

Program EVALPLOT
(Version 2018-1)

by

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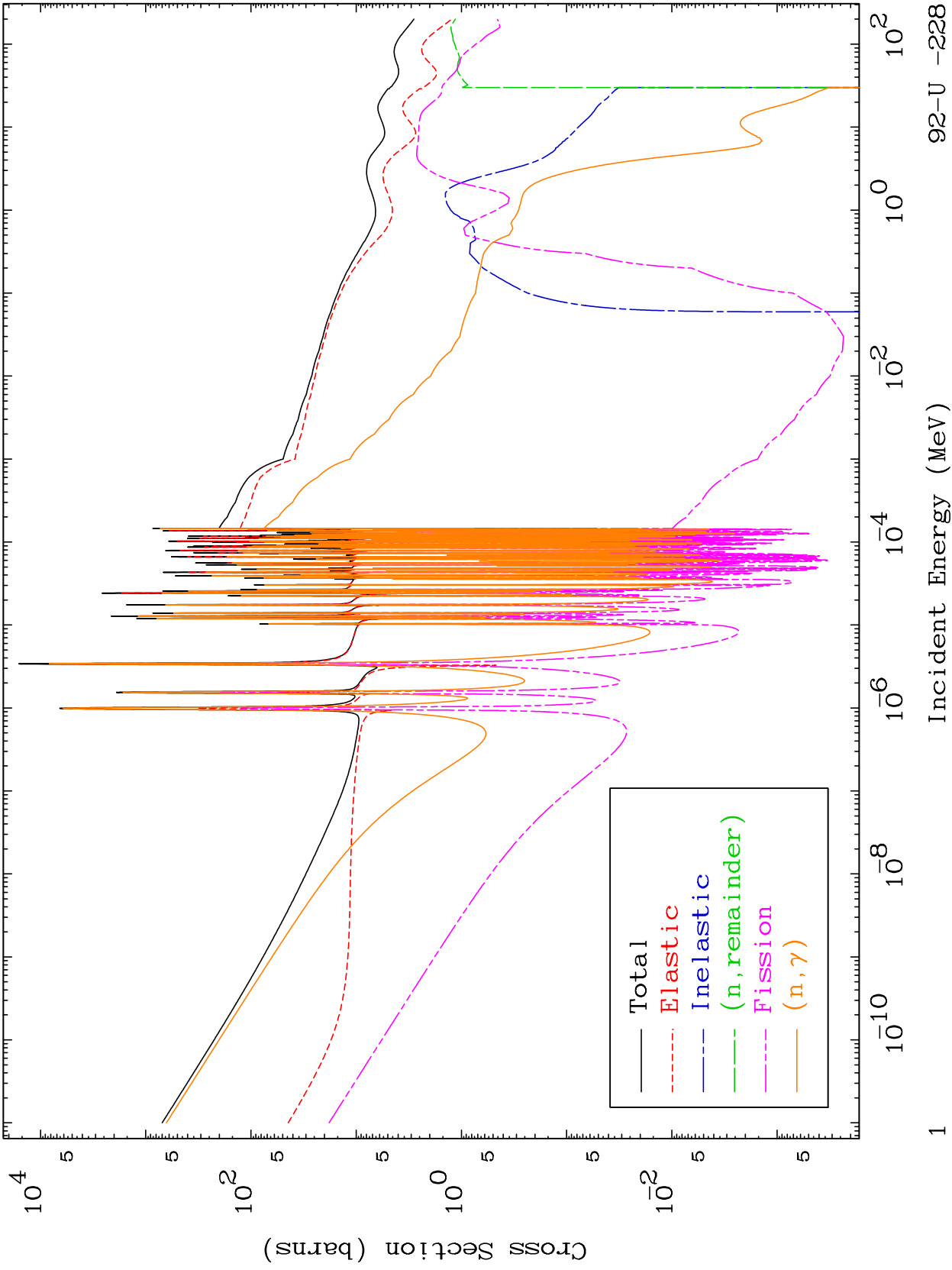
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9207

Major
293 Kelvin Cross Sections

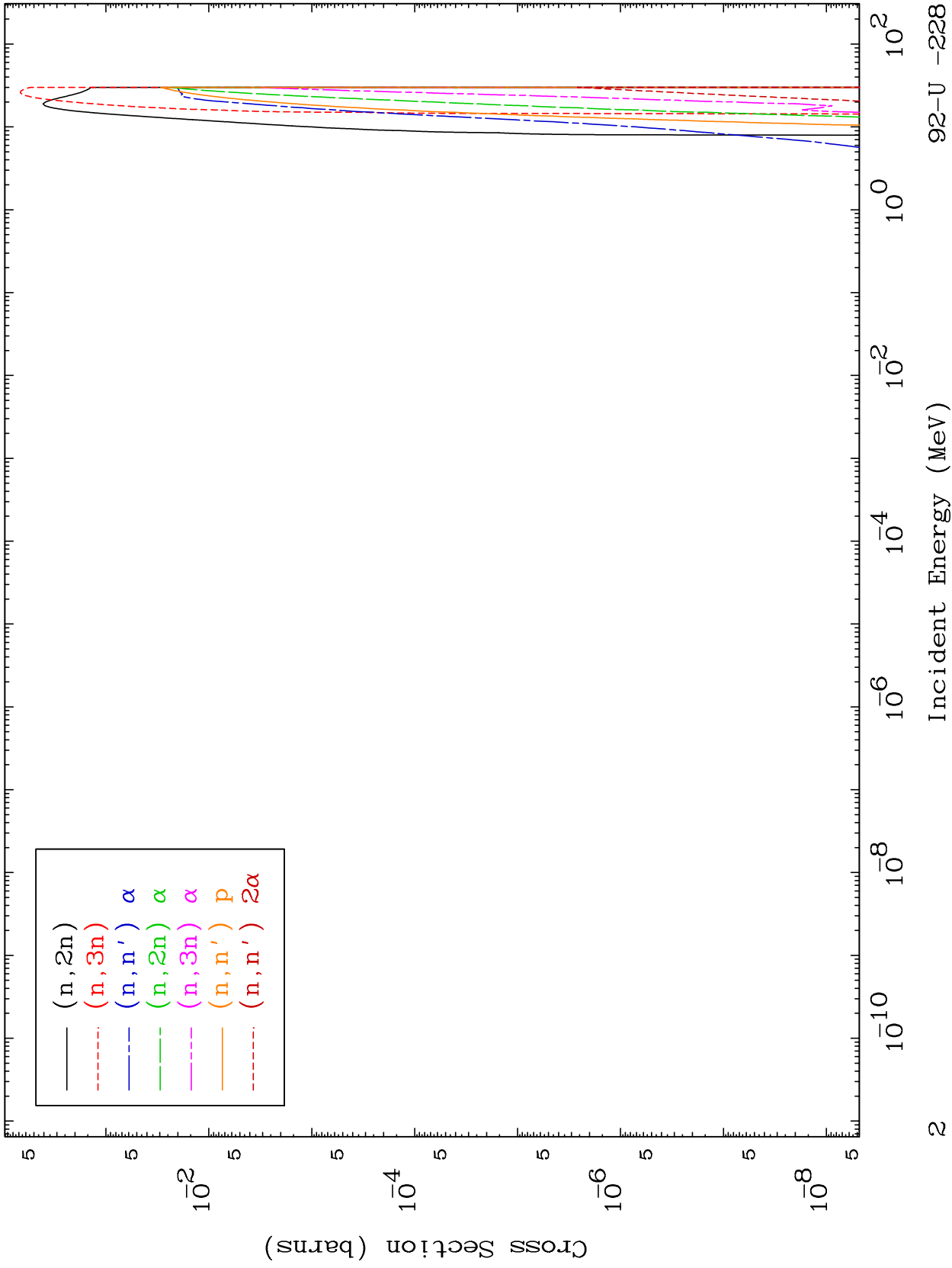
92-U -228



MAT 9207

Neutron Production
293 Kelvin Cross Sections

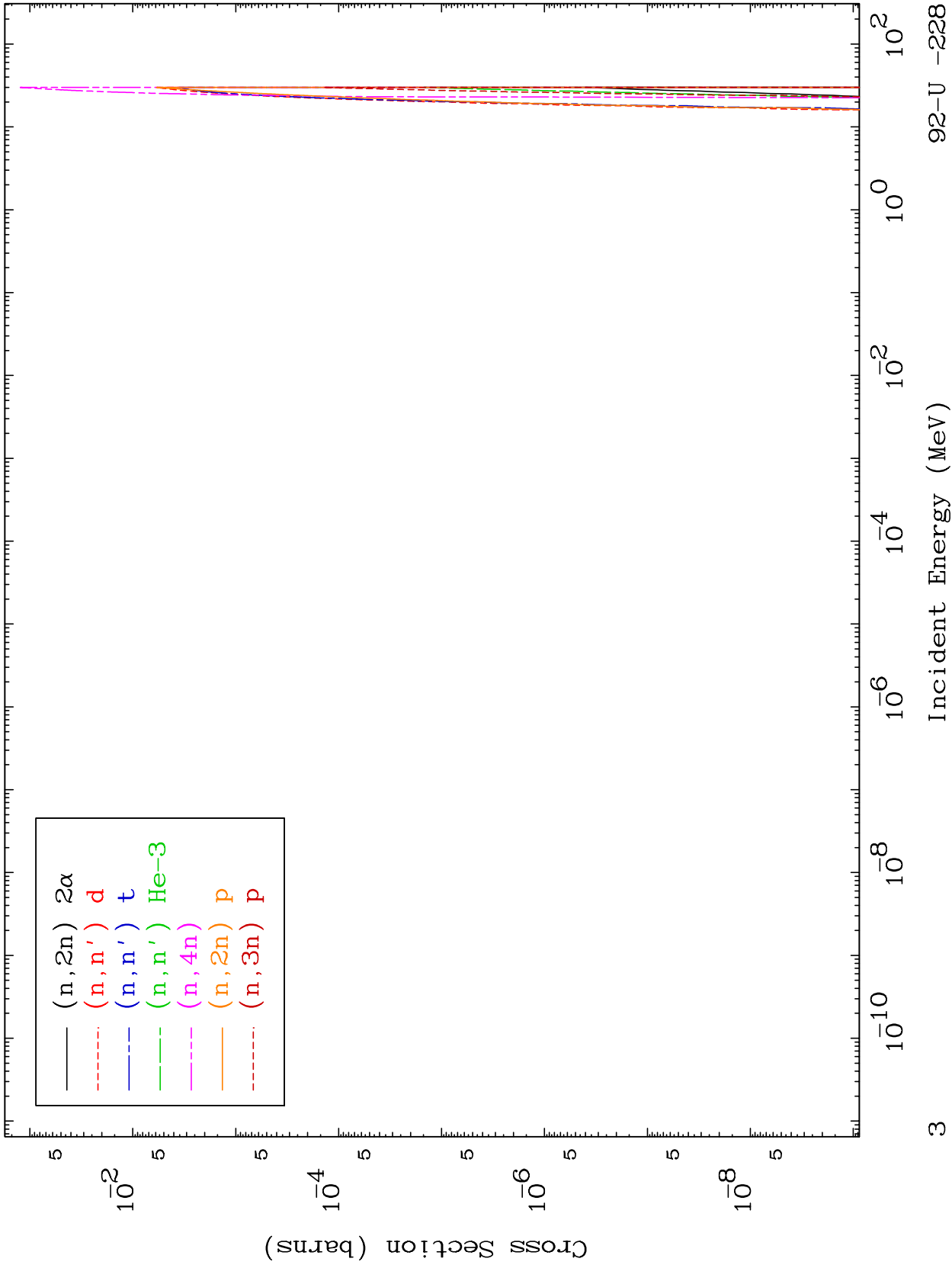
92-U -228



MAT 9207

Neutron Production
293 Kelvin Cross Sections

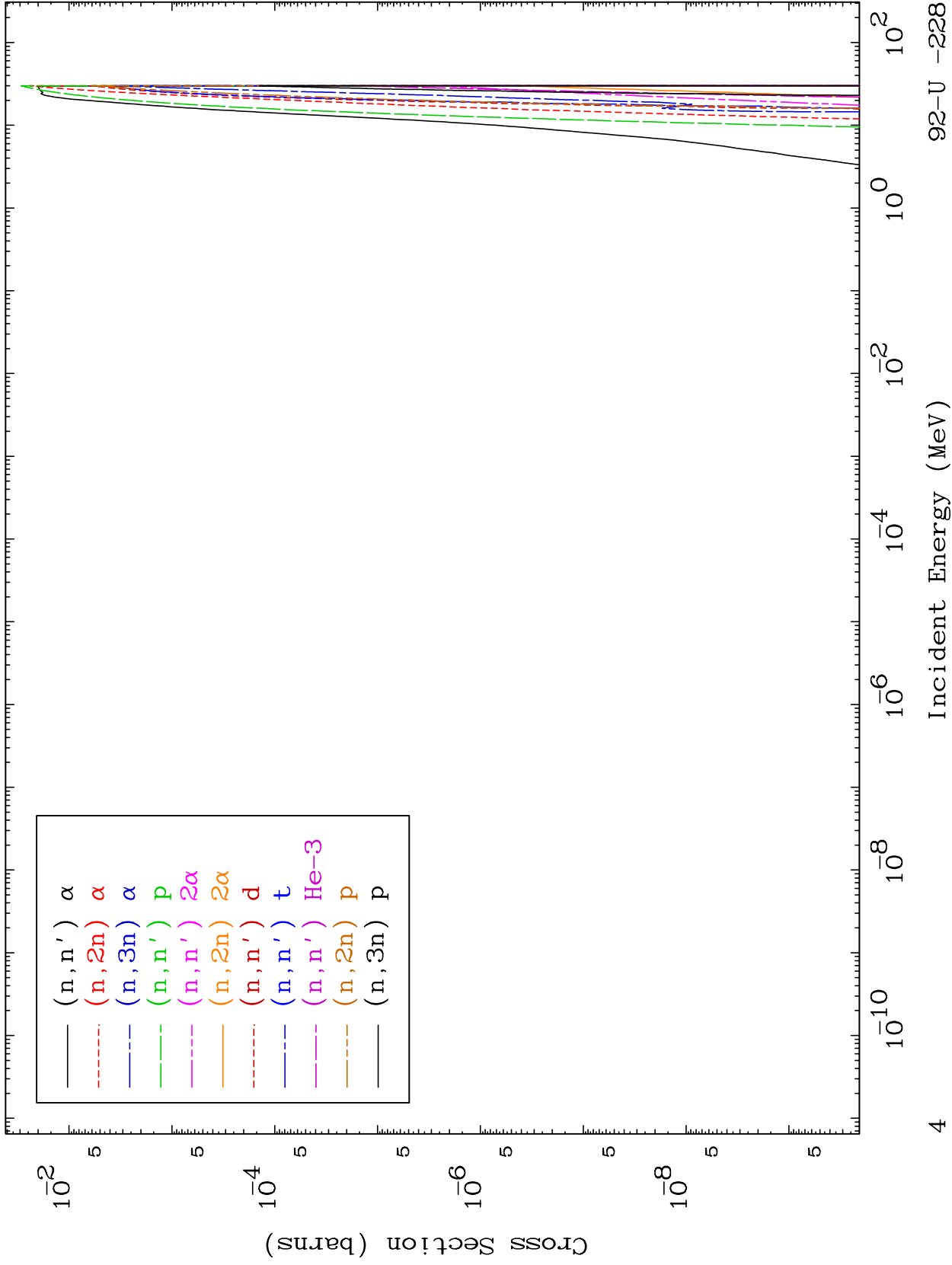
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MAT 9207

Charged Particle
293 Kelvin Cross Sections

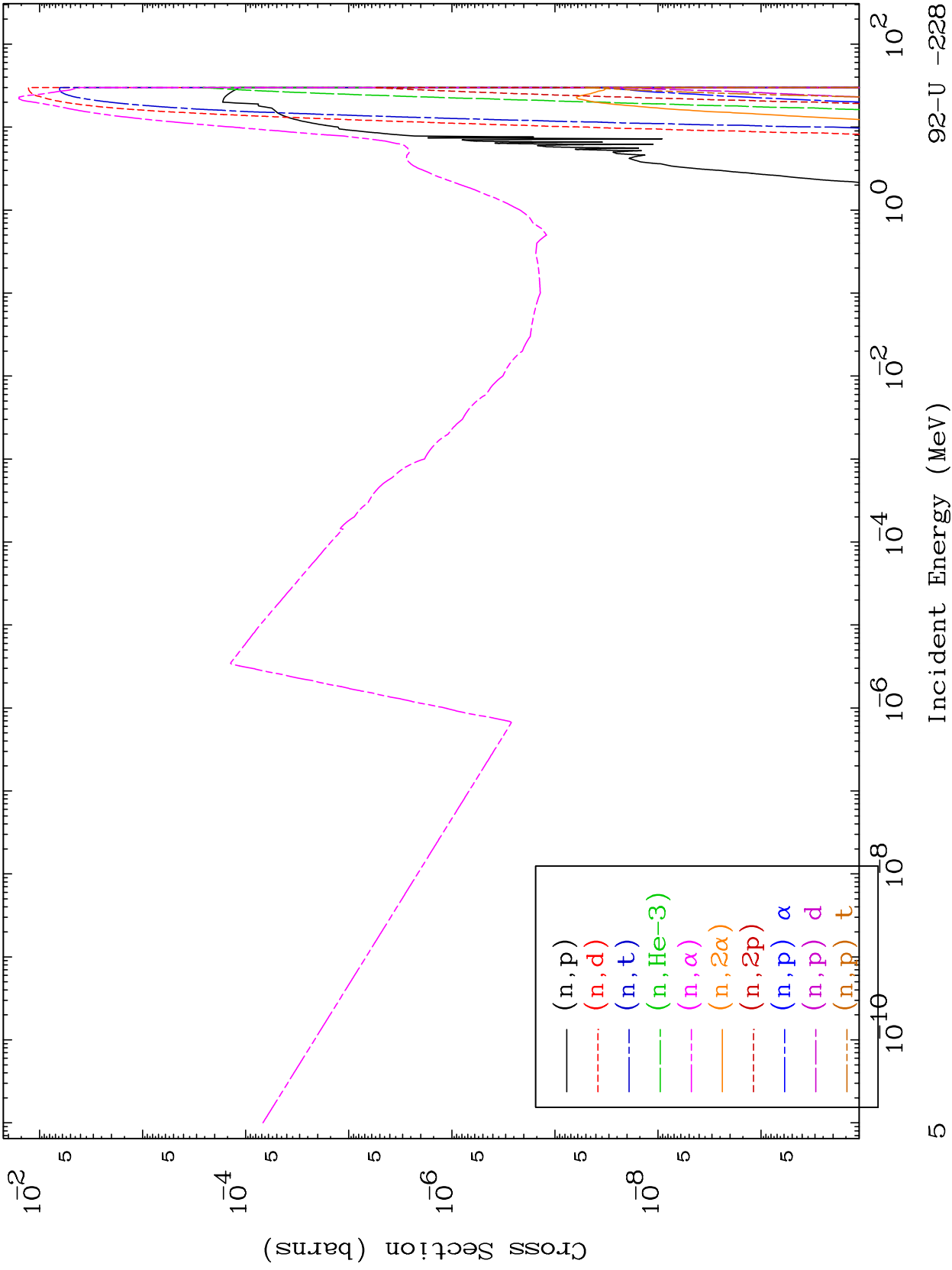
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MAT 9207

Charged Particle
293 Kelvin Cross Sections

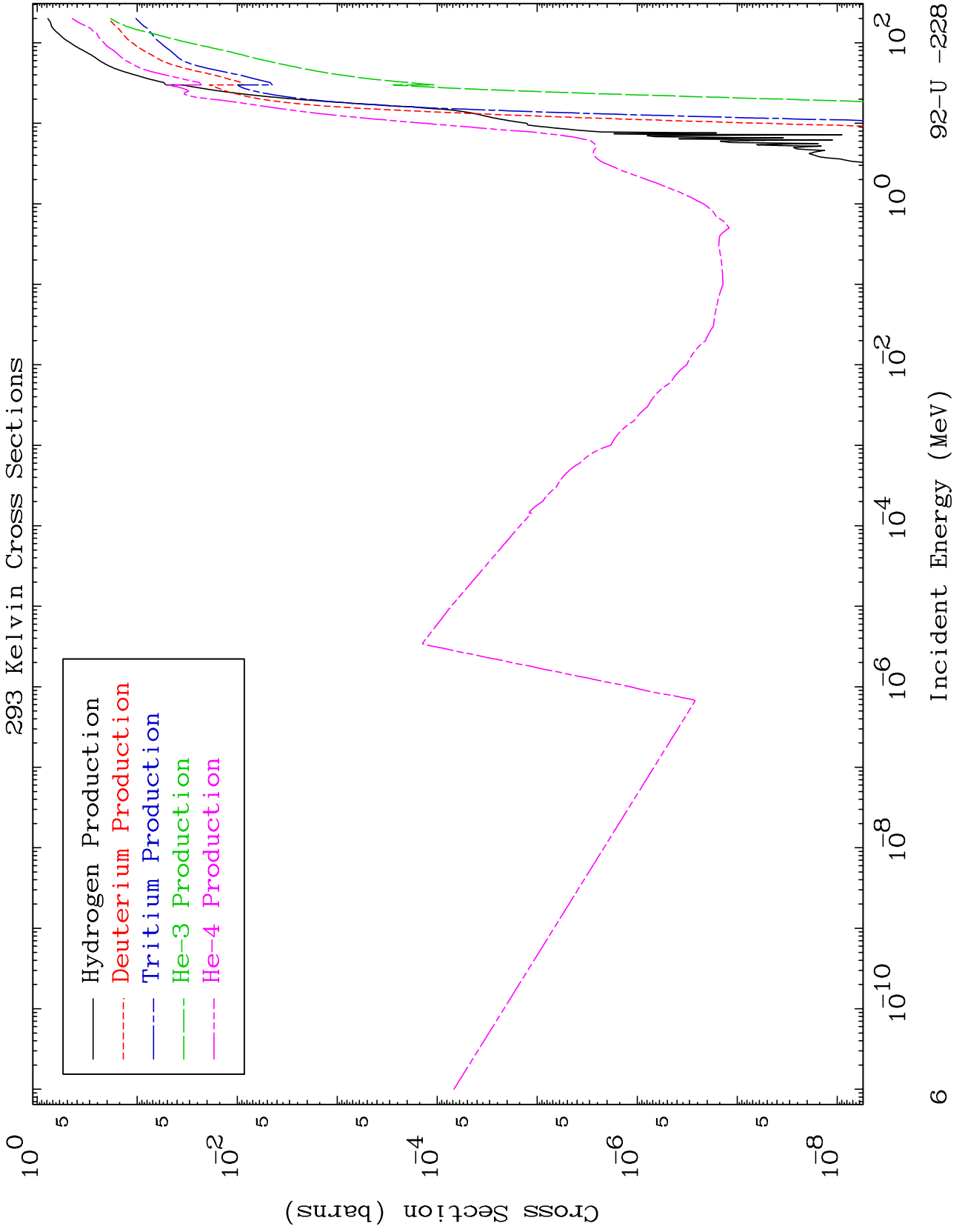
92-U -228



MAT 9207

Particle Production
293 Kelvin Cross Sections

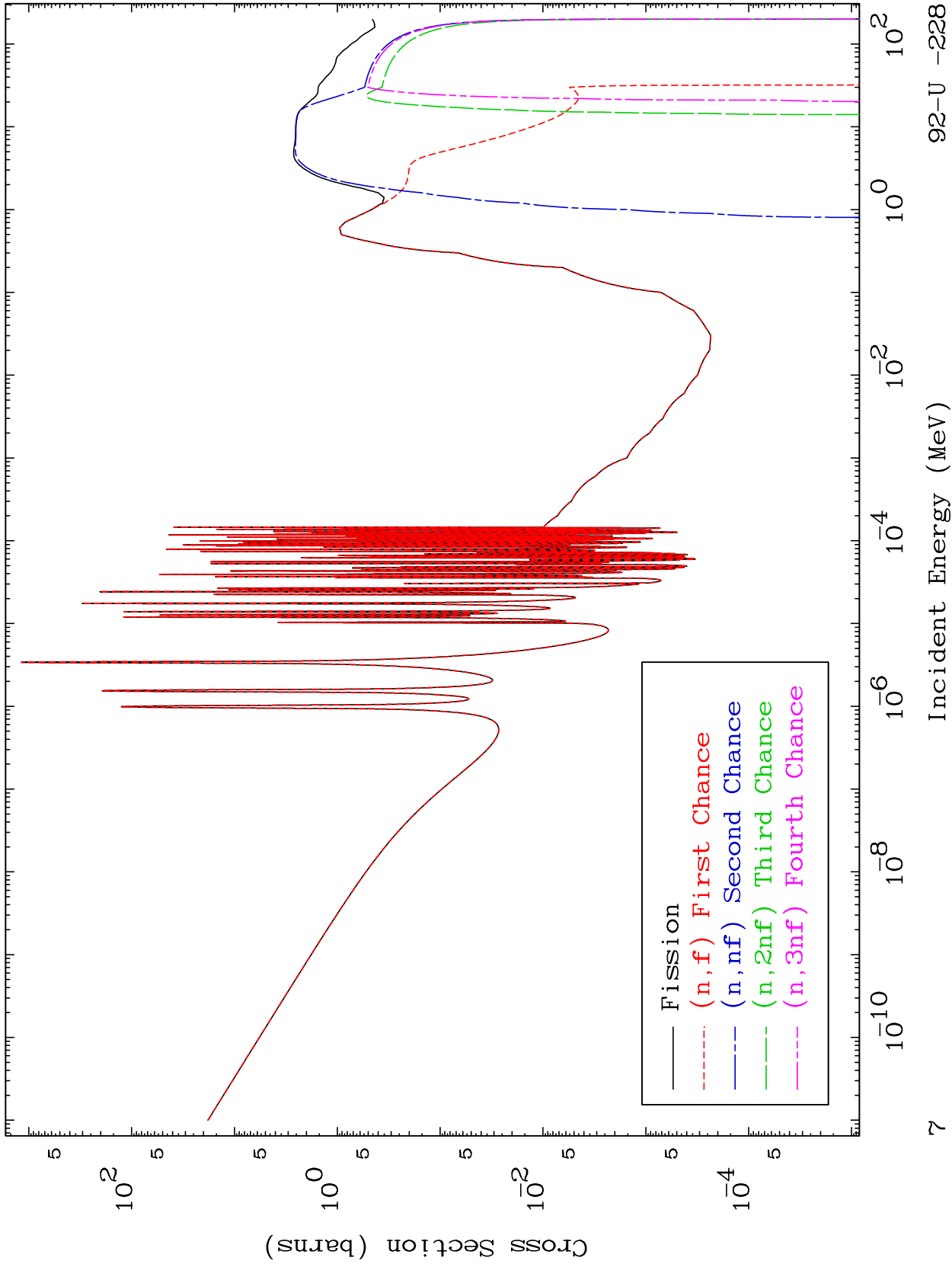
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MAT 9207

Fission
293 Kelvin Cross Sections

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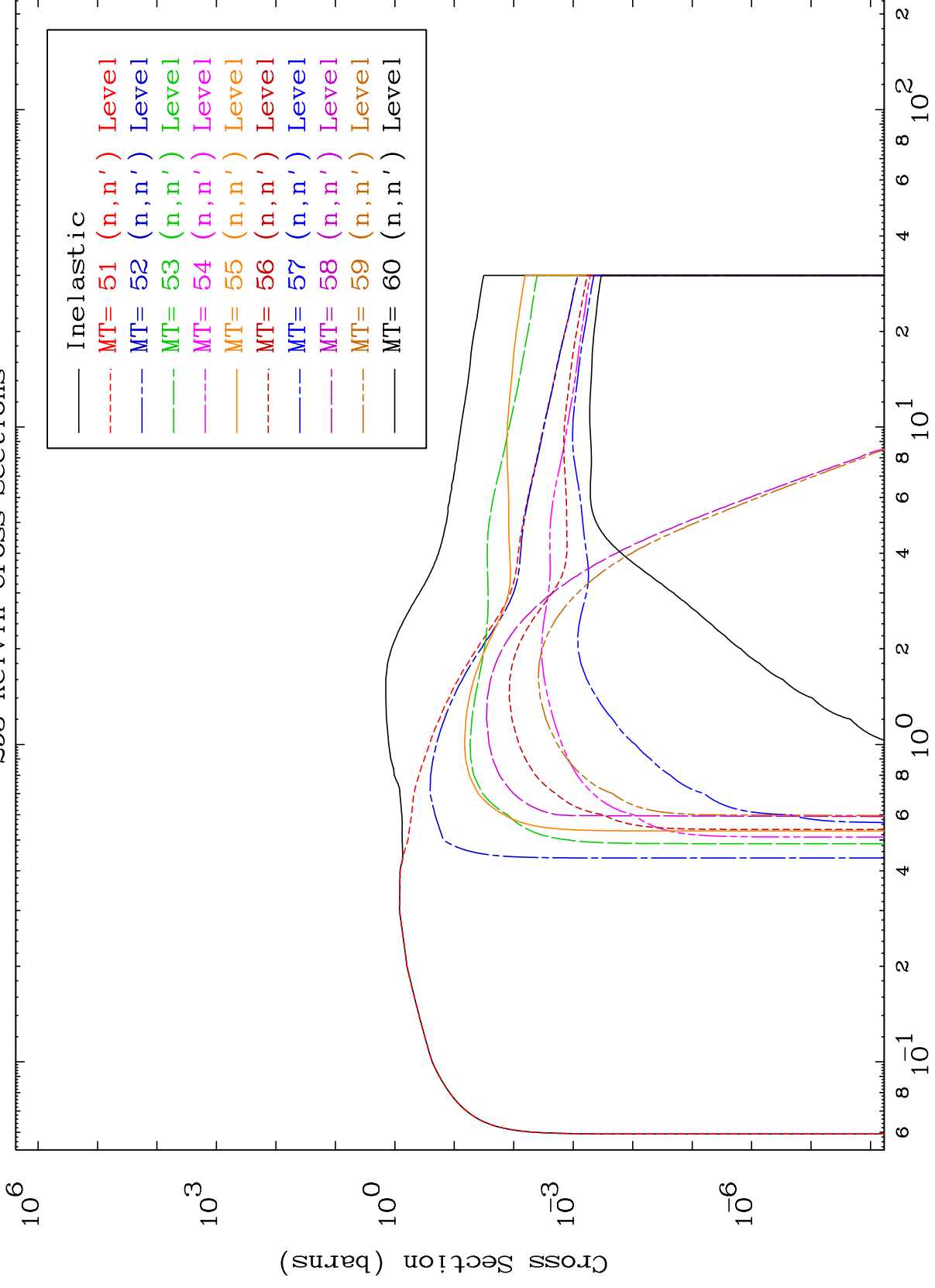


MAT 9207

(n,n') Level

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293 Kelvin Cross Sections

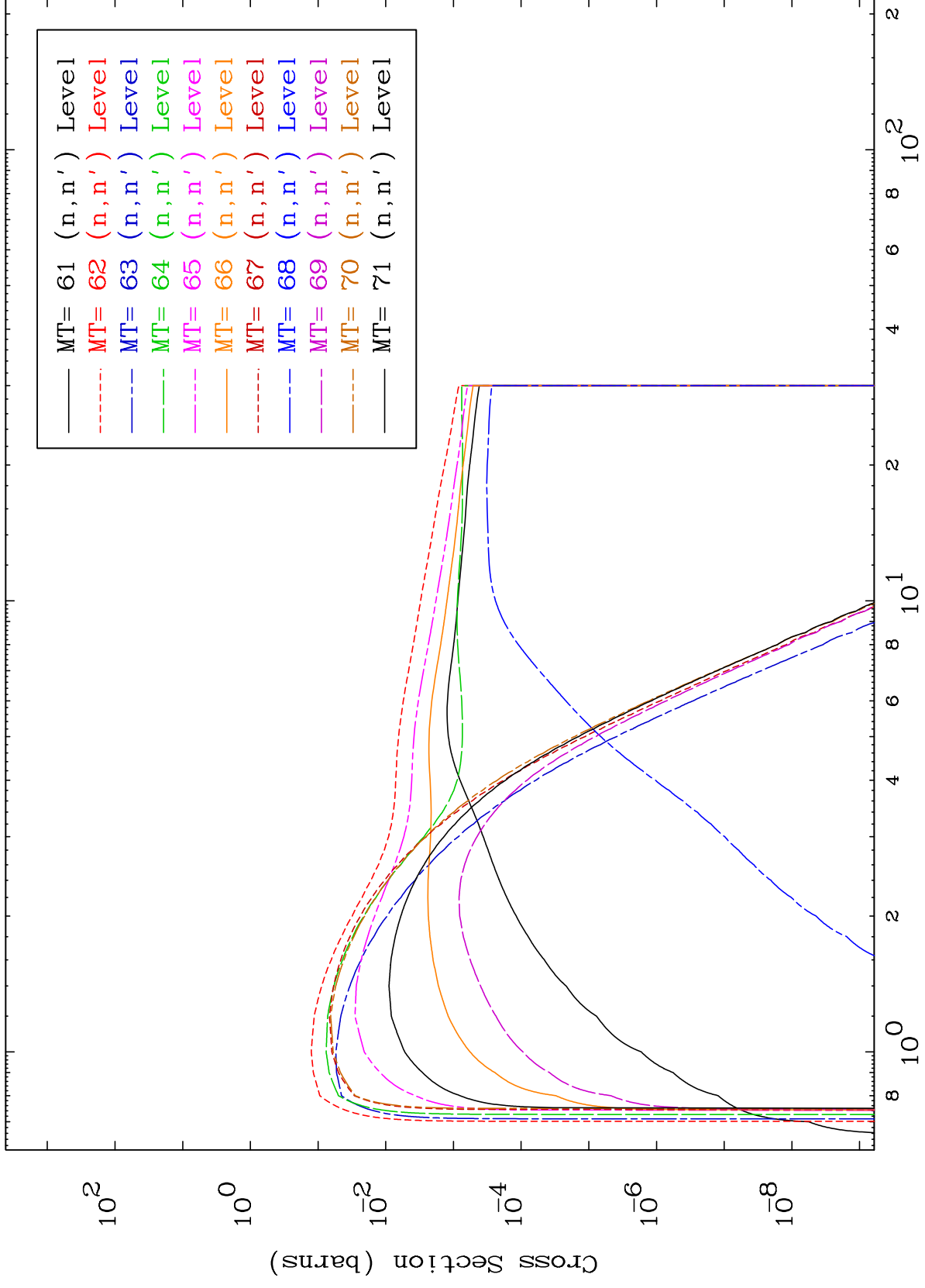


MAT 9207

(n,n') Level

92-U -228

293 Kelvin Cross Sections



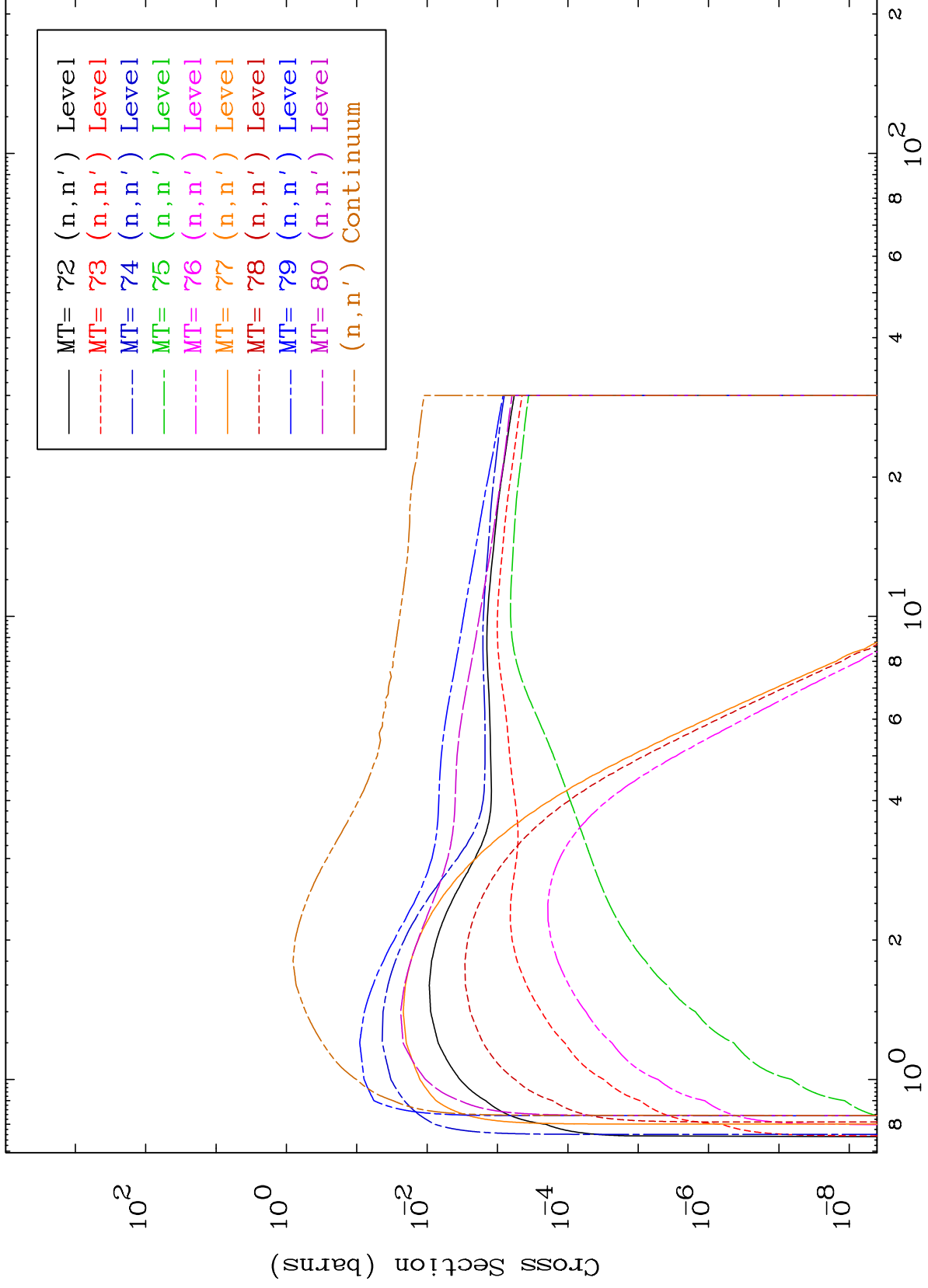
MT=	(n, n')	Level
—	(n, n')	Level 61
- - -	(n, n')	Level 62
- · - ·	(n, n')	Level 63
- - -	(n, n')	Level 64
- · - ·	(n, n')	Level 65
—	(n, n')	Level 66
- - -	(n, n')	Level 67
- · - ·	(n, n')	Level 68
- - -	(n, n')	Level 69
- · - ·	(n, n')	Level 70
—	(n, n')	Level 71

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(n,n') Level

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293 Kelvin Cross Sections



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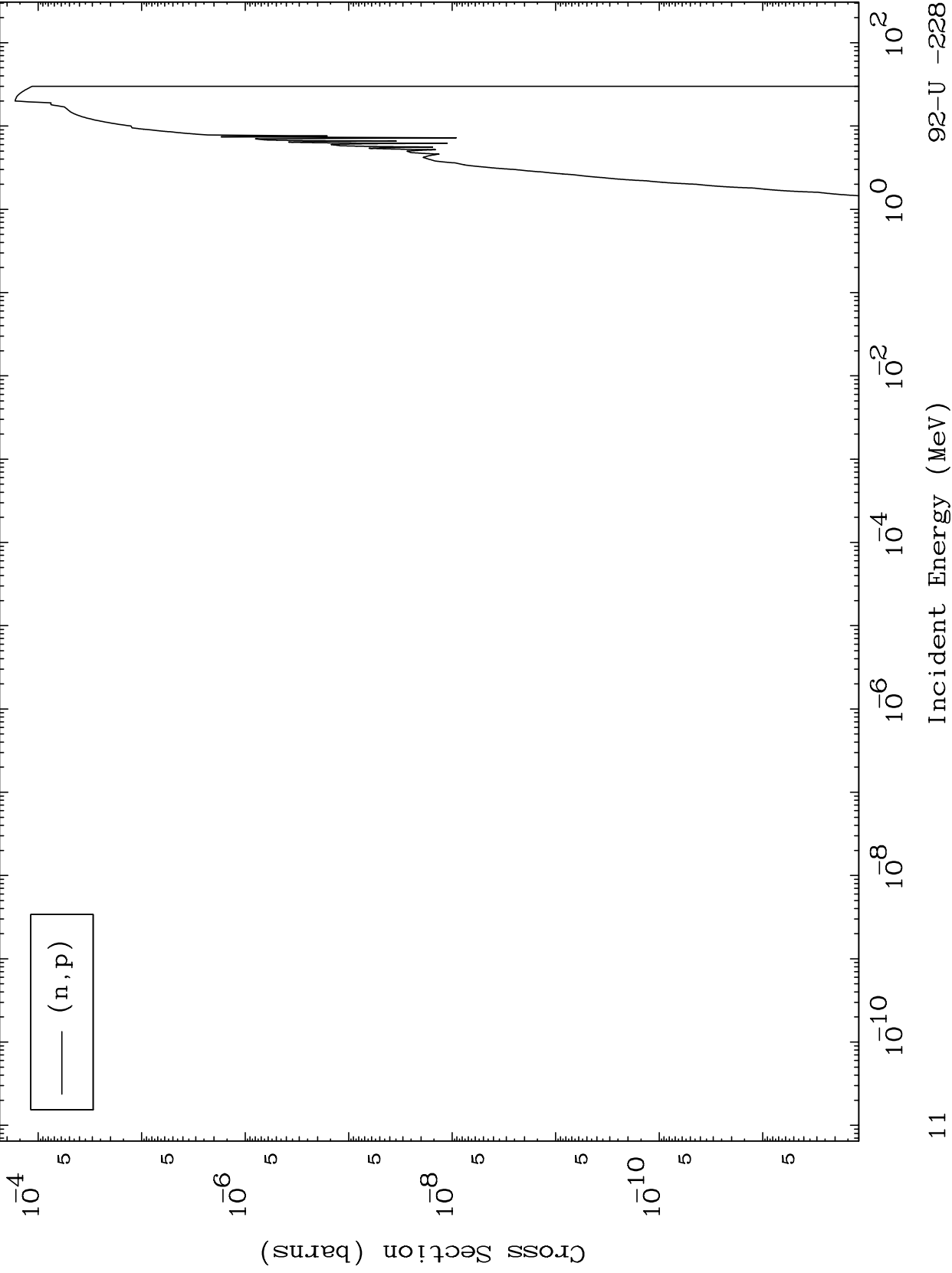
Incident Energy (MeV)

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MAT 9207

(n,p) Levels
293 Kelvin Cross Sections

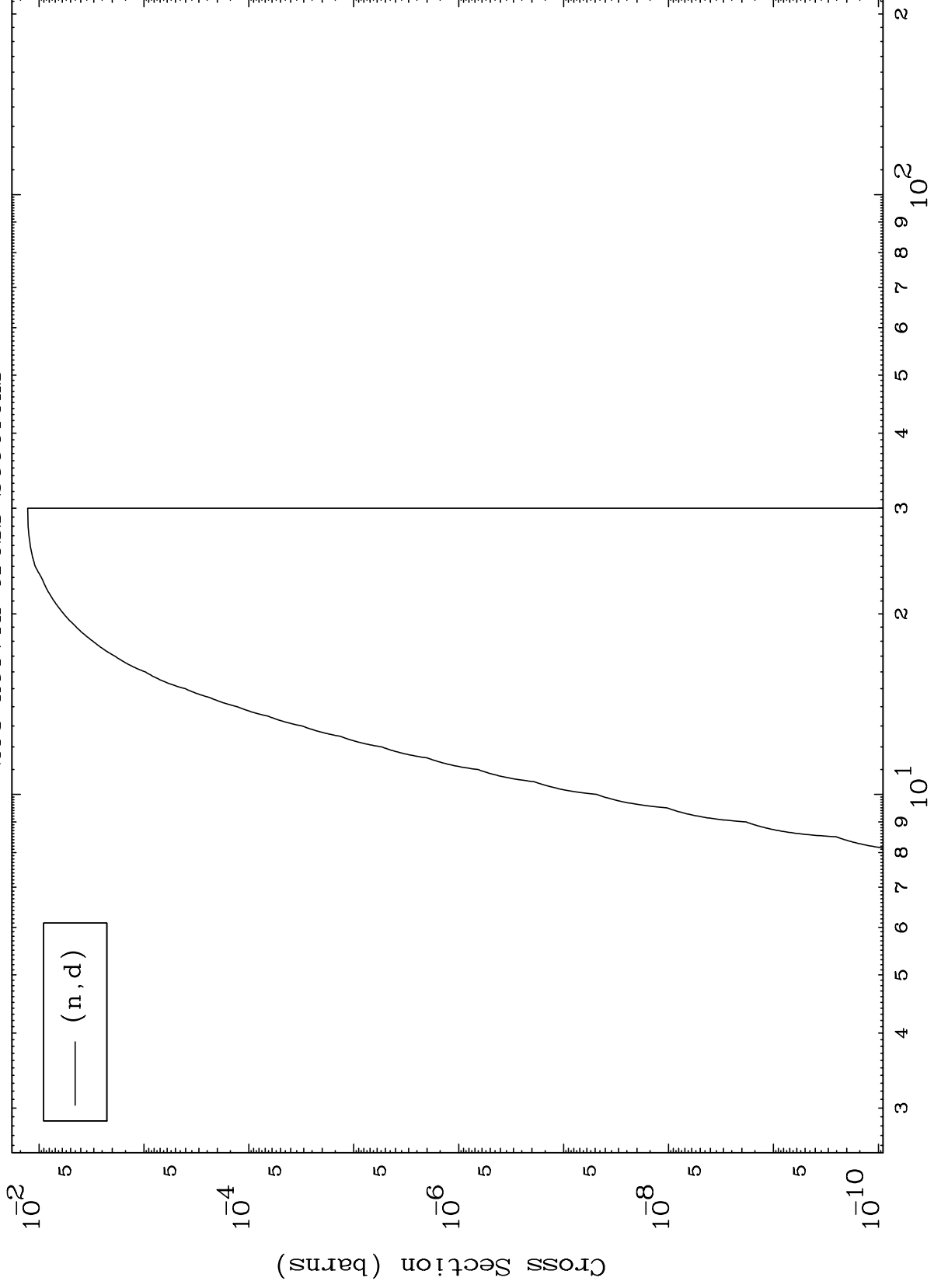
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(n,d) Levels
293 Kelvin Cross Sections

92-U -228



12

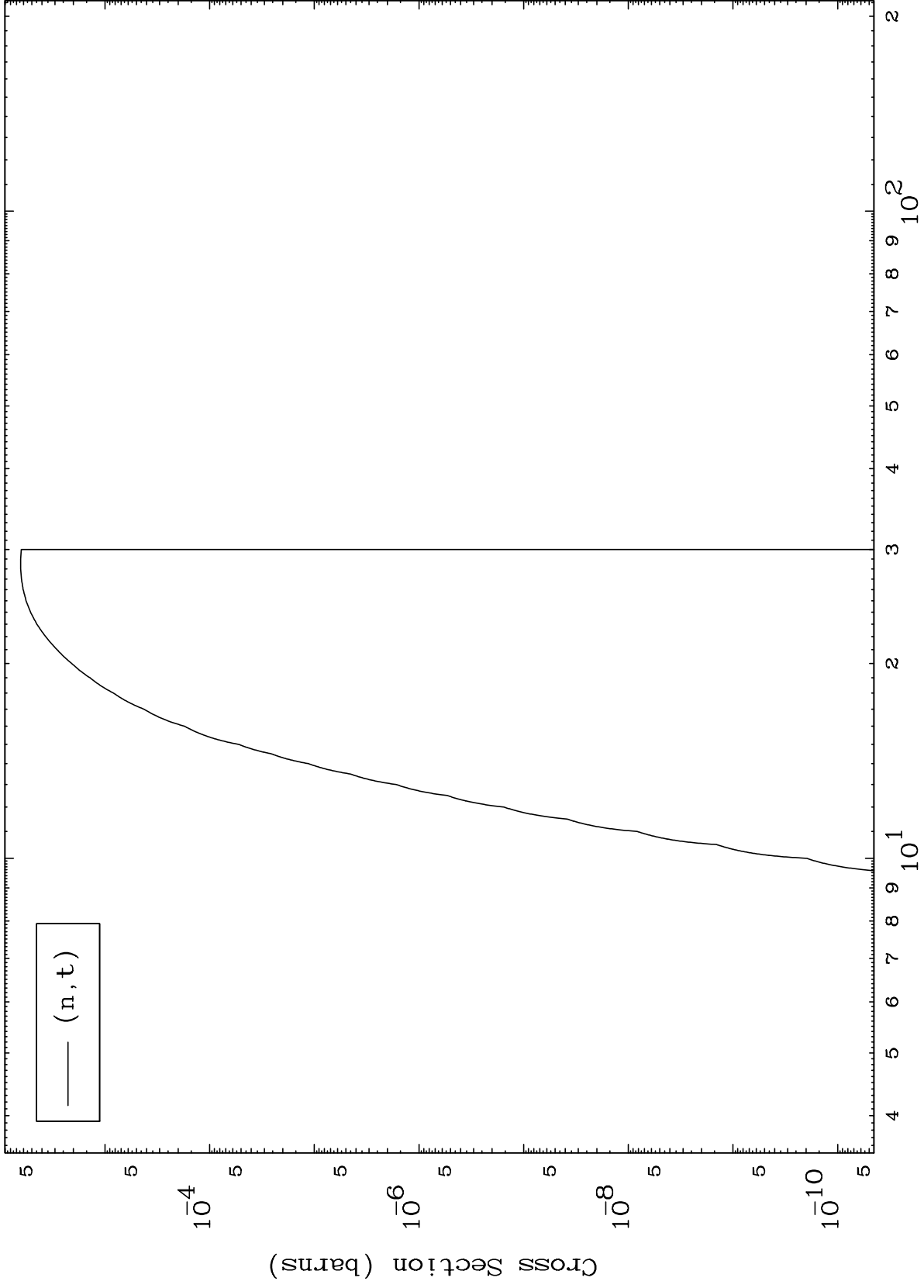
Incident Energy (MeV)

92-U -228

MAT 9207

(n,t) Levels
293 Kelvin Cross Sections

92-U -228



13

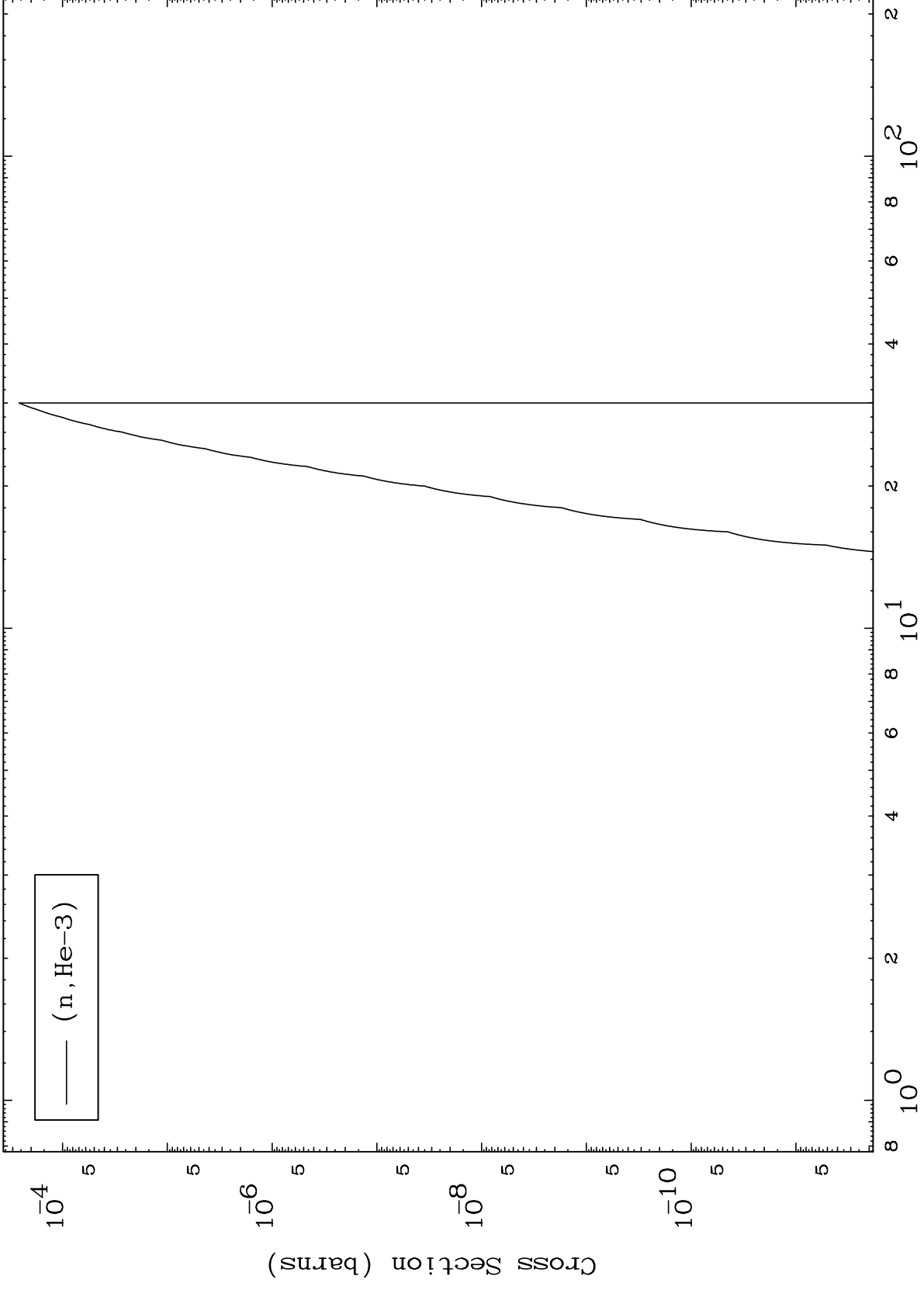
Incident Energy (MeV)

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MAT 9207

(n,He3) Levels
293 Kelvin Cross Sections

92-U -228



14

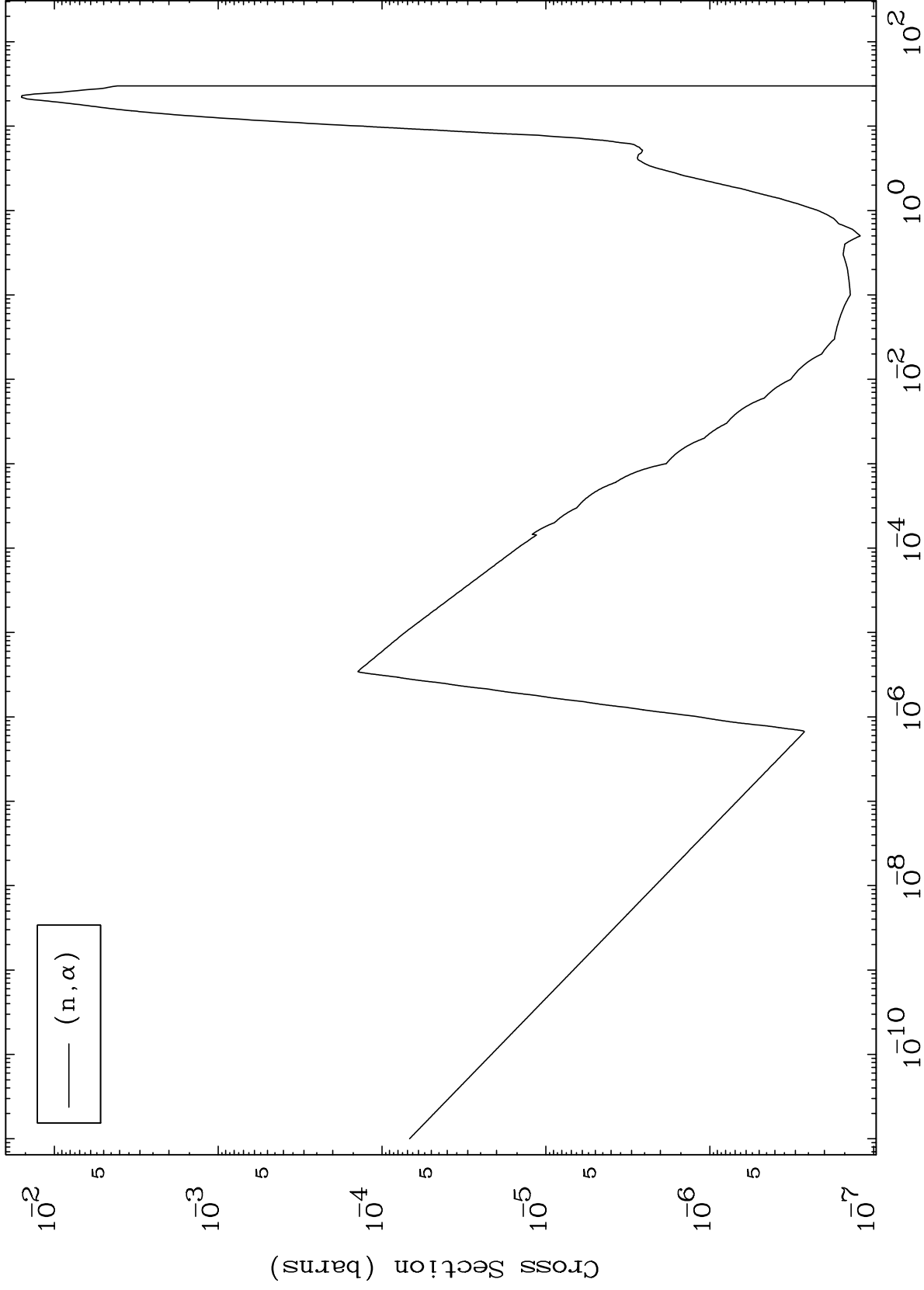
Incident Energy (MeV)

92-U -228

MAT 9207

(n, α) Levels
293 Kelvin Cross Sections

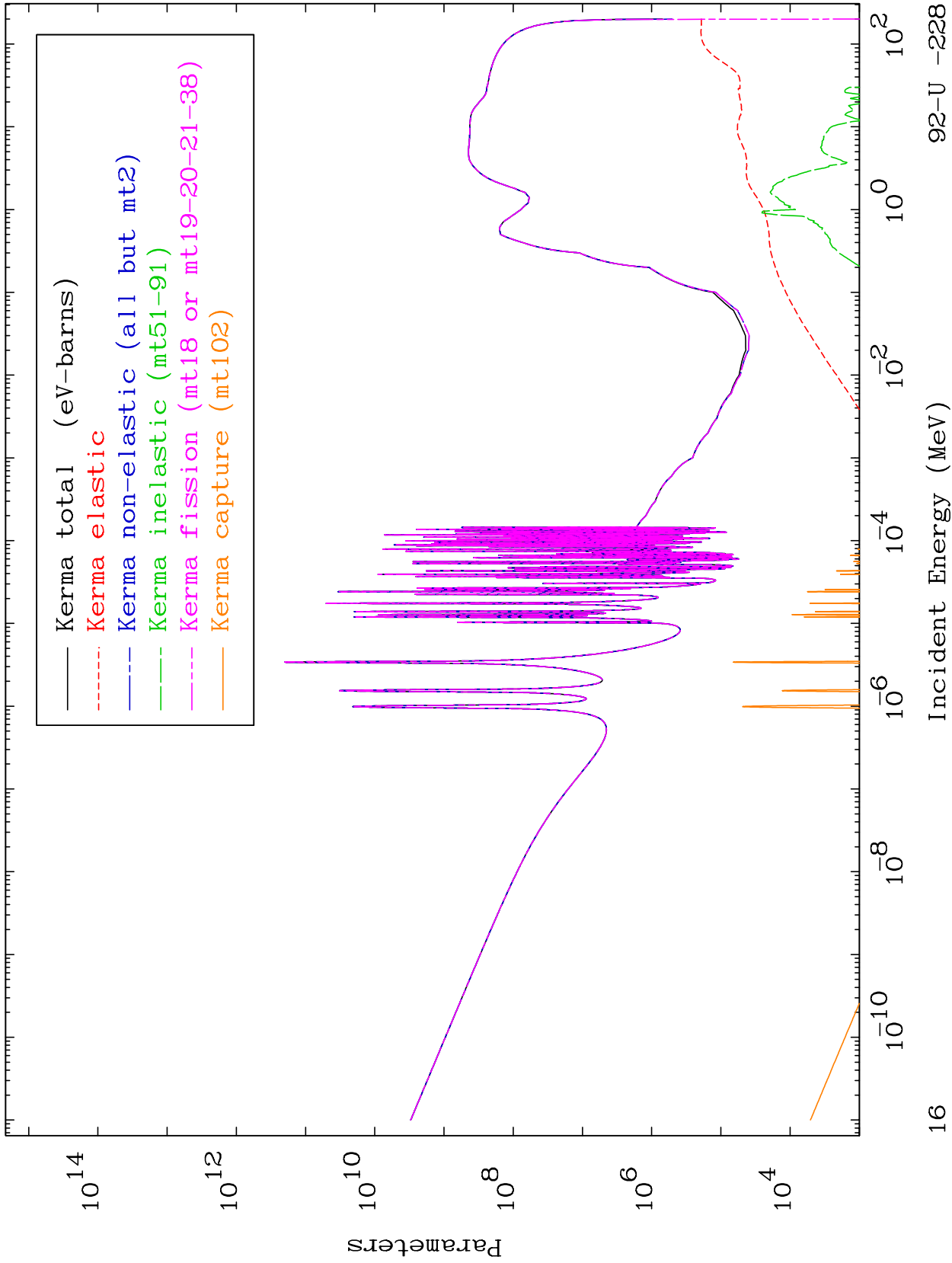
92-U -228

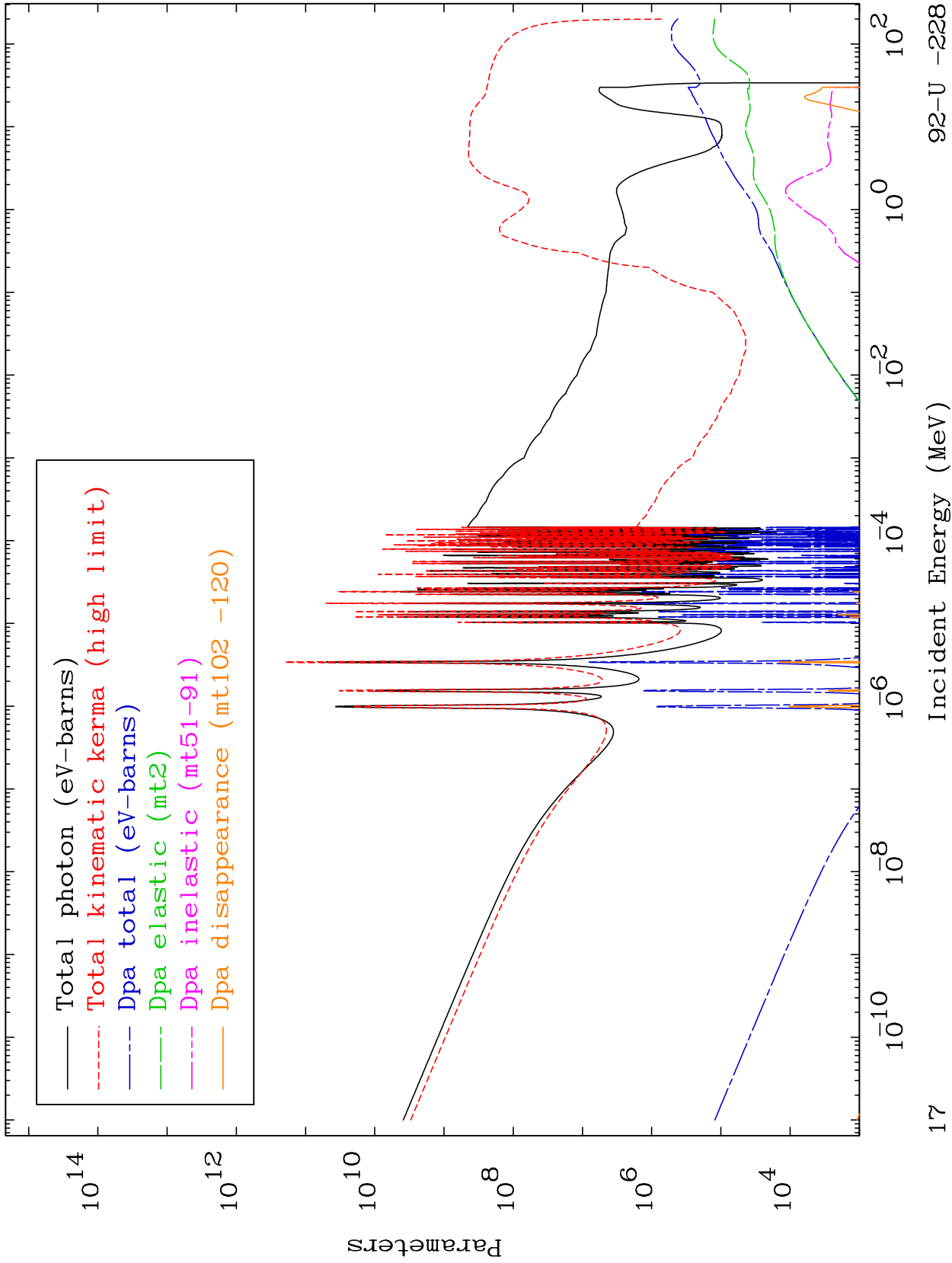


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Incident Energy (MeV)

92-U -228

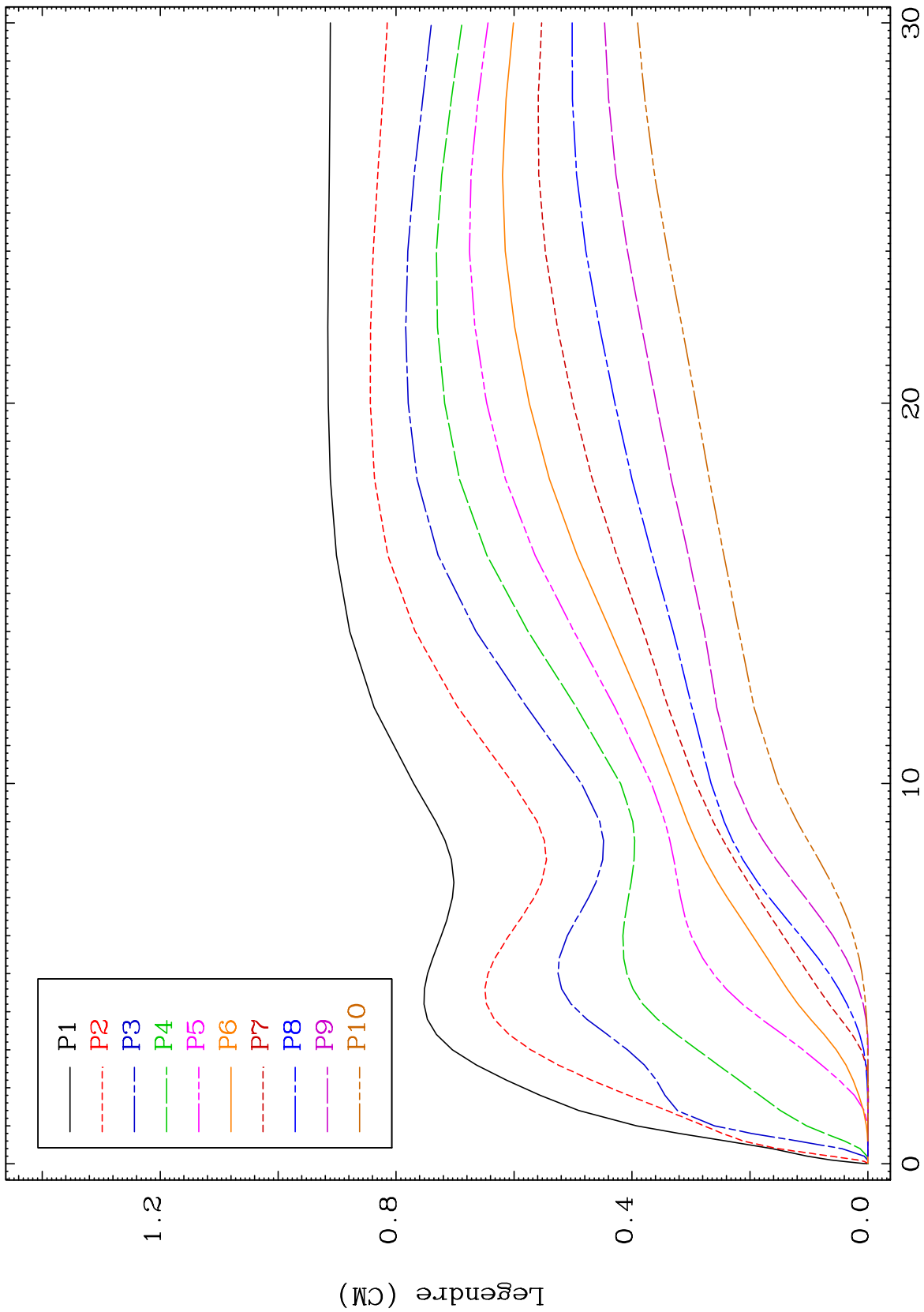




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Elastic Legendre Coefficients

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Incident Energy (MeV)

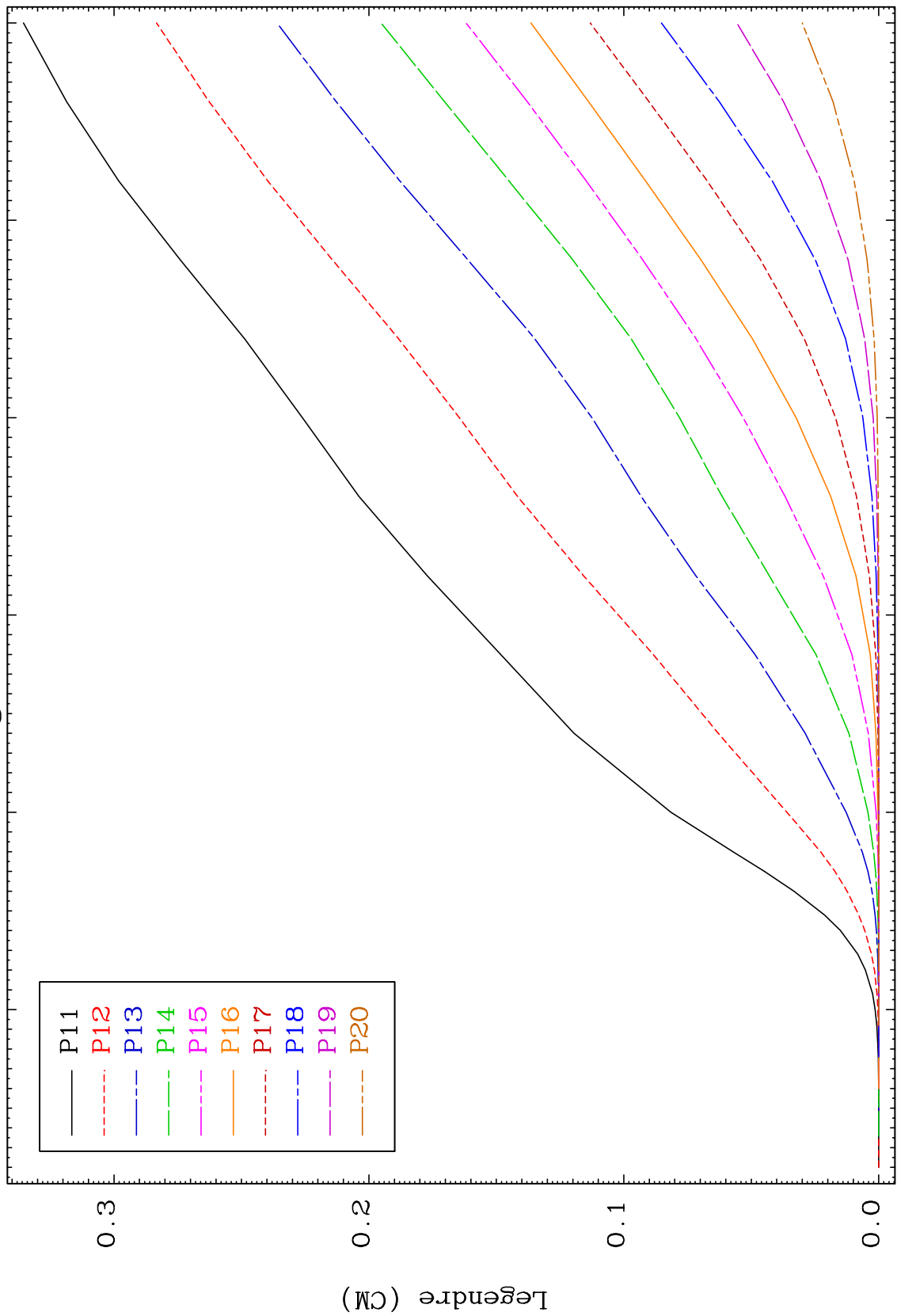
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MAT 9207

Elastic Legendre Coefficients

92-U -228



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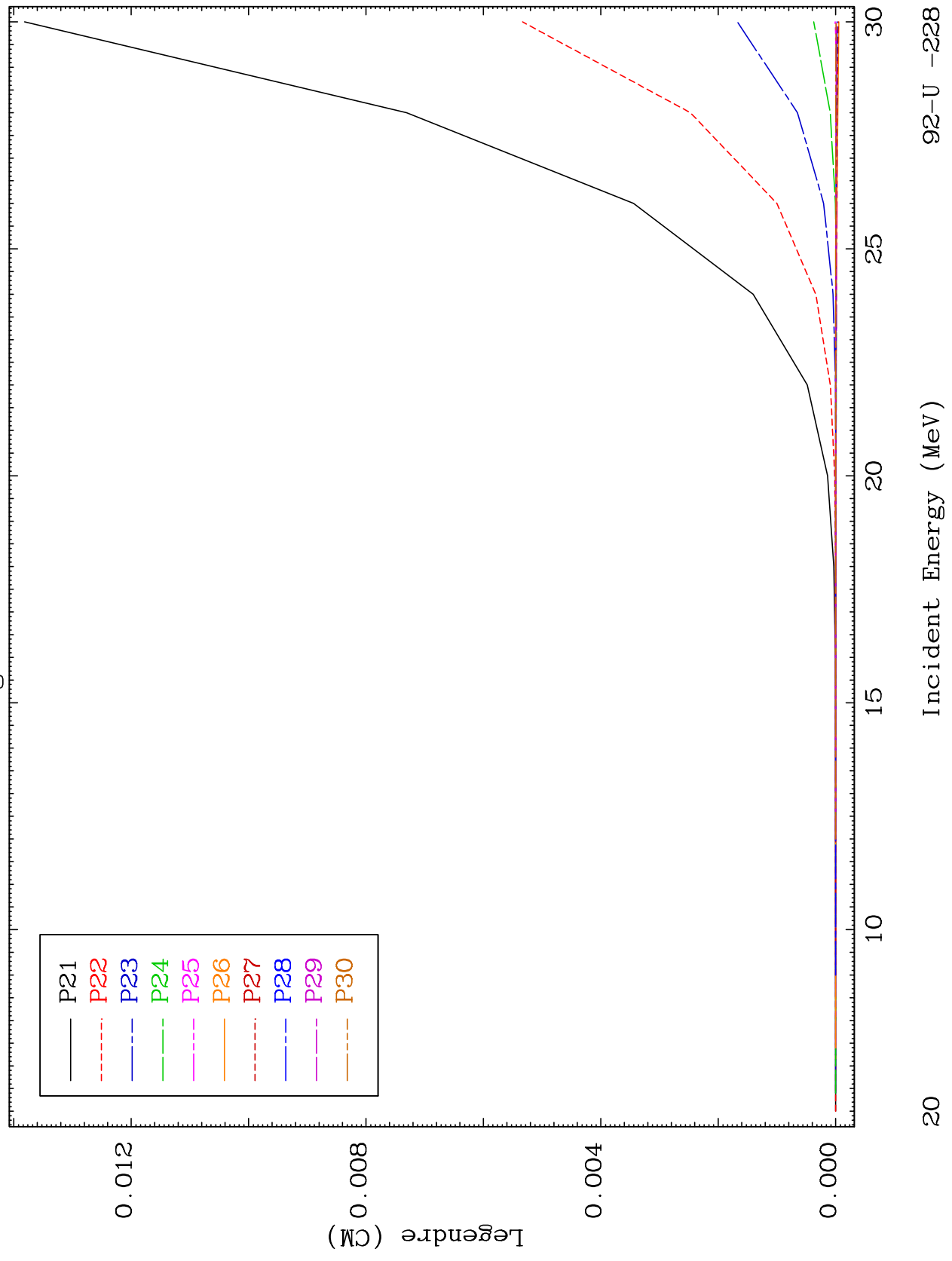
Incident Energy (MeV)

92-U -228

MAT 9207

Elastic Legendre Coefficients

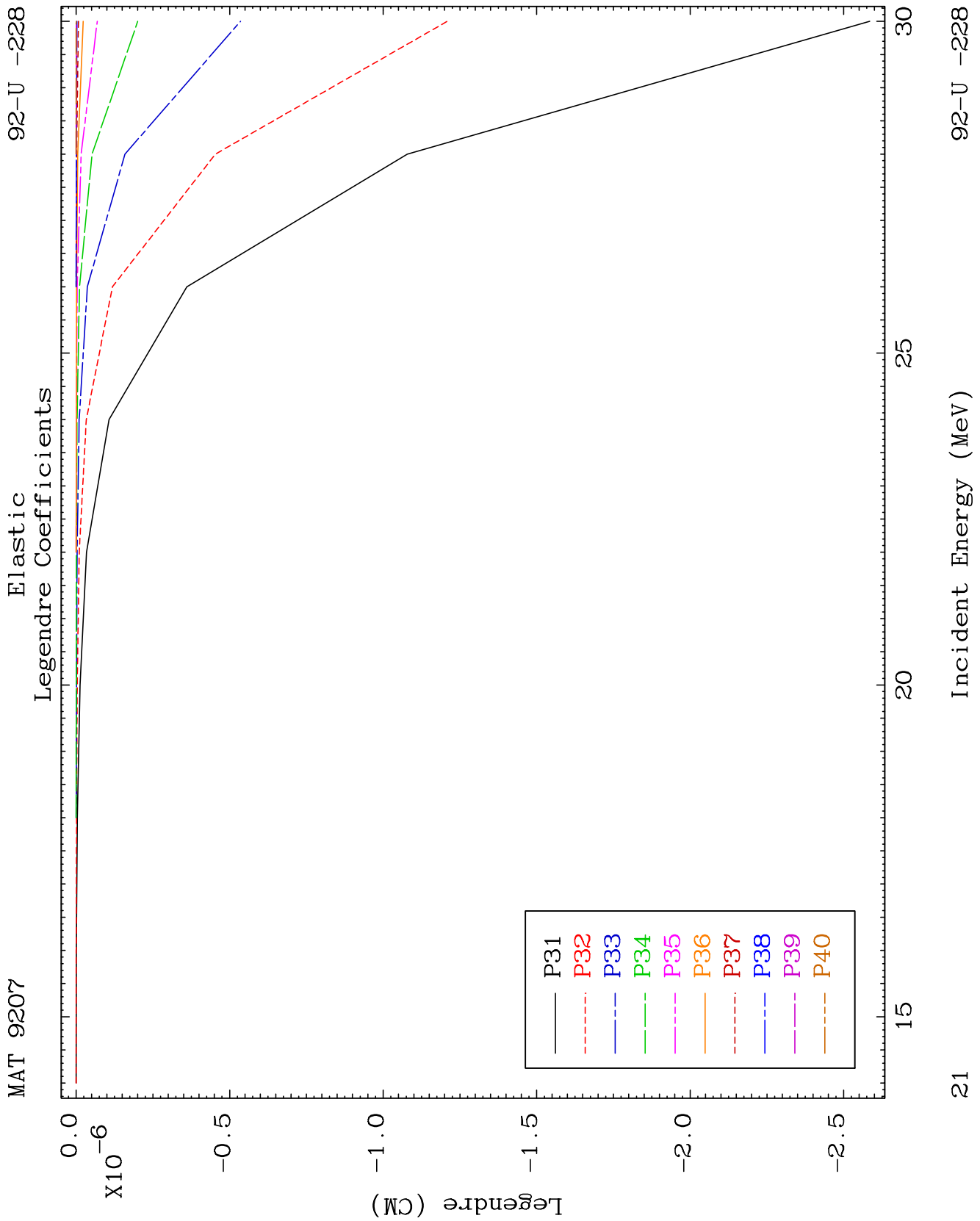
92-U -228



20

Incident Energy (MeV)

92-U -228

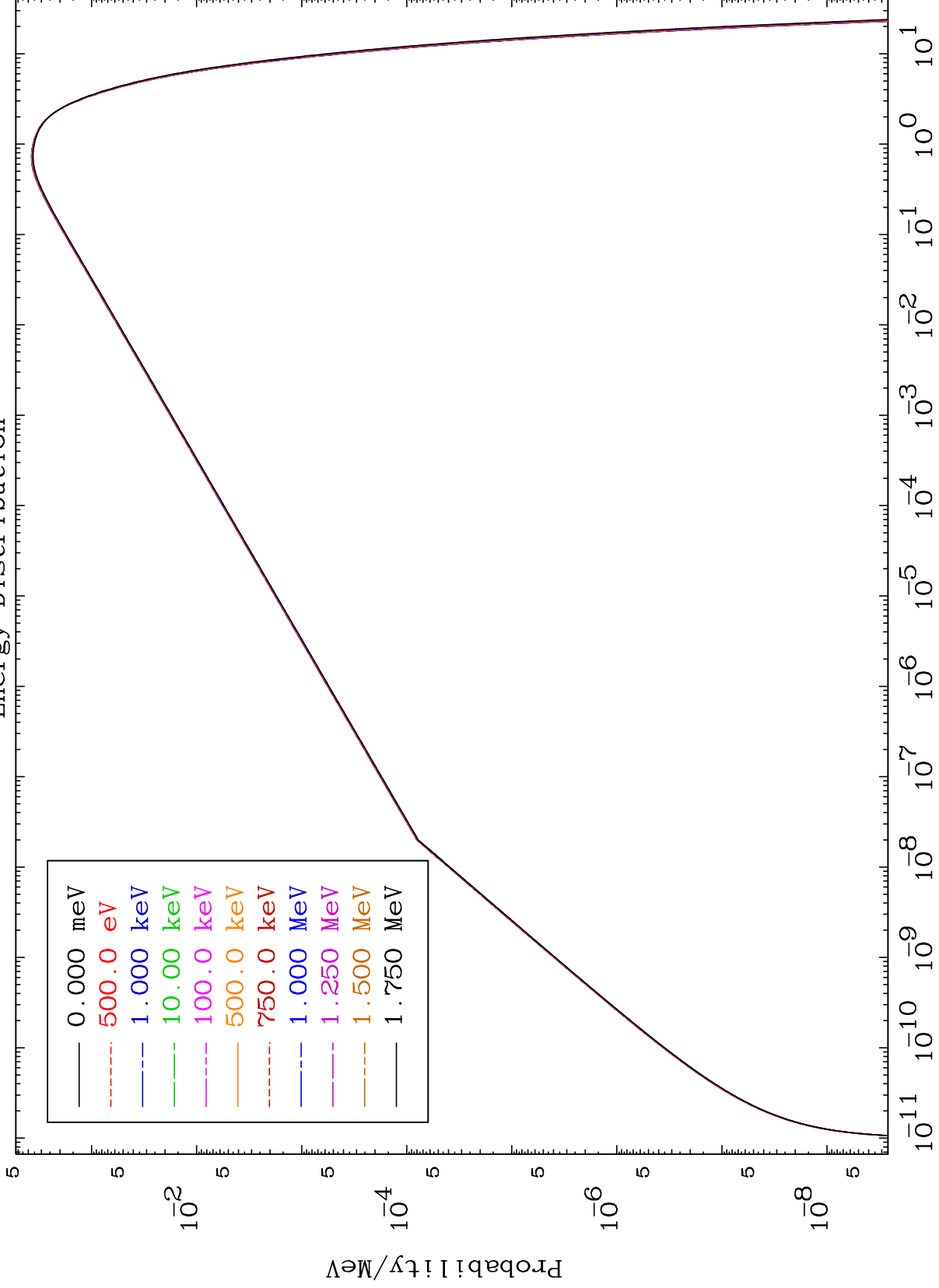


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Fission

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Energy Distribution



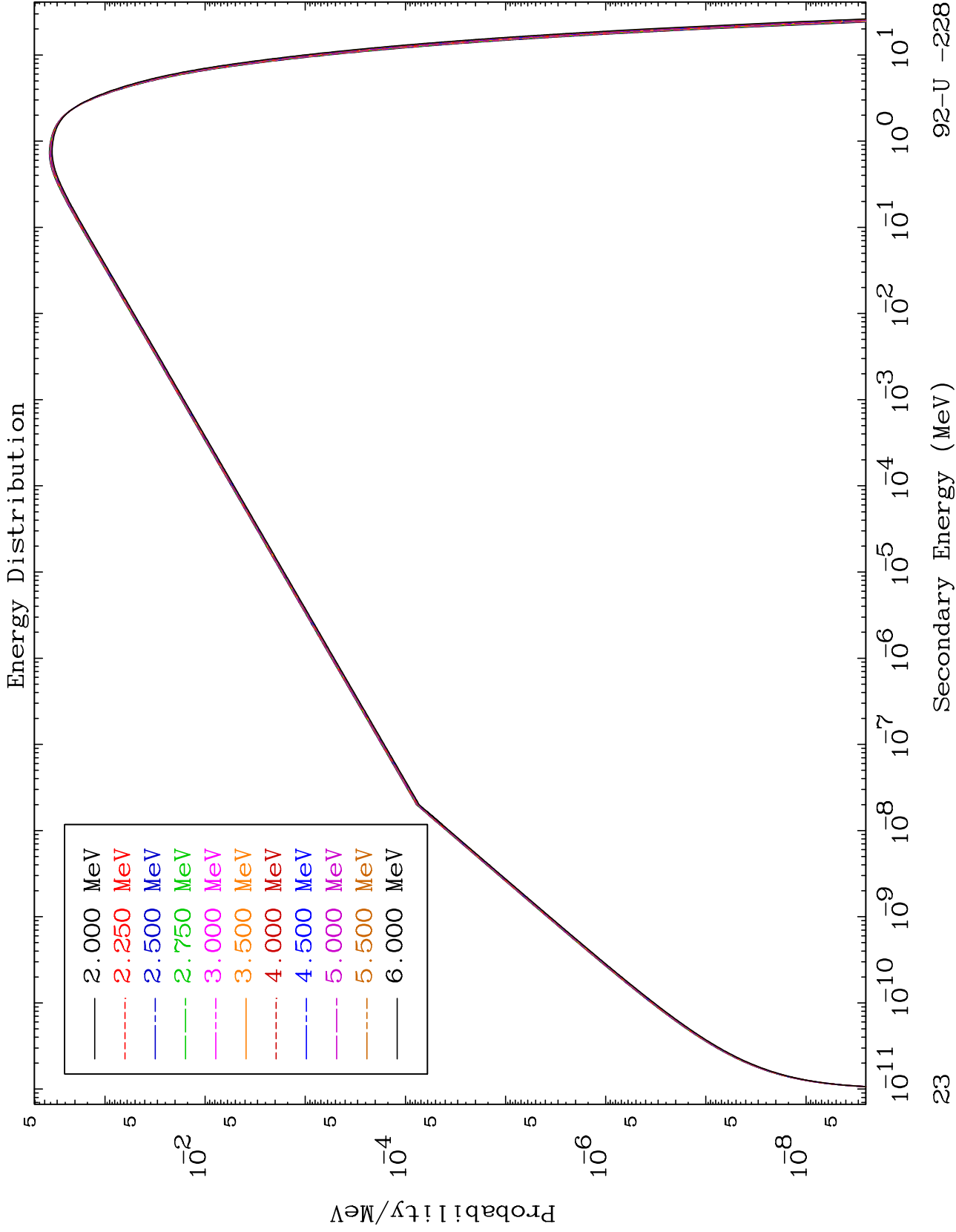
22

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MAT 9207

Fission

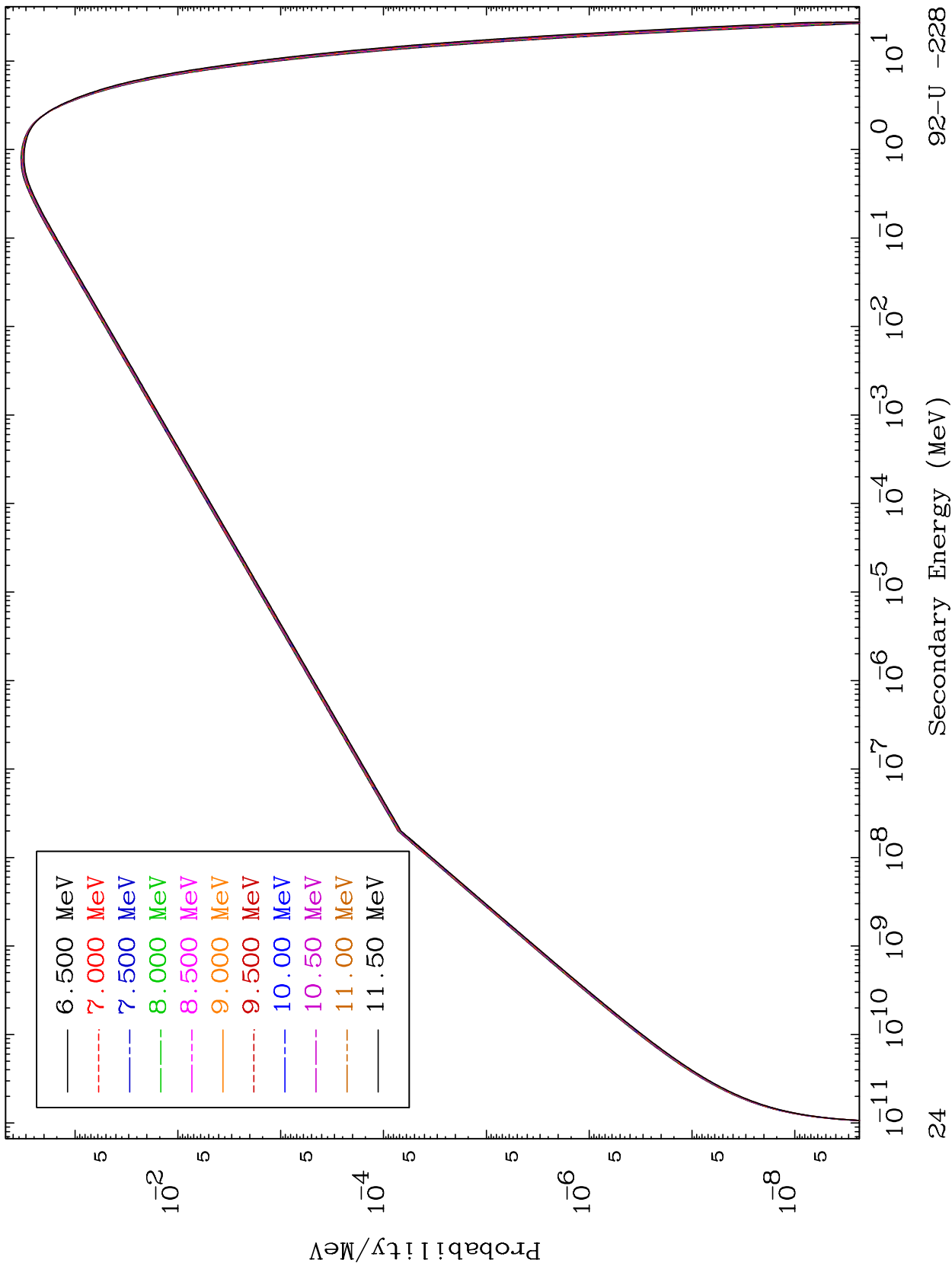
92-U -228



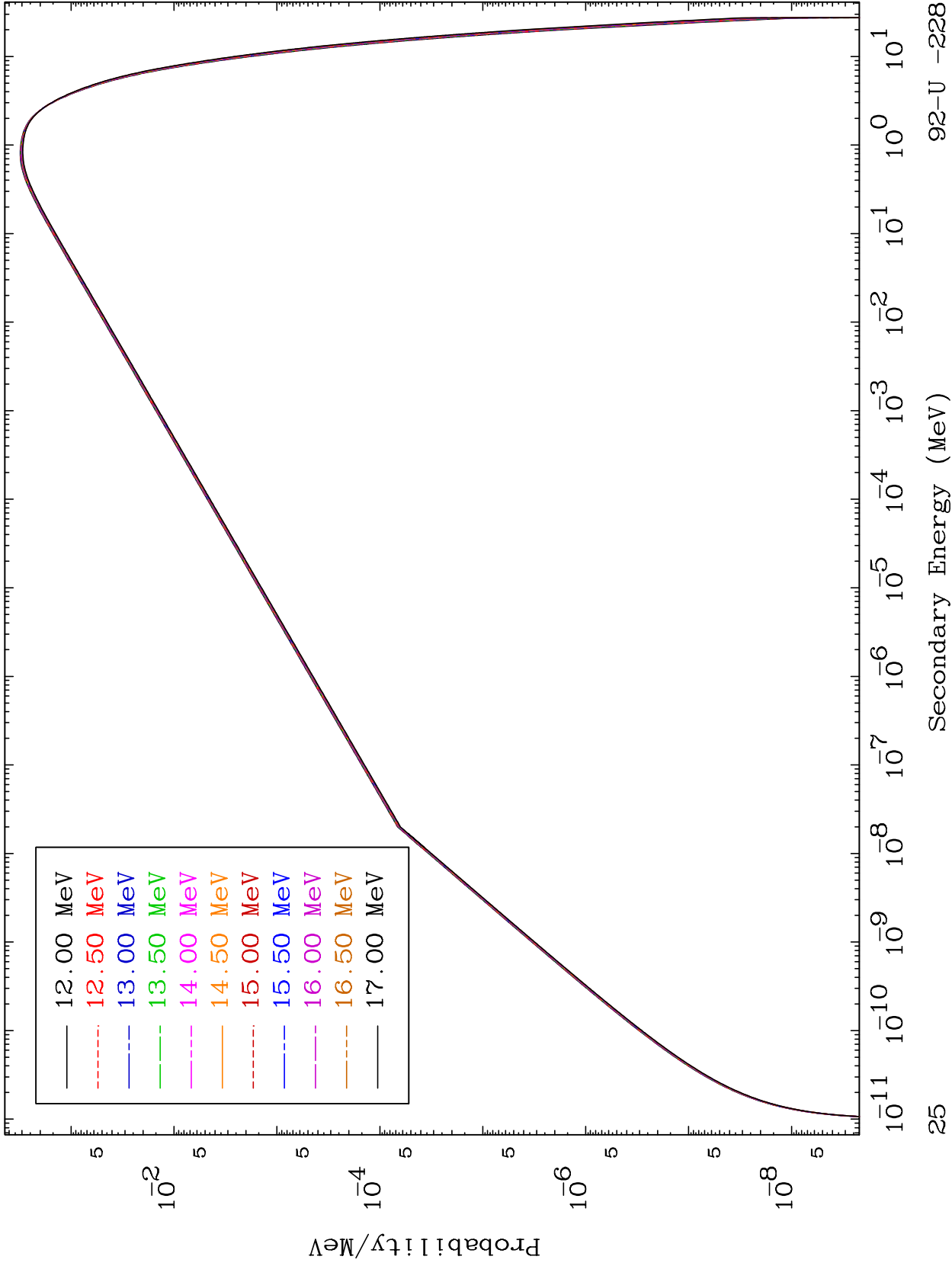
MAT 9207

Fission Energy Distribution

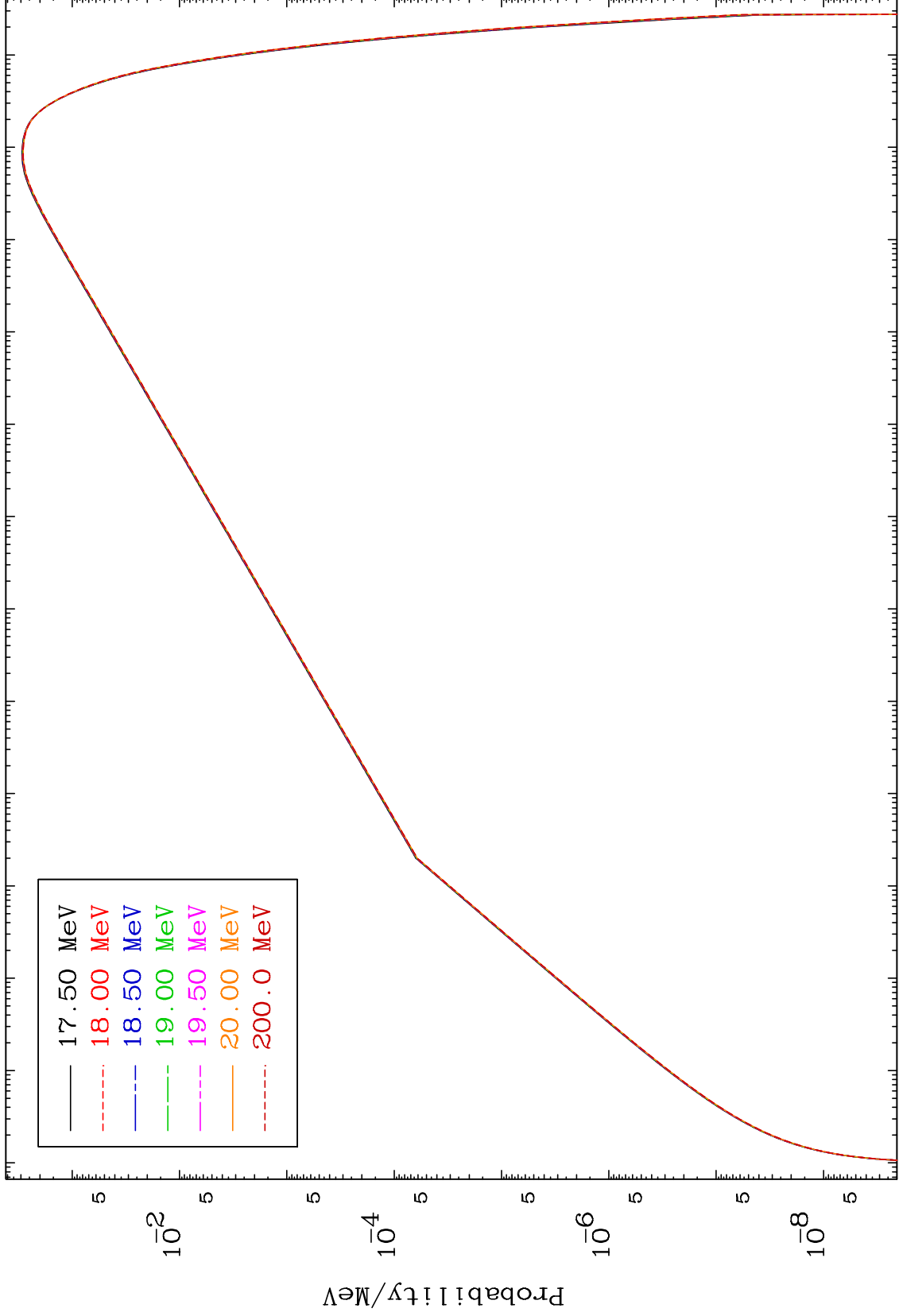
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Fission Energy Distribution



Energy Distribution



10⁻¹¹ 10⁻¹⁰ 10⁻⁹ 10⁻⁸ 10⁻⁷ 10⁻⁶ 10⁻⁵ 10⁻⁴ 10⁻³ 10⁻² 10⁻¹ 10⁰ 10¹

Secondary Energy (MeV)