

Program EVALPLOT  
(Version 2018-1)

by

Dermott E. Cullen  
(Present Contact Information)

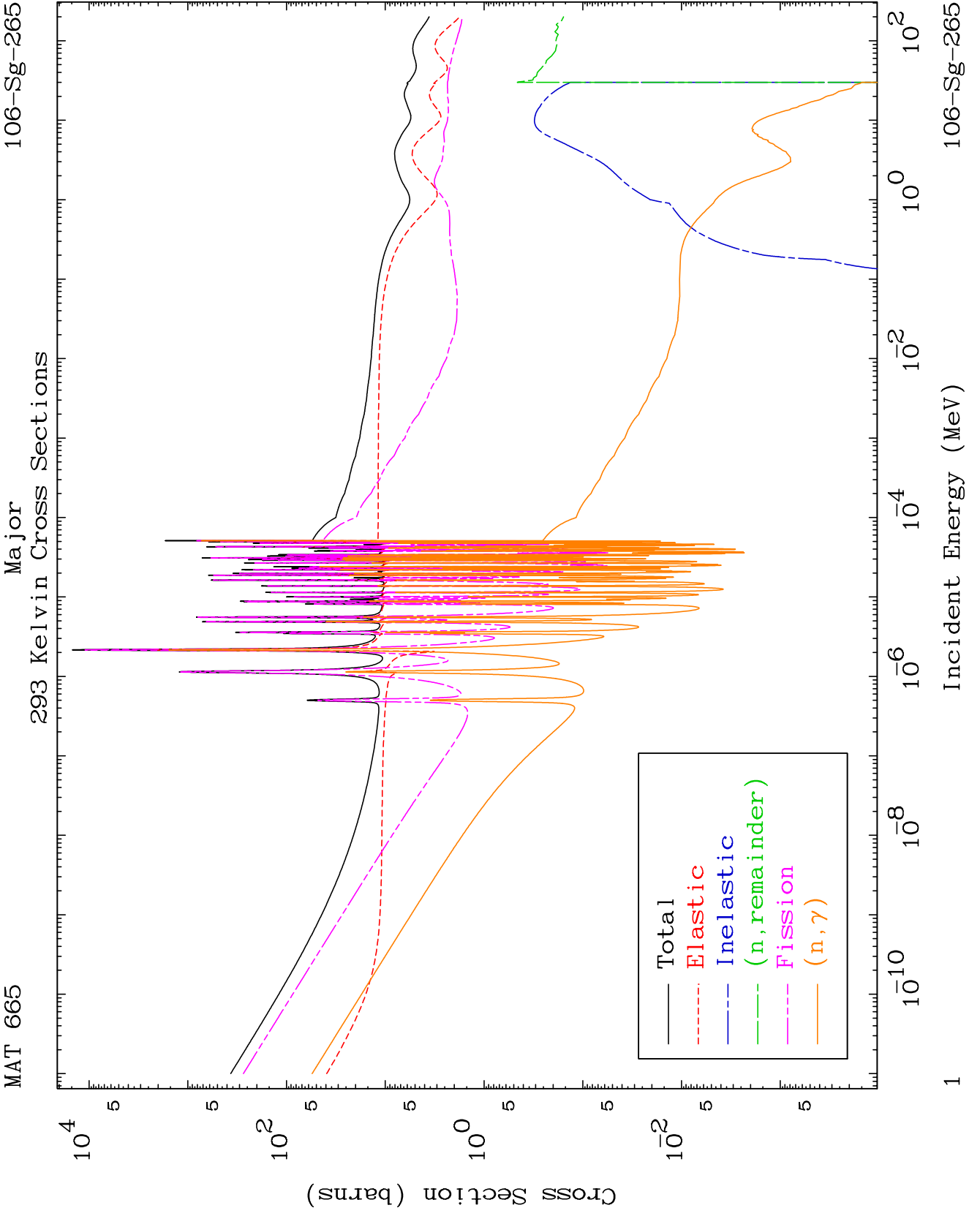
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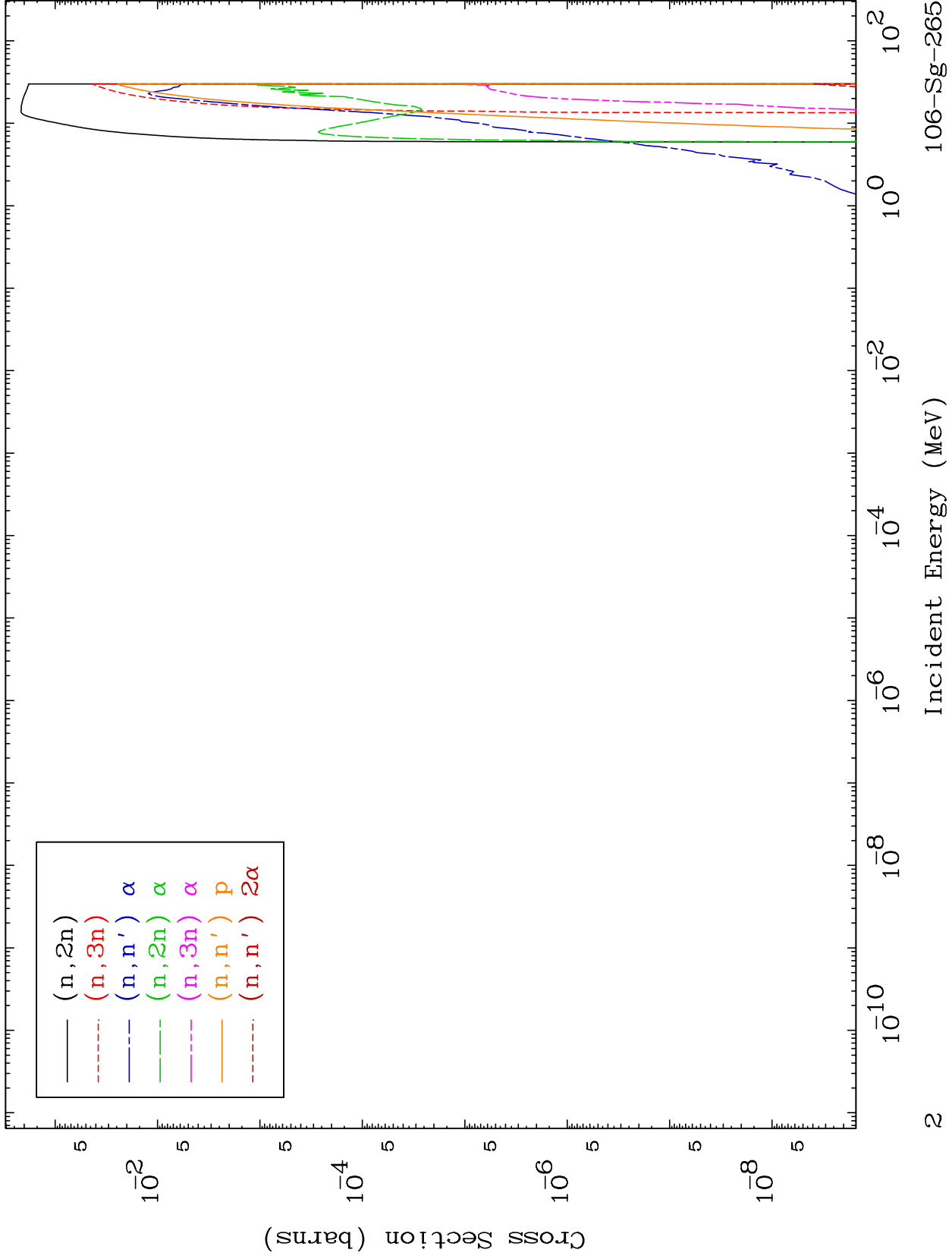
Press Mouse Button to Start

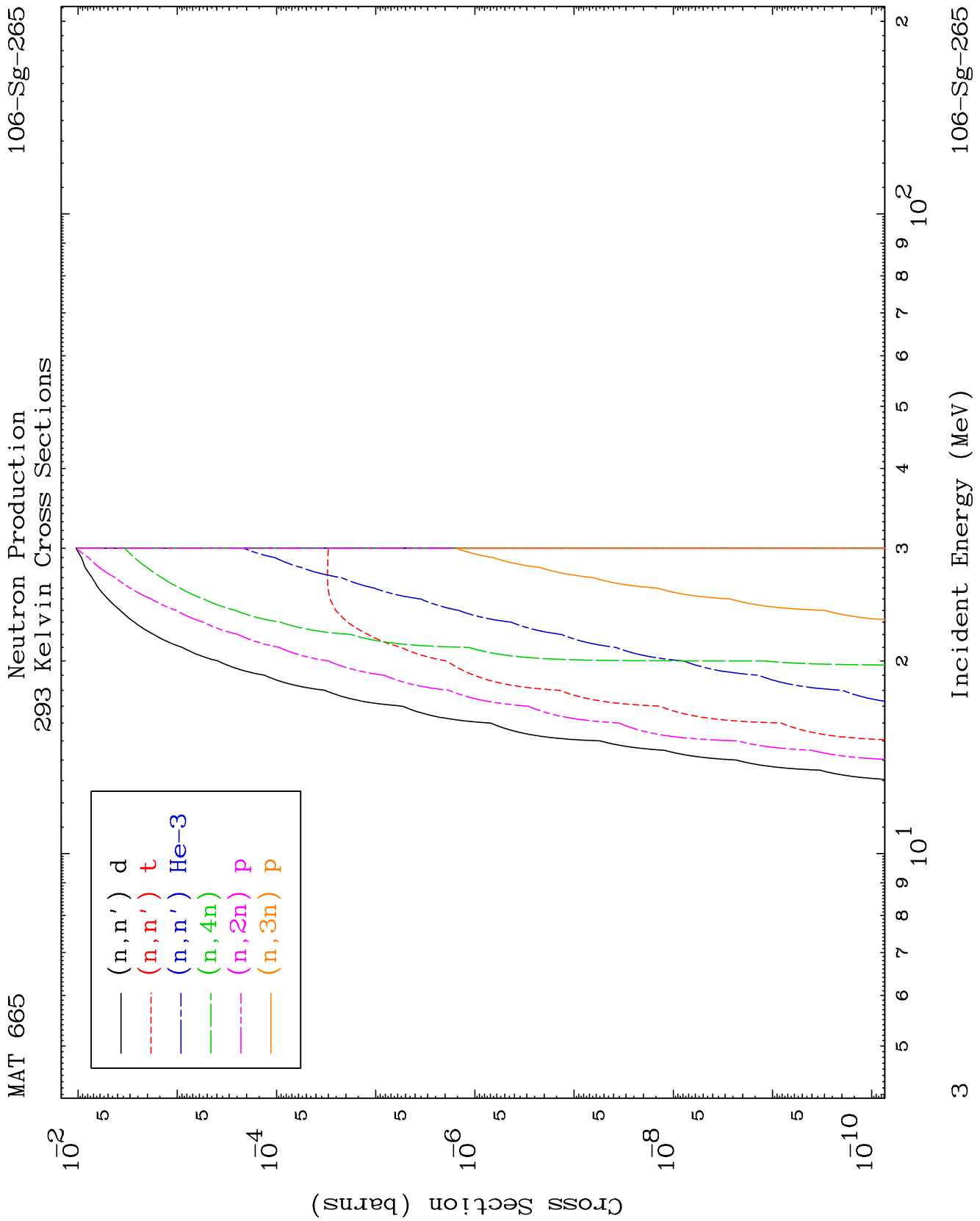


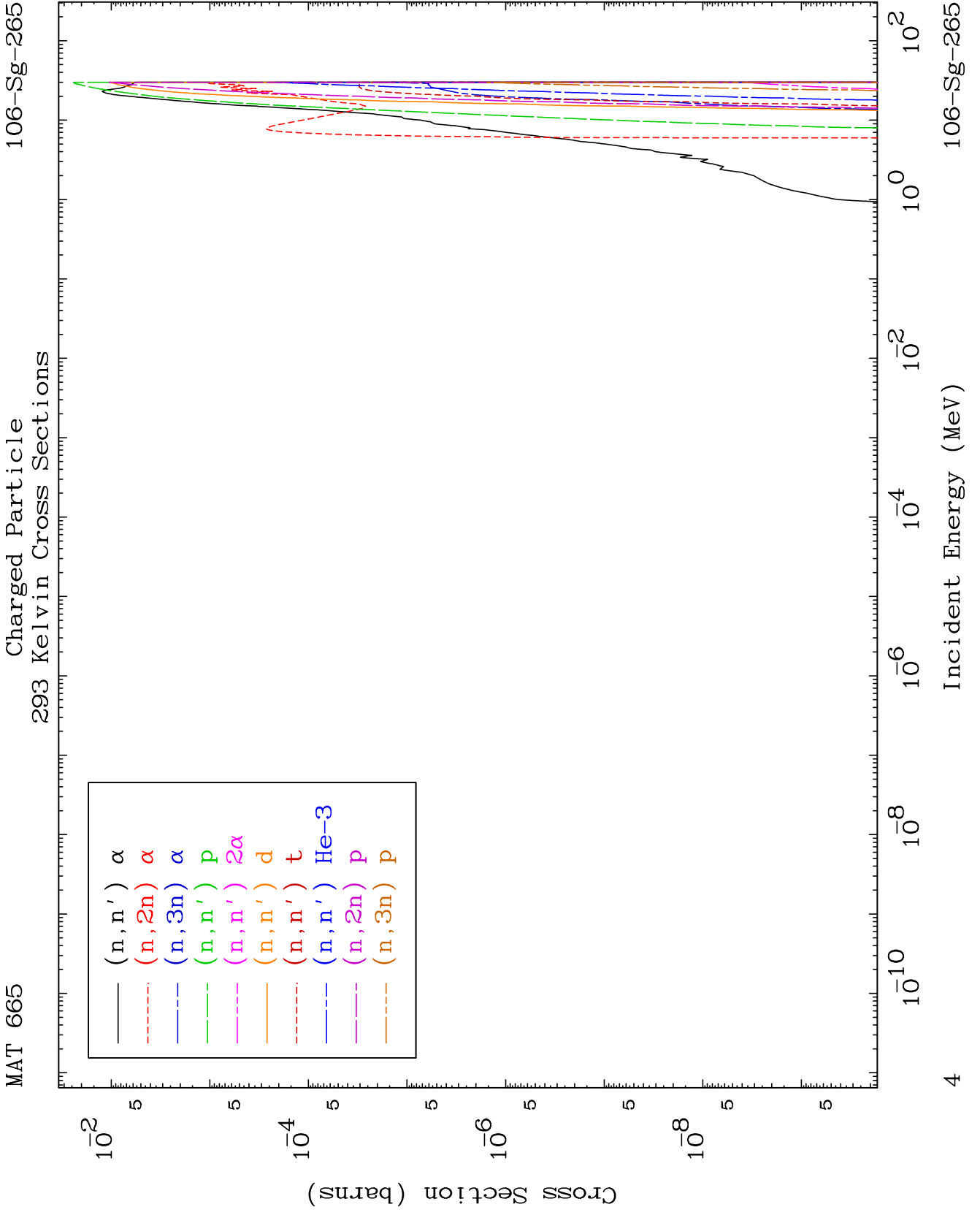
MAT 665

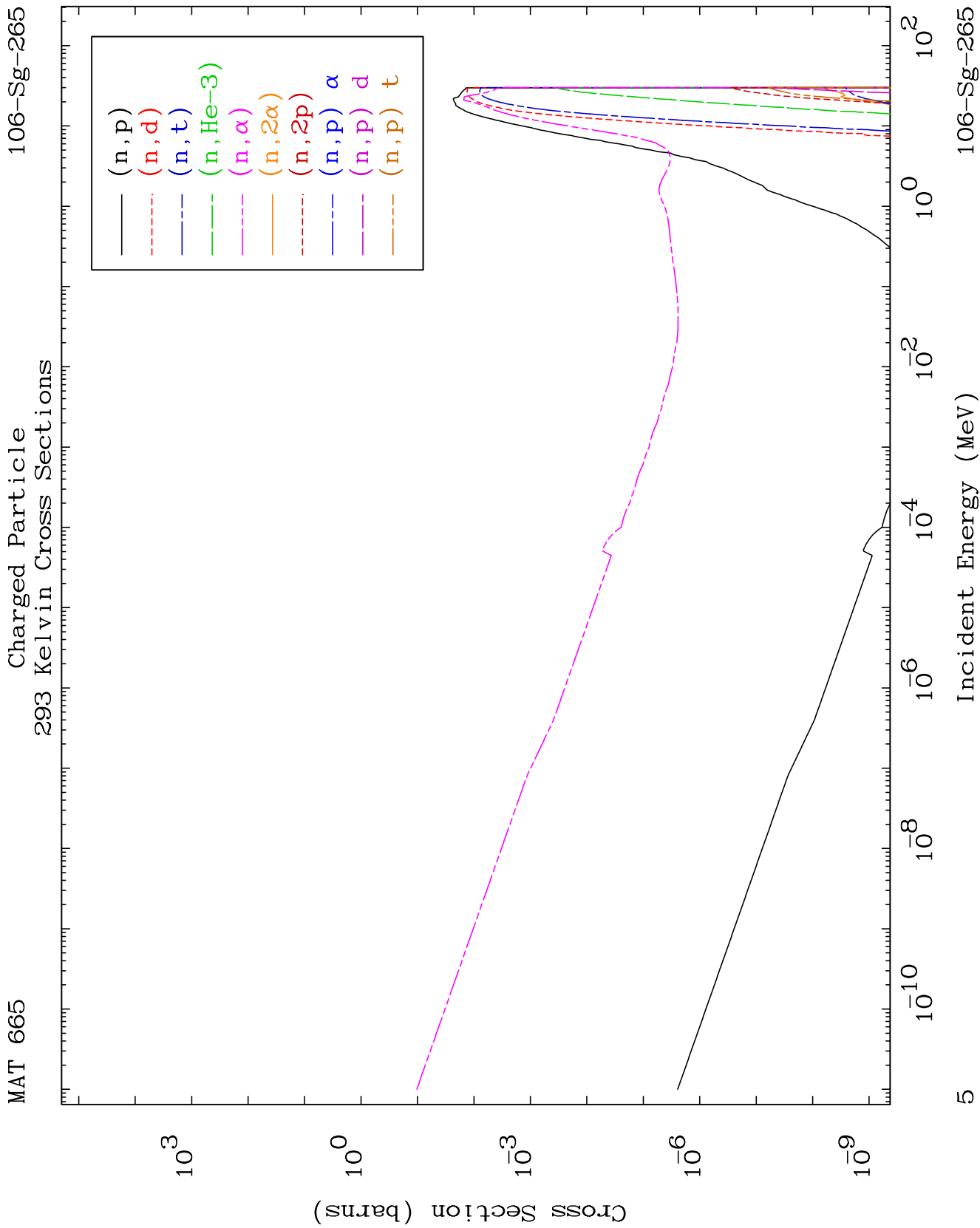
Neutron Production  
293 Kelvin Cross Sections

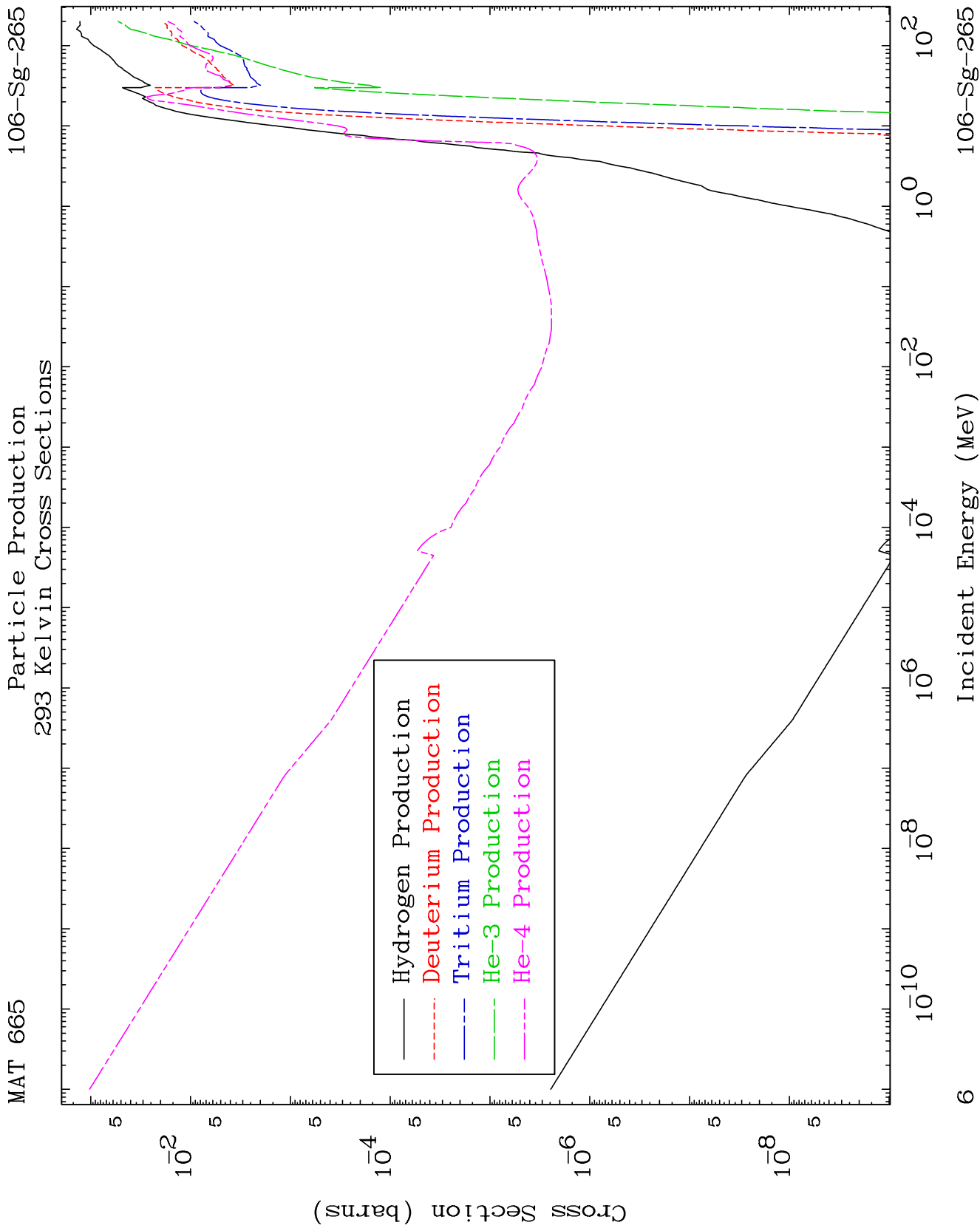
106-Sg-265

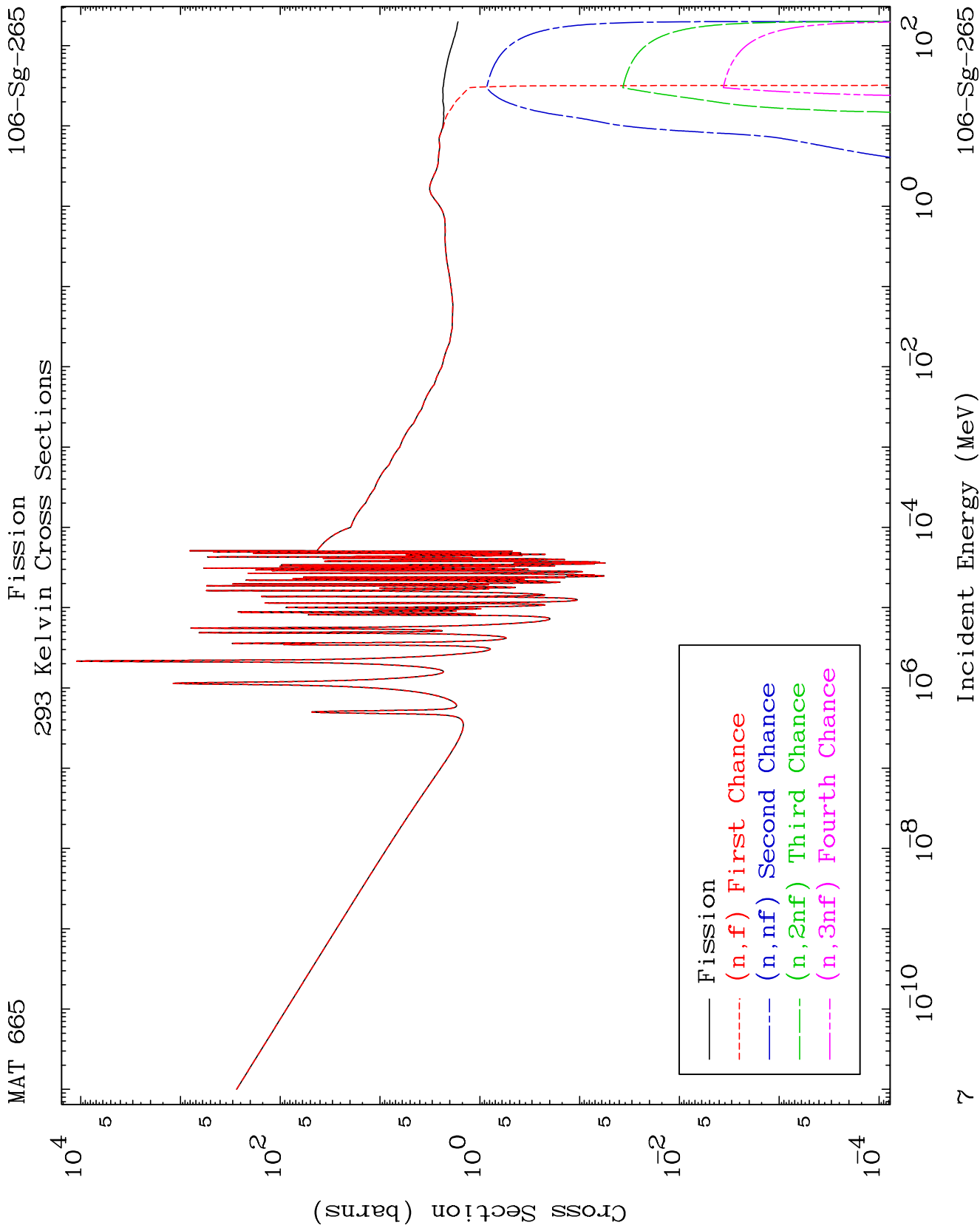




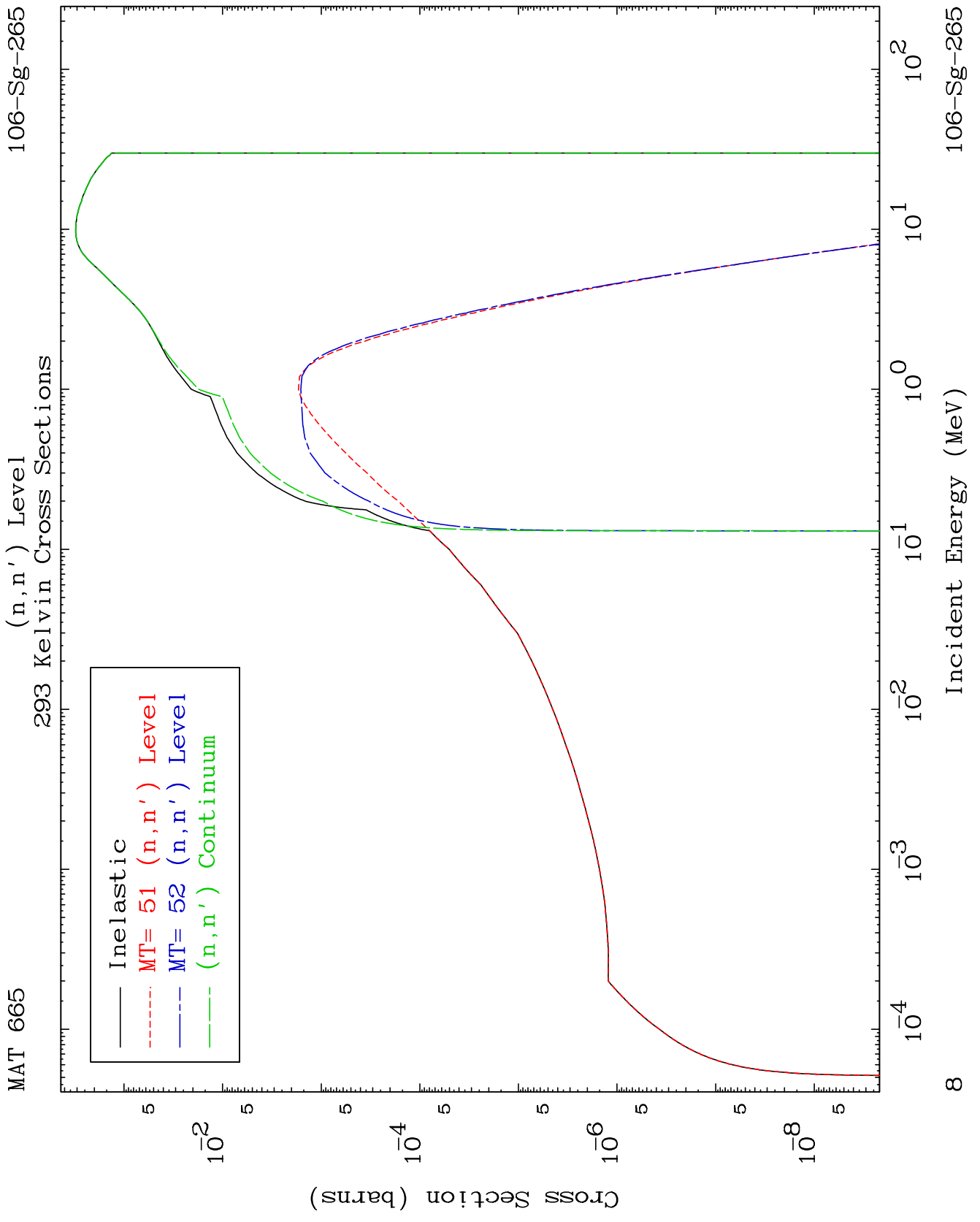


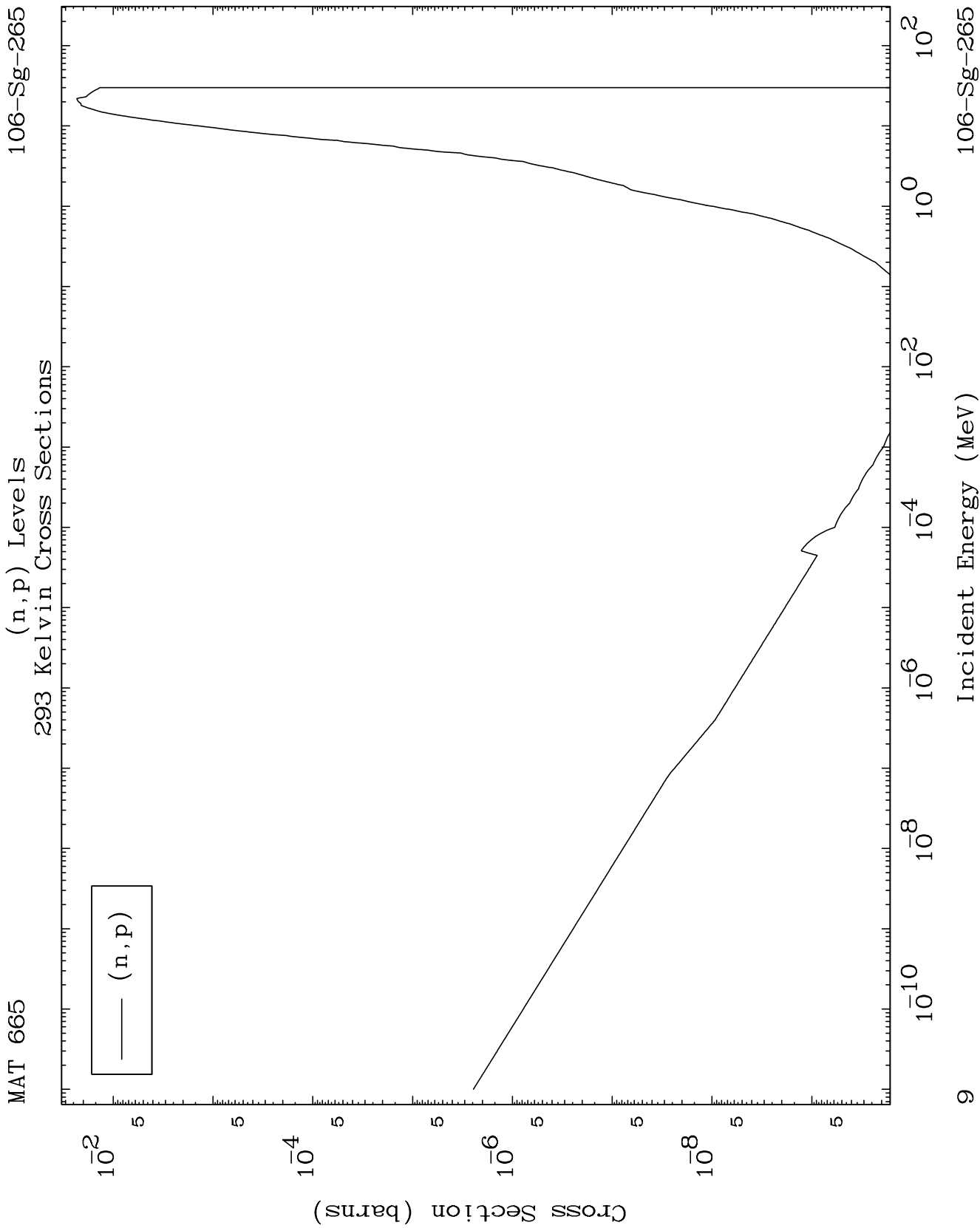


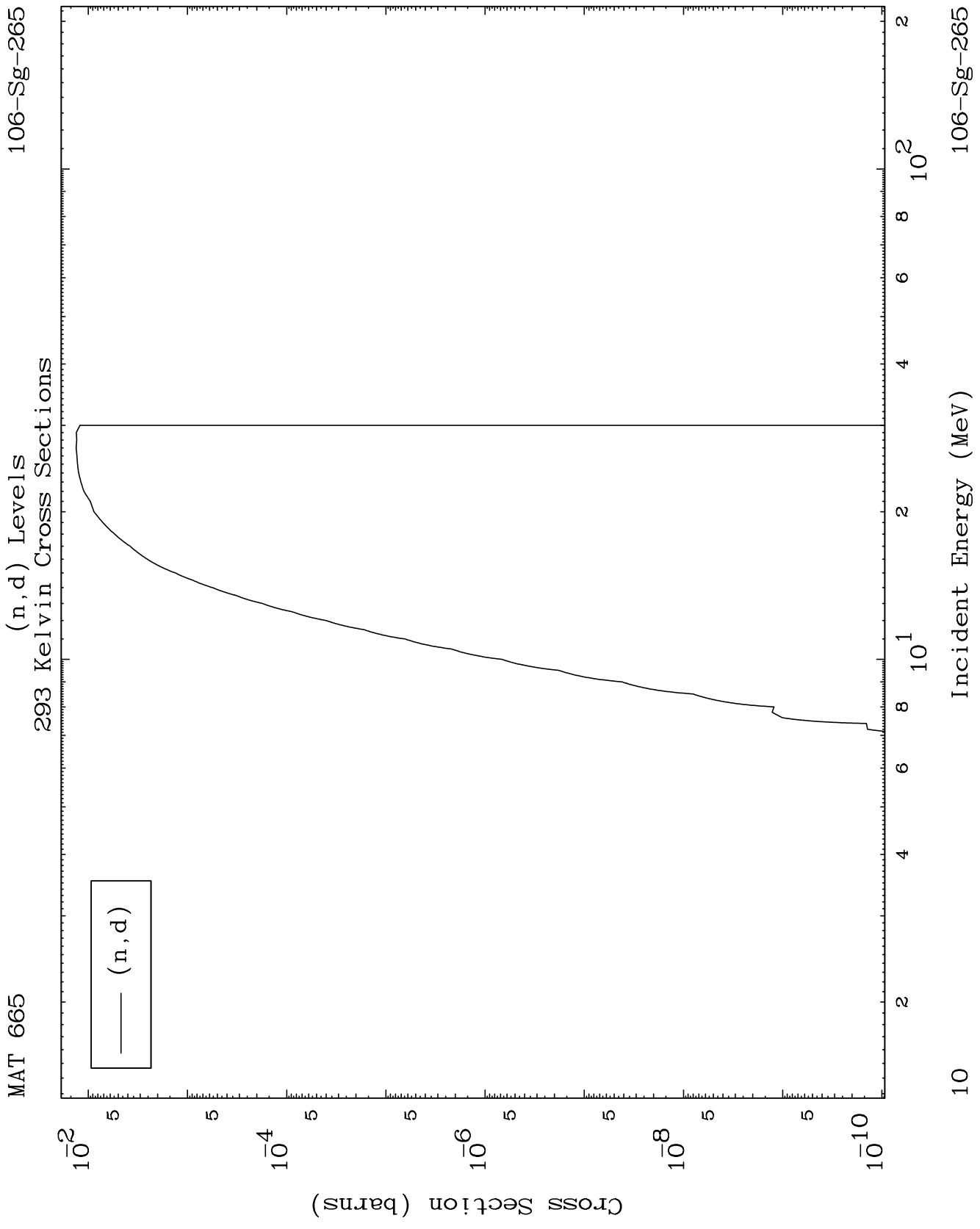


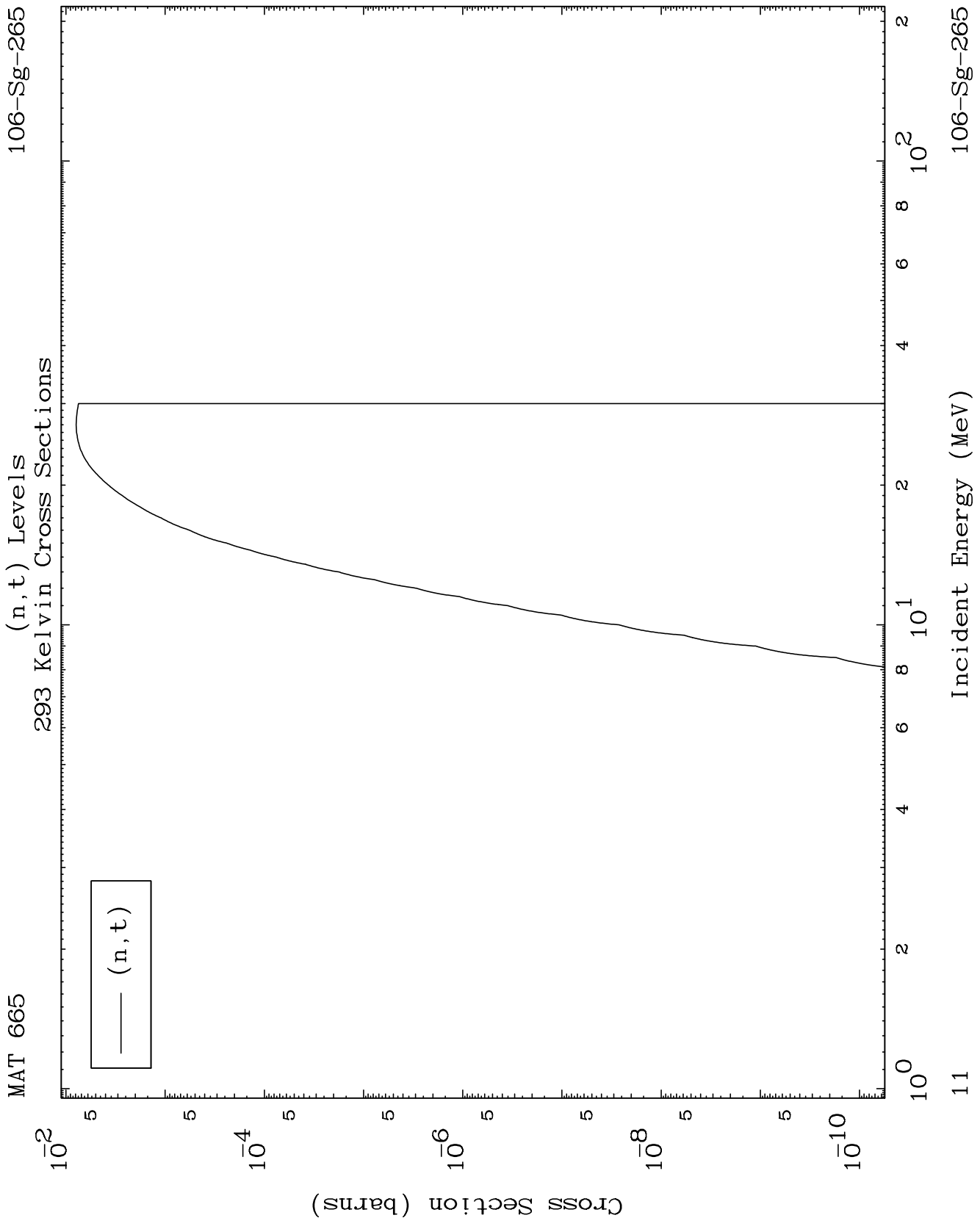


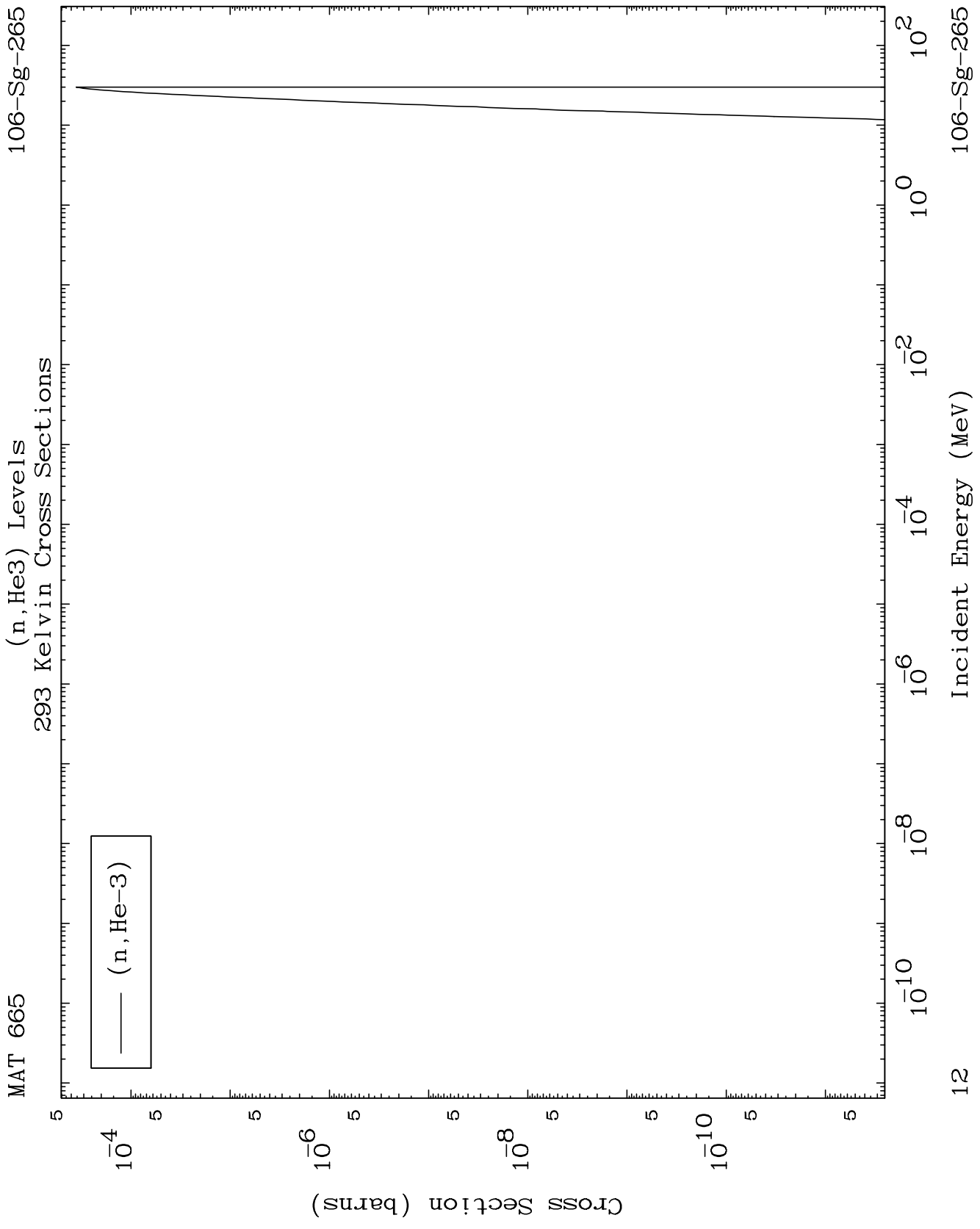


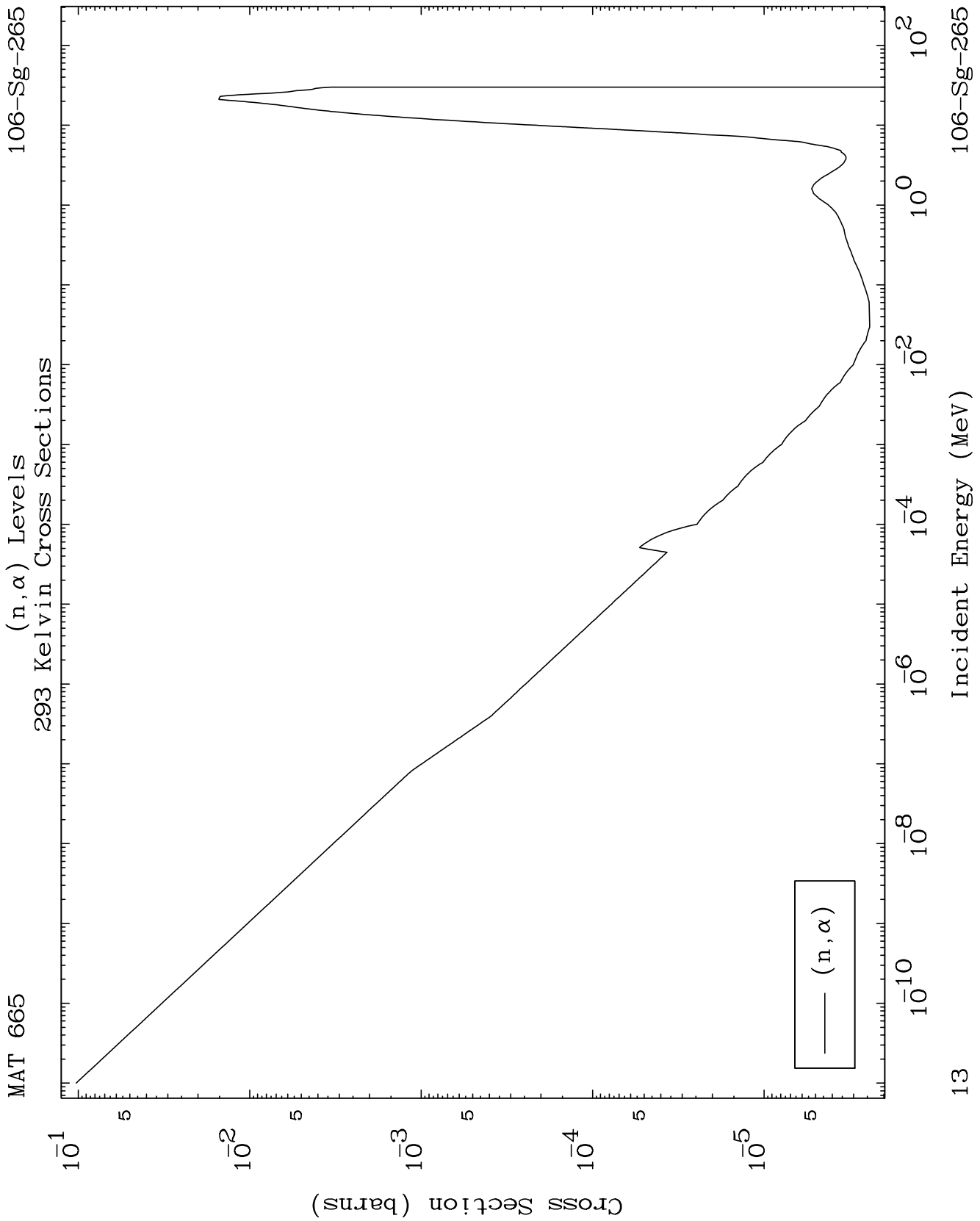


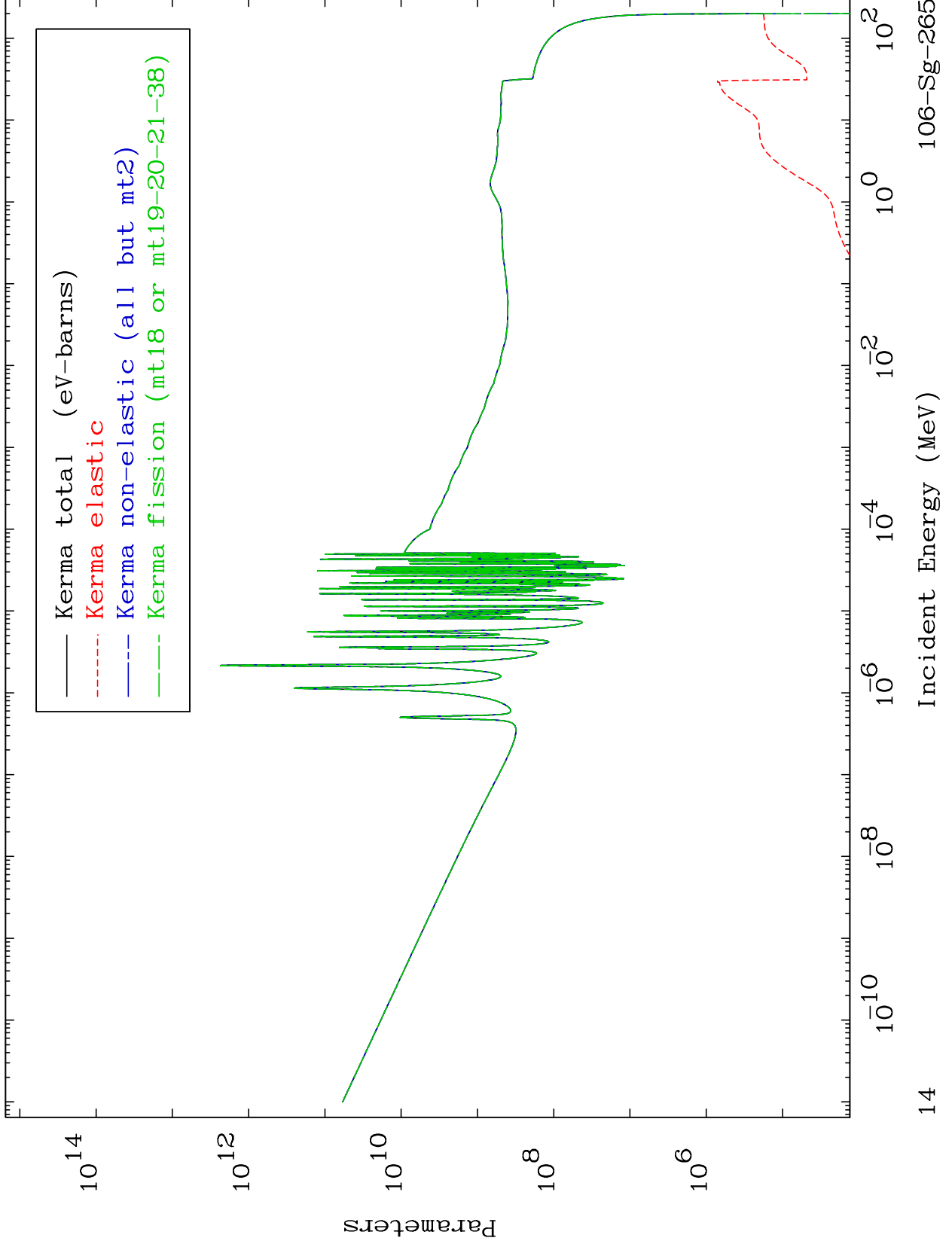


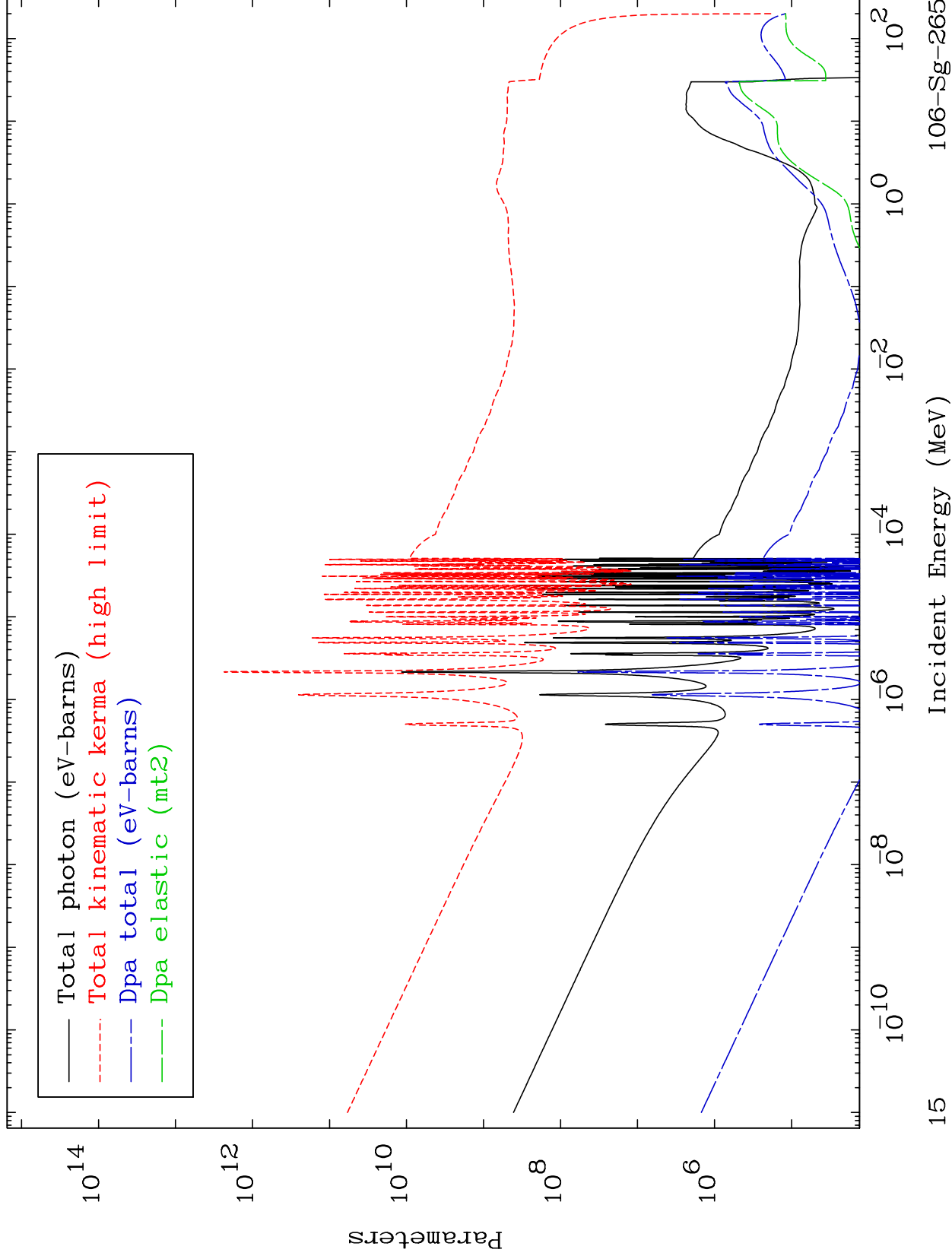






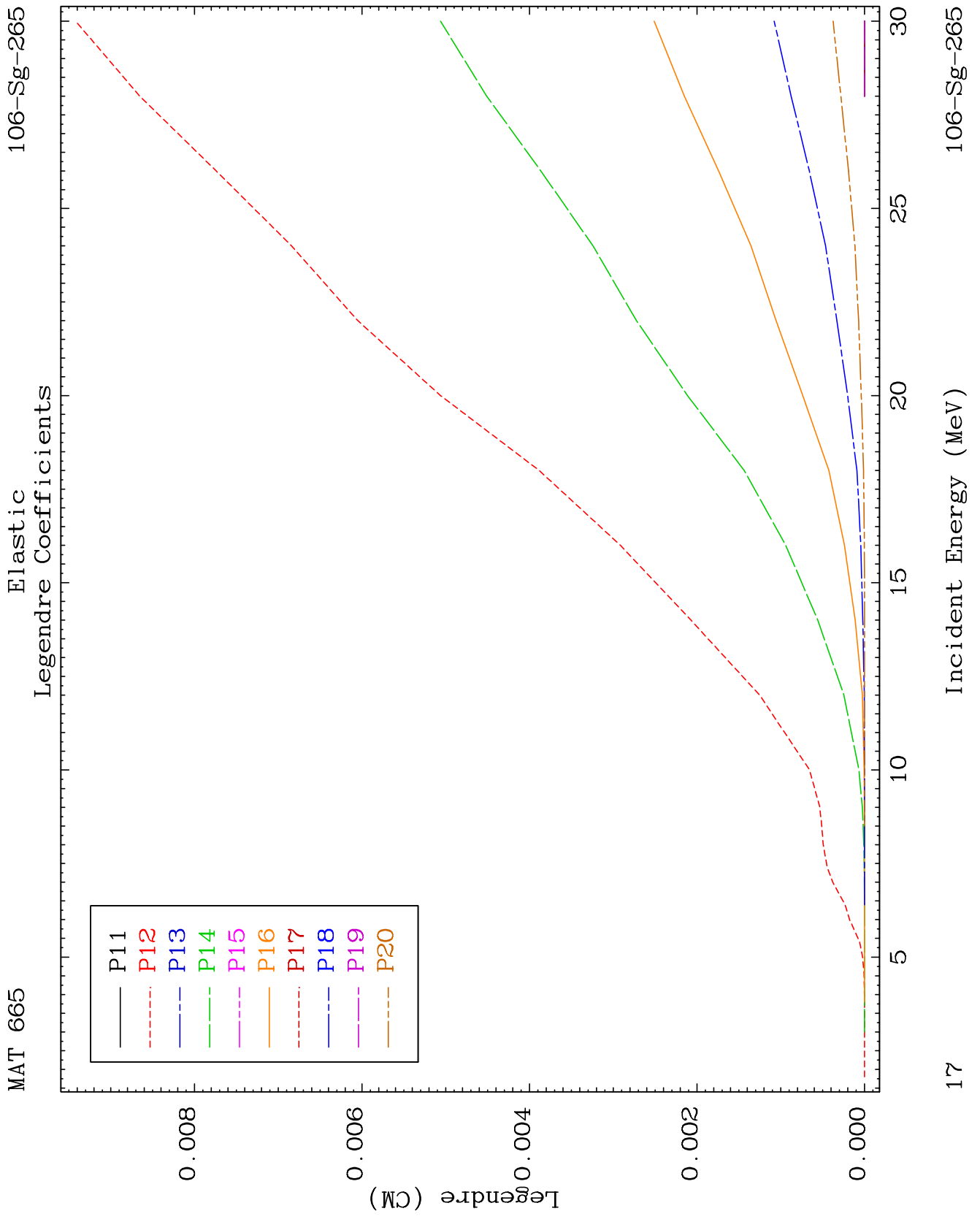


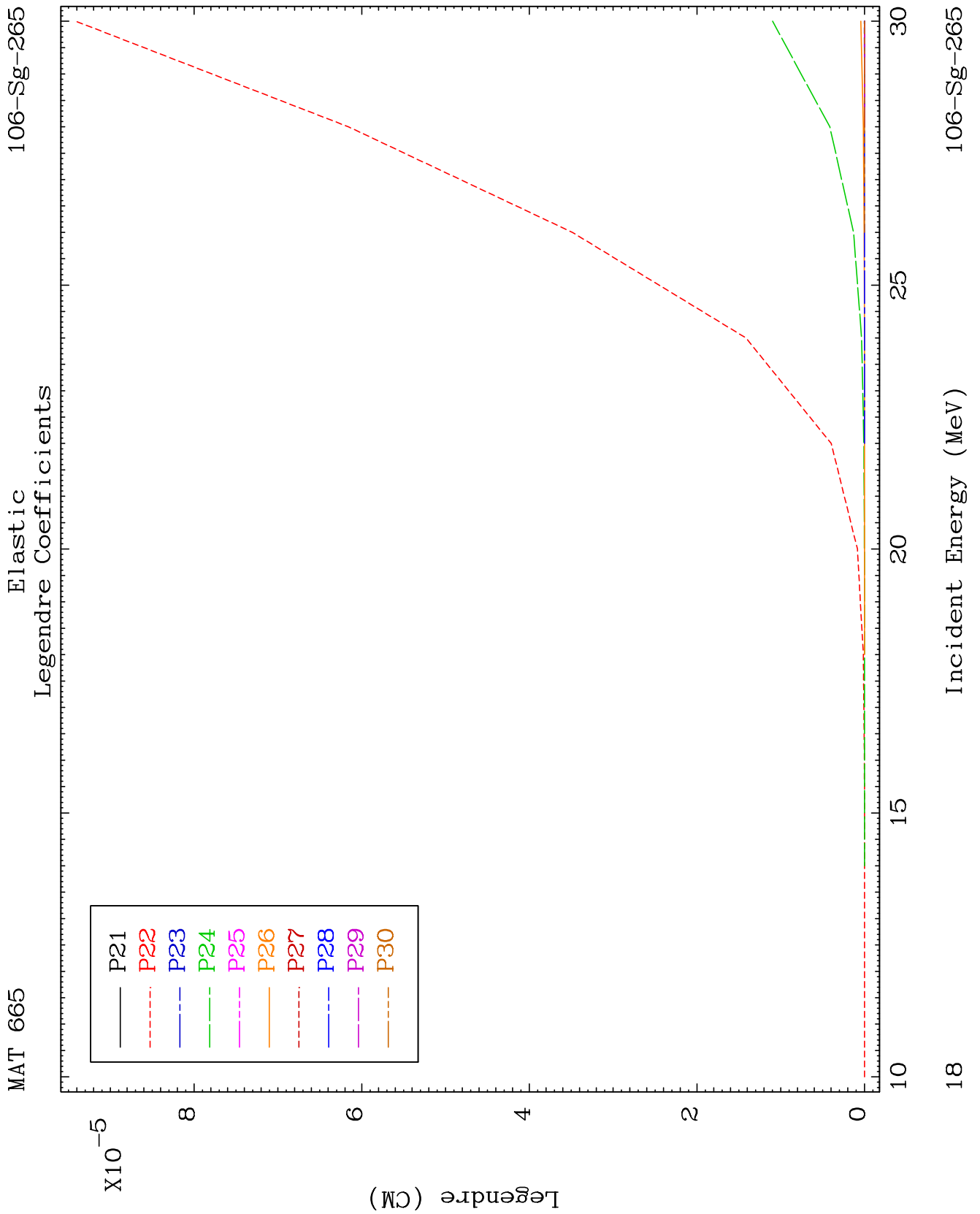








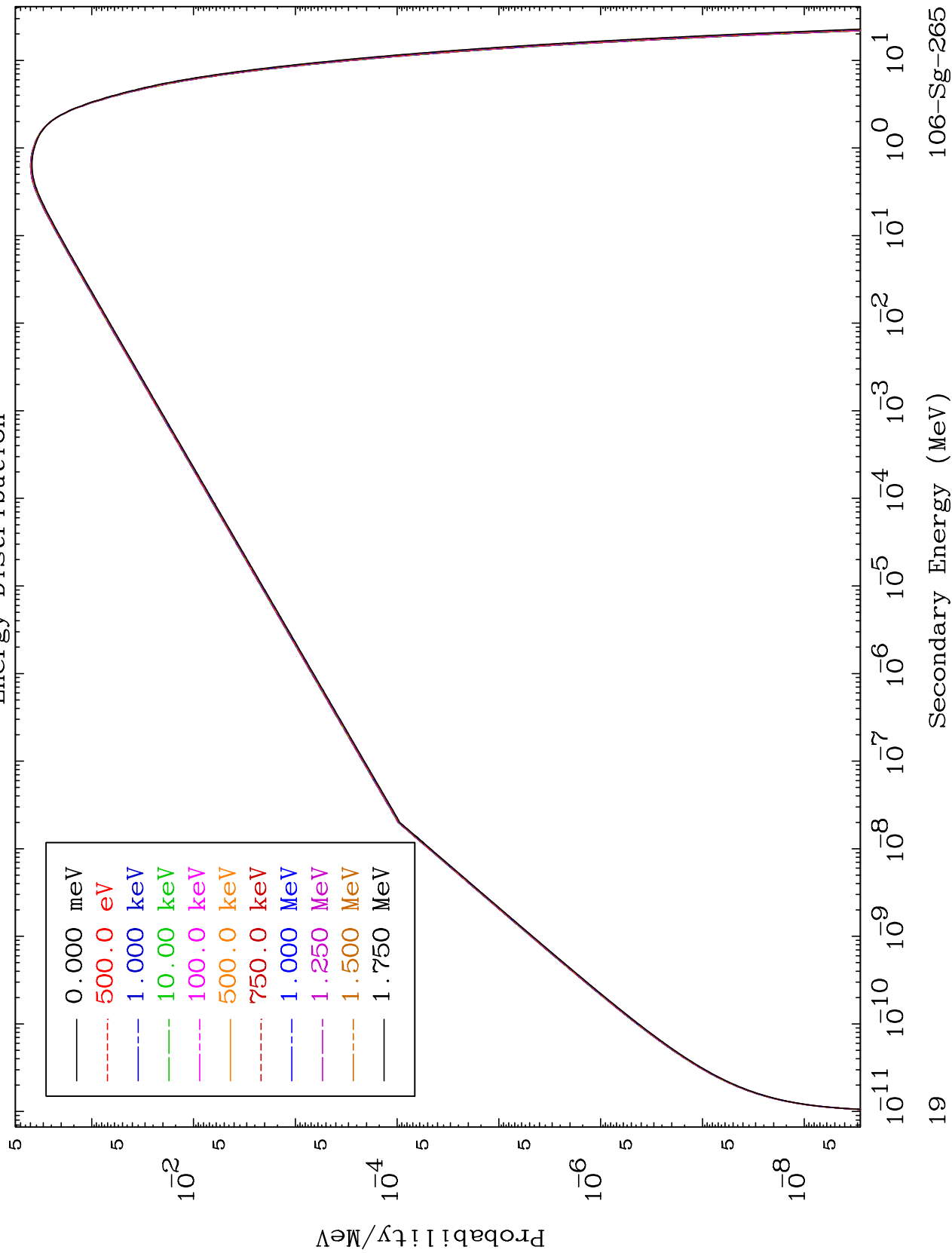




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### Fission Energy Distribution

106-Sg-265



106-Sg-265

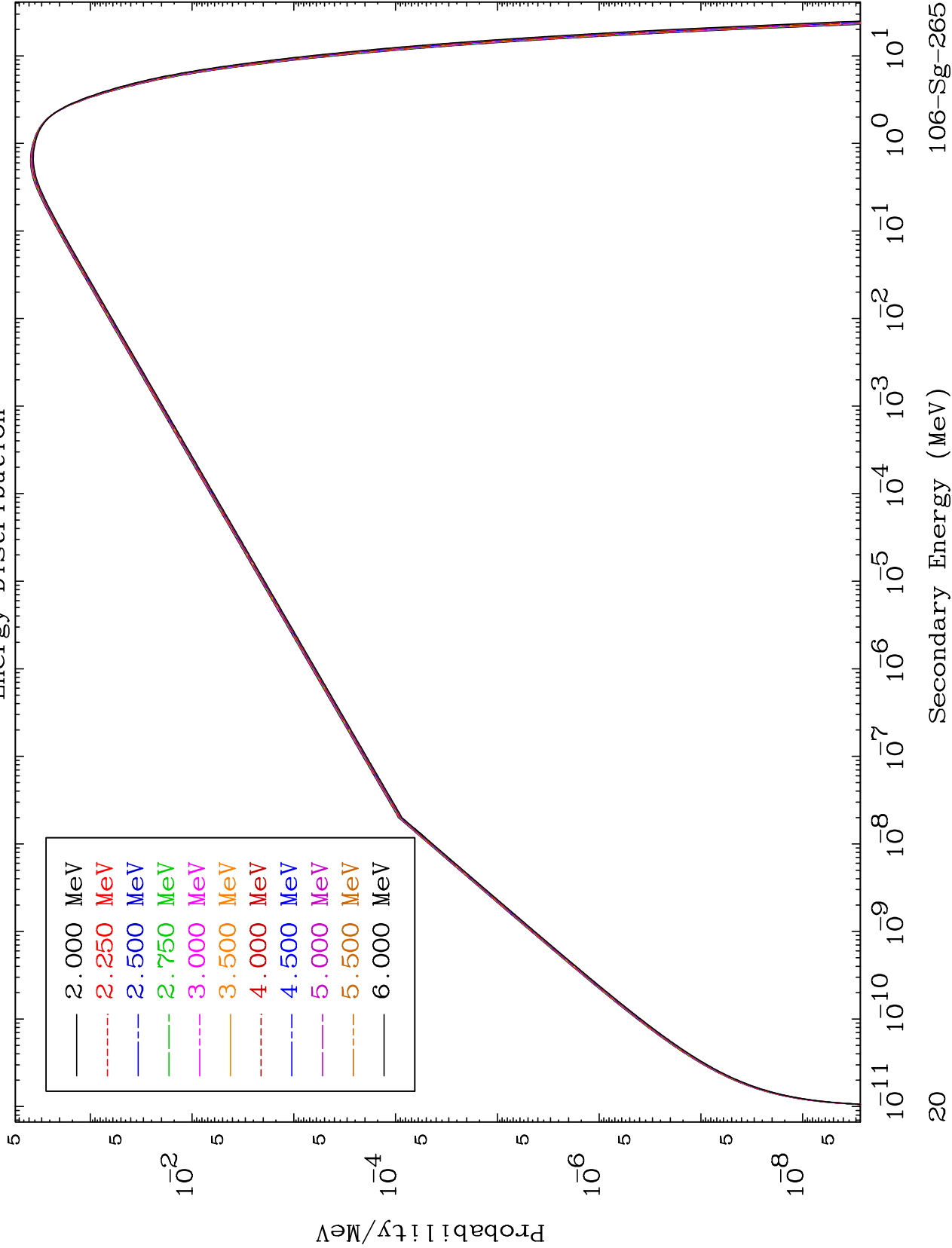
Secondary Energy (MeV)

19

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### Fission Energy Distribution

106-Sg-265



106-Sg-265

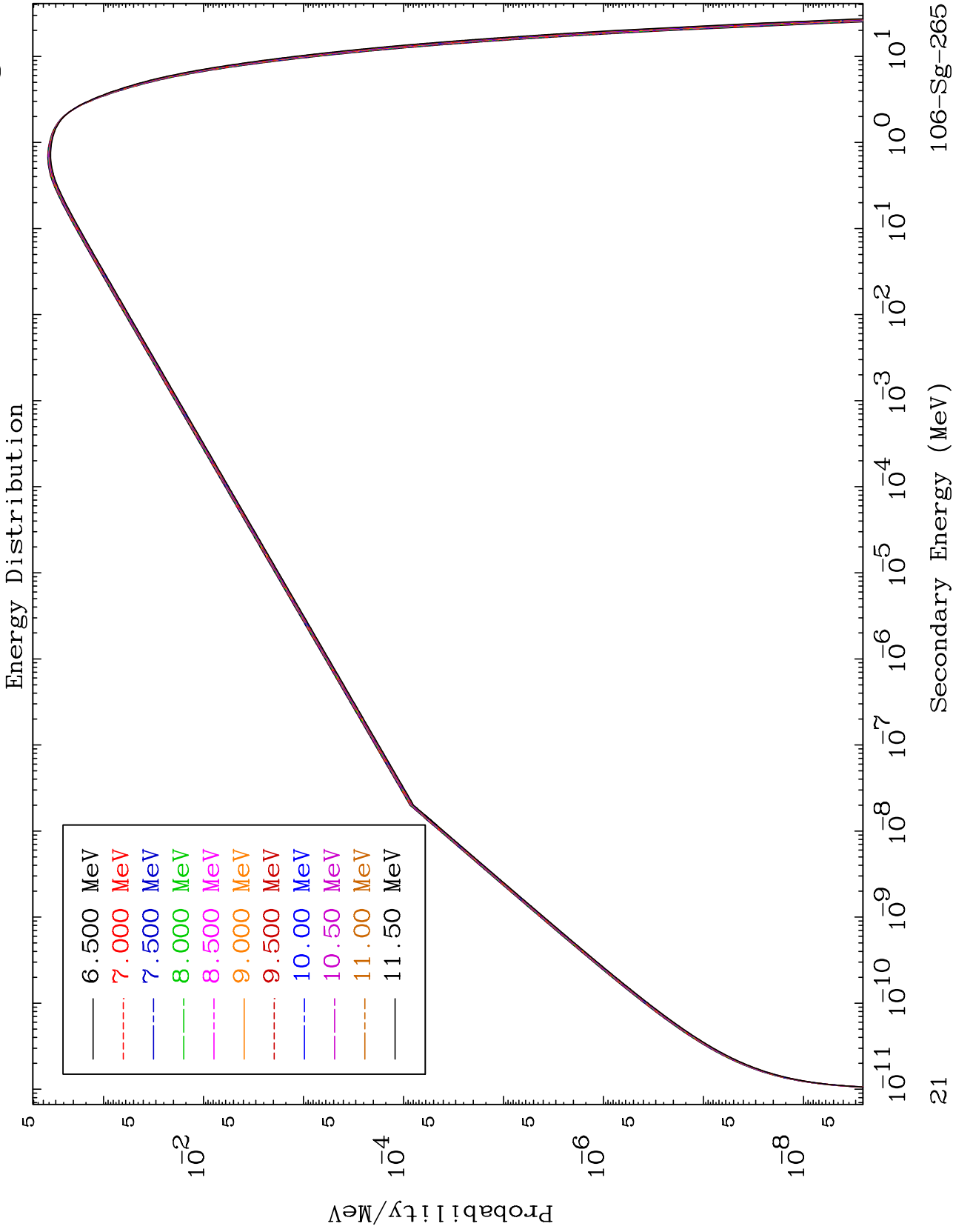
Secondary Energy (MeV)

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Fission

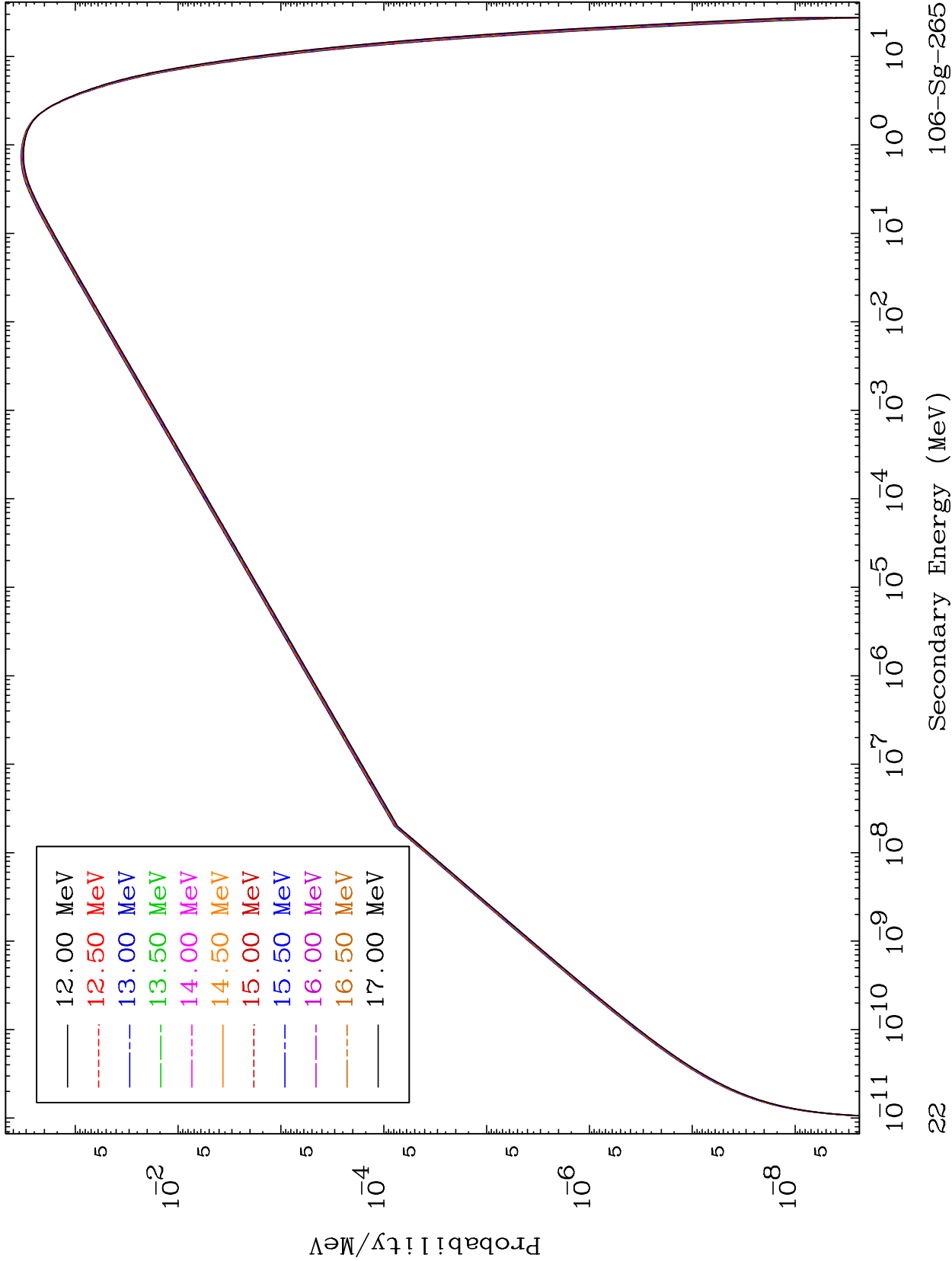
106-Sg-265



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### Fission Energy Distribution

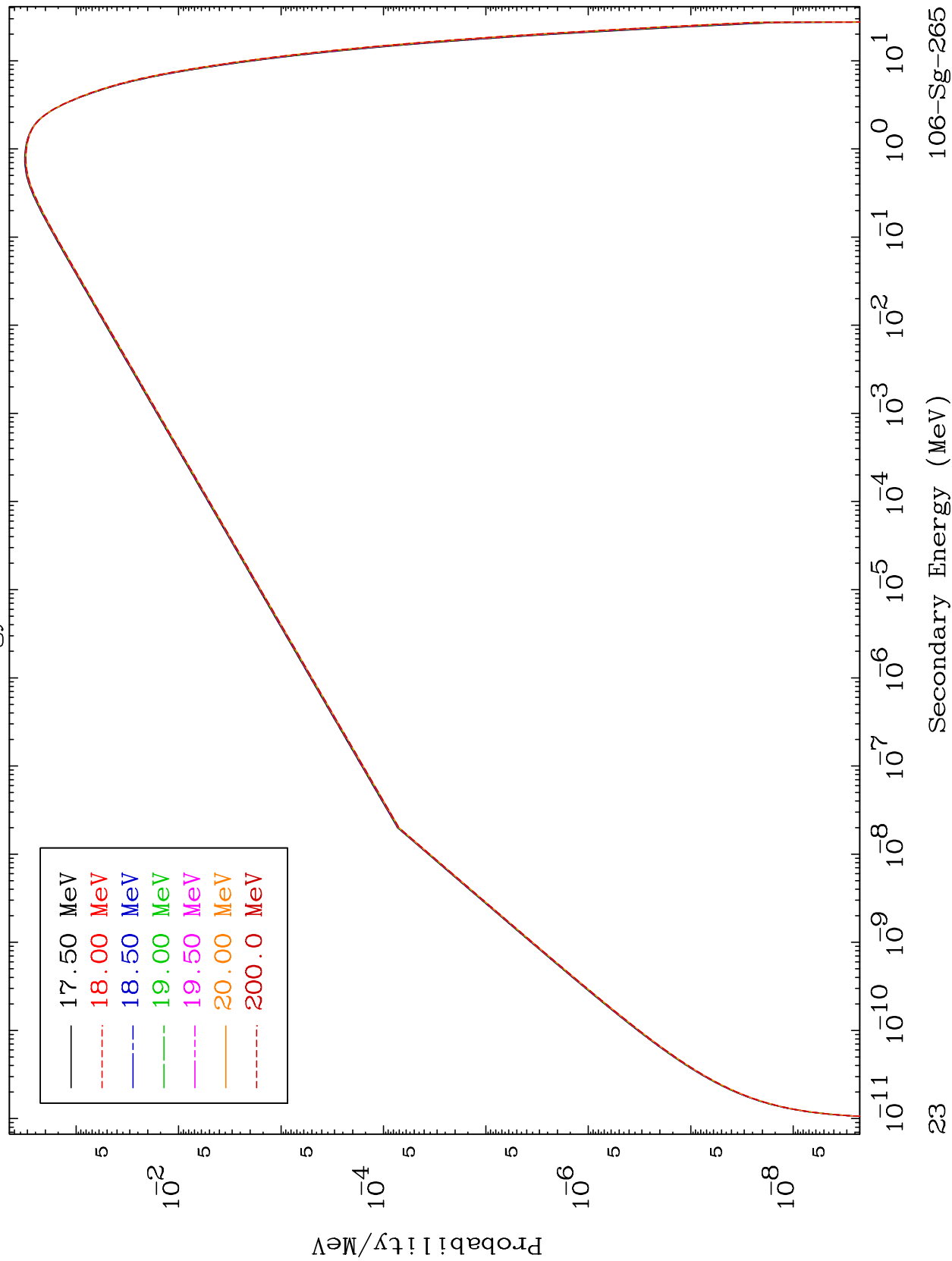
106-Sg-265



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### Fission Energy Distribution

106-Sg-265

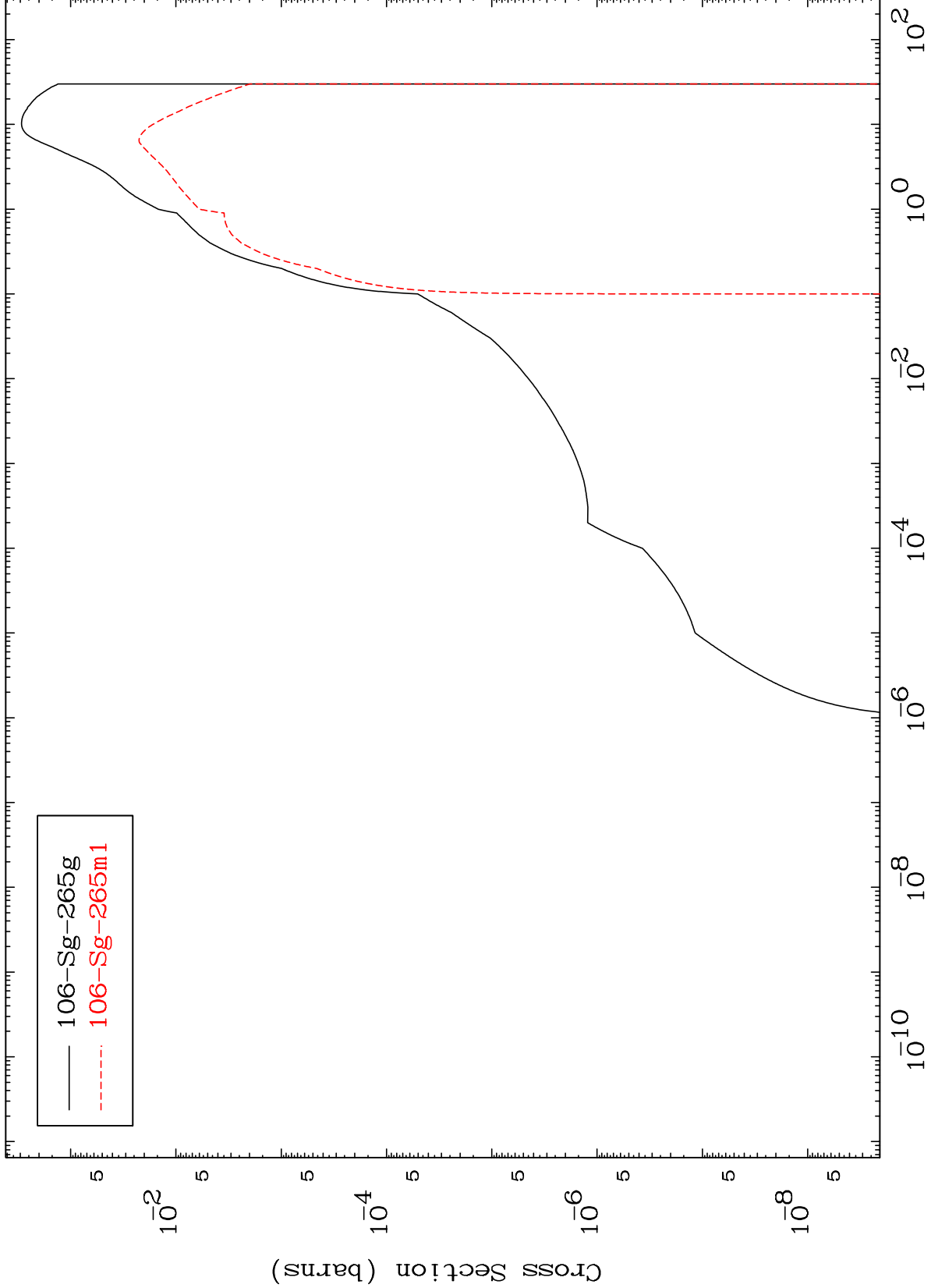




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Inelastic  
Radionuclide Production Cross Section

106-Sg-265



106-Sg-265

