

Program EVALPLOT  
(Version 2018-1)

by

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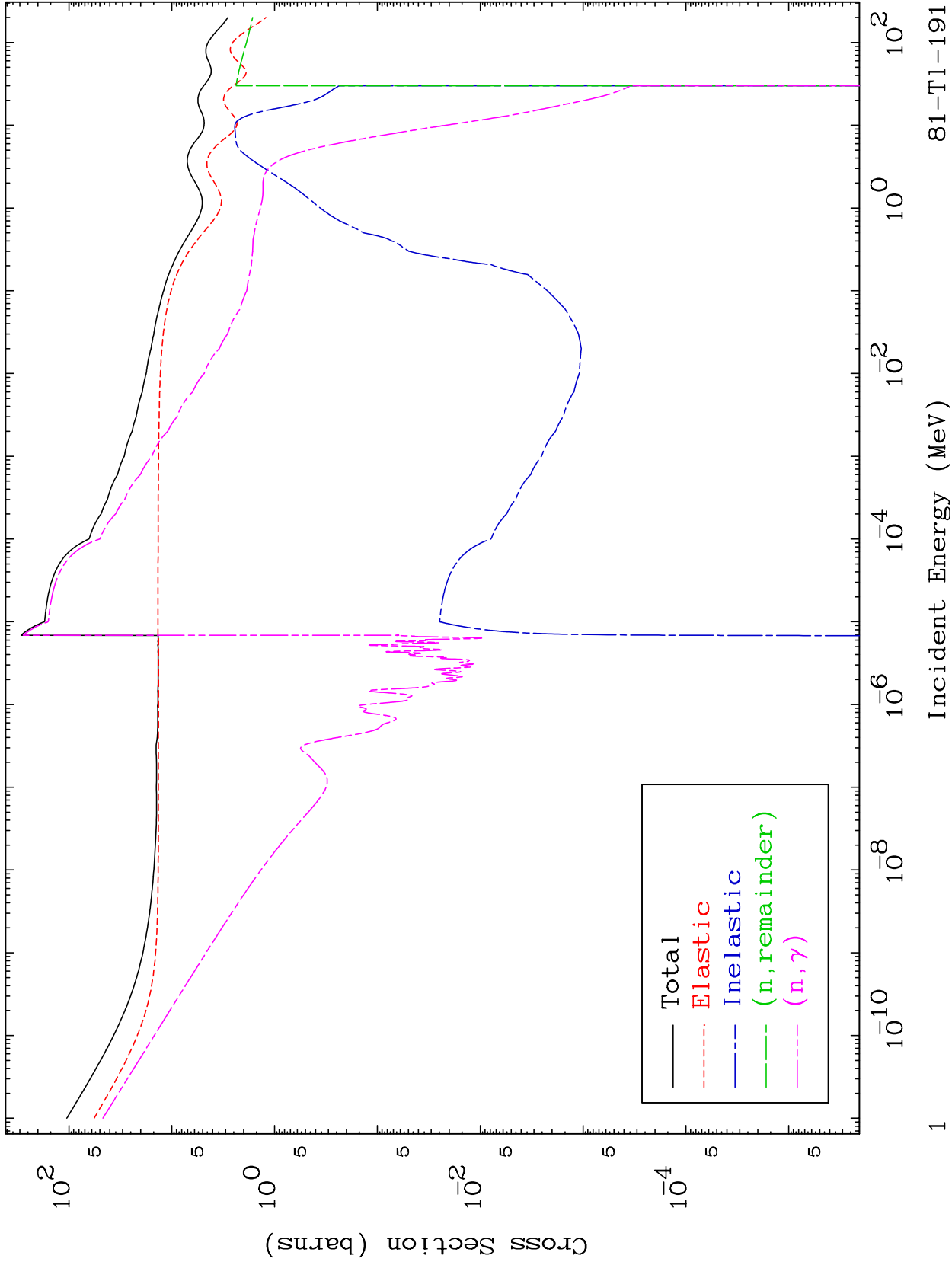
Press Mouse Button to Start

MAT 8090

Major

293 Kelvin Cross Sections

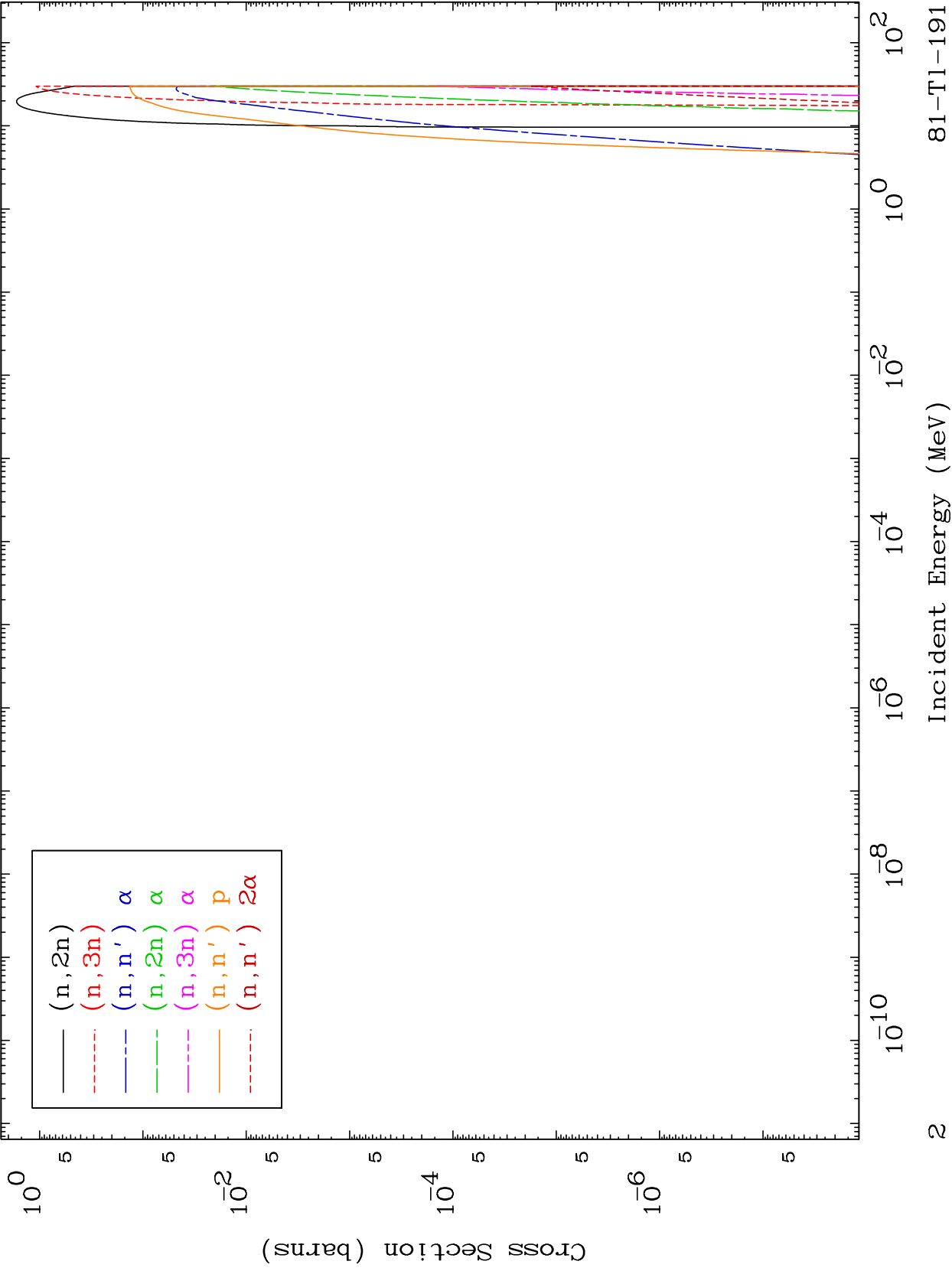
81-Tl-191



MAT 8090

Neutron Production  
293 Kelvin Cross Sections

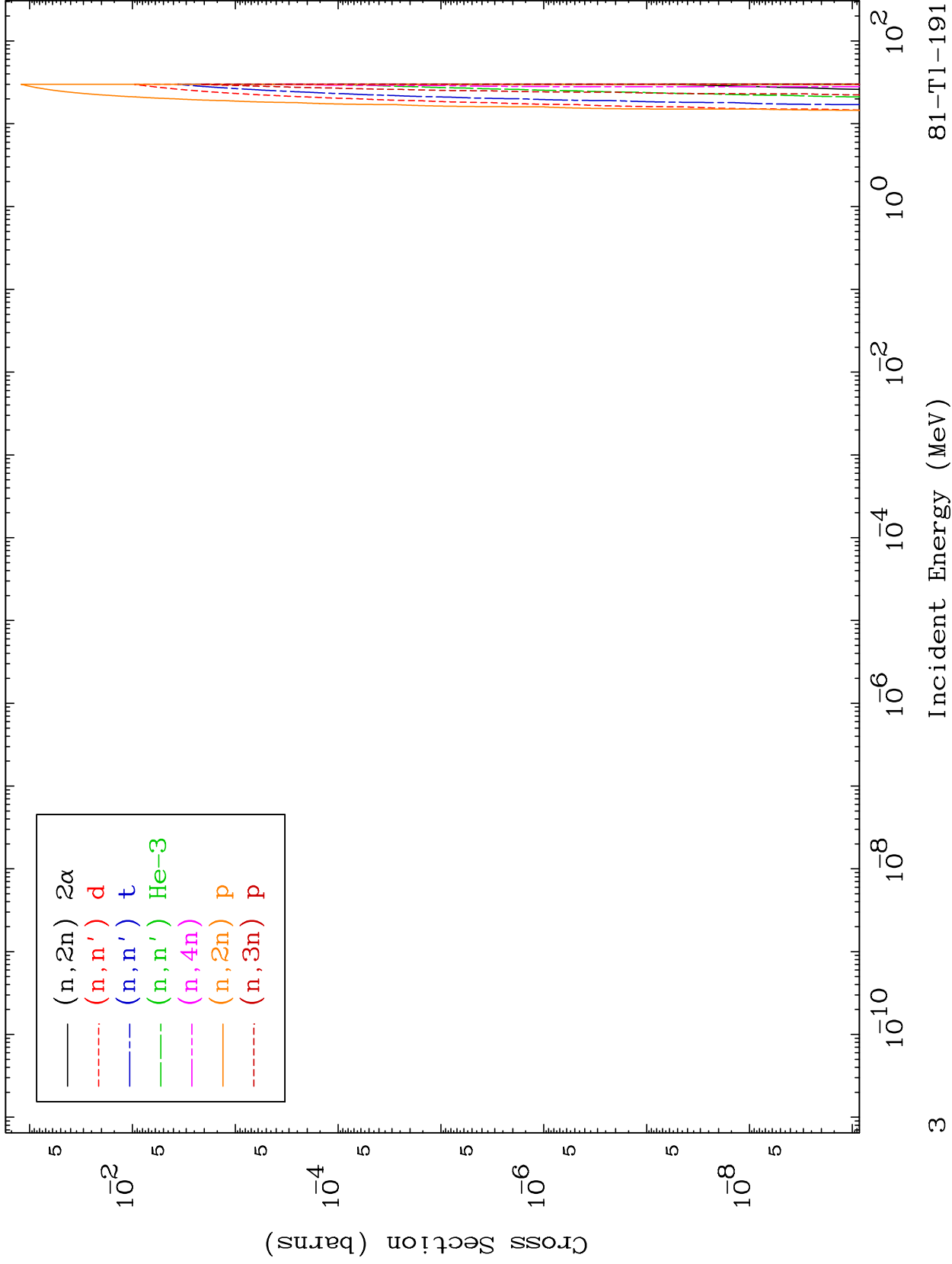
81-Tl-191



MAT 8090

Neutron Production  
293 Kelvin Cross Sections

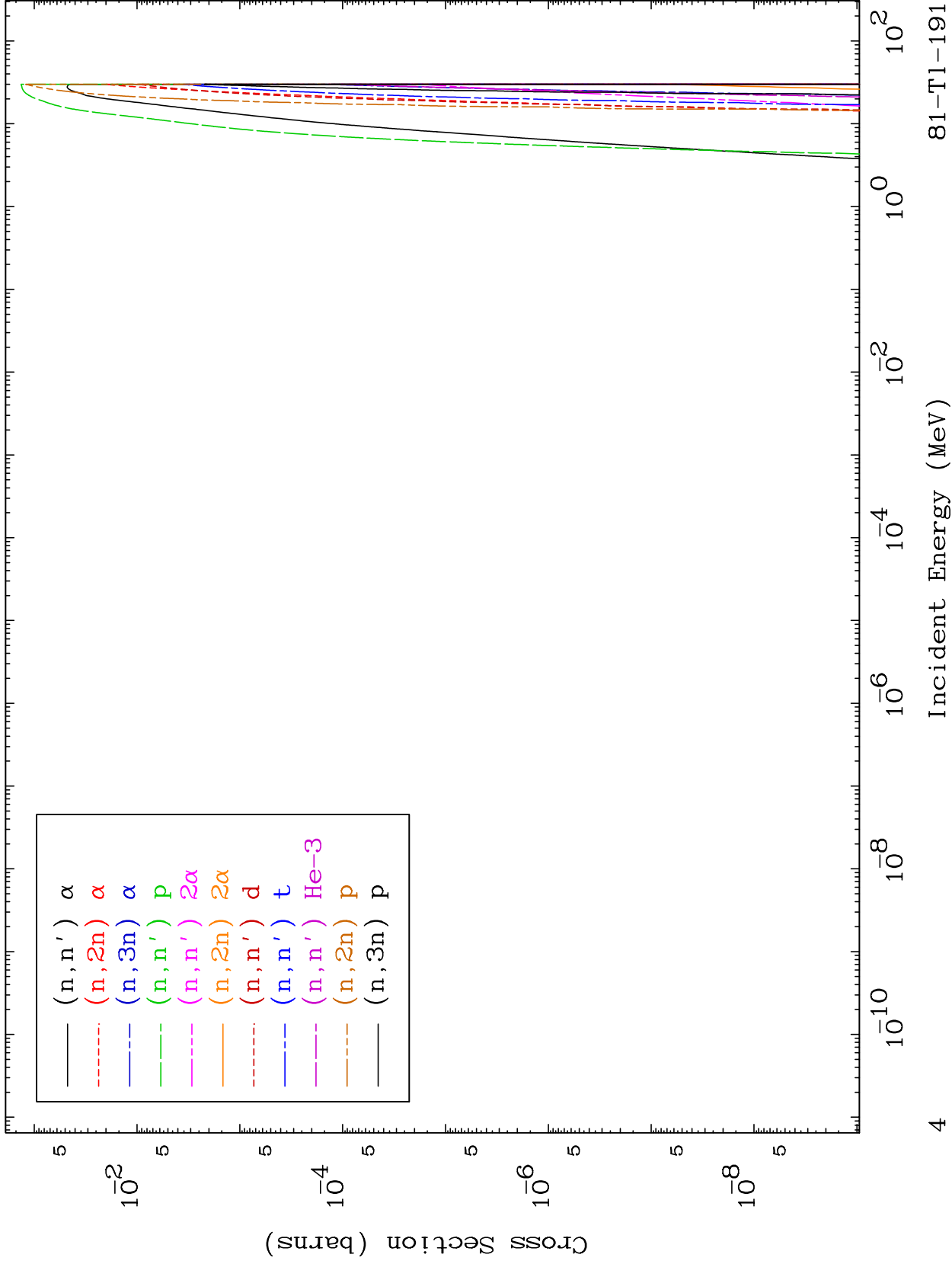
81-Tl-191



MAT 8090

Charged Particle  
293 Kelvin Cross Sections

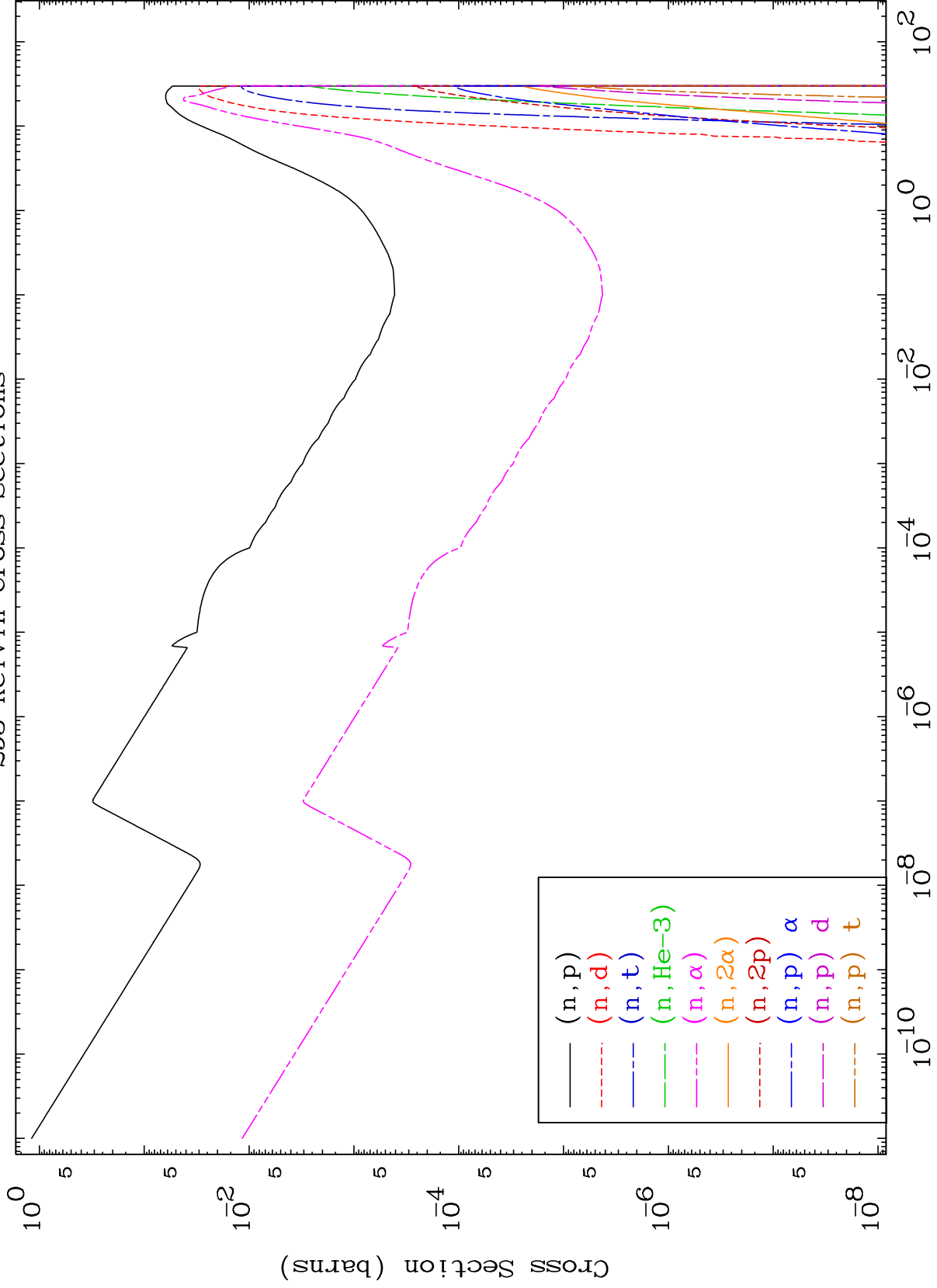
81-Tl-191

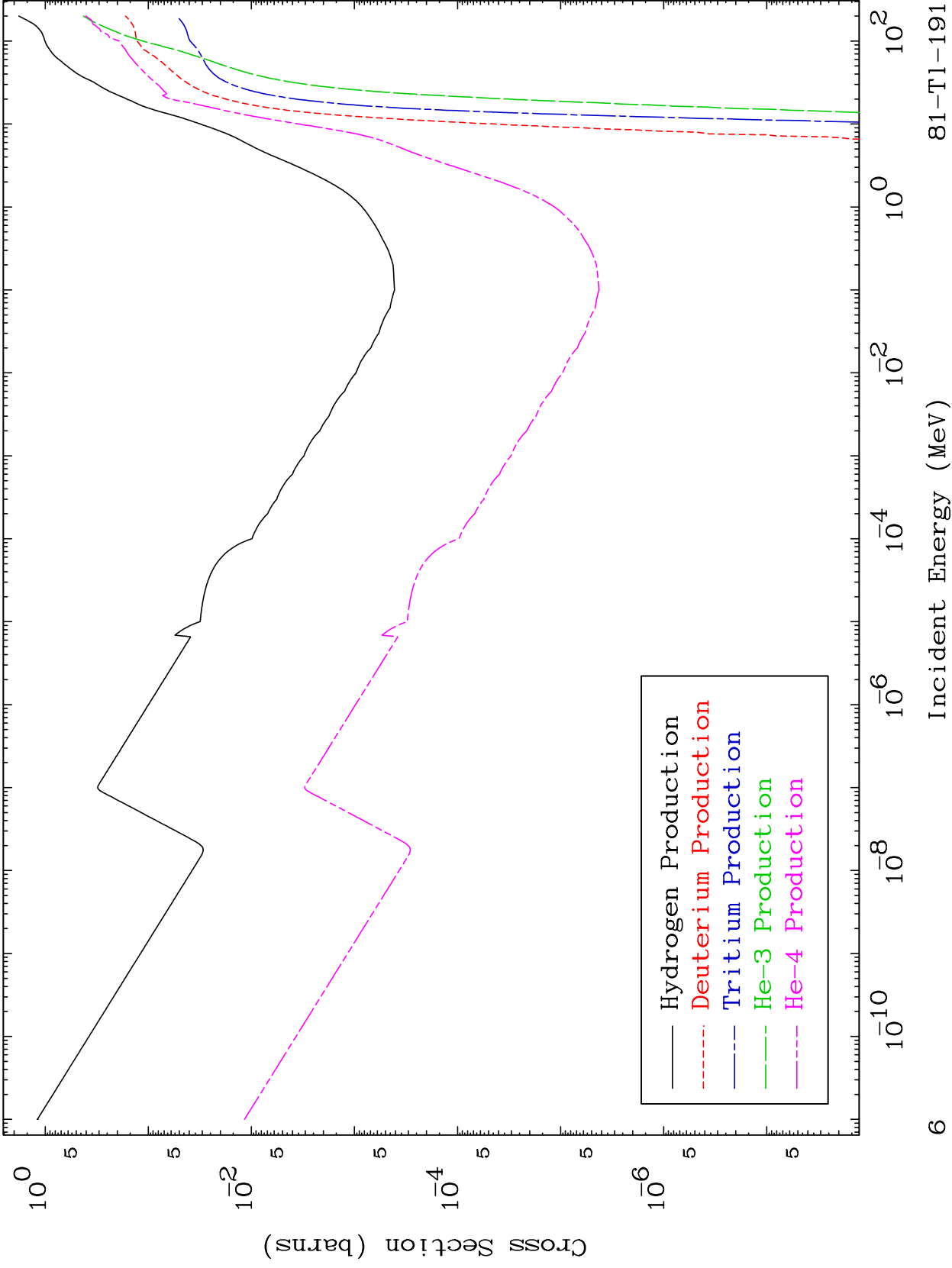


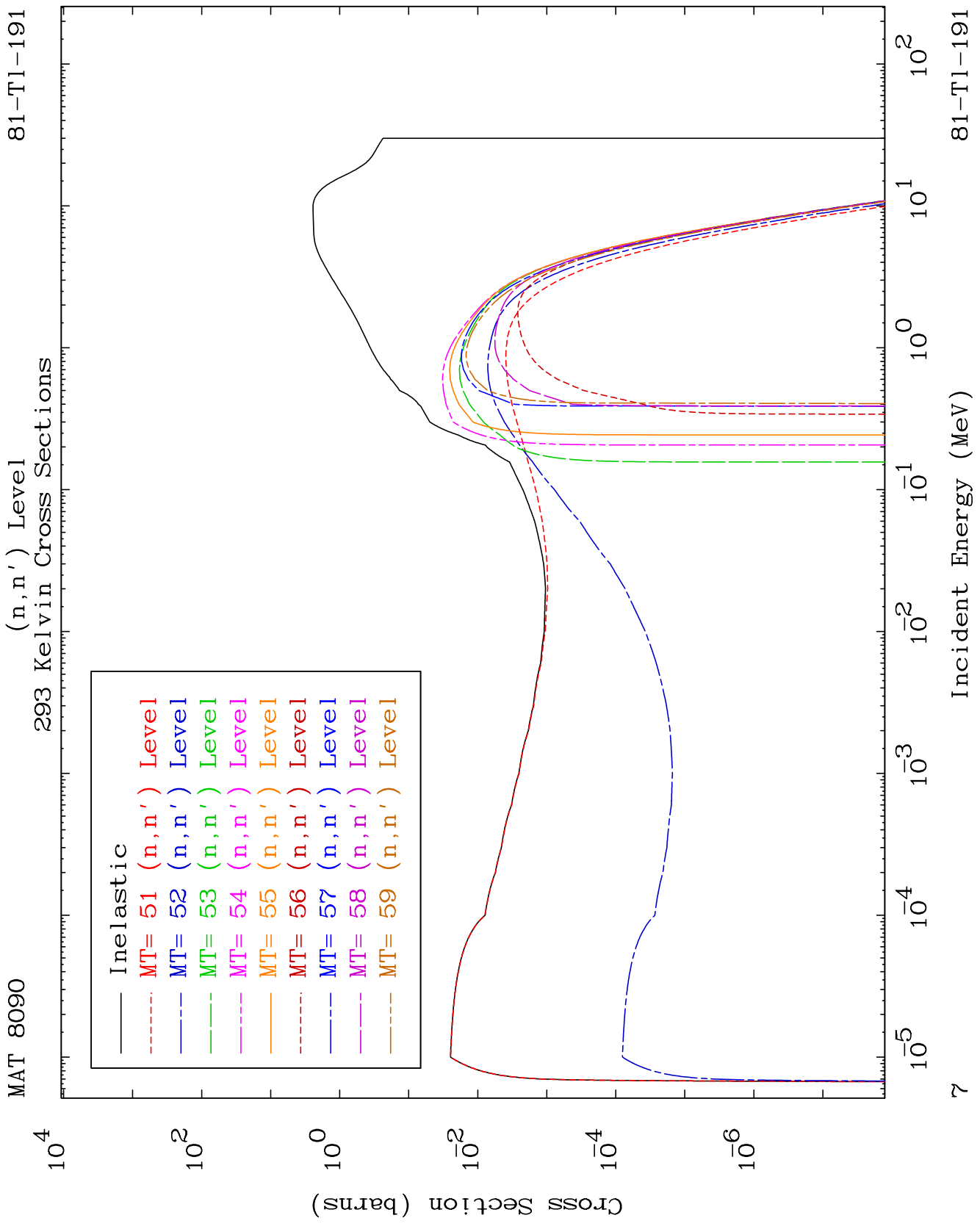
MAT 8090

Charged Particle  
293 Kelvin Cross Sections

81-Tl-191







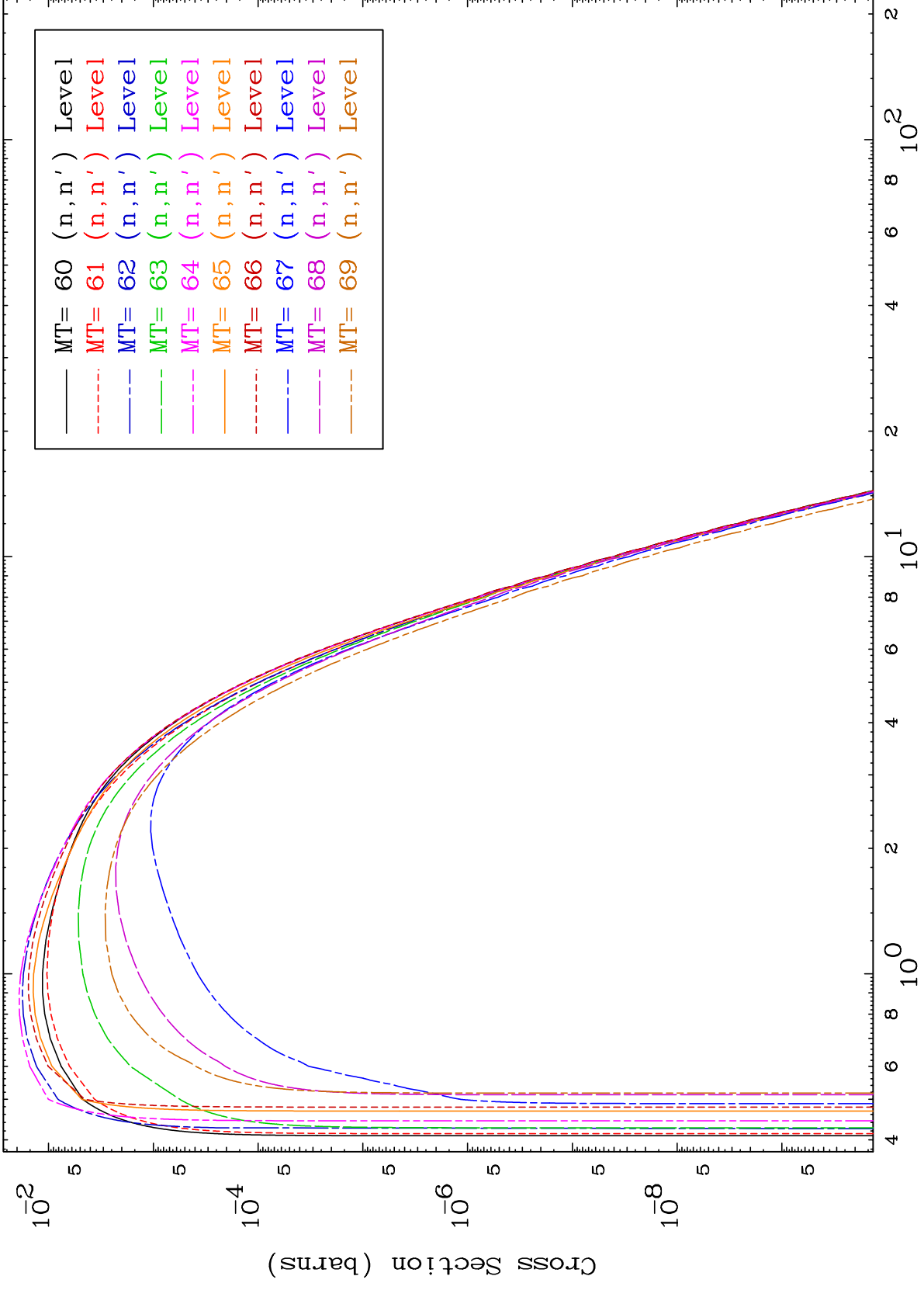


MAT 8090

(n,n') Level

81-Tl-191

293 Kelvin Cross Sections



Incident Energy (MeV)

81-Tl-191

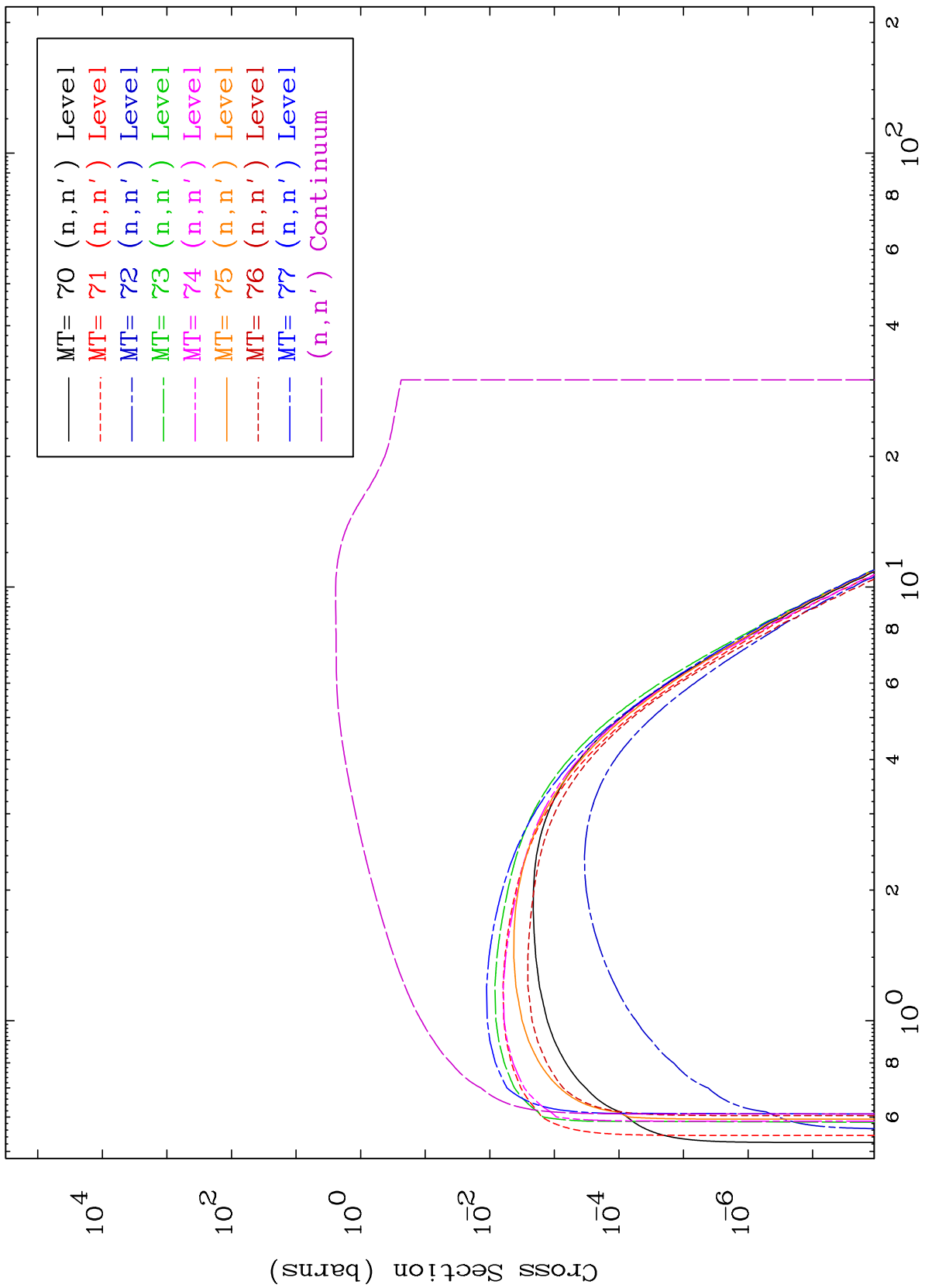
8

MAT 8090

(n,n') Level

81-Tl-191

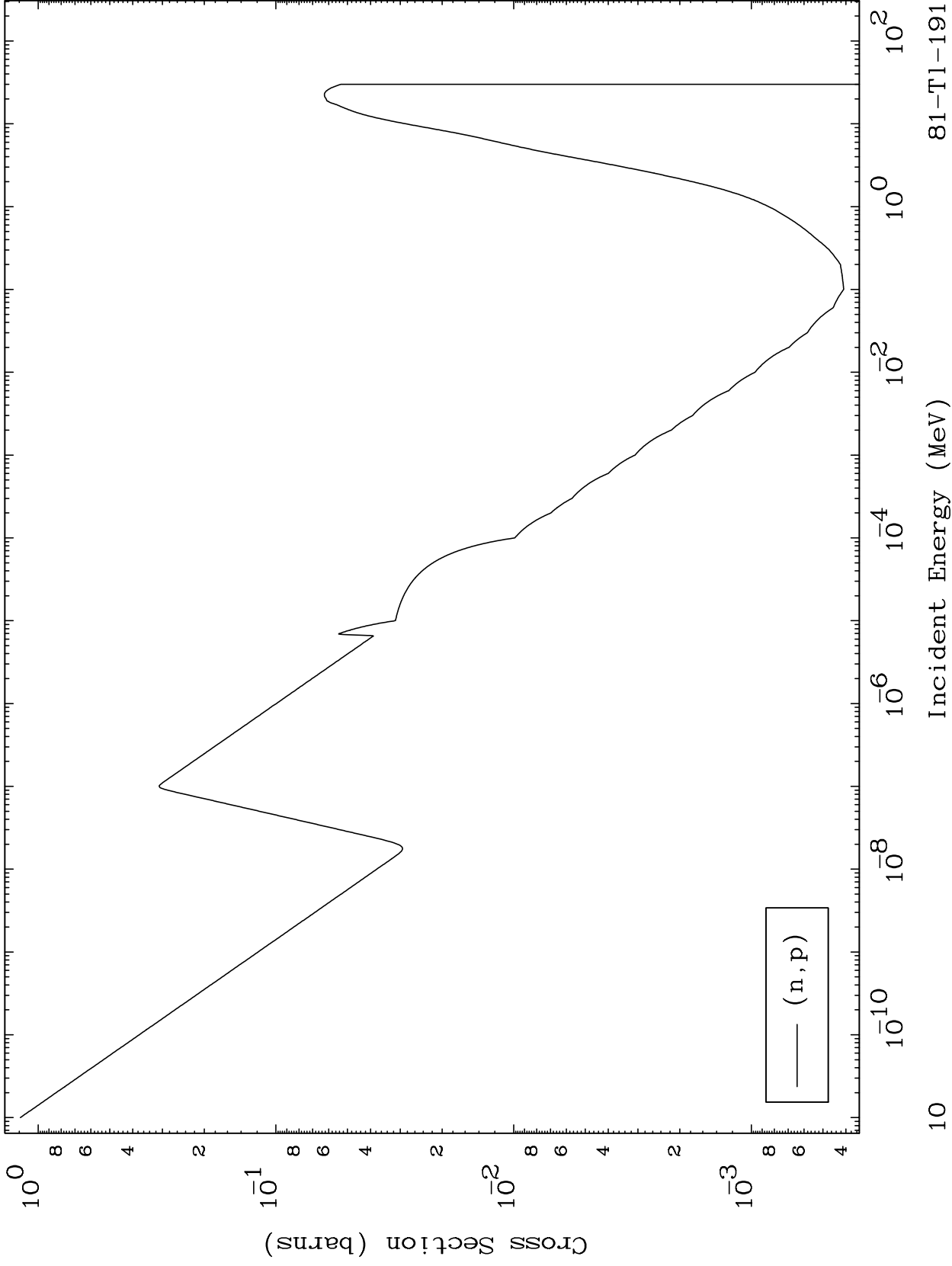
293 Kelvin Cross Sections



MAT 8090

(n,p) Levels  
293 Kelvin Cross Sections

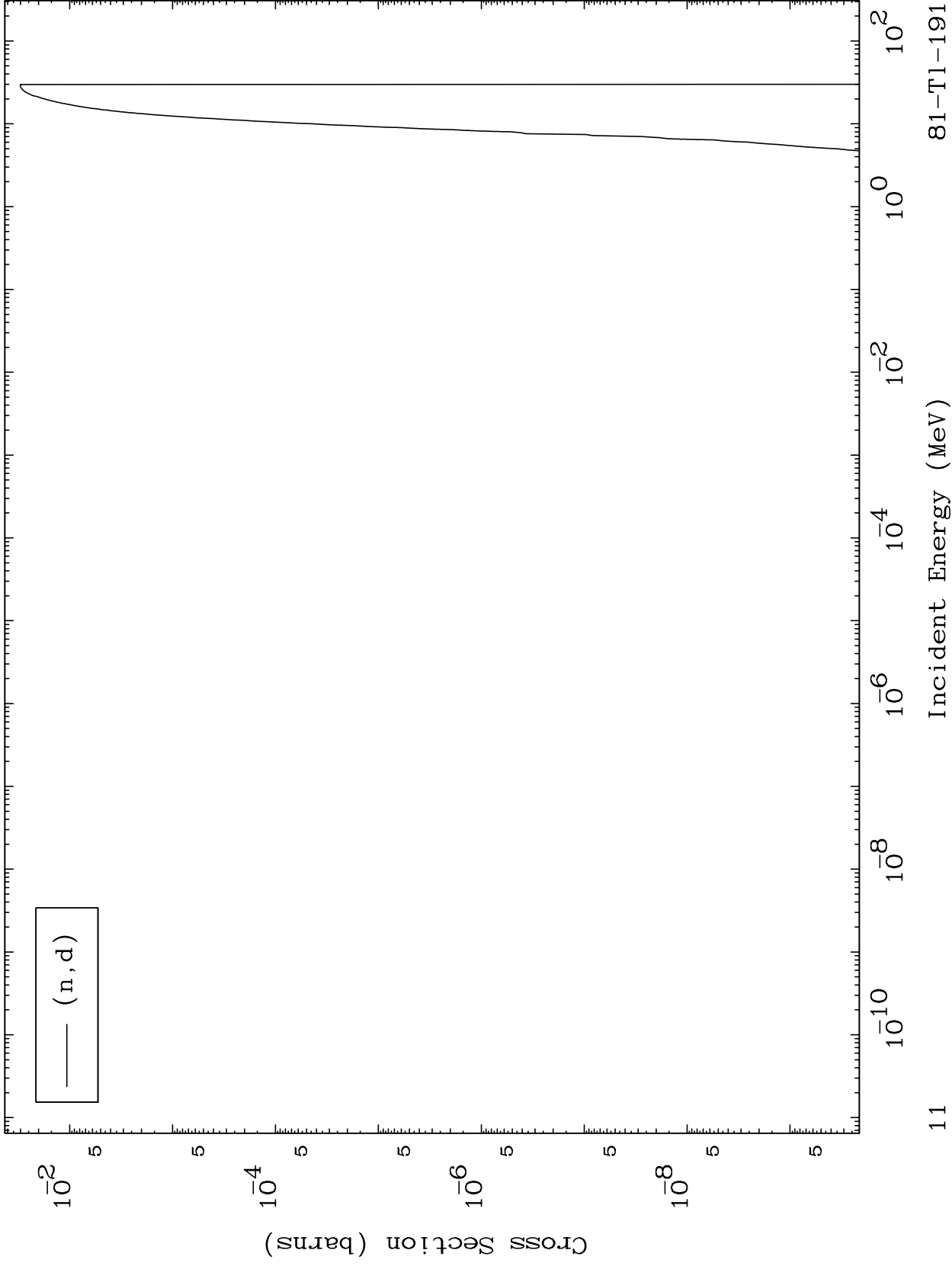
81-Tl-191



MAT 8090

(n,d) Levels  
293 Kelvin Cross Sections

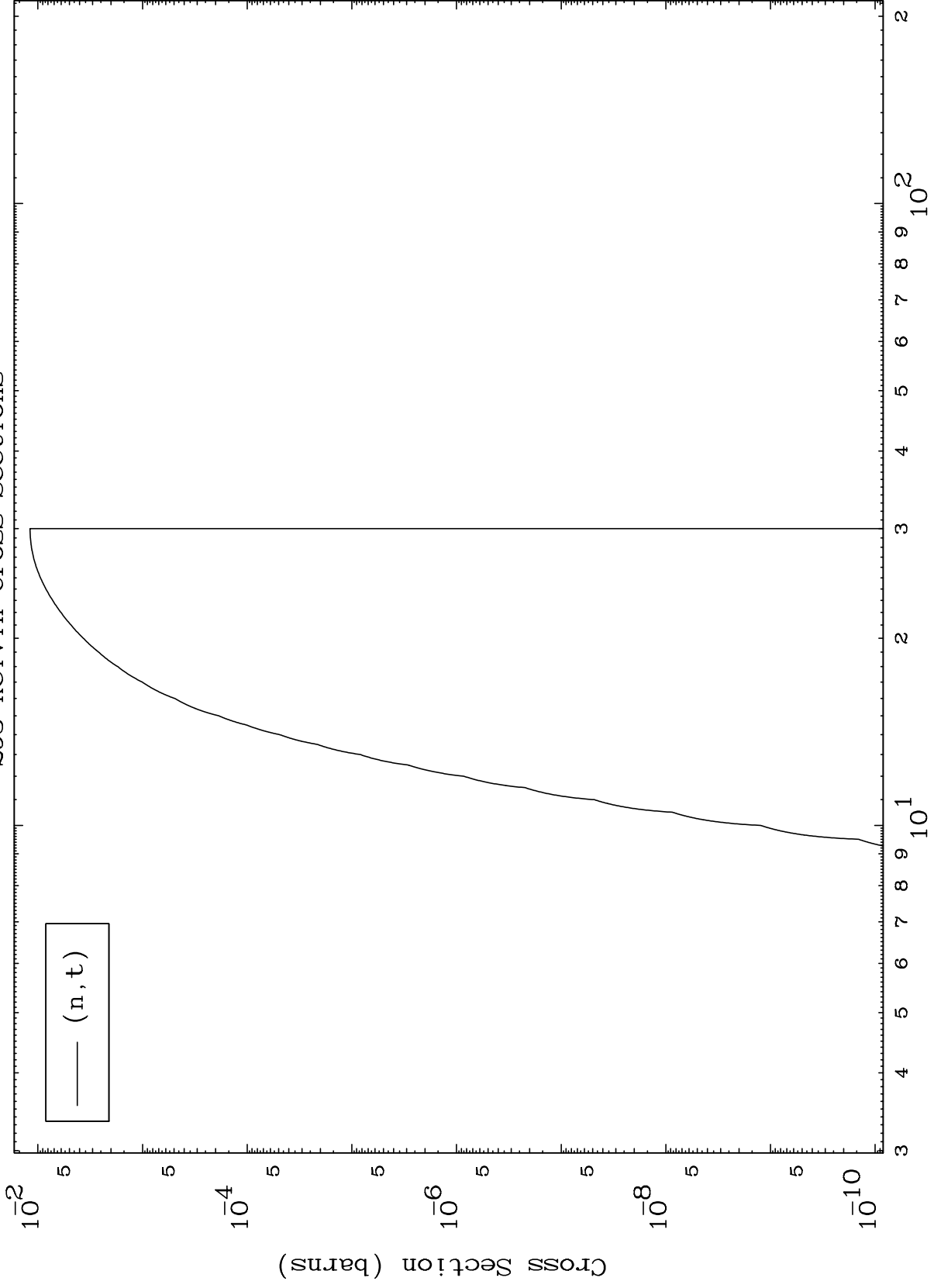
81-Tl-191



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(n,t) Levels  
293 Kelvin Cross Sections

81-Tl-191



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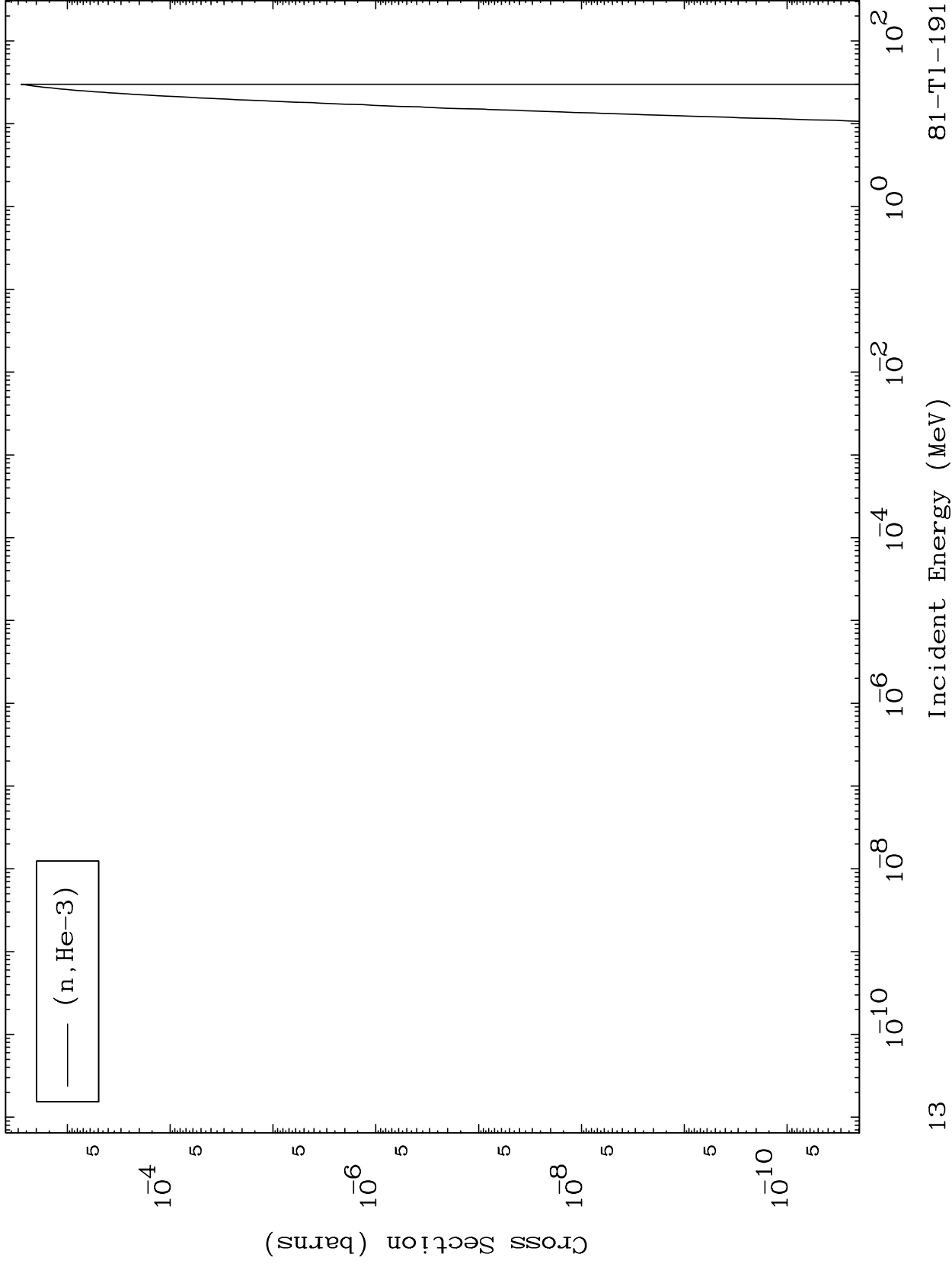
Incident Energy (MeV)

81-Tl-191

MAT 8090

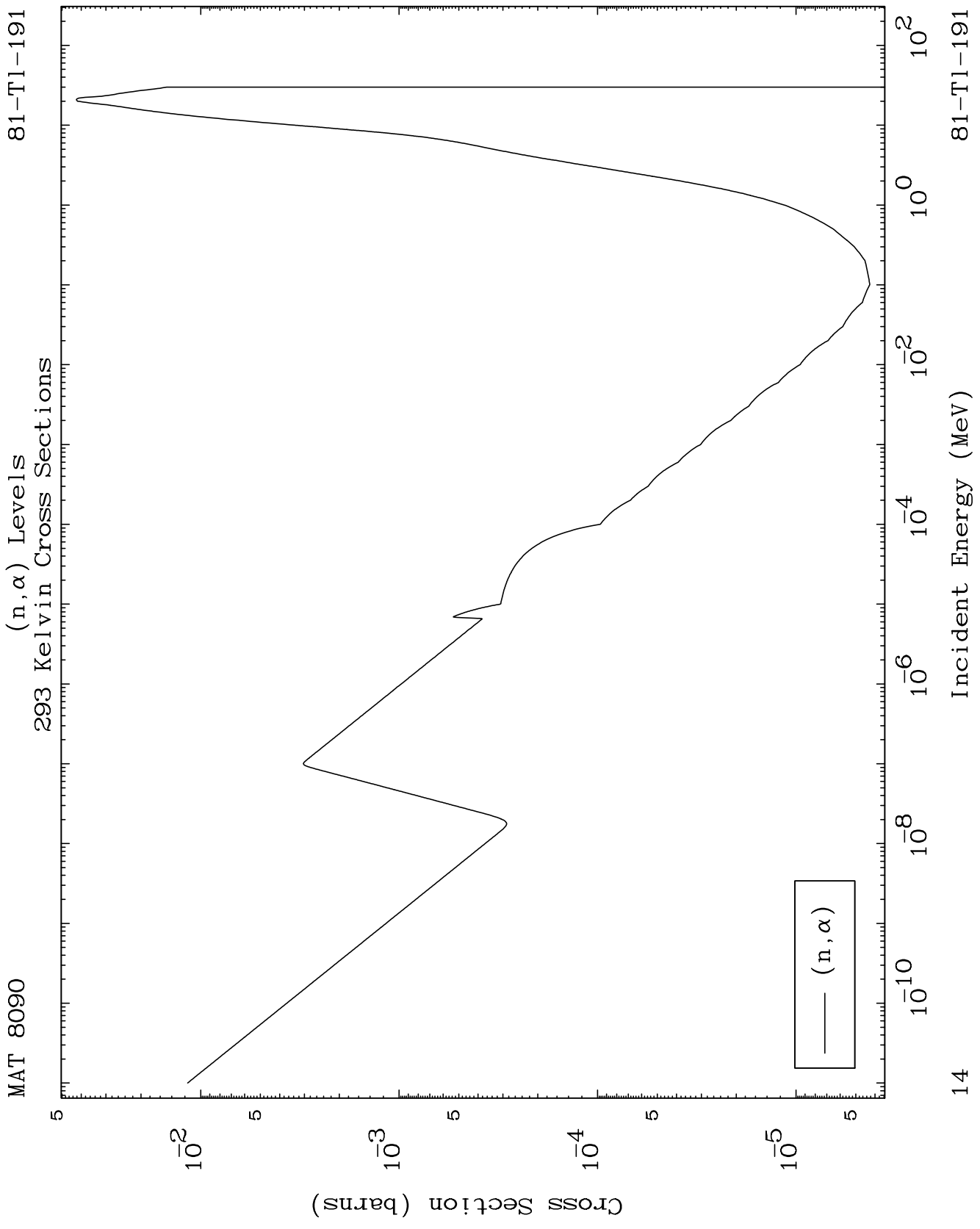
(n,He3) Levels  
293 Kelvin Cross Sections

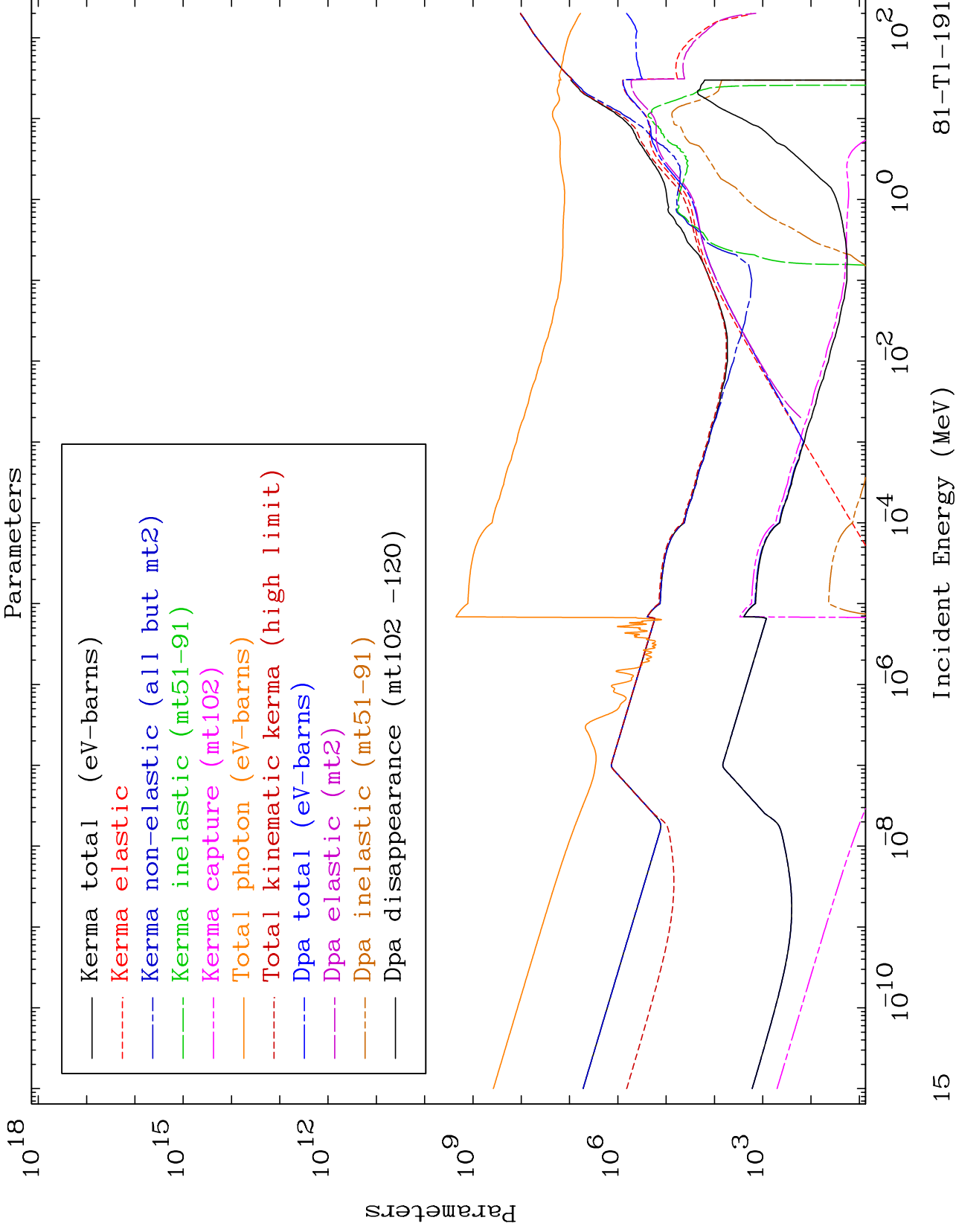
81-Tl-191



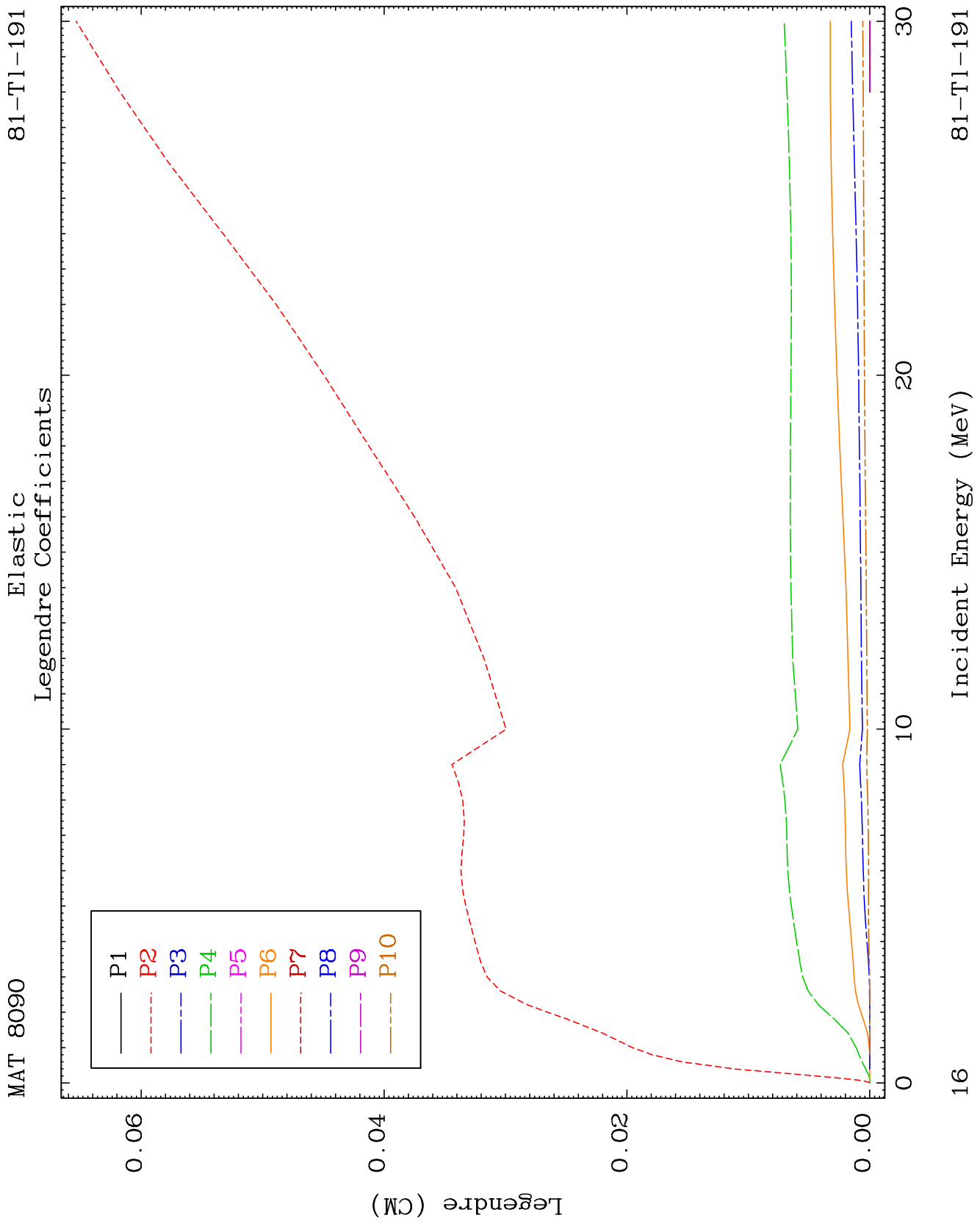
13

81-Tl-191









81-T1-191

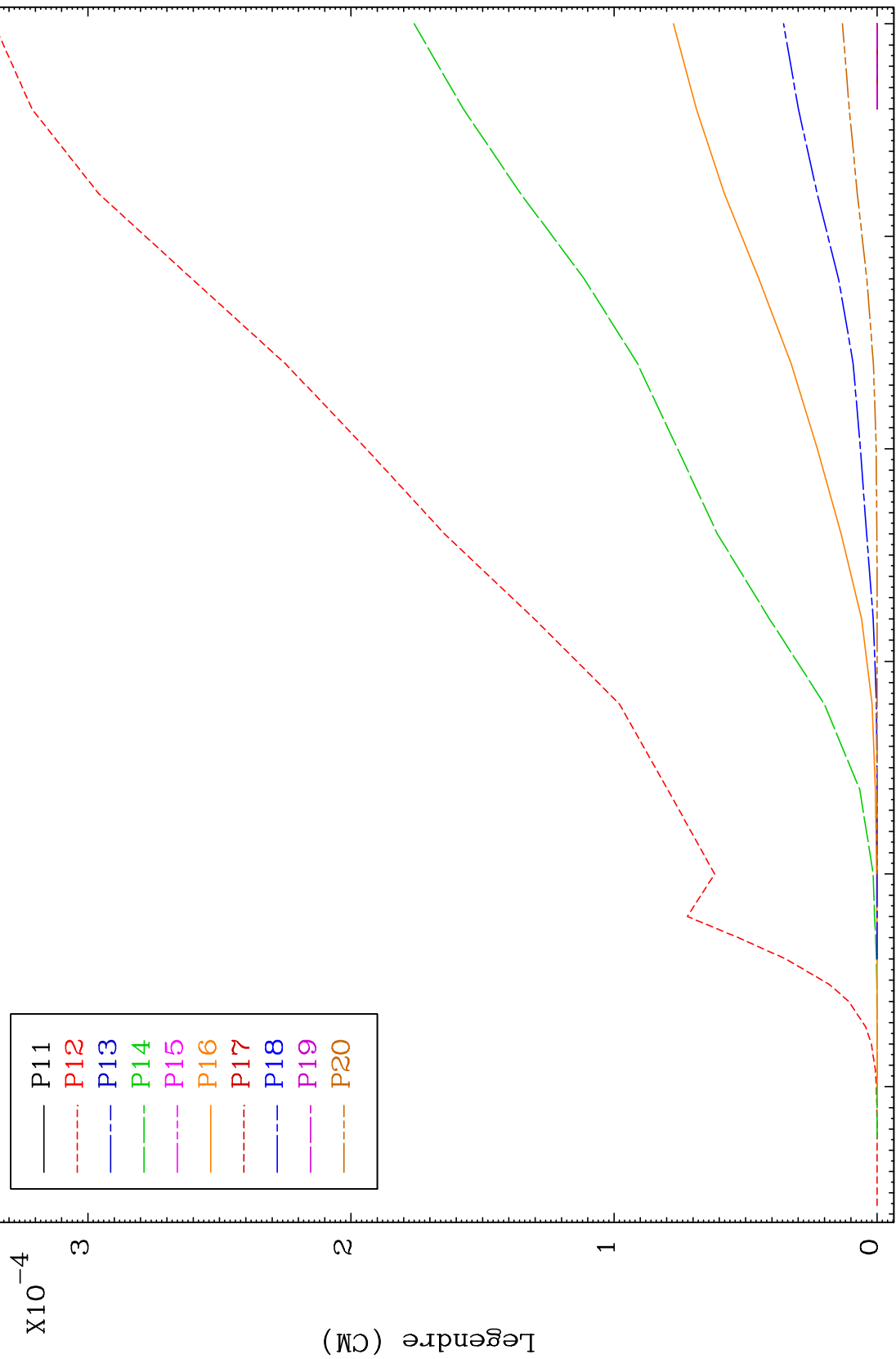
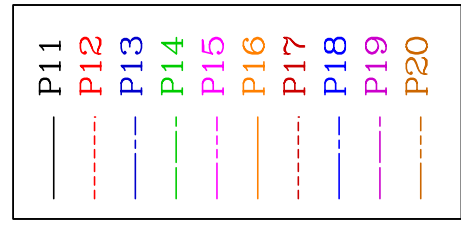
Incident Energy (MeV)

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Elastic Legendre Coefficients

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Incident Energy (MeV)

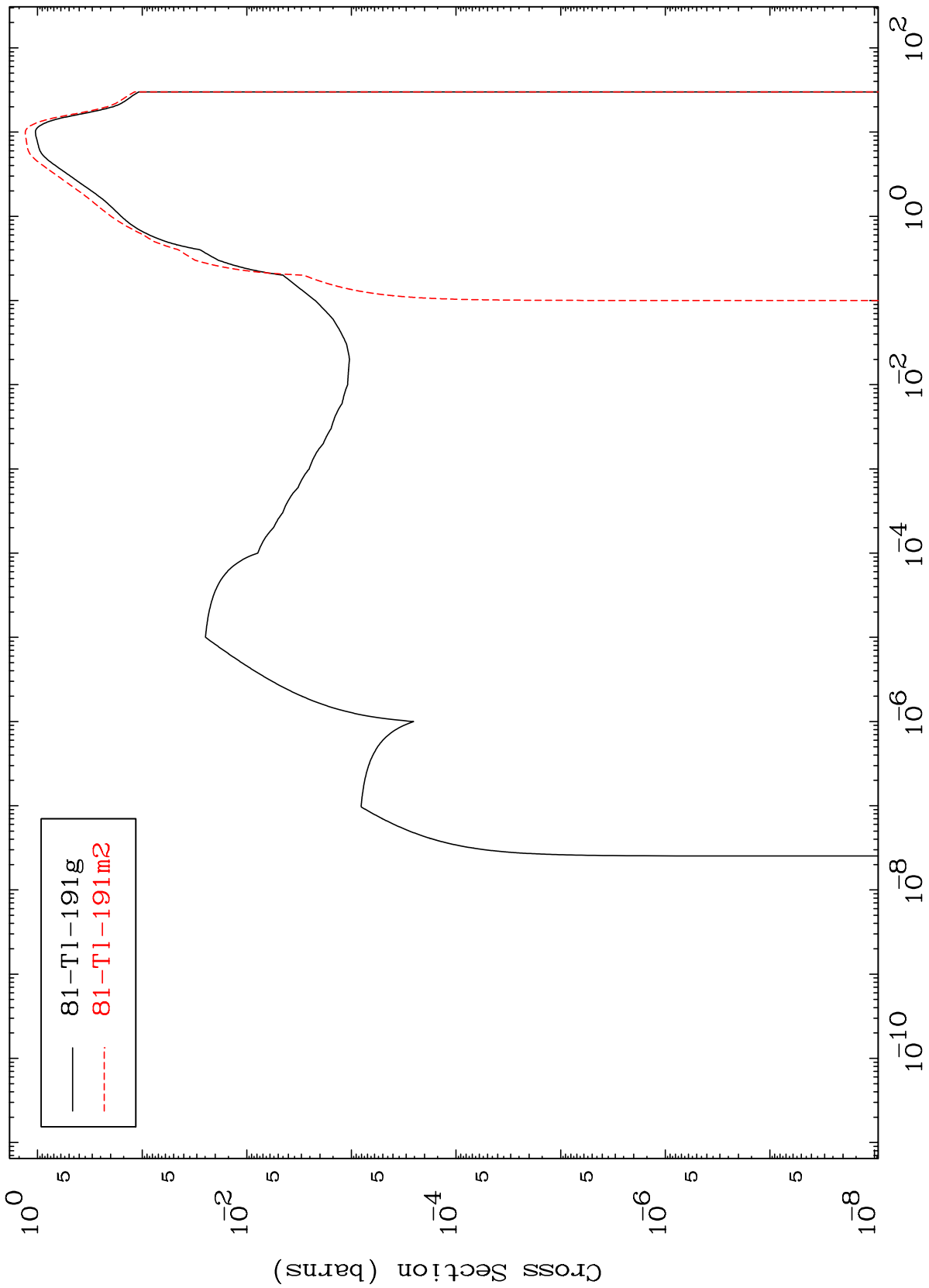
81-Tl-191



MAT 8090

81-Tl-191

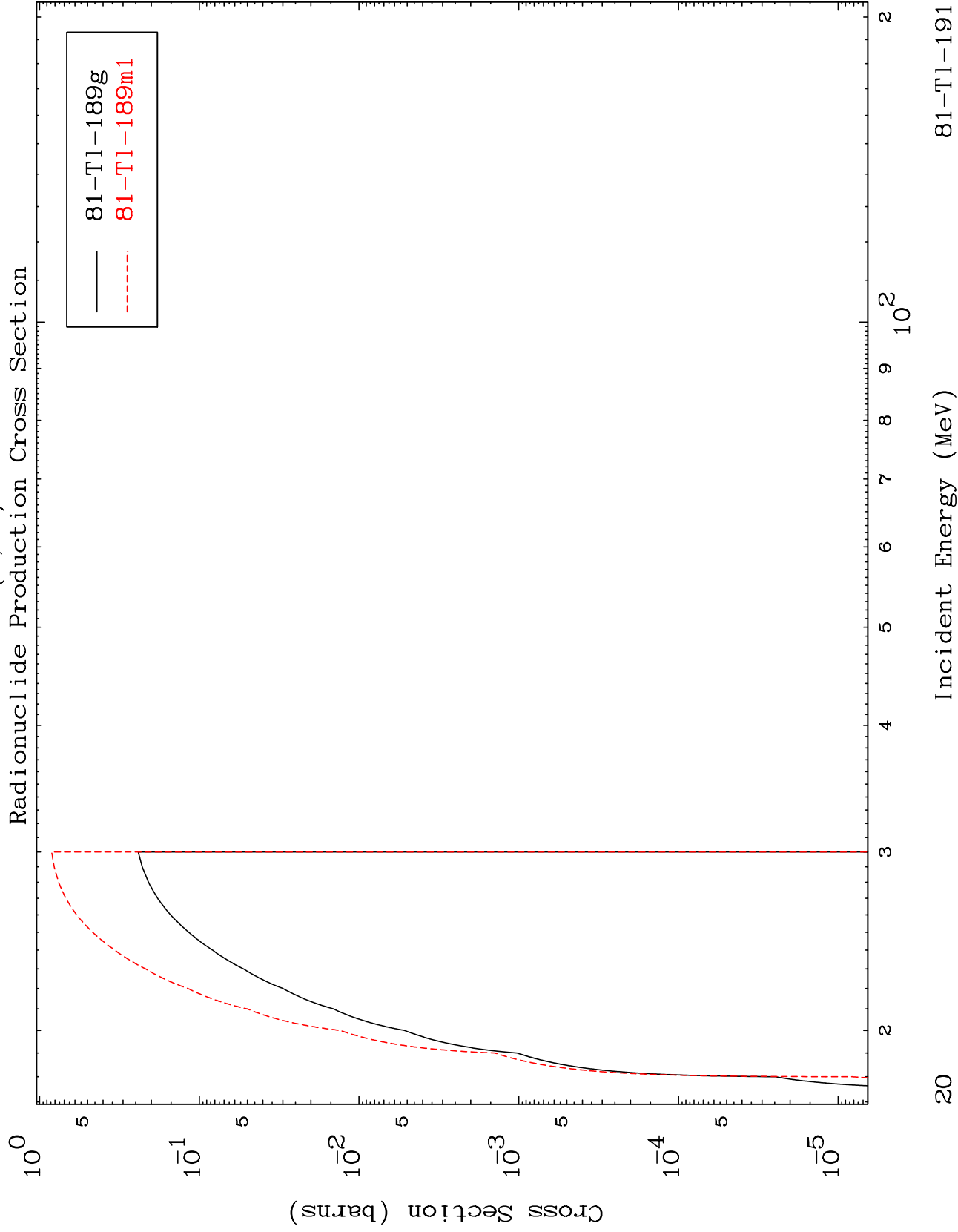
Inelastic  
Radionuclide Production Cross Section



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(n,3n)

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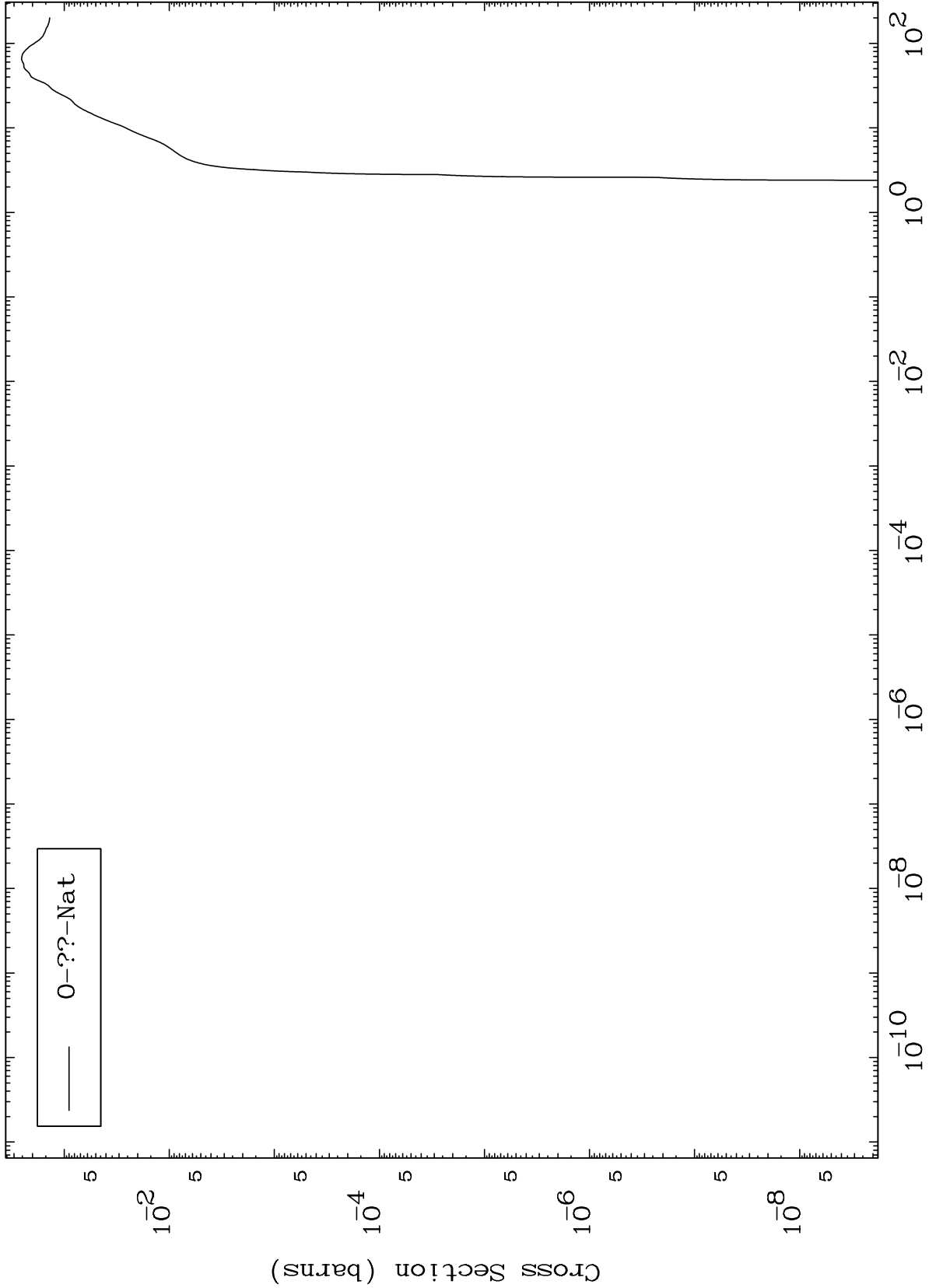
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Fission

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Radionuclide Production Cross Section

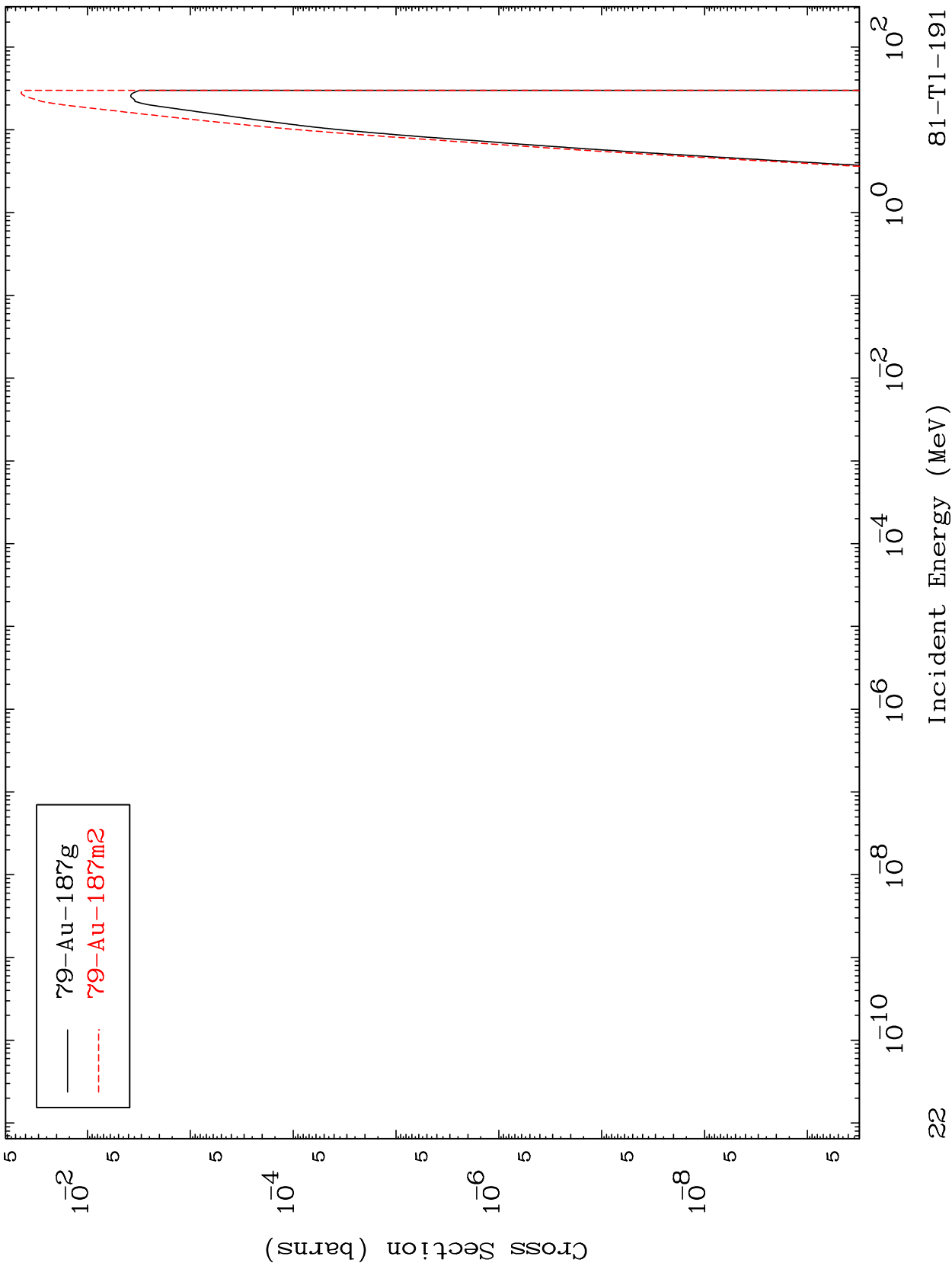


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$(n, n')$   $\alpha$

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Radionuclide Production Cross Section

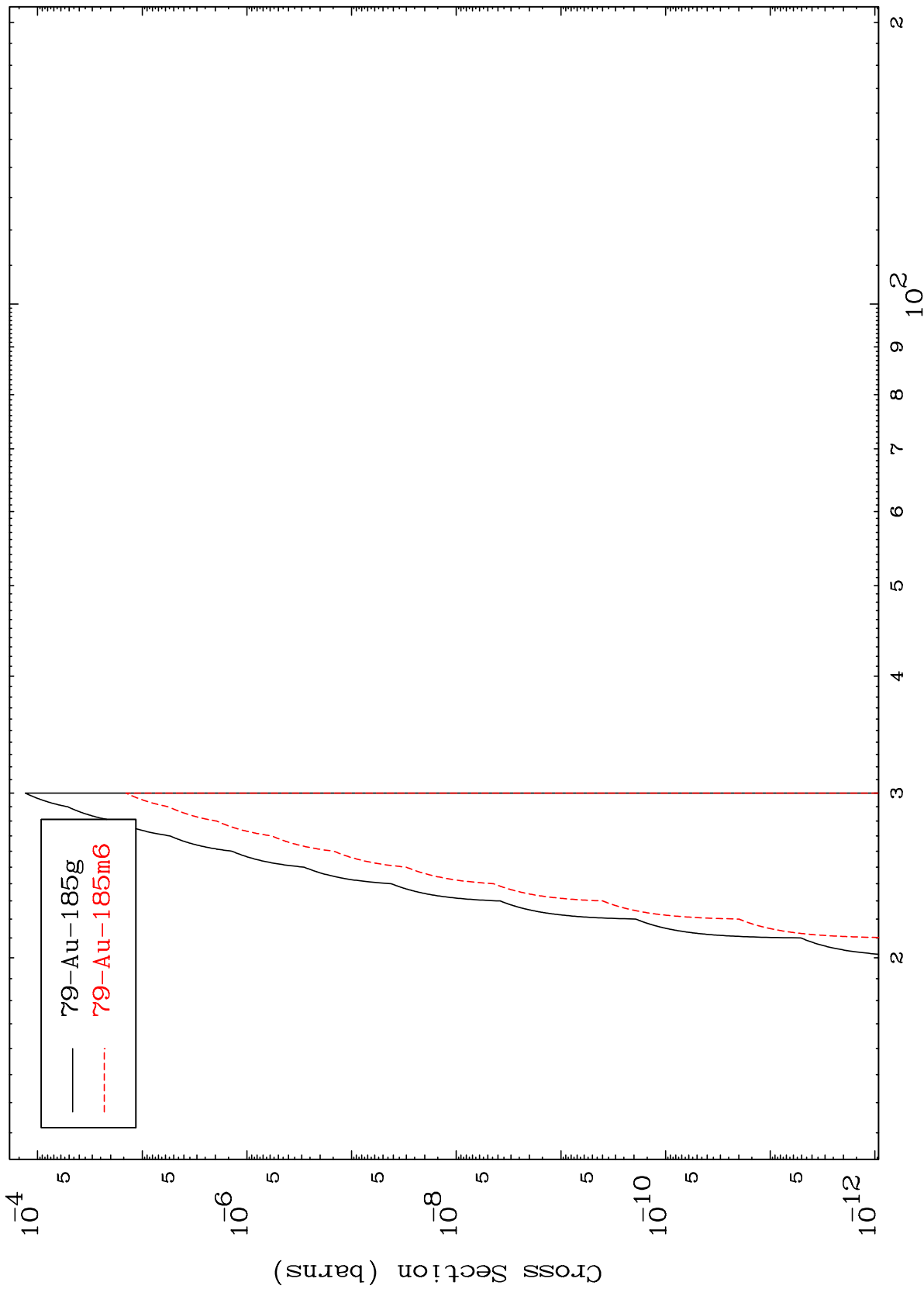


MAT 8090

(n,3n)  $\alpha$

81-Tl-191

Radionuclide Production Cross Section



79-Au-185g  
79-Au-185m6

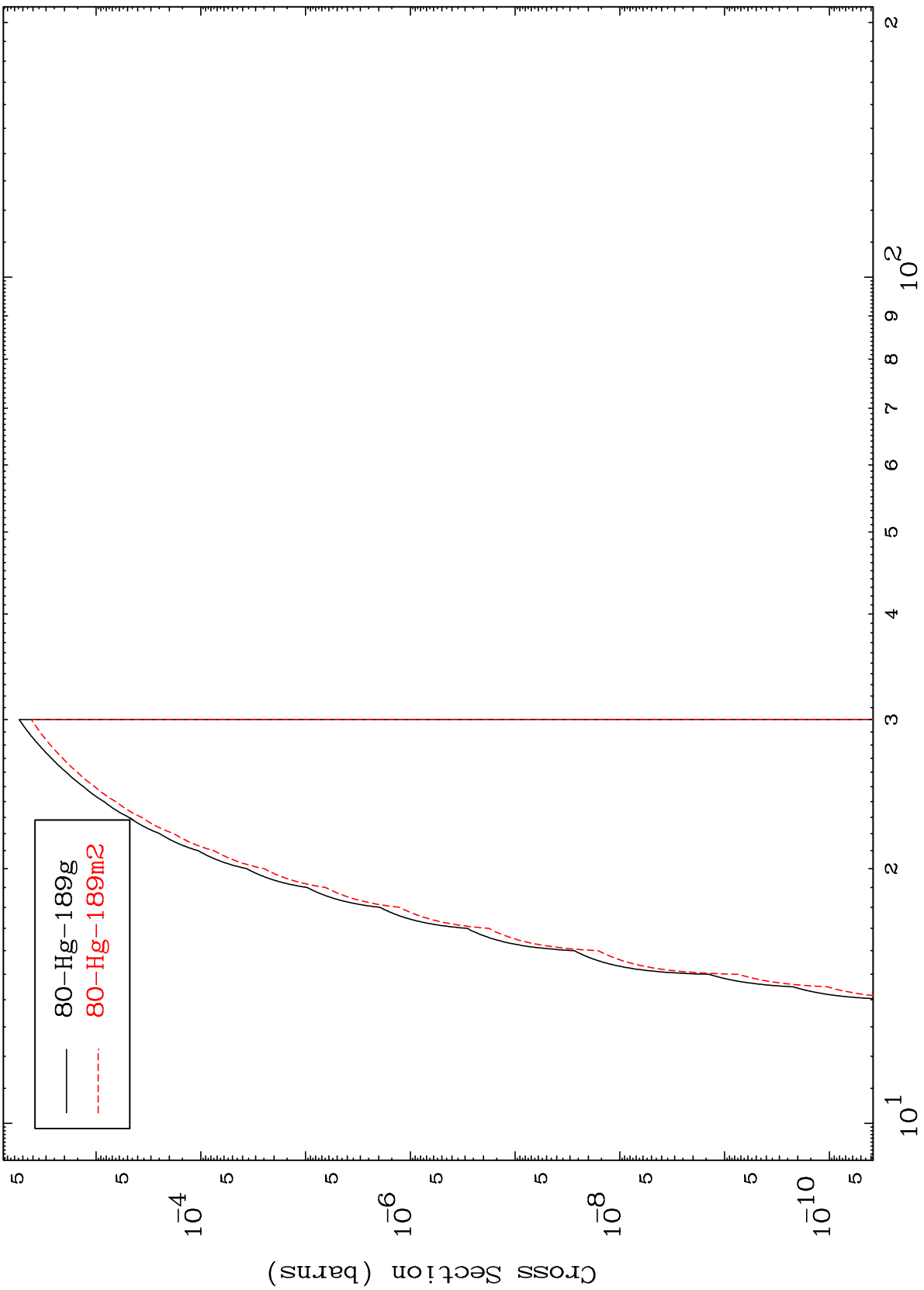


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(n,n') d

81-Tl-191

Radionuclide Production Cross Section



80-Hg-189g  
80-Hg-189m2

24

Incident Energy (MeV)

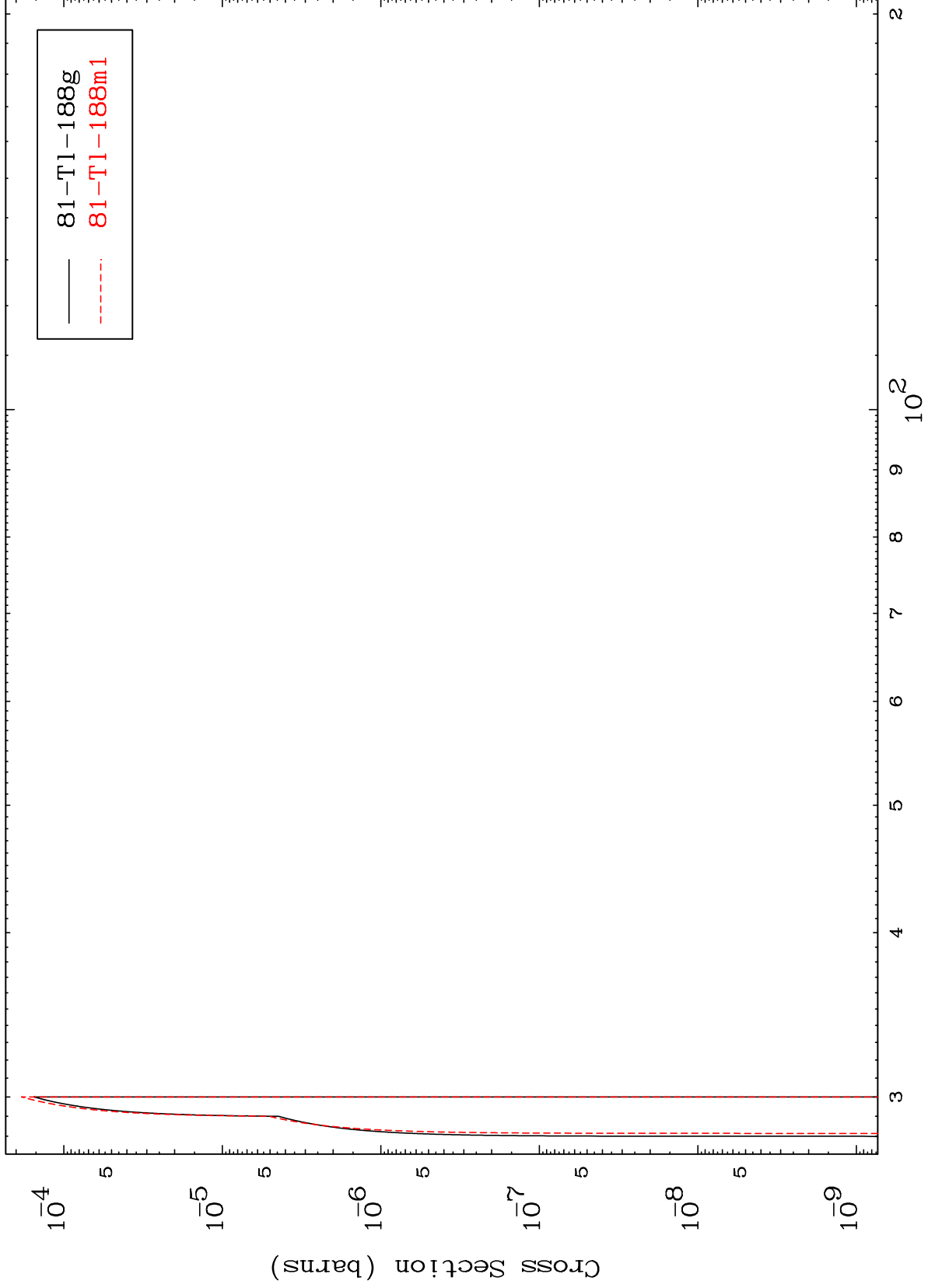
81-Tl-191

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(n,4n)

81-Tl-191

Radionuclide Production Cross Section

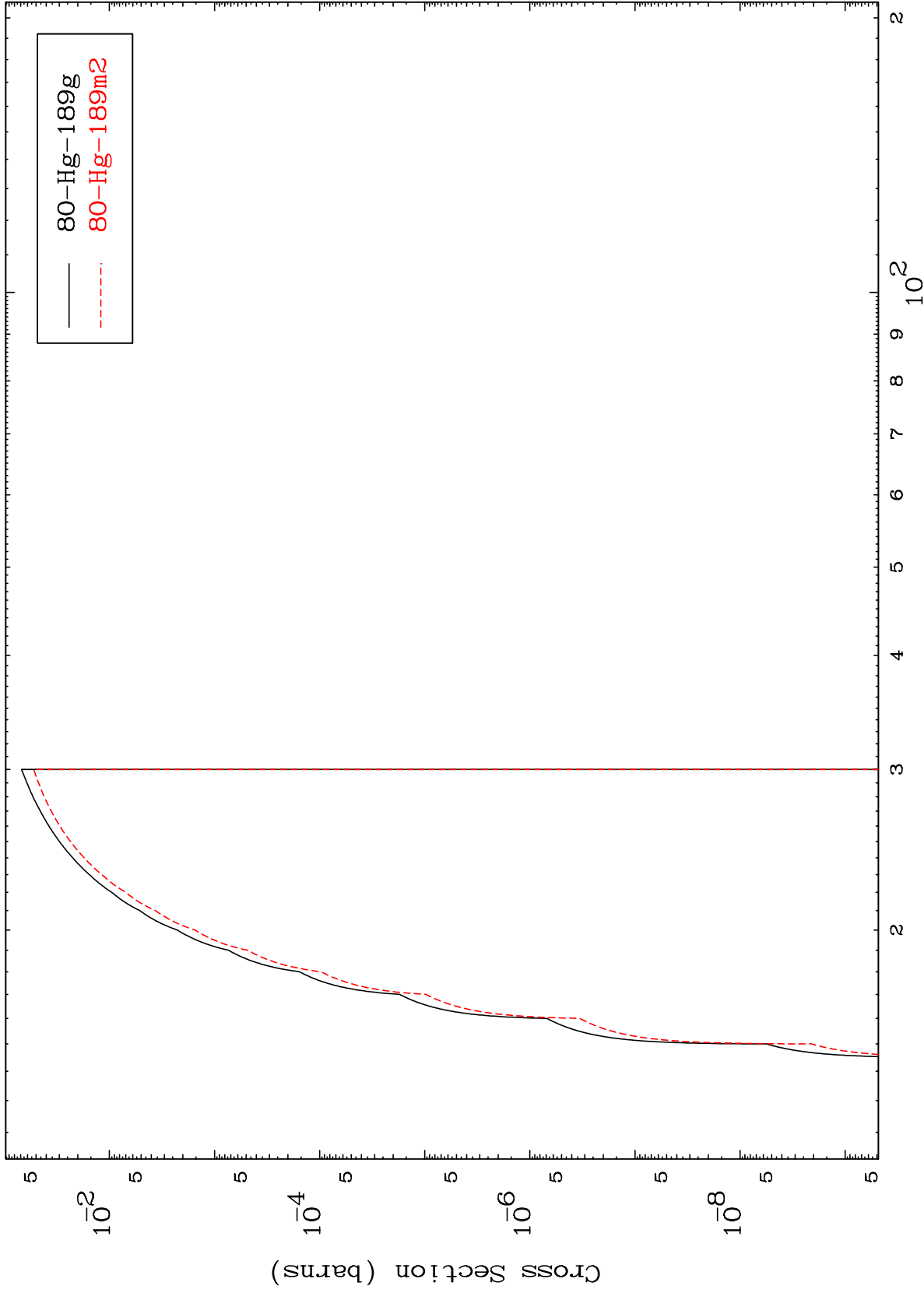


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Incident Energy (MeV)

81-Tl-191

Radionuclide Production Cross Section

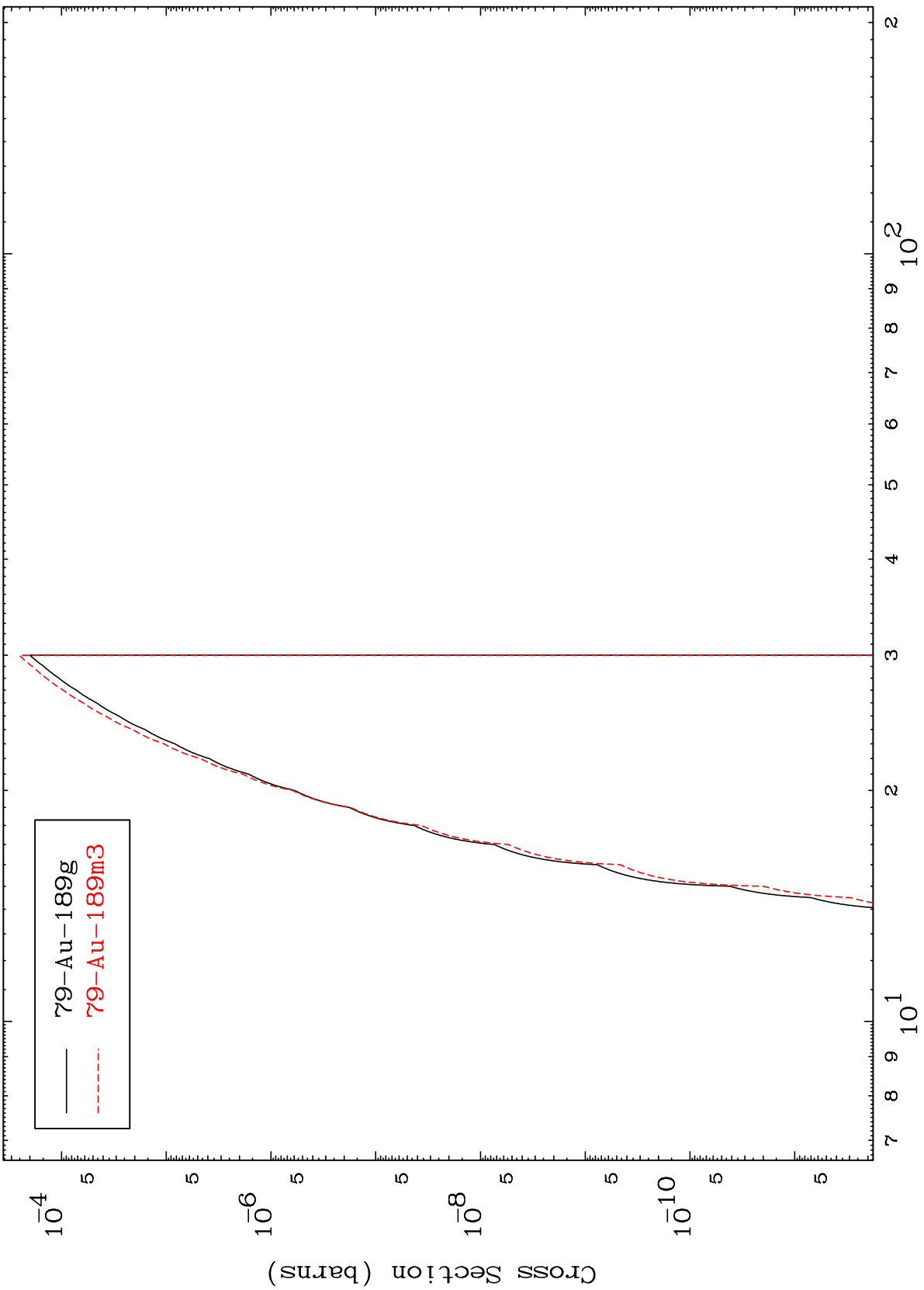


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(n,2n) p

81-Tl-191

Radionuclide Production Cross Section

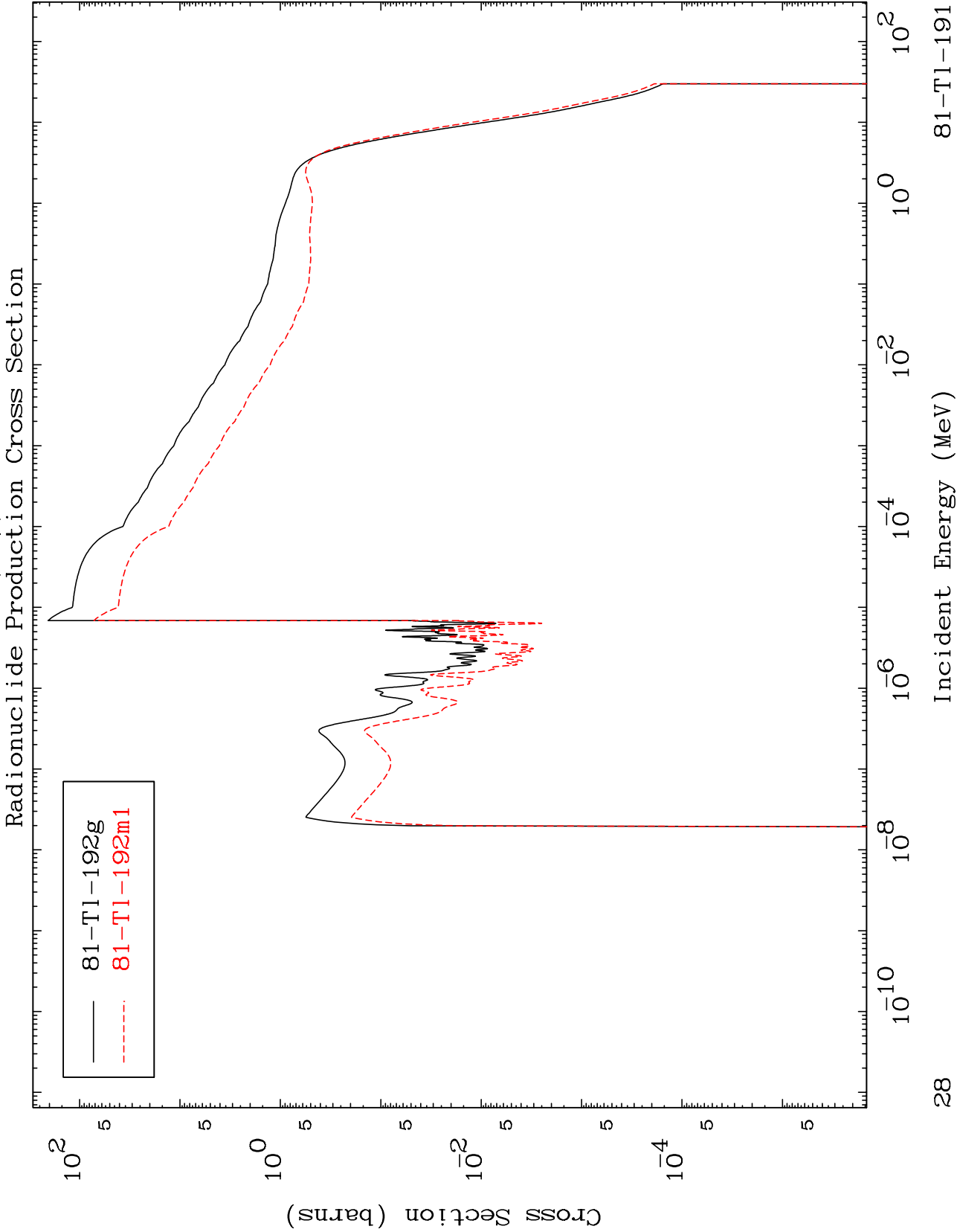


79-Au-189g  
79-Au-189m3

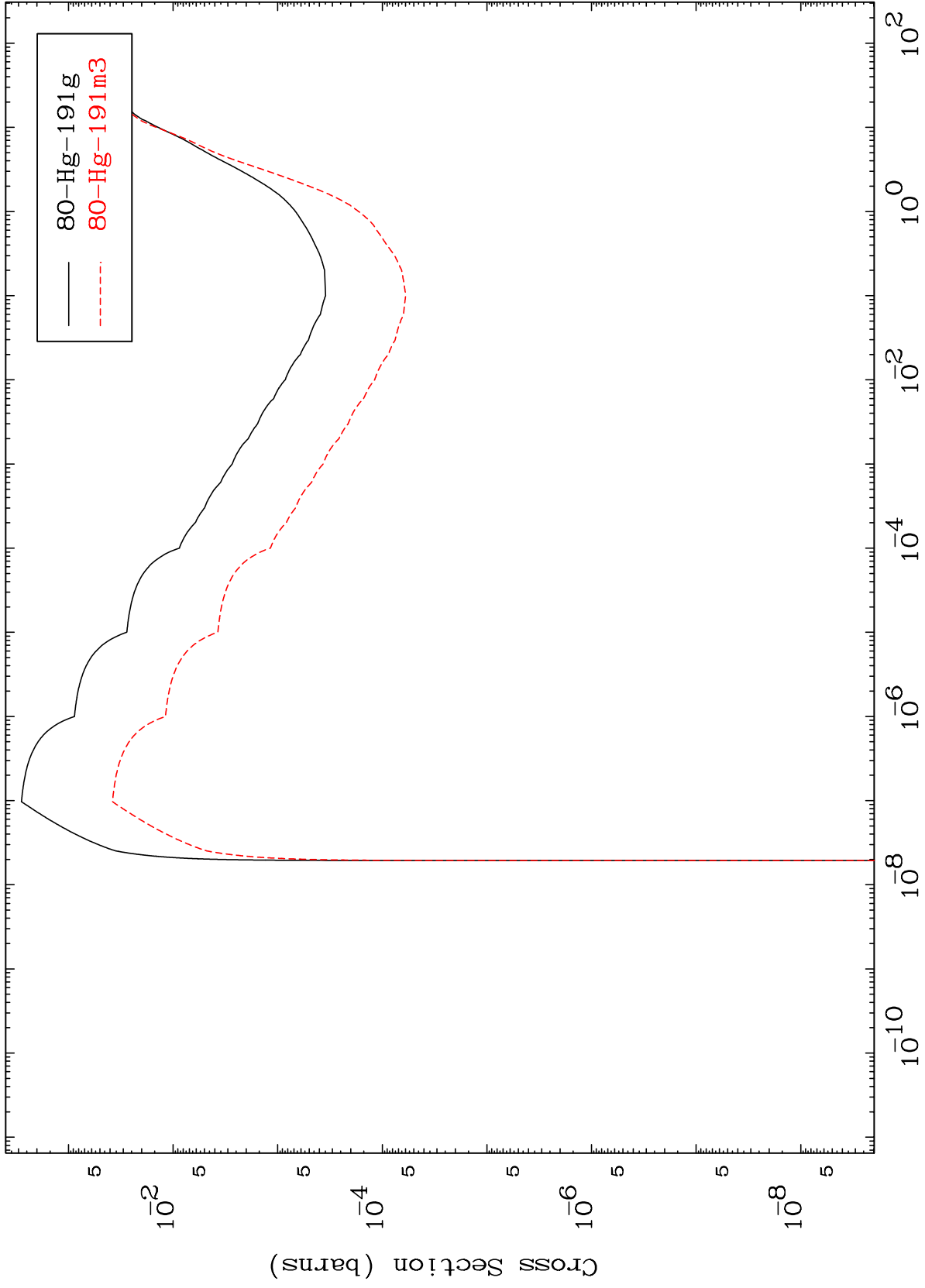
27

Incident Energy (MeV)

81-Tl-191



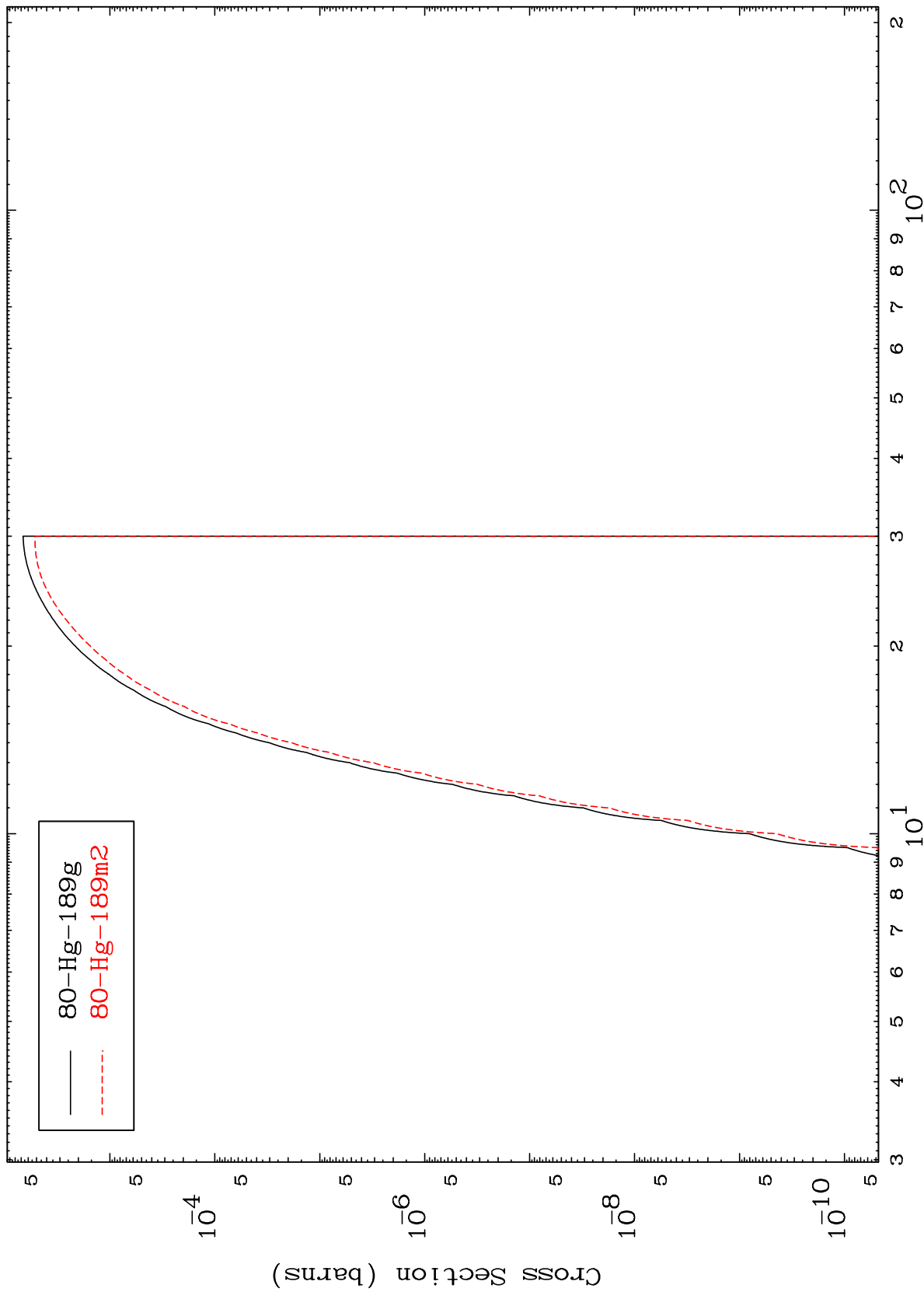
(n,p)  
Radionuclide Production Cross Section



MAT 8090

81-Tl-191

Radionuclide Production Cross Section



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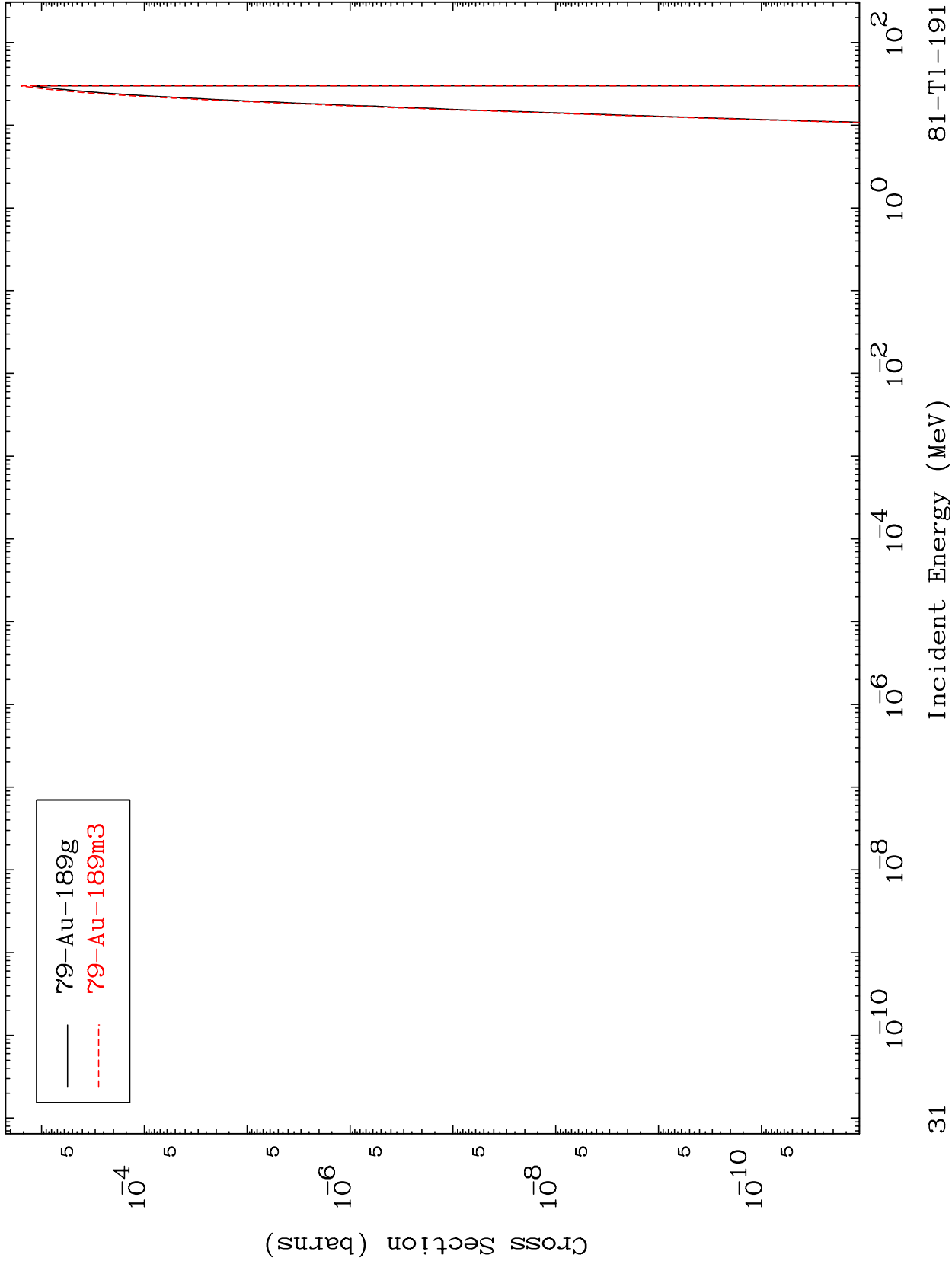
81-Tl-191

MAT 8090

(n,He-3)

81-Tl-191

Radionuclide Production Cross Section



79-Au-189g  
79-Au-189m3

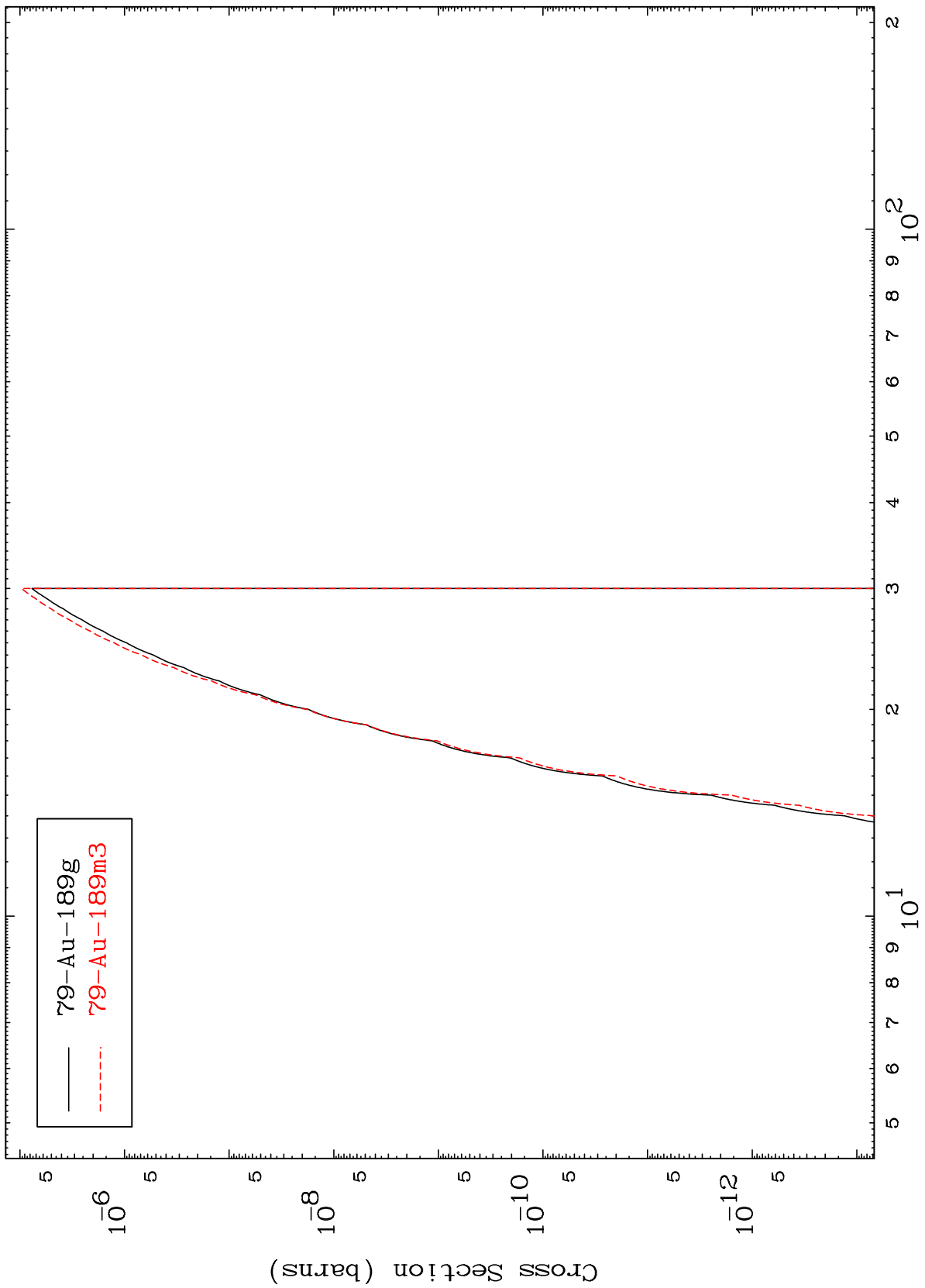


MAT 8090

(n,p) d

81-Tl-191

Radionuclide Production Cross Section



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Incident Energy (MeV)

81-Tl-191