

Program EVALPLOT  
(Version 2018-1)

by

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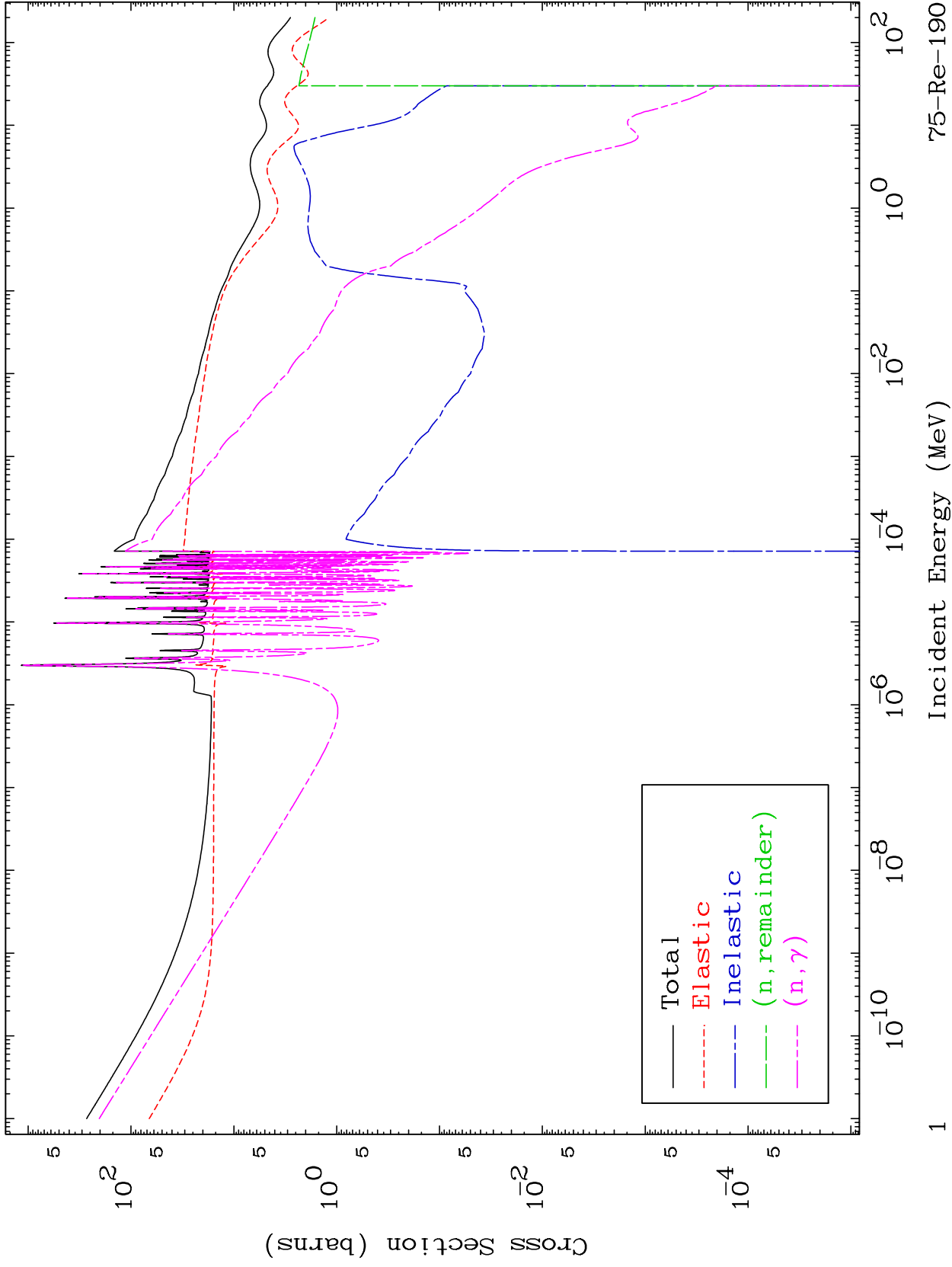
Press Mouse Button to Start

MAT 7541

Major

293 Kelvin Cross Sections

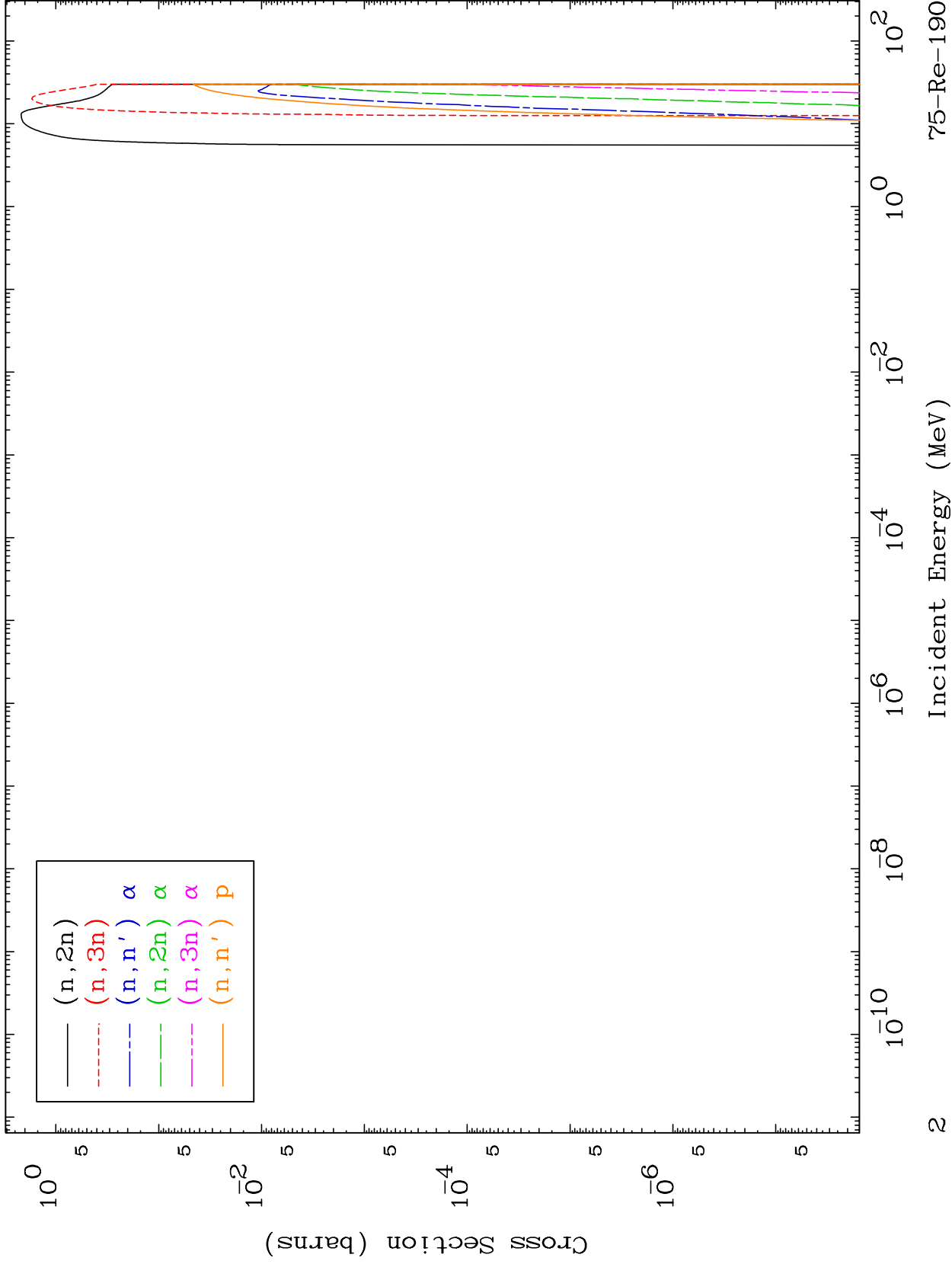
75-Re-190



MAT 7541

Neutron Production  
293 Kelvin Cross Sections

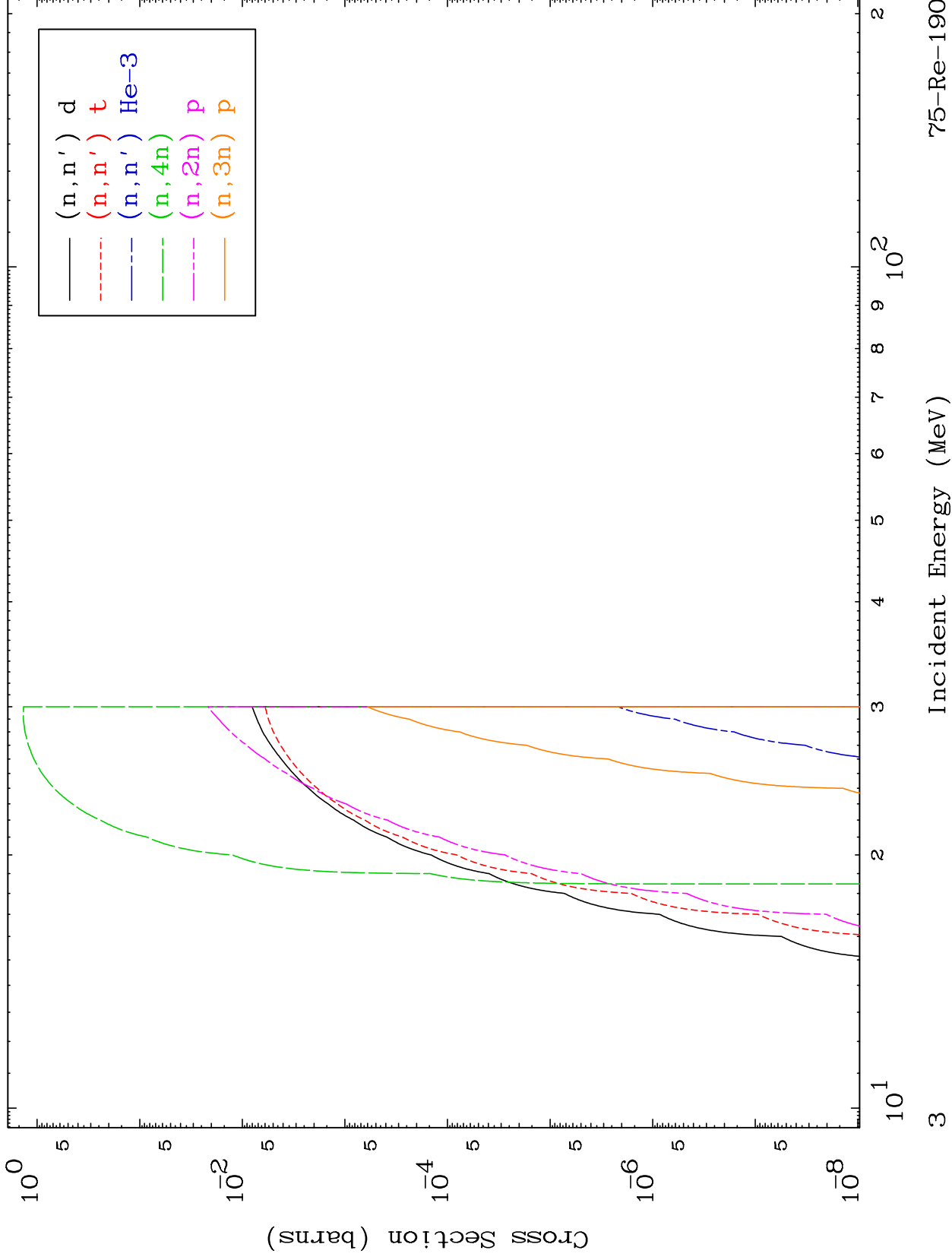
75-Re-190



MAT 7541

Neutron Production  
293 Kelvin Cross Sections

75-Re-190



75-Re-190

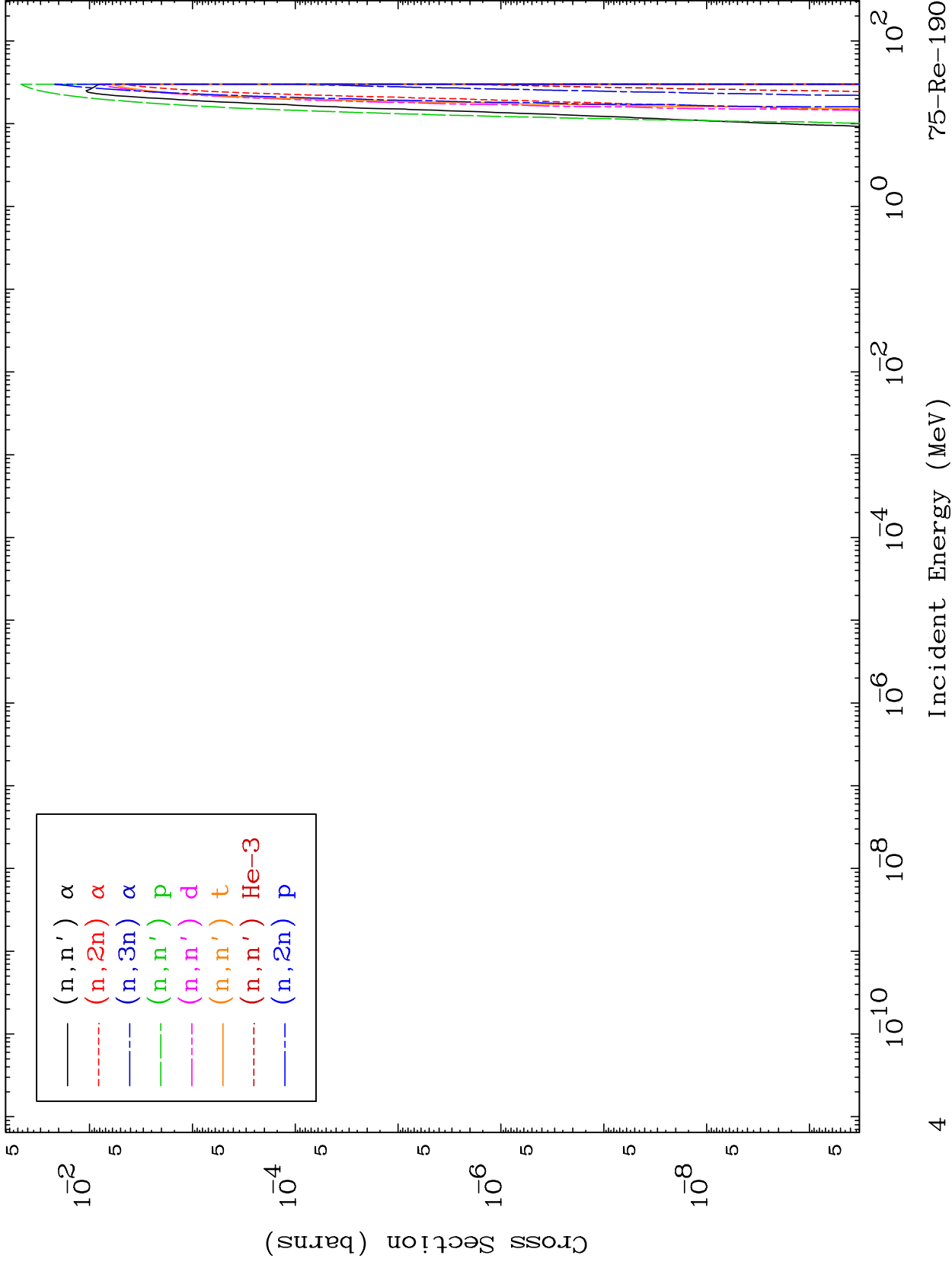
Incident Energy (MeV)

3

MAT 7541

Charged Particle  
293 Kelvin Cross Sections

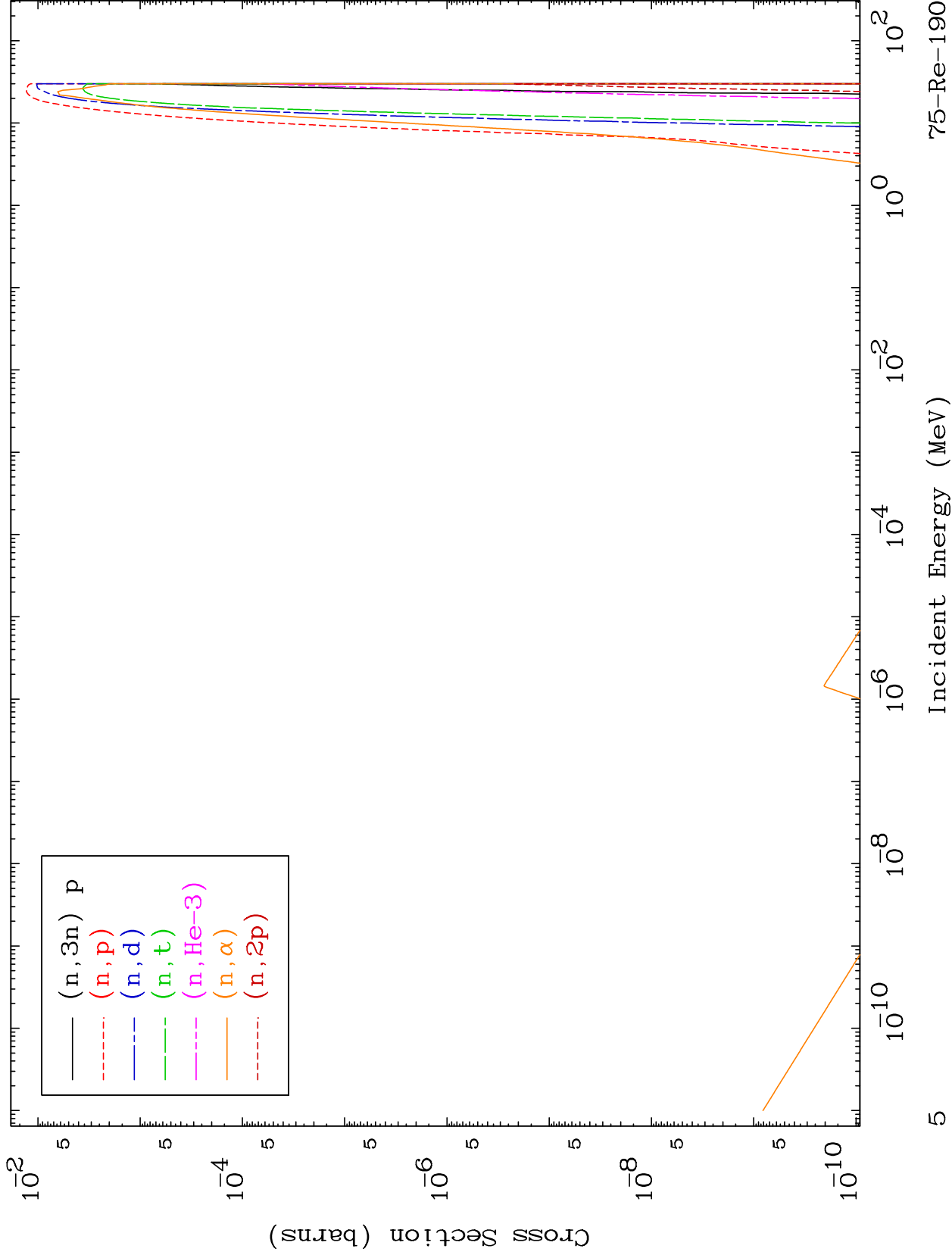
75-Re-190



MAT 7541

Charged Particle  
293 Kelvin Cross Sections

75-Re-190



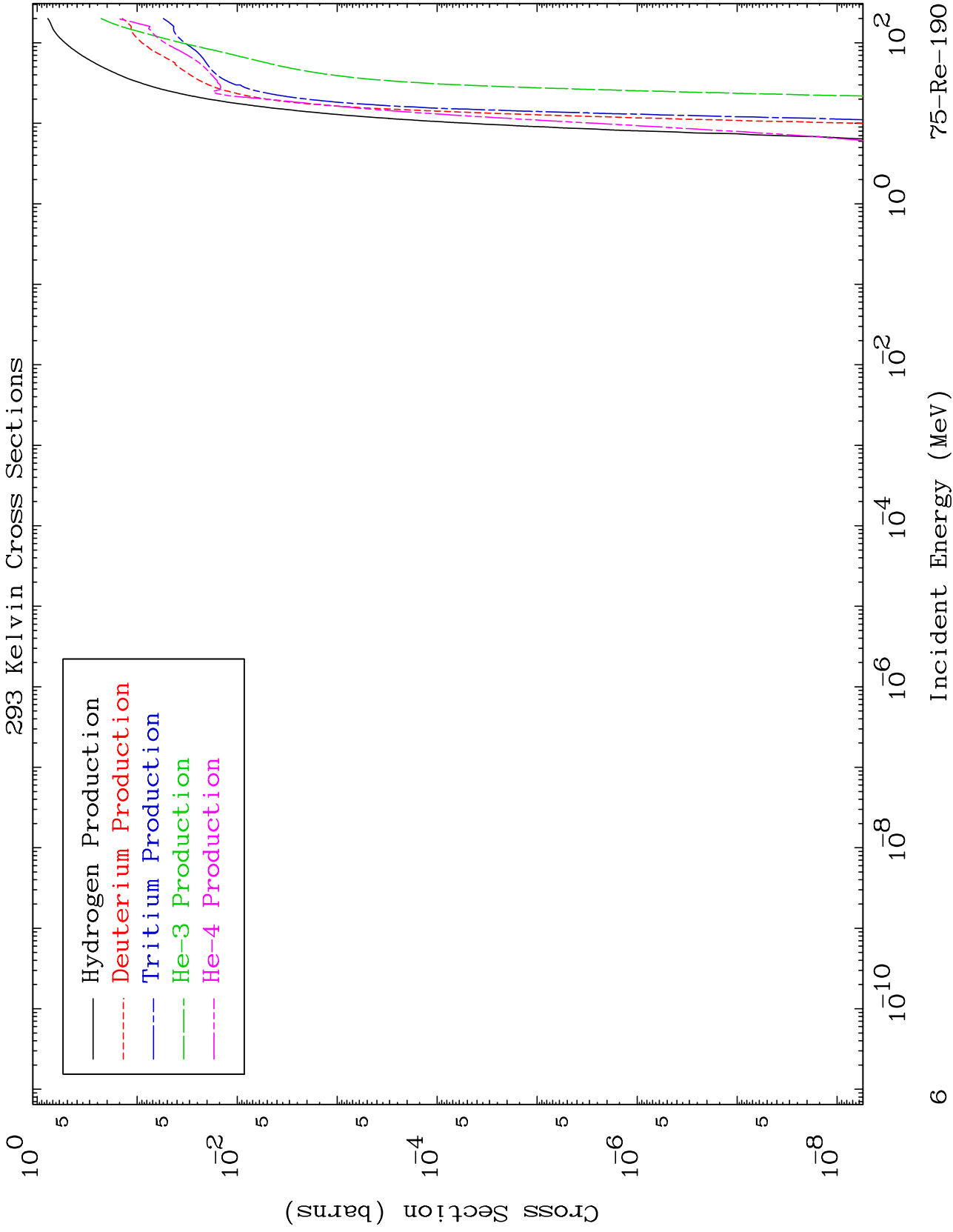
5

75-Re-190

MAT 7541

Particle Production  
293 Kelvin Cross Sections

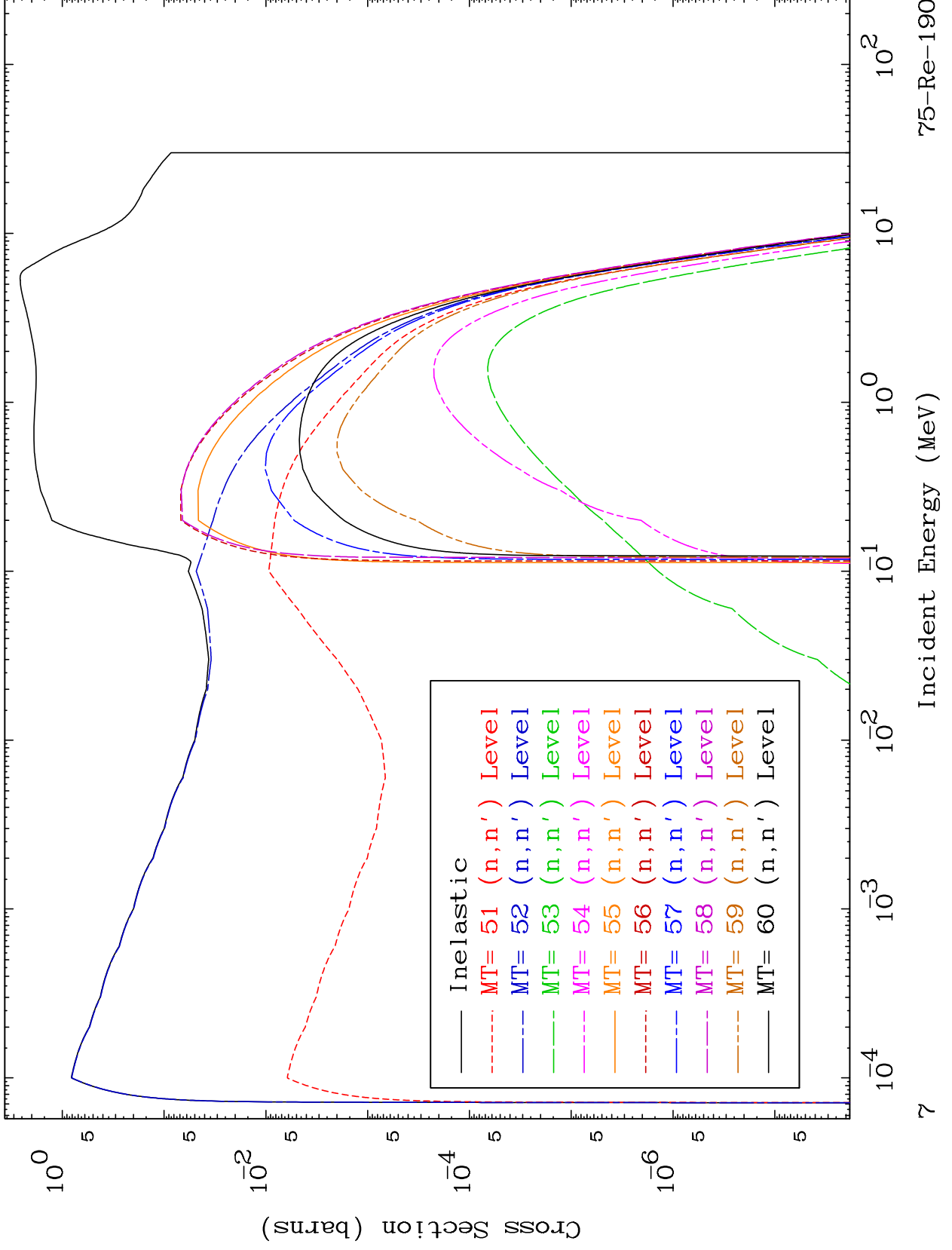
75-Re-190



MAT 7541

(n,n') Level  
293 Kelvin Cross Sections

75-Re-190



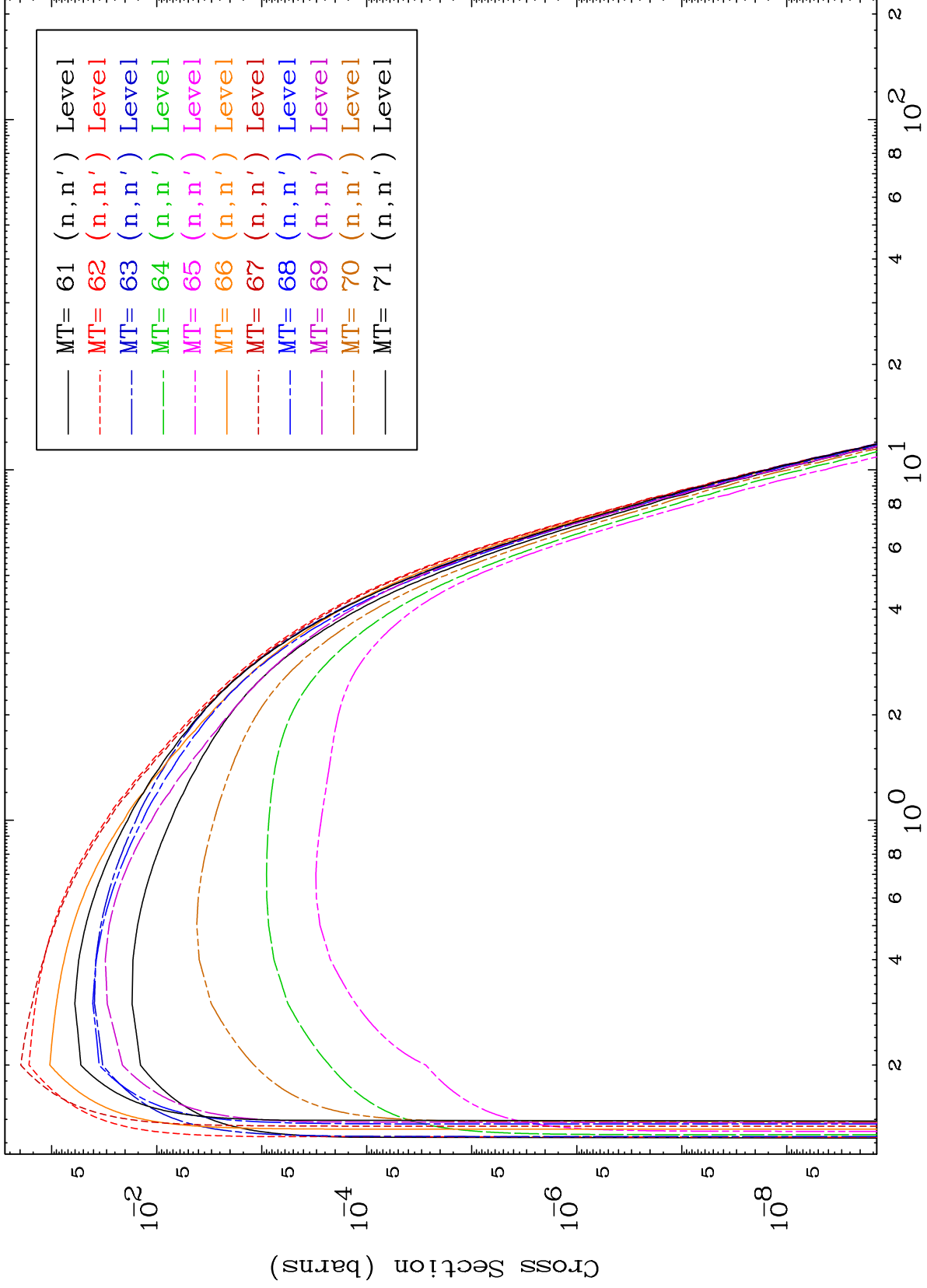


MAT 7541

(n,n') Level

75-Re-190

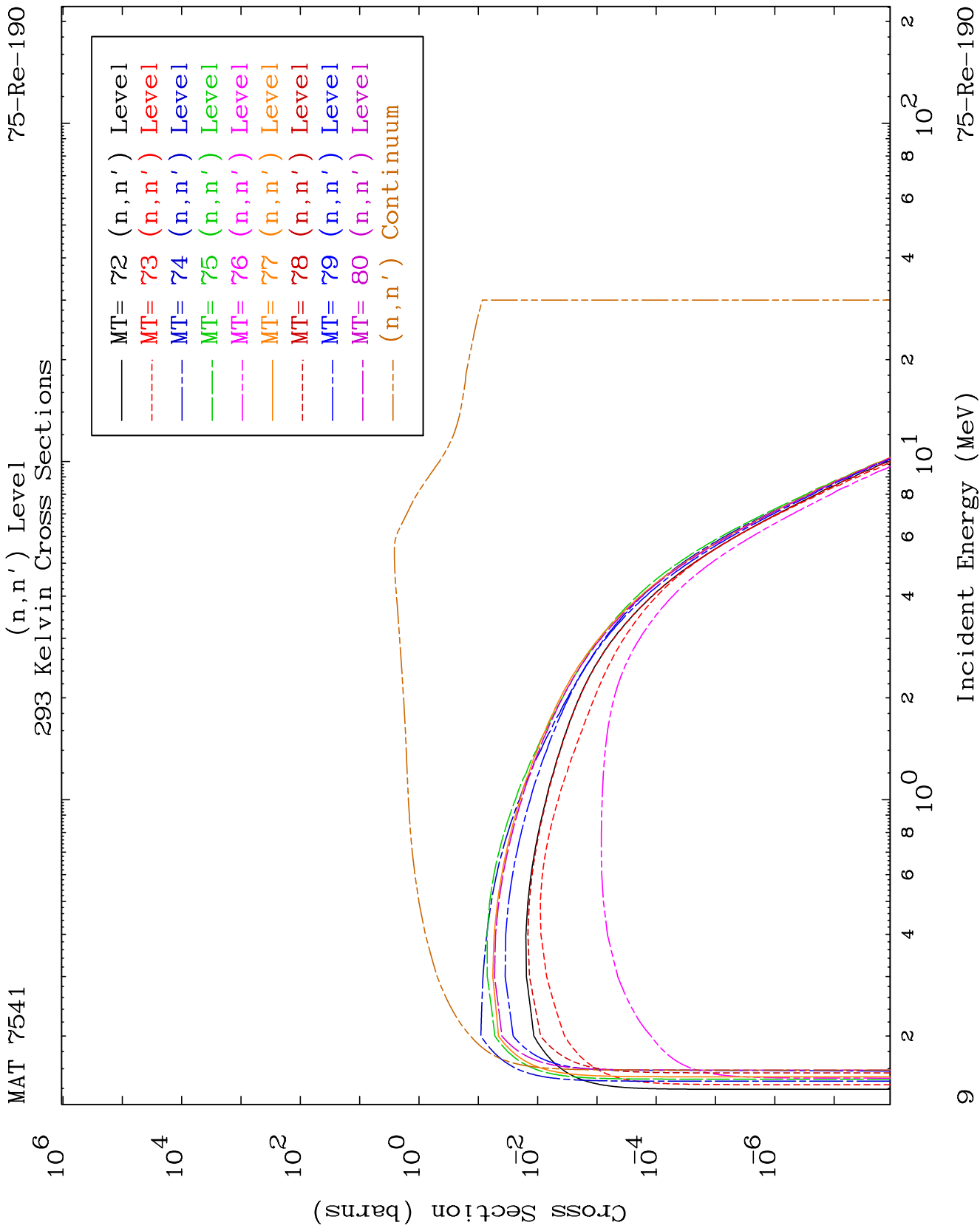
293 Kelvin Cross Sections



8

Incident Energy (MeV)

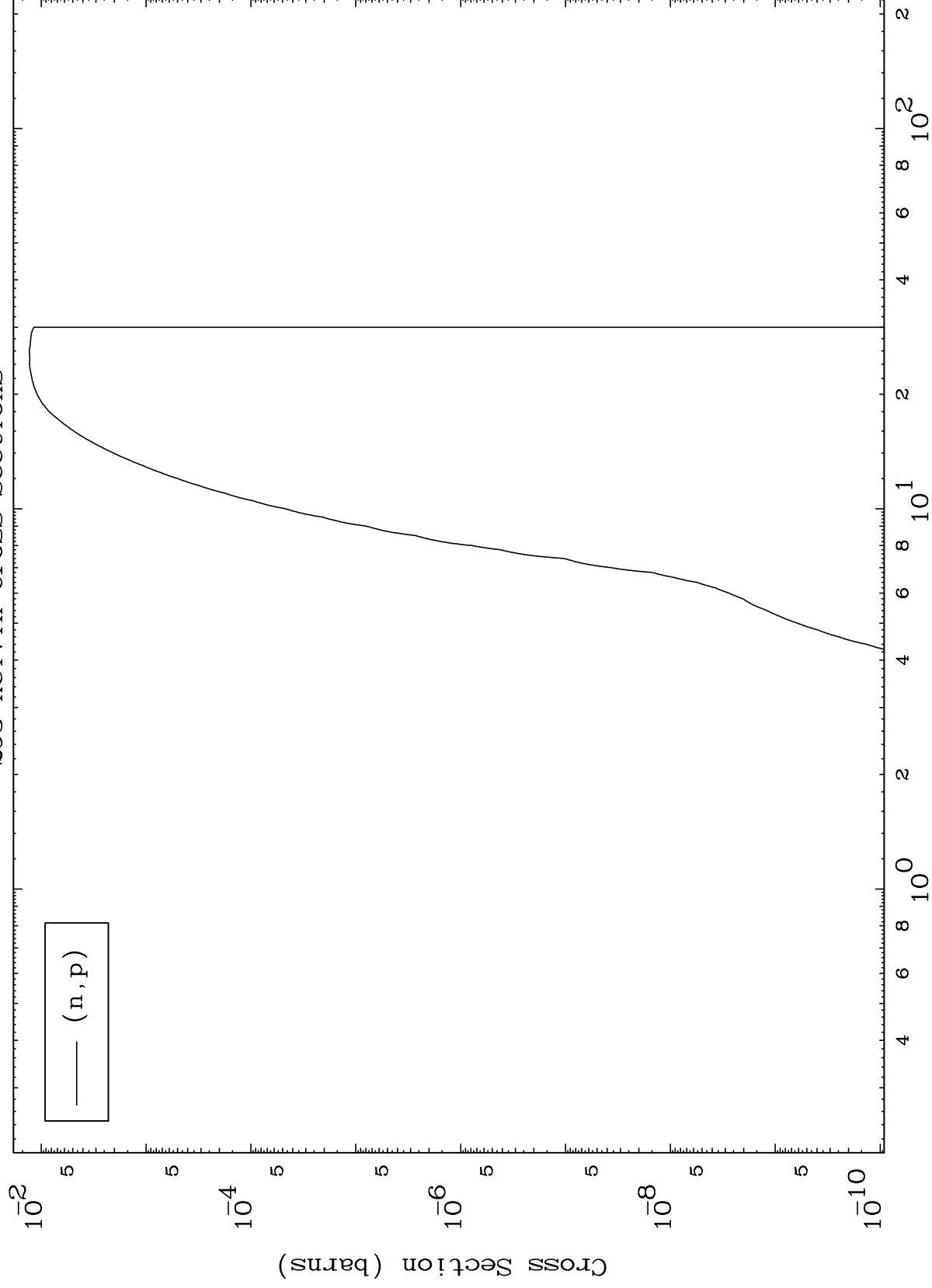
75-Re-190



MAT 7541

(n,p) Levels  
293 Kelvin Cross Sections

75-Re-190



10

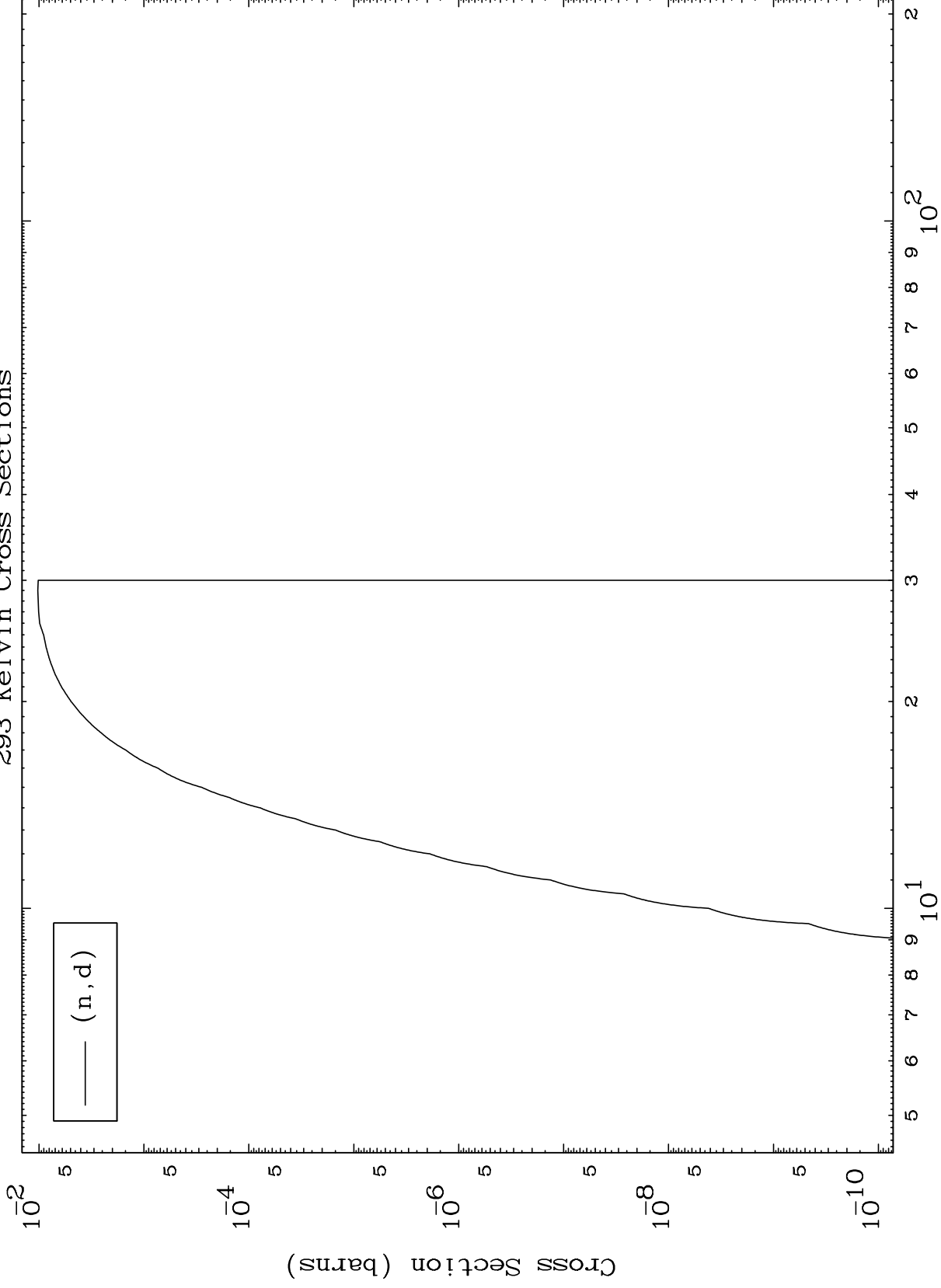
Incident Energy (MeV)

75-Re-190

MAT 7541

(n,d) Levels  
293 Kelvin Cross Sections

75-Re-190



11

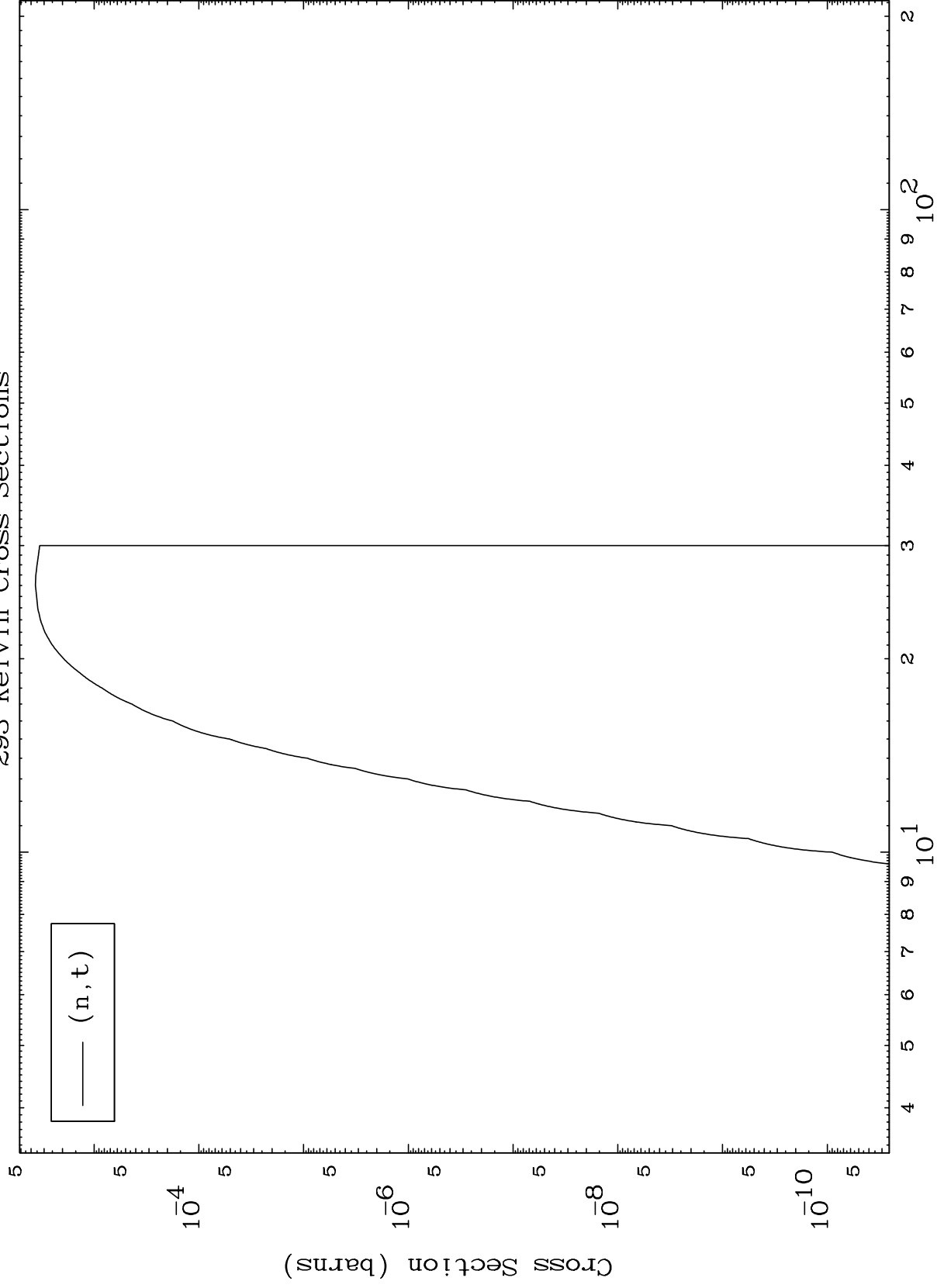
Incident Energy (MeV)

75-Re-190

MAT 7541

(n,t) Levels  
293 Kelvin Cross Sections

75-Re-190



Incident Energy (MeV)

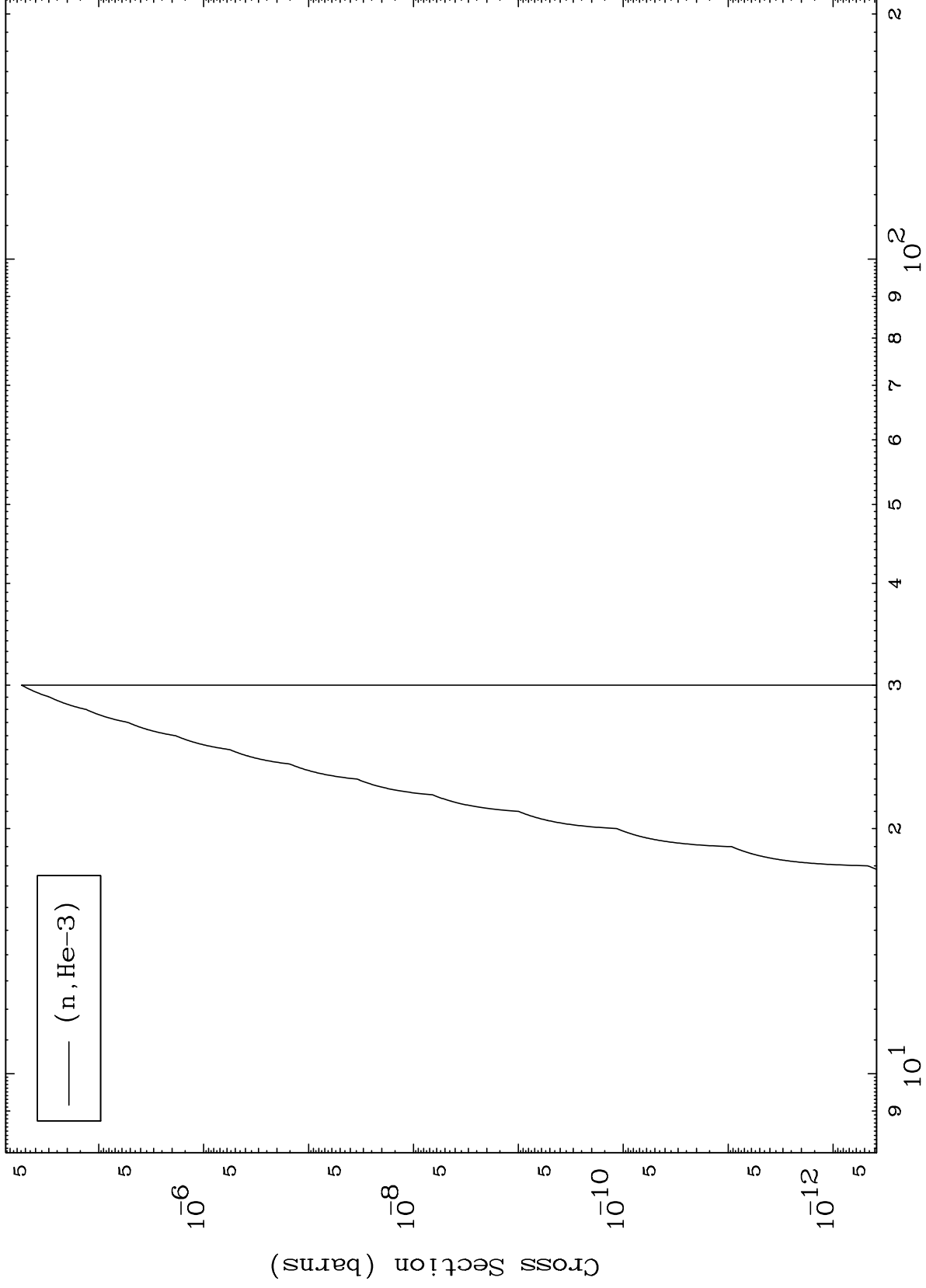
75-Re-190

12

MAT 7541

(n,He3) Levels  
293 Kelvin Cross Sections

75-Re-190



13

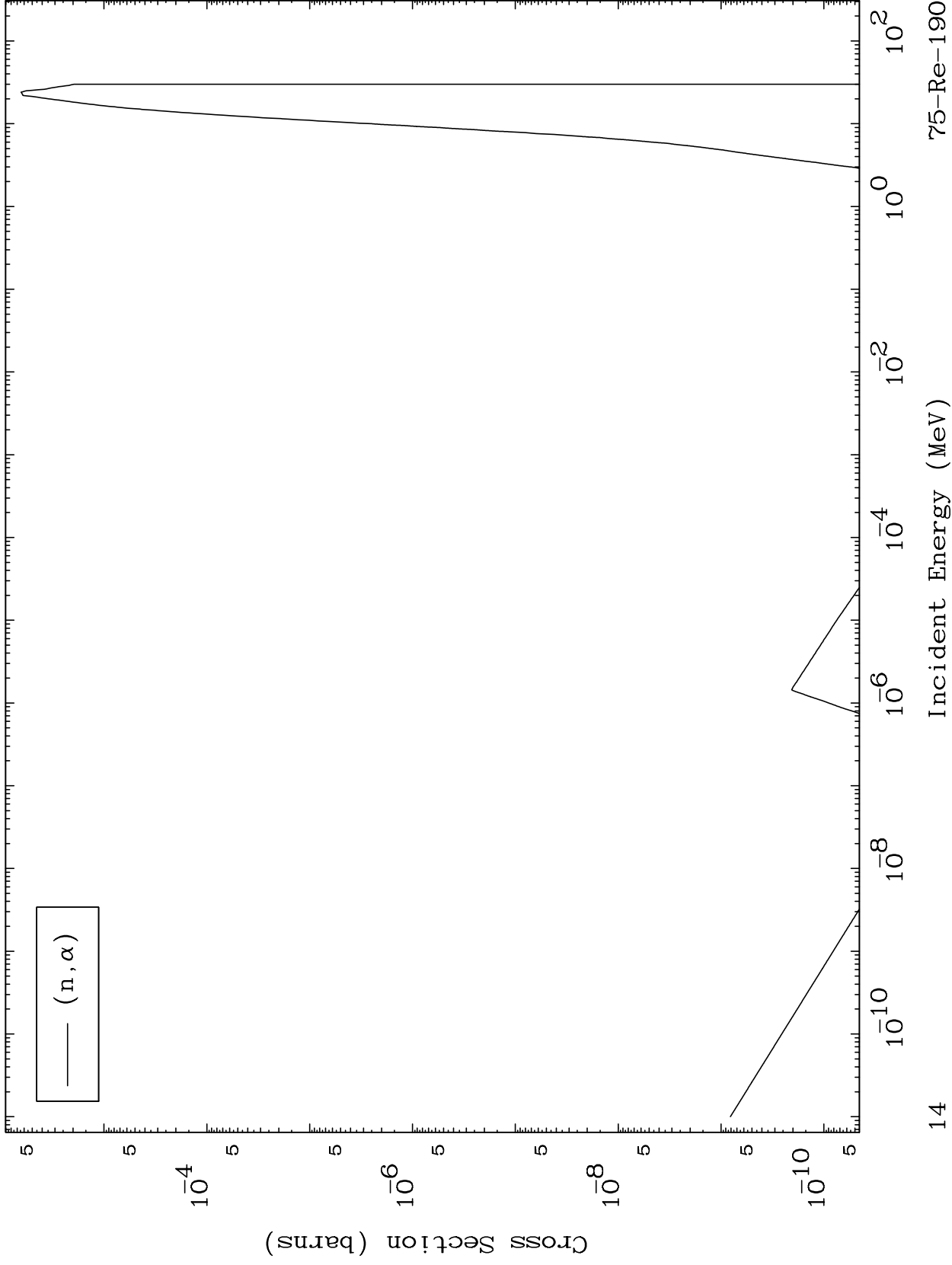
Incident Energy (MeV)

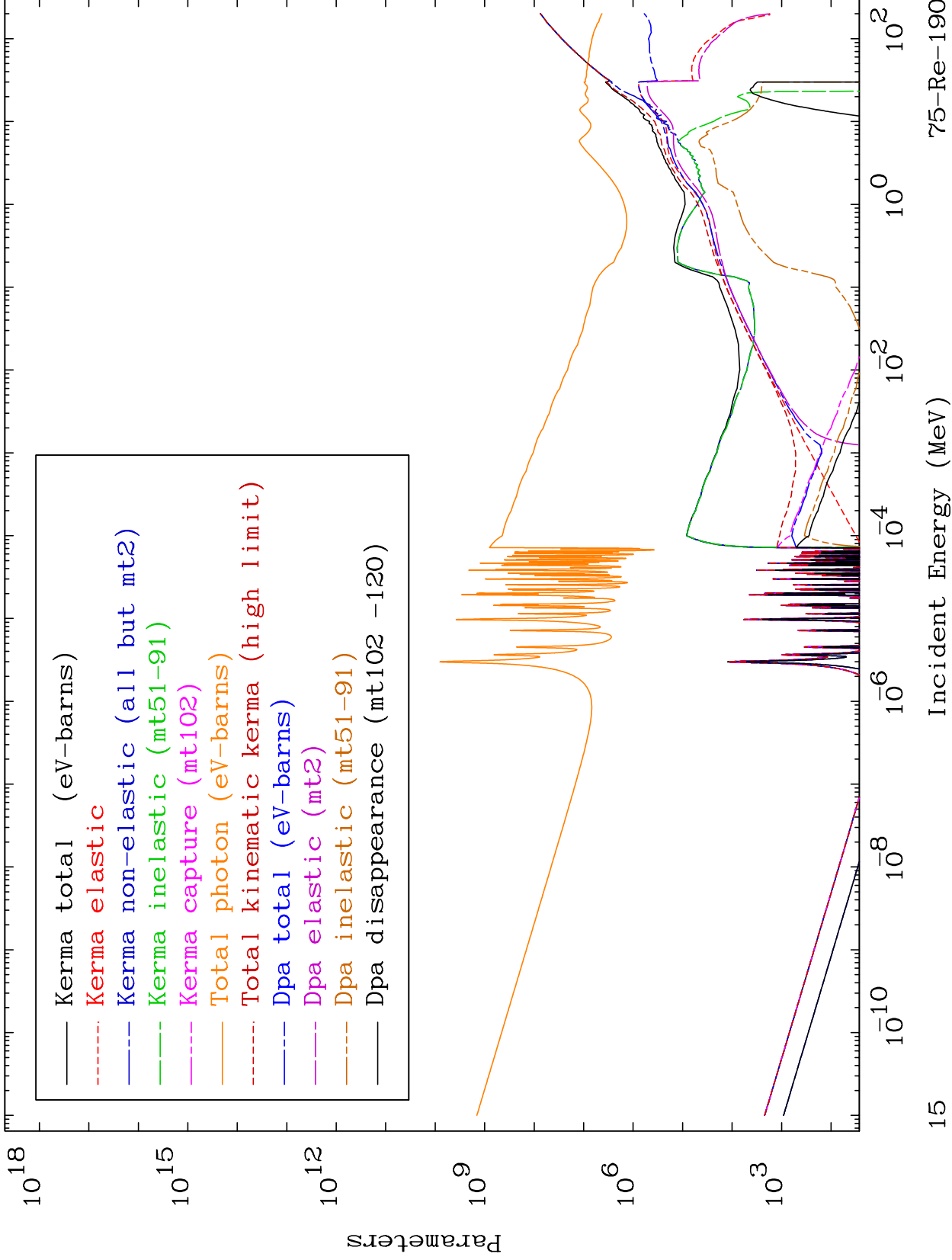
75-Re-190

MAT 7541

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

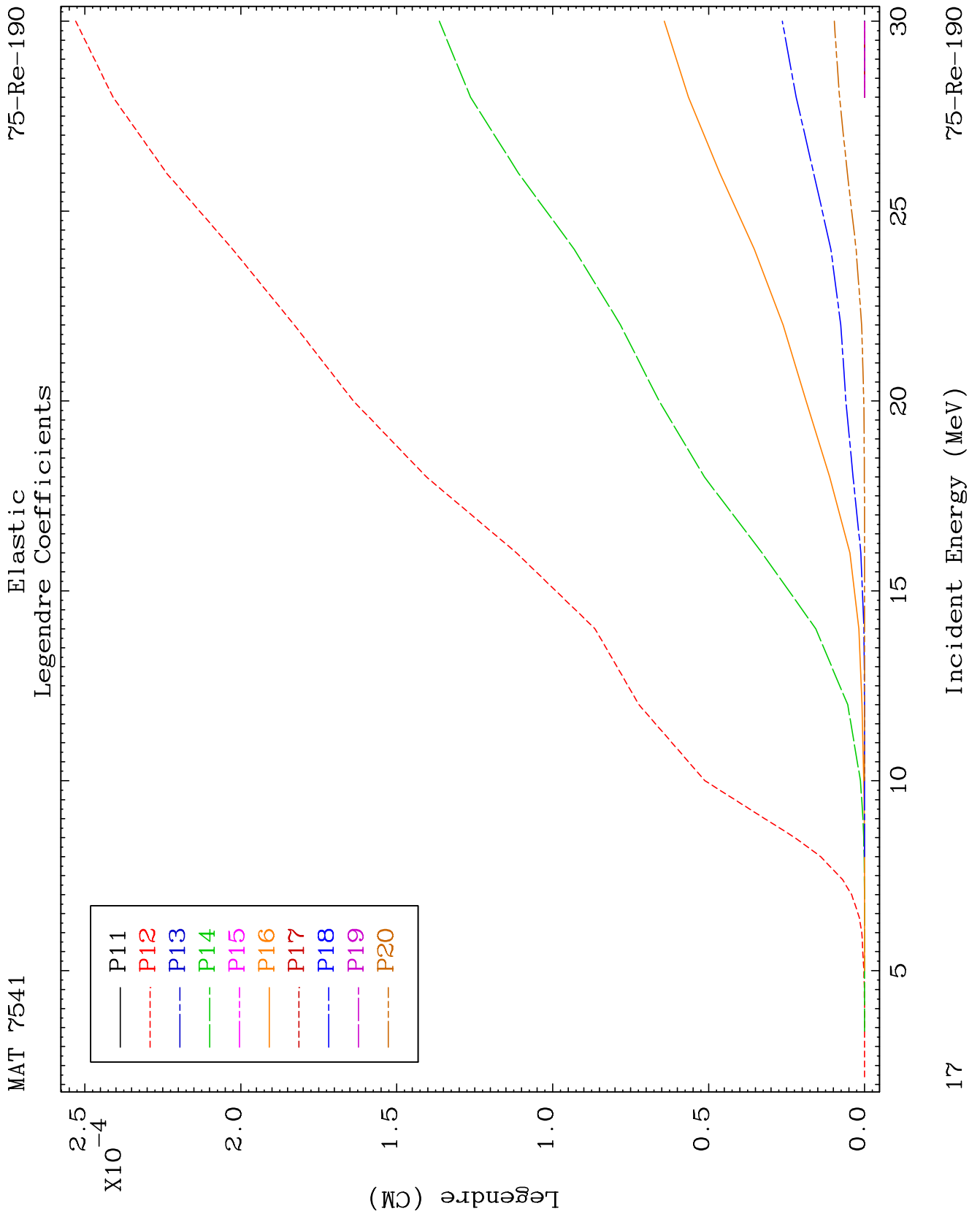
75-Re-190









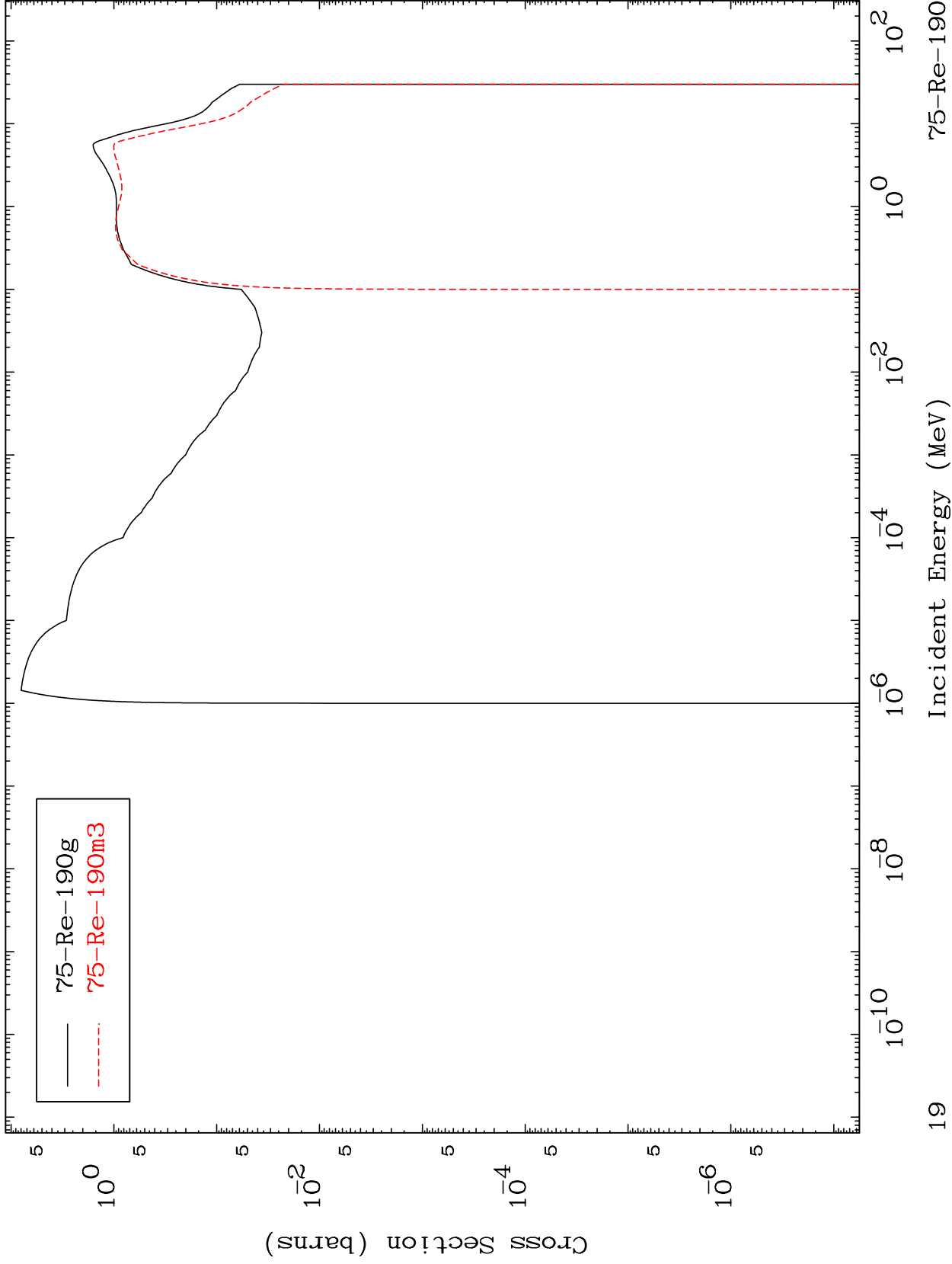




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Radionuclide Production Cross Section

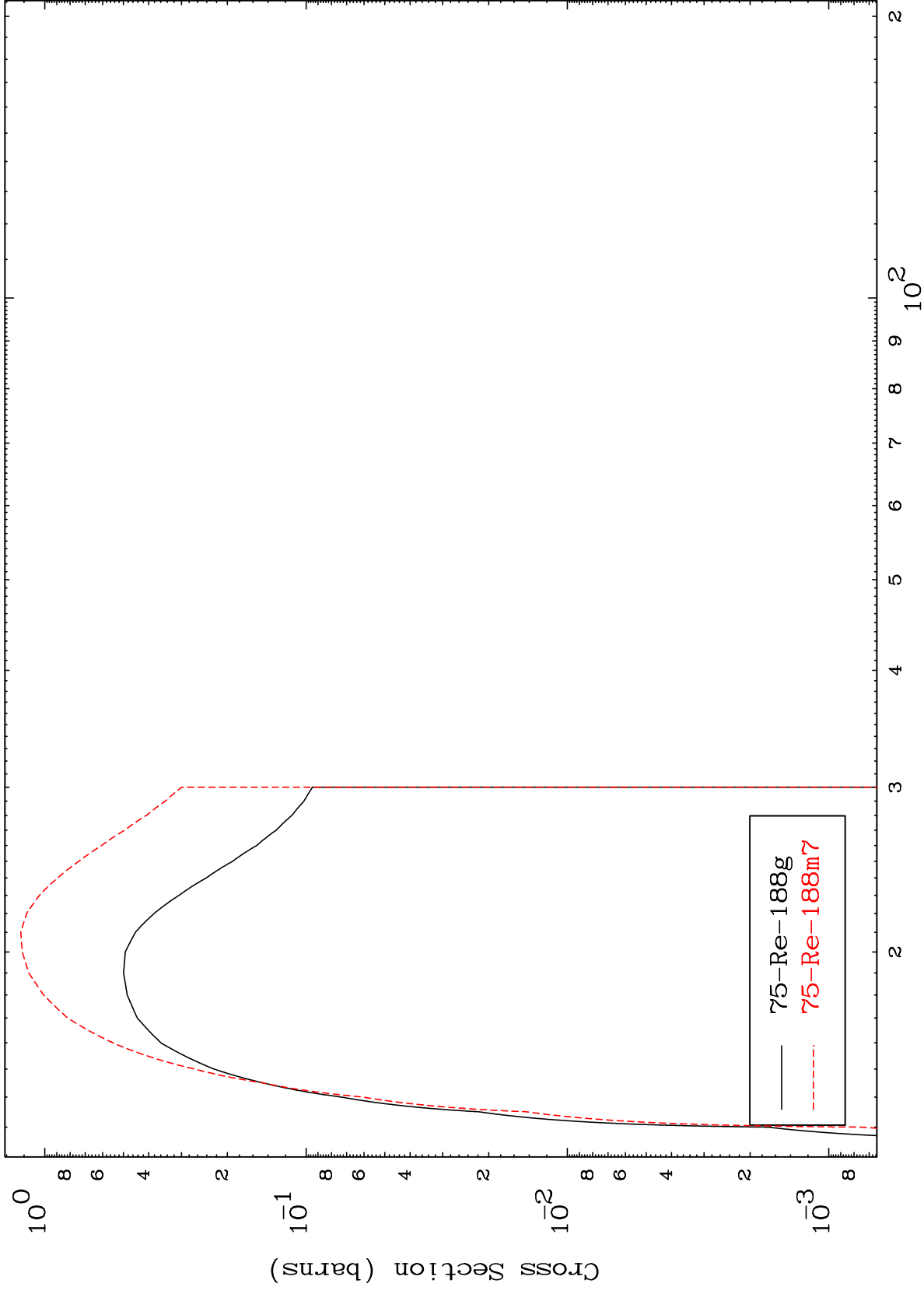
75-Re-190



MAT 7541

75-Re-190

(n,3n)  
Radionuclide Production Cross Section



75-Re-190

Incident Energy (MeV)

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MAT 7541

Fission

<sup>75</sup>Re-190

Radionuclide Production Cross Section

