

Program EVALPLOT
(Version 2018-1)

by

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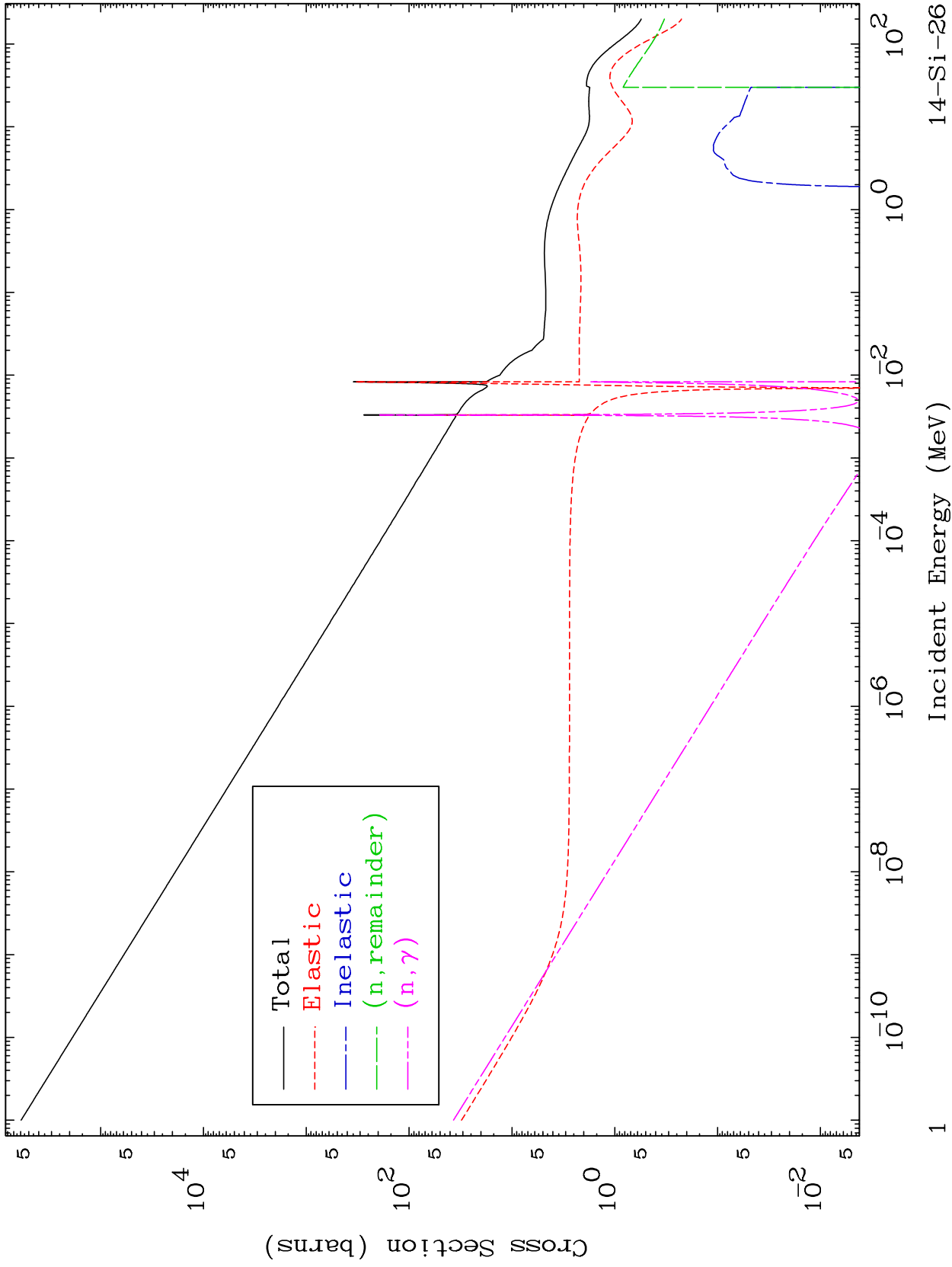
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 1419

Major
293 Kelvin Cross Sections

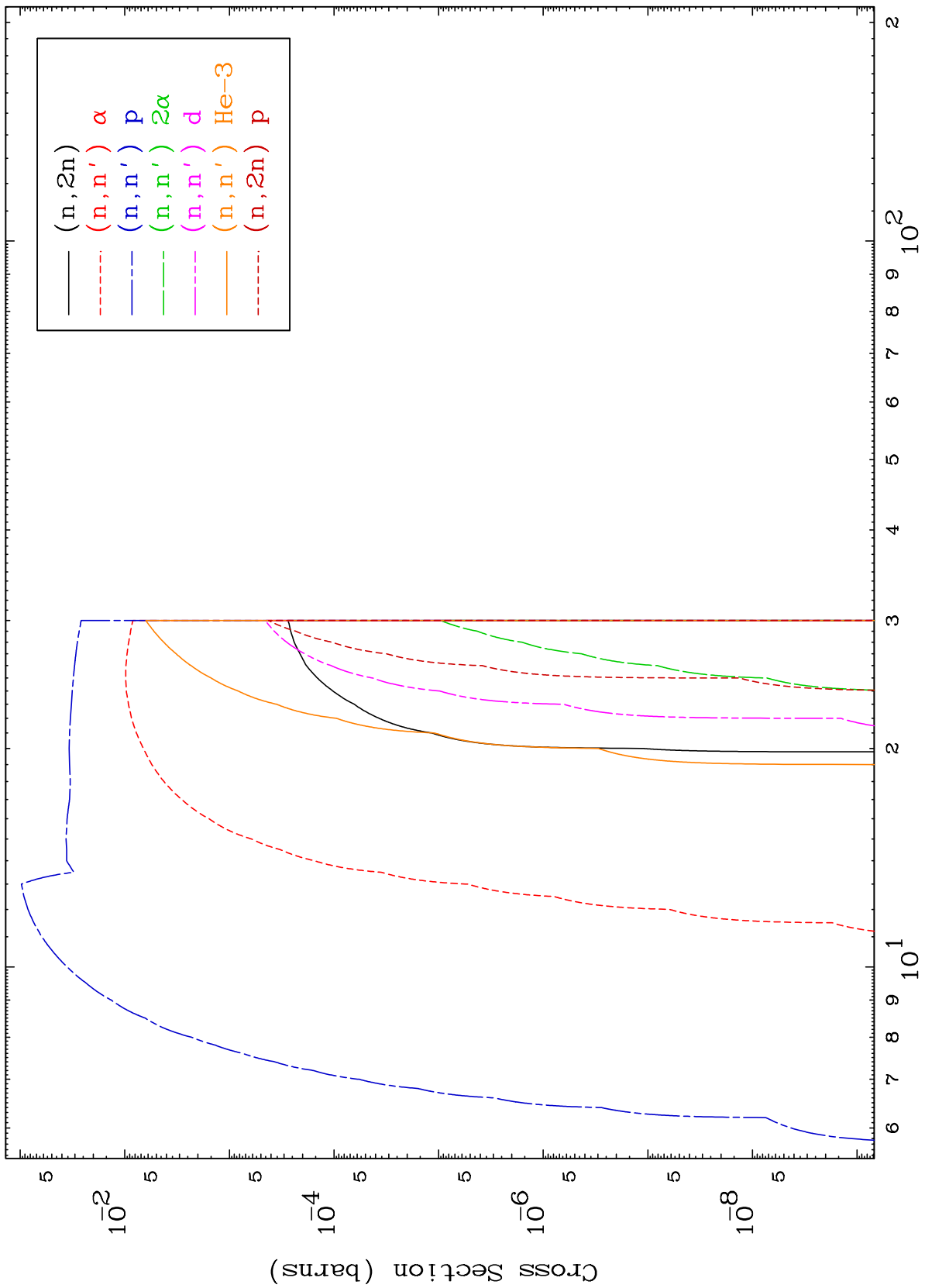
14-Si-26



MAT 1419

Neutron Production
293 Kelvin Cross Sections

14-Si-26



2

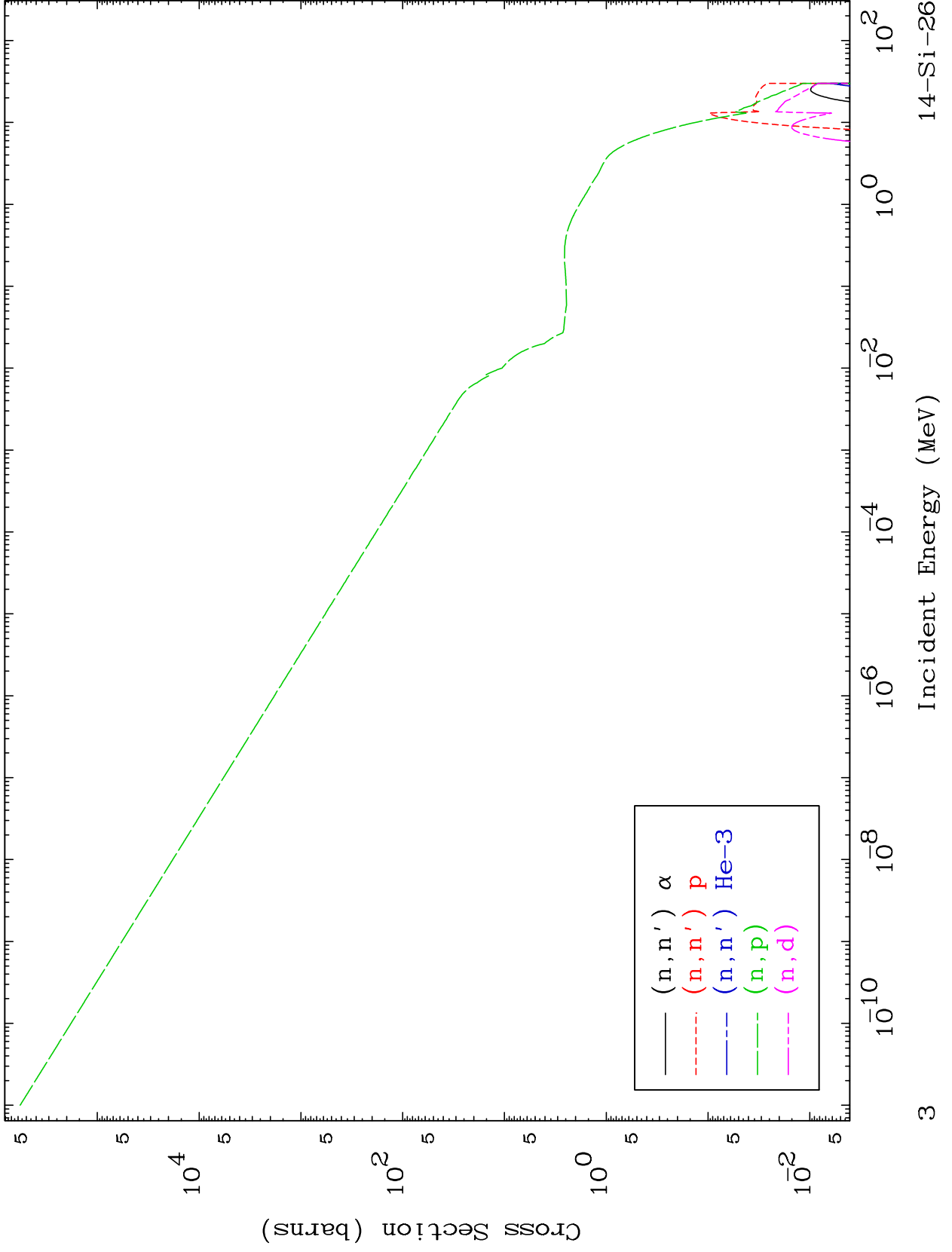
Incident Energy (MeV)

14-Si-26

MAT 1419

Charged Particle
293 Kelvin Cross Sections

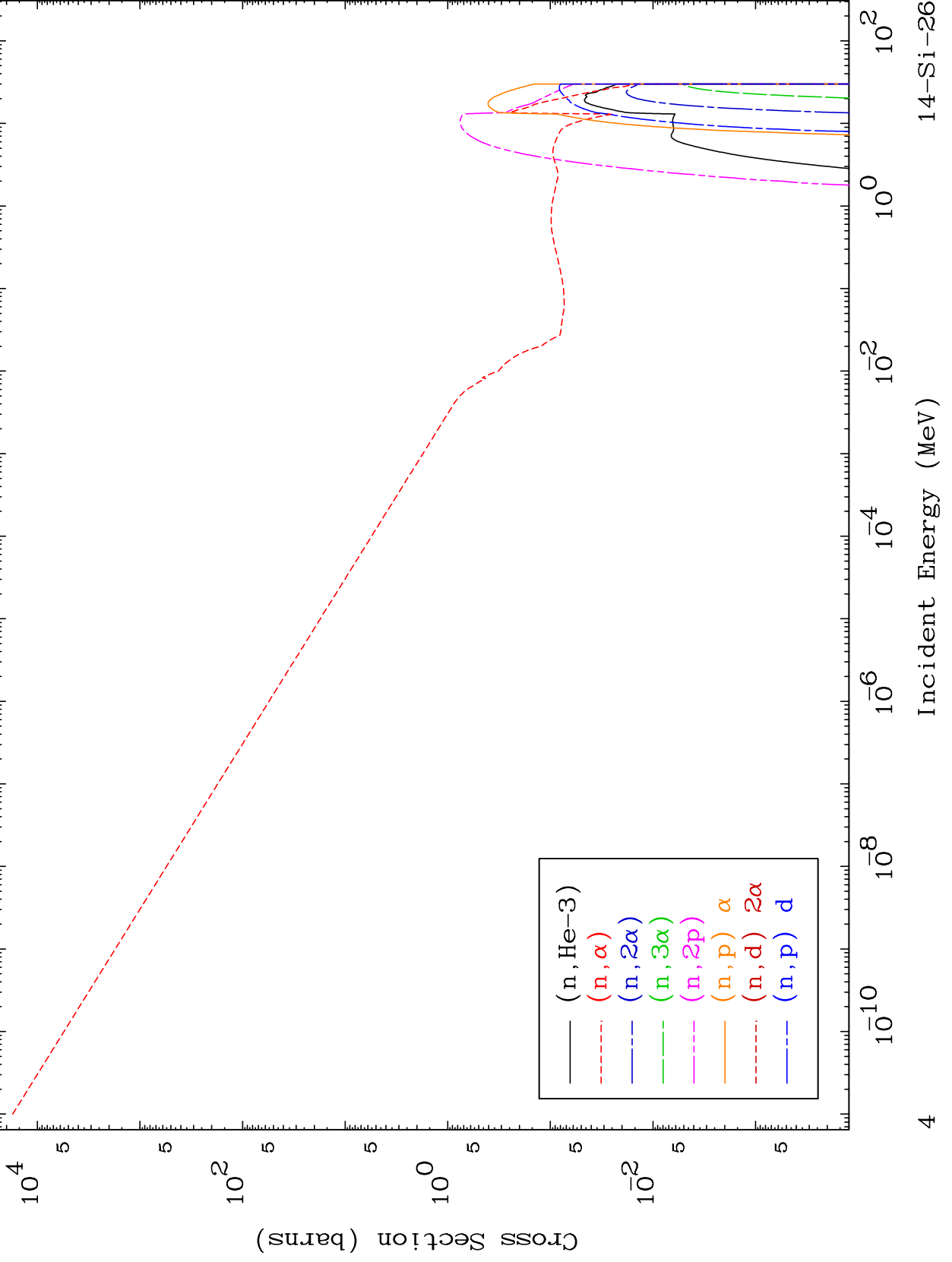
14-Si-26

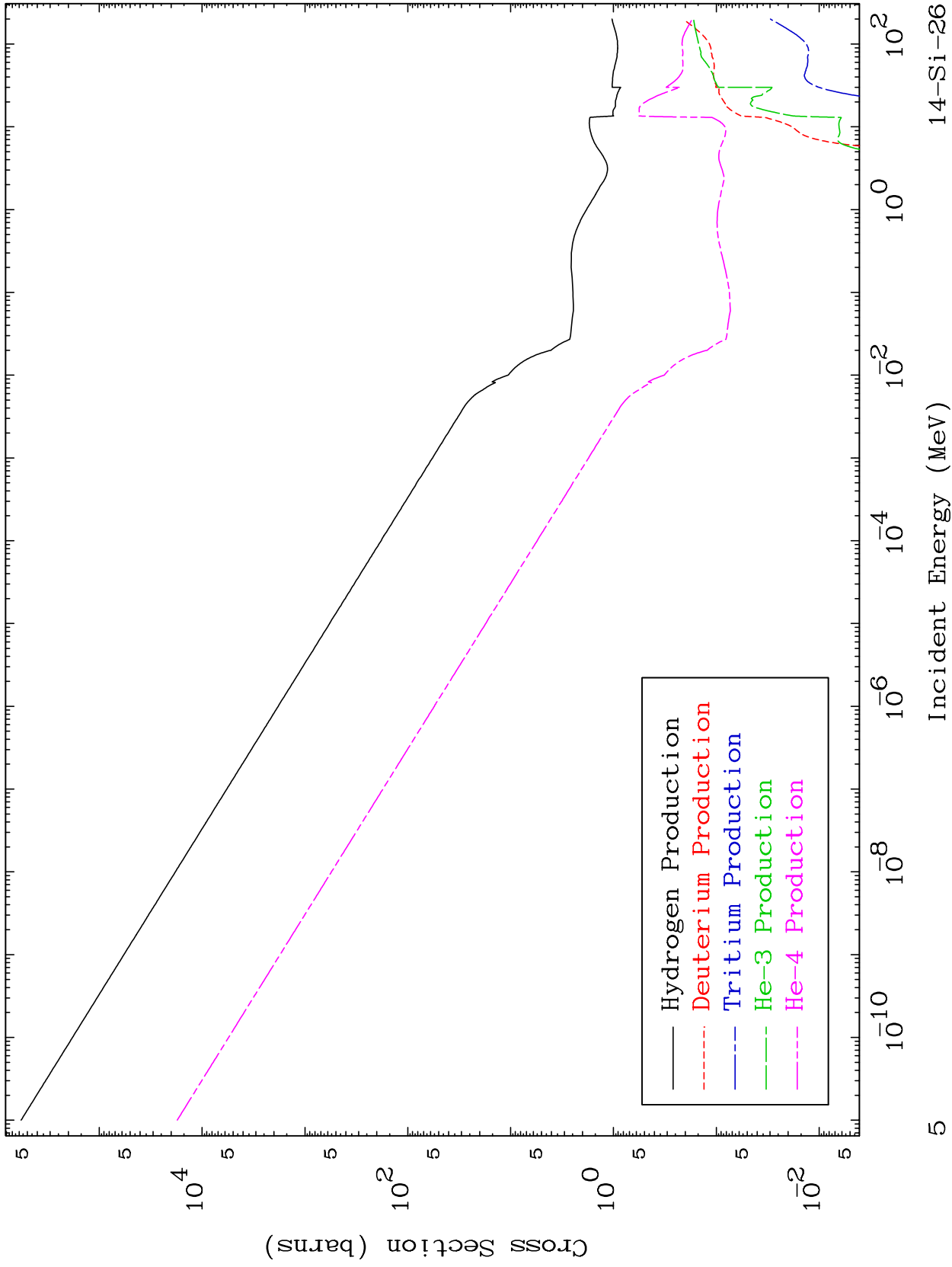


MAT 1419

Charged Particle
293 Kelvin Cross Sections

14-Si-26



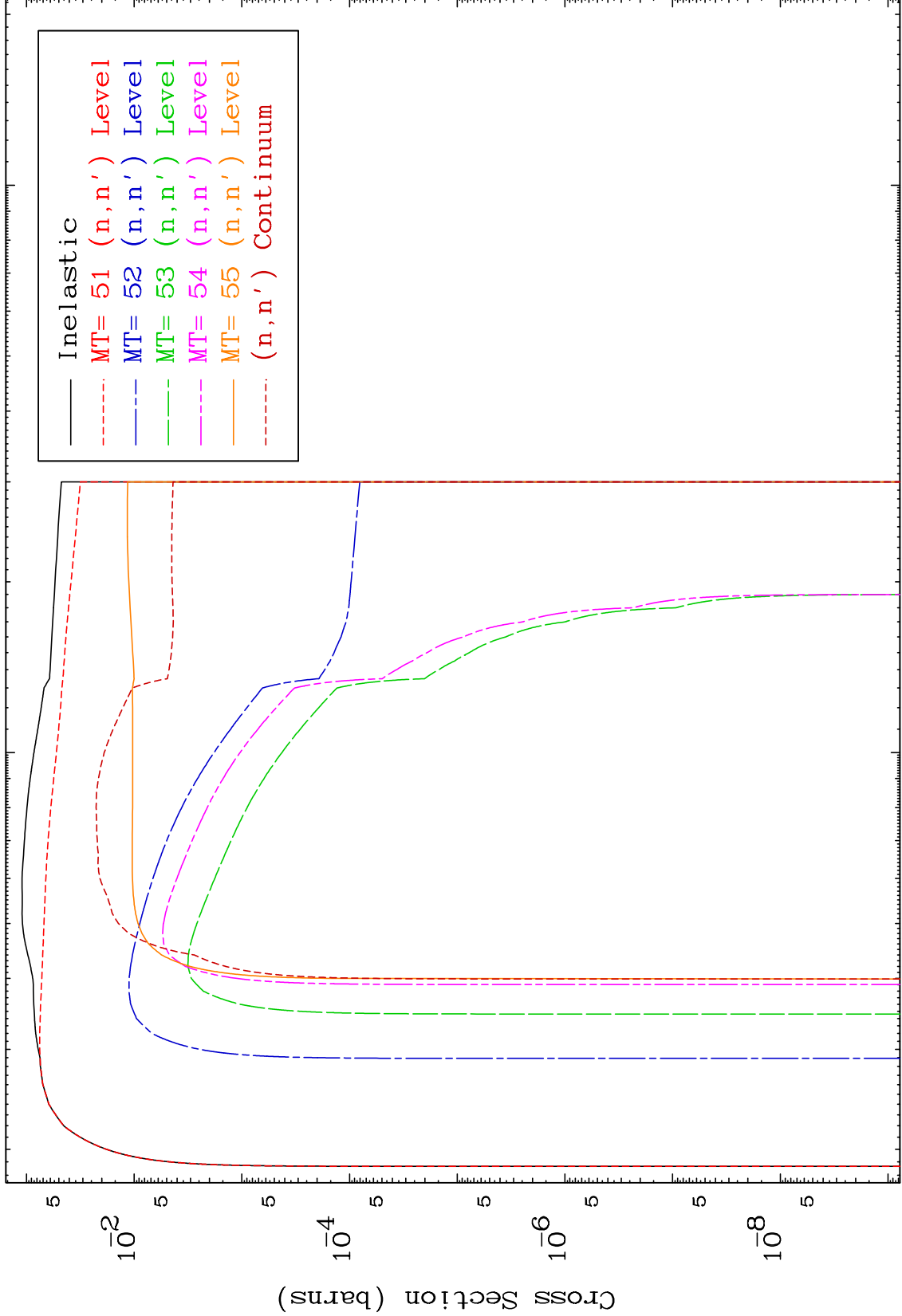


MAT 1419

(n,n') Level

14-Si-26

293 Kelvin Cross Sections



14-Si-26

Incident Energy (MeV)

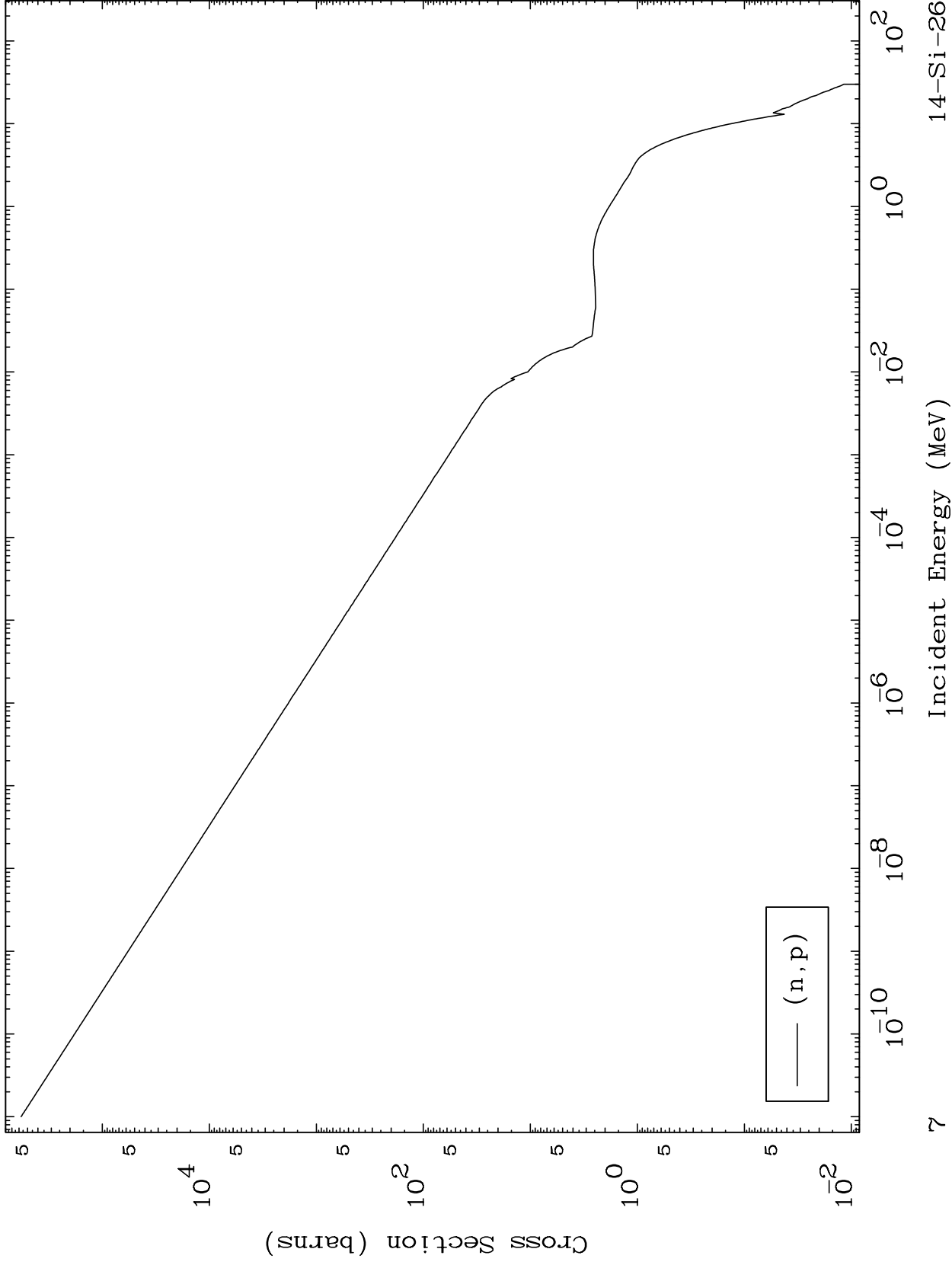
14-Si-26

6

MAT 1419

(n,p) Levels
293 Kelvin Cross Sections

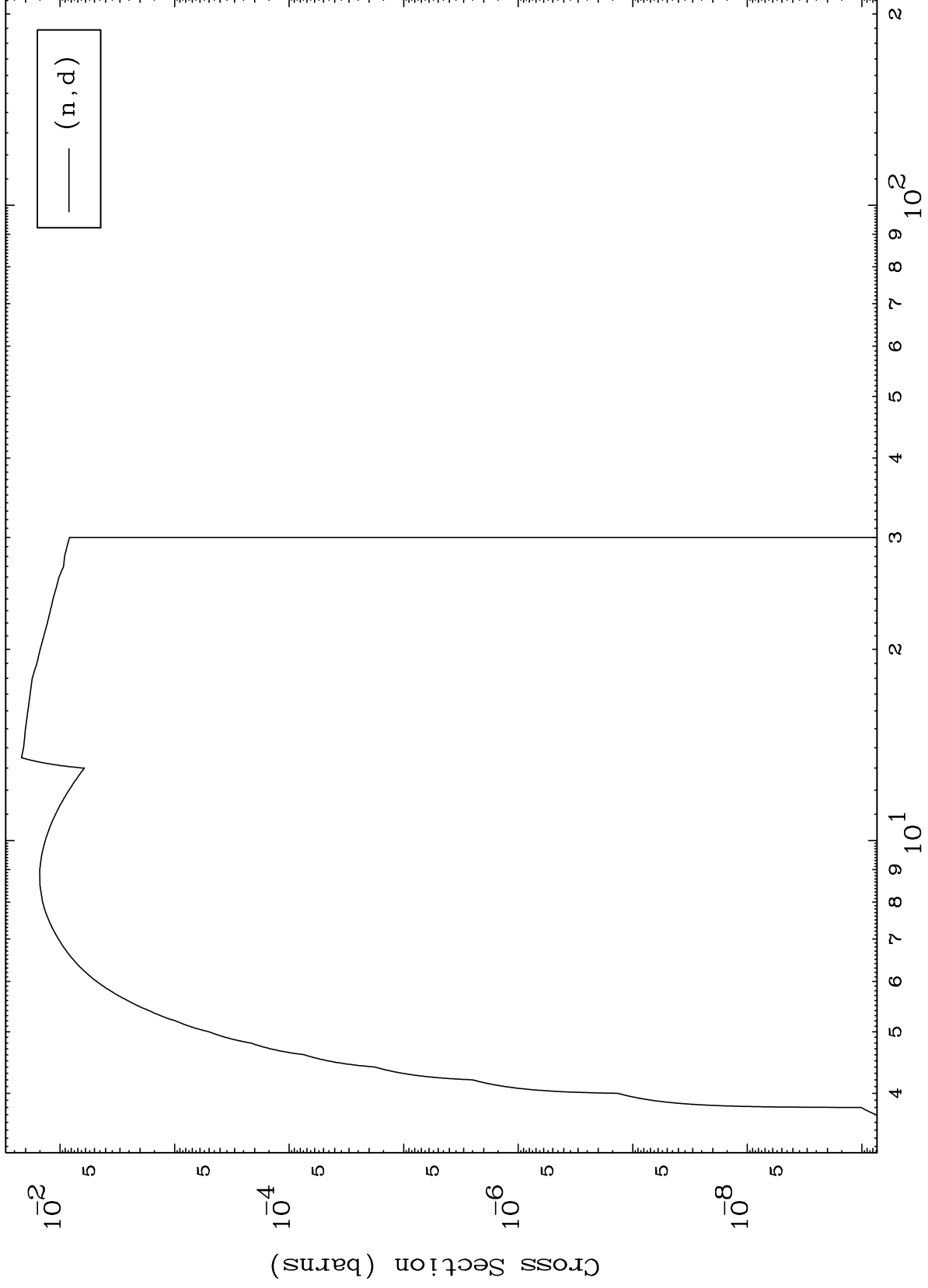
14-Si-26



MAT 1419

(n,d) Levels
293 Kelvin Cross Sections

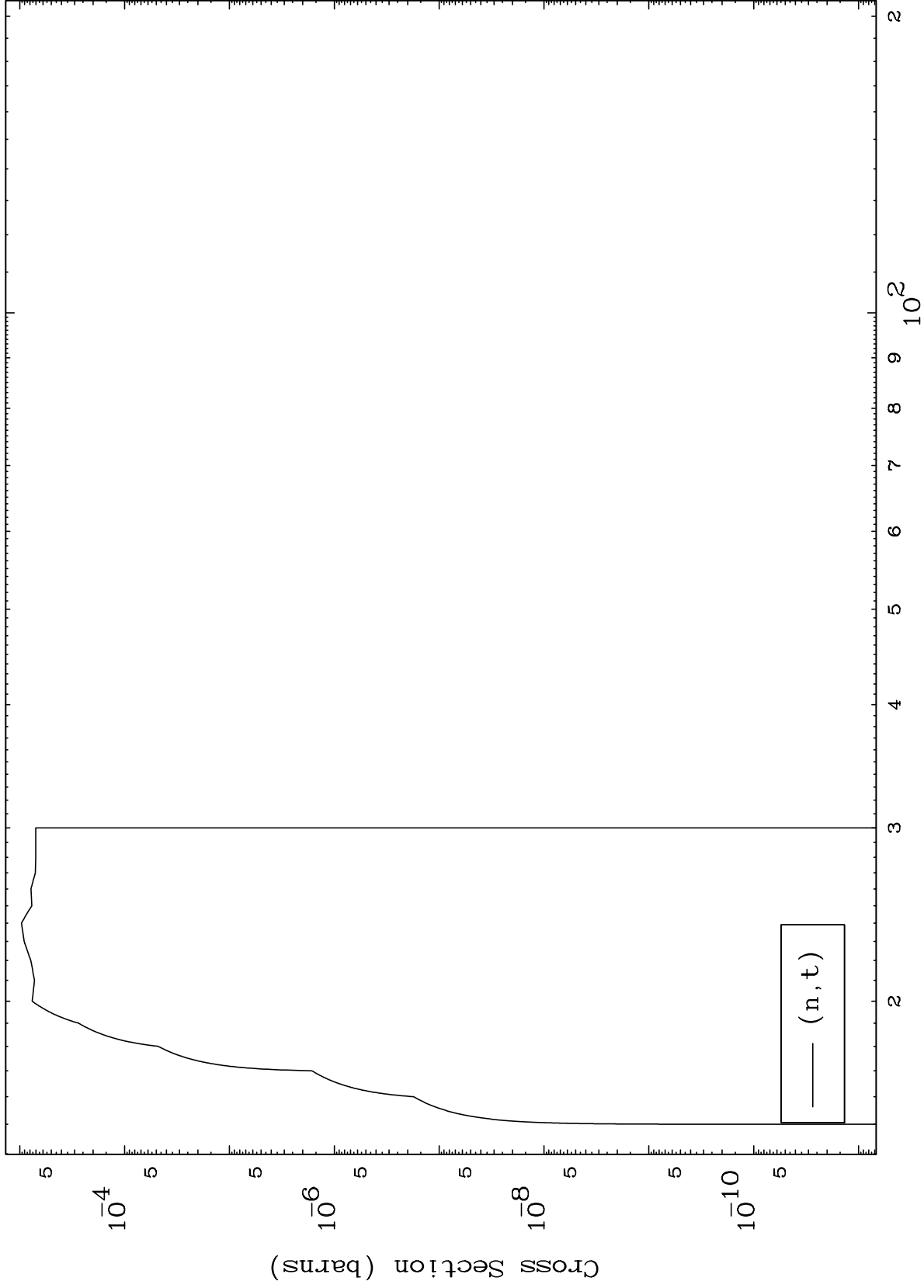
14-Si-26



MAT 1419

(n,t) Levels
293 Kelvin Cross Sections

14-Si-26



9

14-Si-26

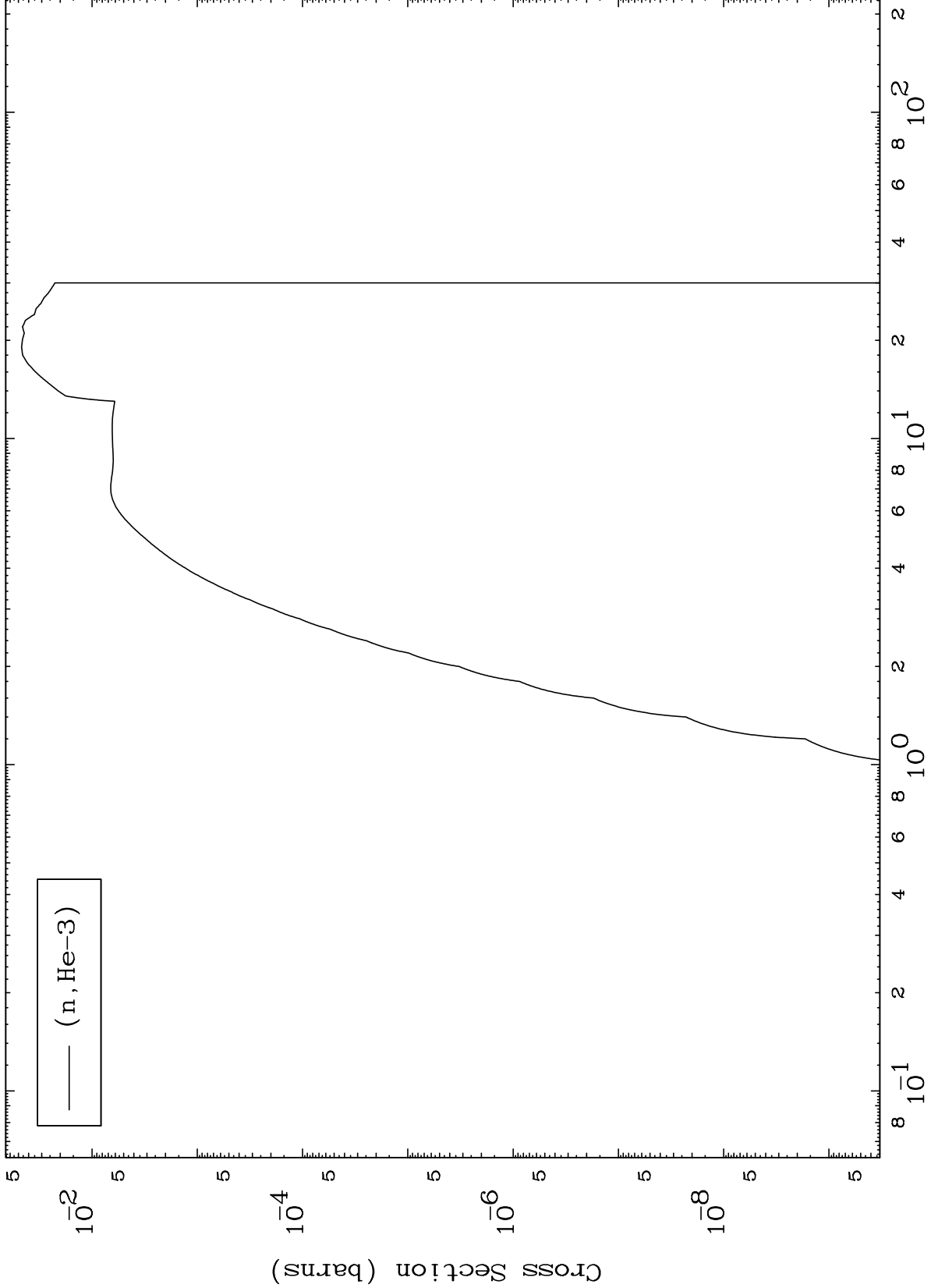
14-Si-26

MAT 1419

(n,He3) Levels

14-Si-26

293 Kelvin Cross Sections



10

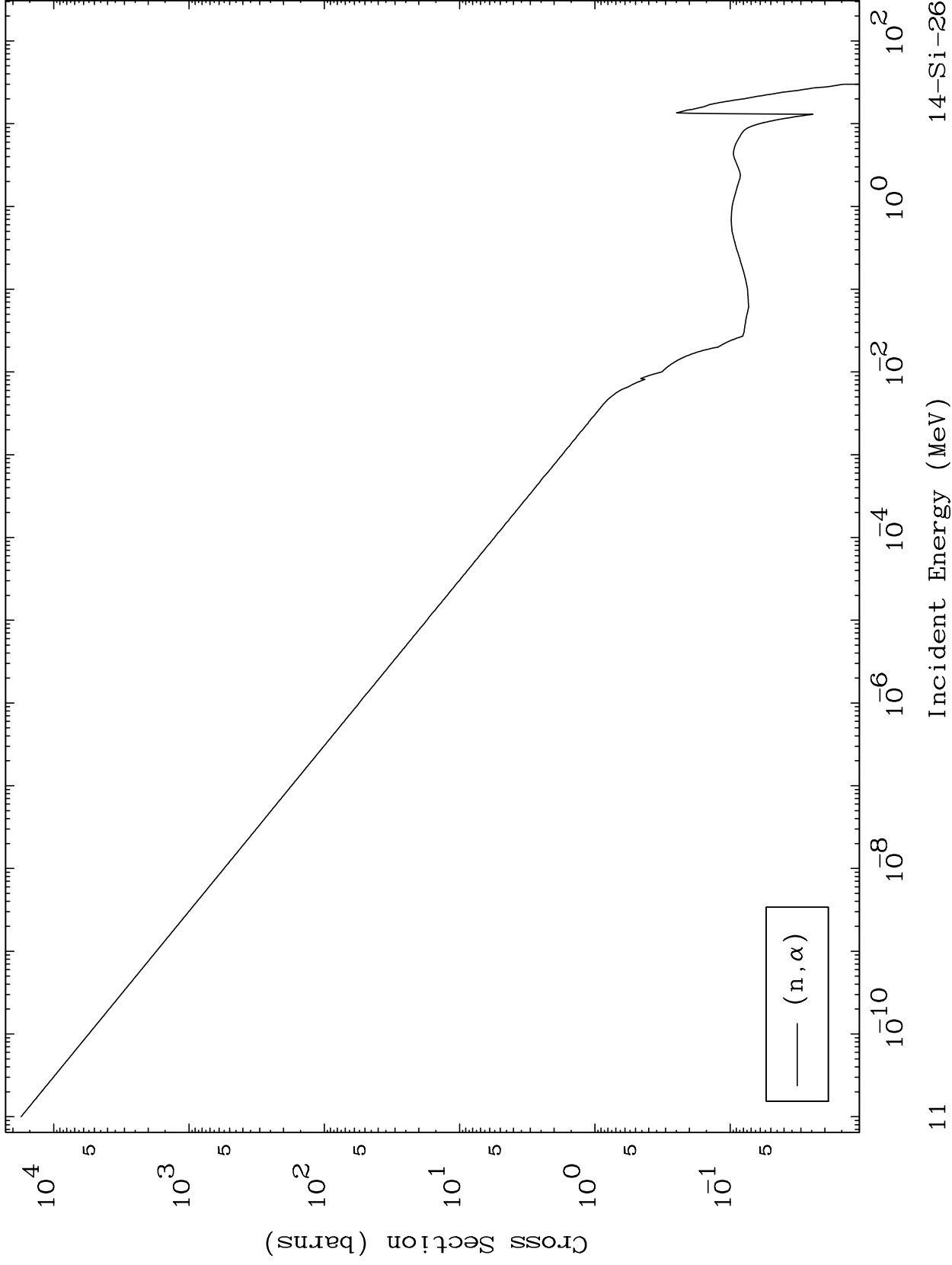
Incident Energy (MeV)

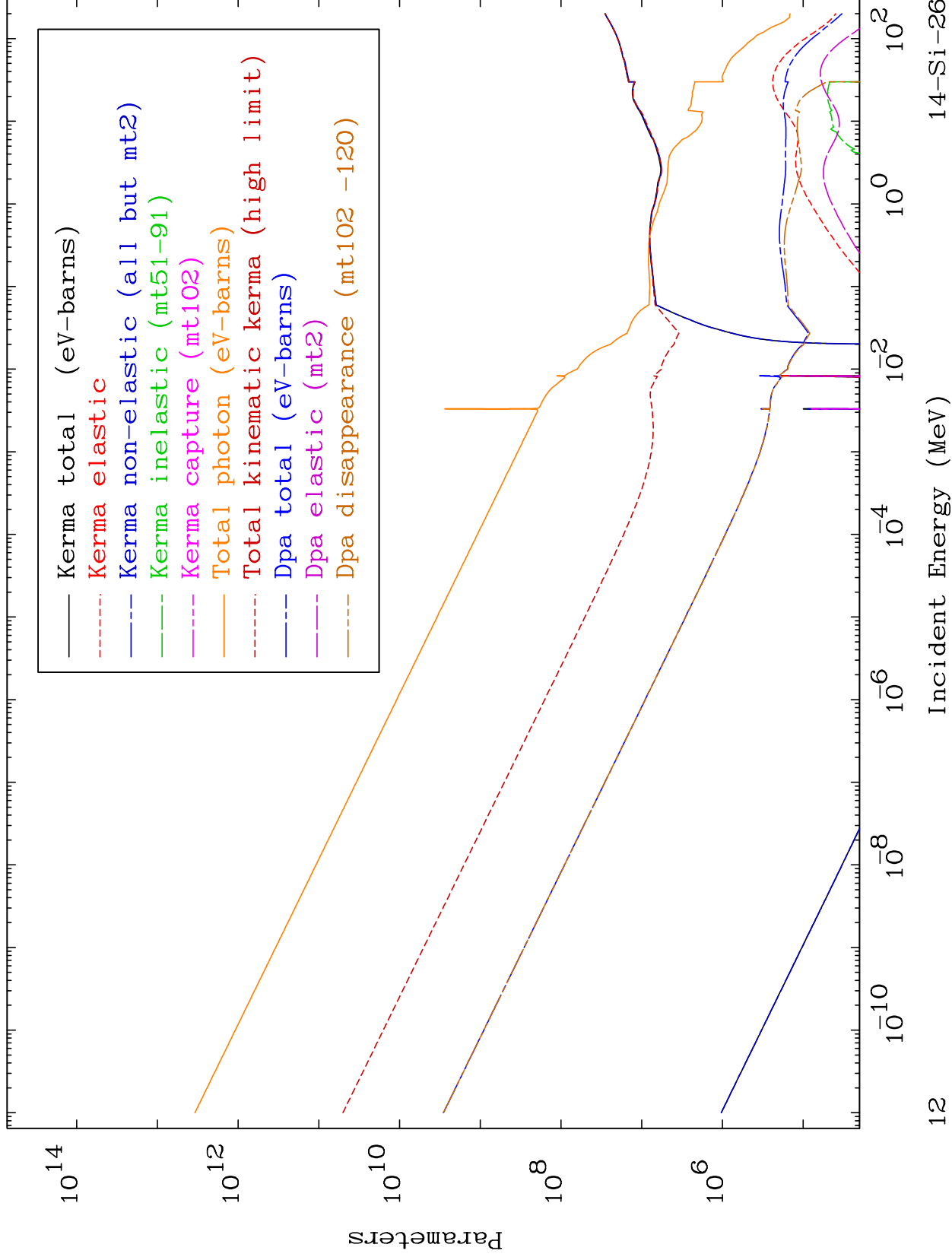
14-Si-26

MAT 1419

(n,α) Levels
293 Kelvin Cross Sections

14-Si-26

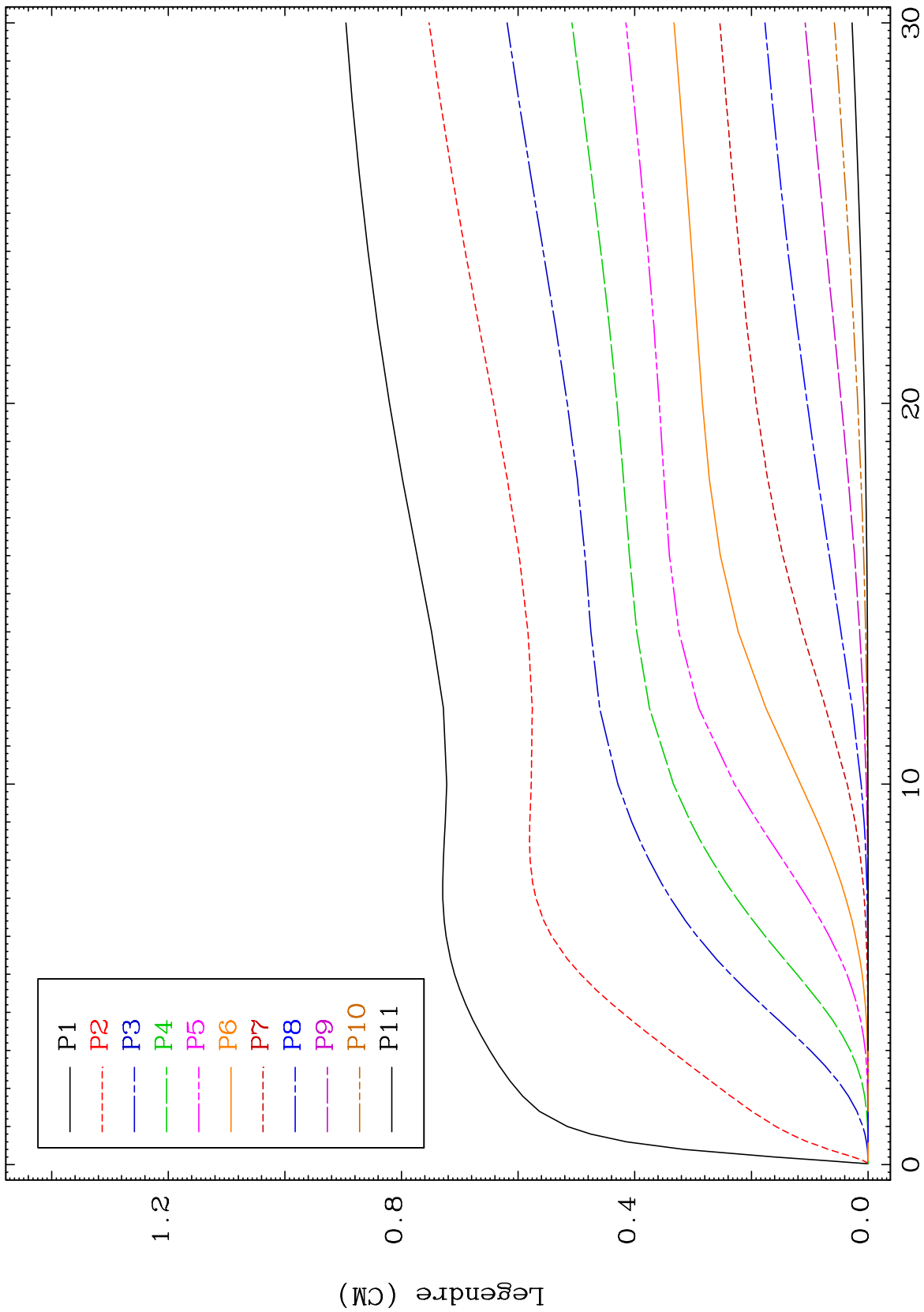




MAT 1419

Elastic Legendre Coefficients

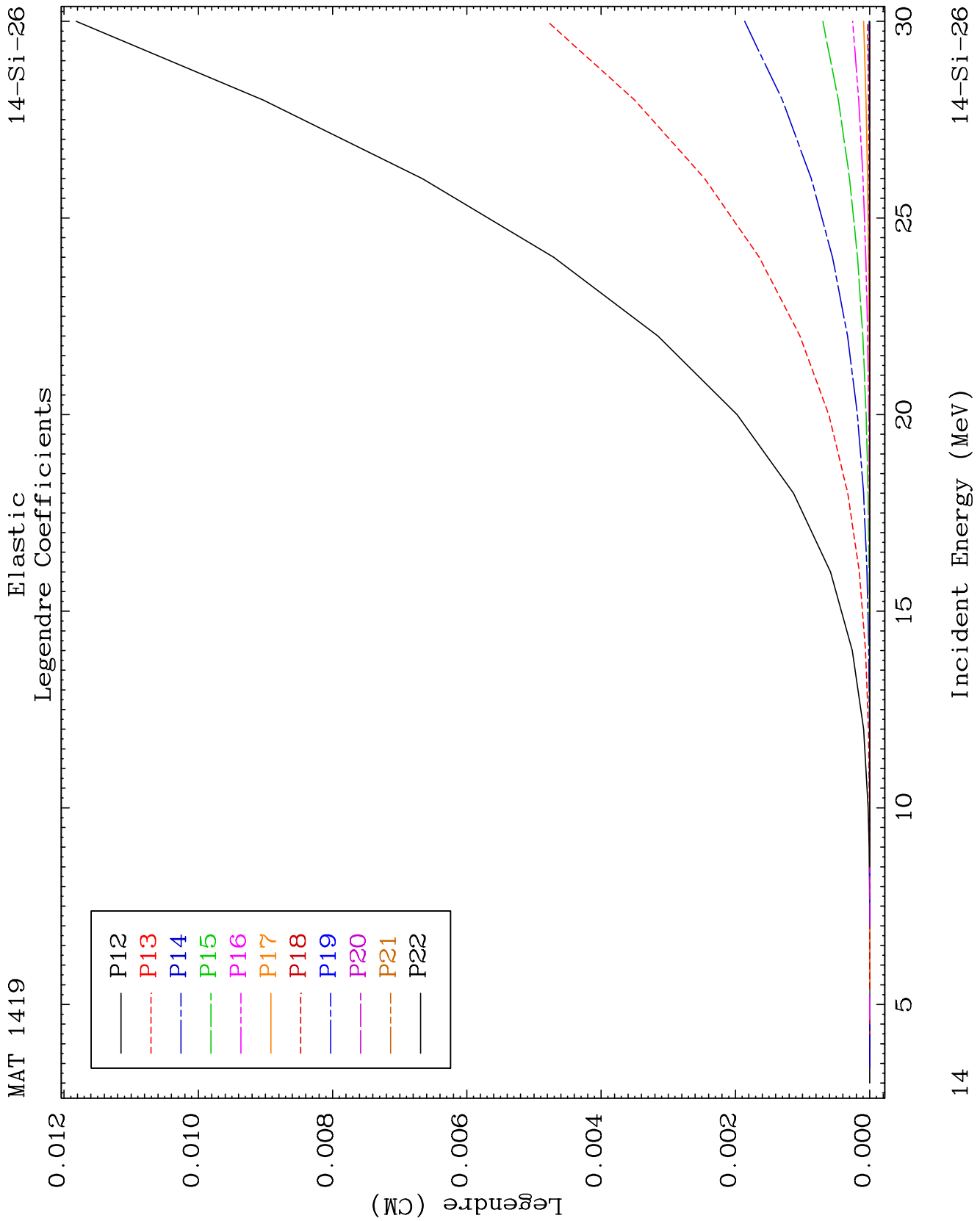
¹⁴Si-26



13

Incident Energy (MeV)

¹⁴Si-26



MAT 1419

14-Si-26

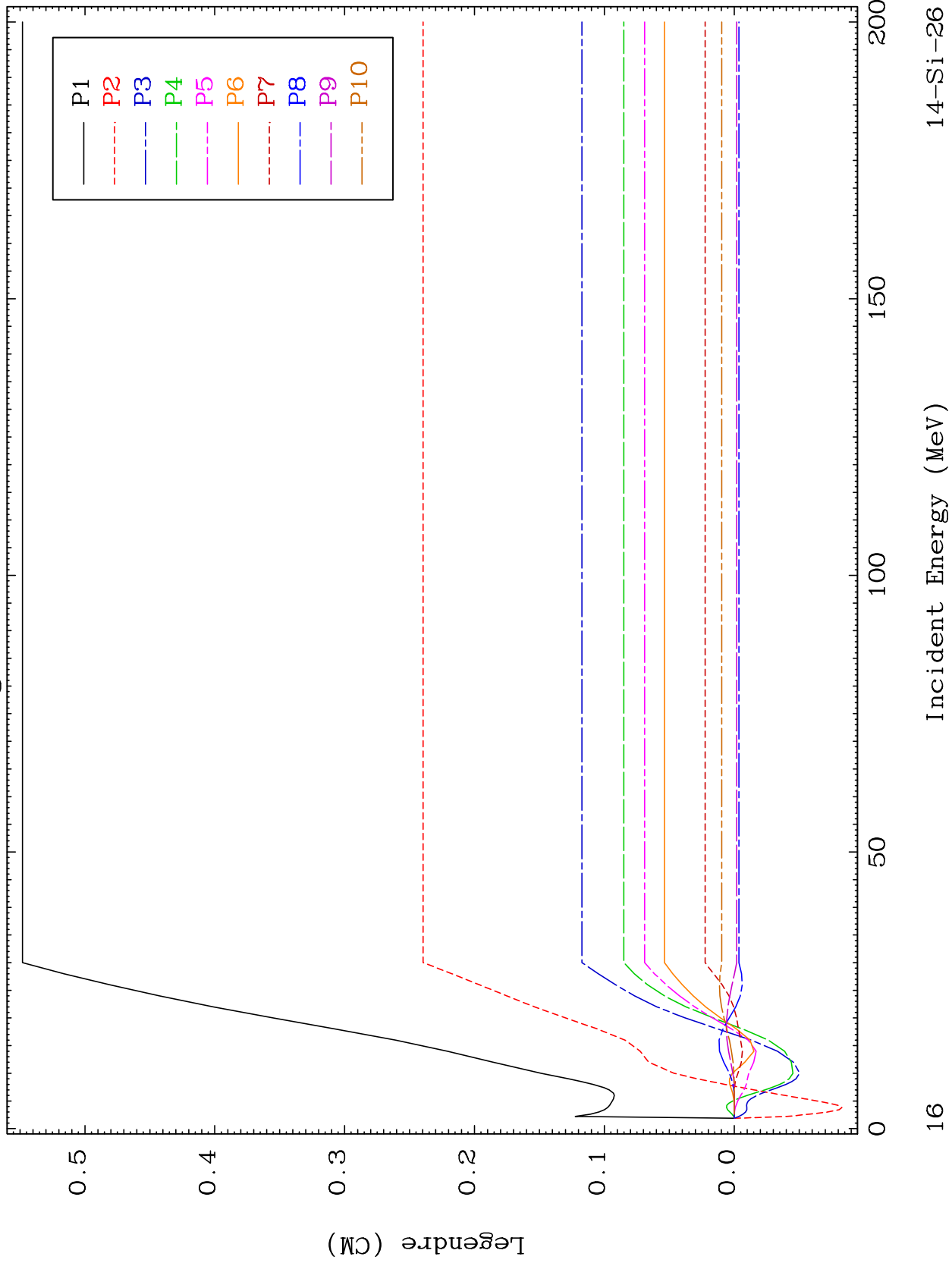
14

14-Si-26

MAT 1419

MT= 51 (n,n') Level
Legendre Coefficients

14-Si-26



16

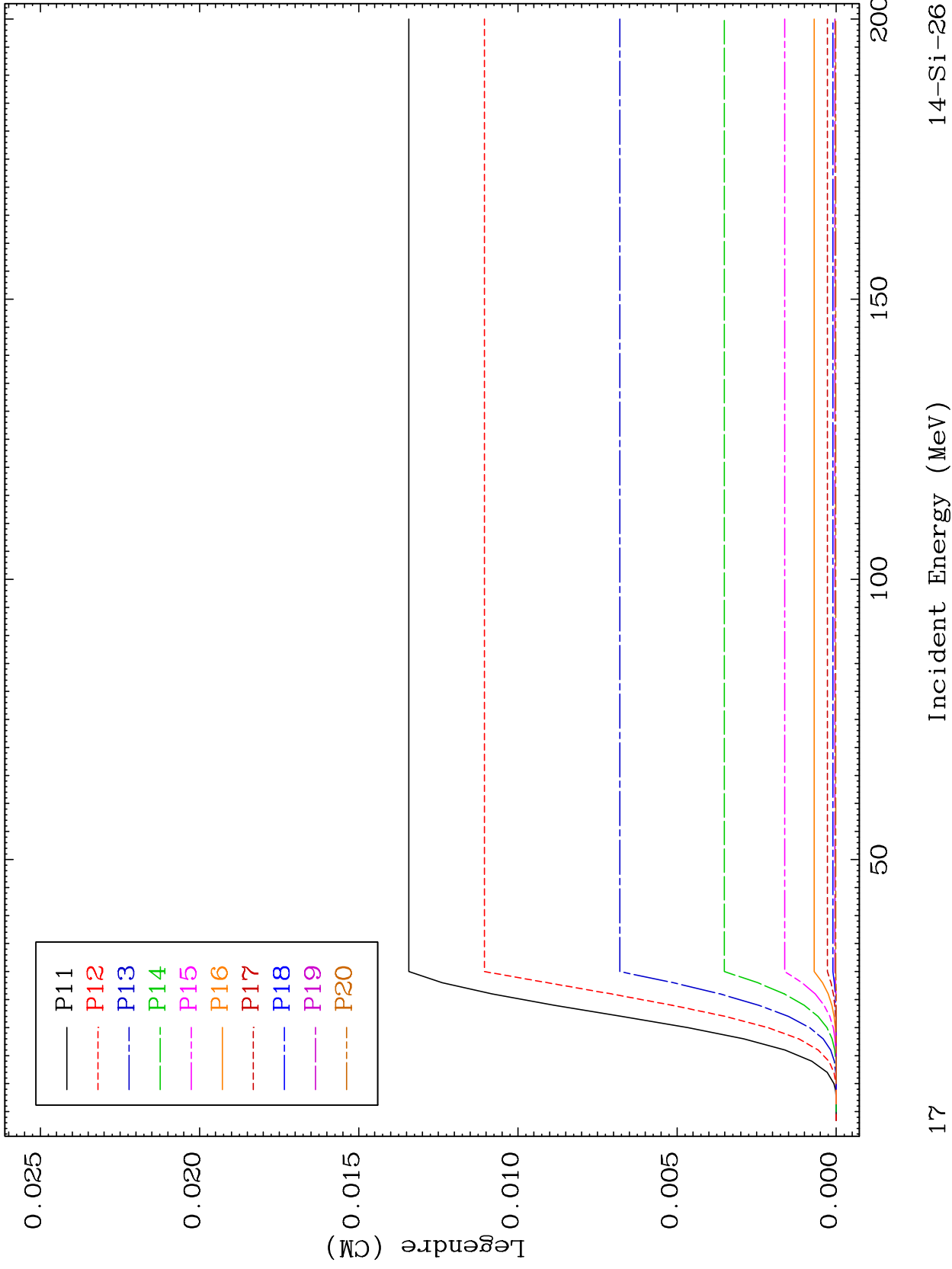
Incident Energy (MeV)

14-Si-26

MAT 1419

MT= 51 (n,n') Level
Legendre Coefficients

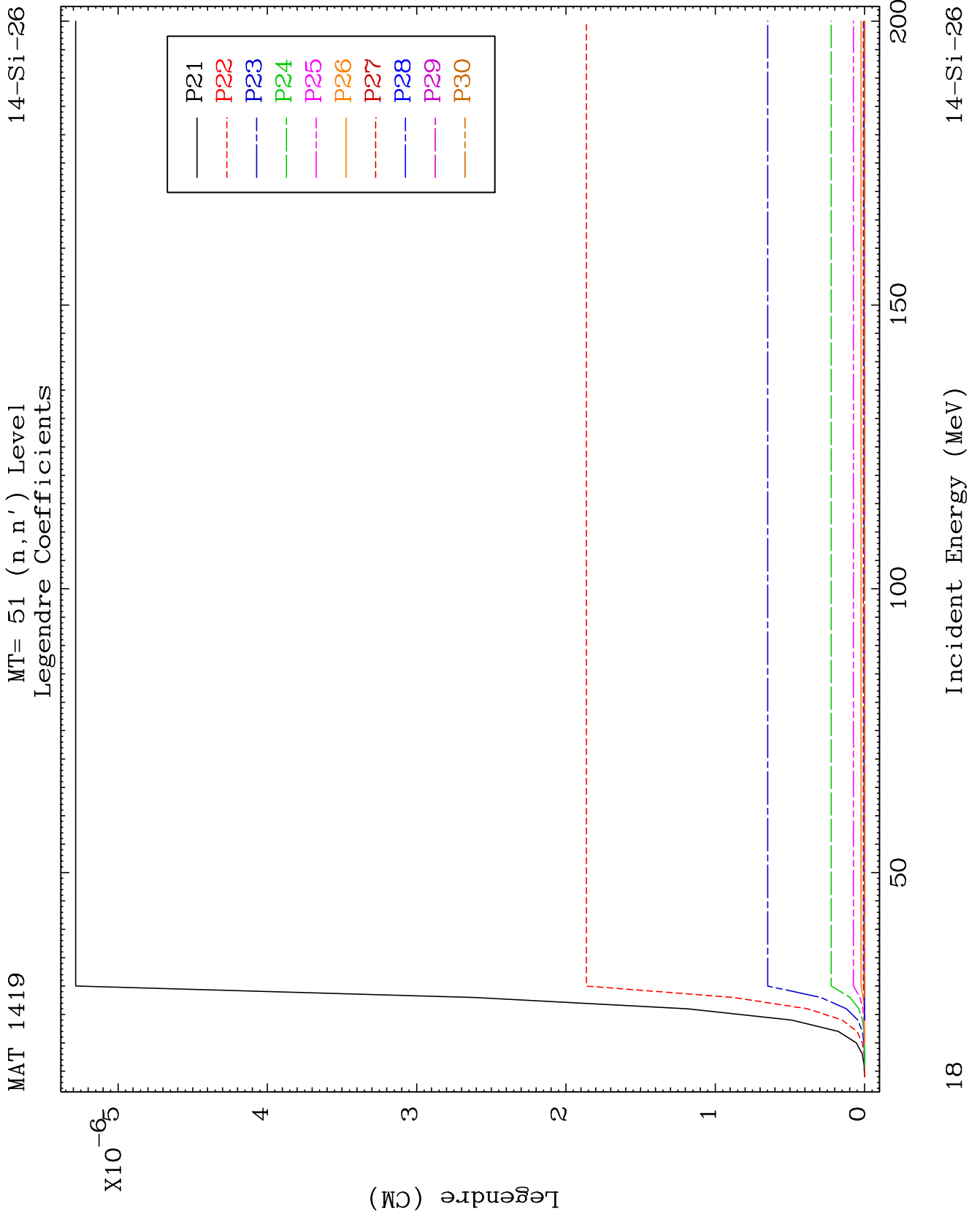
14-Si-26



17

Incident Energy (MeV)

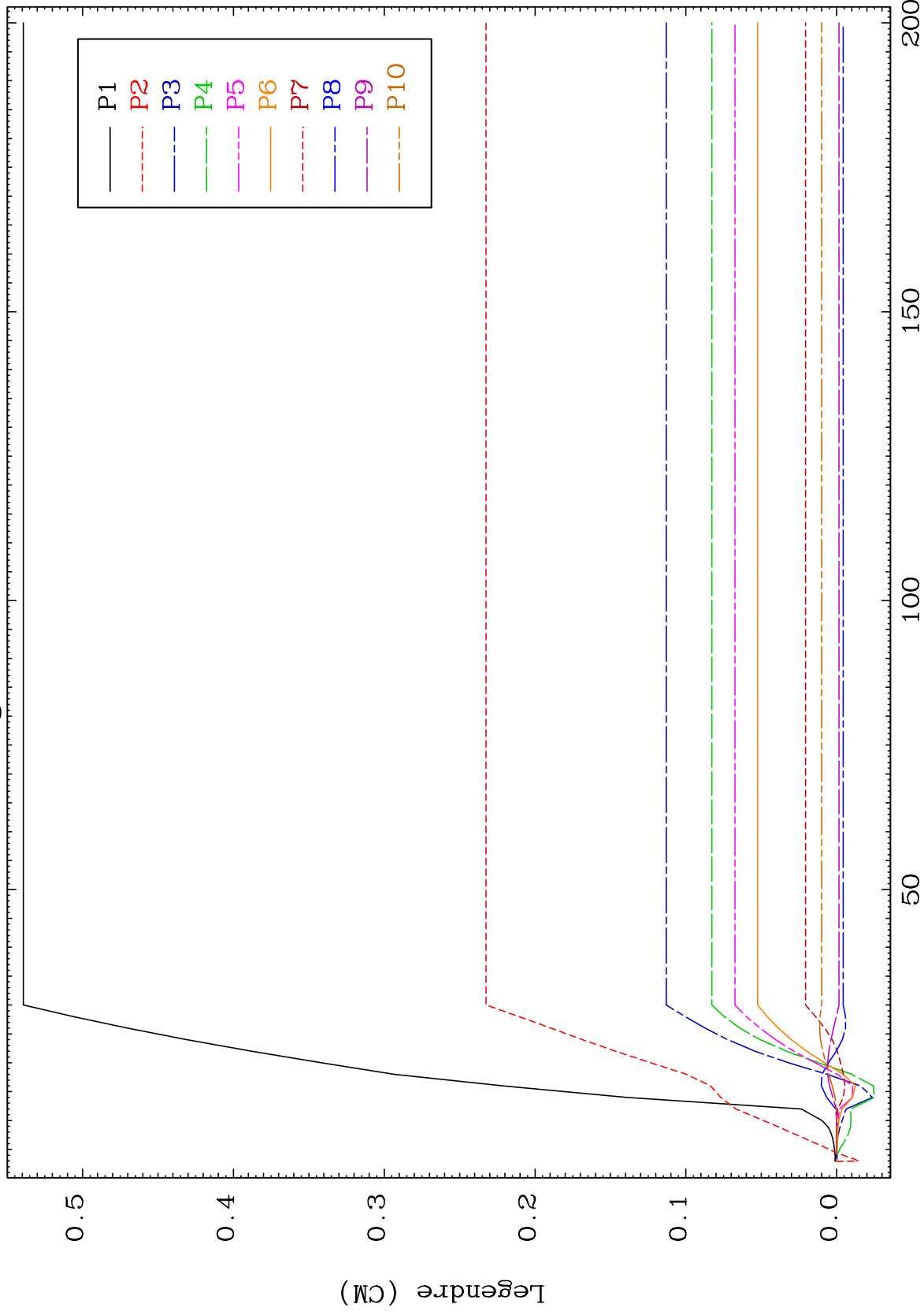
14-Si-26



MAT 1419

MT= 52 (n,n') Level
Legendre Coefficients

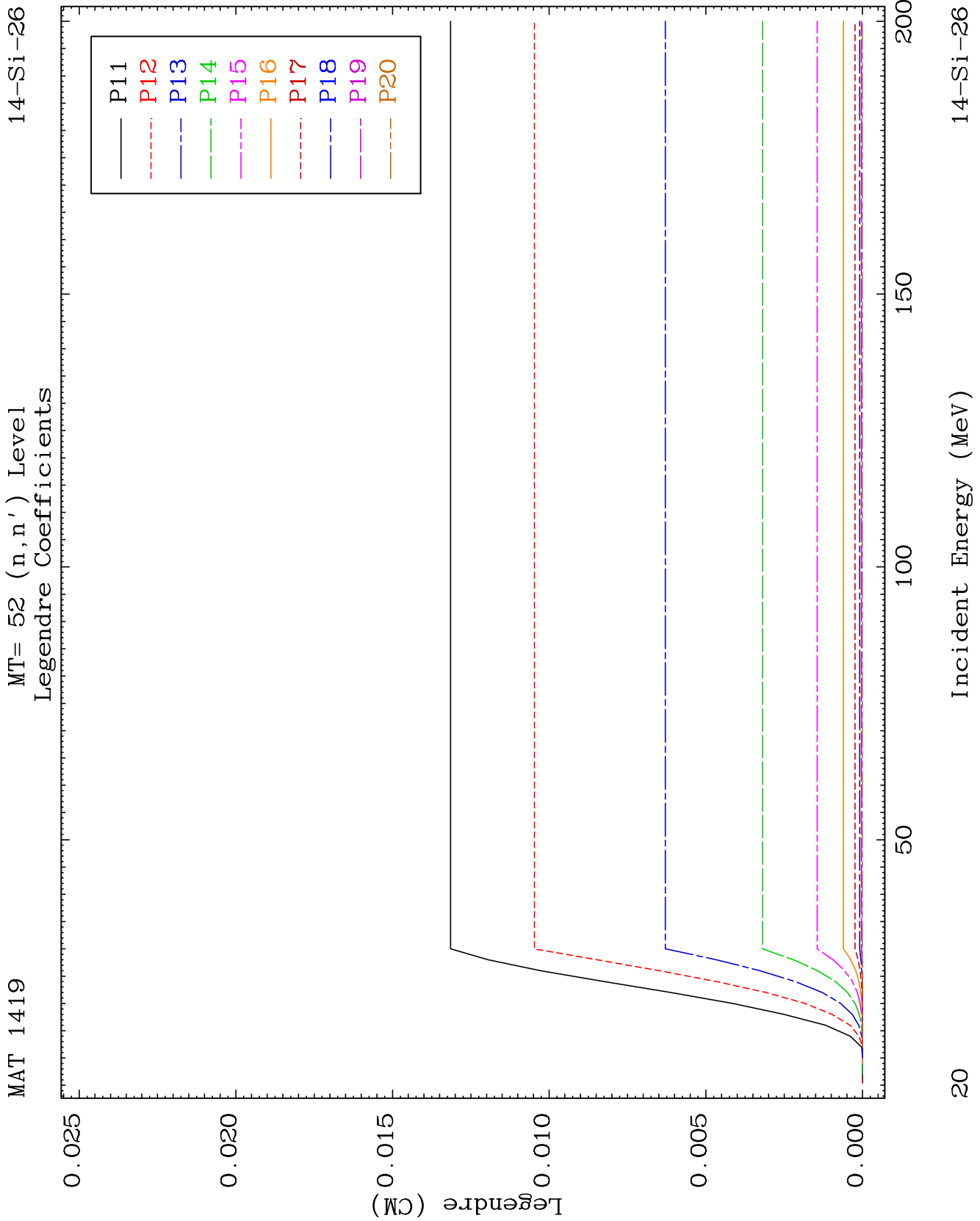
14-Si-26



19

Incident Energy (MeV)

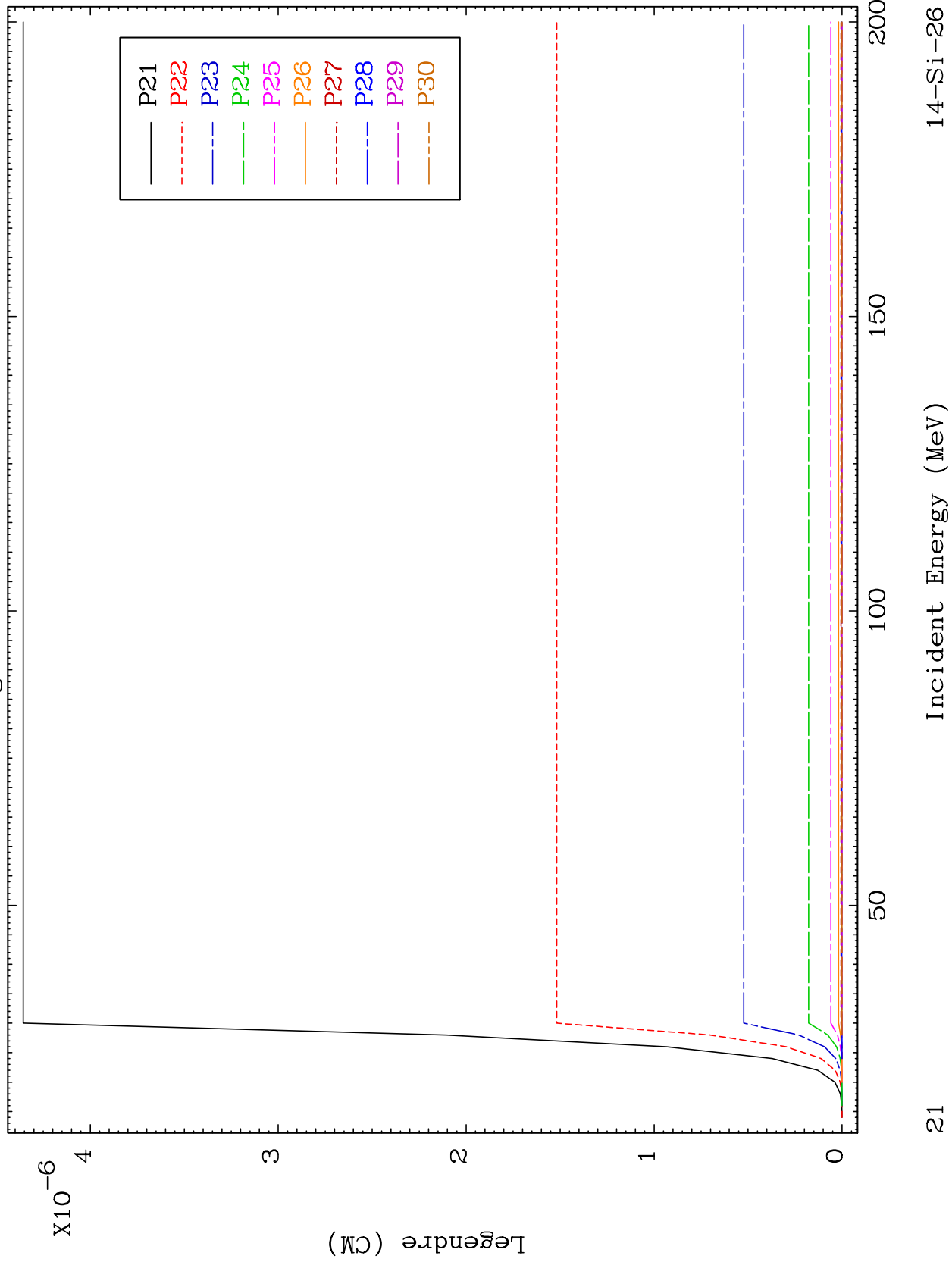
14-Si-26



MAT 1419

MT= 52 (n,n') Level
Legendre Coefficients

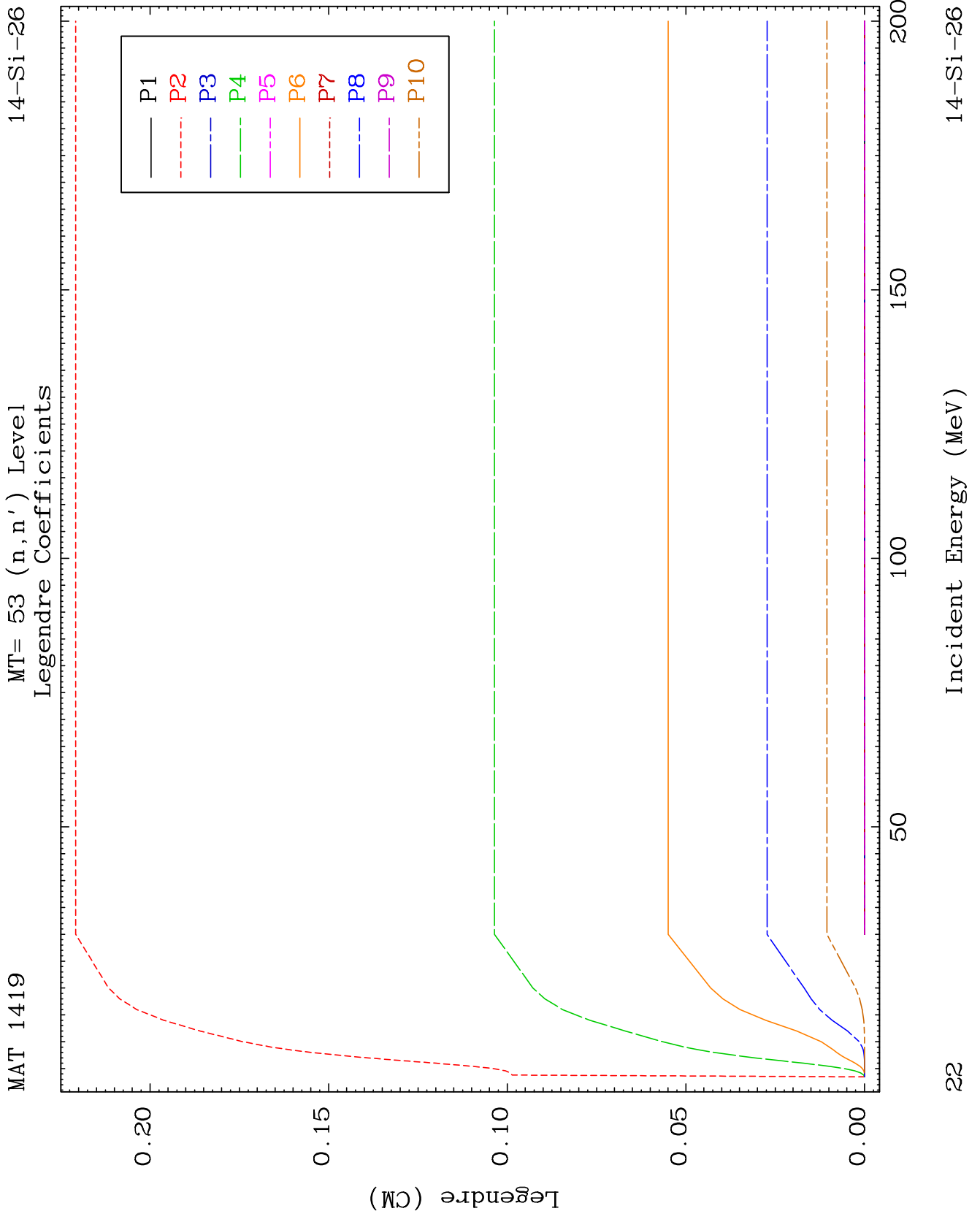
14-Si-26

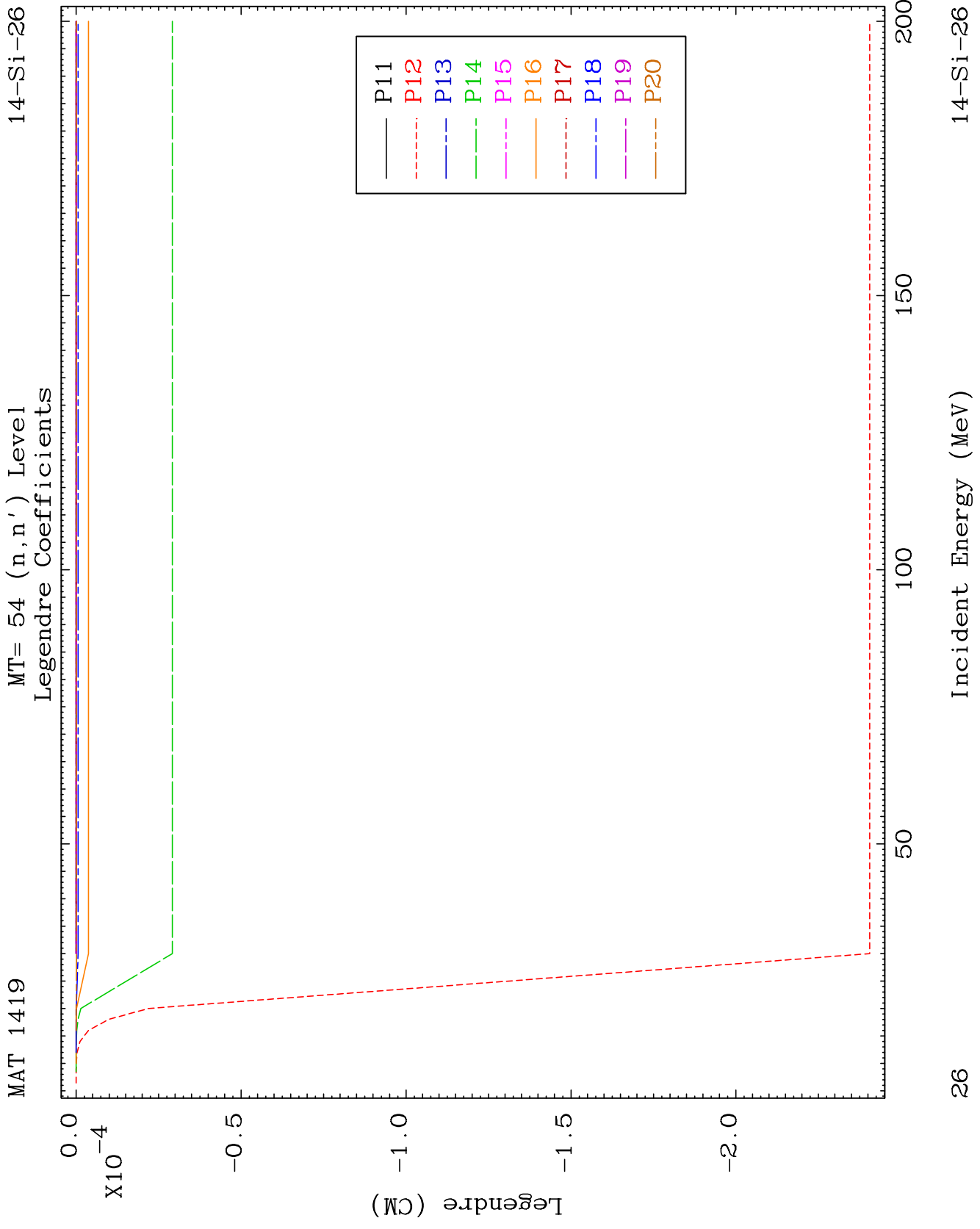


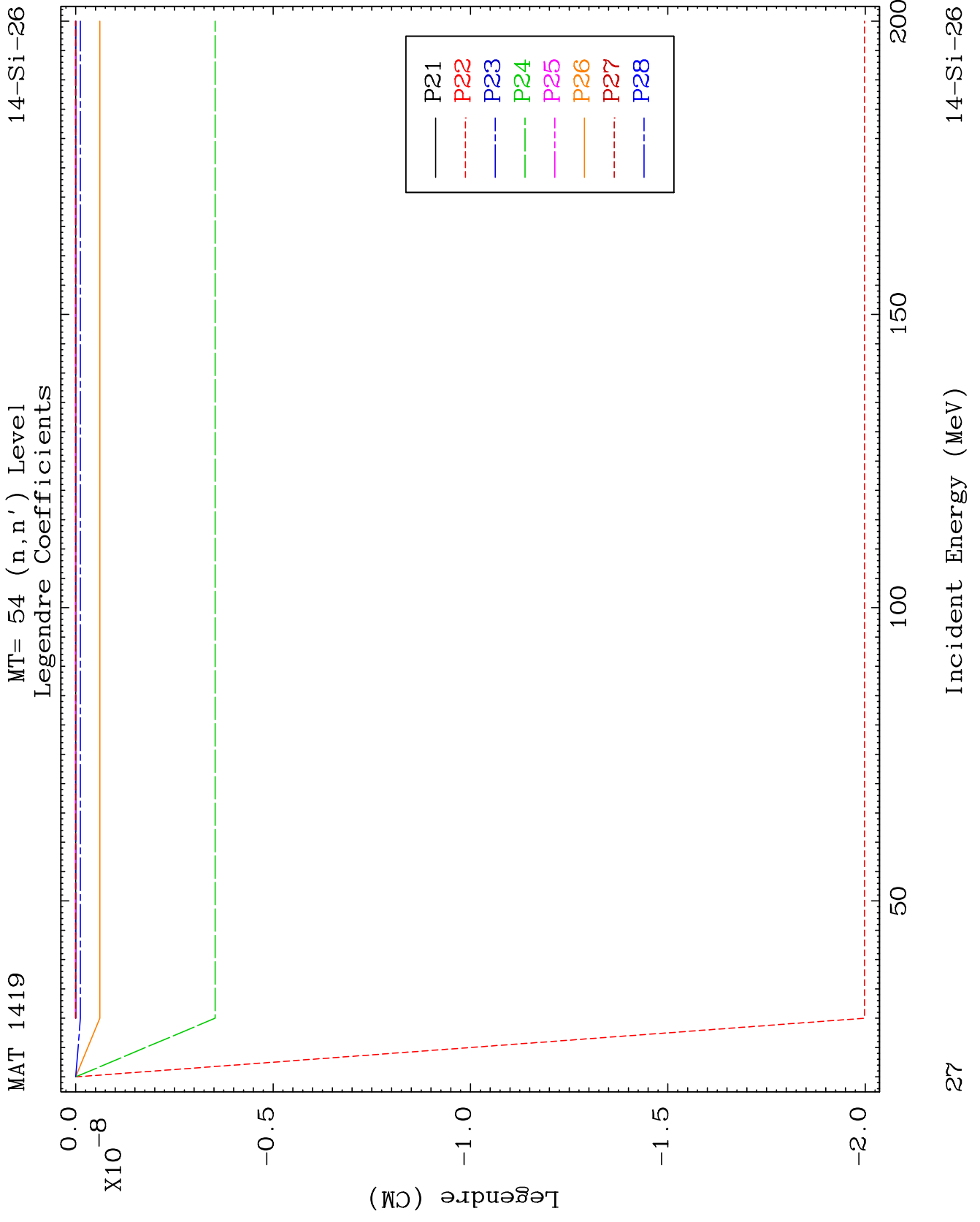
21

Incident Energy (MeV)

14-Si-26



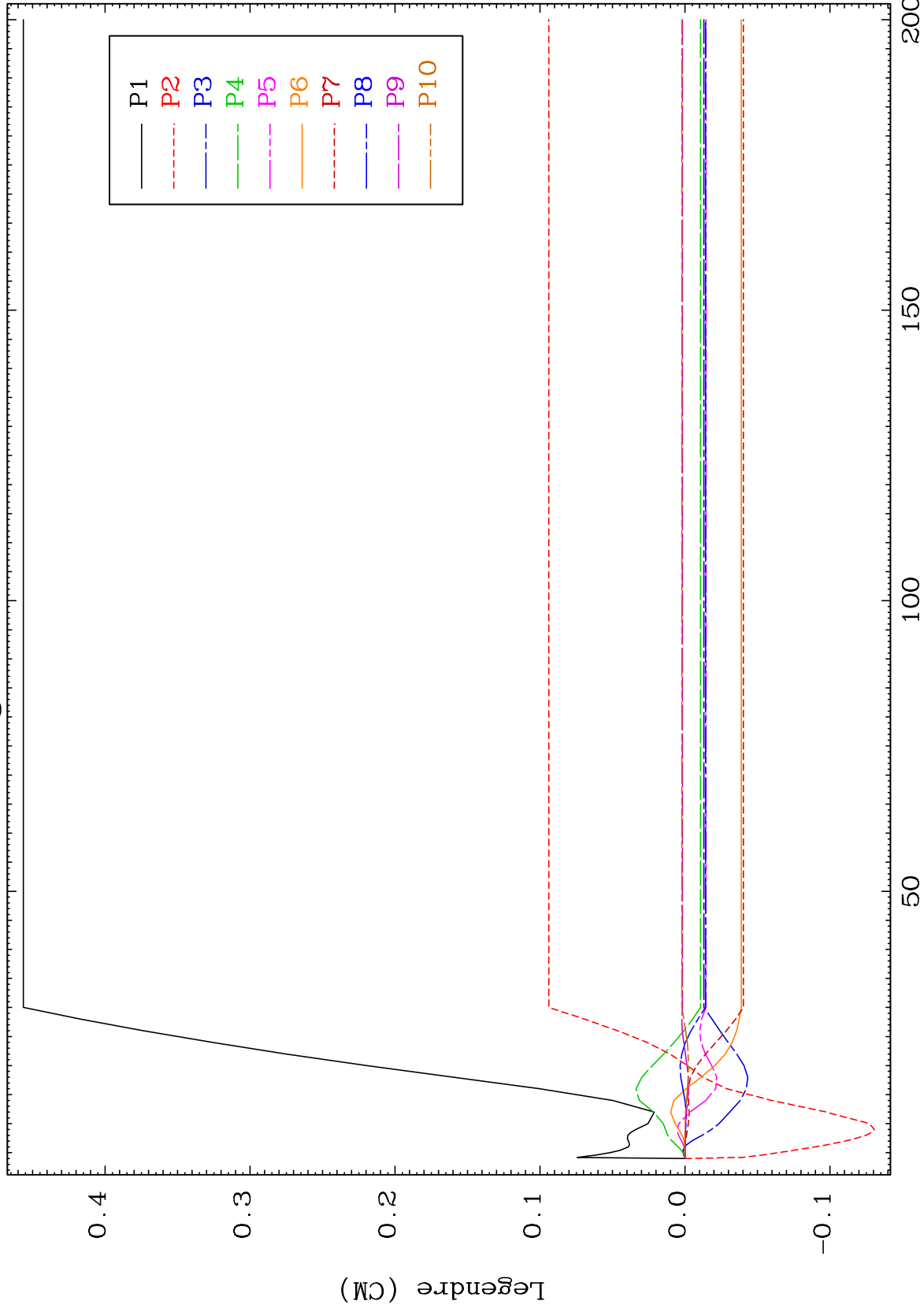




MAT 1419

MT= 55 (n,n') Level
Legendre Coefficients

14-Si-26



28

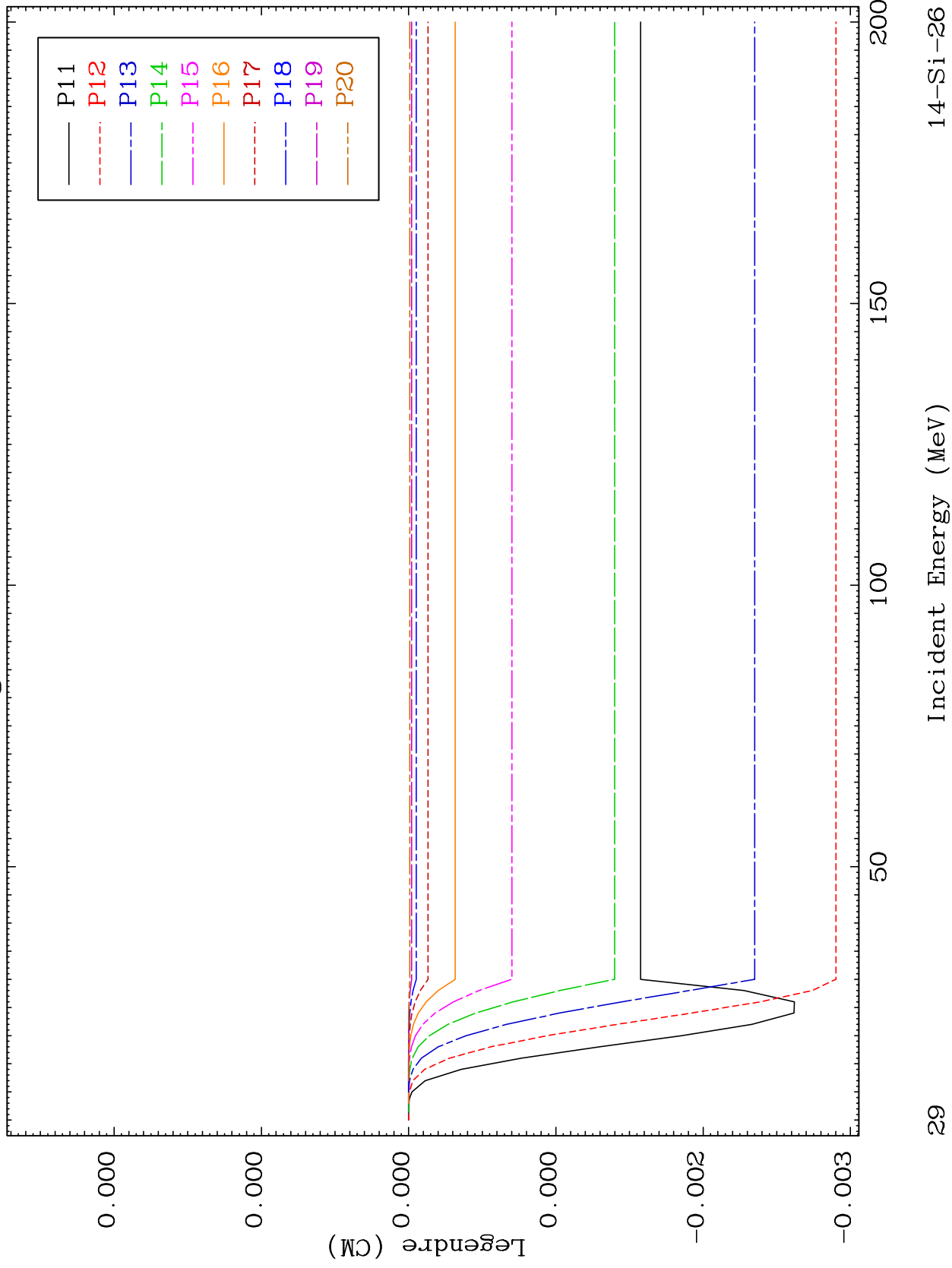
Incident Energy (MeV)

14-Si-26

MAT 1419

MT= 55 (n,n') Level
Legendre Coefficients

14-Si-26

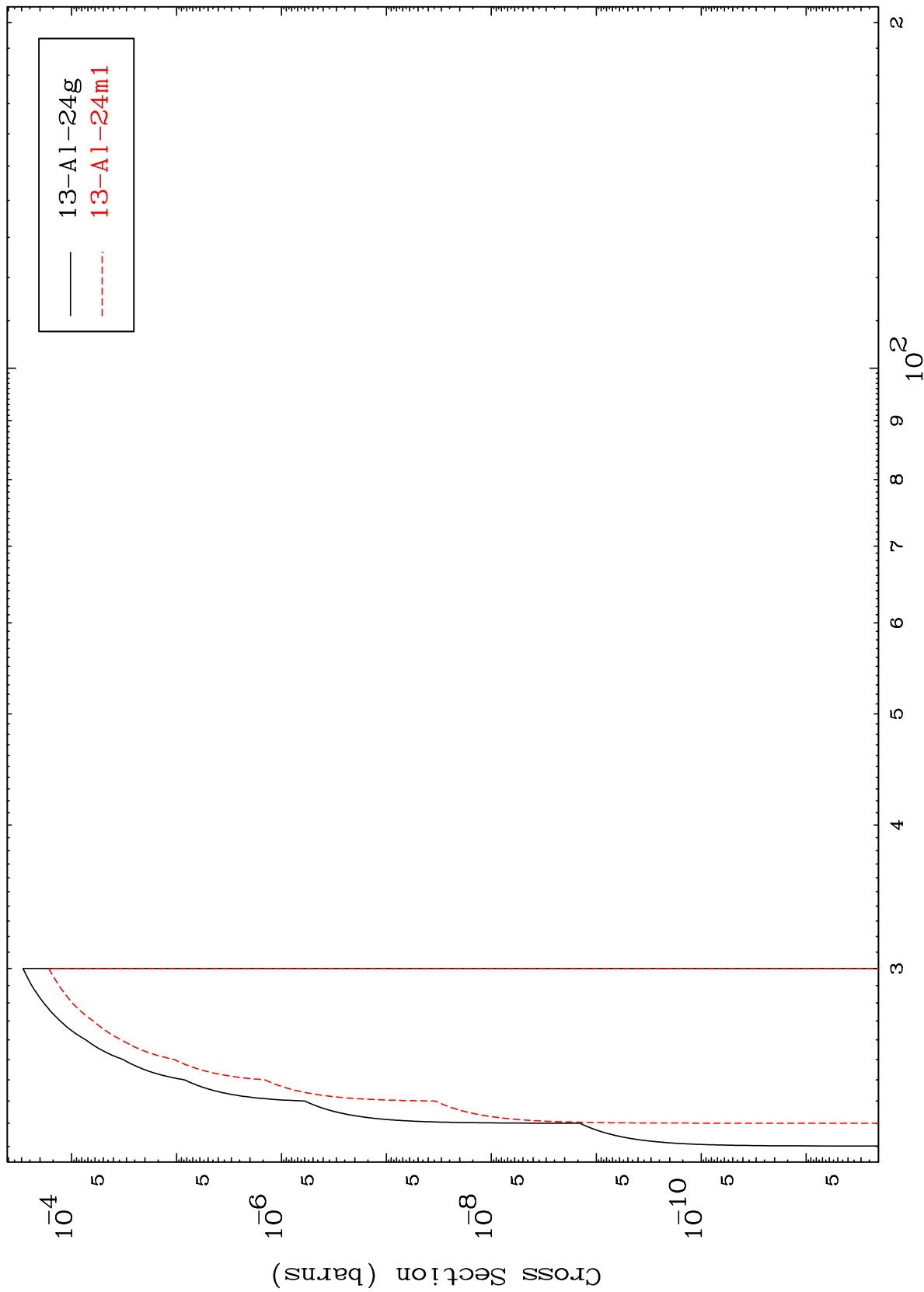


29

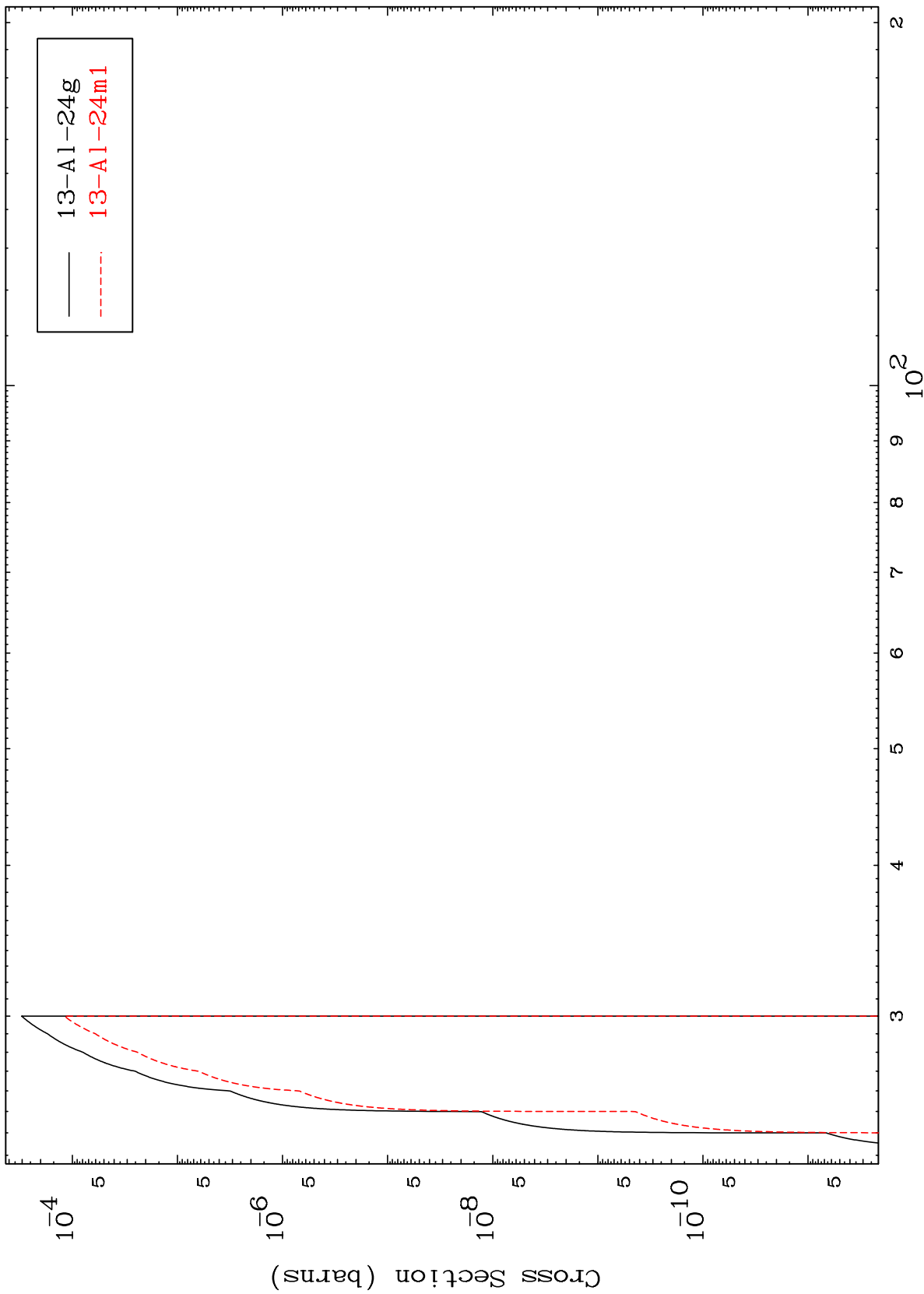
Incident Energy (MeV)

14-Si-26

Radionuclide Production Cross Section

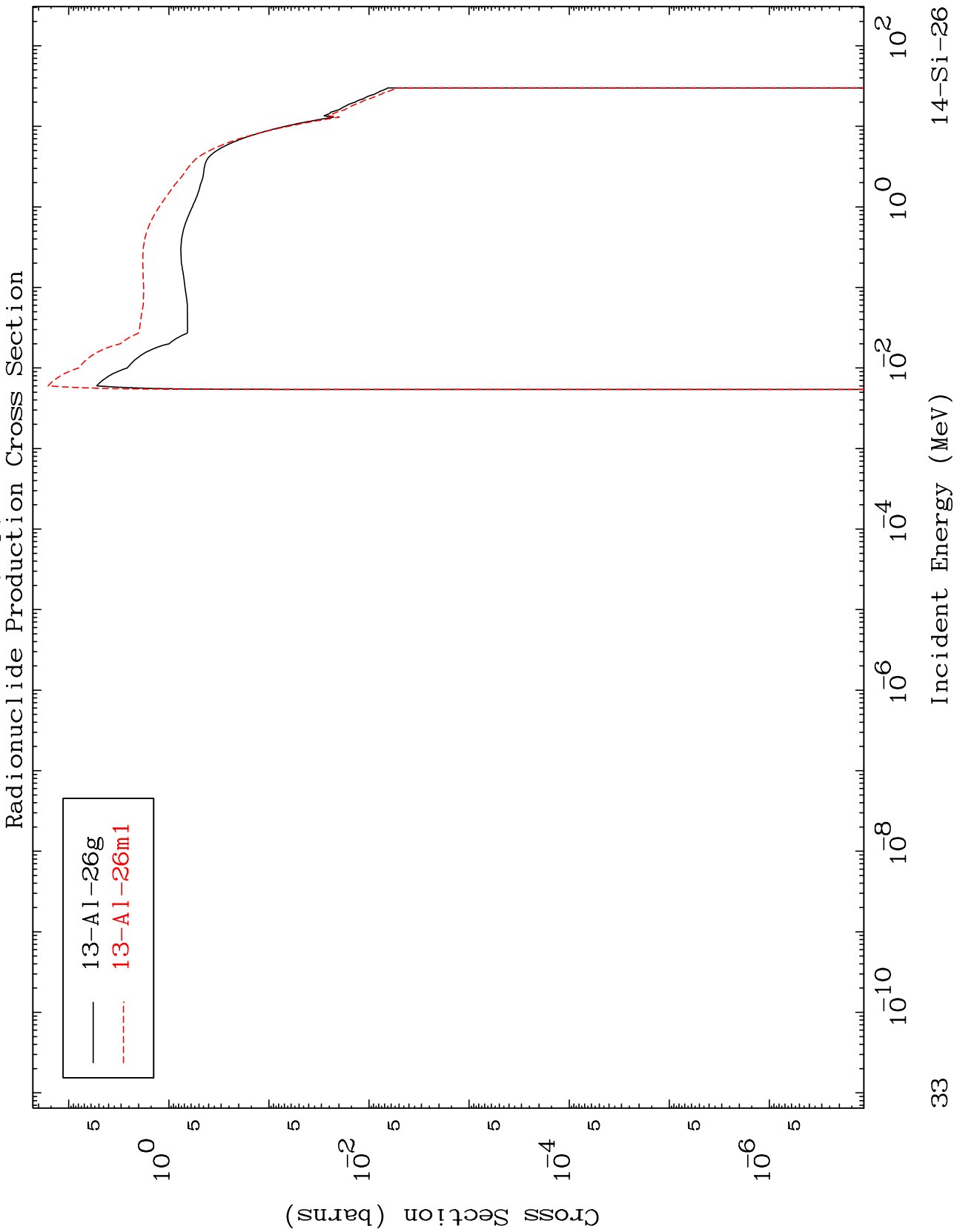


Radionuclide Production Cross Section



MAT 1419

14-Si-26



— 13-Al-26g
- - - 13-Al-26m1

(n,t)
Radionuclide Production Cross Section

