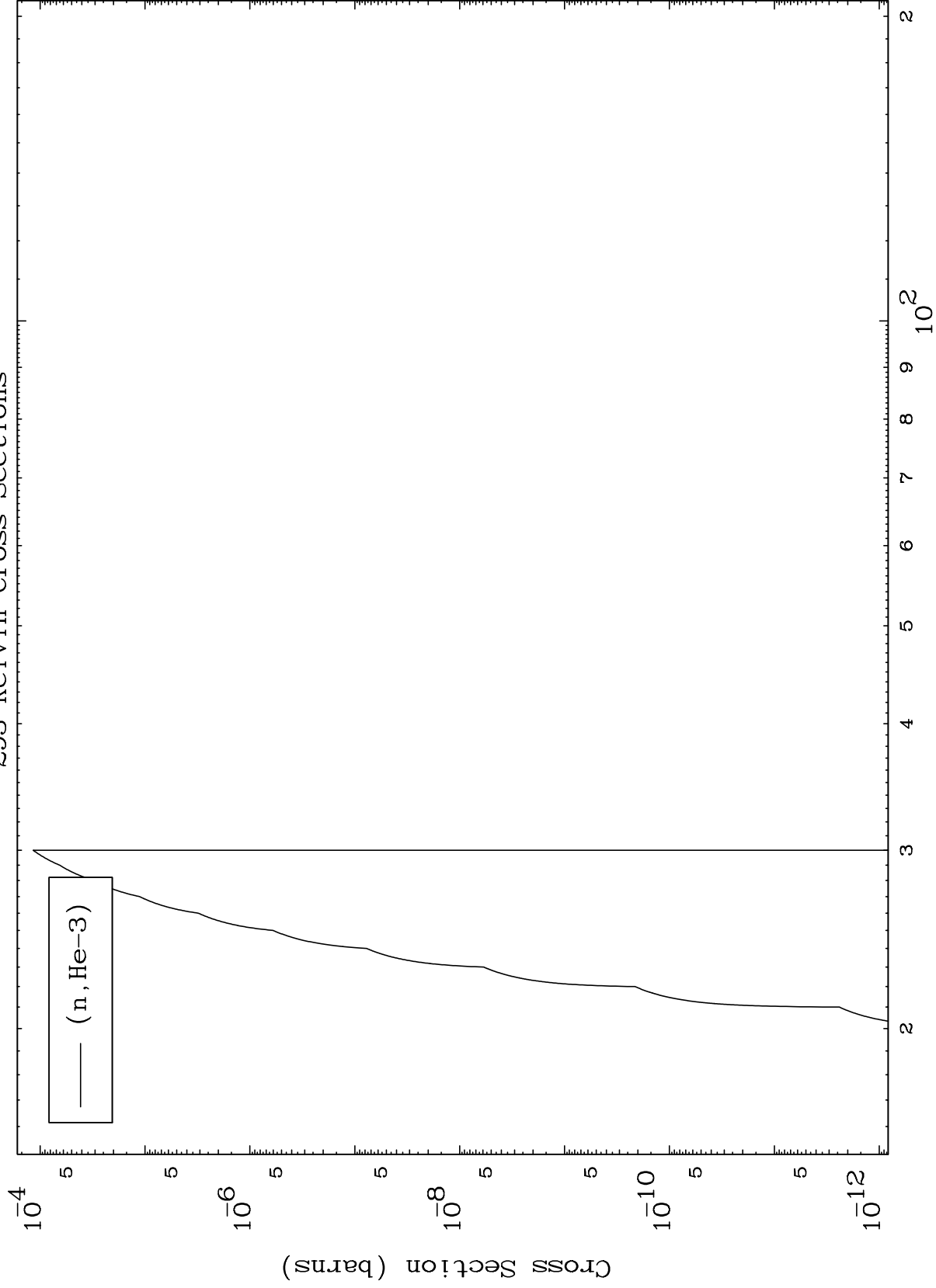
Incident Energy (MeV)

37-Rb-90

MAT 3741

(n,He3) Levels  
293 Kelvin Cross Sections

37-Rb-90



13

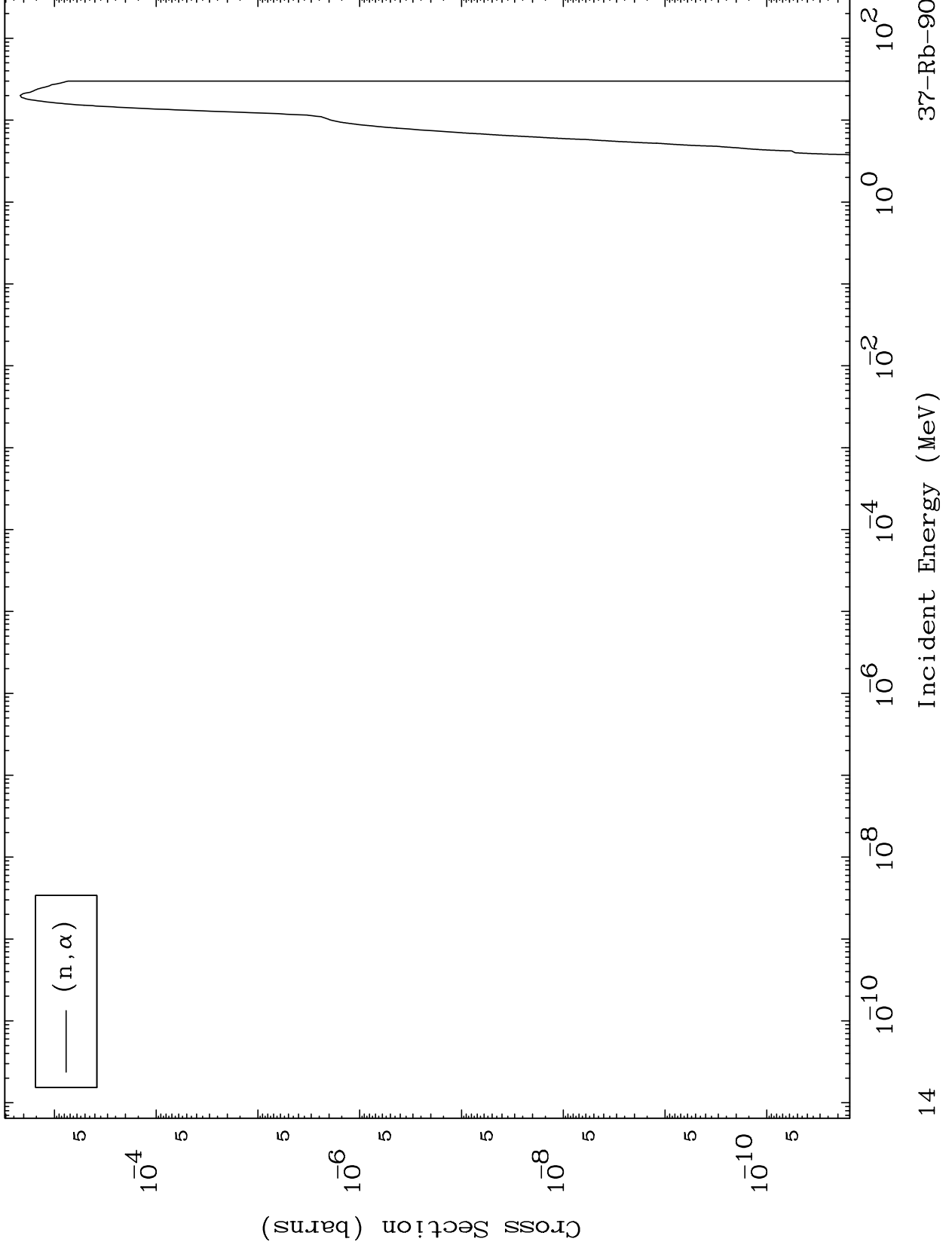
Incident Energy (MeV)

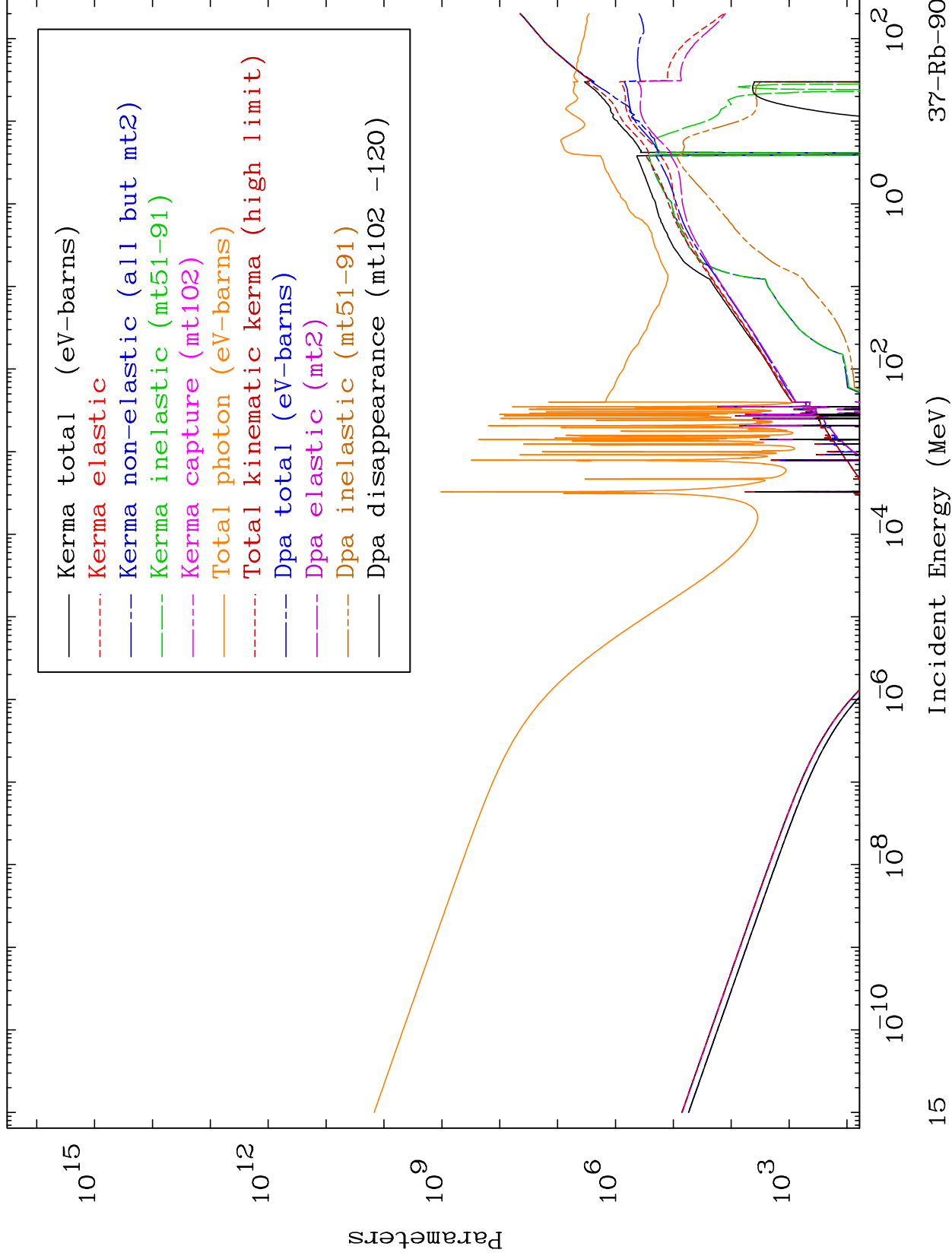
37-Rb-90

MAT 3741

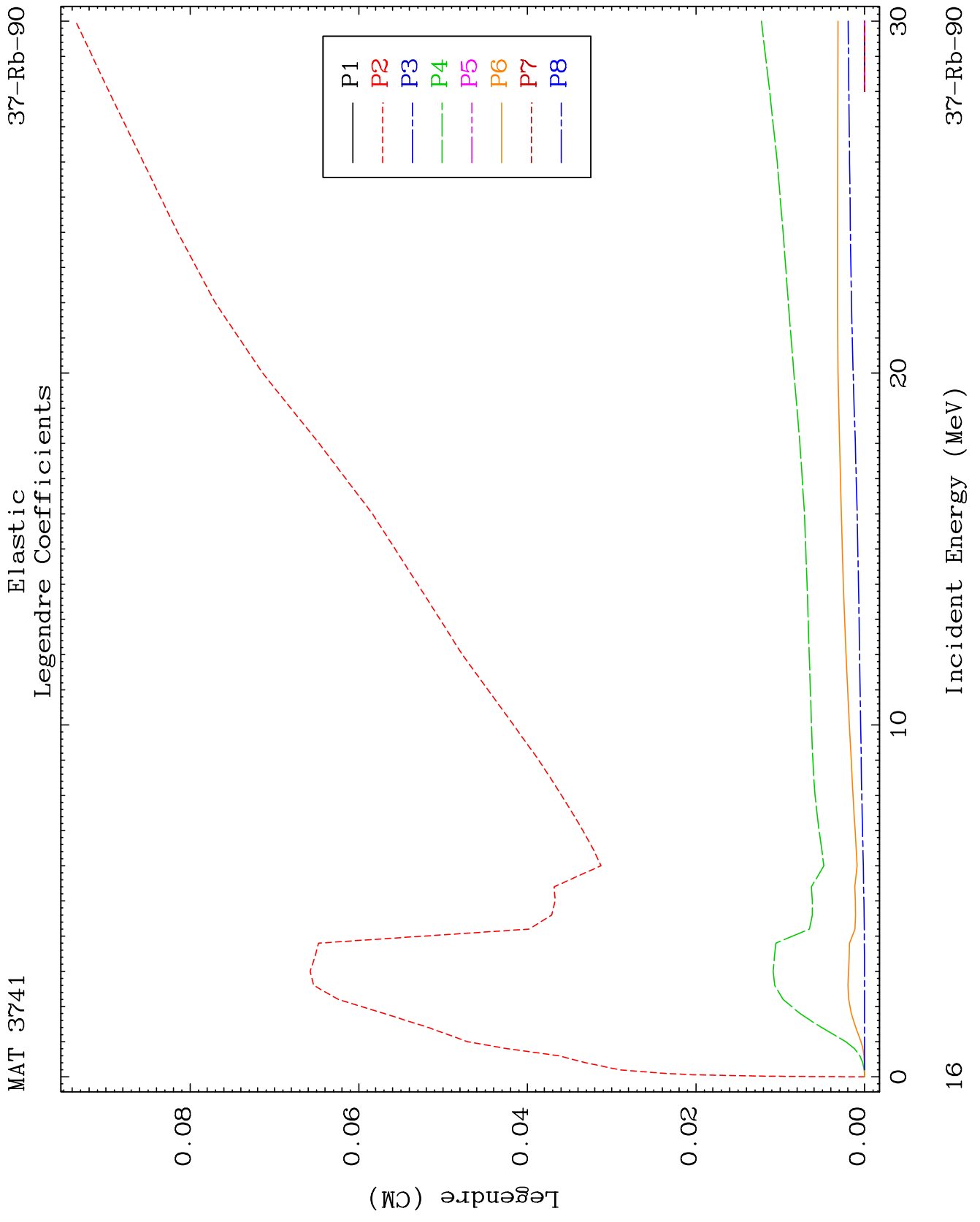
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

37-Rb-90







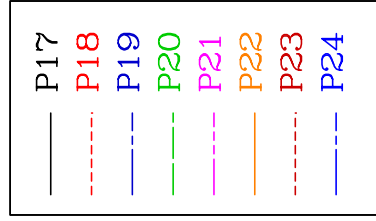




MAT 3741

Elastic Legendre Coefficients

37-Rb-90



$\times 10^{-6}$   
2.5  
2.0

Legendre (CM)

1.5  
1.0  
0.5  
0.0

15

20

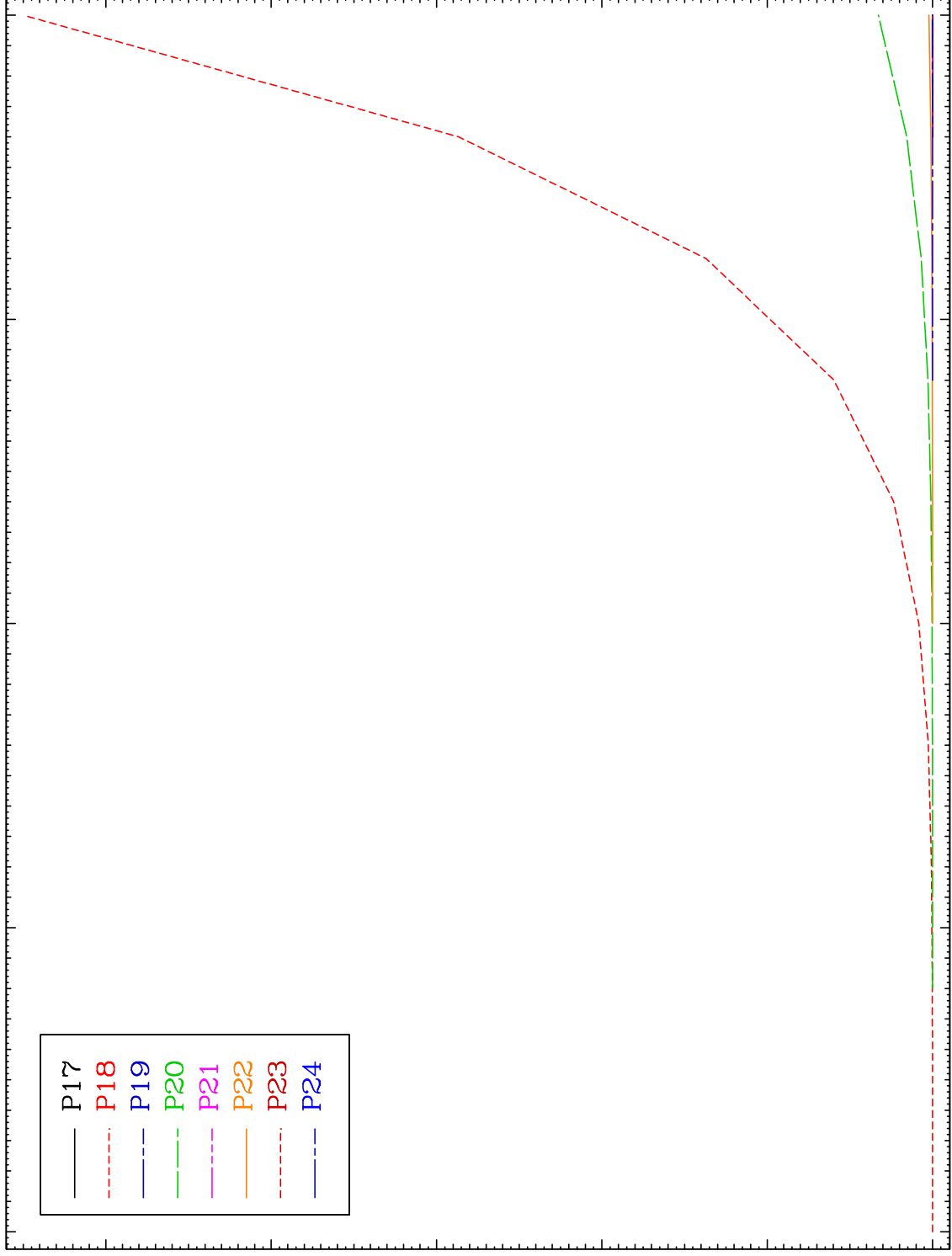
25

30

18

Incident Energy (MeV)

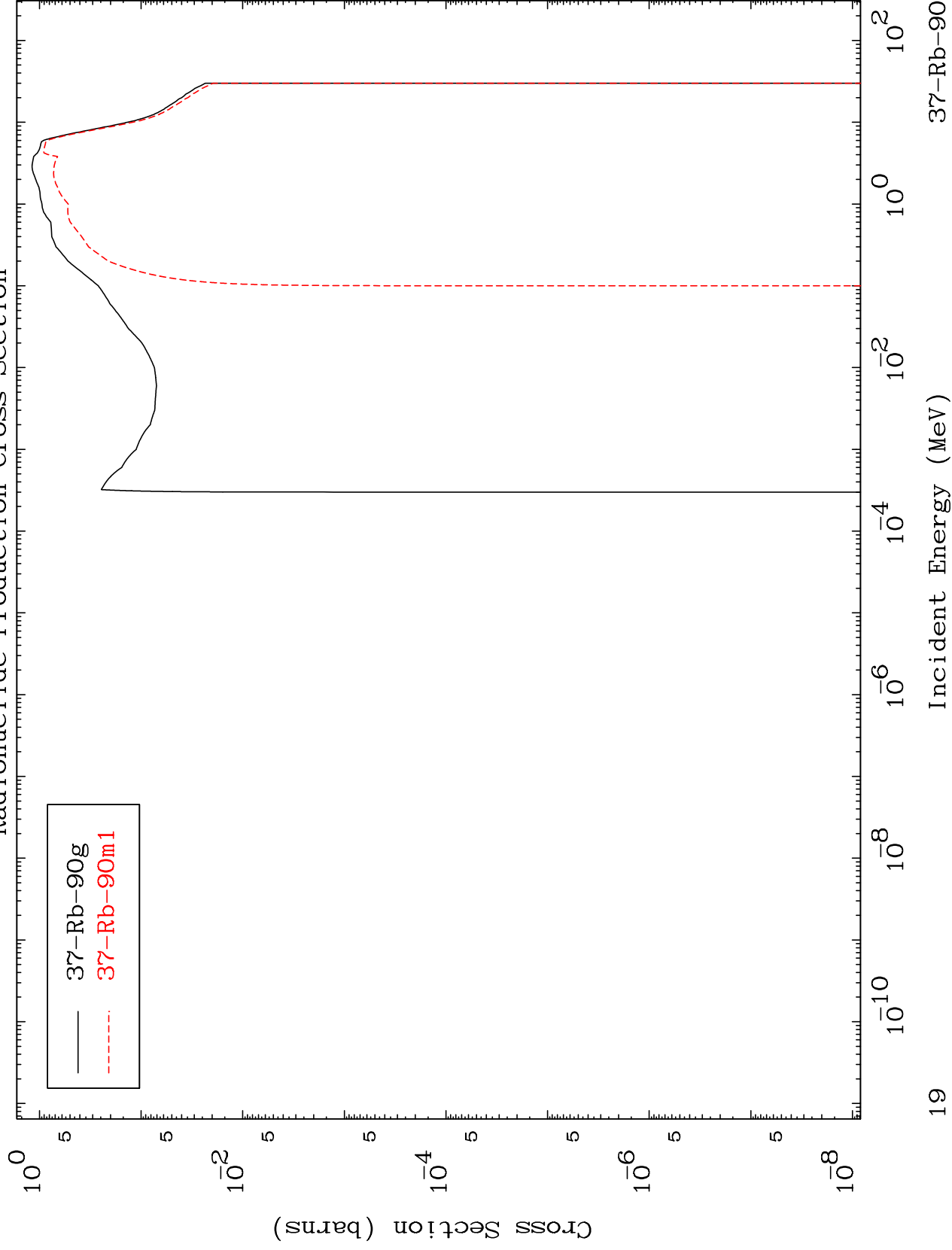
37-Rb-90



MAT 3741

37-Rb-90

Inelastic  
Radionuclide Production Cross Section

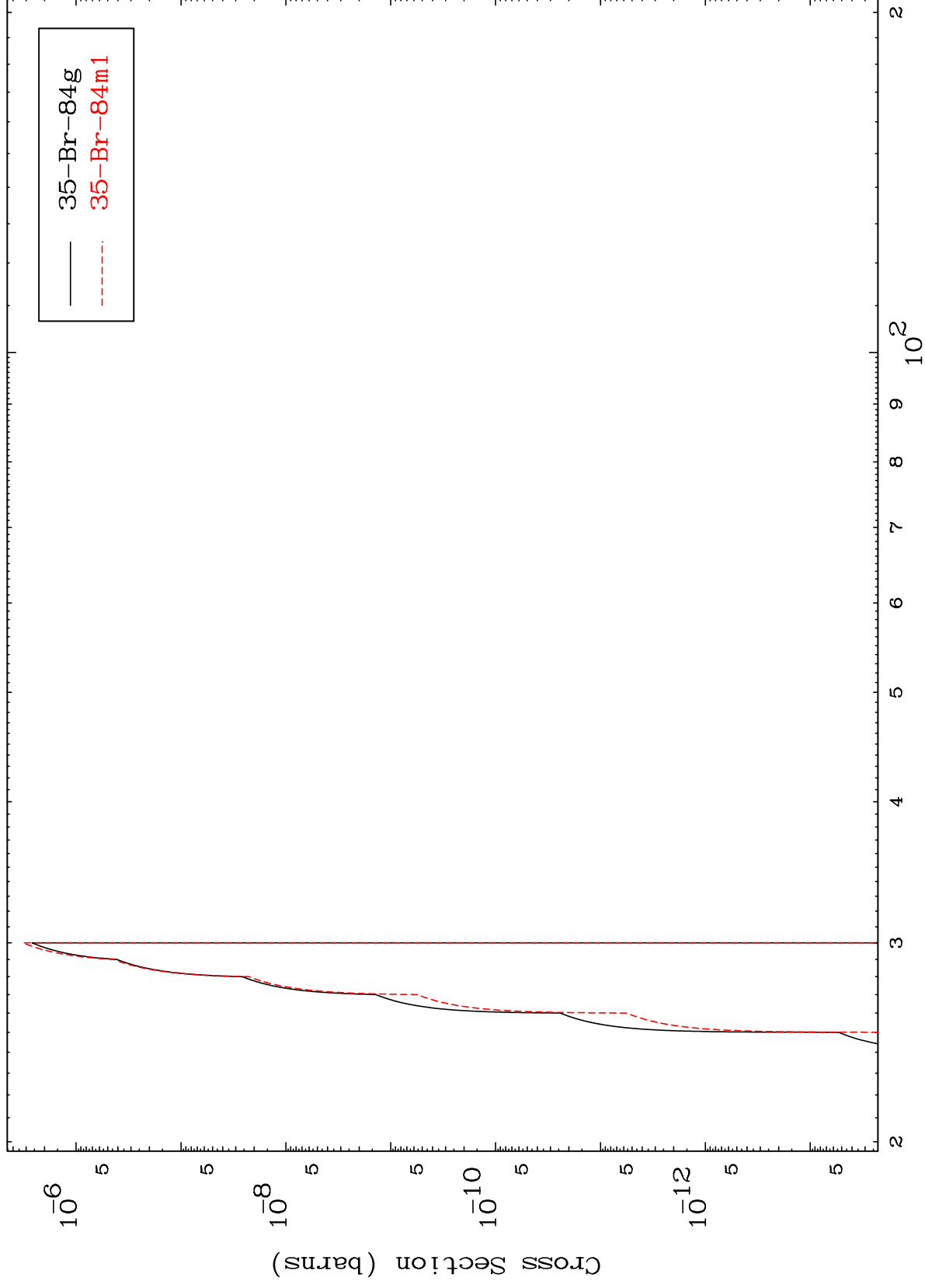


MAT 3741

$(n,3n) \alpha$

$^{37}\text{Rb-90}$

Radionuclide Production Cross Section



20

Incident Energy (MeV)

$^{37}\text{Rb-90}$