

Program EVALPLOT
(Version 2018-1)

by

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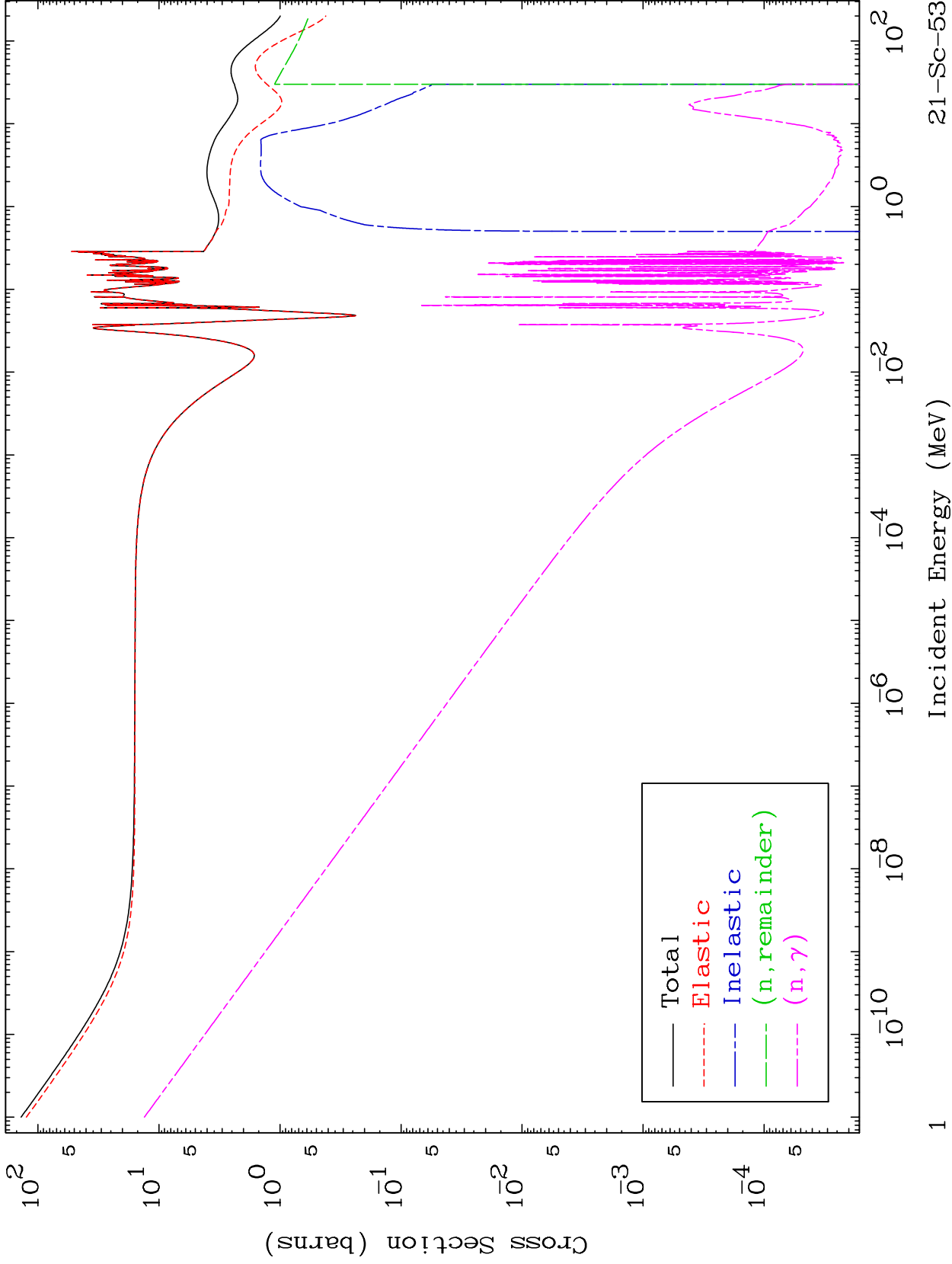
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 2149

Major
293 Kelvin Cross Sections

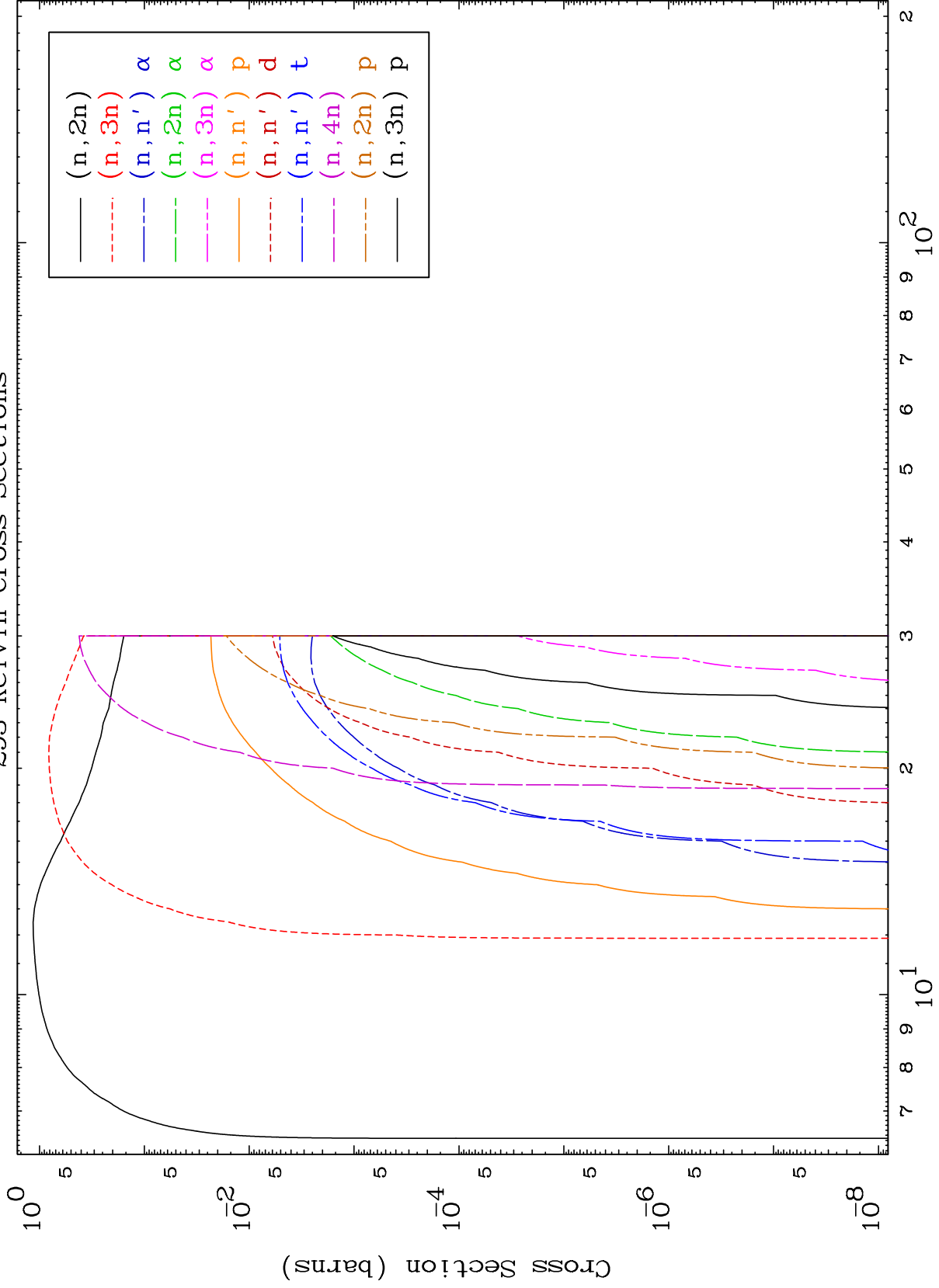
21-Sc-53

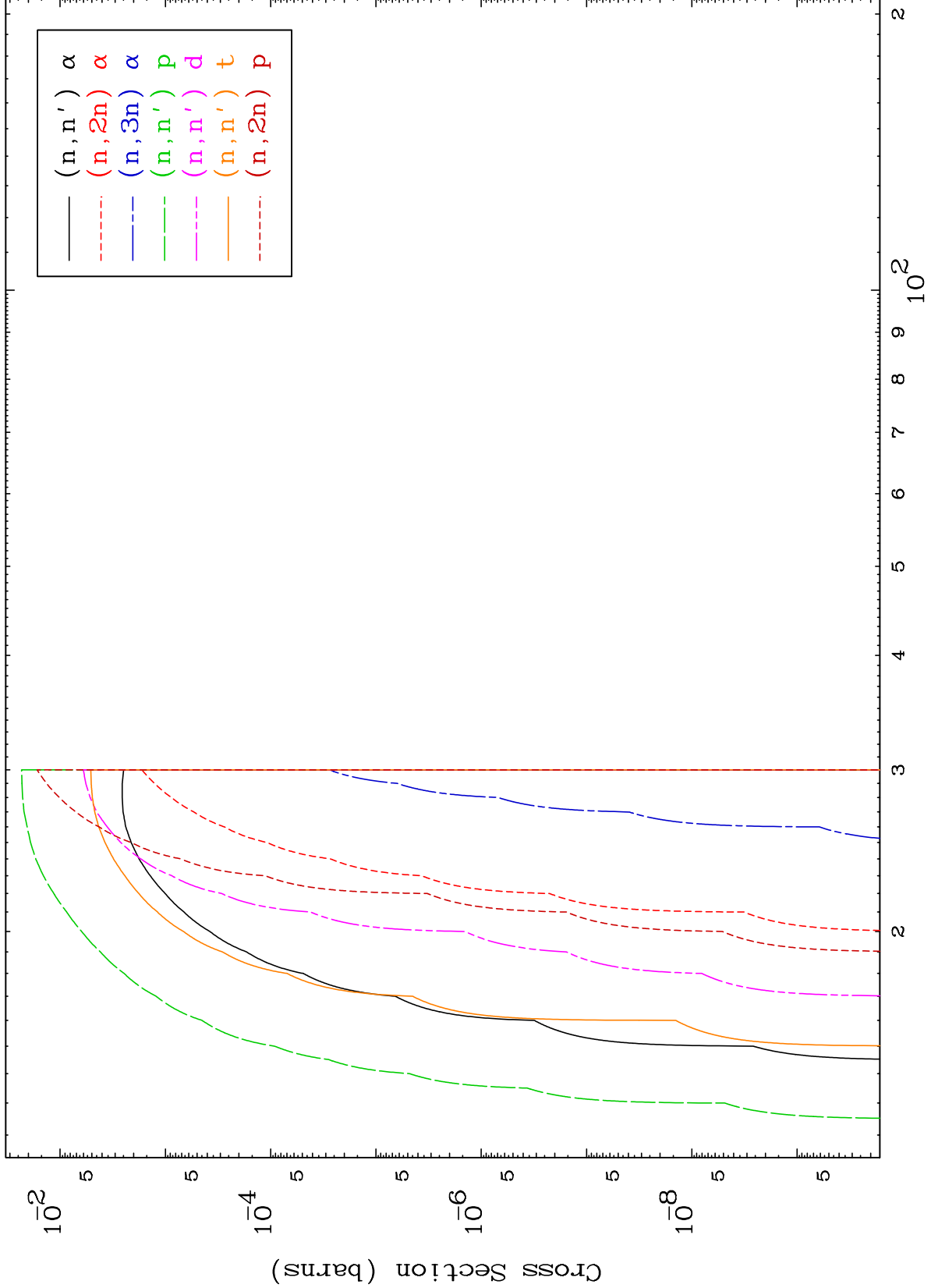


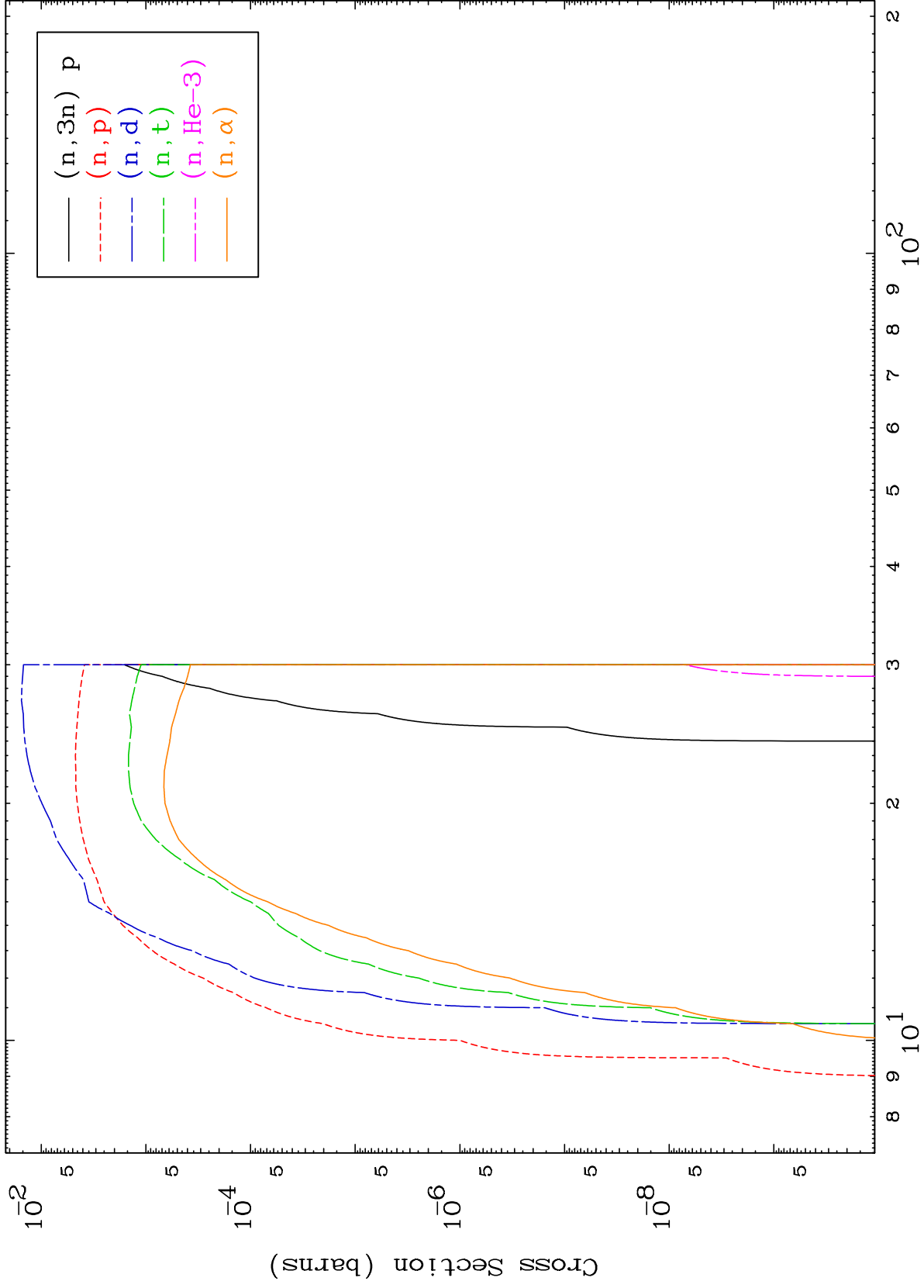
MAT 2149

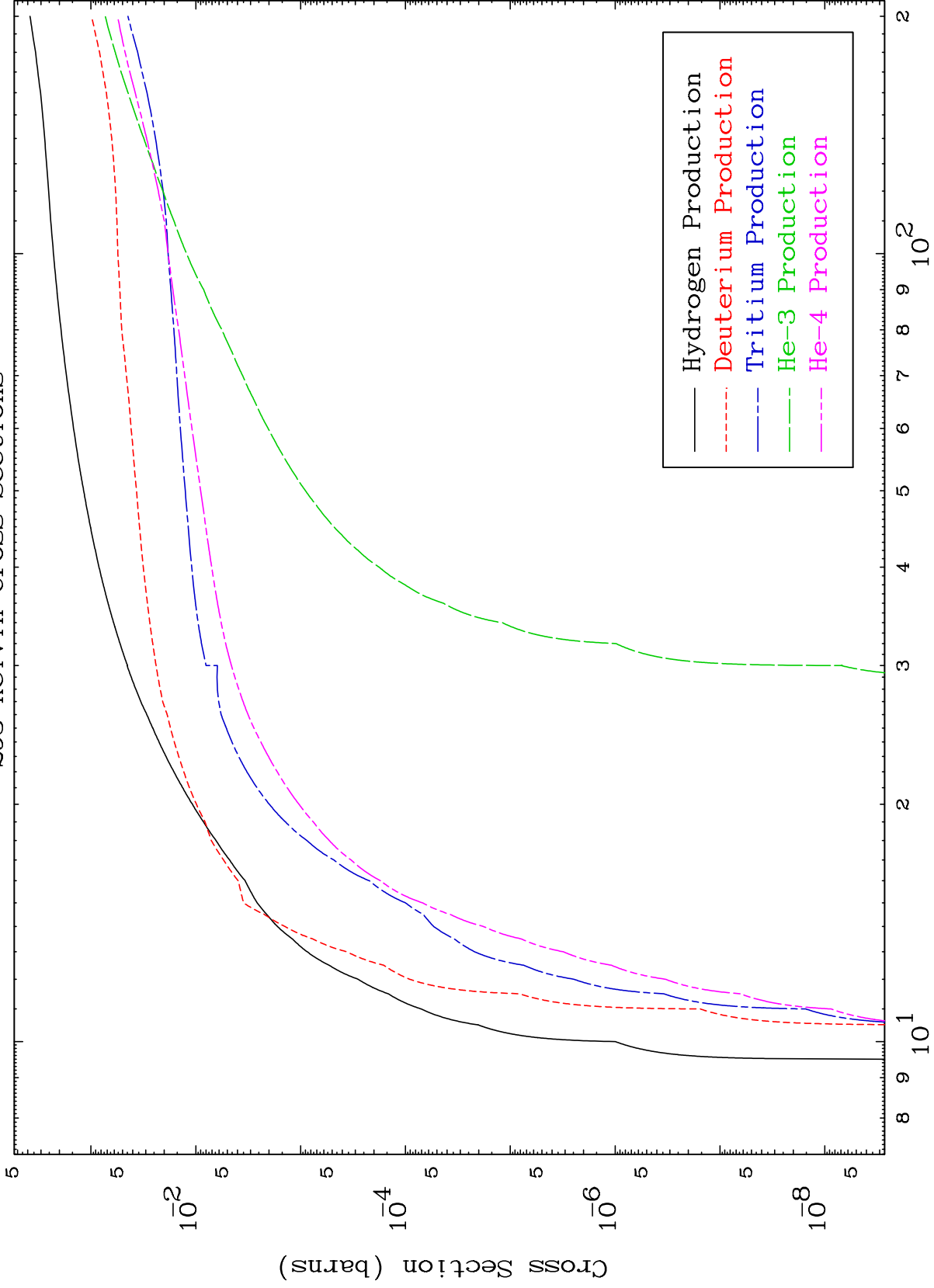
Neutron Production
293 Kelvin Cross Sections

21-Sc-53







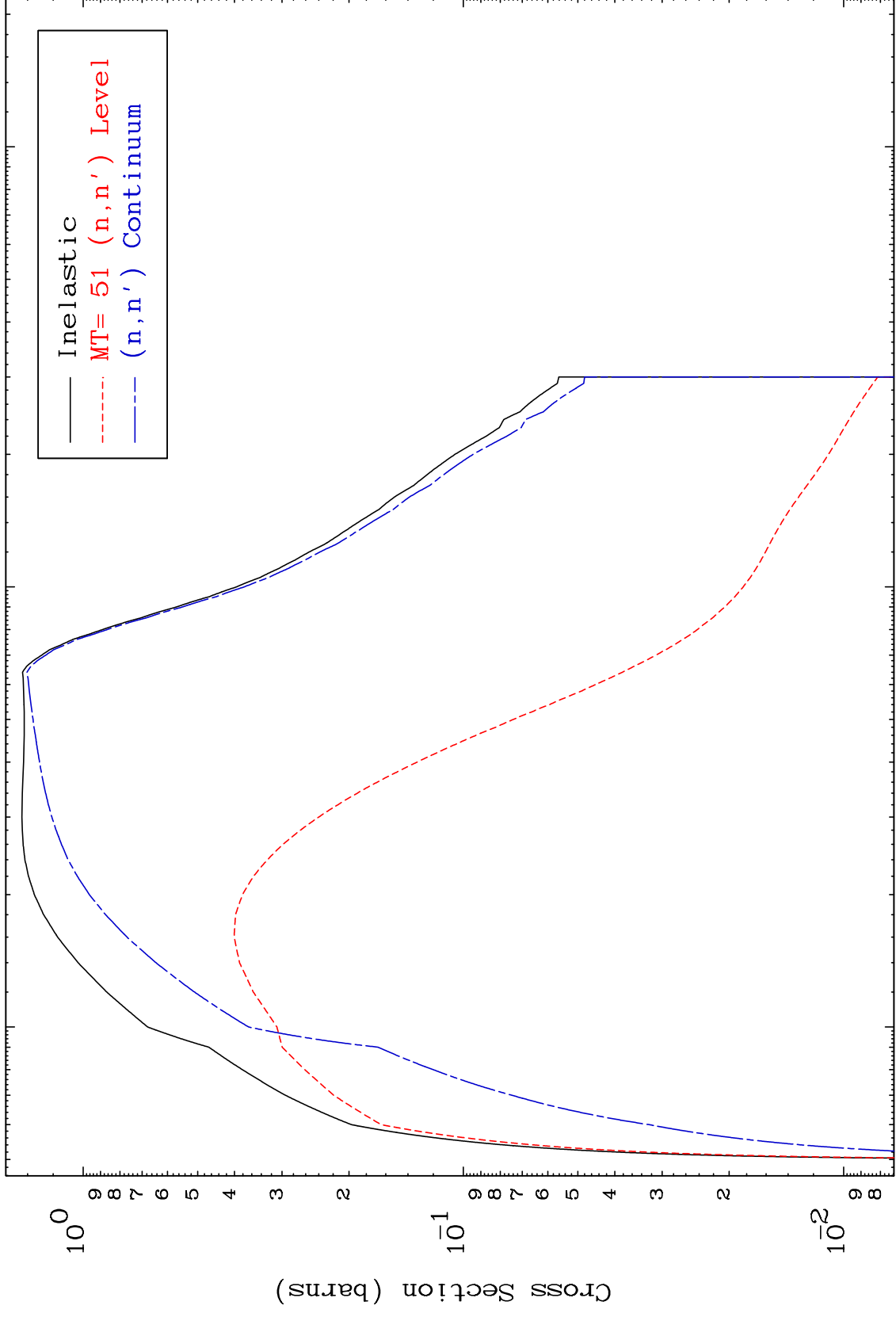


MAT 2149

(n,n') Level

21-Sc-53

293 Kelvin Cross Sections



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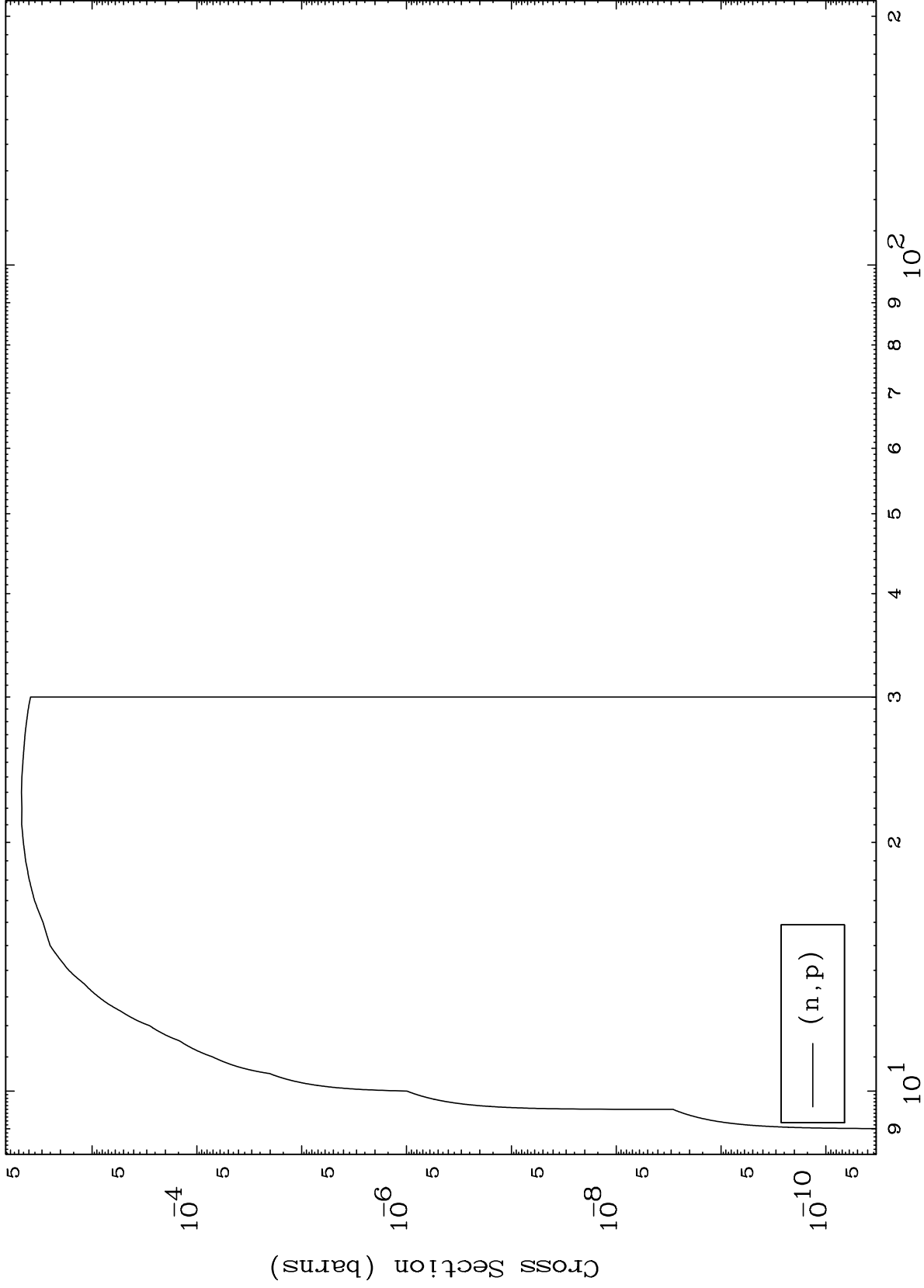
Incident Energy (MeV)

21-Sc-53

MAT 2149

(n,p) Levels
293 Kelvin Cross Sections

21-Sc-53



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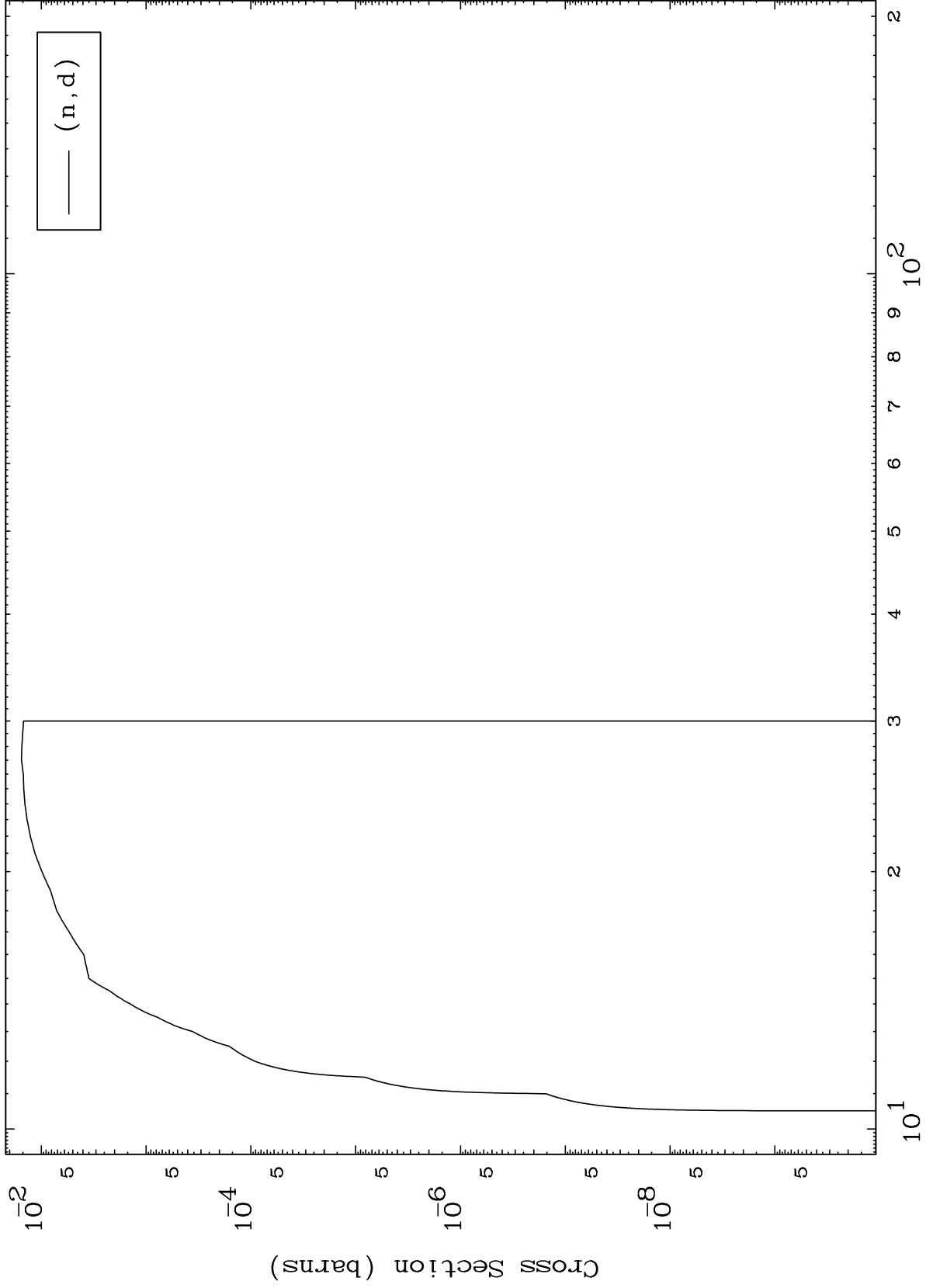
Incident Energy (MeV)

21-Sc-53

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(n,d) Levels
293 Kelvin Cross Sections

21-Sc-53



(n,d)

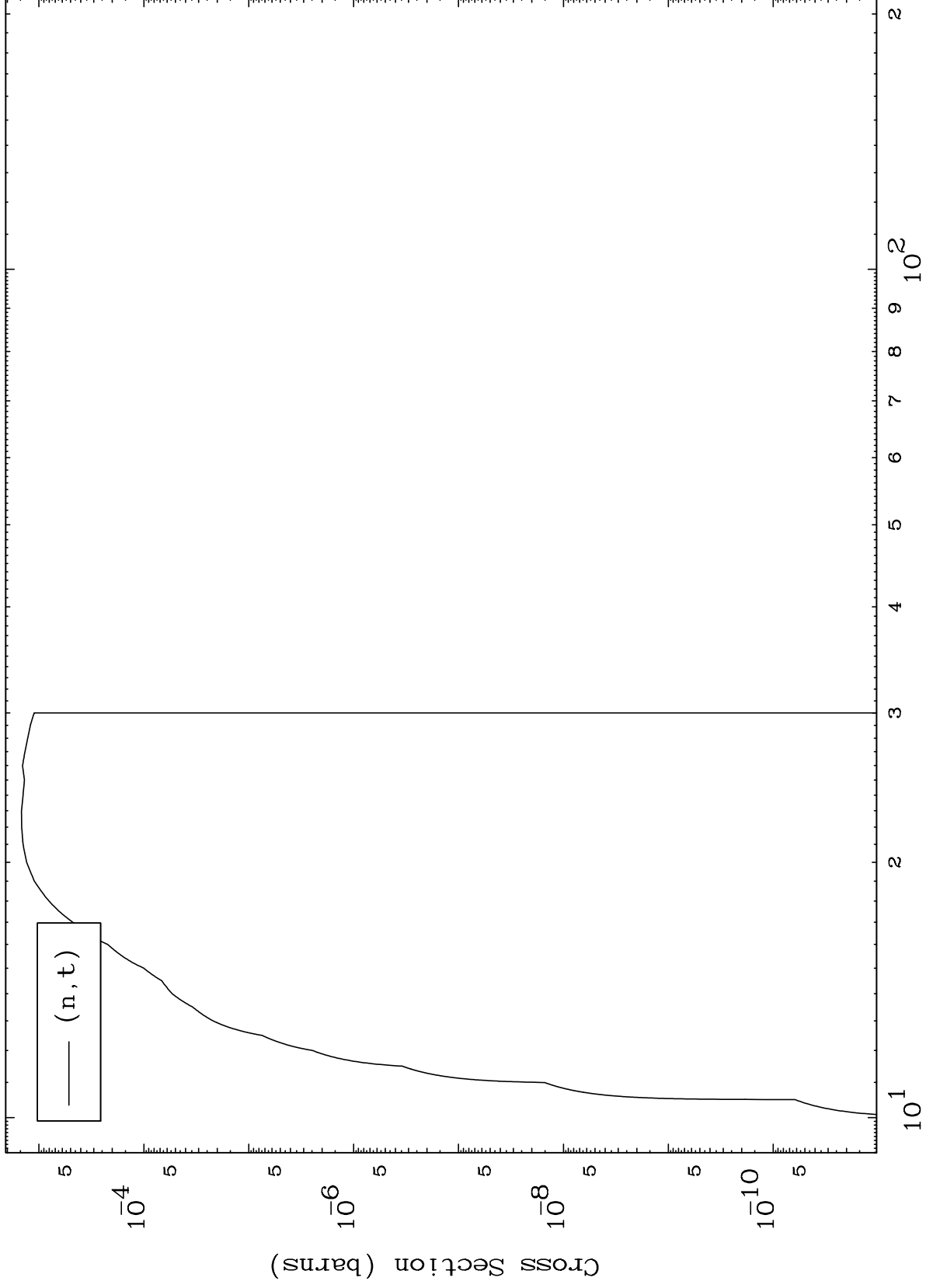
Incident Energy (MeV)

21-Sc-53

MAT 2149

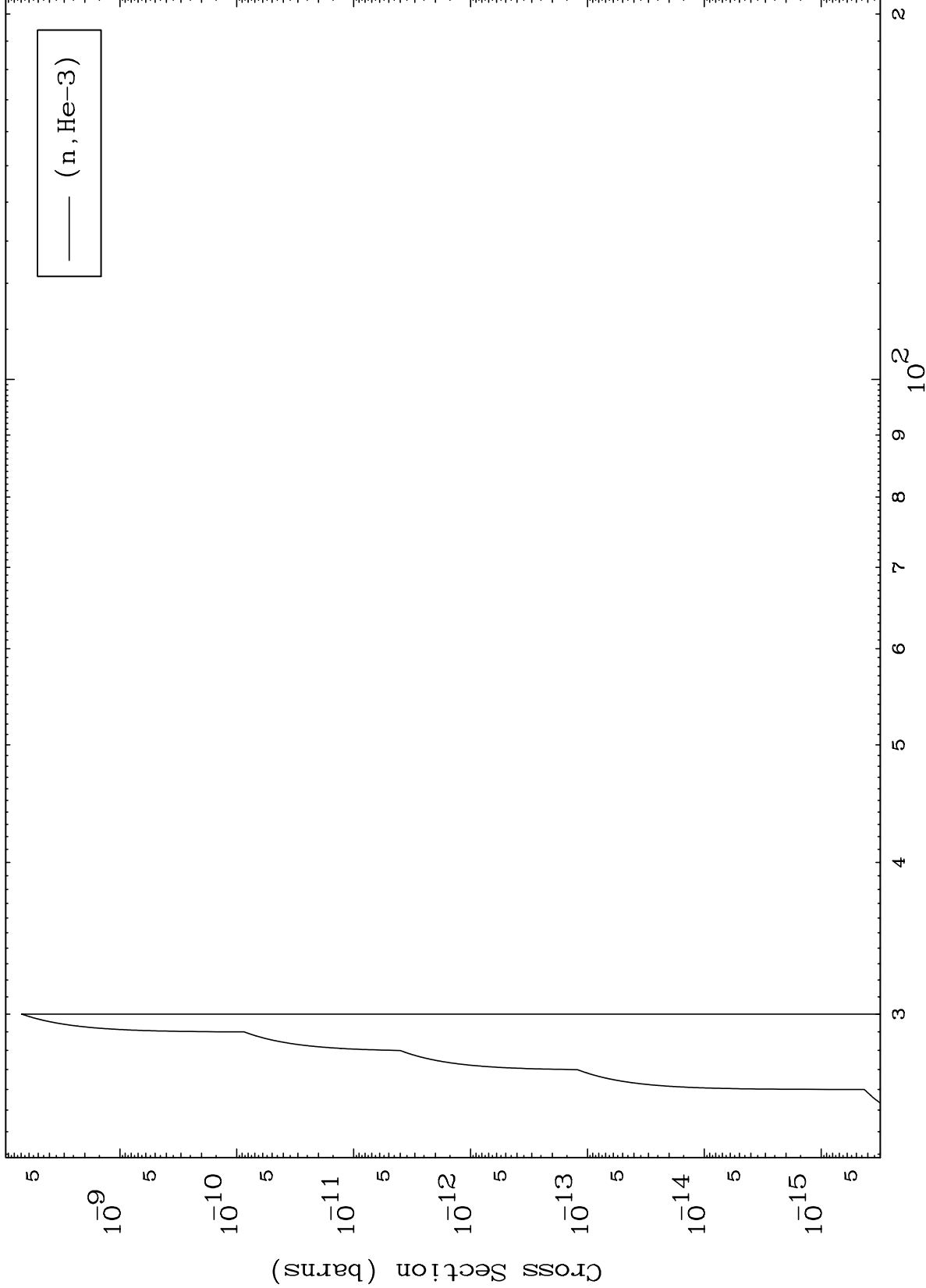
(n,t) Levels
293 Kelvin Cross Sections

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Incident Energy (MeV)

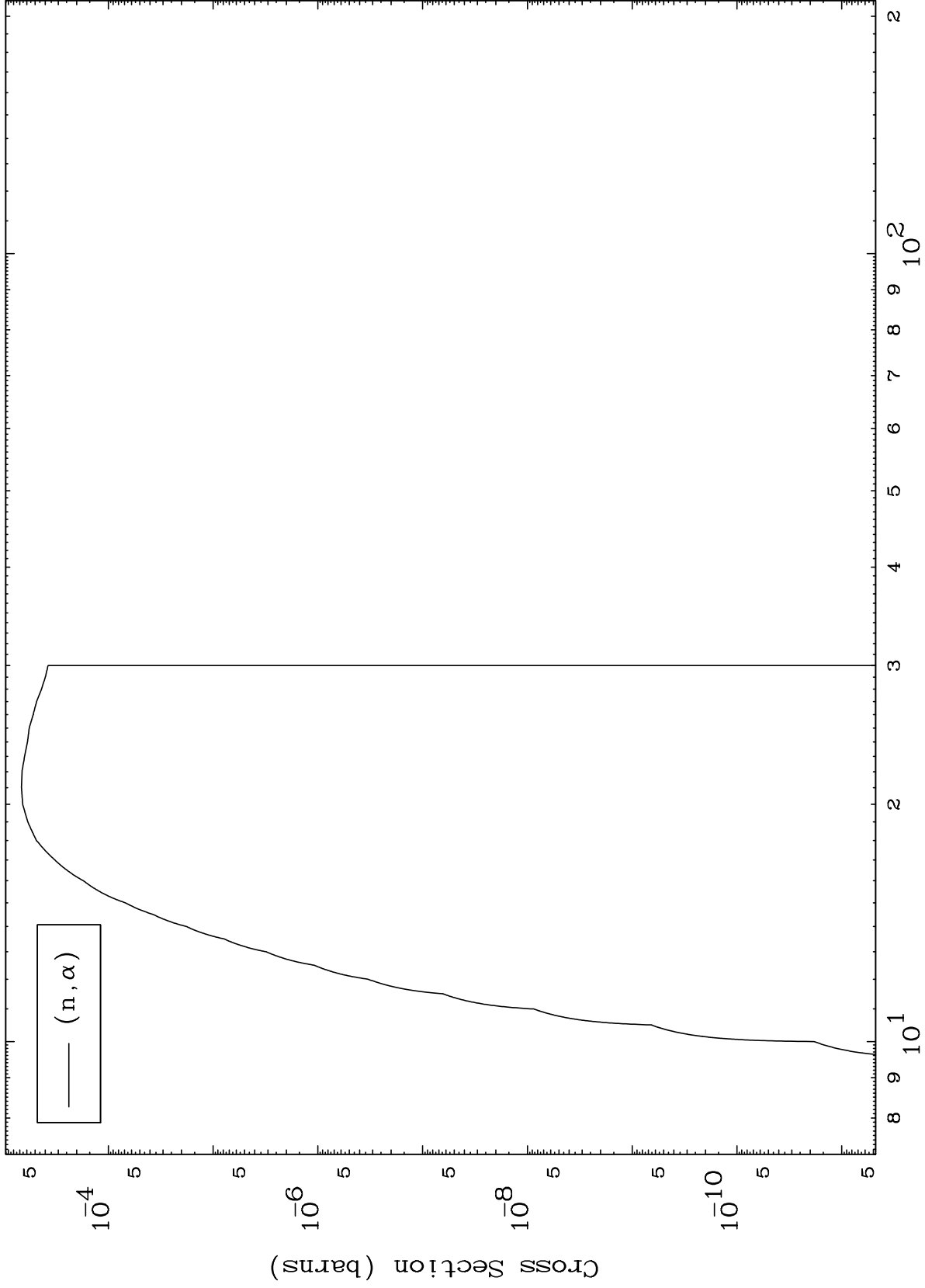
21-Sc-53



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(n, α) Levels
293 Kelvin Cross Sections

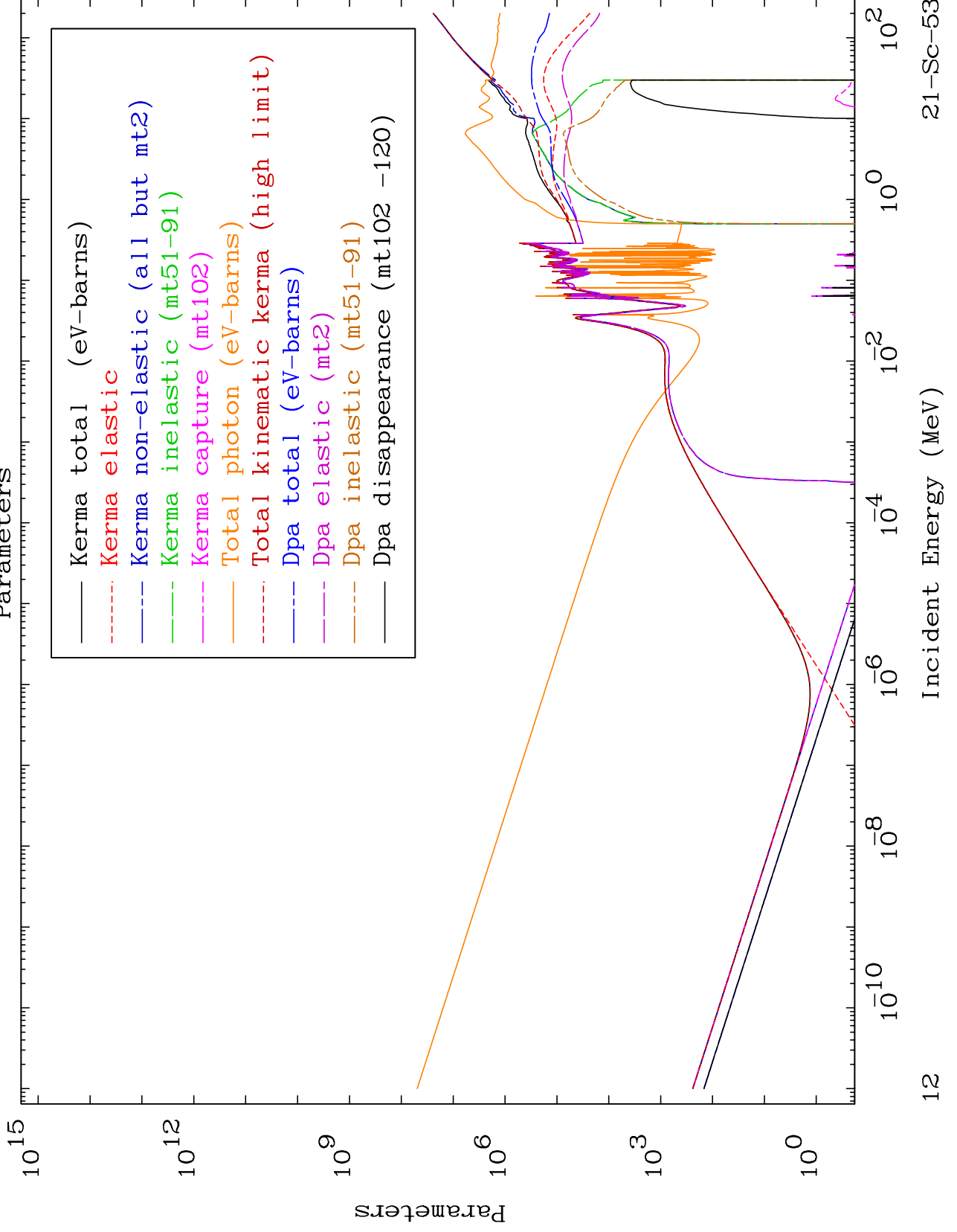
21-Sc-53



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Incident Energy (MeV)

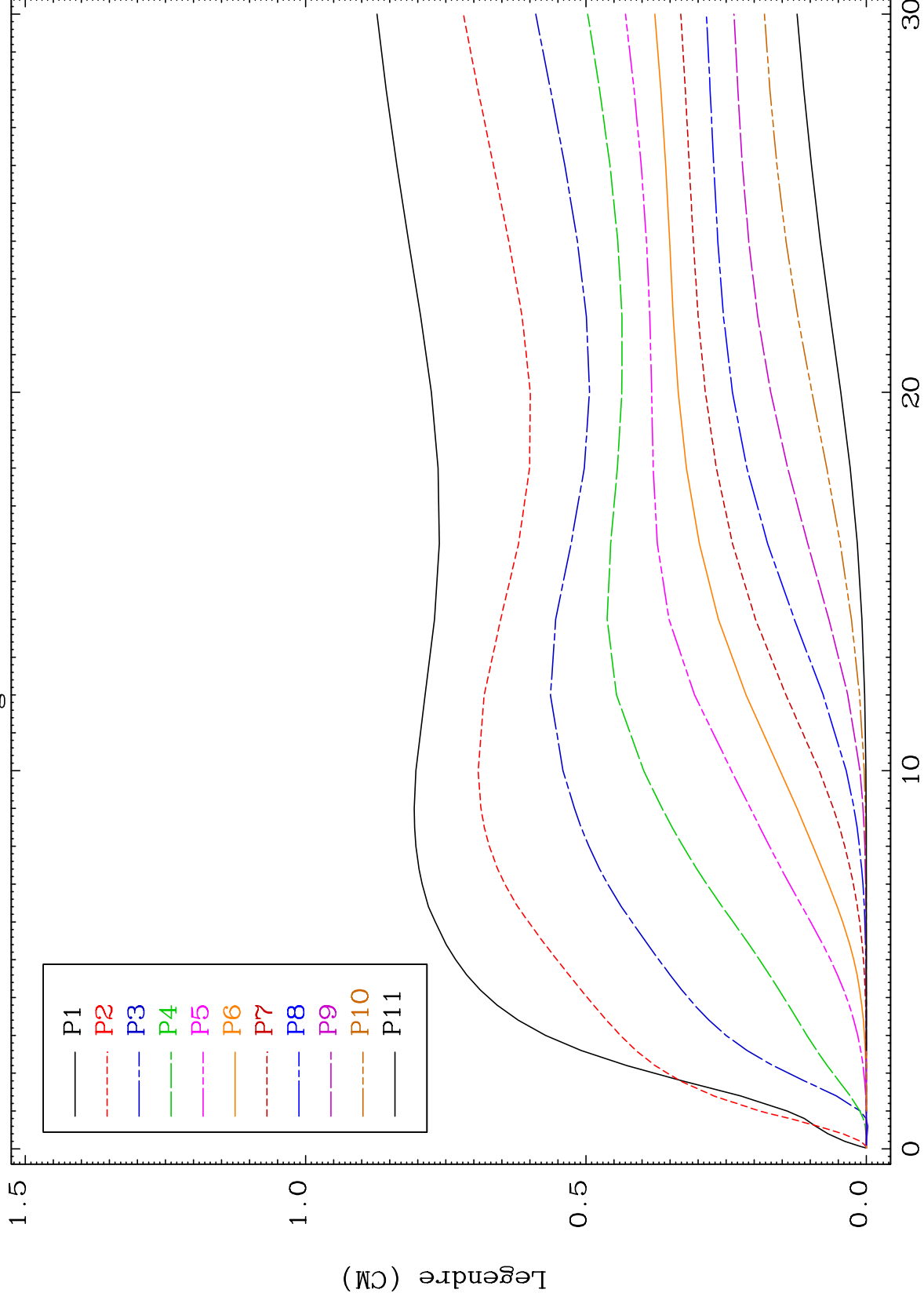
21-Sc-53



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Elastic Legendre Coefficients

21-Sc-53



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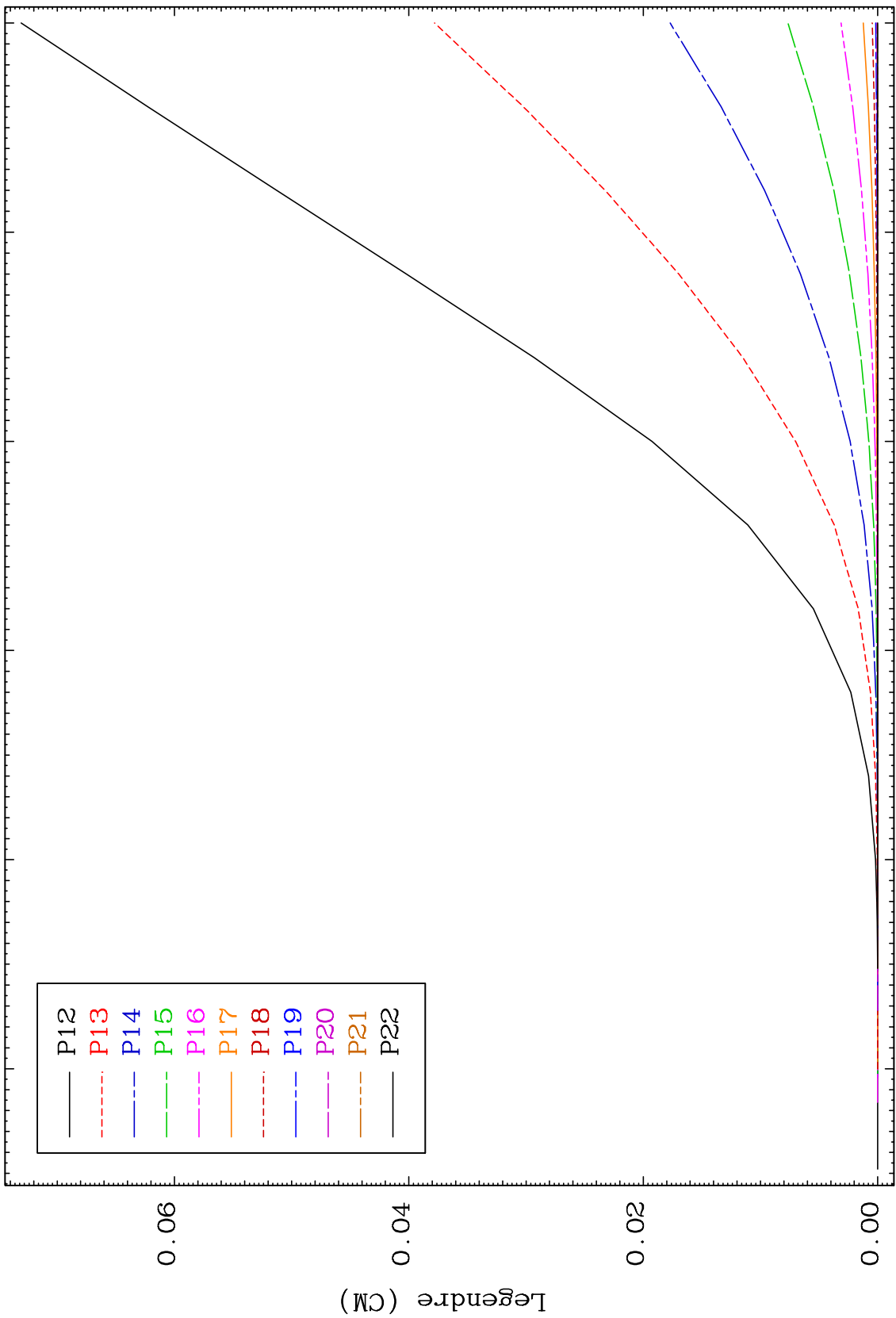
Incident Energy (MeV)

21-Sc-53

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Elastic
Legendre Coefficients

21-Sc-53



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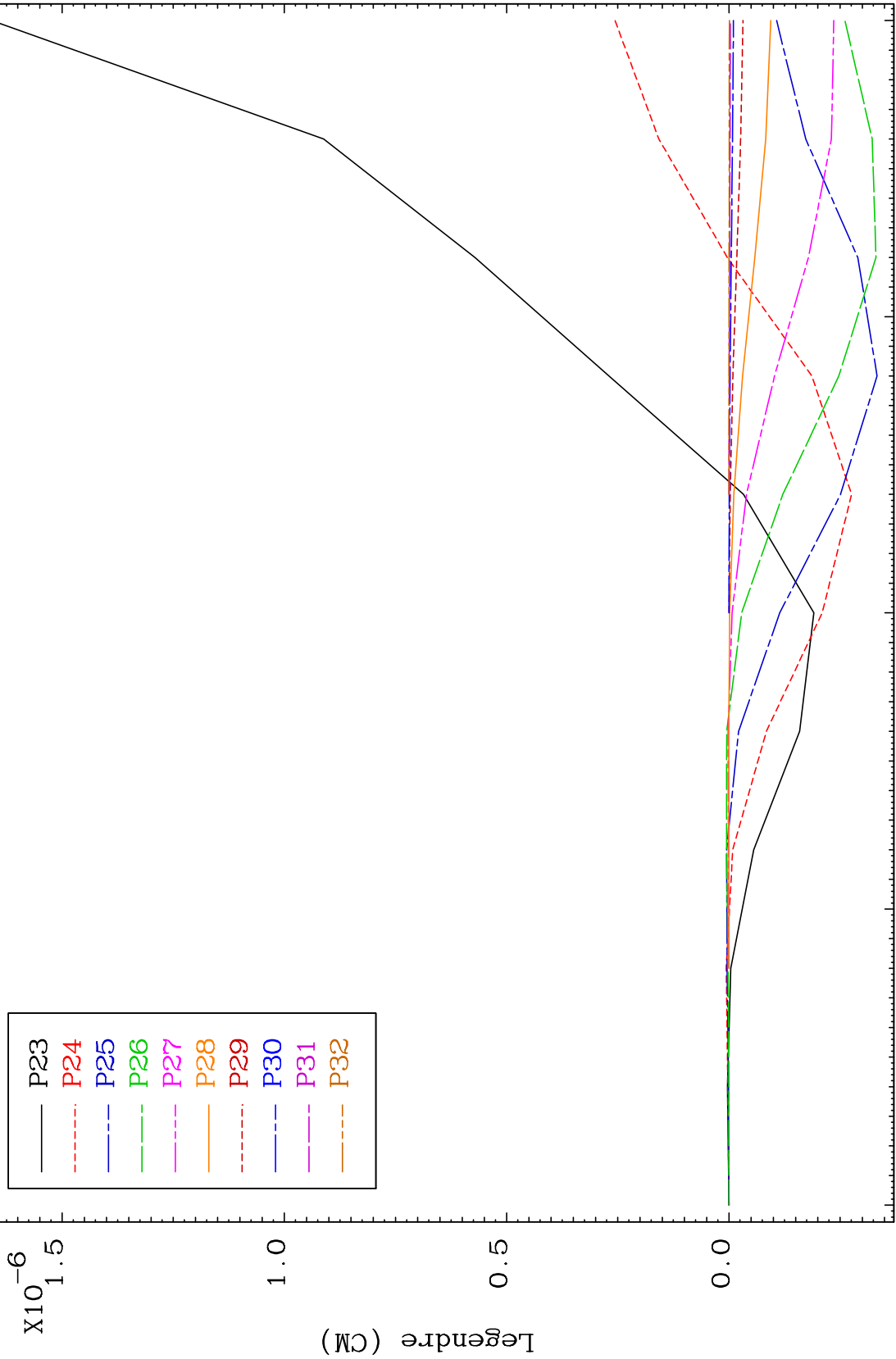
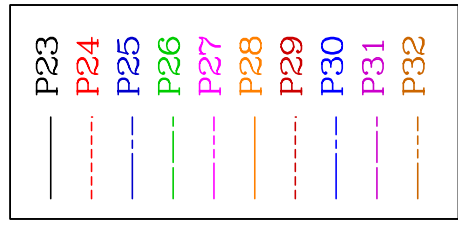
Incident Energy (MeV)

21-Sc-53

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Elastic
Legendre Coefficients

21-Sc-53



21-Sc-53

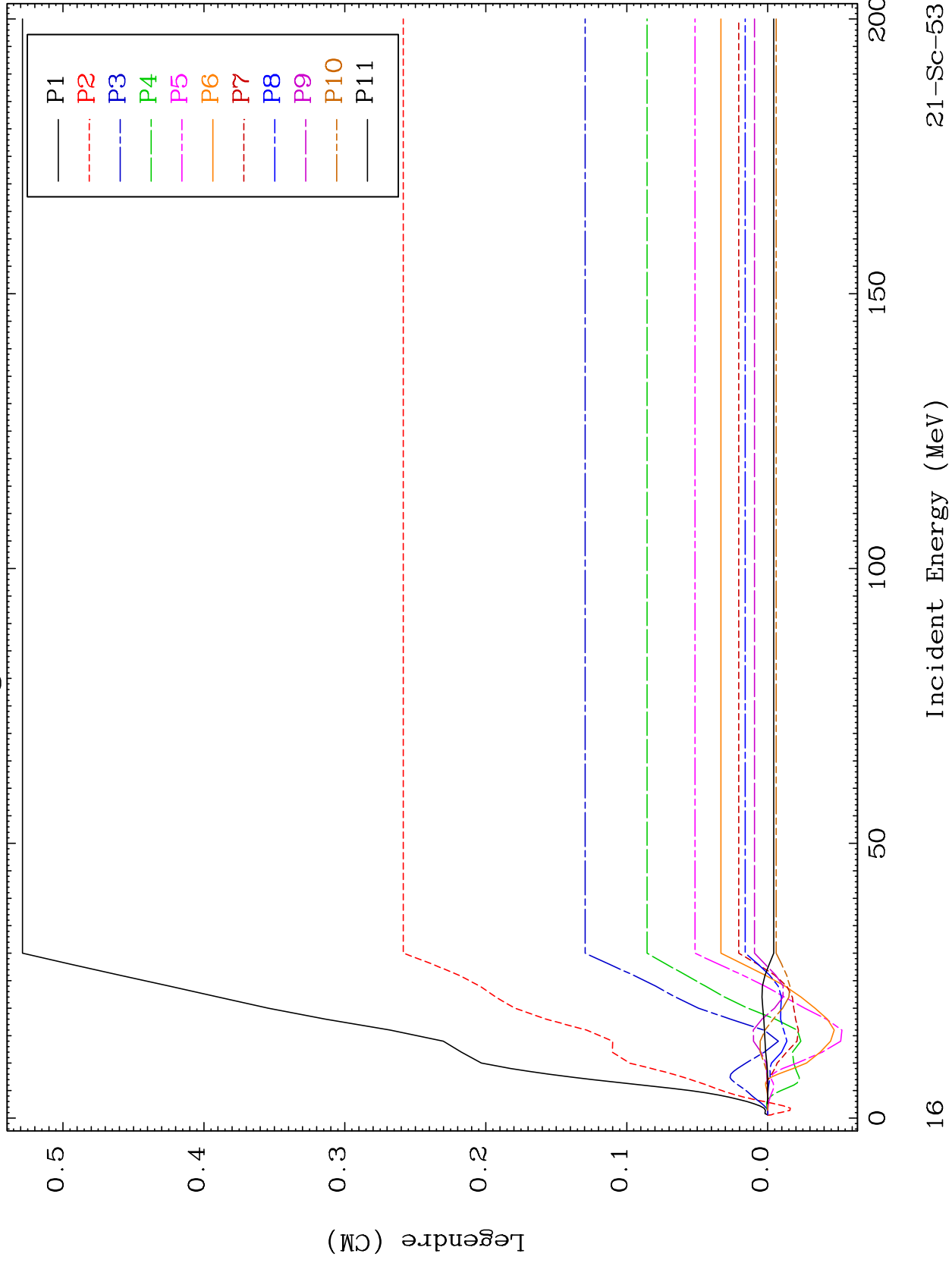
Incident Energy (MeV)

15

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MT= 51 (n,n') Level
Legendre Coefficients

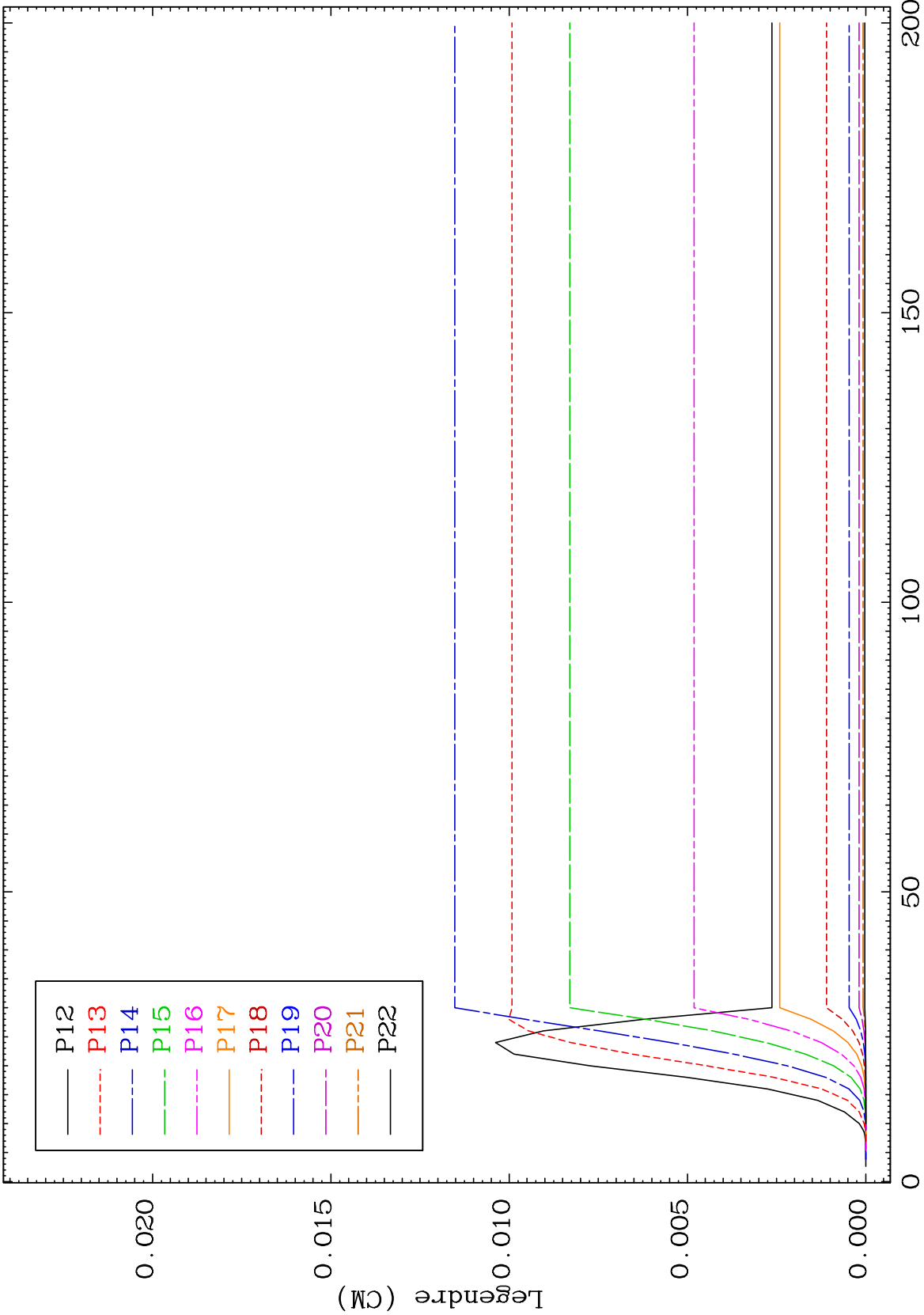
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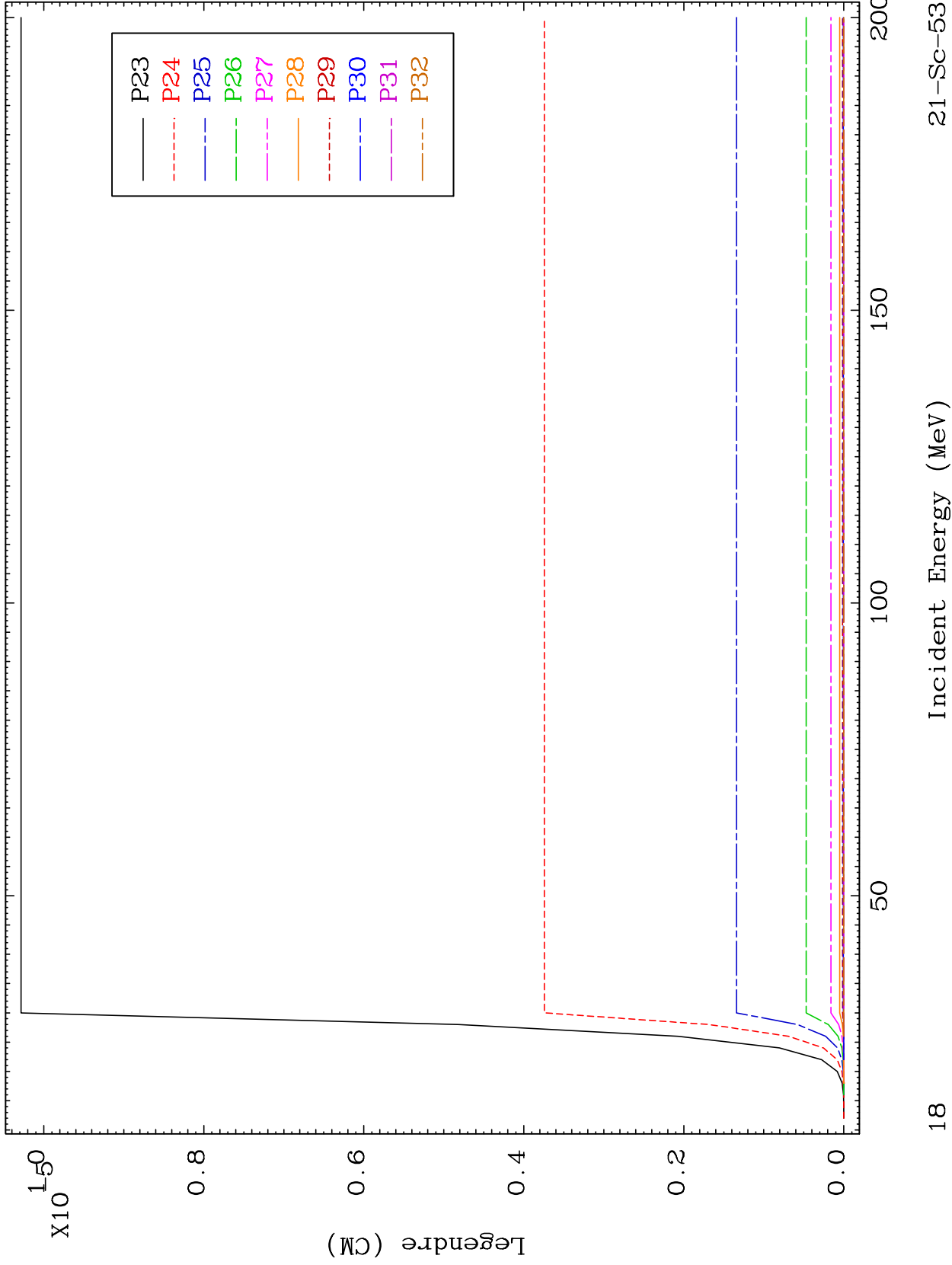


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Incident Energy (MeV)

16

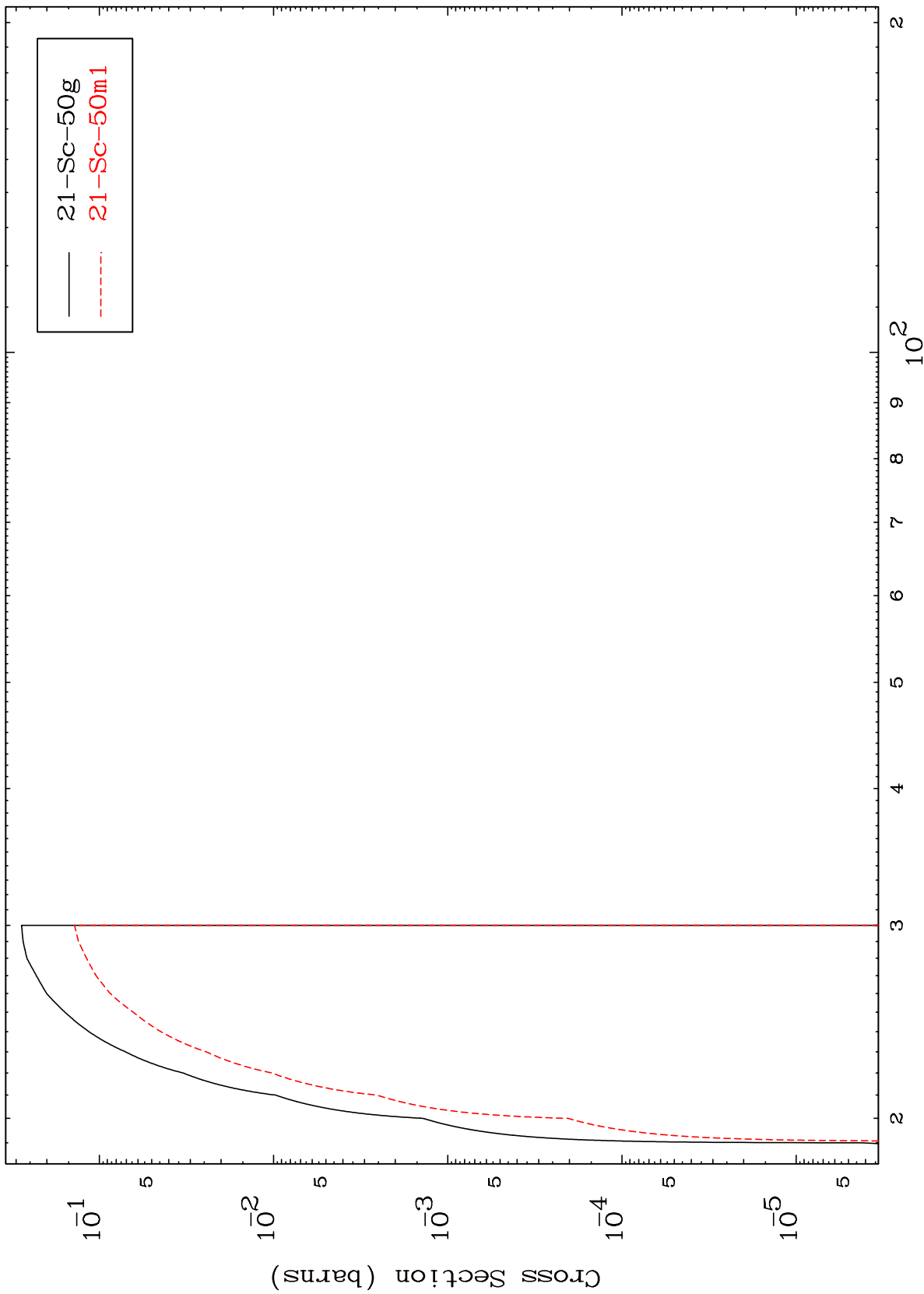




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(n,4n)
Radionuclide Production Cross Section



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21-Sc-53

Incident Energy (MeV)