

Program EVALPLOT  
(Version 2018-1)

by

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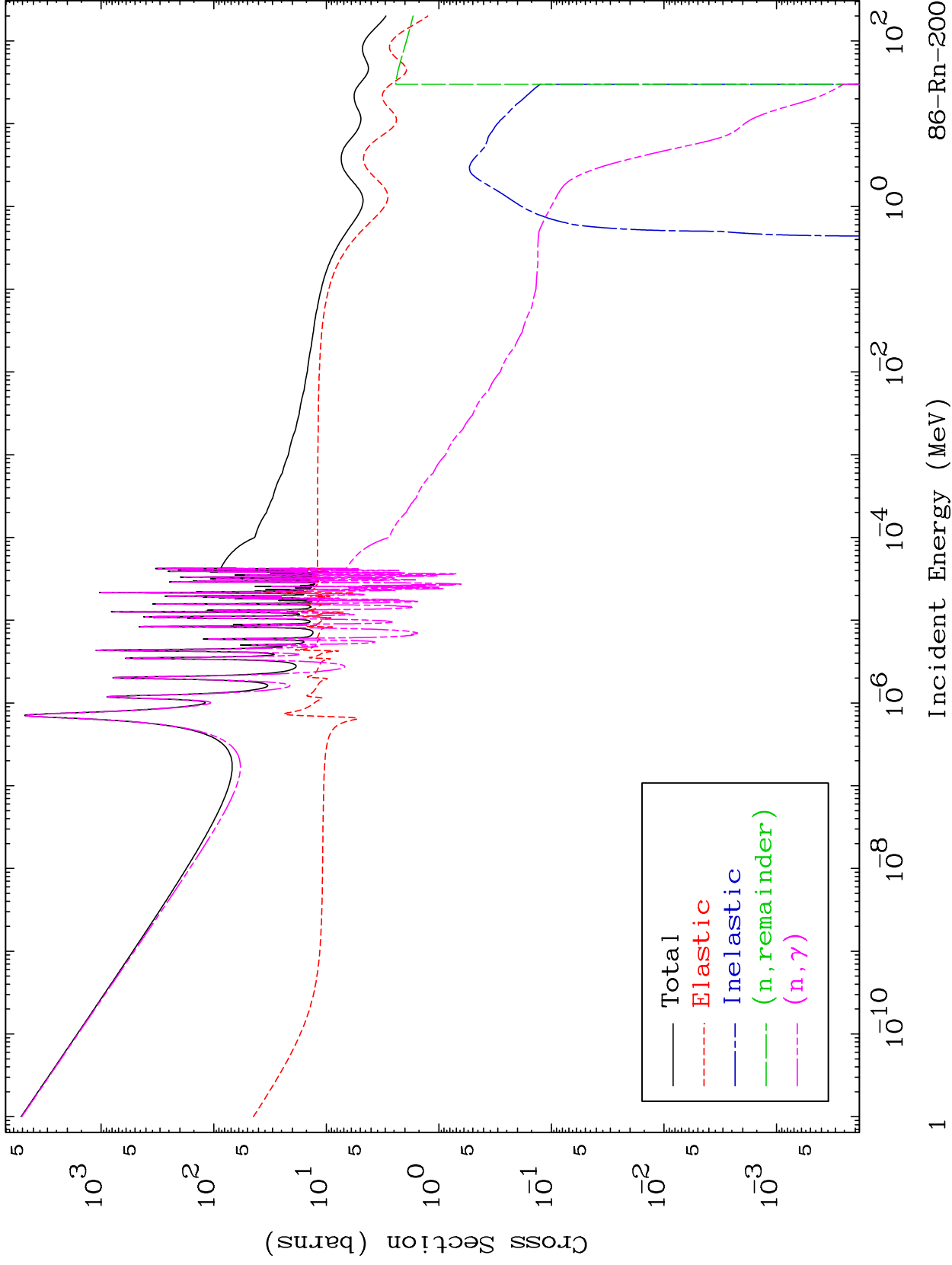
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Press Mouse Button to Start

MAT 8592

Major  
293 Kelvin Cross Sections

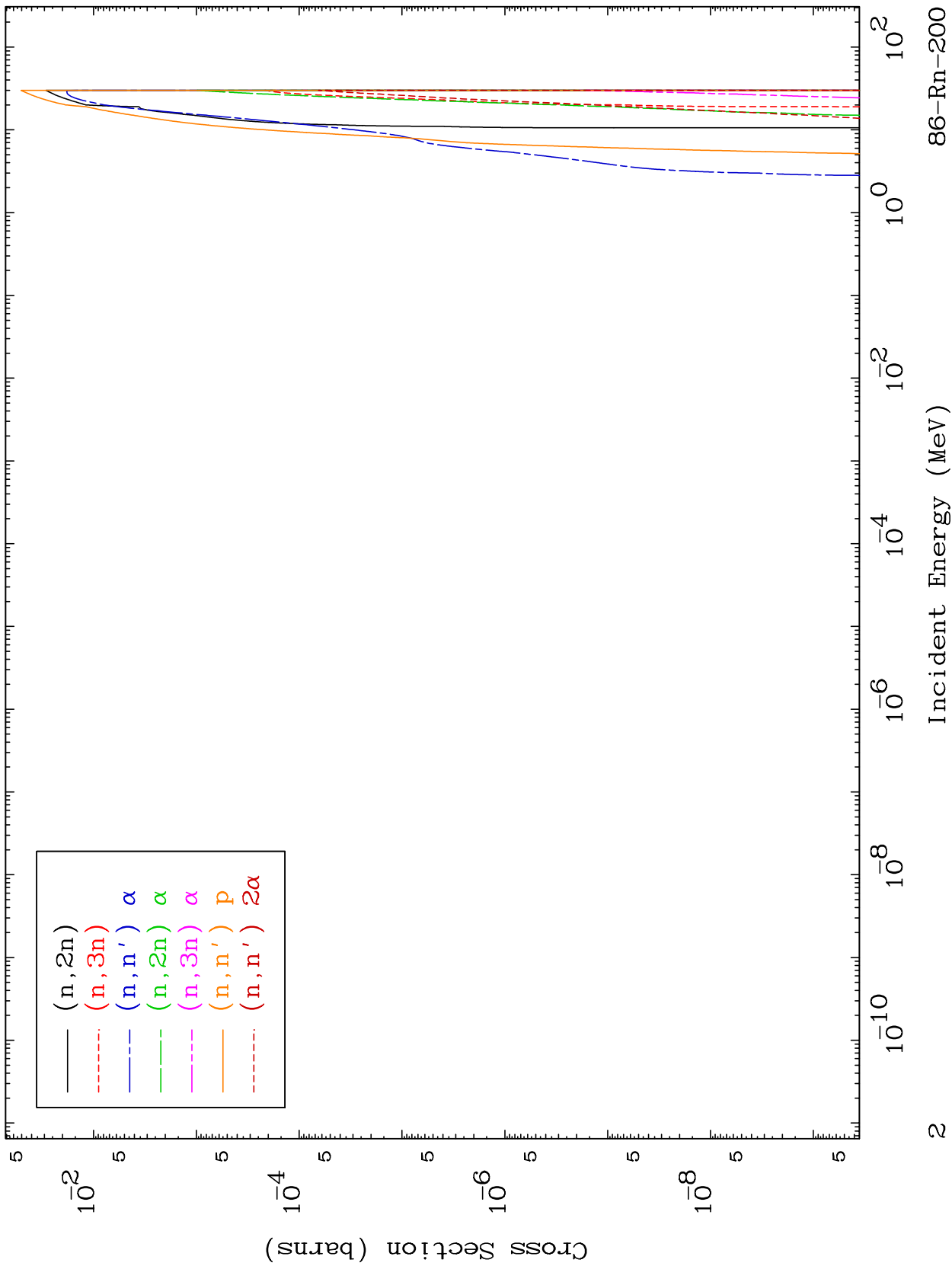
86-Rn-200



MAT 8592

Neutron Production  
293 Kelvin Cross Sections

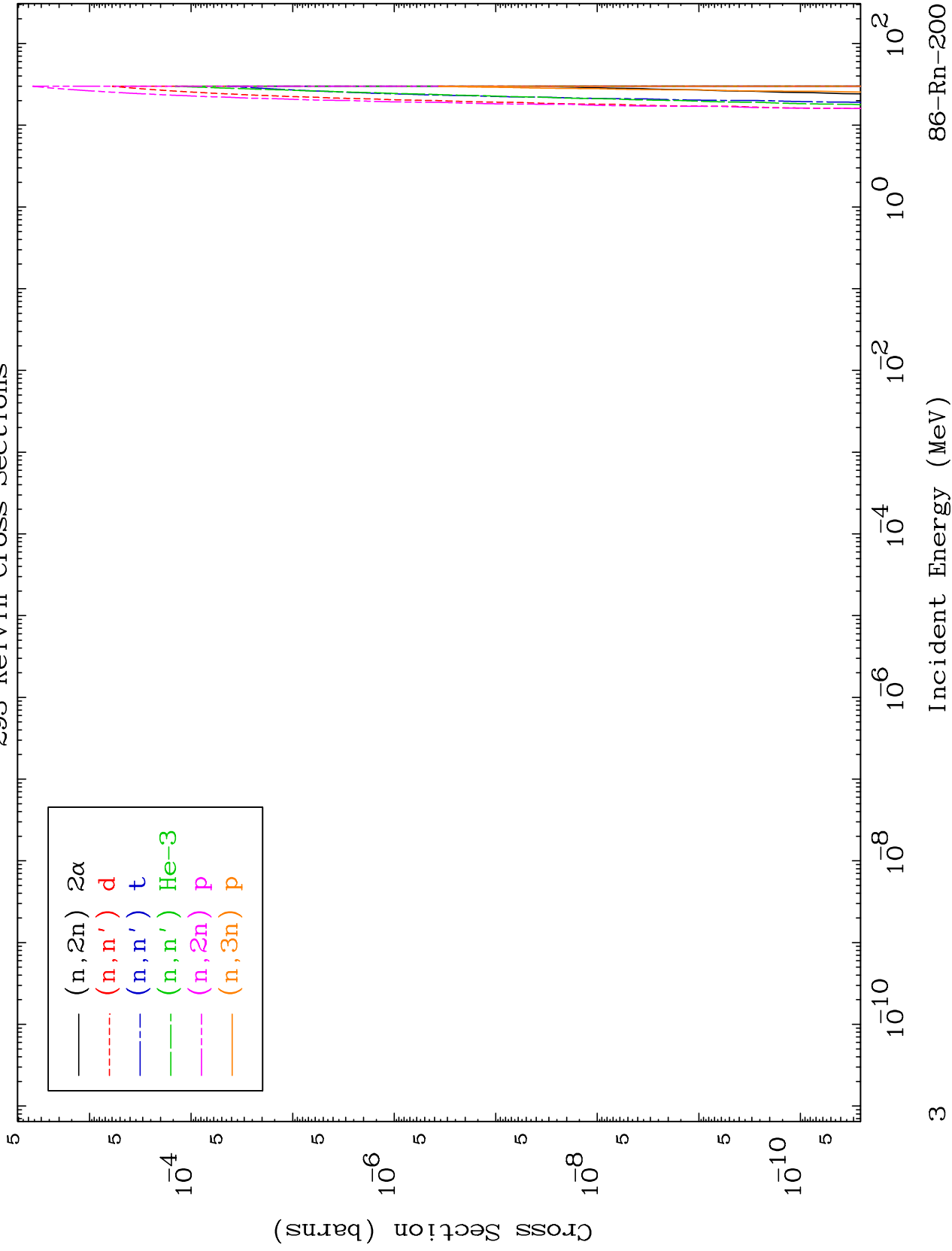
86-Rn-200



MAT 8592

Neutron Production  
293 Kelvin Cross Sections

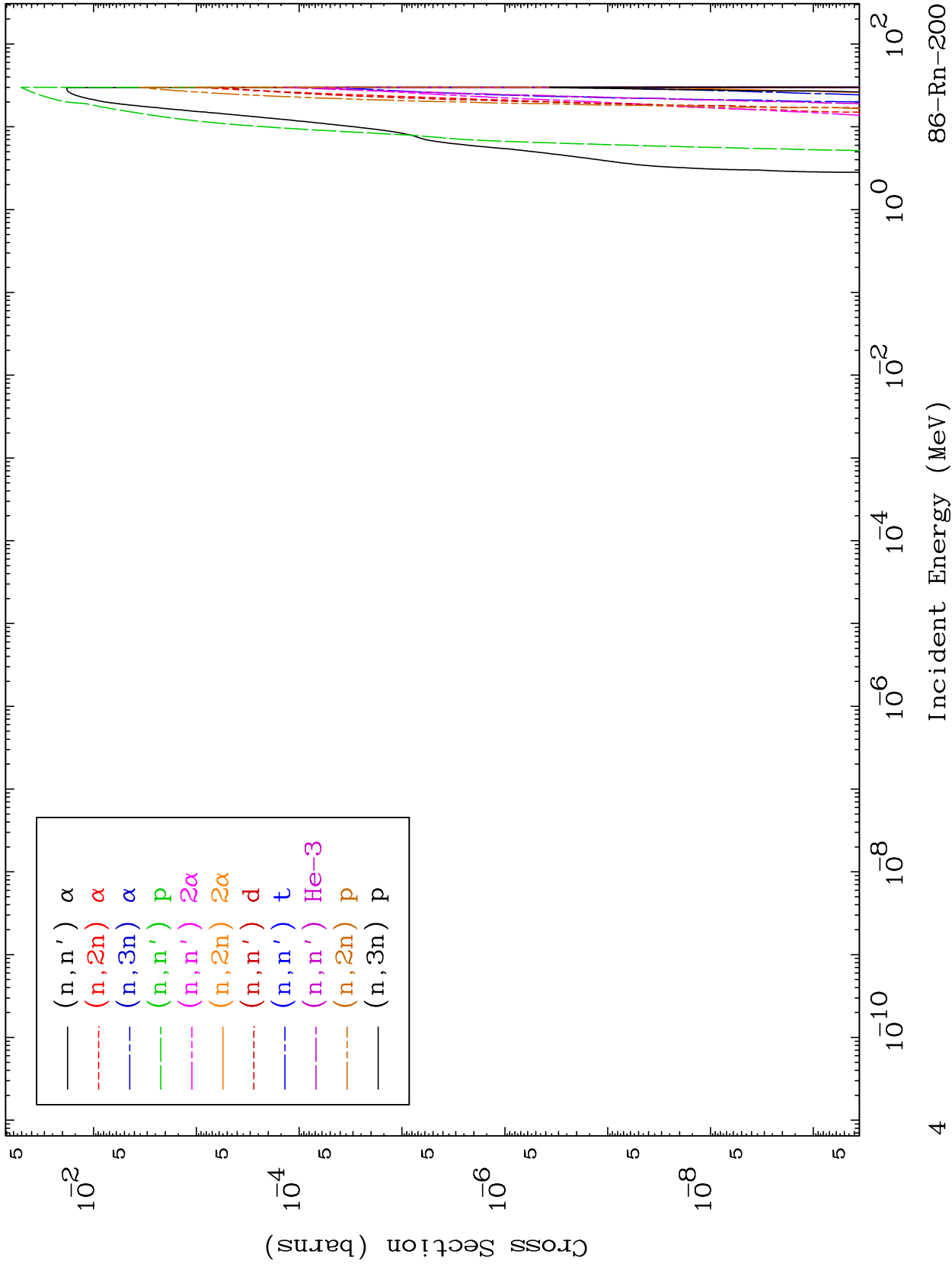
86-Rn-200



MAT 8592

Charged Particle  
293 Kelvin Cross Sections

86-Rn-200

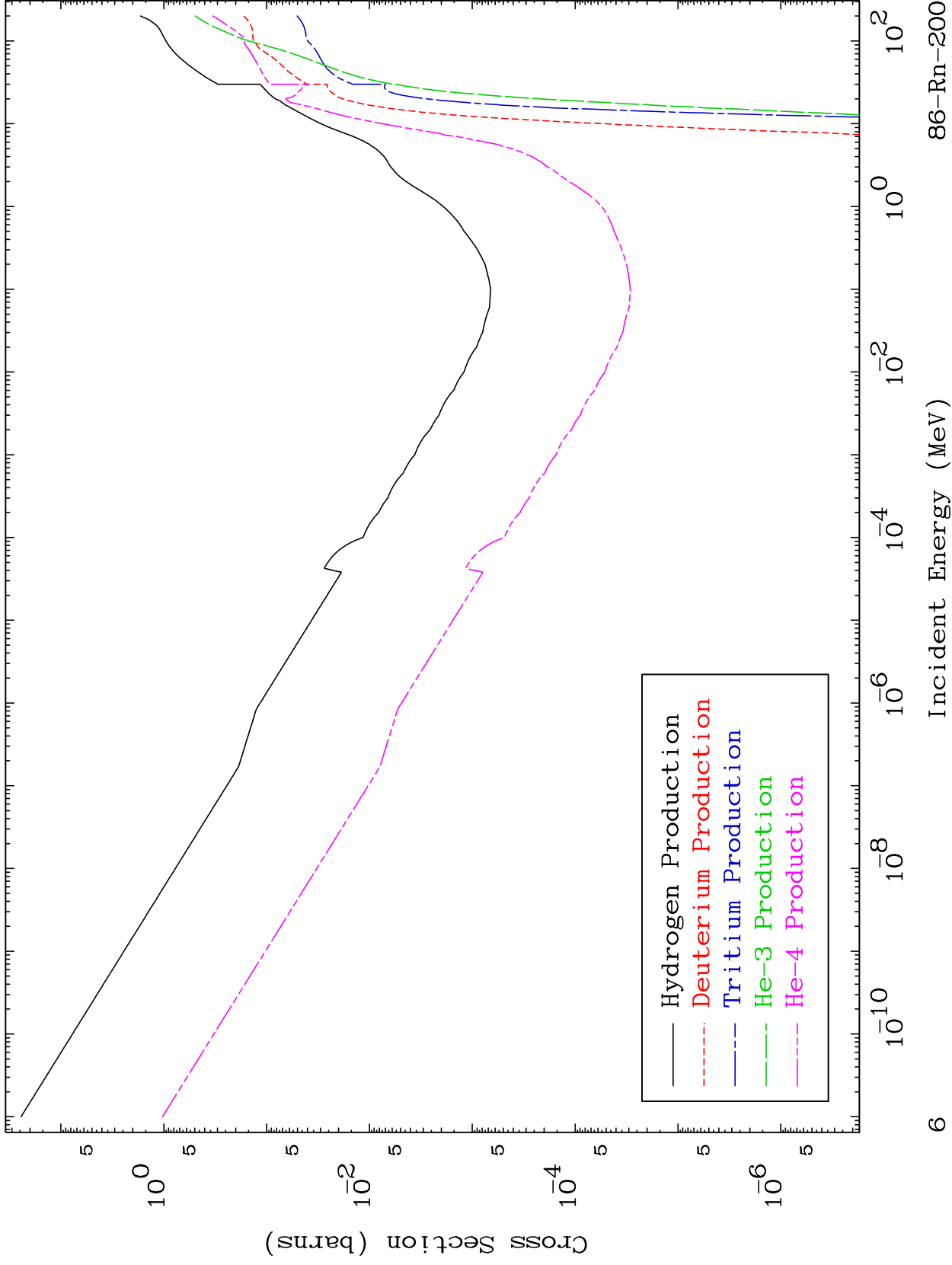




MAT 8592

Particle Production  
293 Kelvin Cross Sections

86-Rn-200

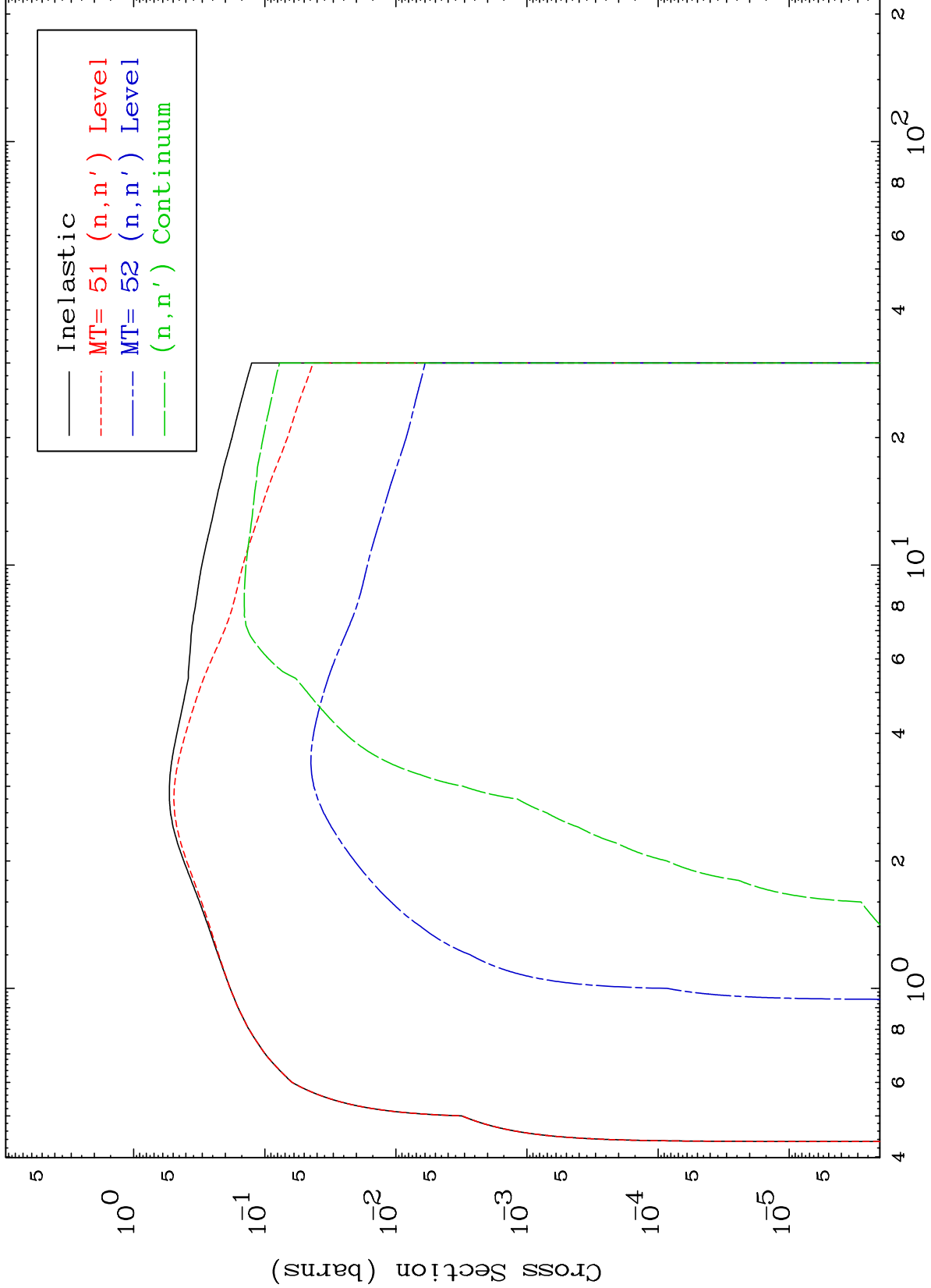


MAT 8592

(n,n') Level

86-Rn-200

293 Kelvin Cross Sections

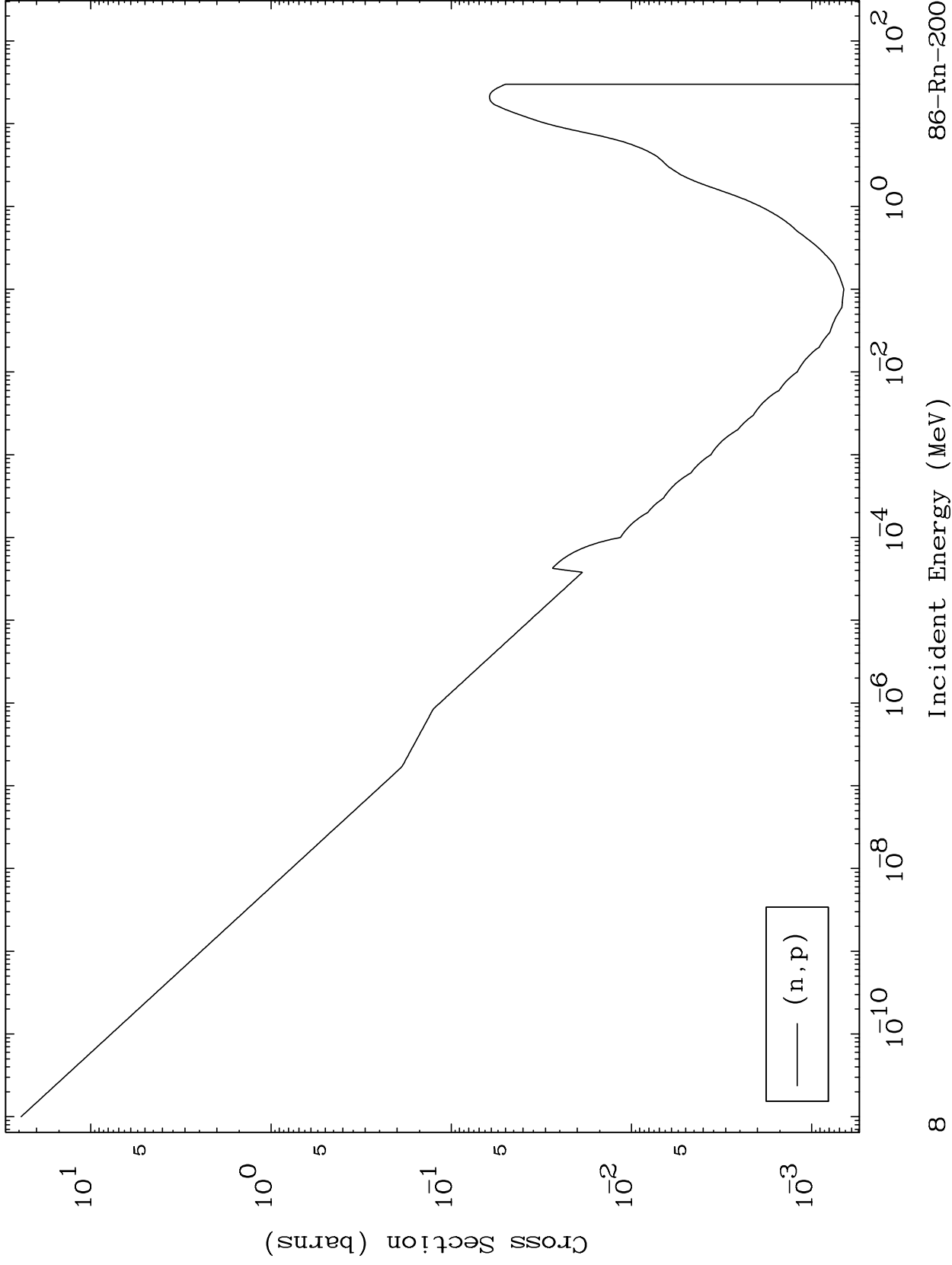




MAT 8592

(n,p) Levels  
293 Kelvin Cross Sections

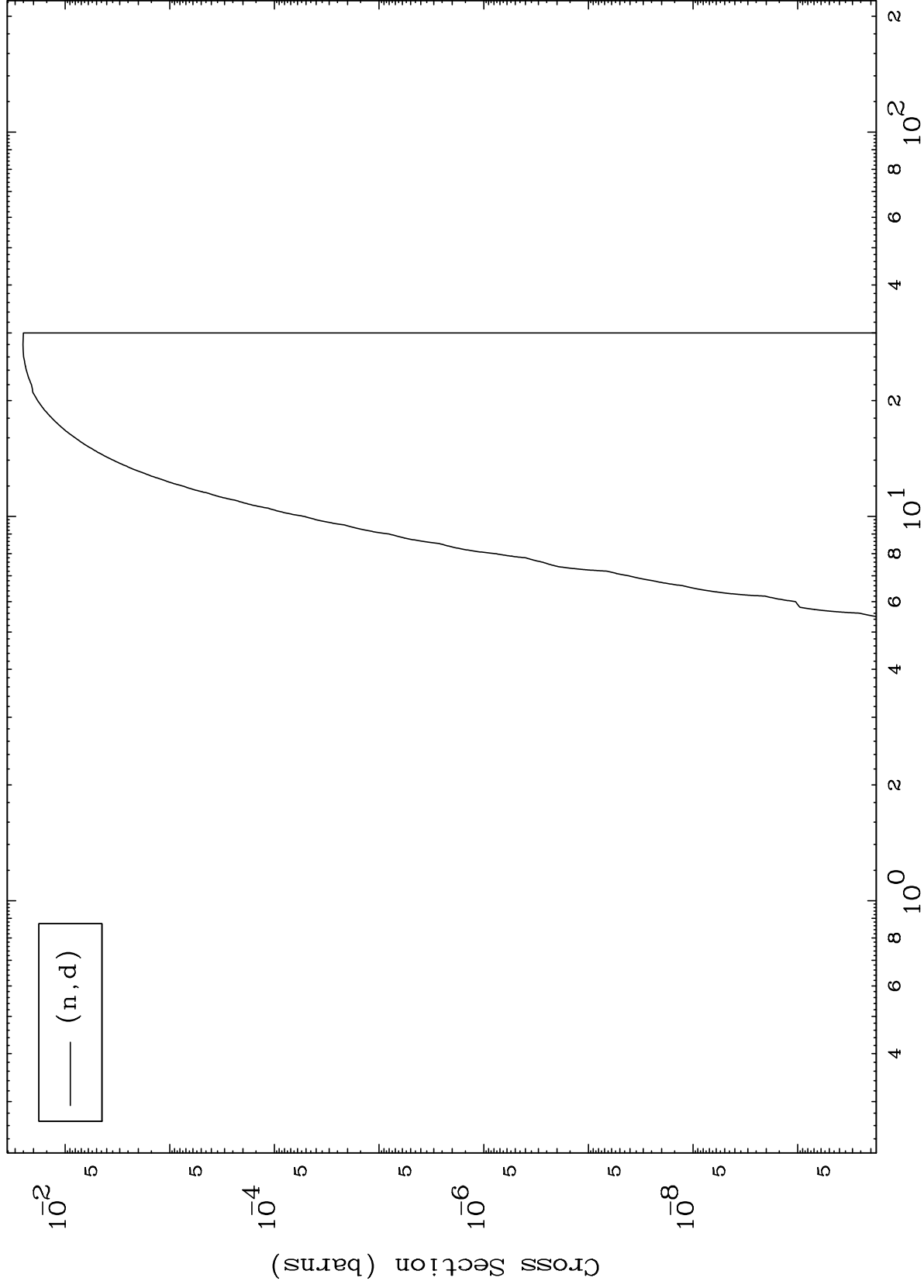
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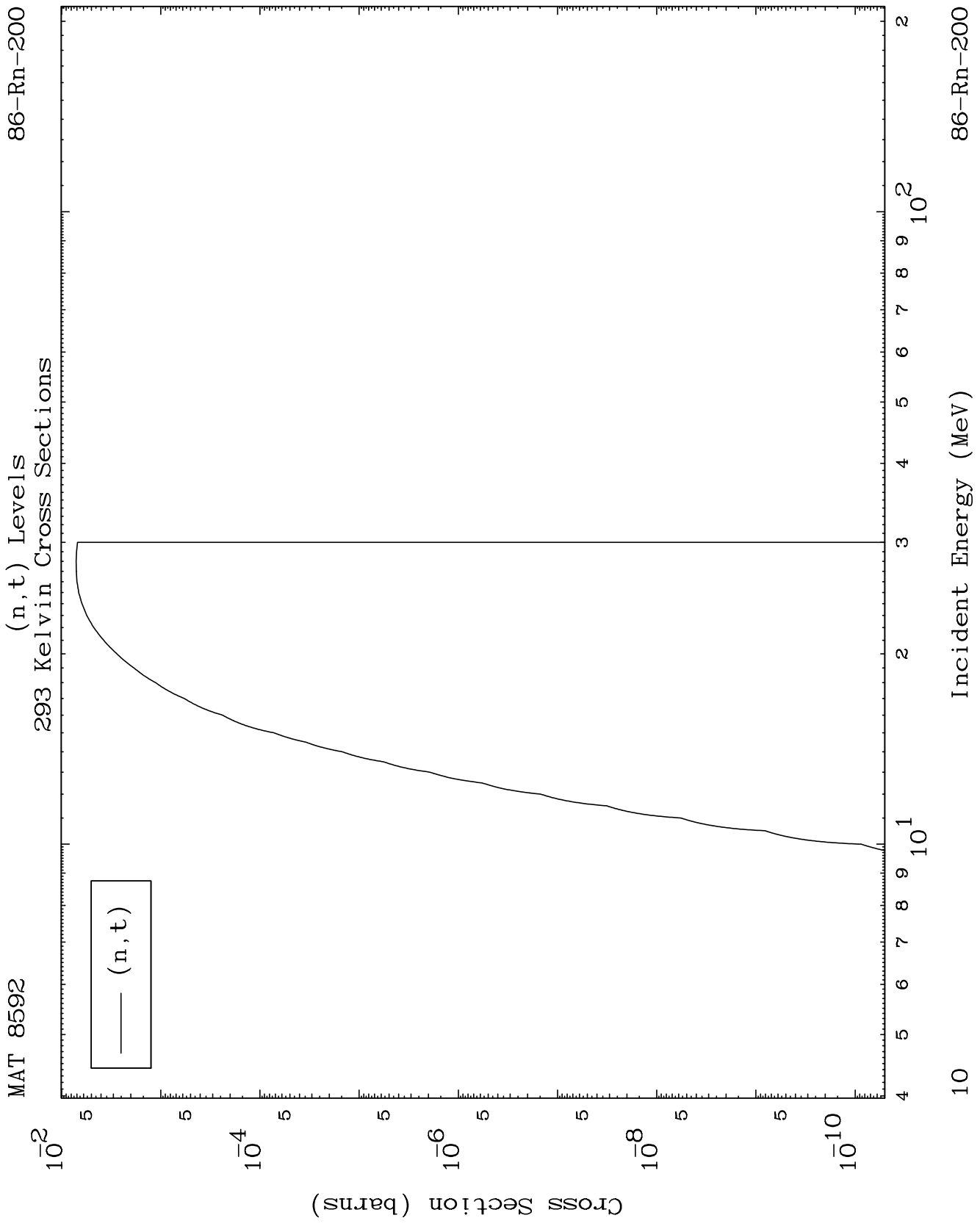


MAT 8592

(n,d) Levels  
293 Kelvin Cross Sections

86-Rn-200

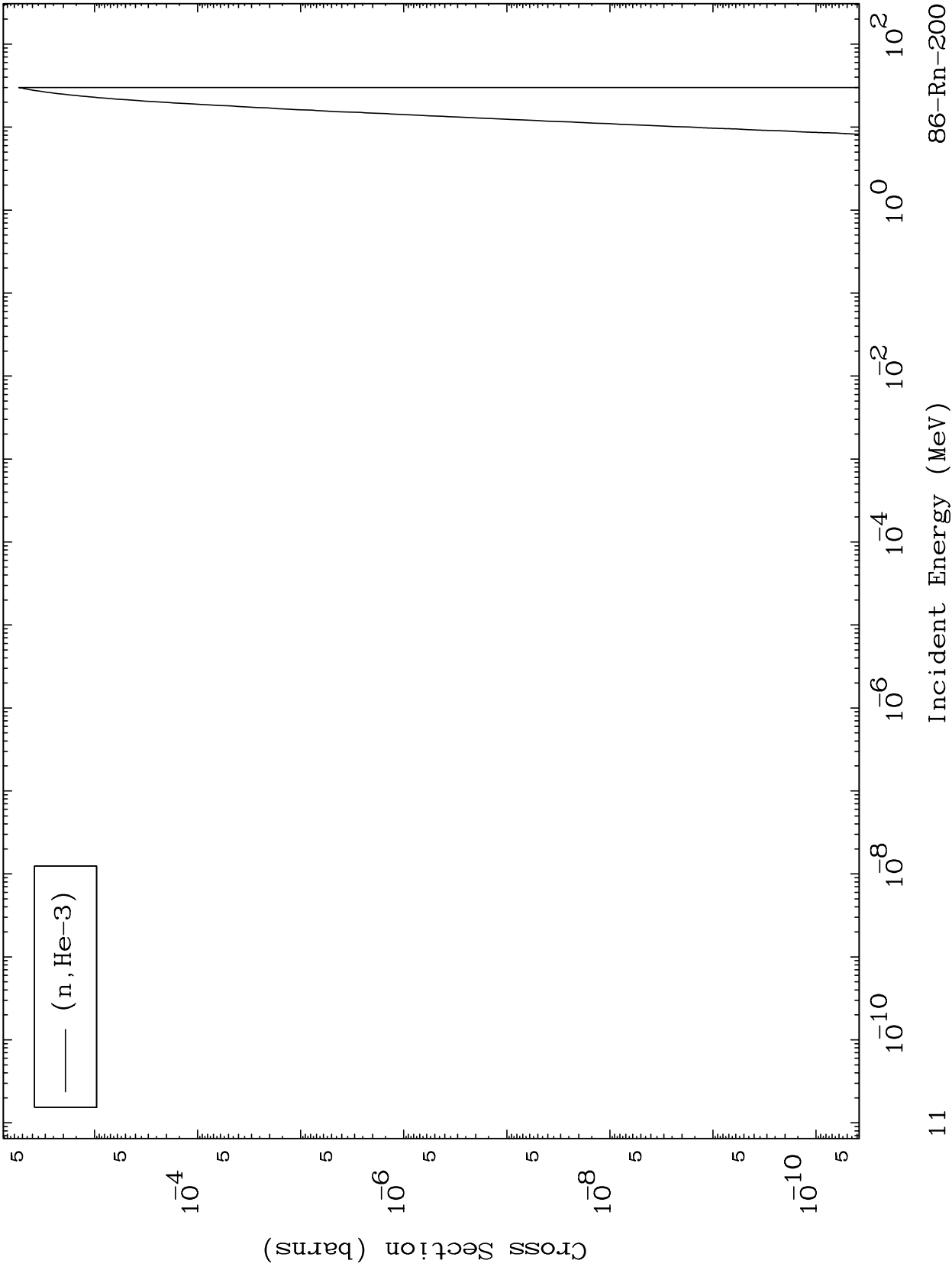


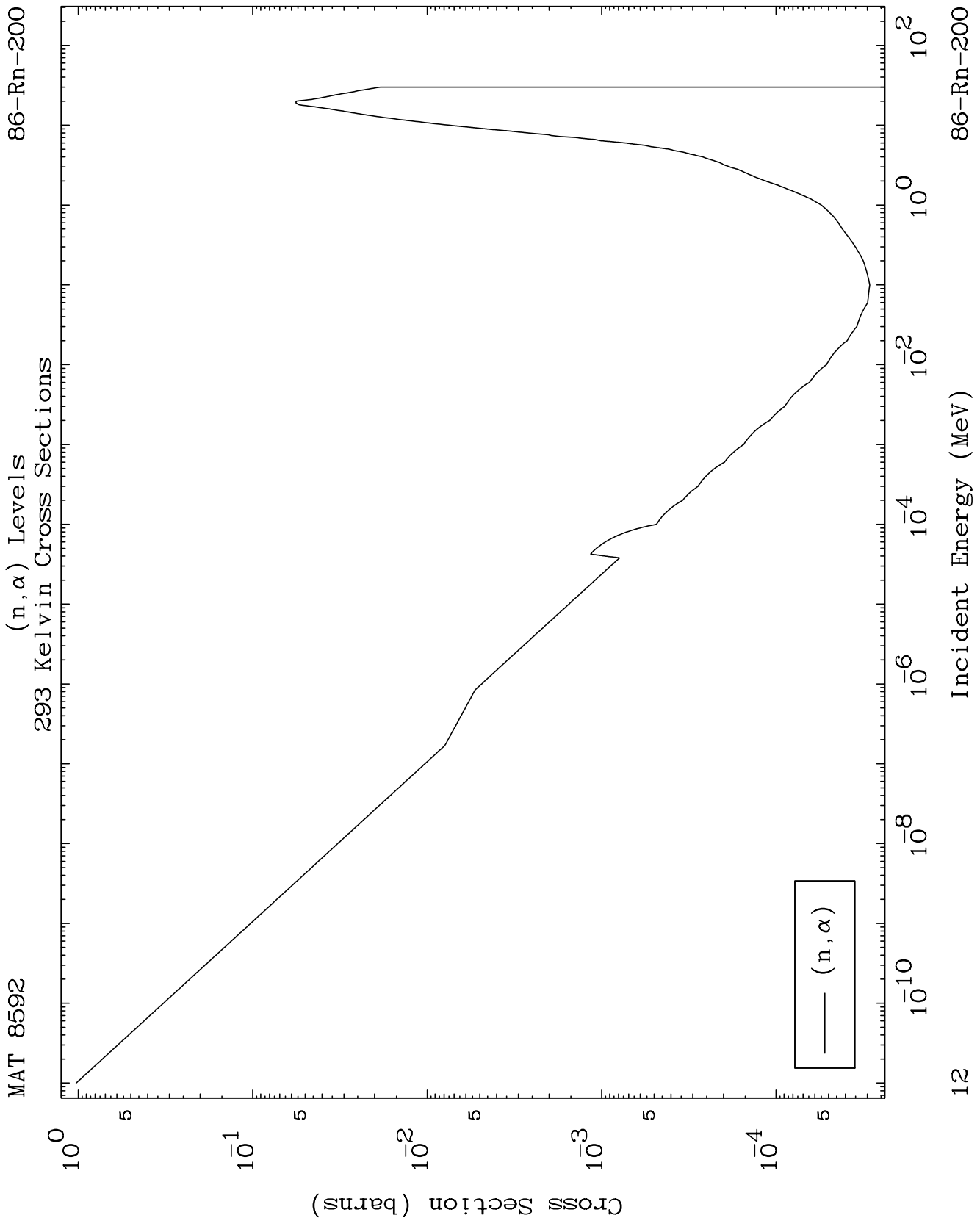


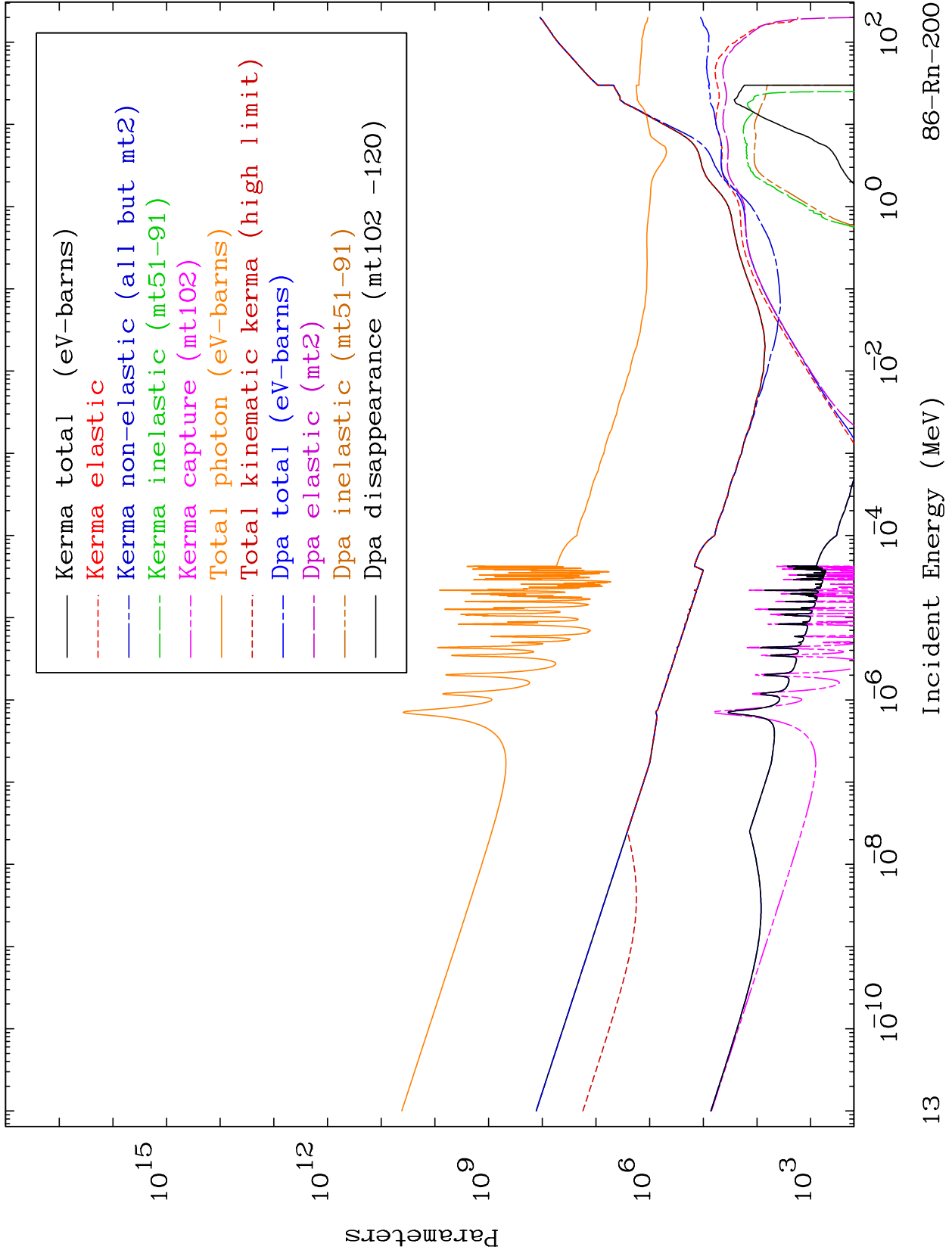
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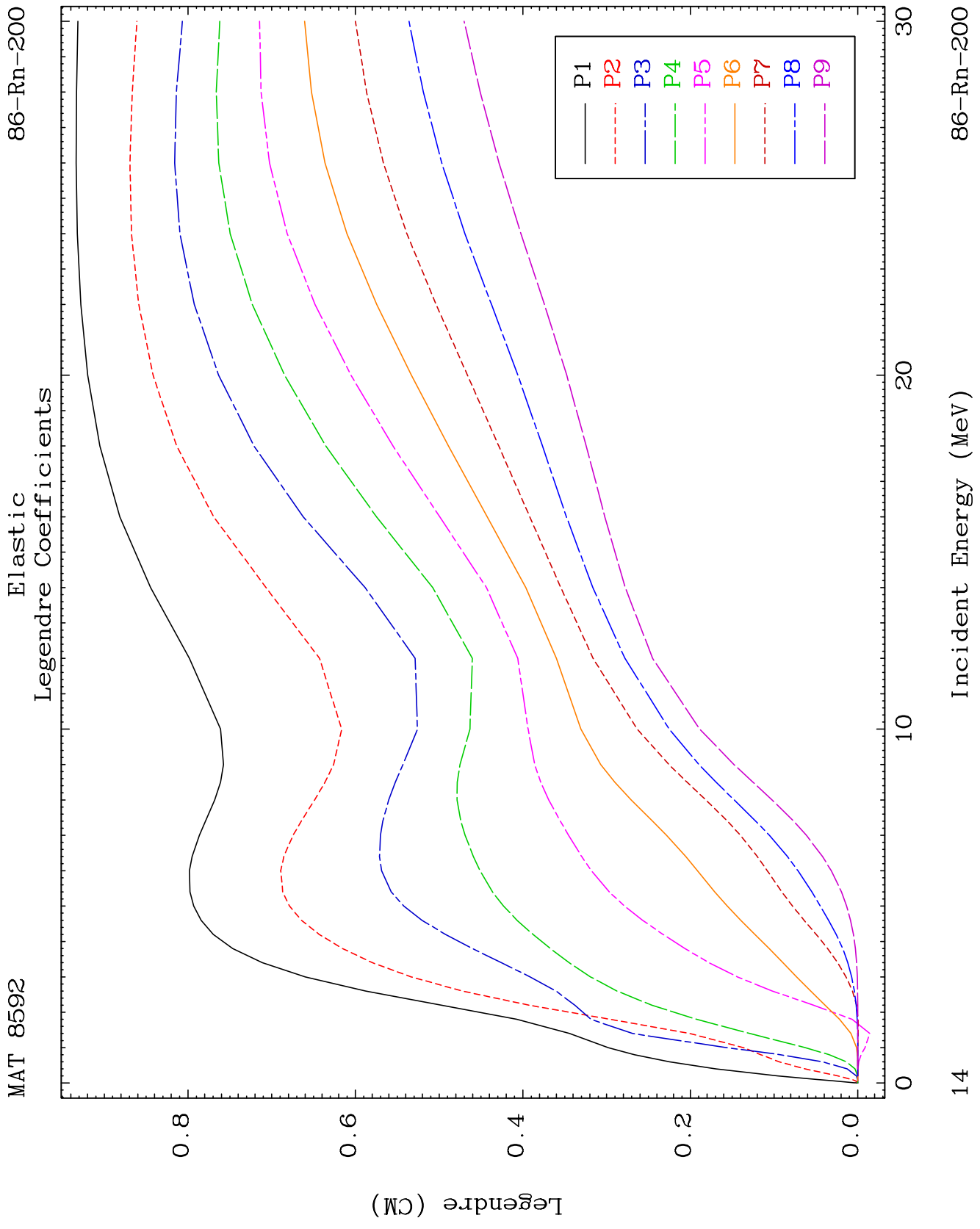
(n,He3) Levels  
293 Kelvin Cross Sections

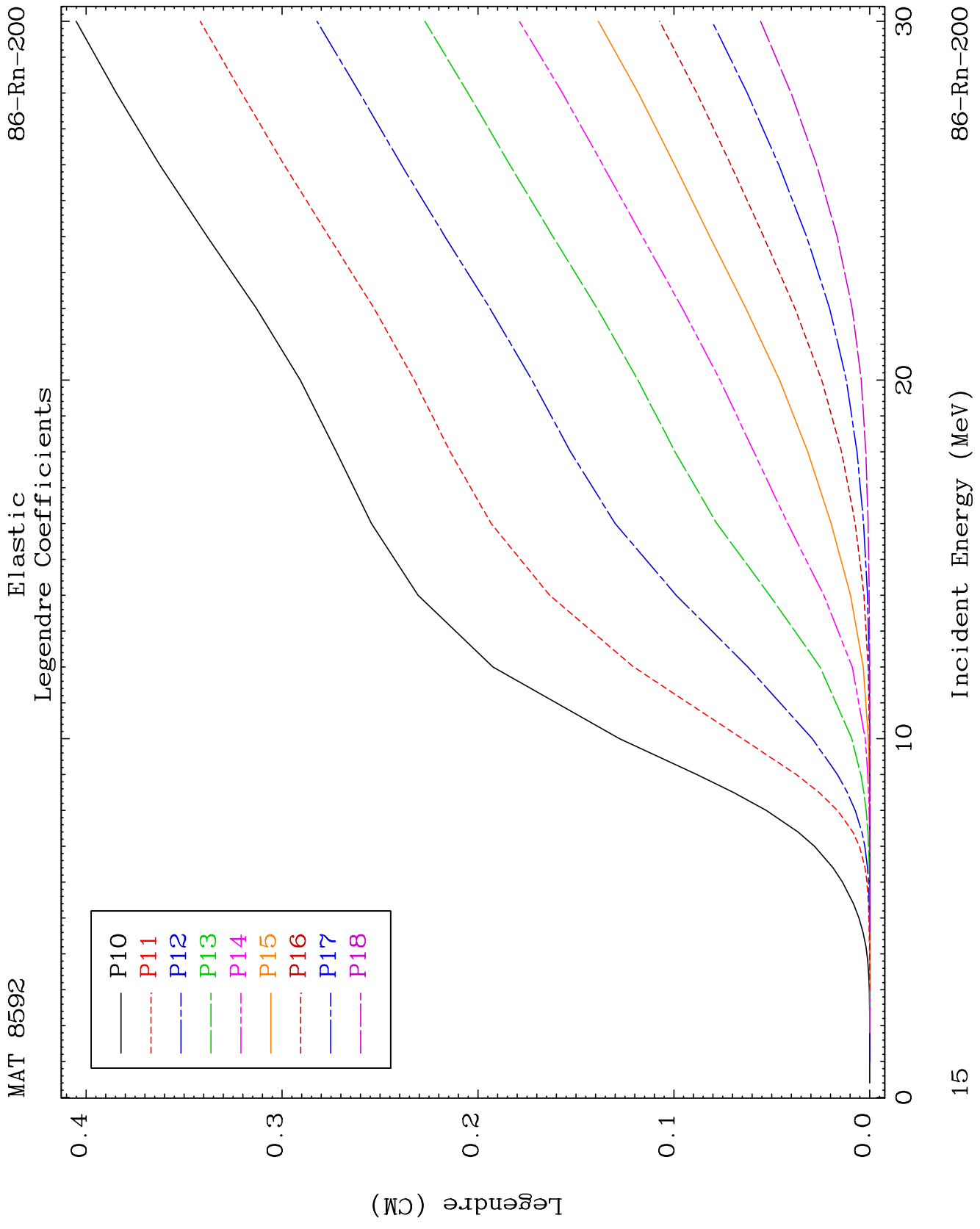
86-Rn-200









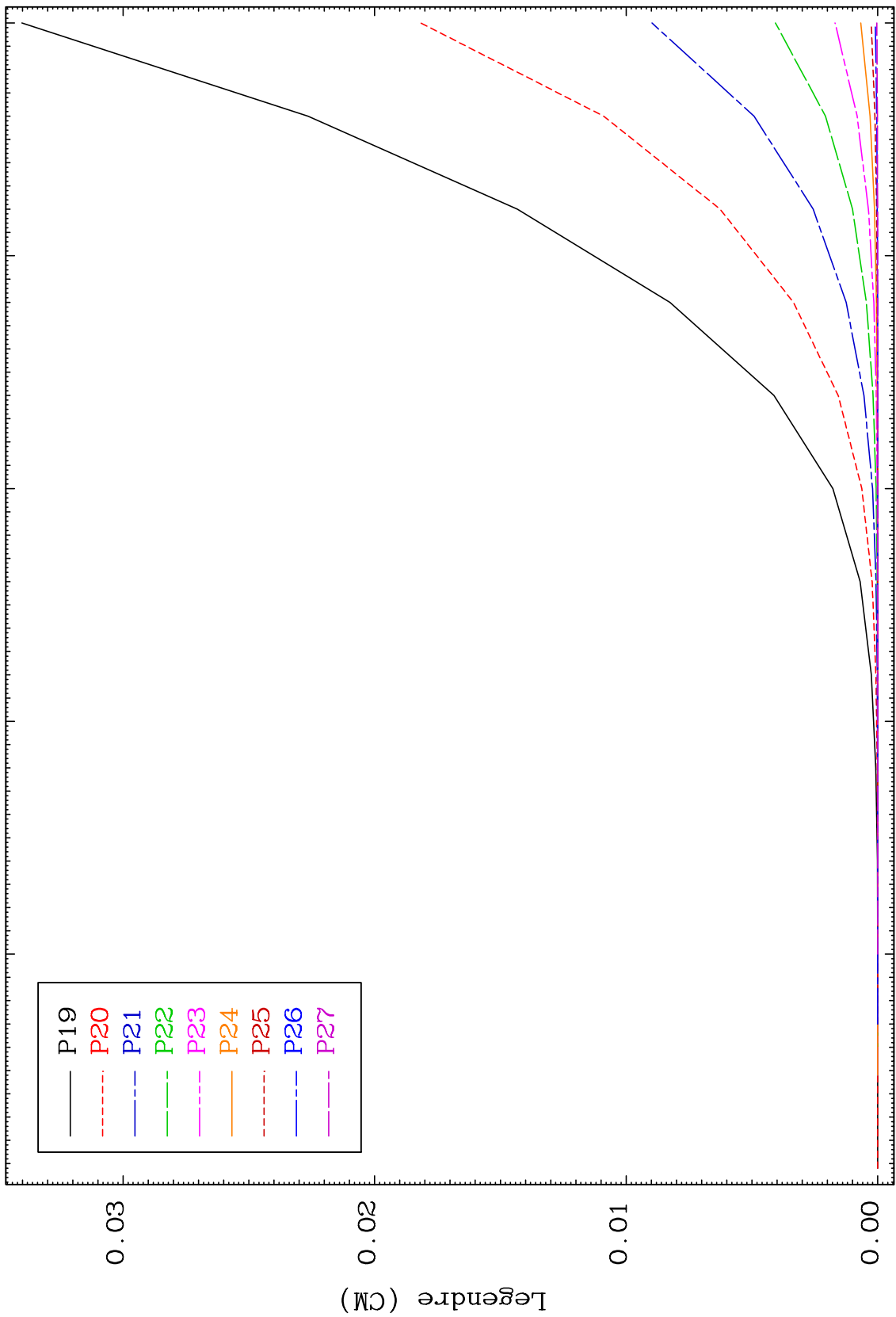




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Elastic Legendre Coefficients

86-Rn-200



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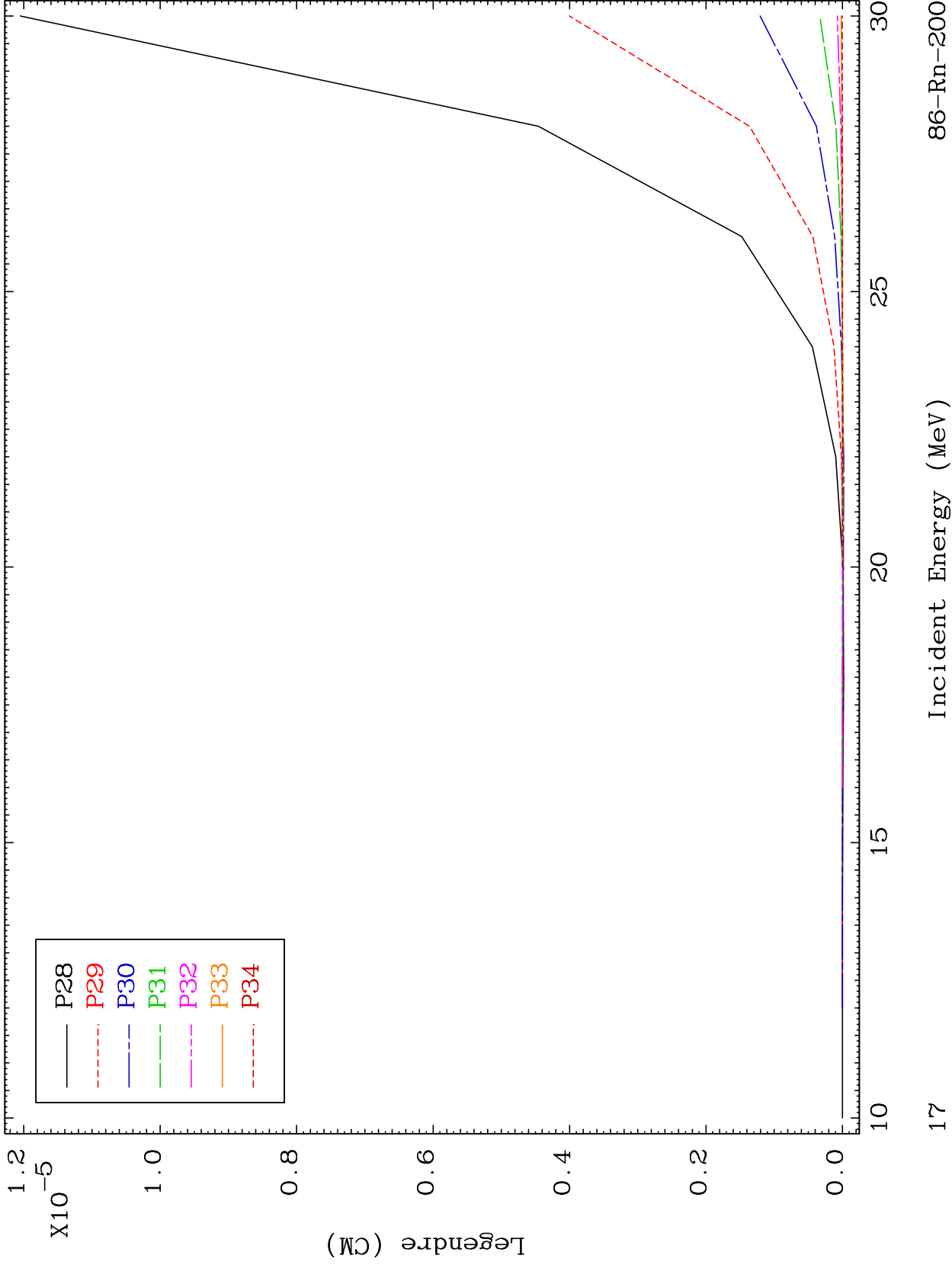
Incident Energy (MeV)

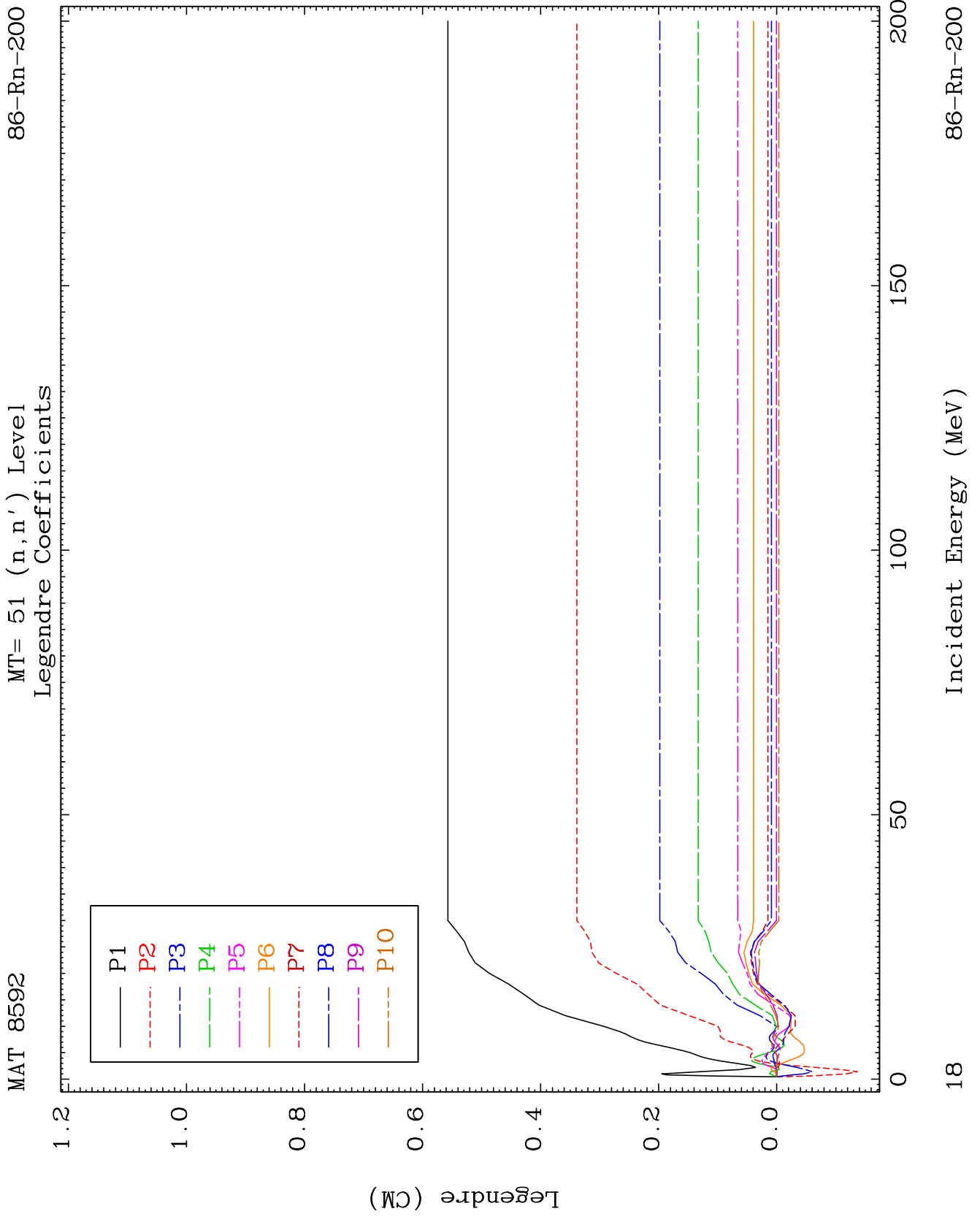
86-Rn-200

MAT 8592

Elastic  
Legendre Coefficients

86-Rn-200

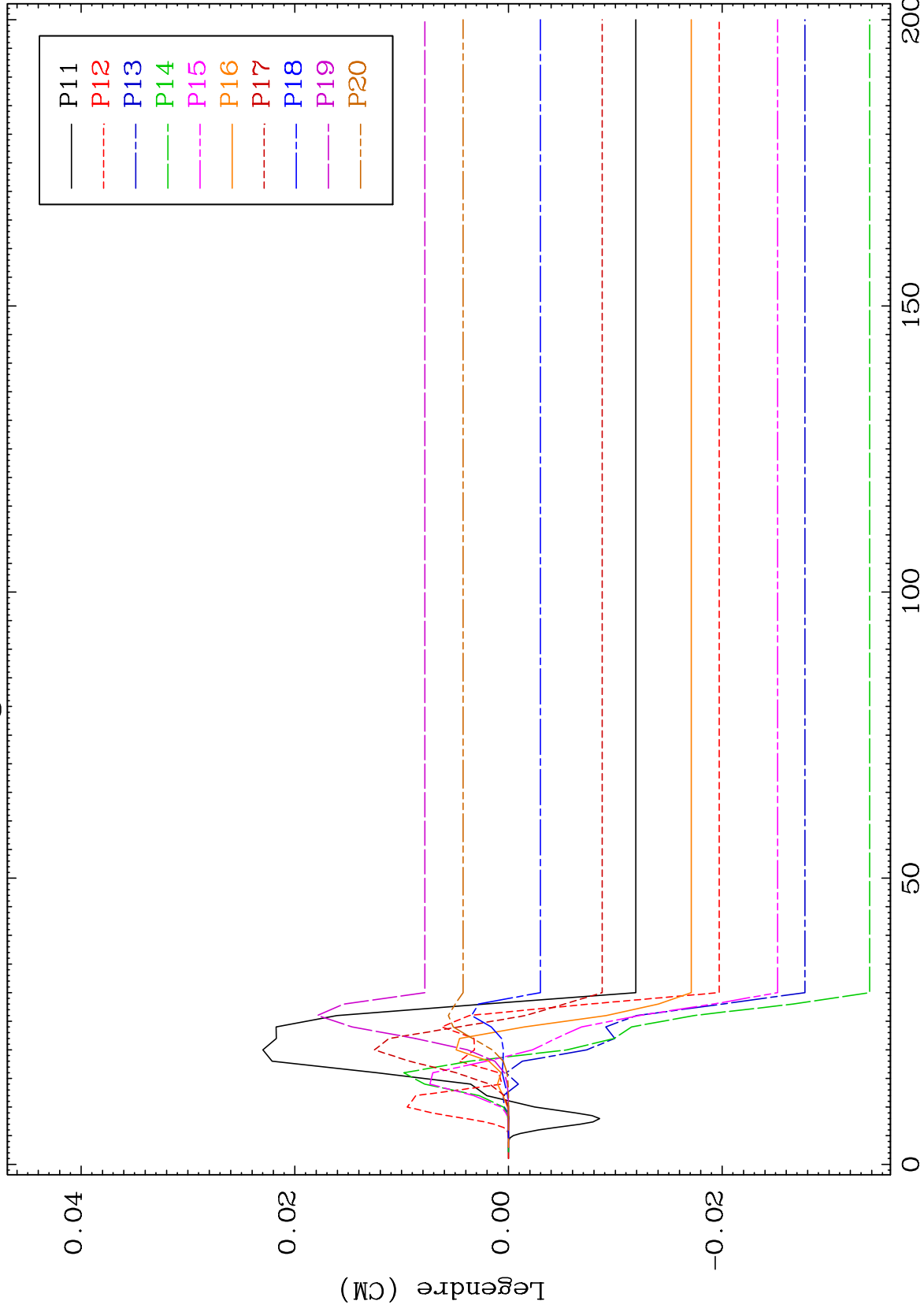




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MT= 51 (n,n') Level  
Legendre Coefficients

86-Rn-200



19

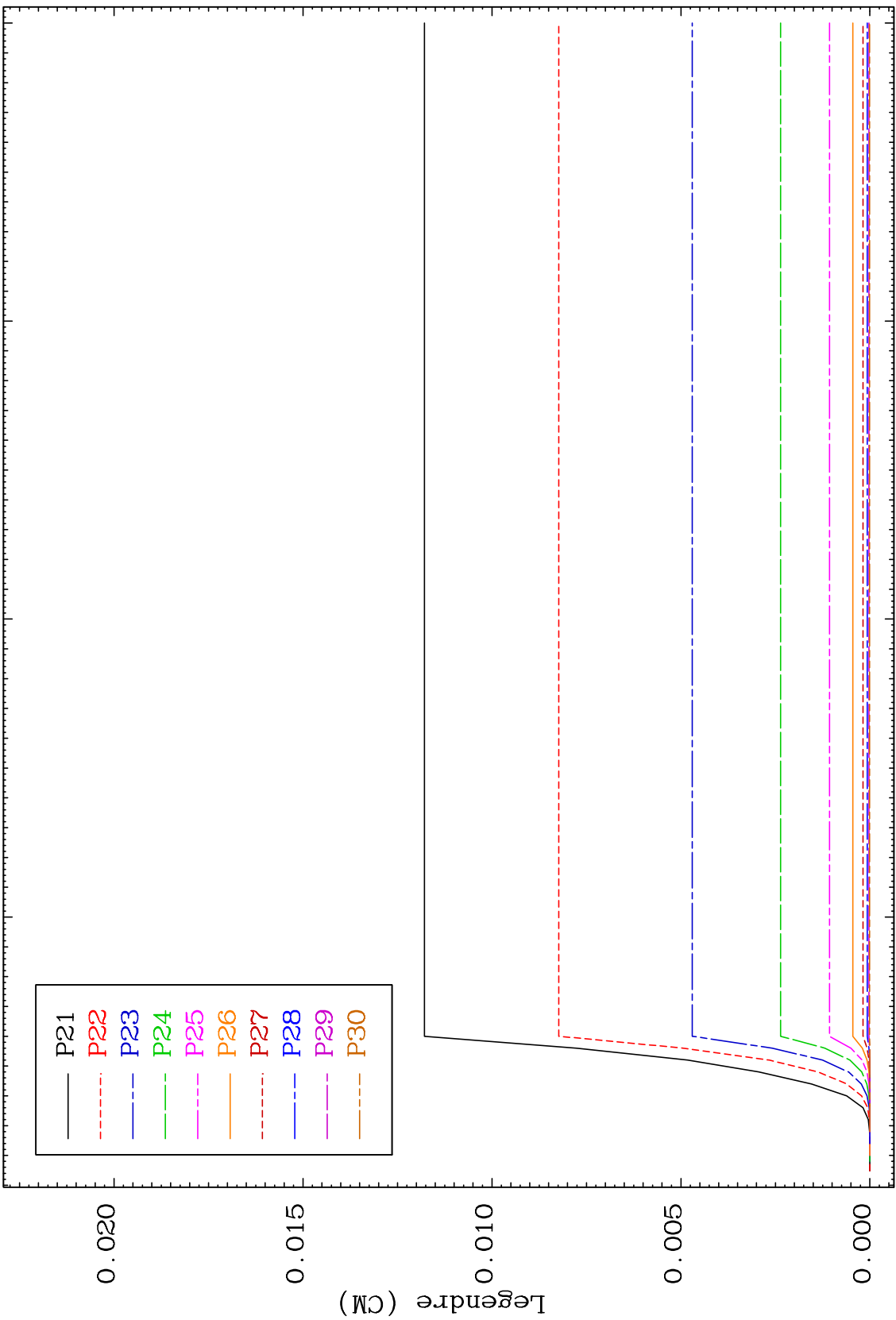
Incident Energy (MeV)

86-Rn-200

MAT 8592

MT= 51 (n,n') Level  
Legendre Coefficients

86-Rn-200



20

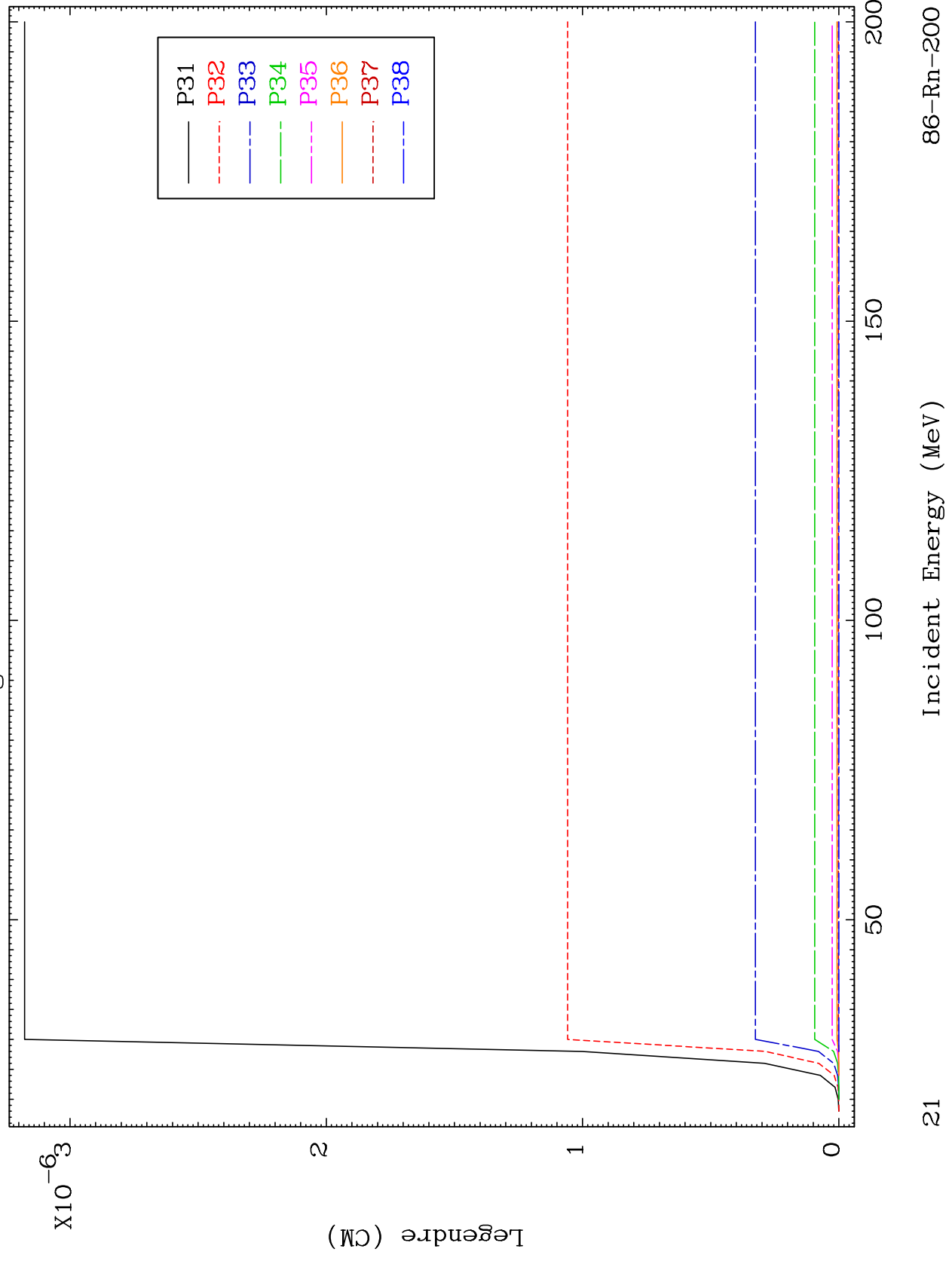
Incident Energy (MeV)

86-Rn-200

MAT 8592

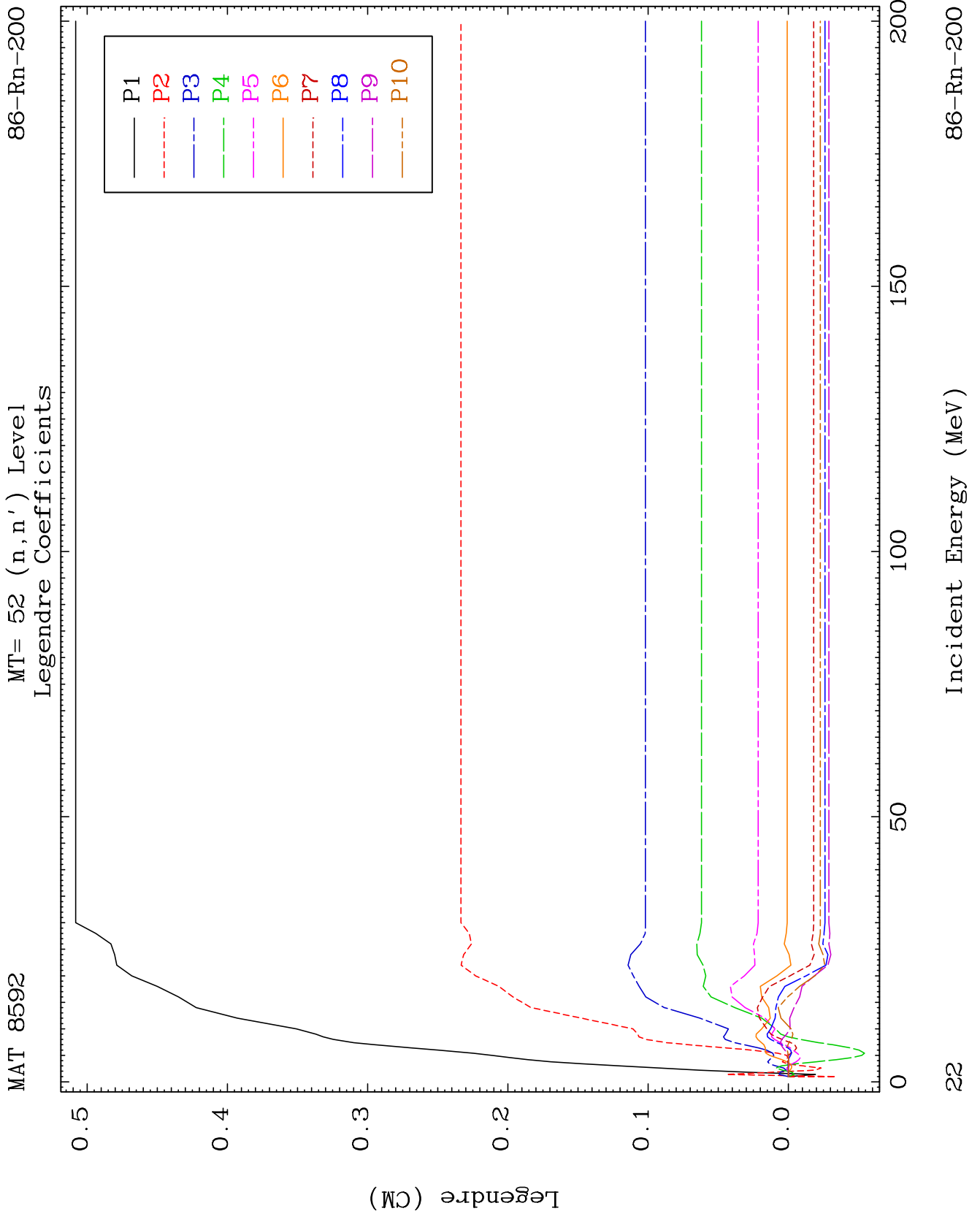
MT= 51 (n,n') Level  
Legendre Coefficients

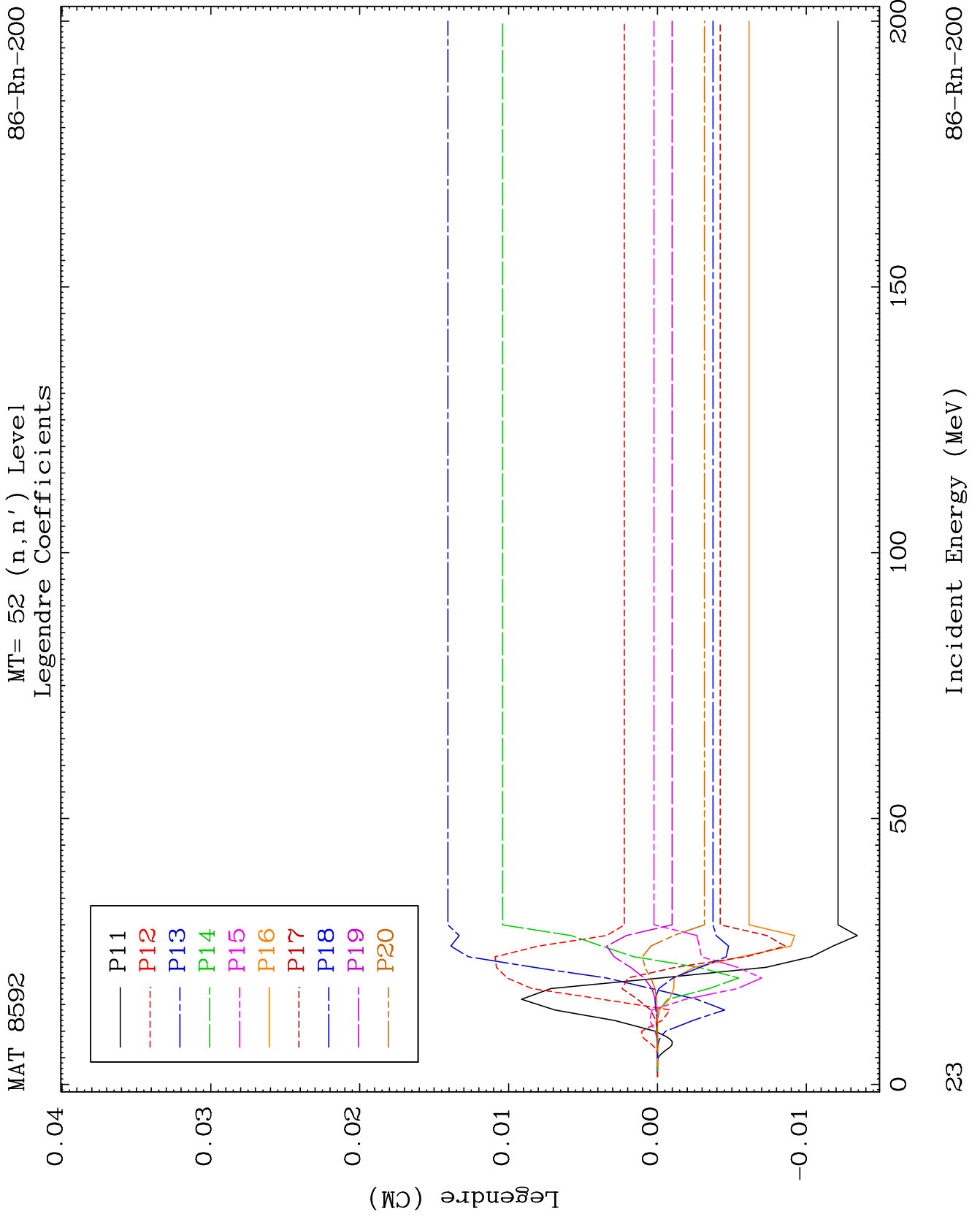
86-Rn-200



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86-Rn-200



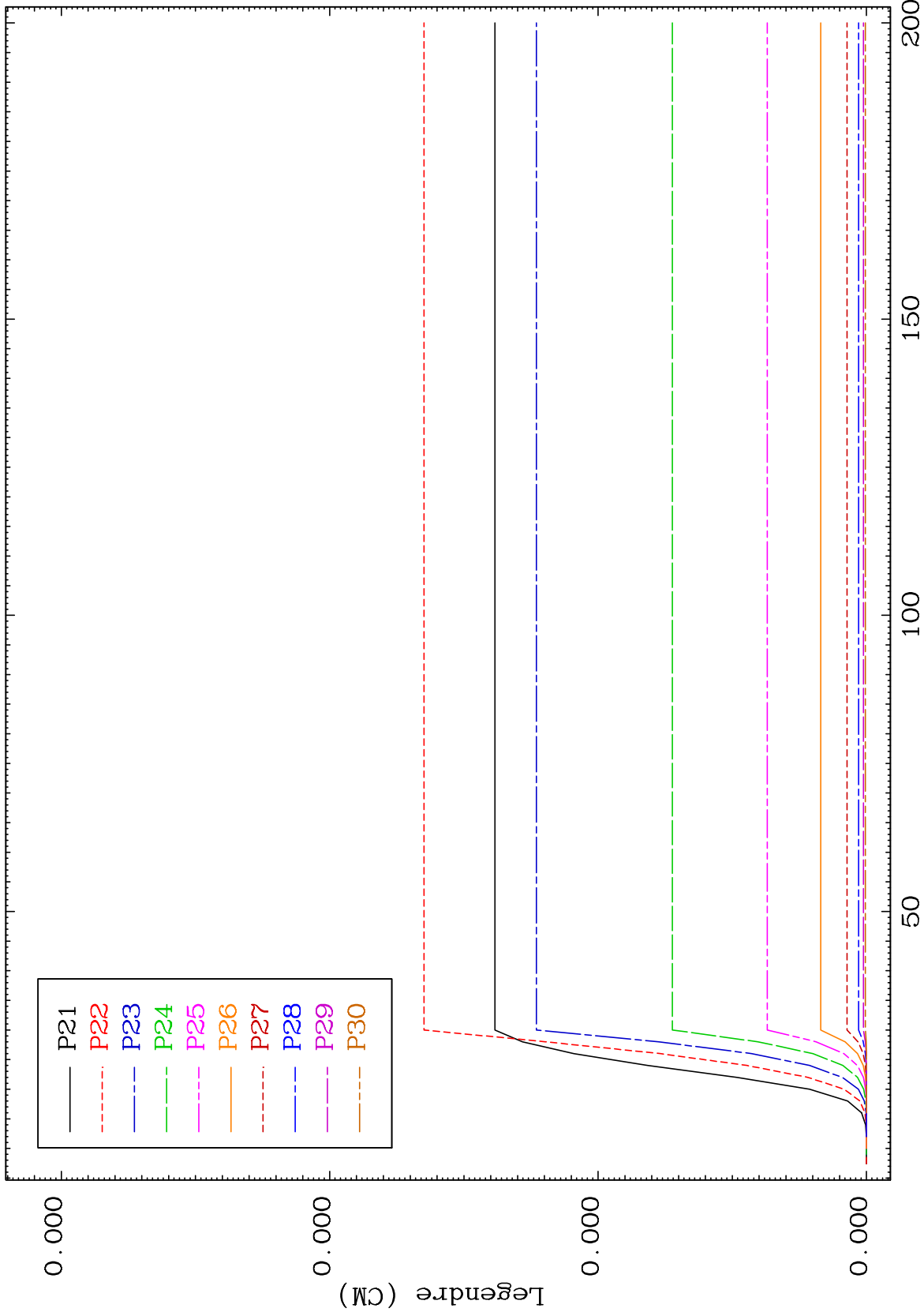




MAT 8592

MT= 52 (n,n') Level  
Legendre Coefficients

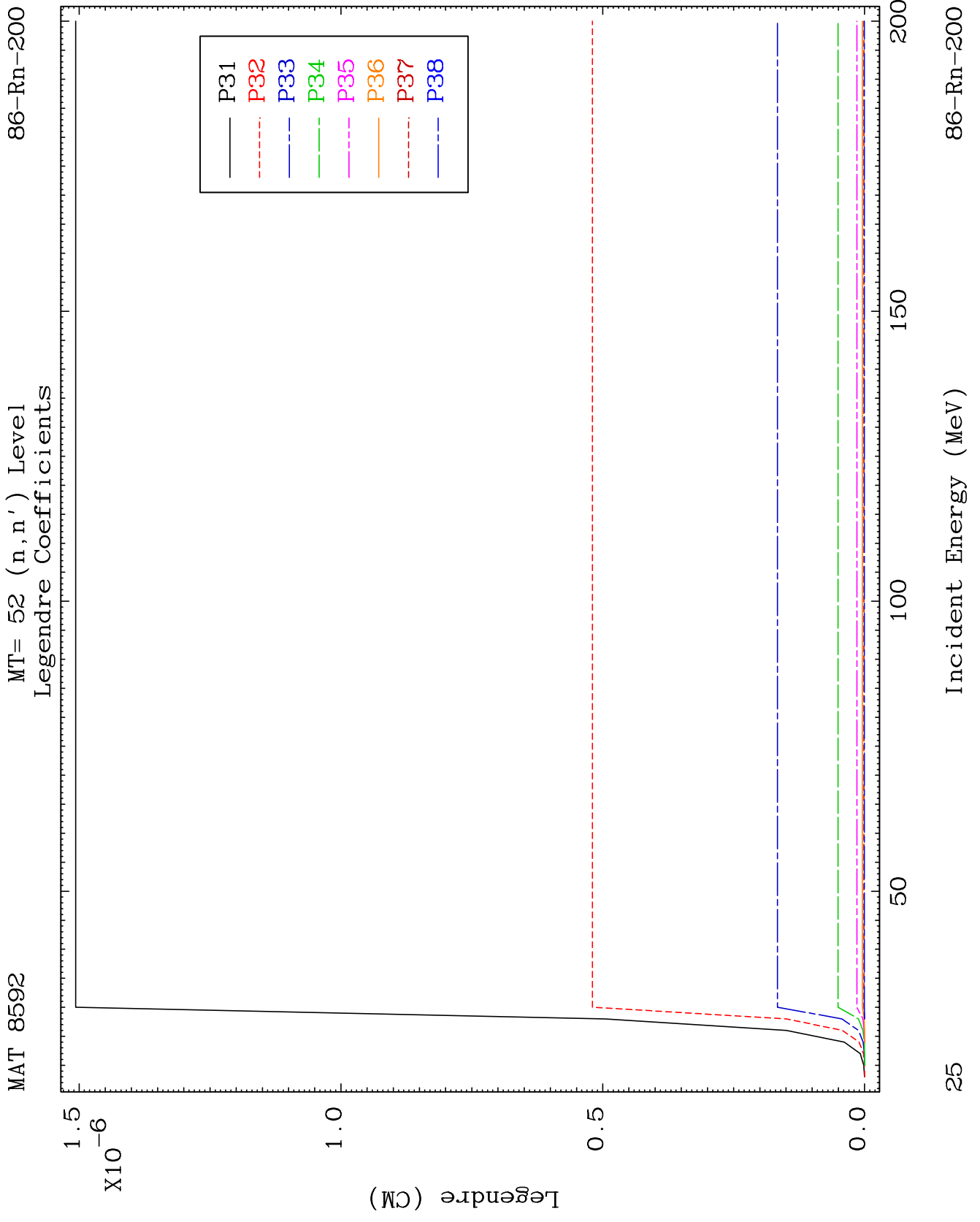
86-Rn-200



24

Incident Energy (MeV)

86-Rn-200

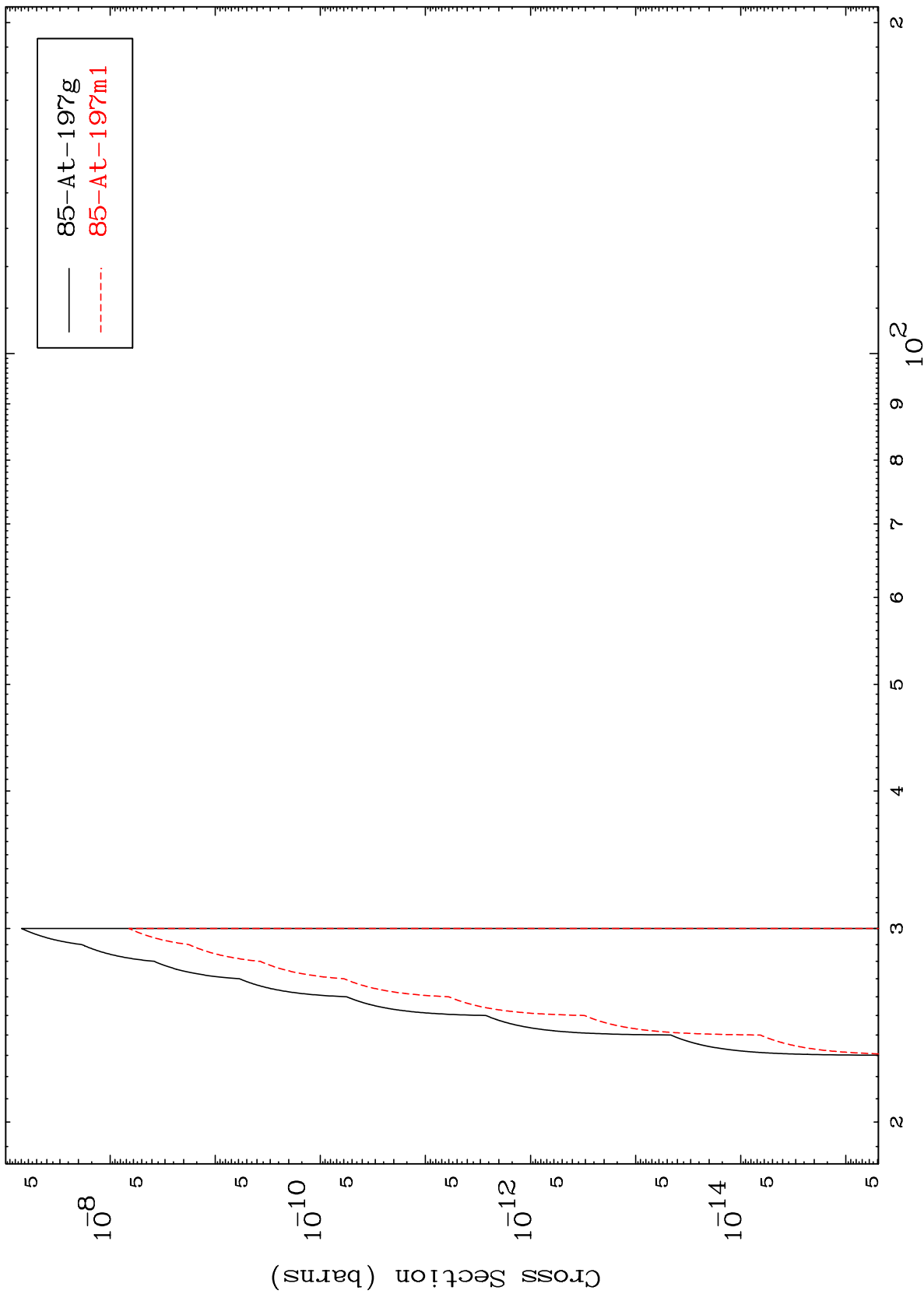


MAT 8592

(n,2n) d

86-Rn-200

Radionuclide Production Cross Section



26

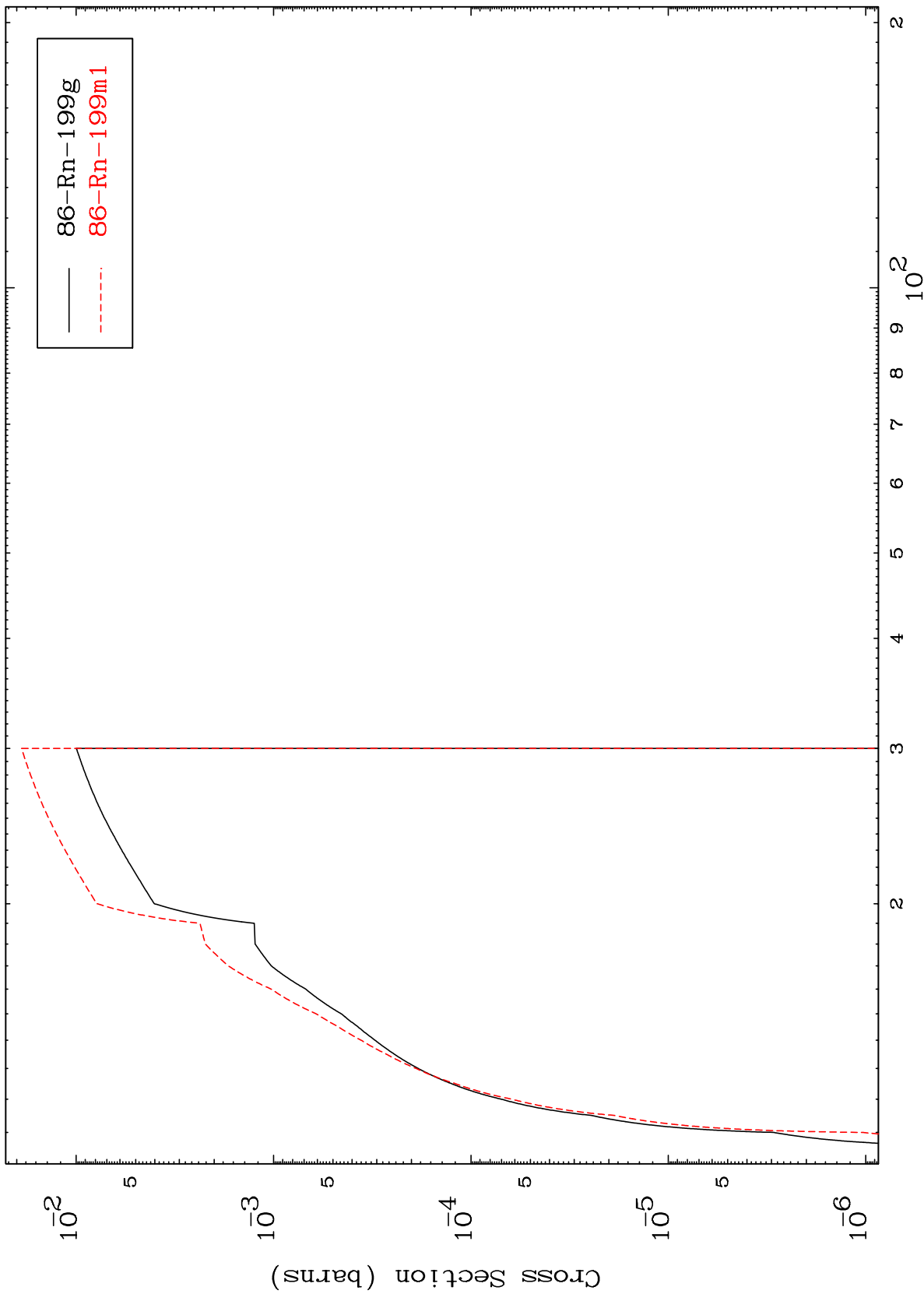
Incident Energy (MeV)

86-Rn-200

MAT 8592

86-Rn-200

(n,2n)  
Radionuclide Production Cross Section



27

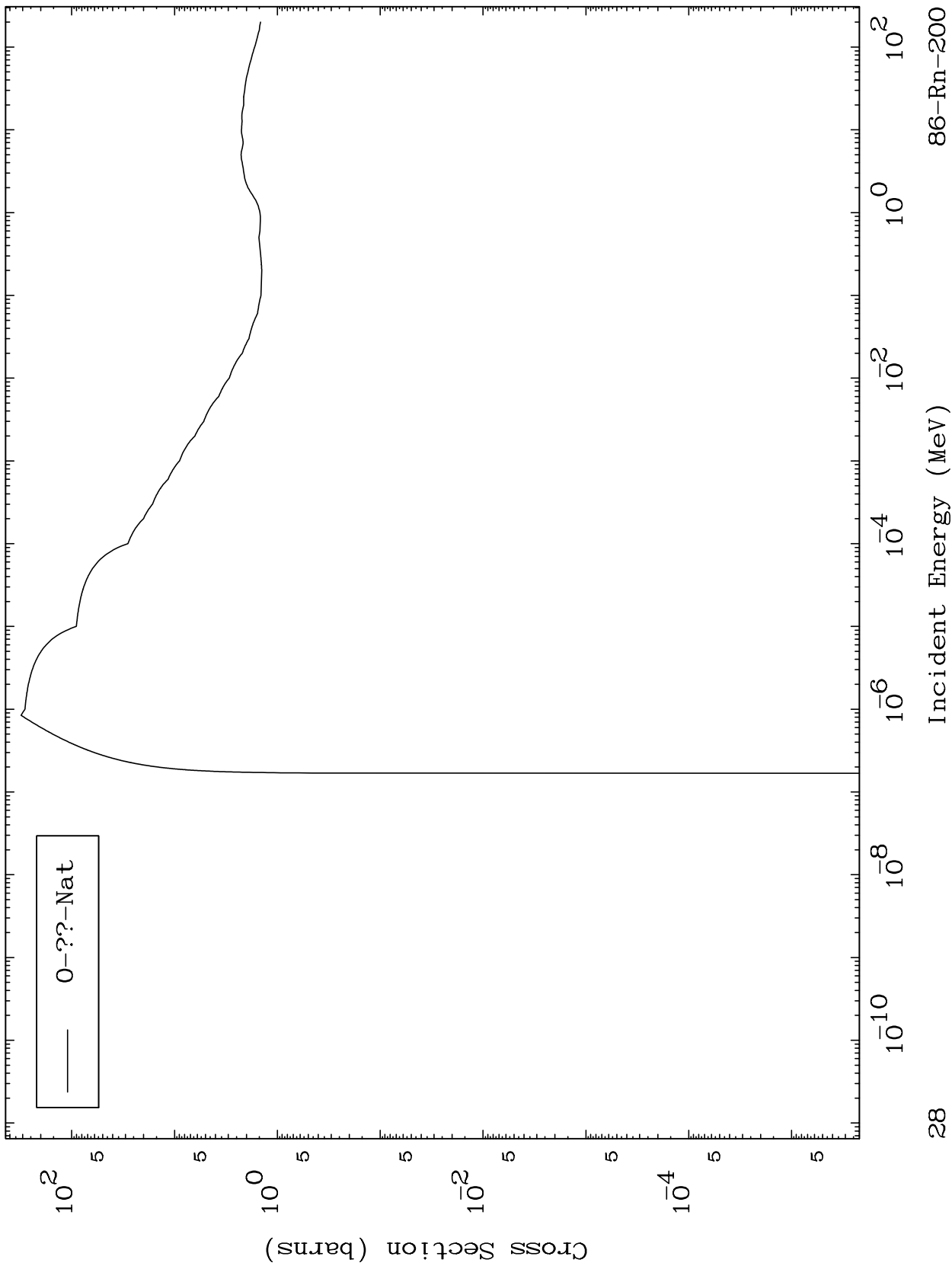
Incident Energy (MeV)

86-Rn-200

MAT 8592

86-Rn-200

Fission  
Radionuclide Production Cross Section

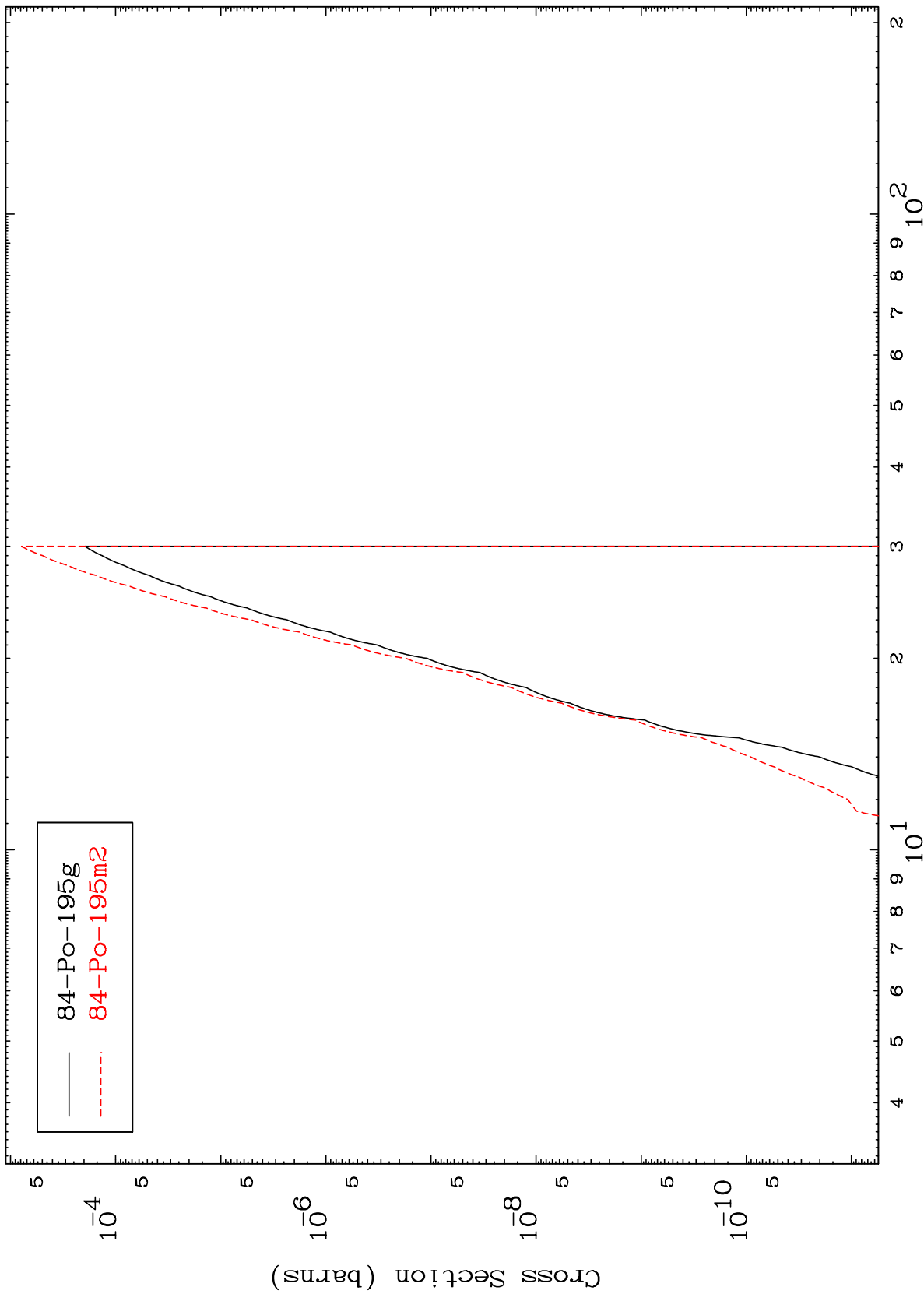


MAT 8592

$(n,2n) \alpha$

86-Rn-200

Radionuclide Production Cross Section

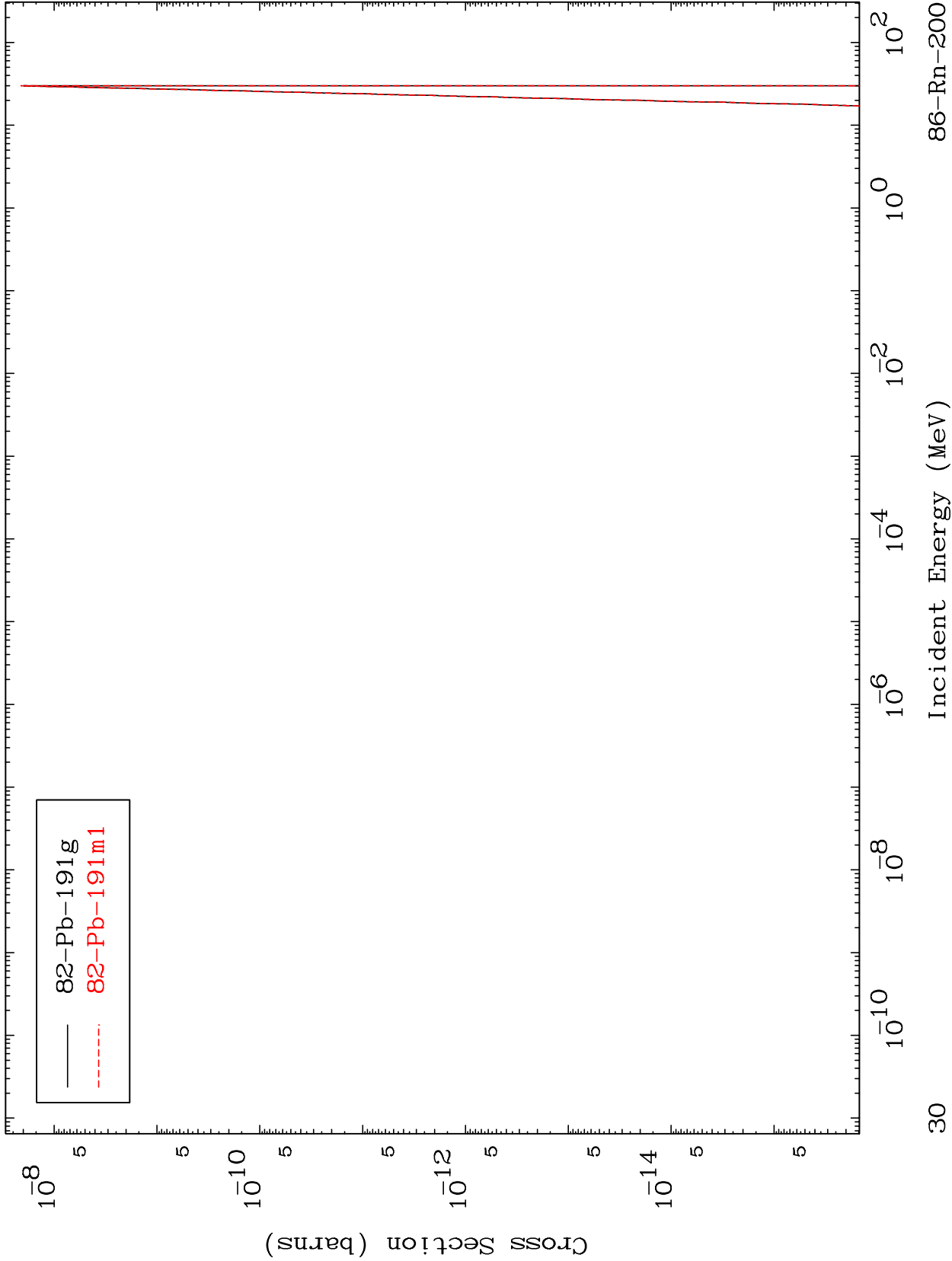


MAT 8592

$(n,2n) 2\alpha$

86-Rn-200

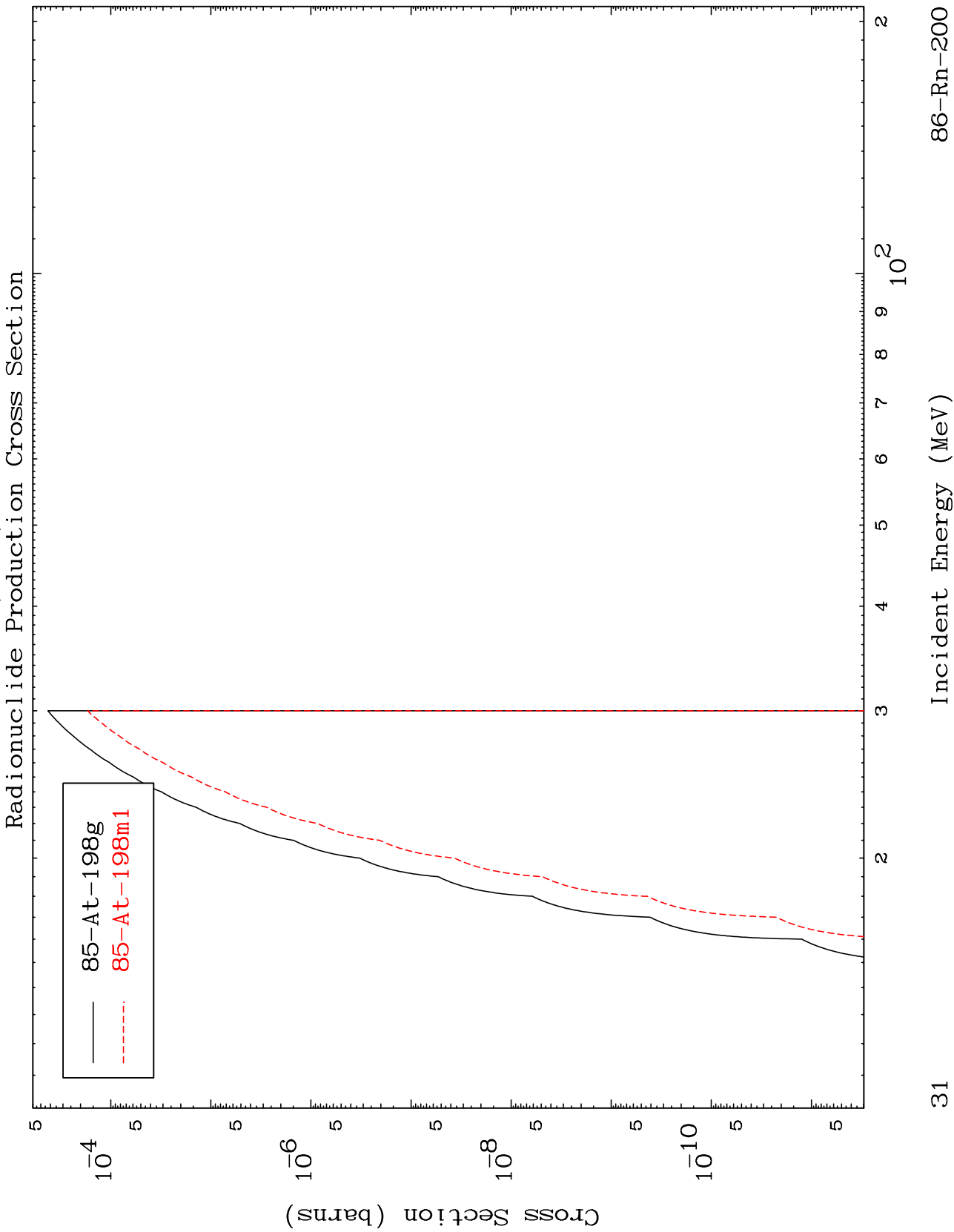
Radionuclide Production Cross Section



MAT 8592

(n,n') d

86-Rn-200

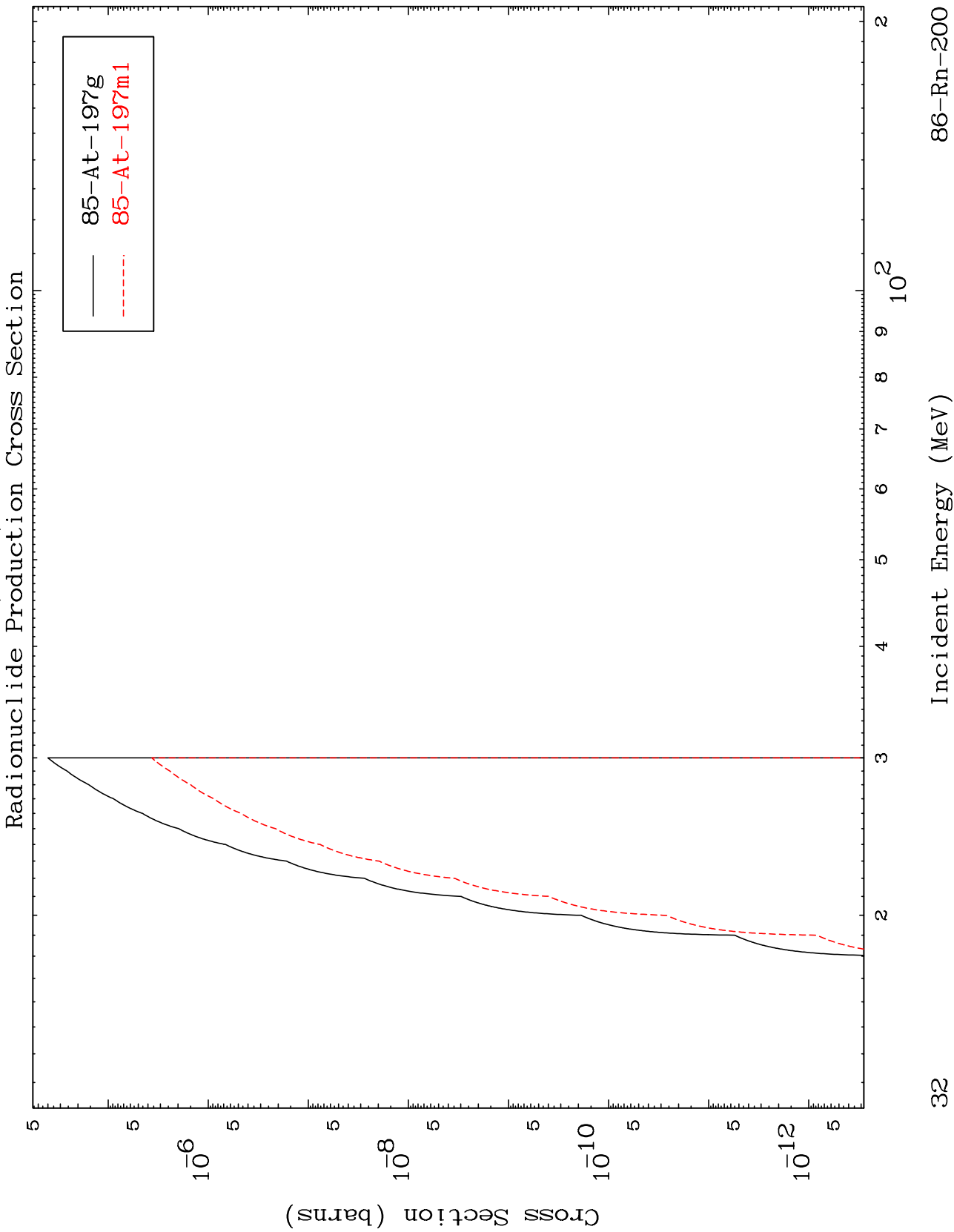




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(n,n') t

86-Rn-200



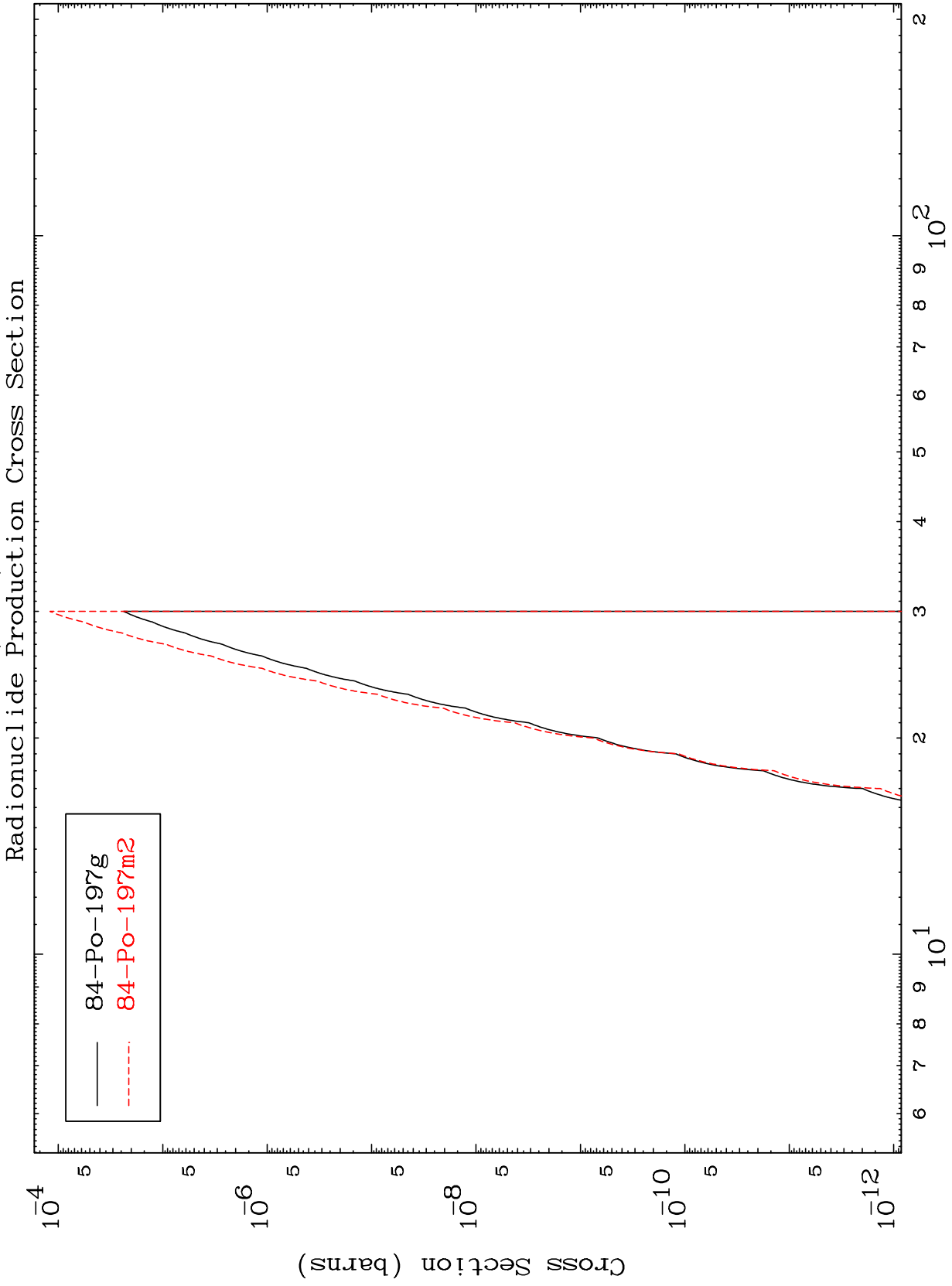
32

86-Rn-200

MAT 8592

(n,n') He-3

86-Rn-200



33

Incident Energy (MeV)

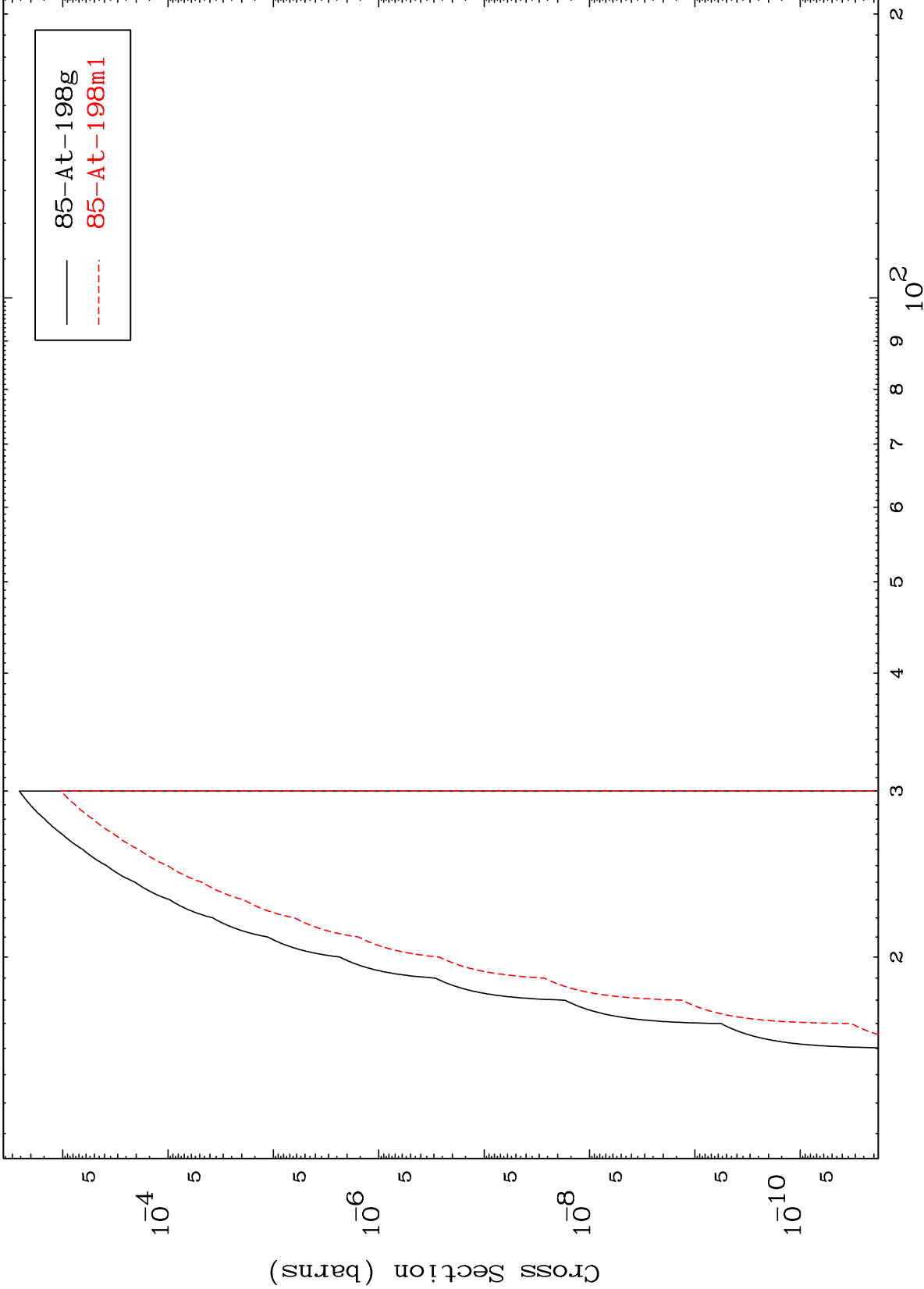
86-Rn-200

MAT 8592

(n,2n) p

86-Rn-200

Radionuclide Production Cross Section



34

Incident Energy (MeV)

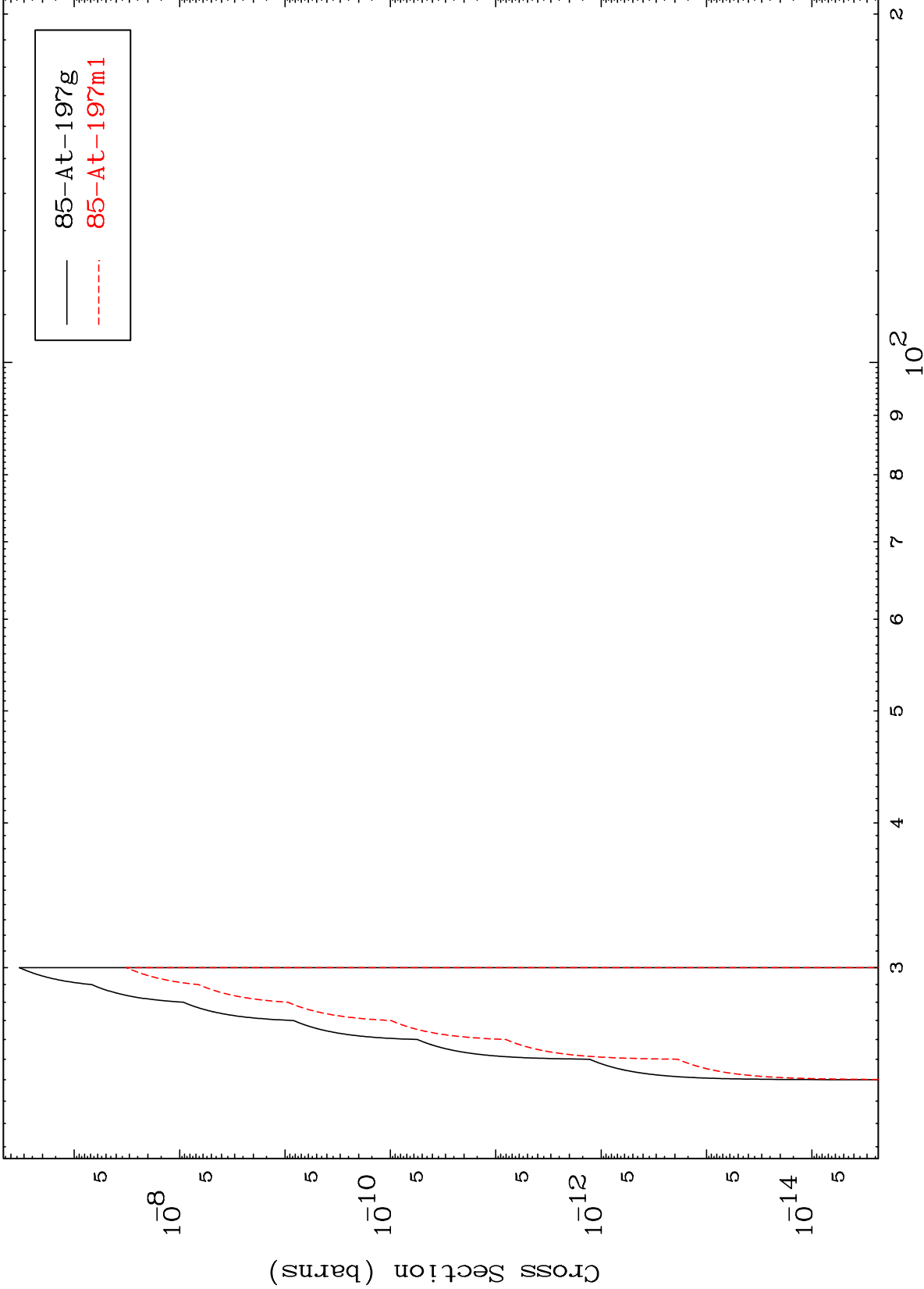
86-Rn-200

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(n,3n) p

86-Rn-200

Radionuclide Production Cross Section



35

Incident Energy (MeV)

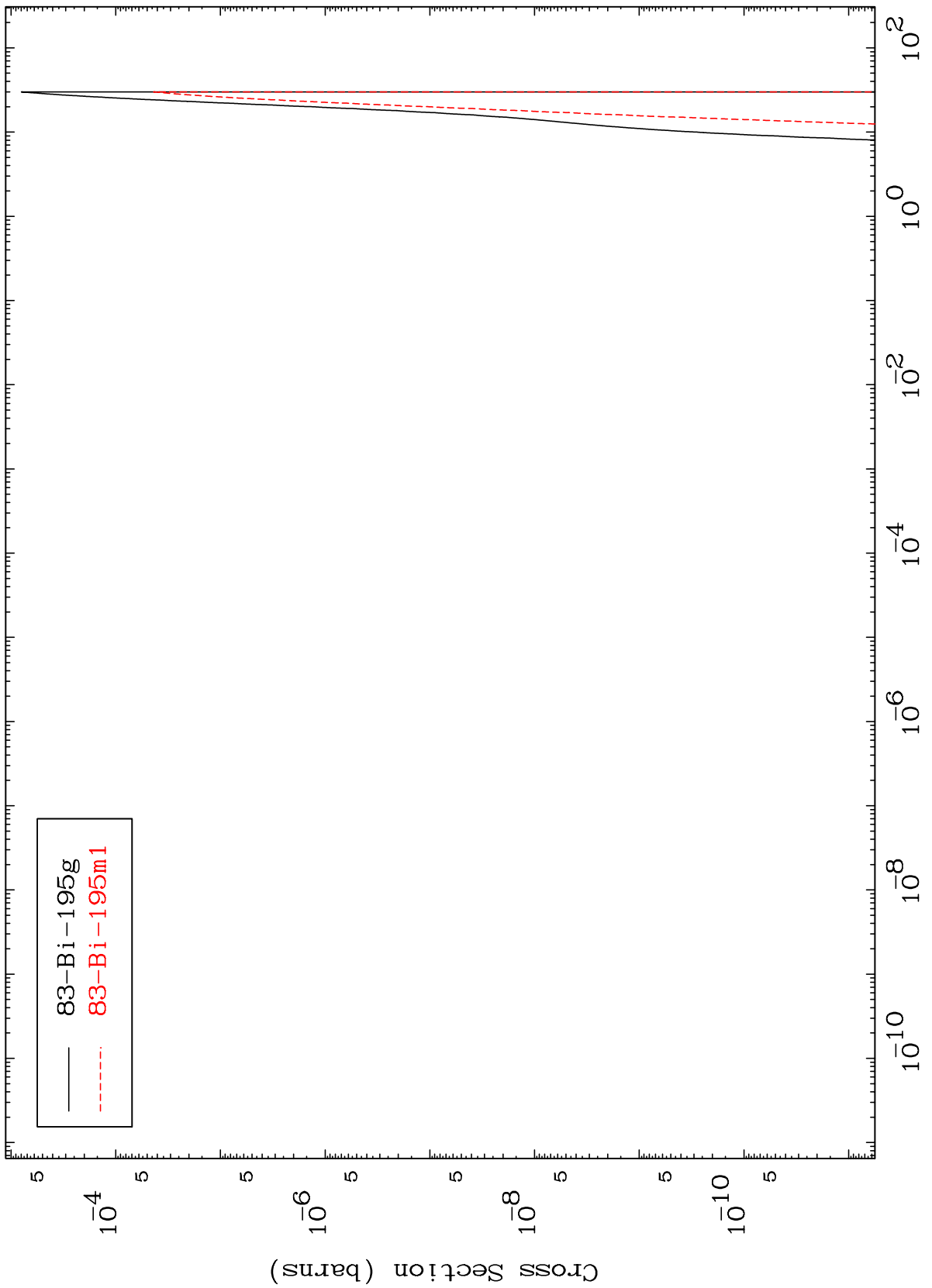
86-Rn-200

MAT 8592

(n,n') p  $\alpha$

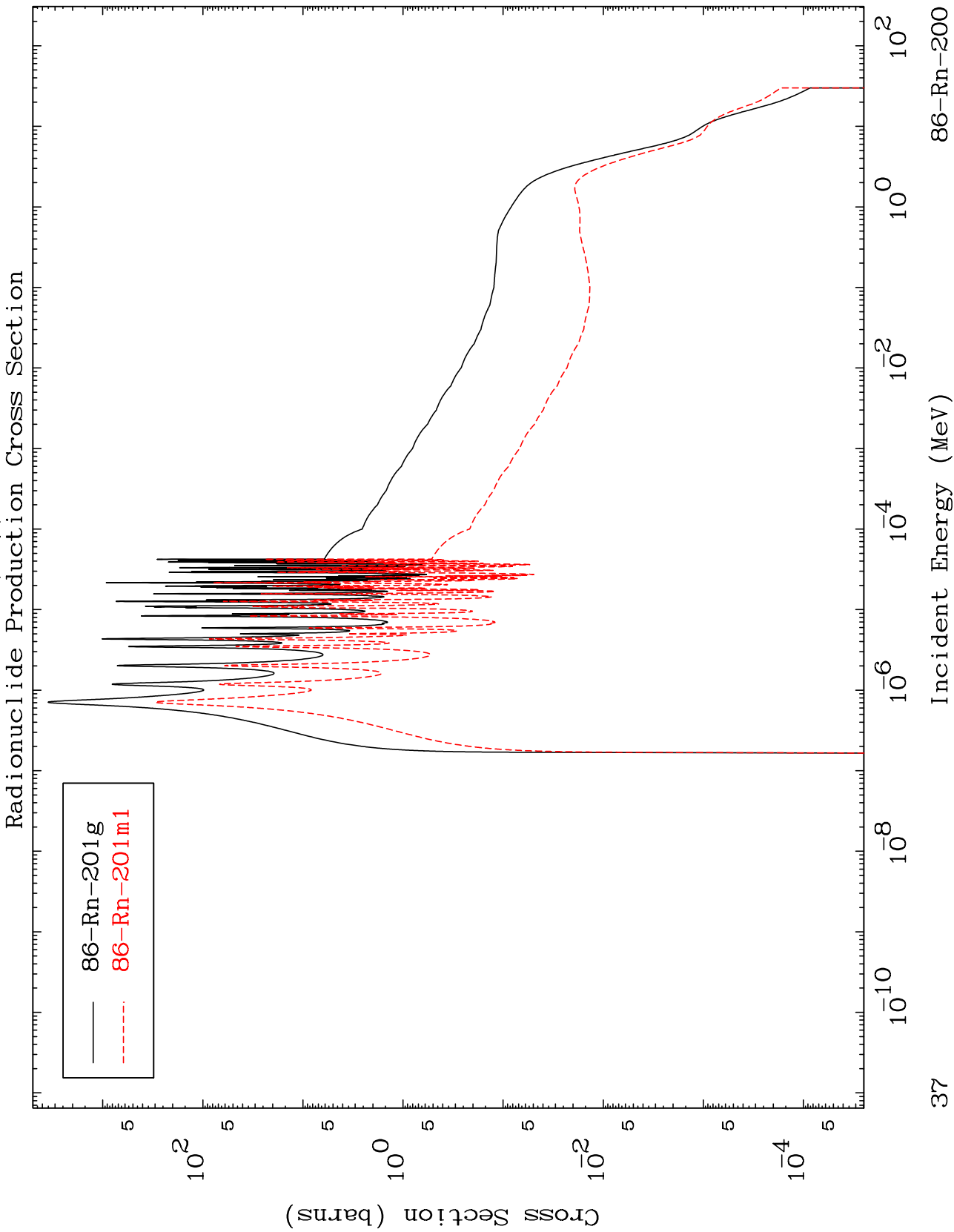
86-Rn-200

Radionuclide Production Cross Section



MAT 8592

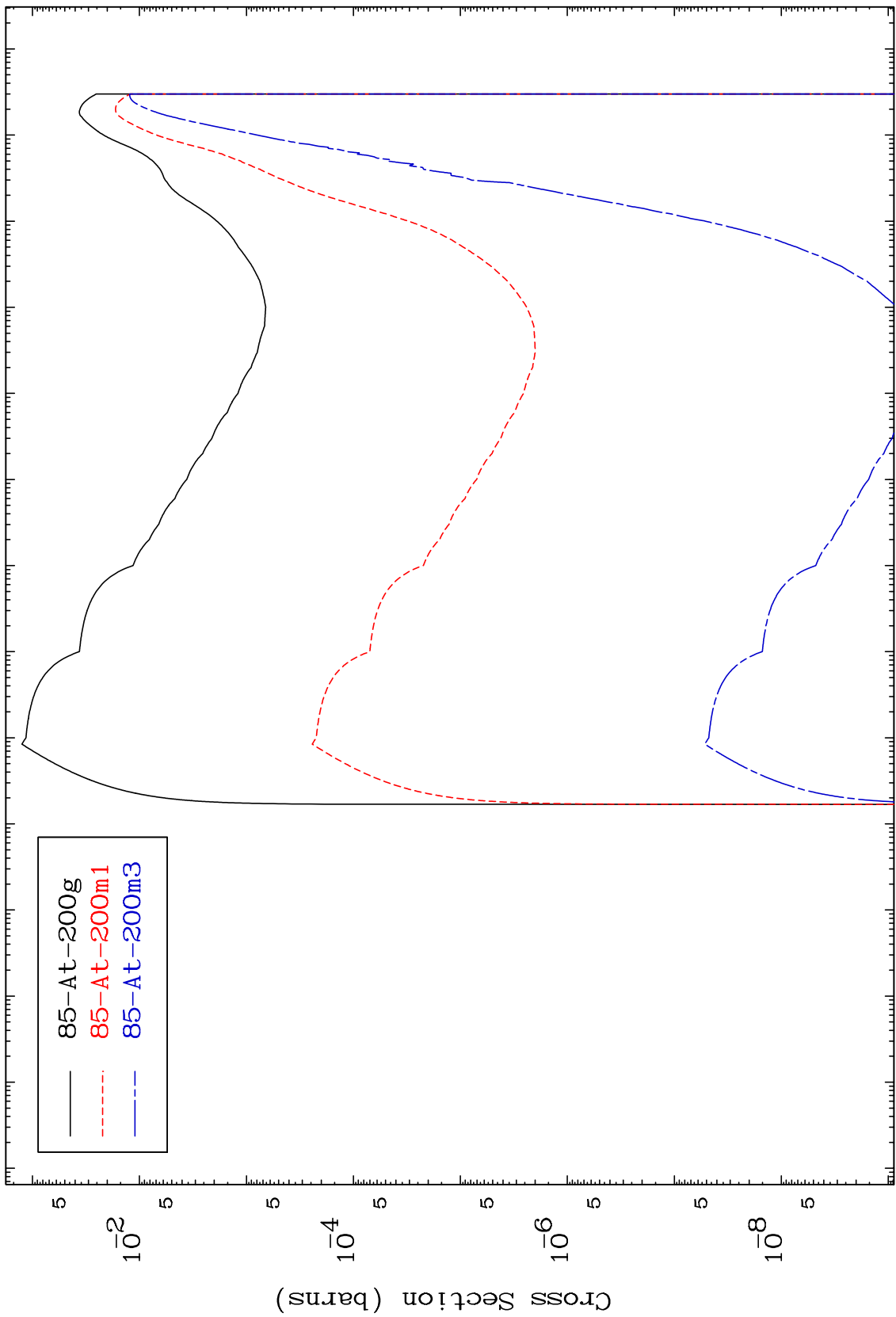
86-Rn-200



MAT 8592

86-Rn-200

(n,p)  
Radionuclide Production Cross Section



85-At-200g  
85-At-200m1  
85-At-200m3

86-Rn-200

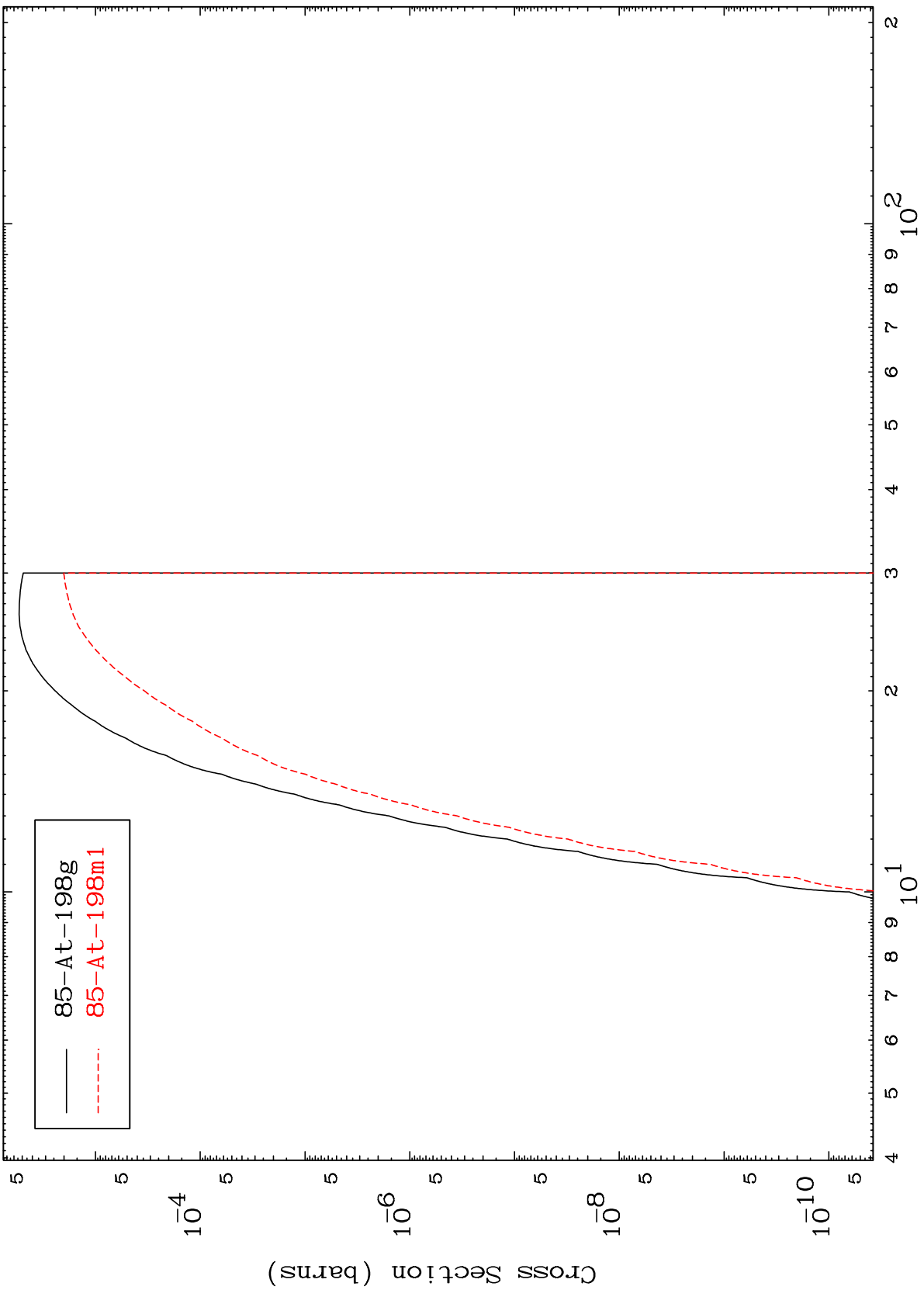
Incident Energy (MeV)

38

MAT 8592

86-Rn-200

(n, t)  
Radionuclide Production Cross Section



85-At-198g  
85-At-198m1

39

Incident Energy (MeV)

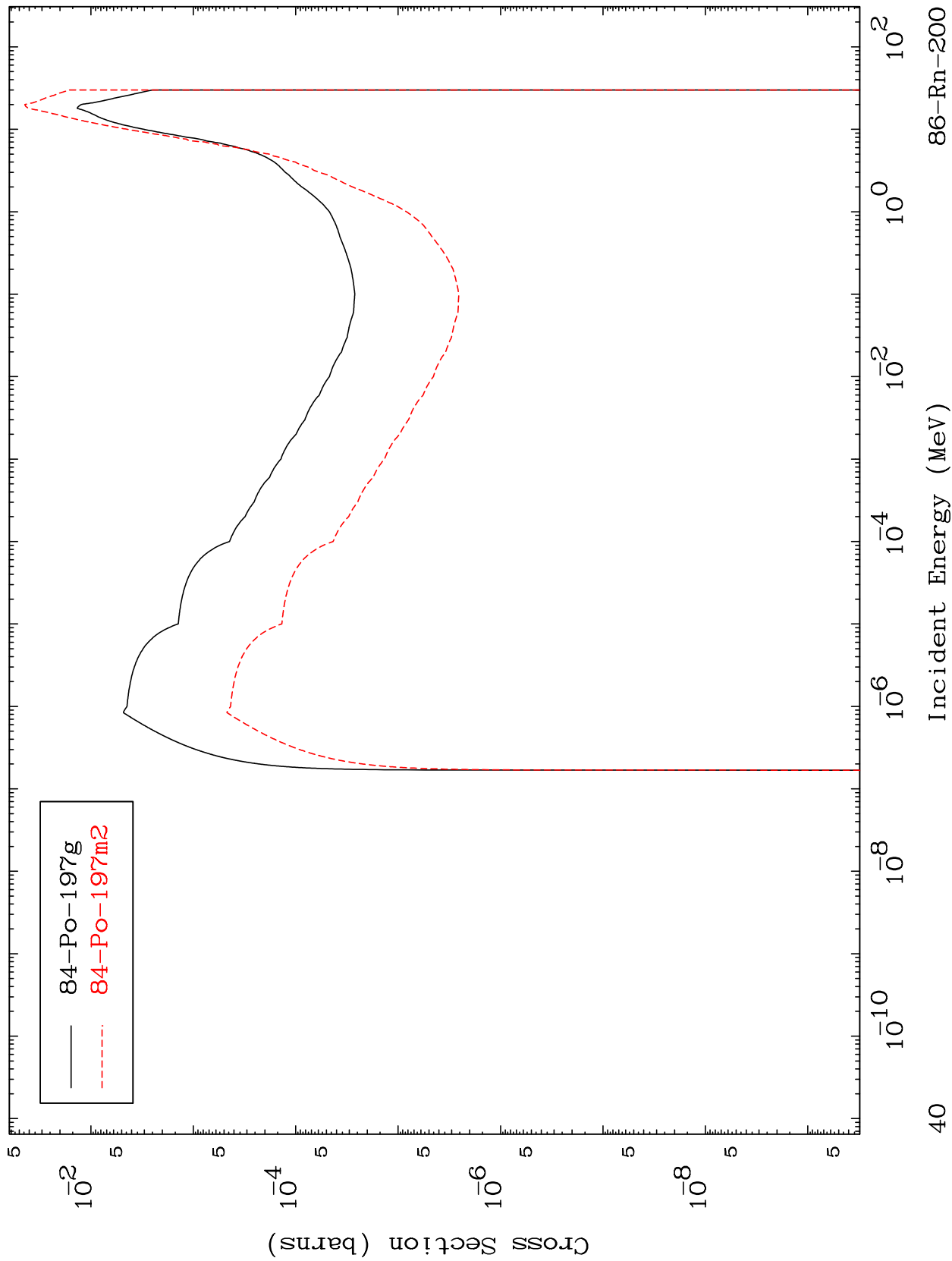
86-Rn-200



MAT 8592

86-Rn-200

(n,  $\alpha$ )  
Radionuclide Production Cross Section



— 84-Po-197g  
- - - 84-Po-197m2

40

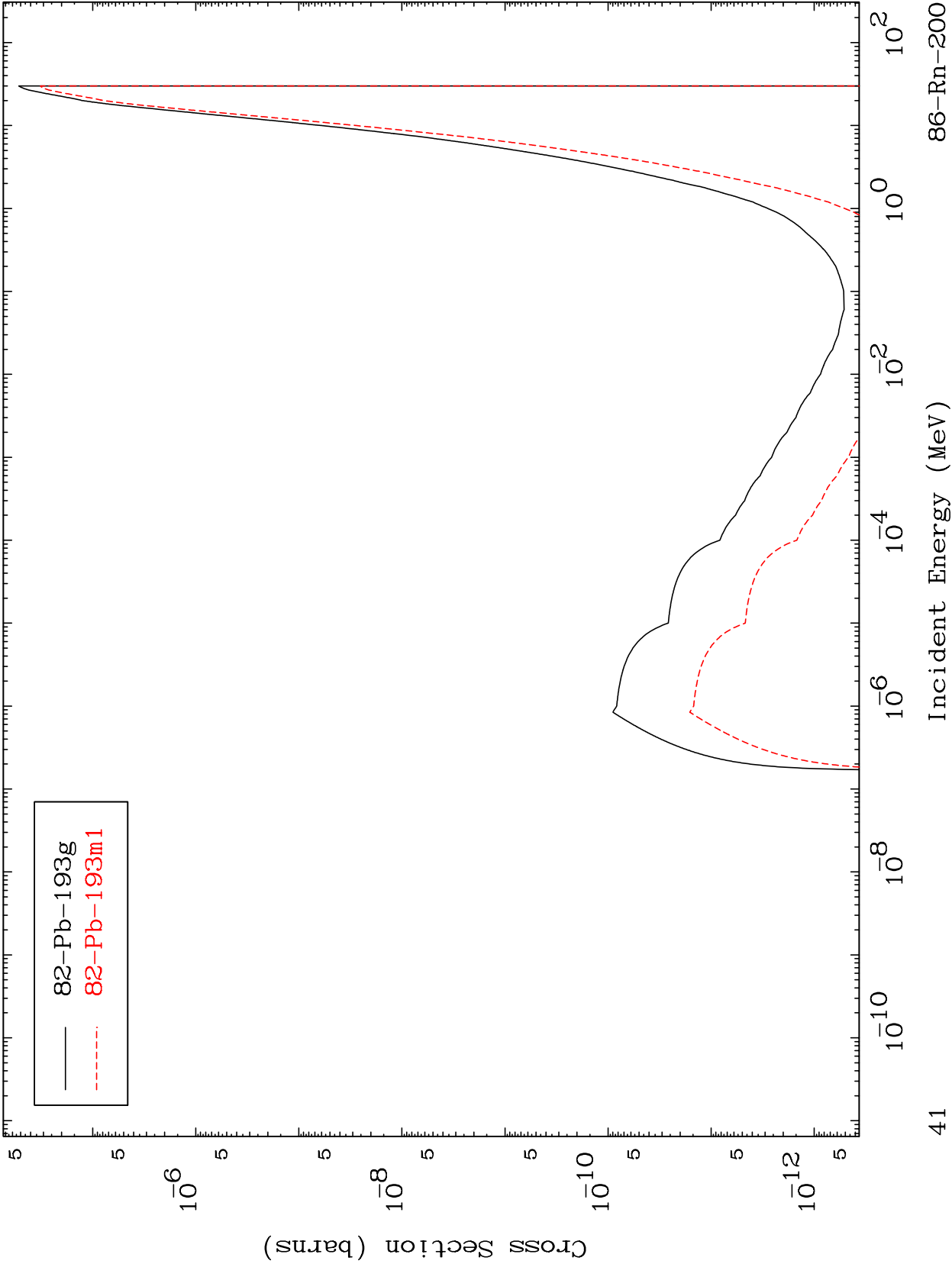
86-Rn-200

MAT 8592

(n,2 $\alpha$ )

86-Rn-200

Radionuclide Production Cross Section

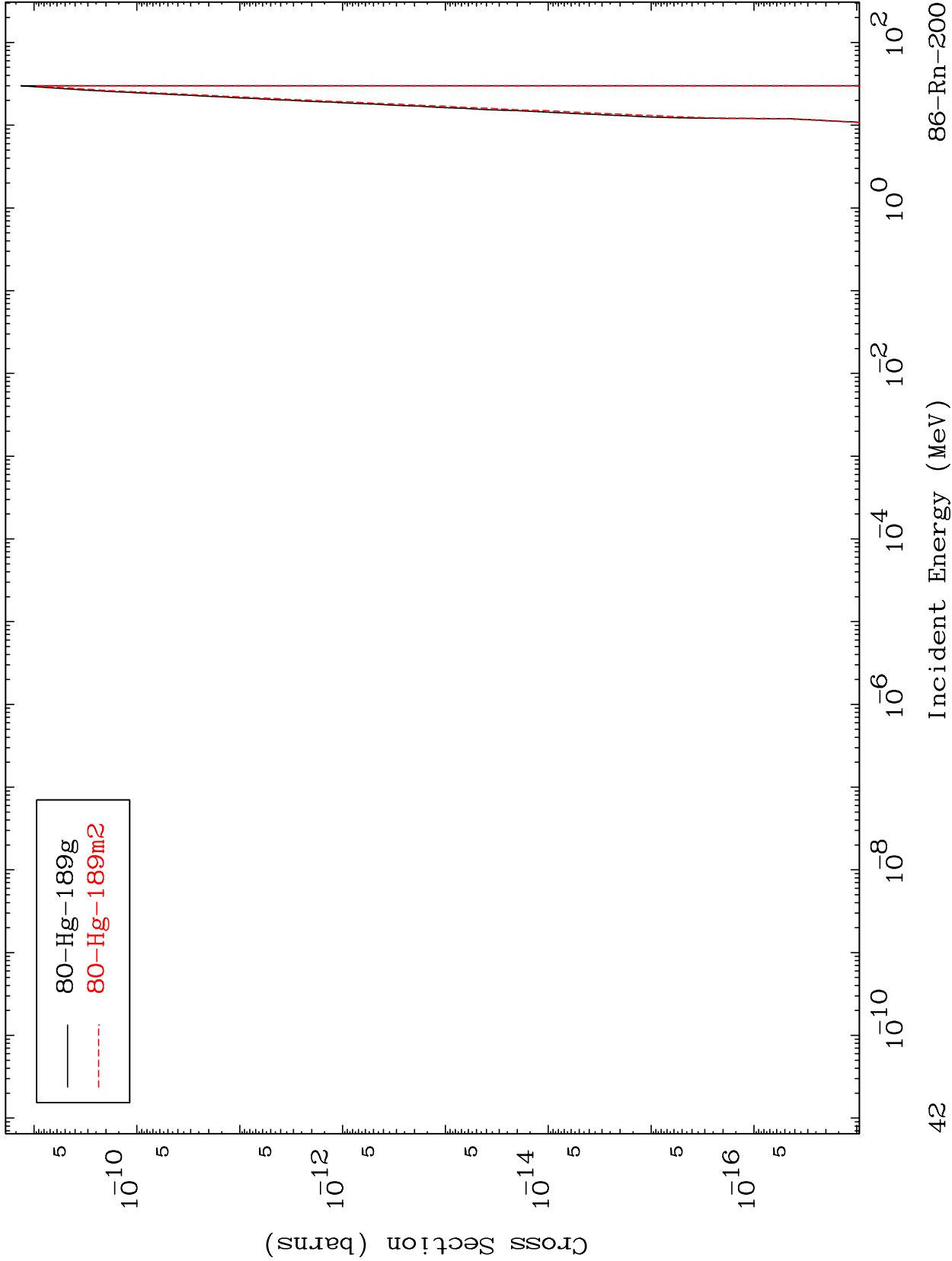


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(n, 3α)

86-Rn-200

Radionuclide Production Cross Section

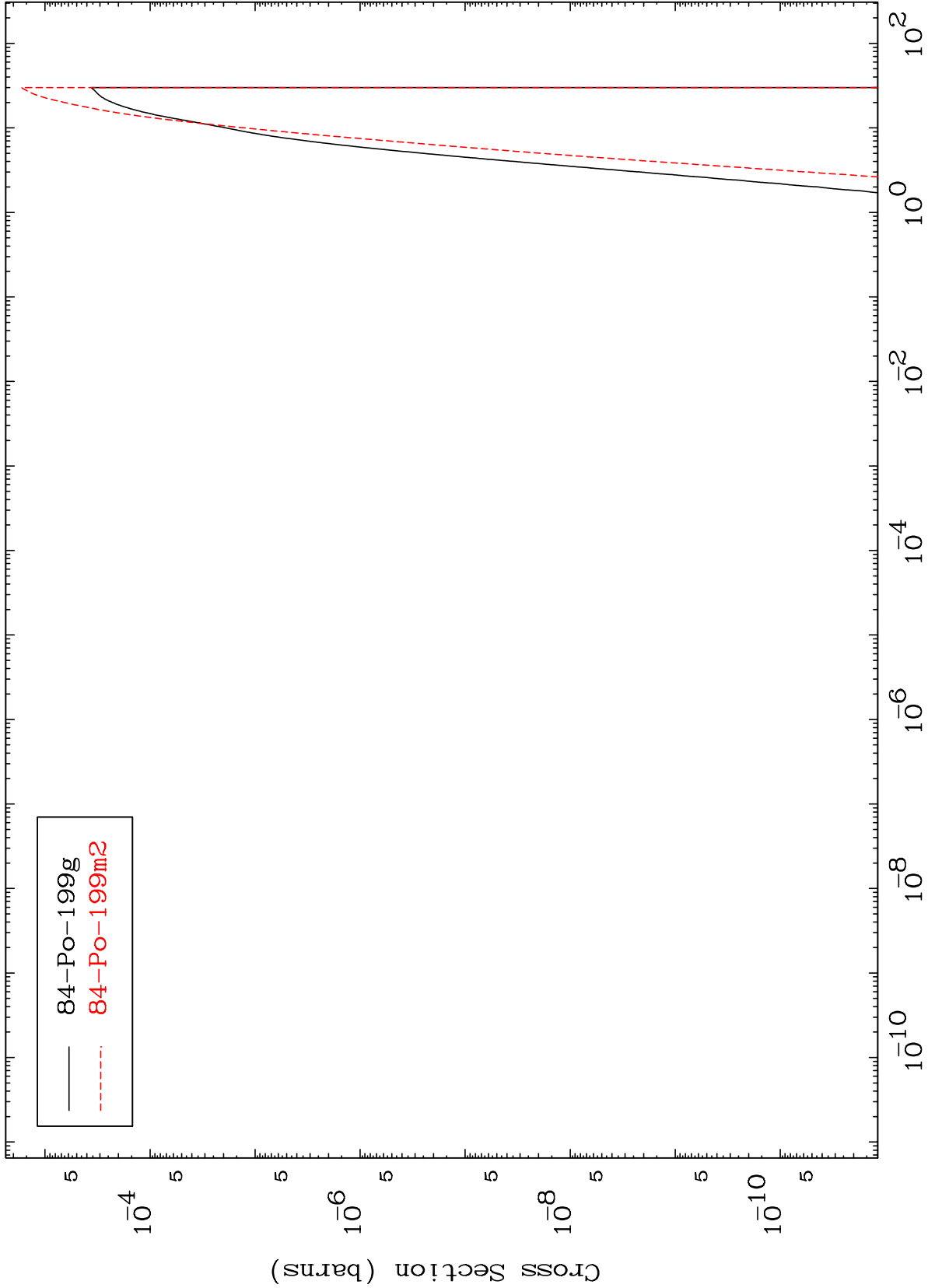


MAT 8592

(n,2p)

86-Rn-200

Radionuclide Production Cross Section

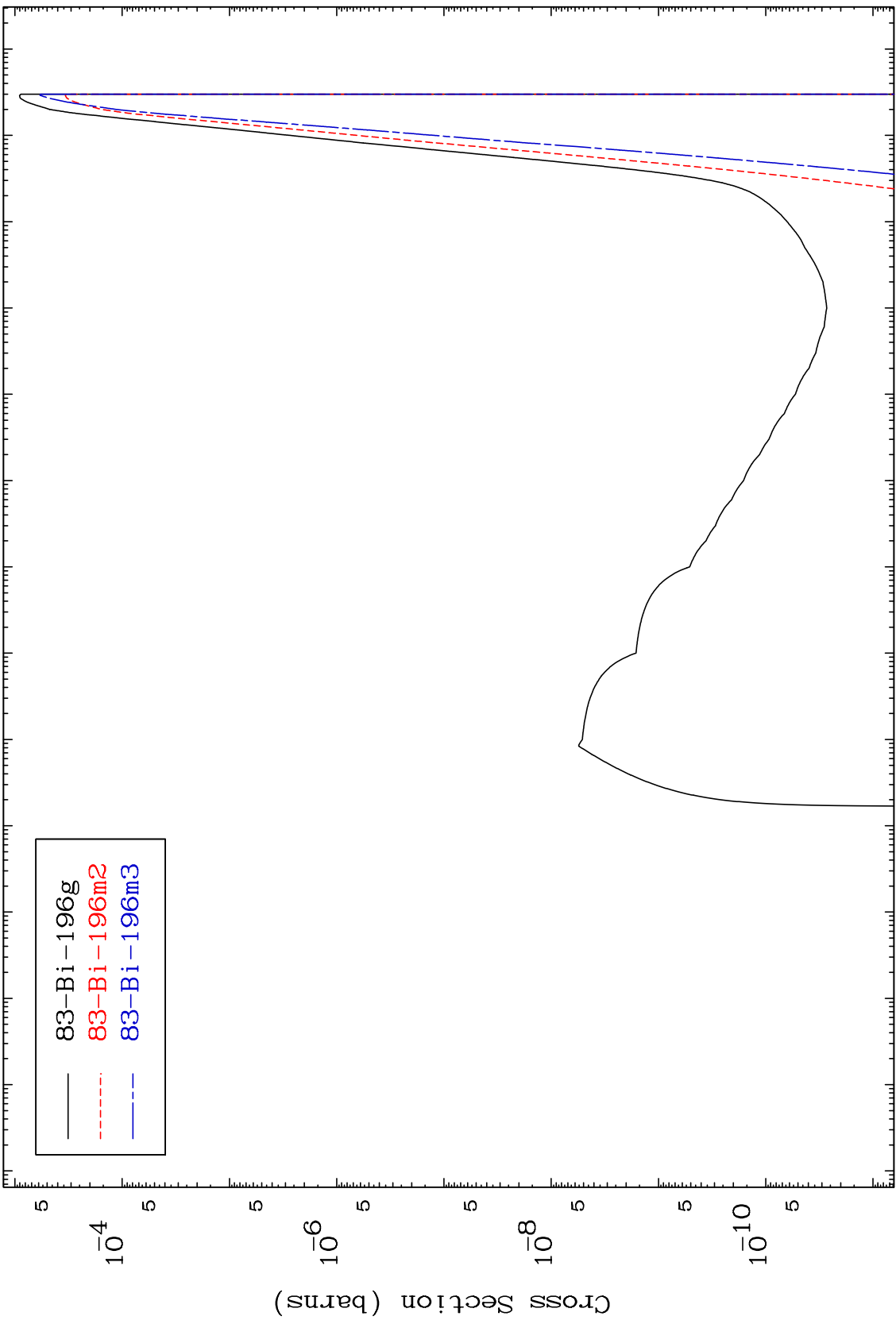
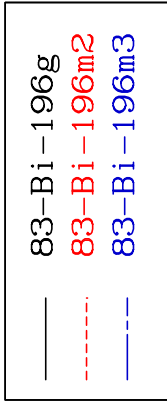


MAT 8592

(n,p)  $\alpha$

86-Rn-200

Radionuclide Production Cross Section



86-Rn-200

10<sup>2</sup>

10<sup>0</sup>

10<sup>-2</sup>

10<sup>-4</sup>

10<sup>-6</sup>

10<sup>-8</sup>

10<sup>-10</sup>

Incident Energy (MeV)

86-Rn-200

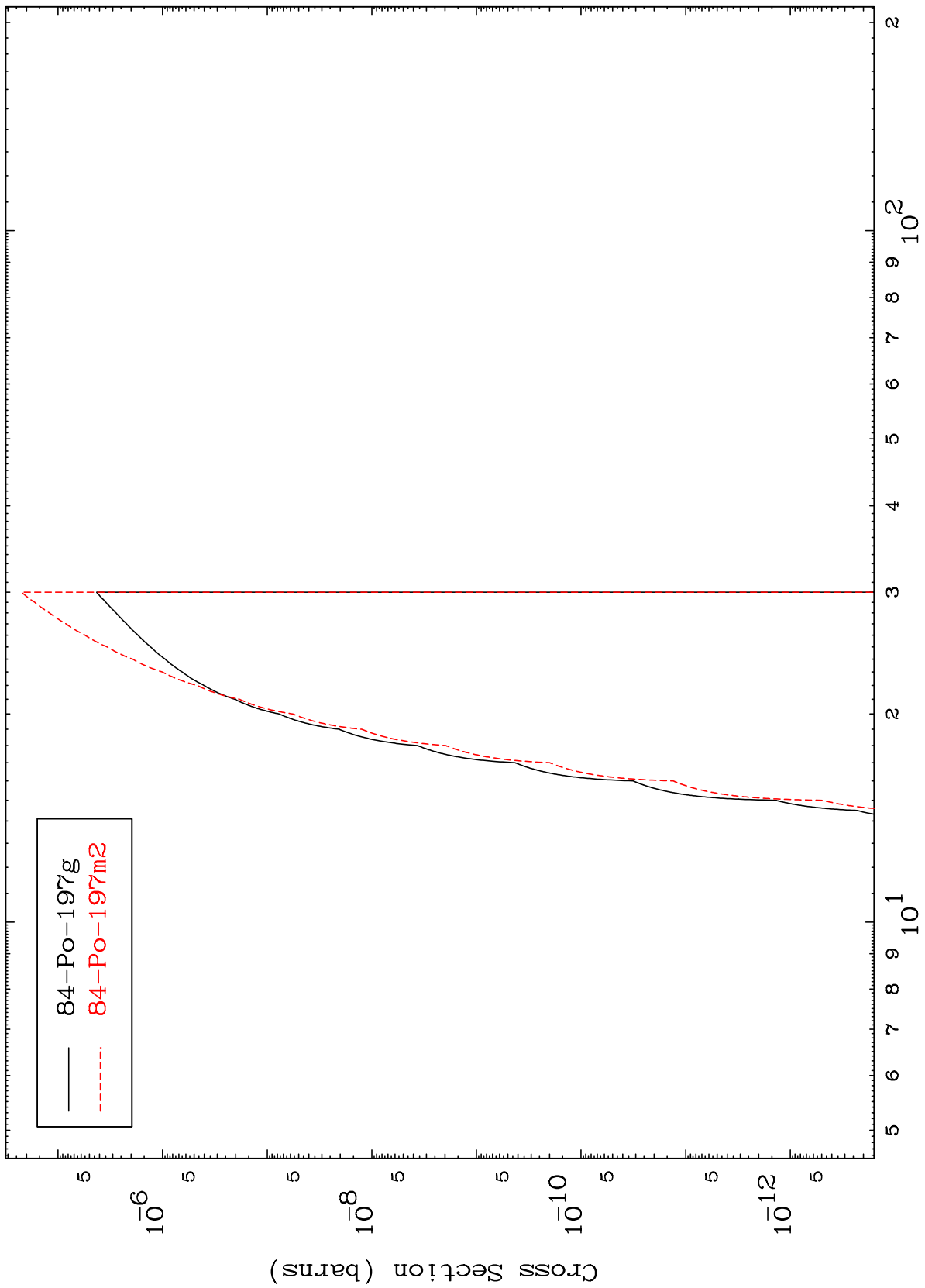
44

MAT 8592

(n,p) t

86-Rn-200

Radionuclide Production Cross Section



45

Incident Energy (MeV)

86-Rn-200

MAT 8592

(n,d)  $\alpha$

86-Rn-200

Radionuclide Production Cross Section

