

Program EVALPLOT
(Version 2018-1)

by

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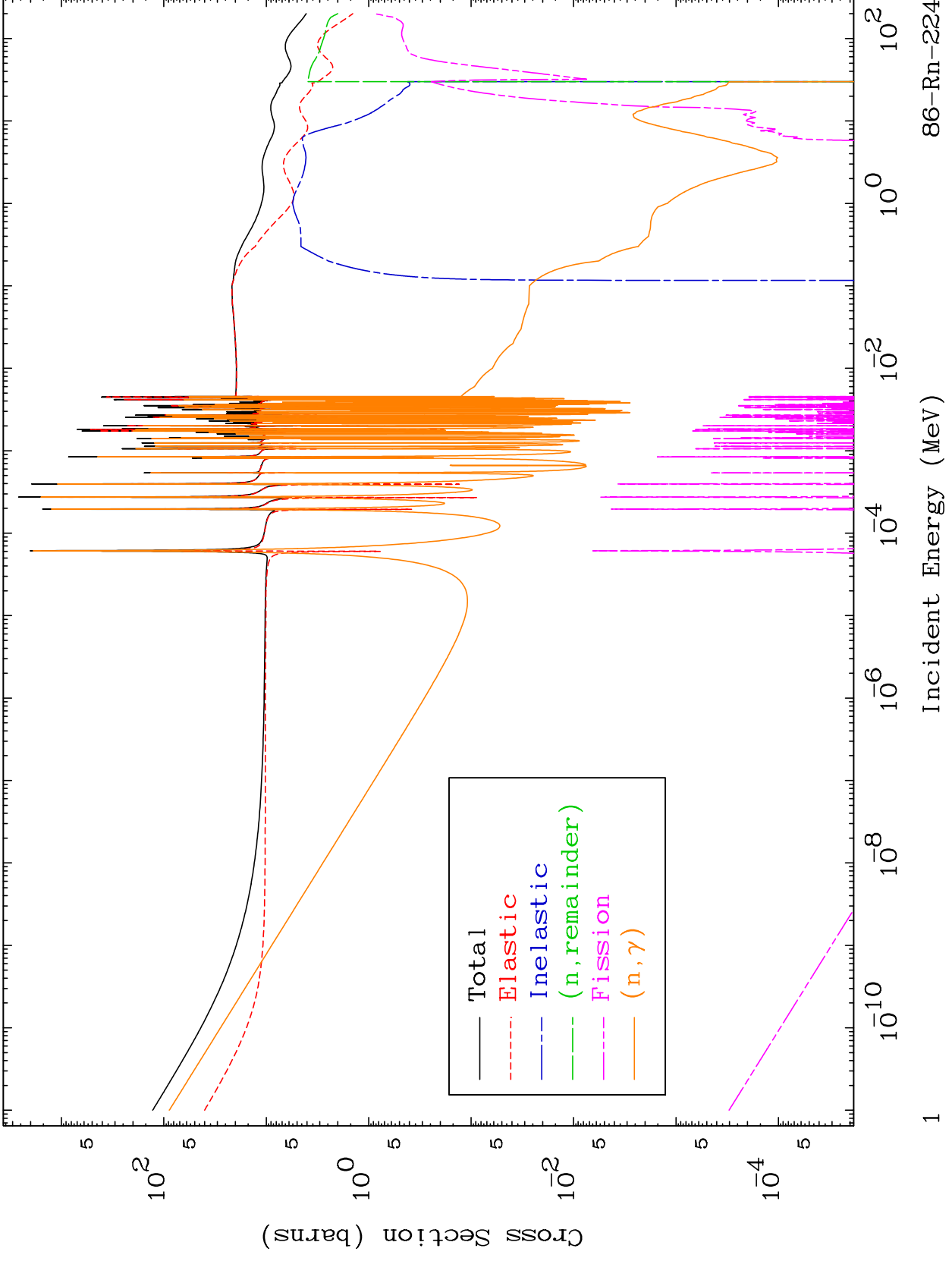
Press Mouse Button to Start

MAT 8664

Major

293 Kelvin Cross Sections

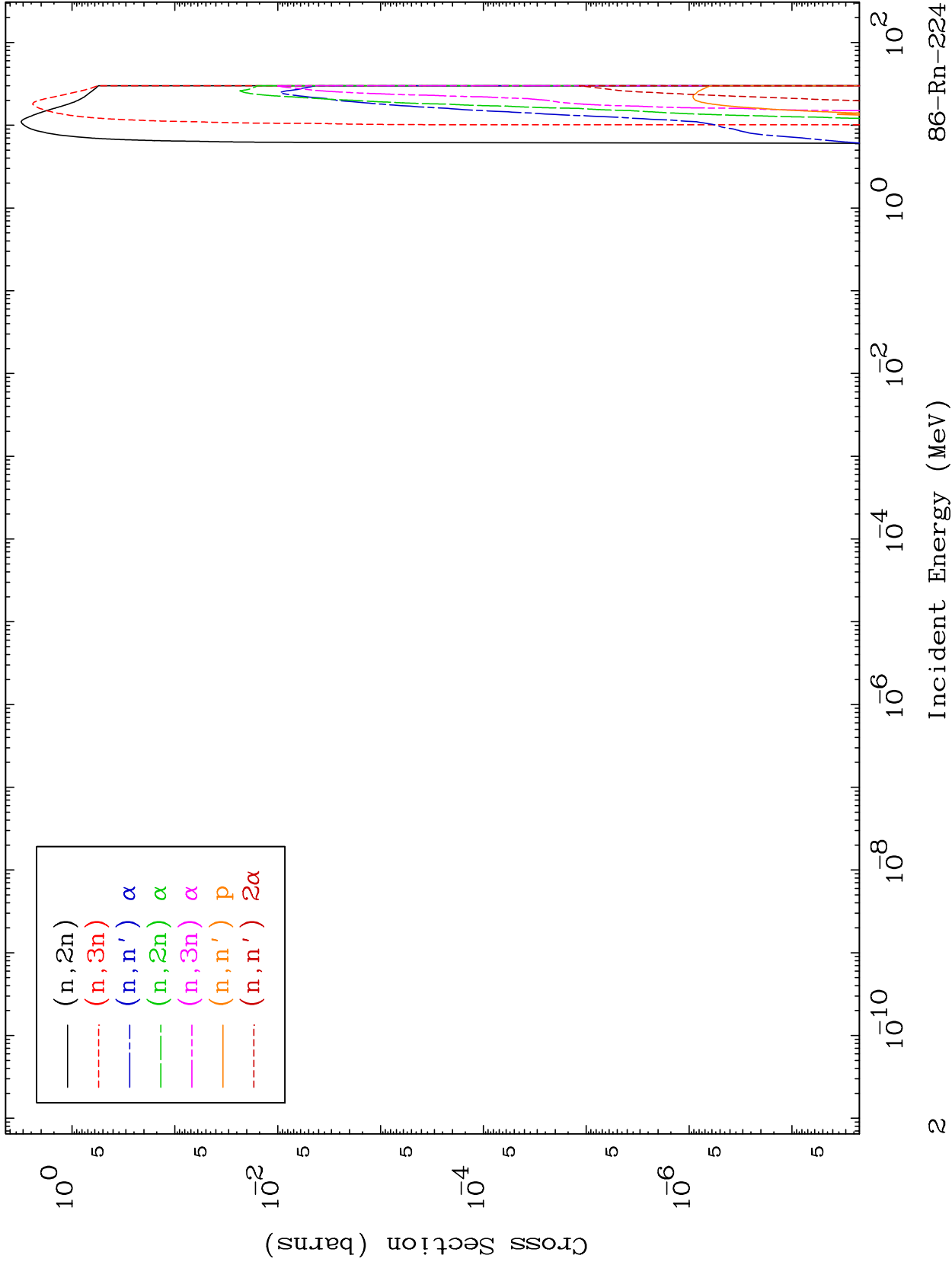
86-Rn-224



MAT 8664

Neutron Production
293 Kelvin Cross Sections

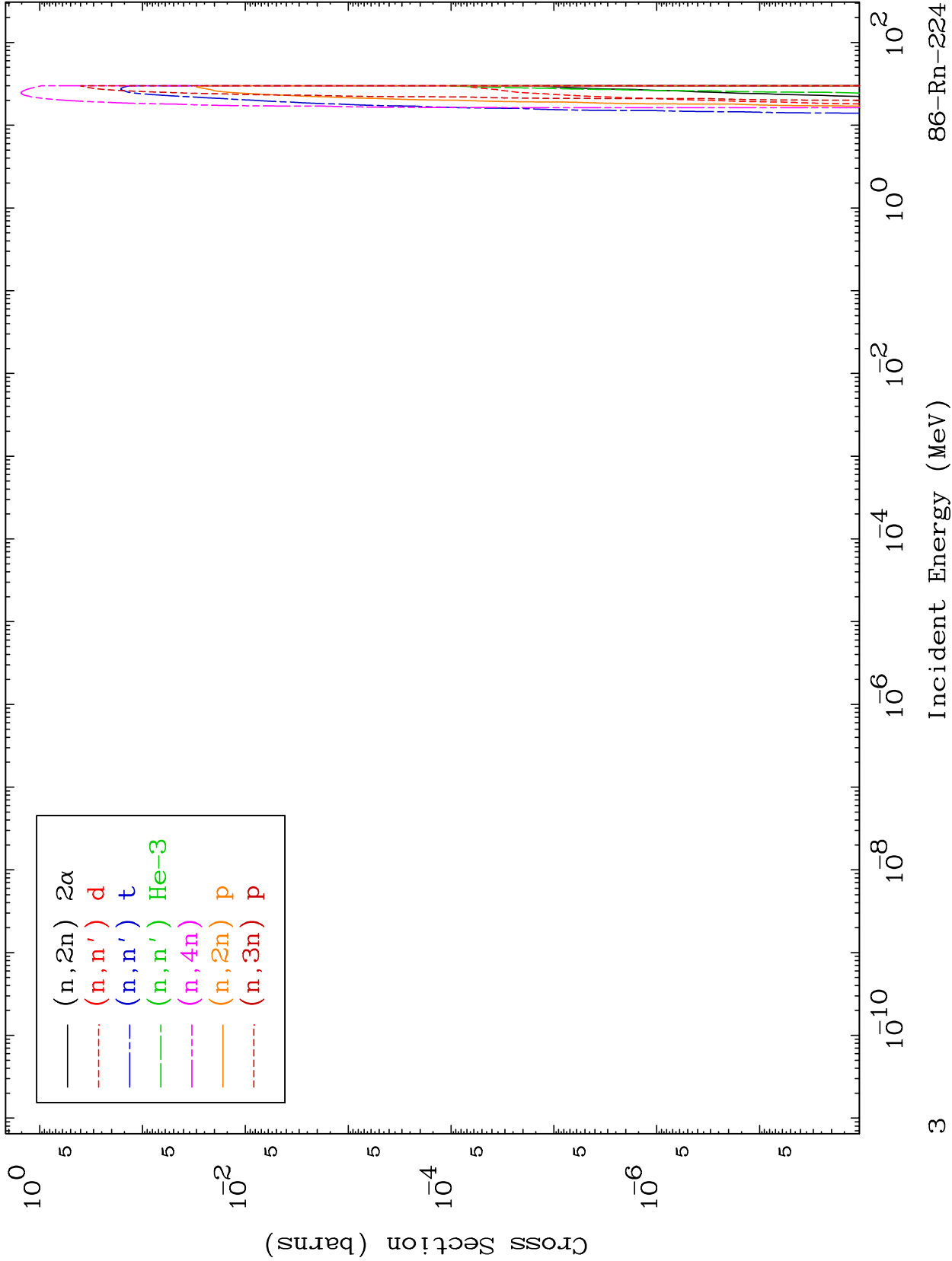
86-Rn-224



MAT 8664

Neutron Production
293 Kelvin Cross Sections

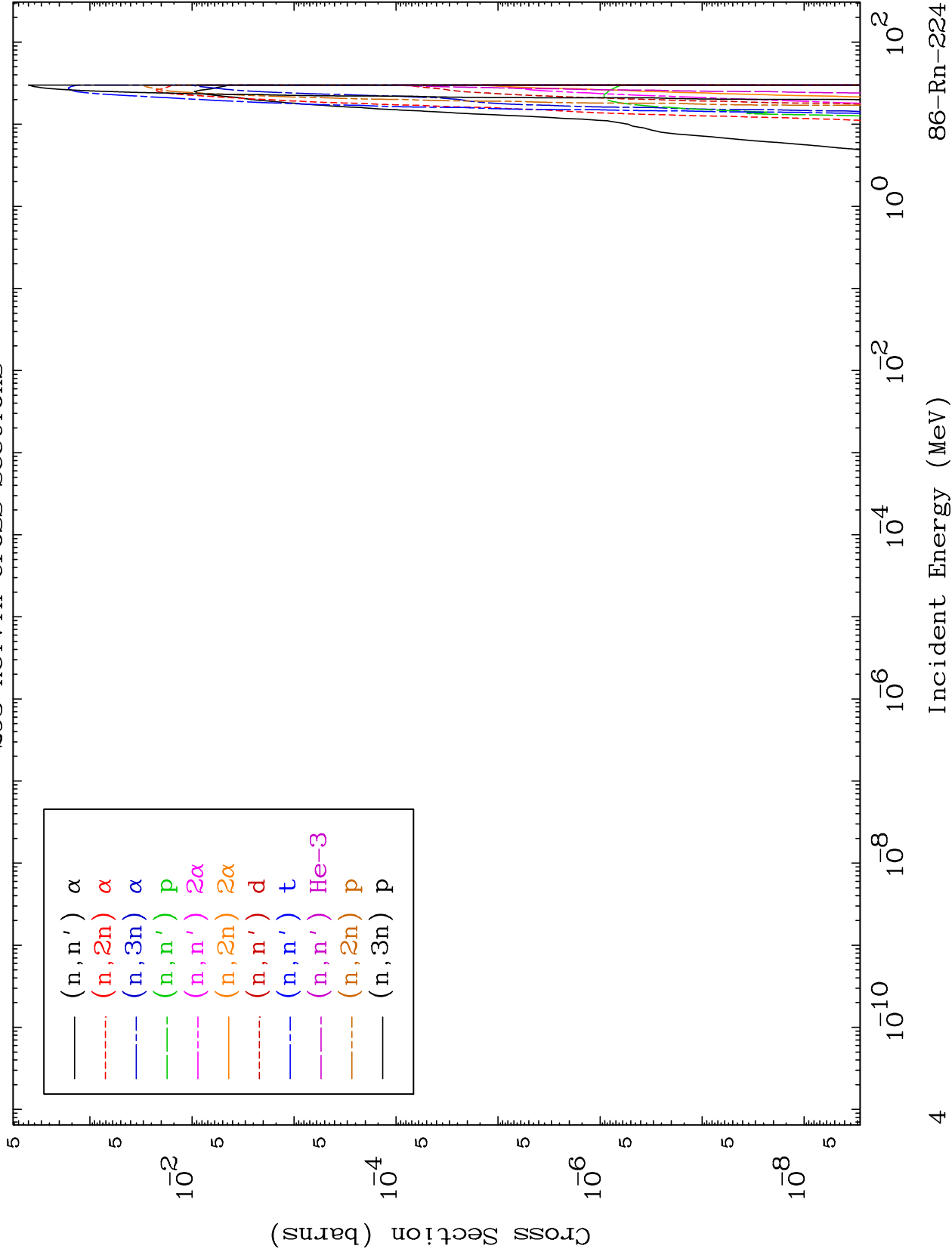
86-Rn-224



MAT 8664

Charged Particle
293 Kelvin Cross Sections

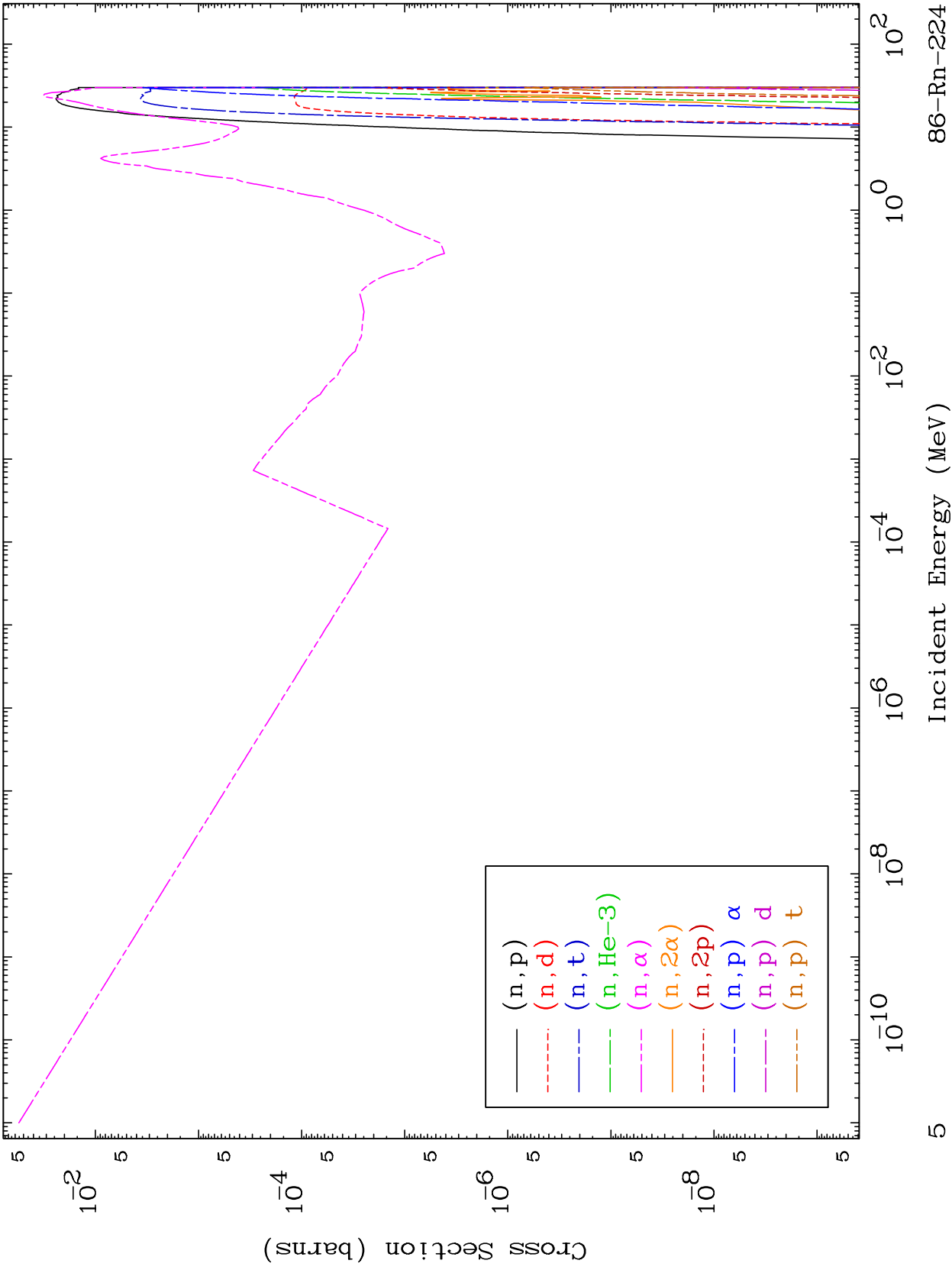
86-Rn-224



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Charged Particle
293 Kelvin Cross Sections

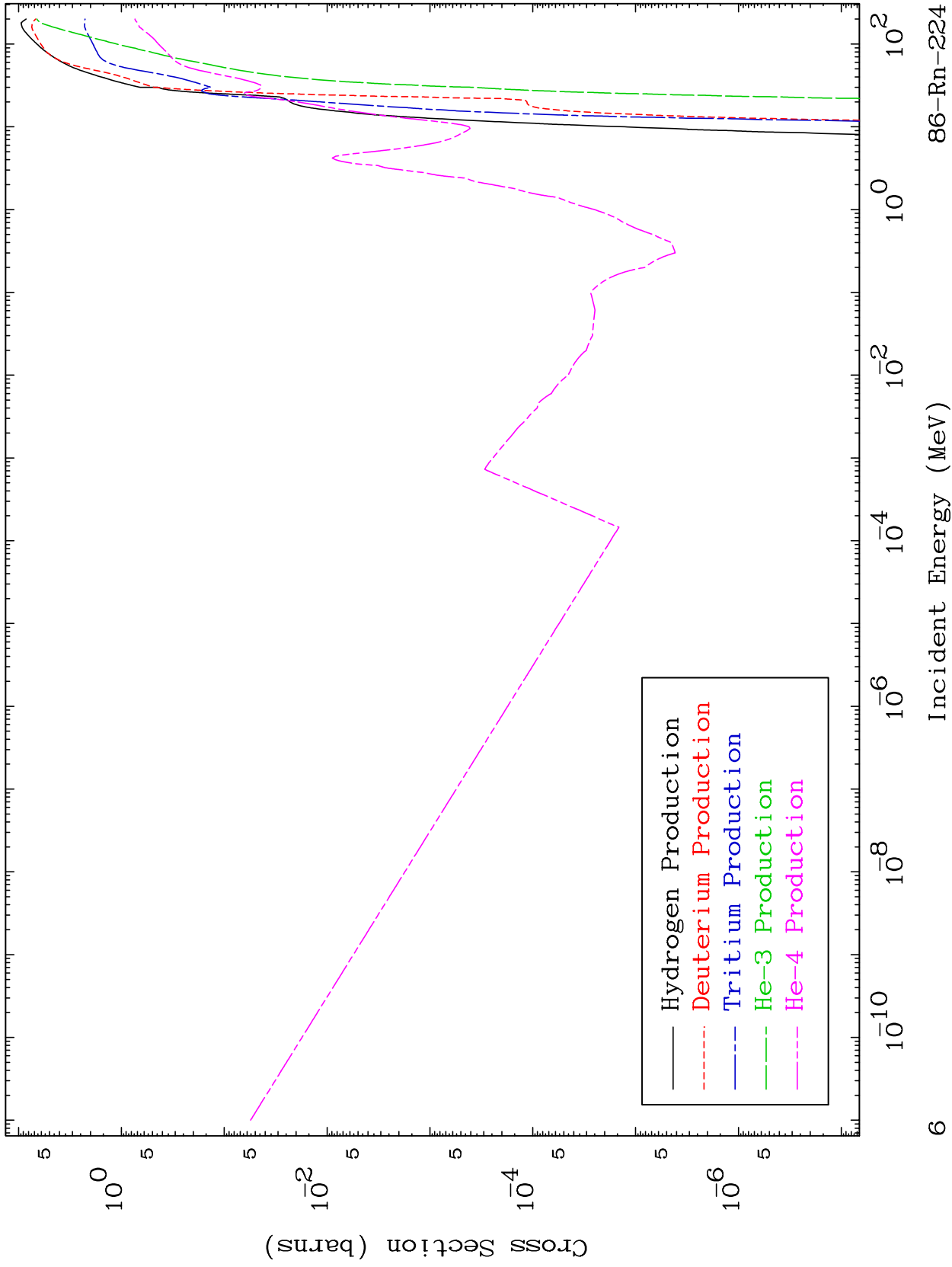
86-Rn-224



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Particle Production
293 Kelvin Cross Sections

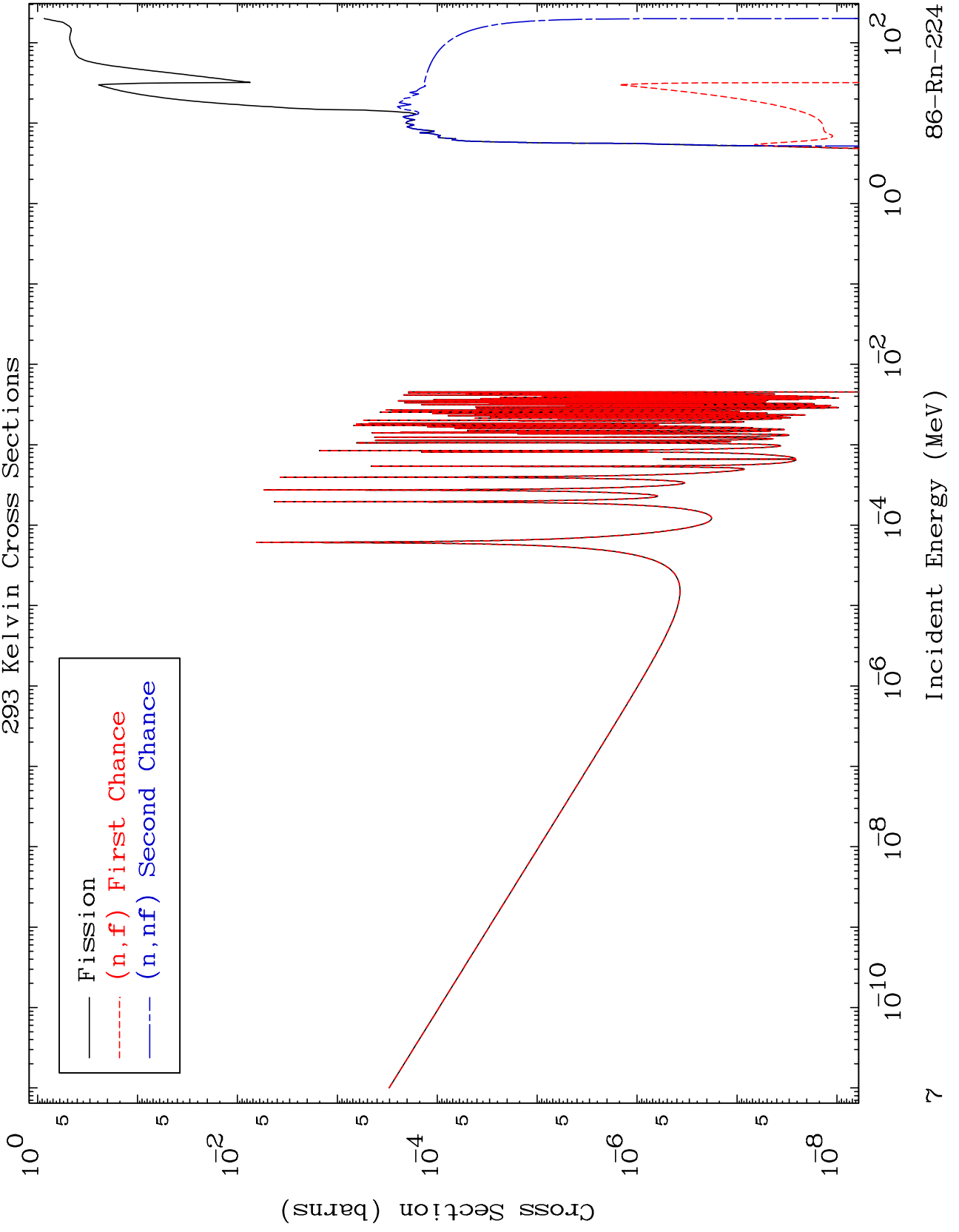
86-Rn-224



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Fission
293 Kelvin Cross Sections

86-Rn-224

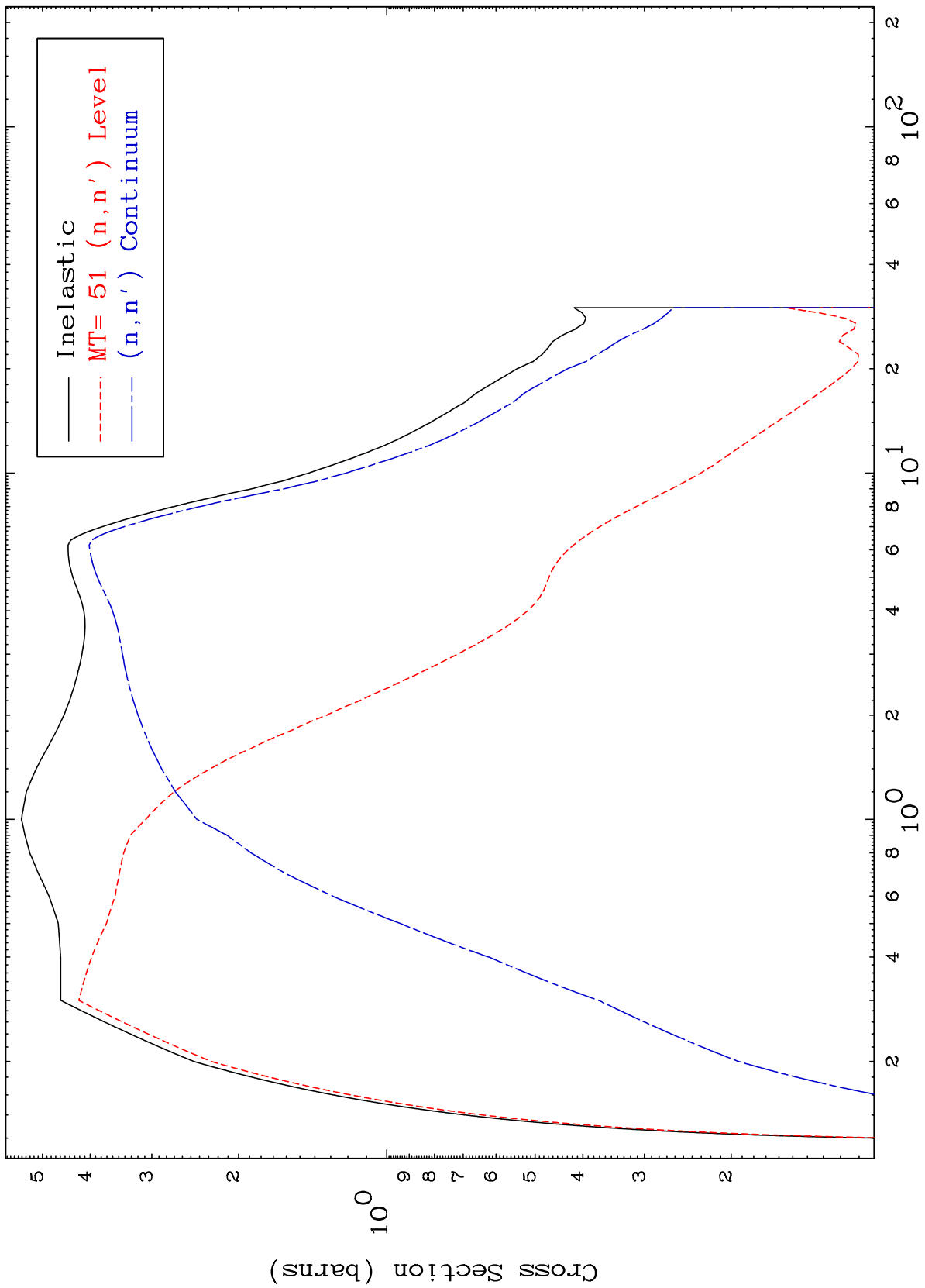


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(n,n') Level

86-Rn-224

293 Kelvin Cross Sections



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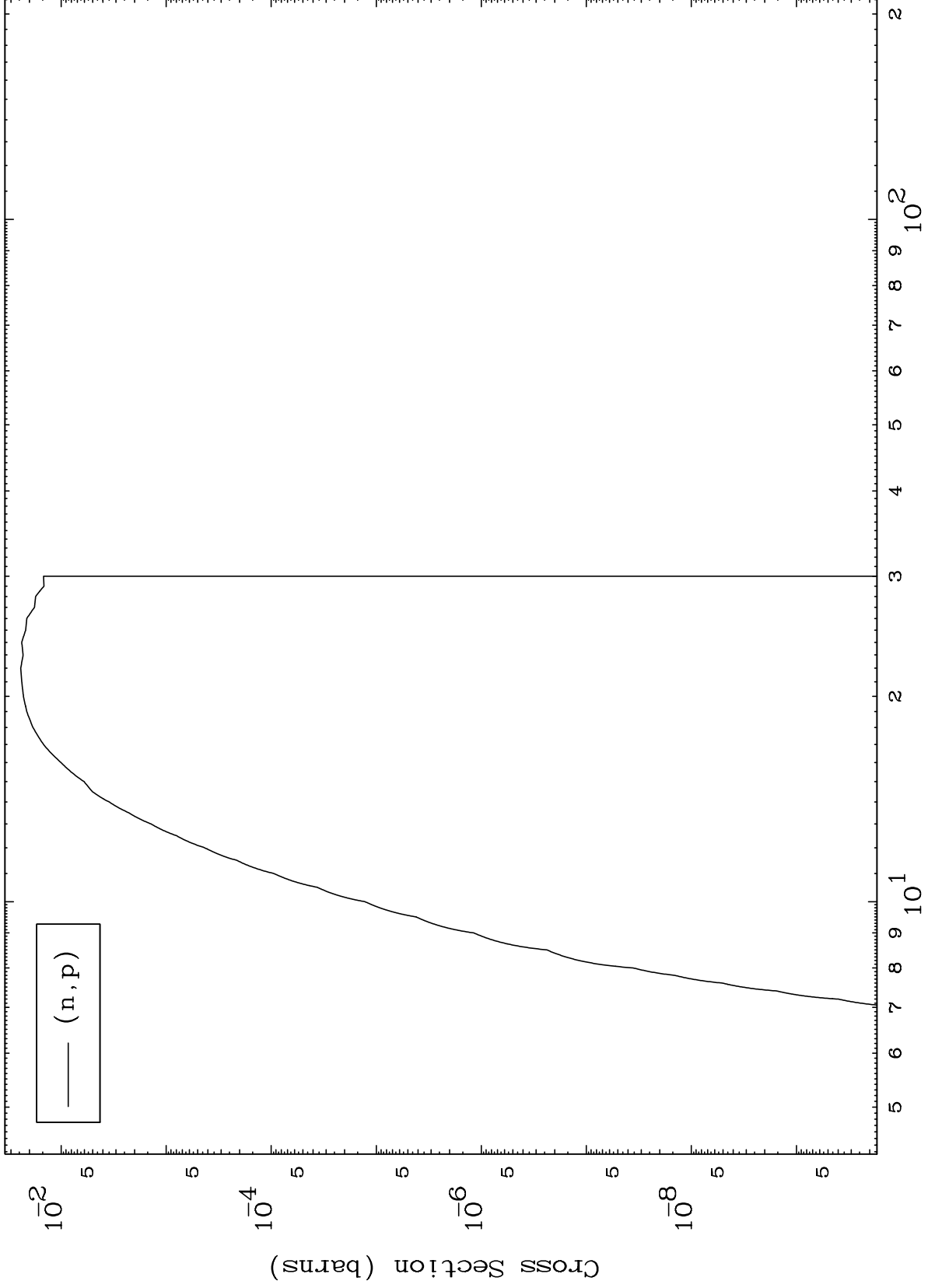
Incident Energy (MeV)

86-Rn-224

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(n,p) Levels
293 Kelvin Cross Sections

86-Rn-224



9

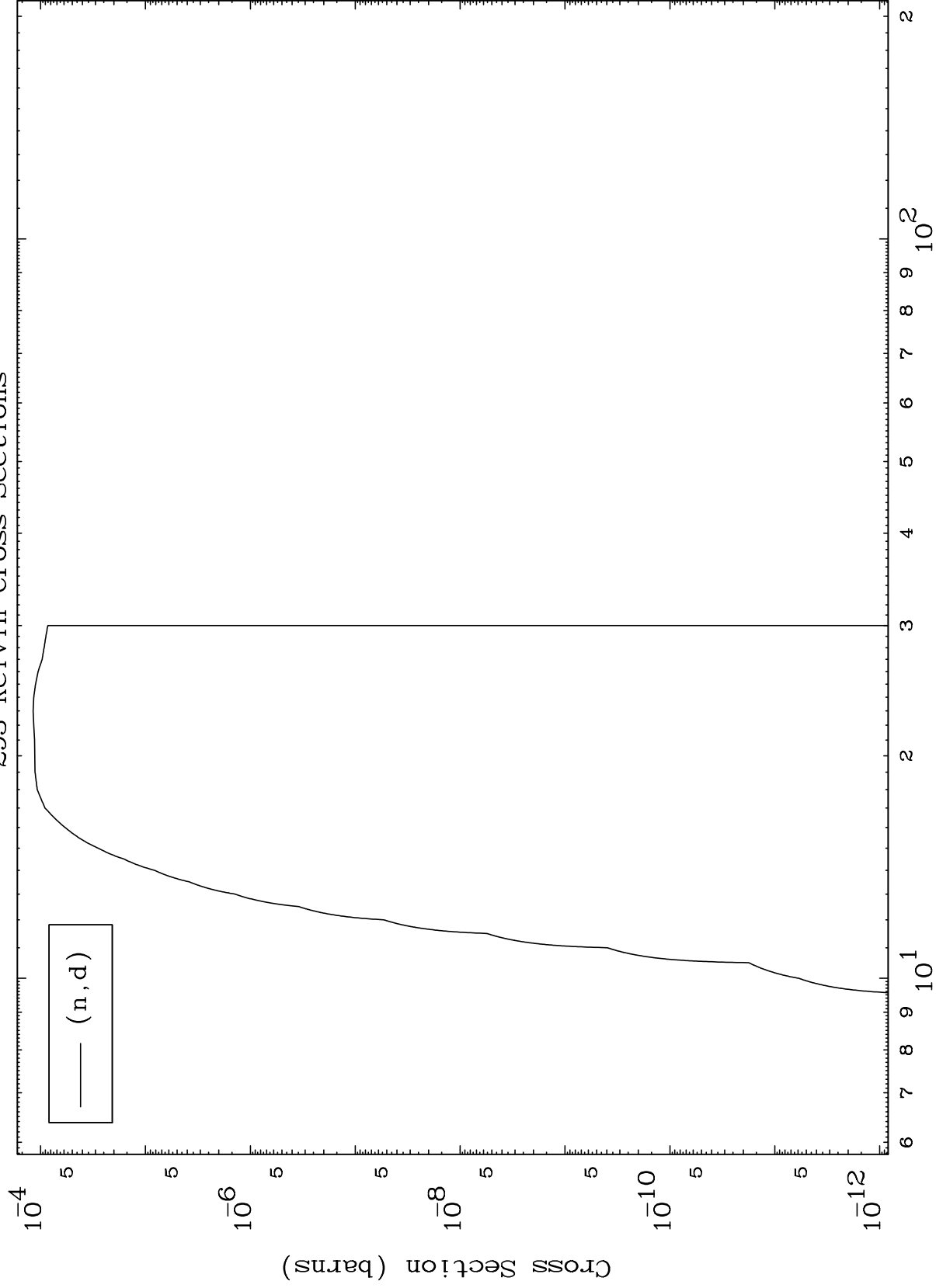
Incident Energy (MeV)

86-Rn-224

MAT 8664

(n,d) Levels
293 Kelvin Cross Sections

86-Rn-224



10

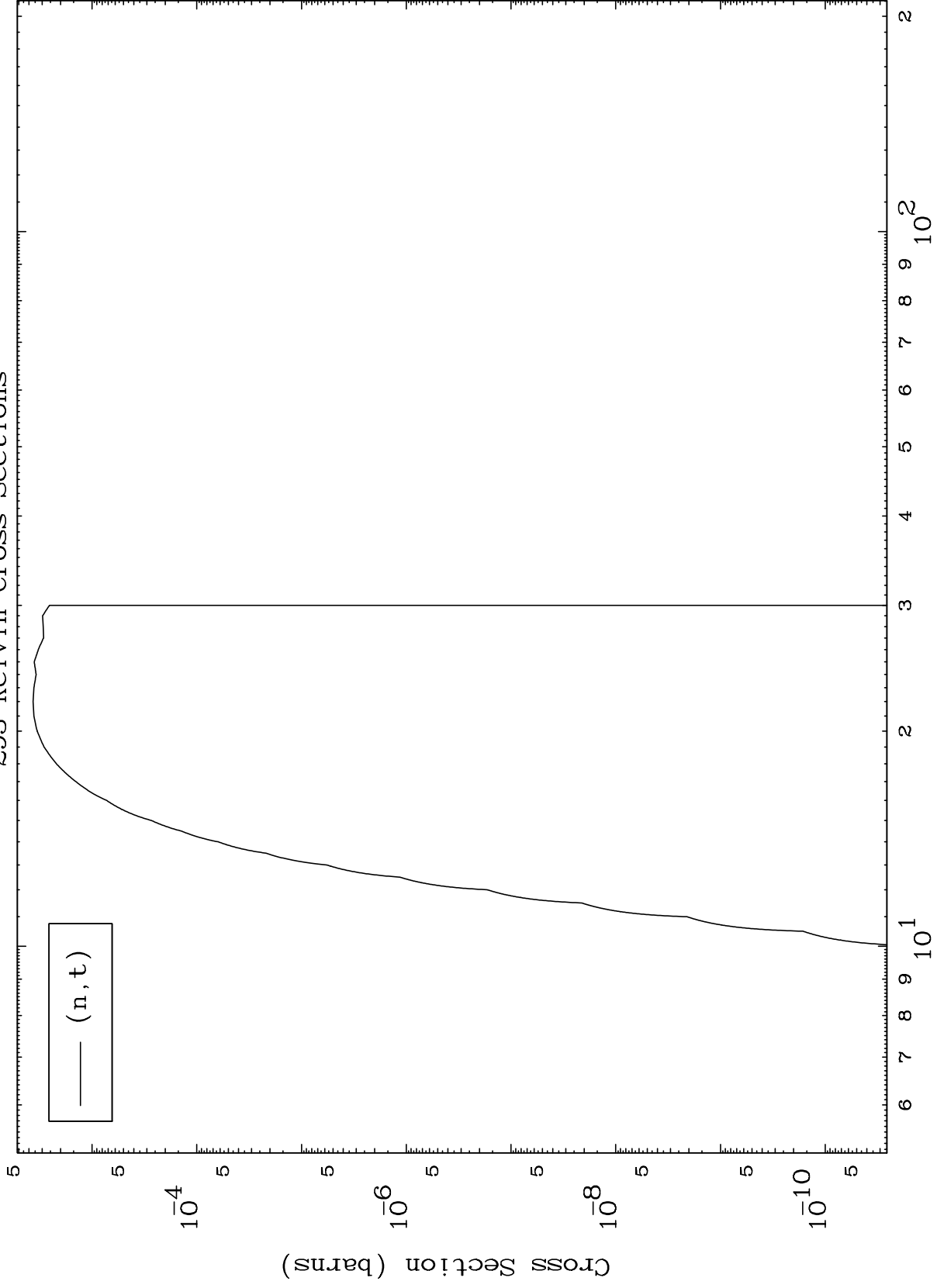
Incident Energy (MeV)

86-Rn-224

MAT 8664

(n, t) Levels
293 Kelvin Cross Sections

86-Rn-224



11

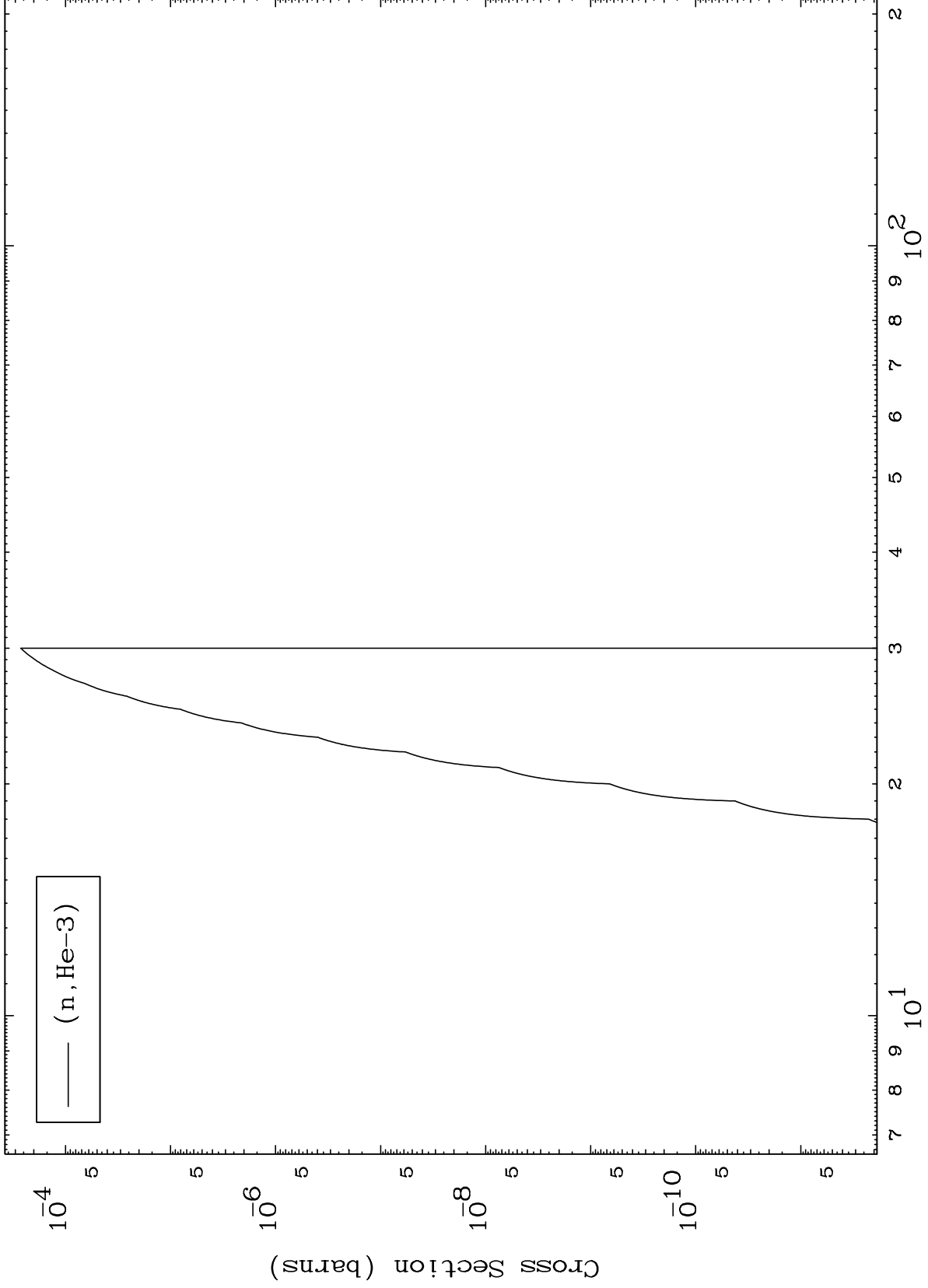
Incident Energy (MeV)

86-Rn-224

MAT 8664

(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-224



12

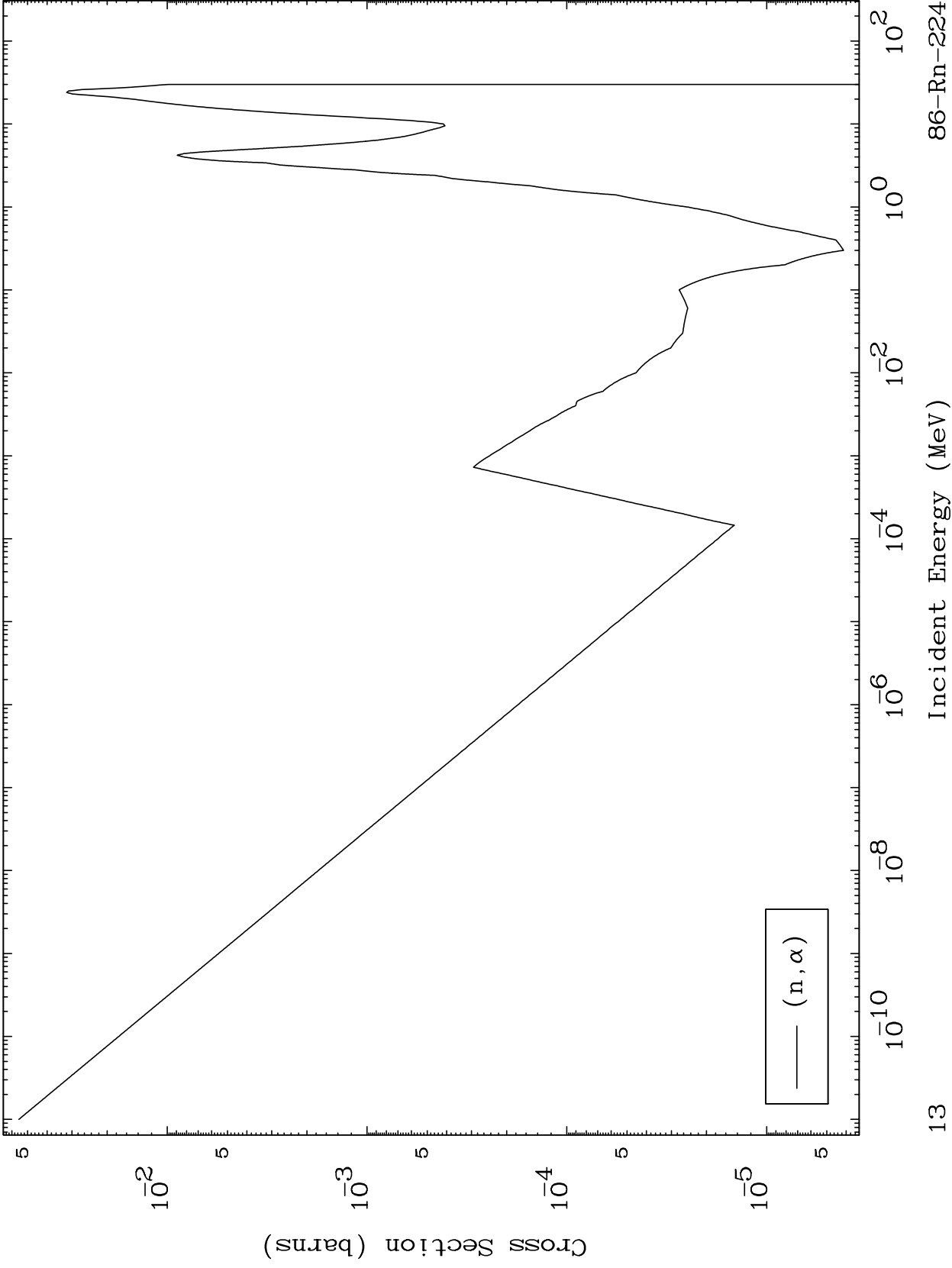
Incident Energy (MeV)

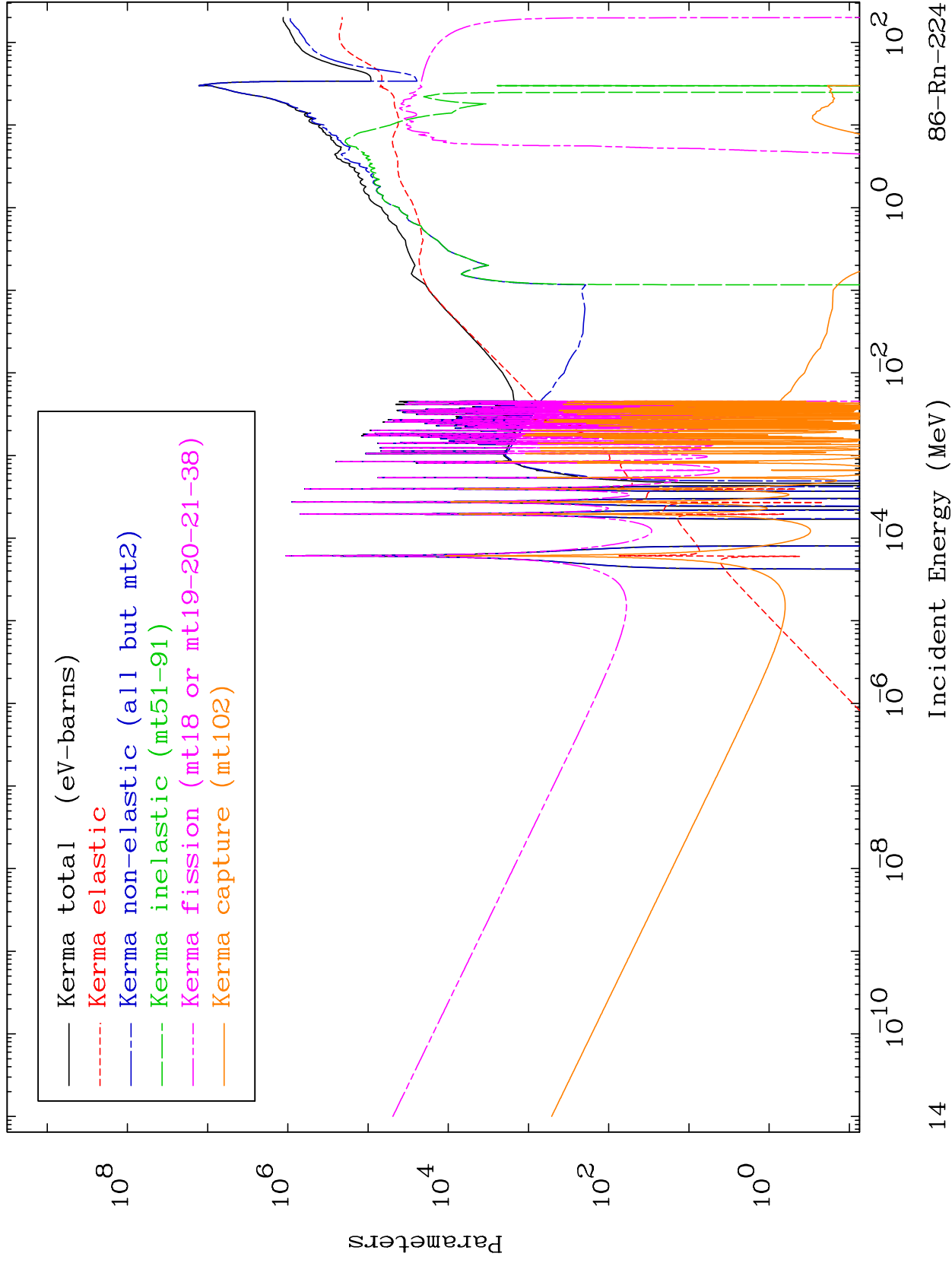
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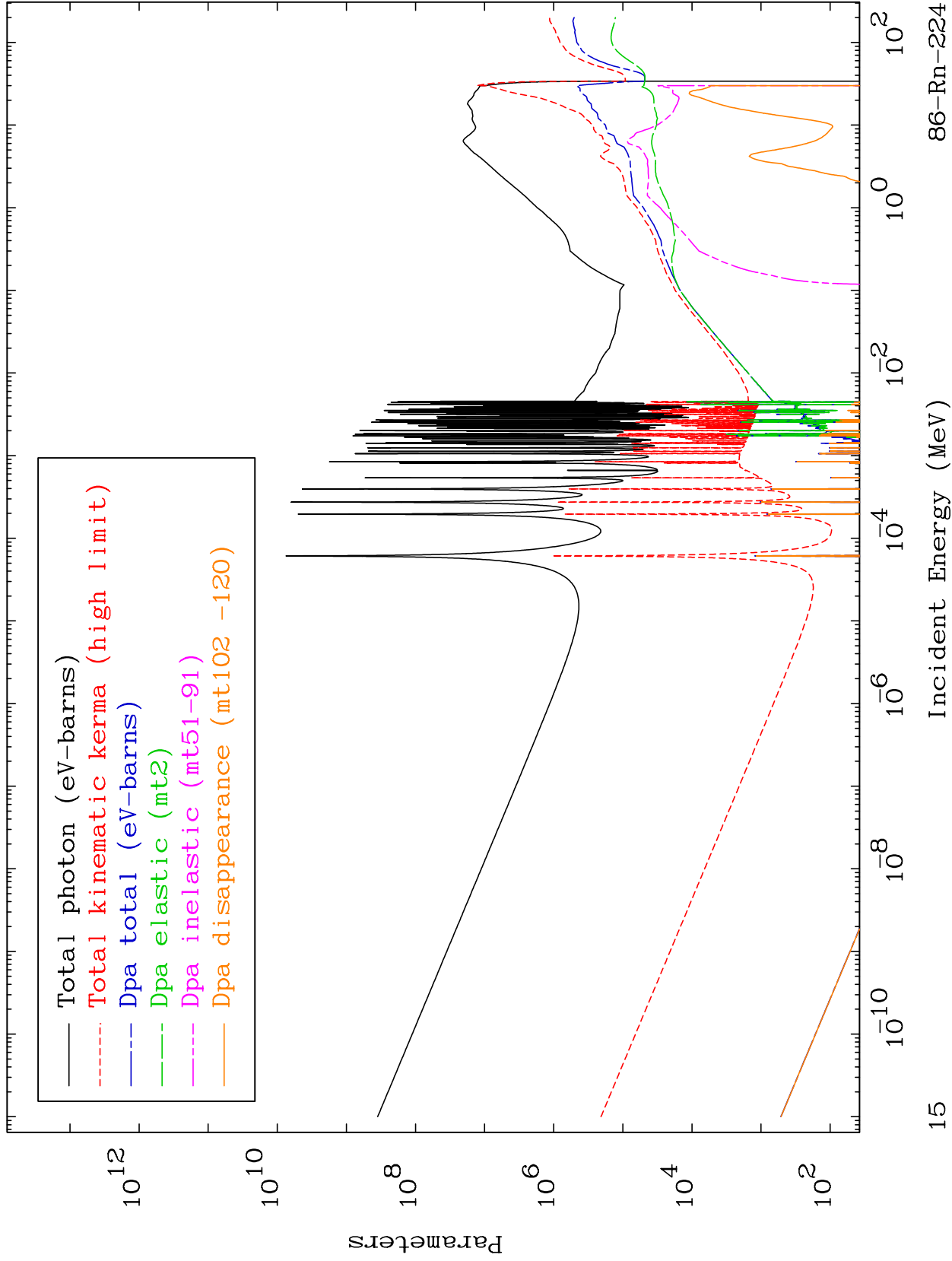
MAT 8664

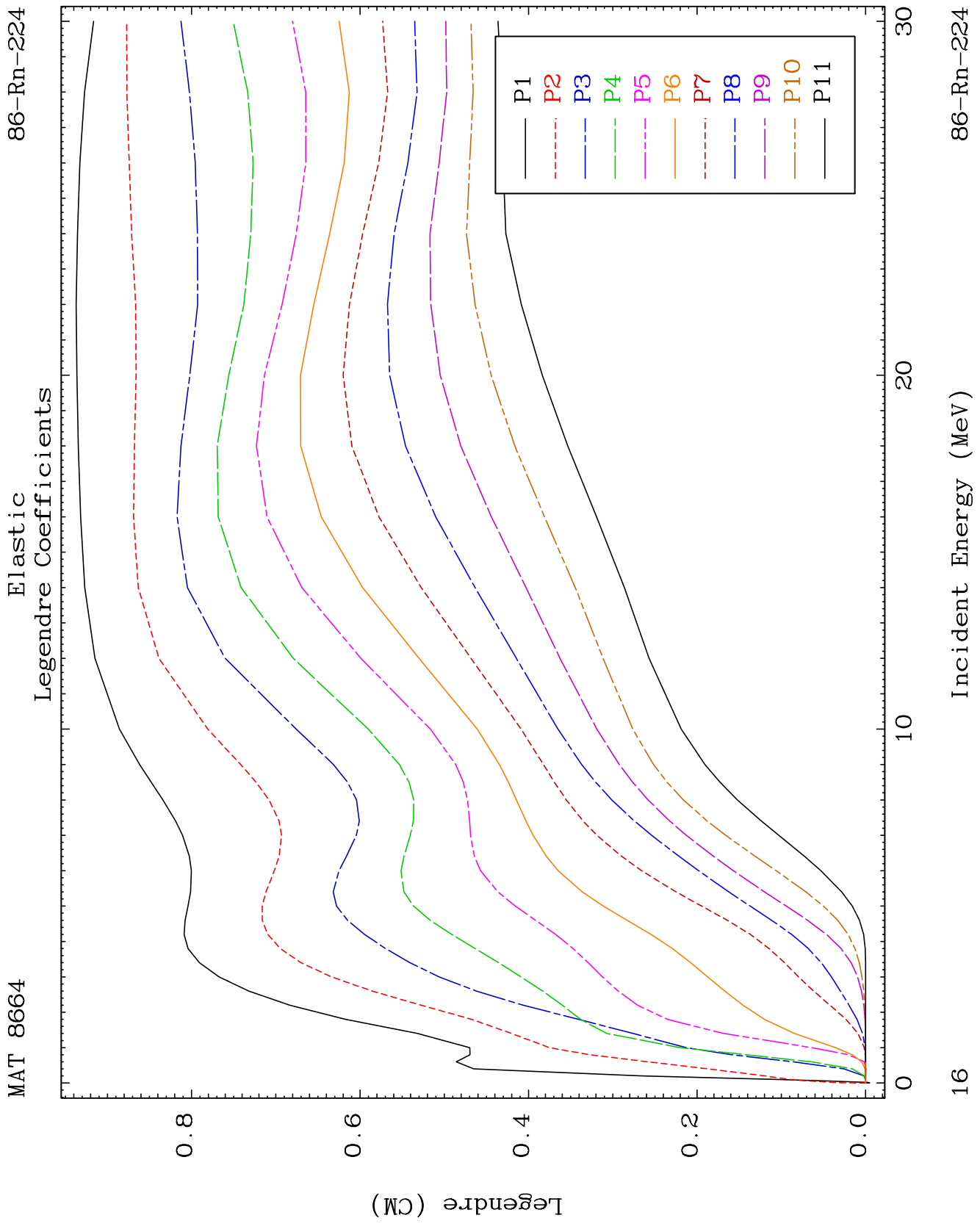
(n, α) Levels
293 Kelvin Cross Sections

86-Rn-224





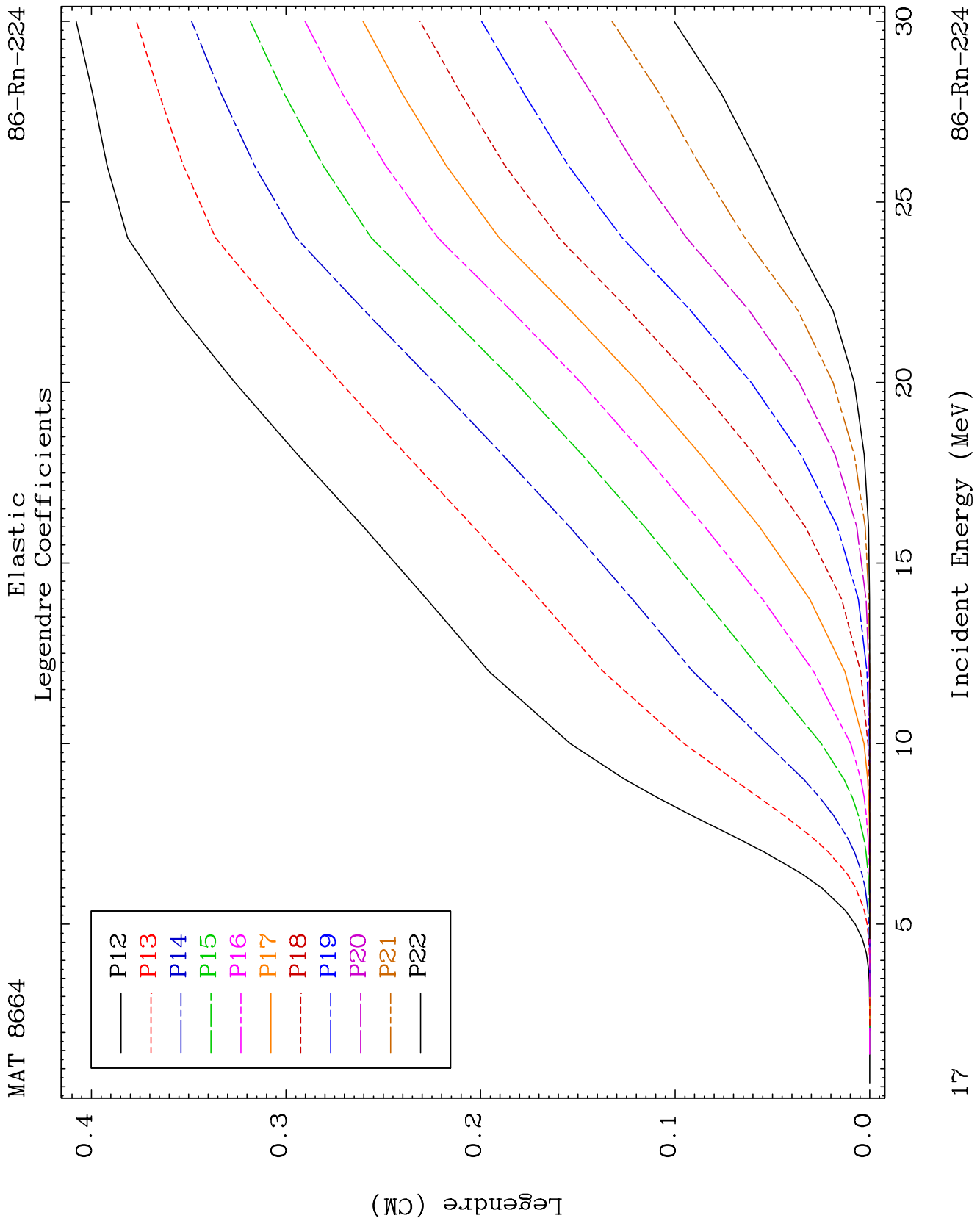




86-Rn-224

Incident Energy (MeV)

16



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Elastic Legendre Coefficients

$^{86}\text{Rn-224}$

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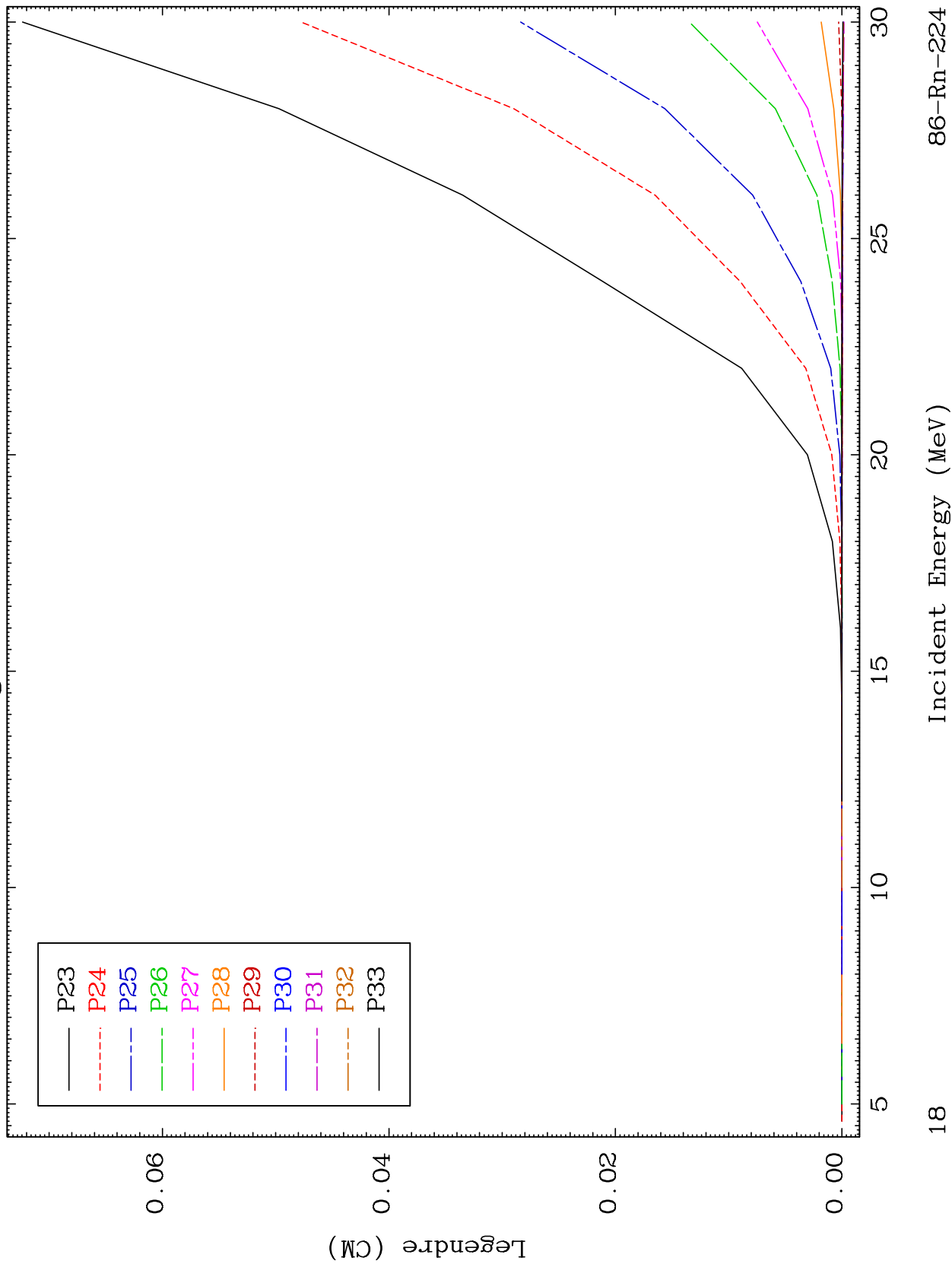
Incident Energy (MeV)

$^{86}\text{Rn-224}$

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Elastic Legendre Coefficients

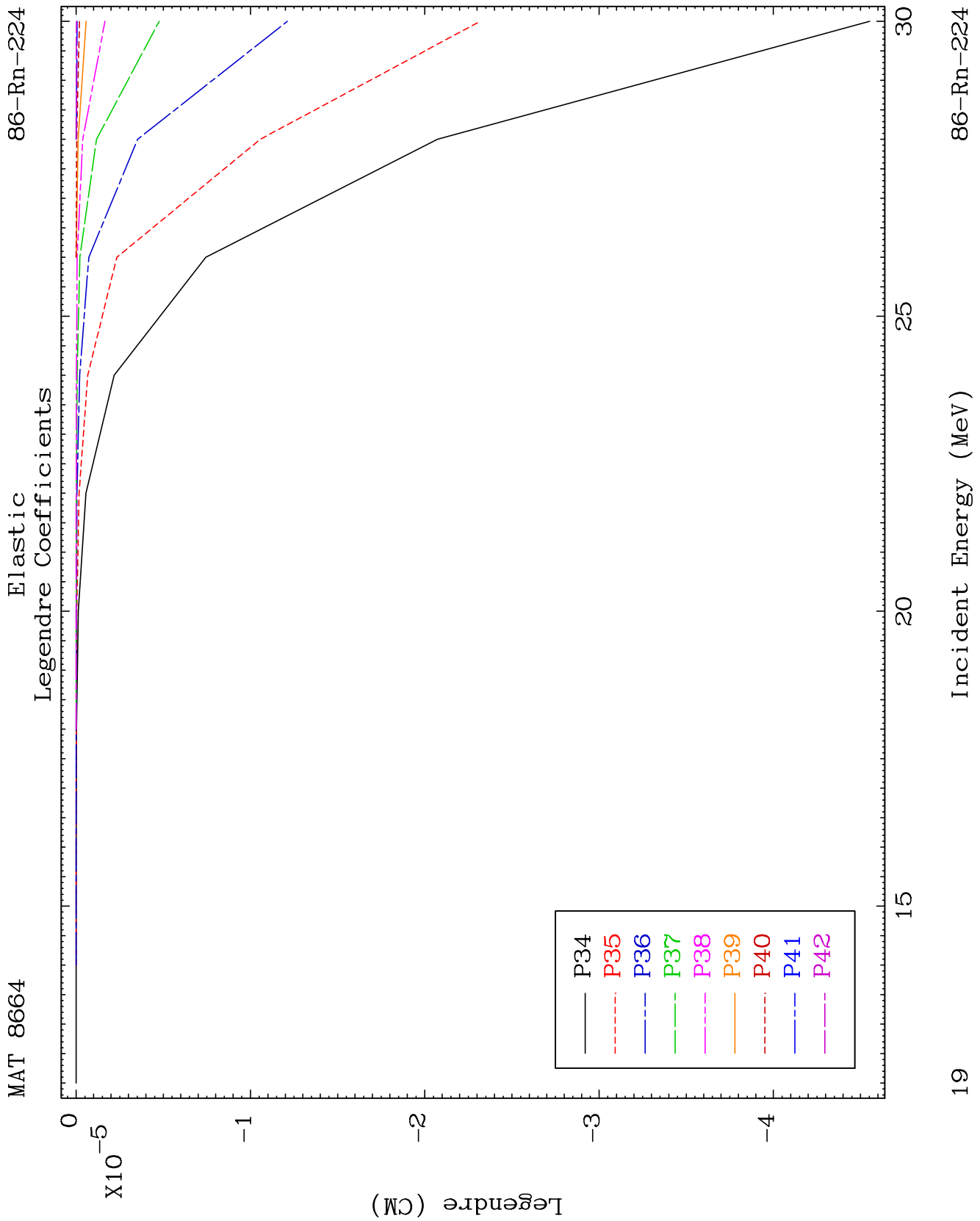
86-Rn-224



86-Rn-224

Incident Energy (MeV)

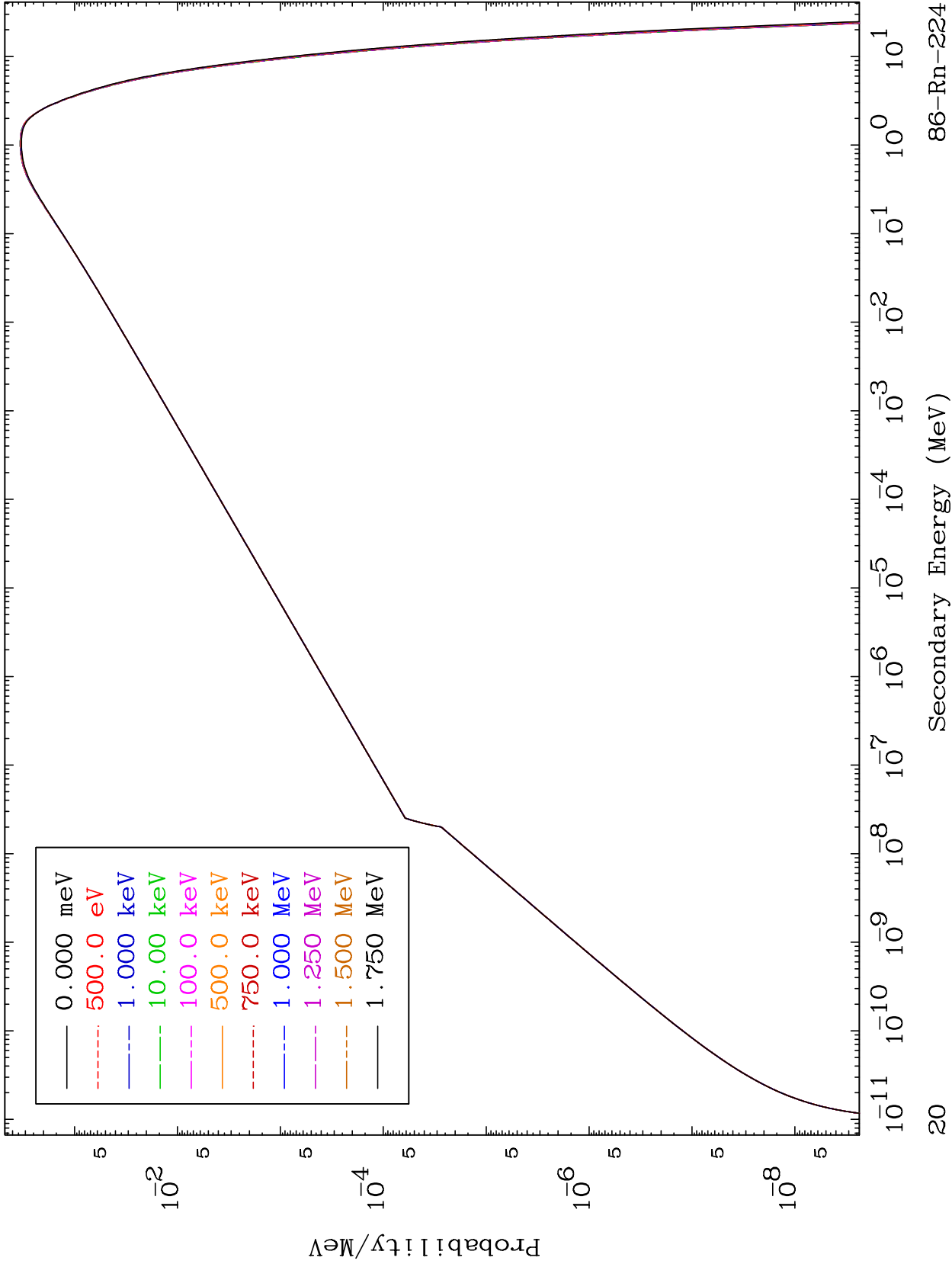
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Fission Energy Distribution

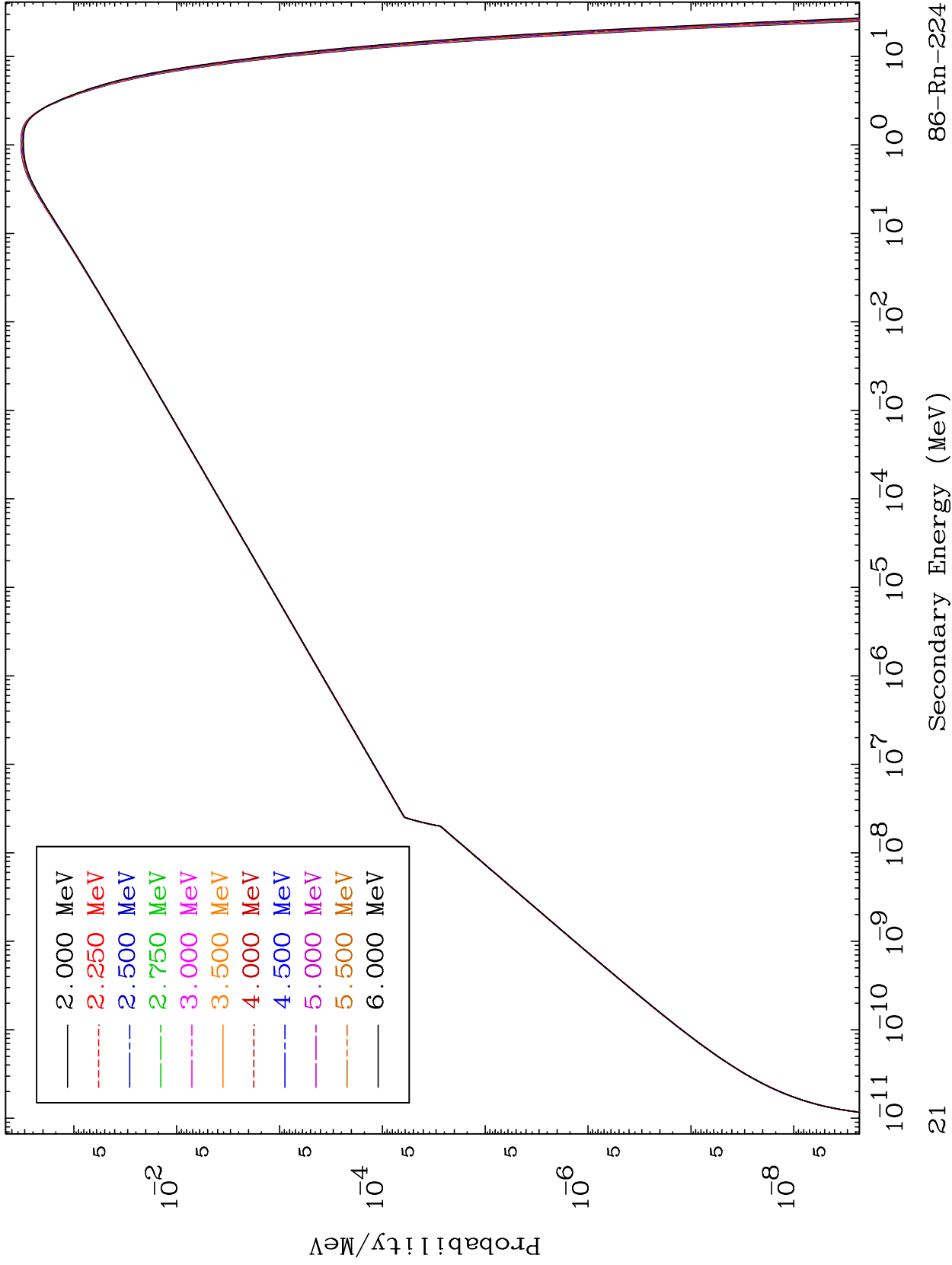
86-Rn-224



MAT 8664

Fission Energy Distribution

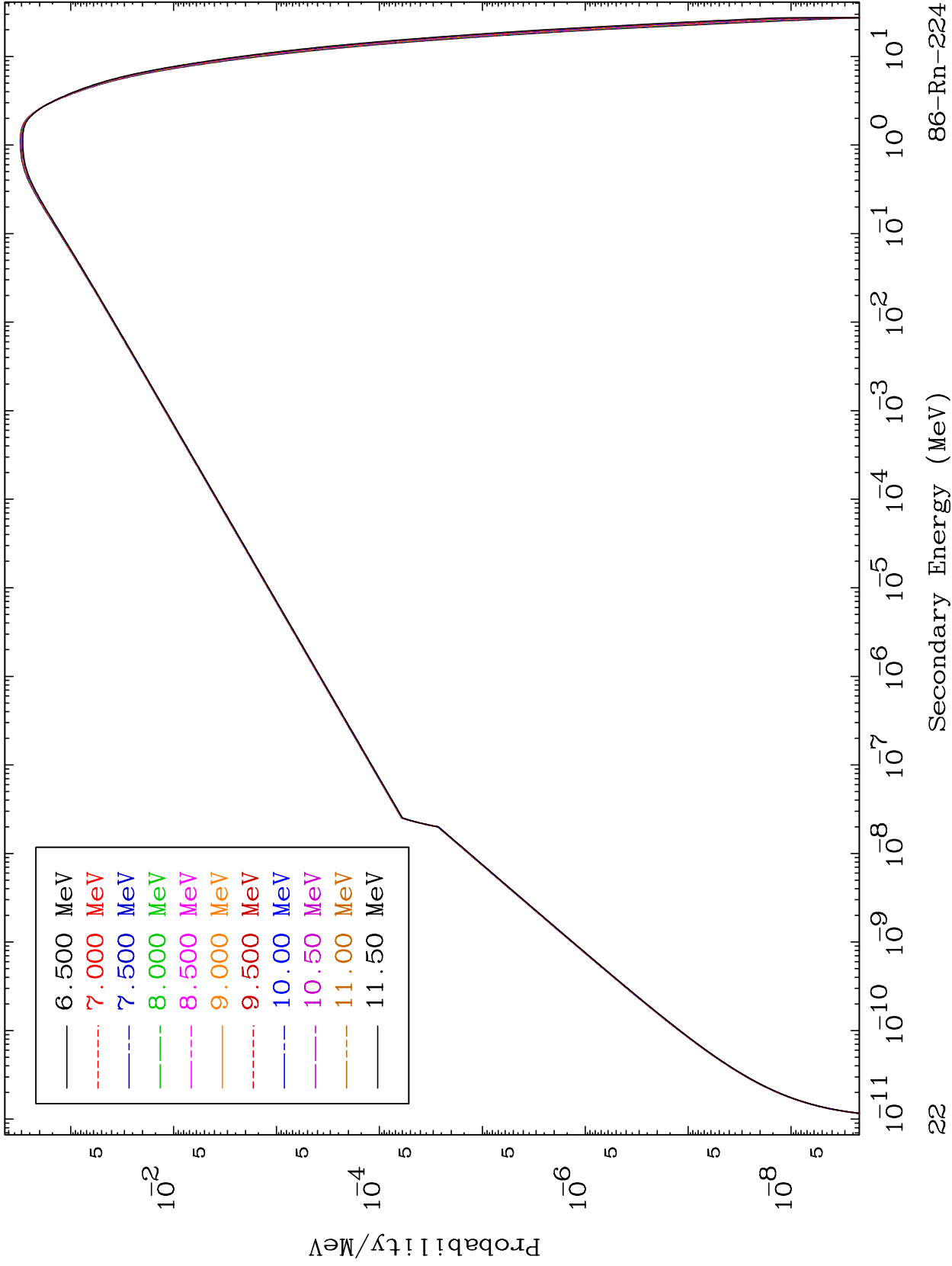
86-Rn-224



MAT 8664

Fission Energy Distribution

86-Rn-224



86-Rn-224

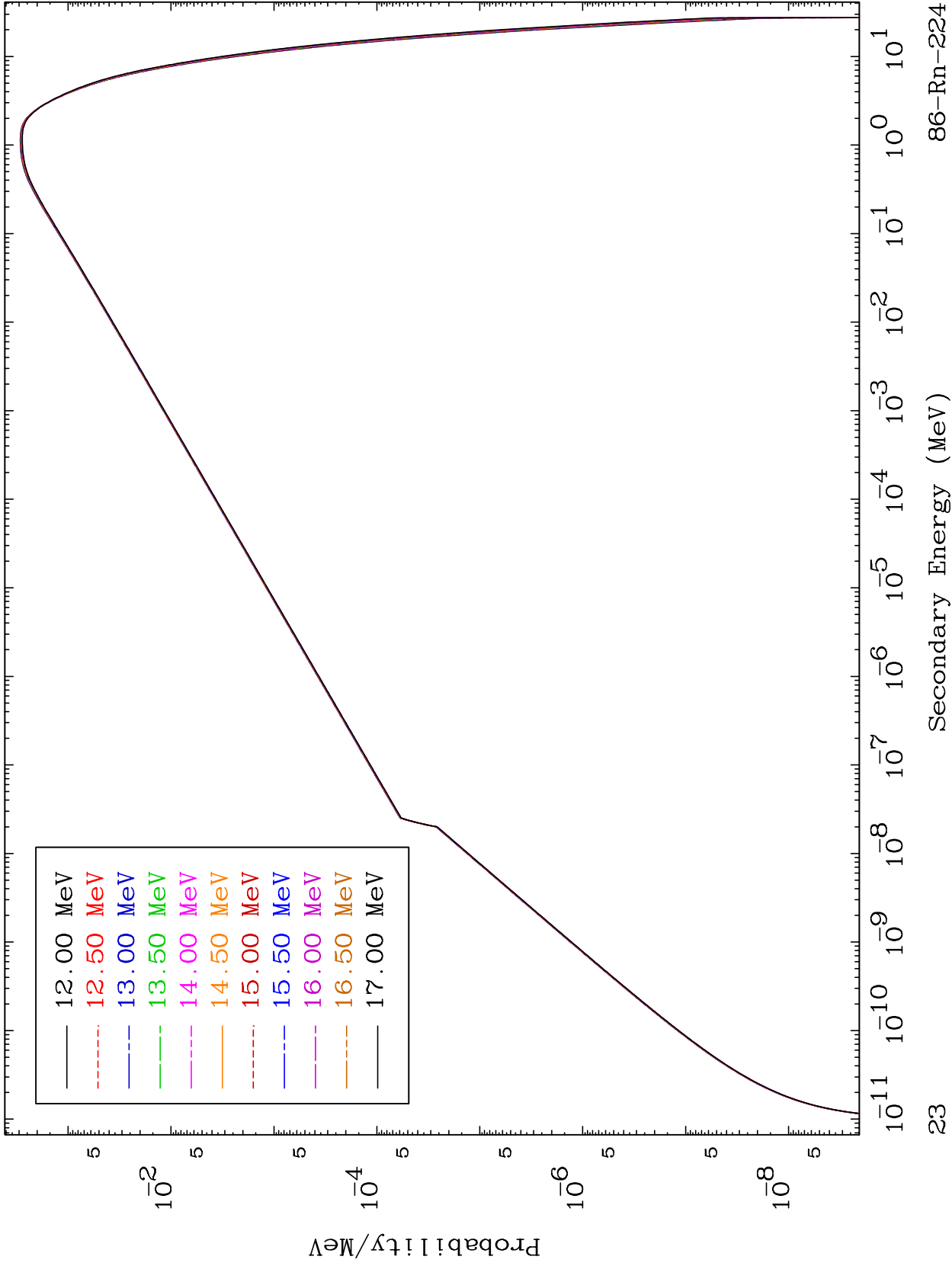
Secondary Energy (MeV)

22

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Fission Energy Distribution

86-Rn-224



86-Rn-224

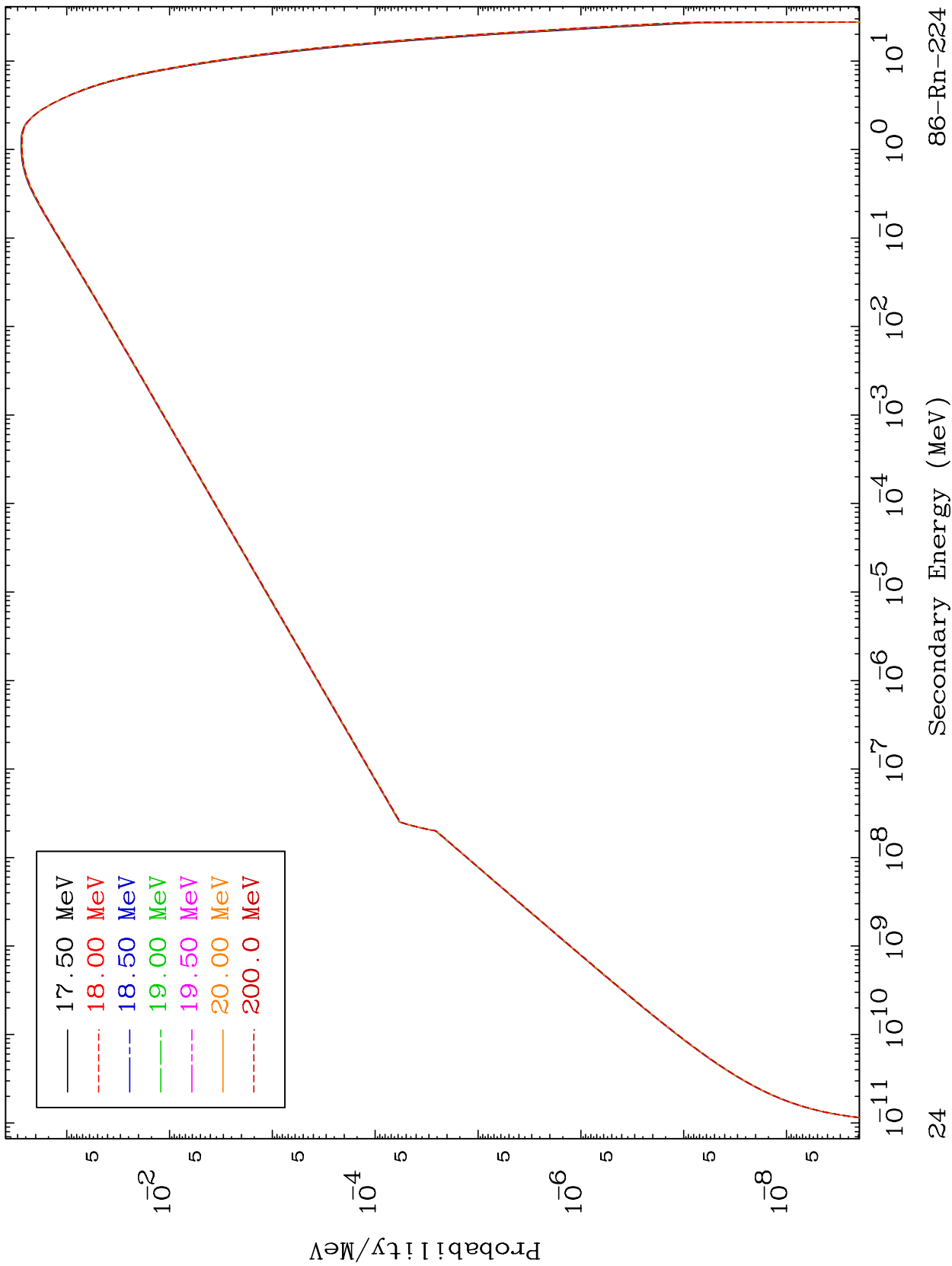
Secondary Energy (MeV)

23

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Fission Energy Distribution

86-Rn-224



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