

Program EVALPLOT
(Version 2018-1)

by

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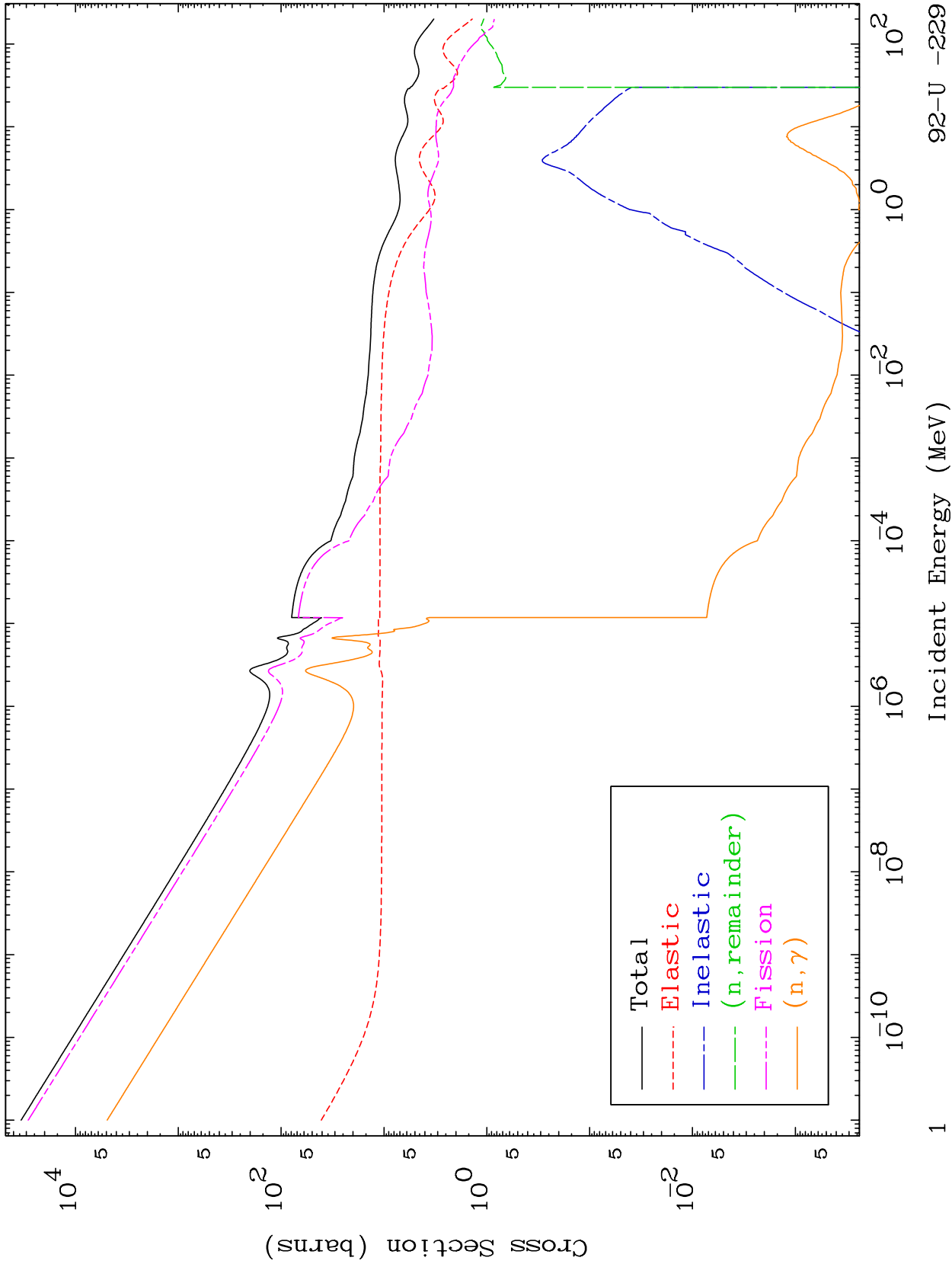
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Press Mouse Button to Start

MAT 9210

Major
293 Kelvin Cross Sections

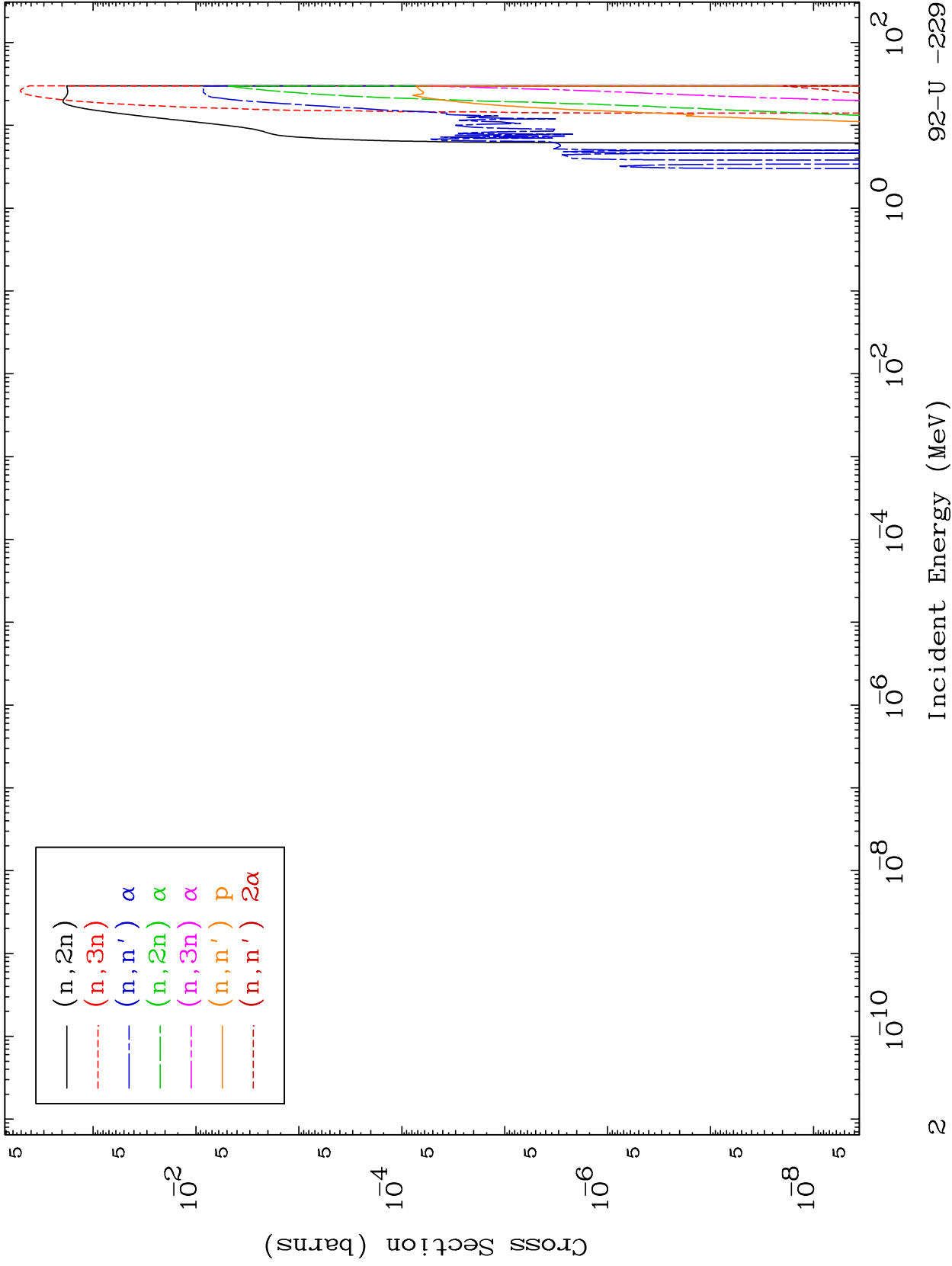
92-U -229



MAT 9210

Neutron Production
293 Kelvin Cross Sections

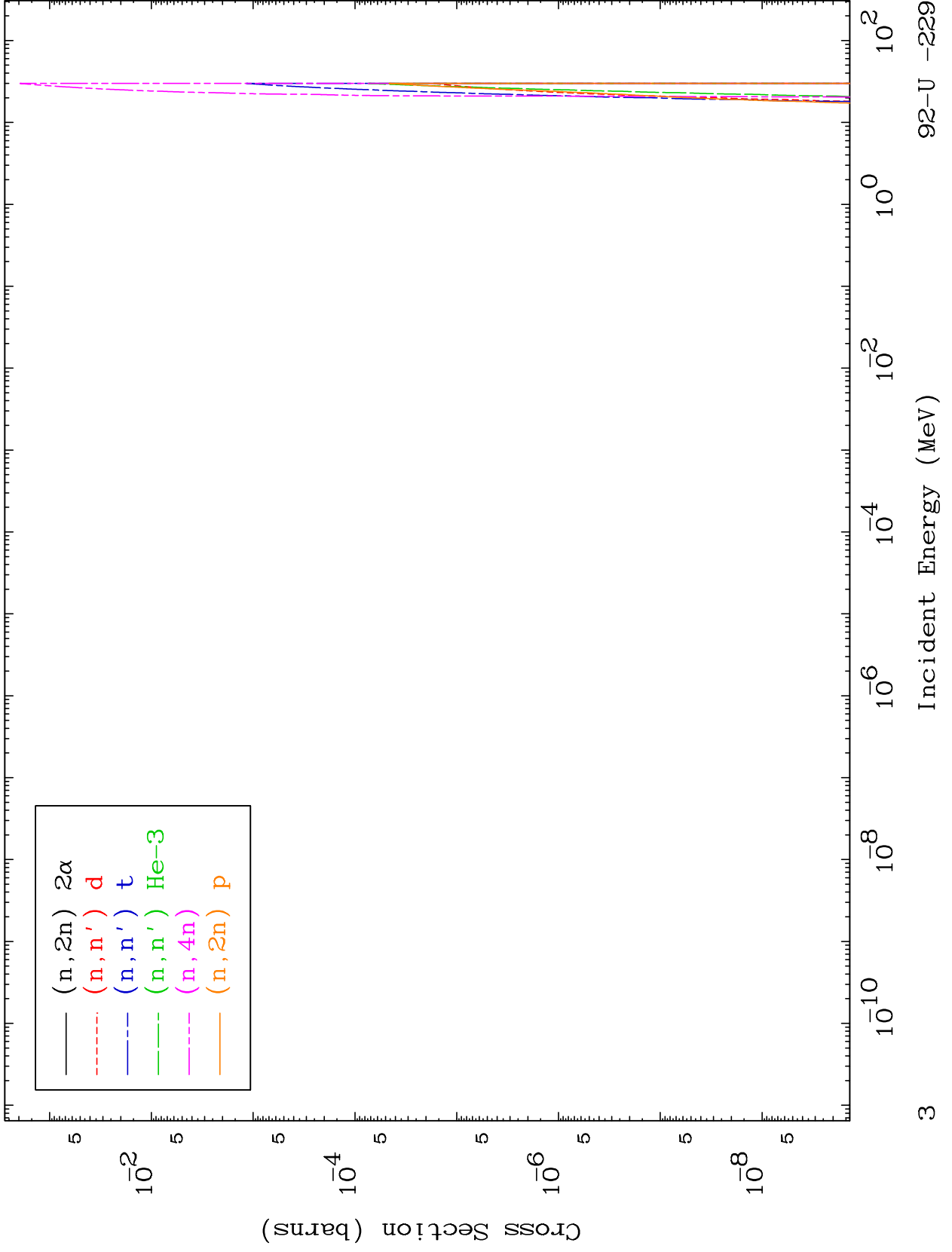
92-U -229

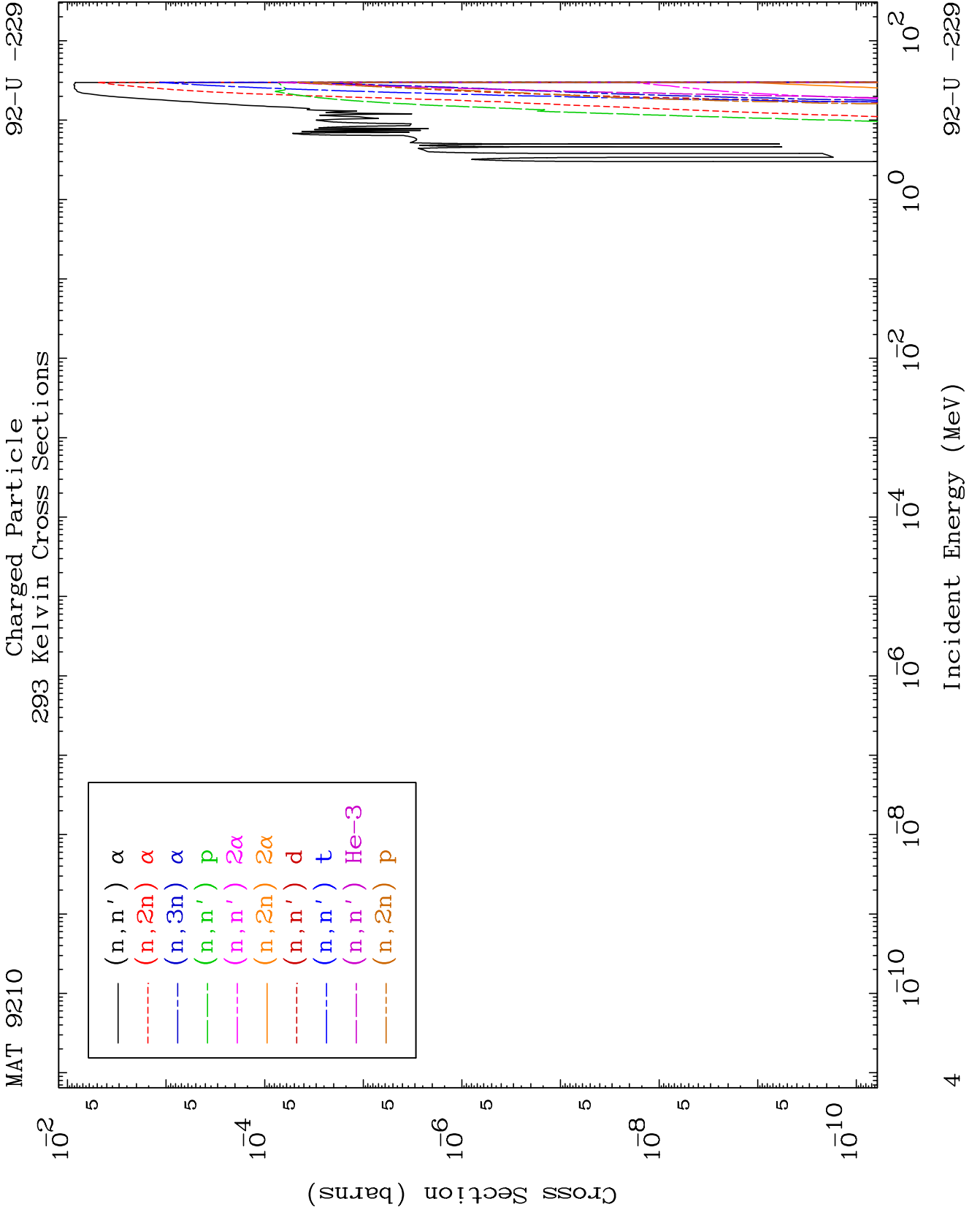


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Neutron Production
293 Kelvin Cross Sections

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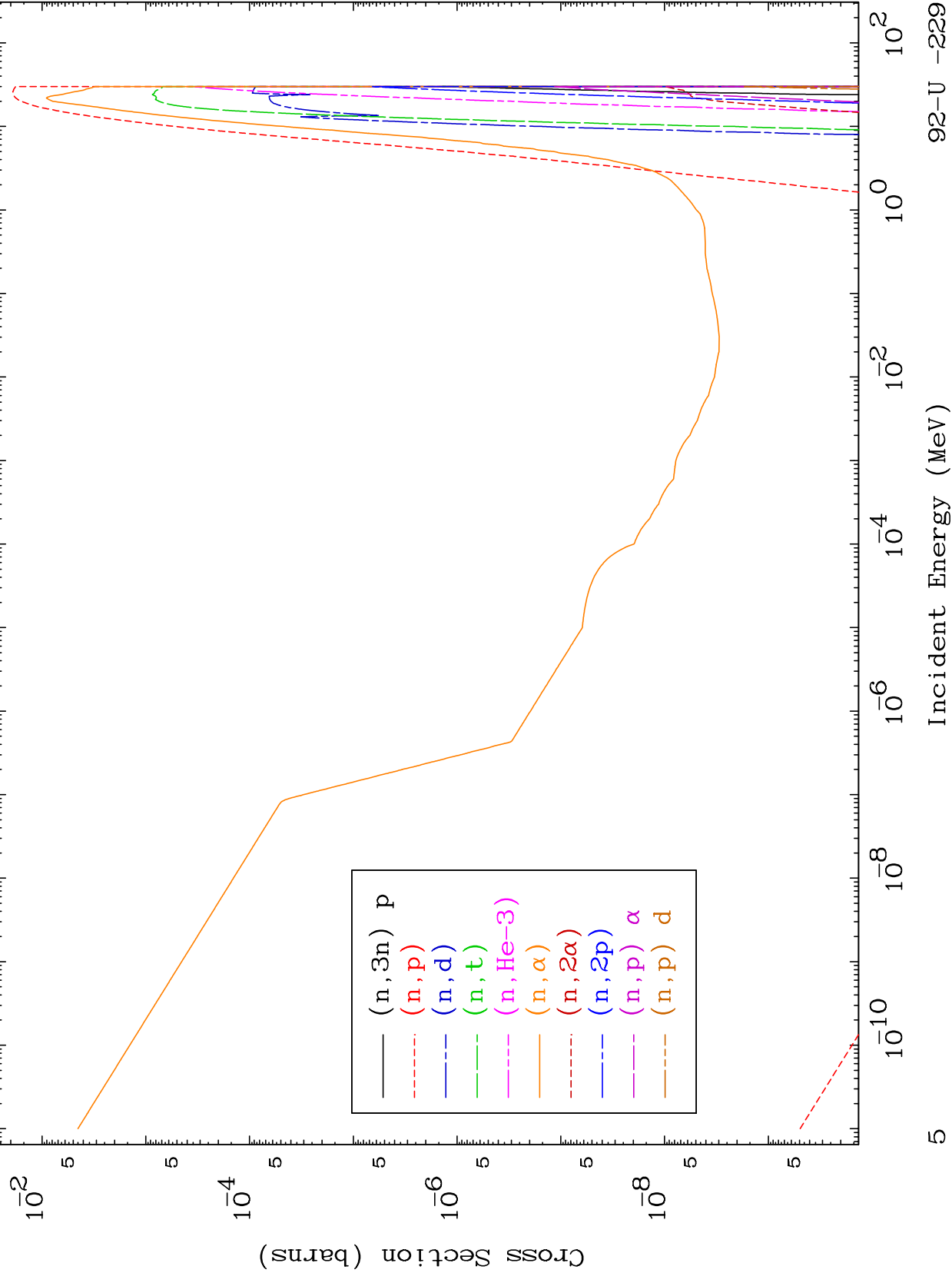




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Charged Particle
293 Kelvin Cross Sections

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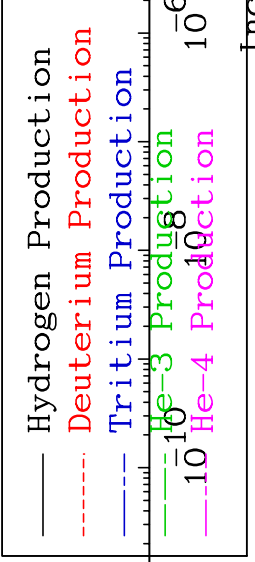
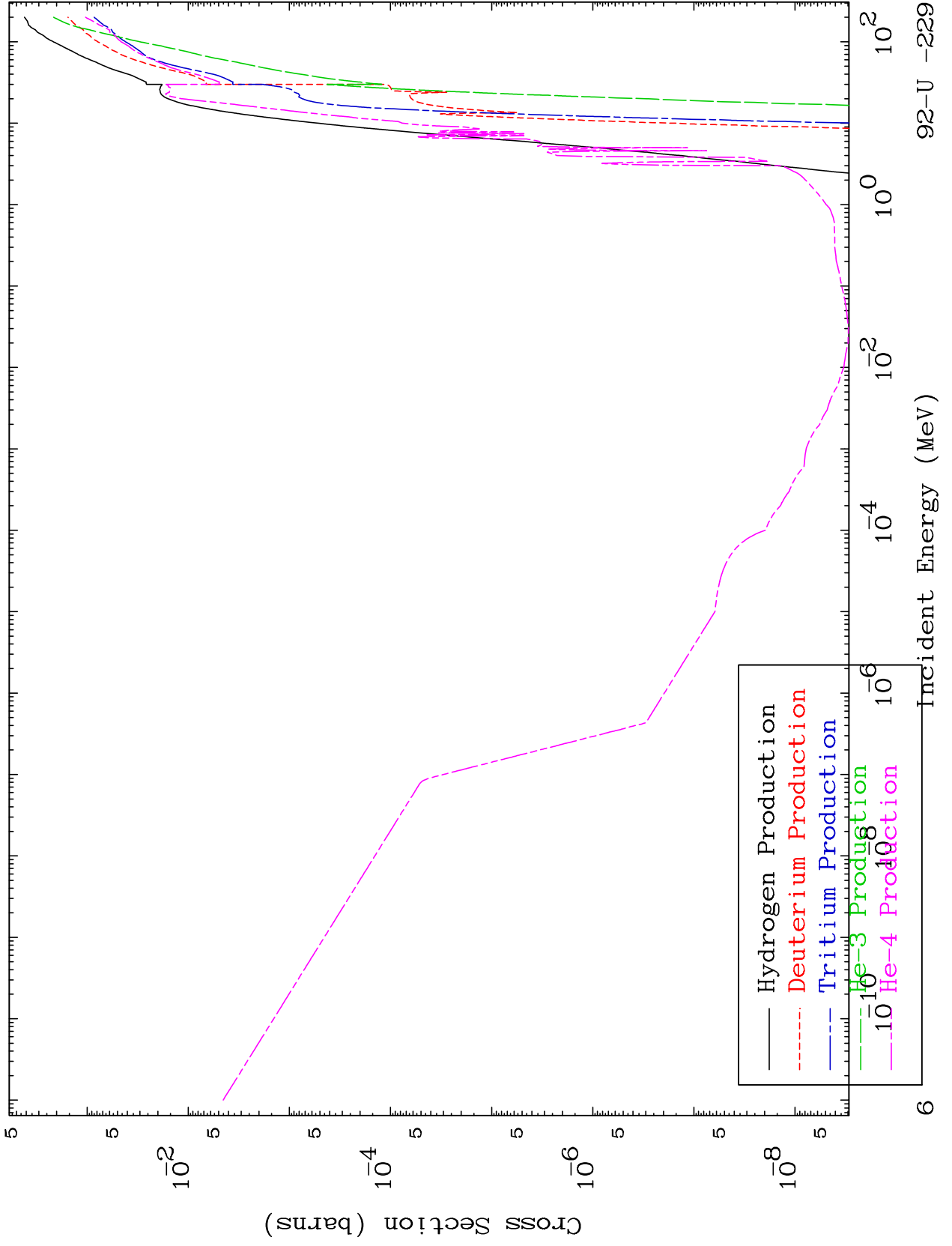


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Particle Production
293 Kelvin Cross Sections

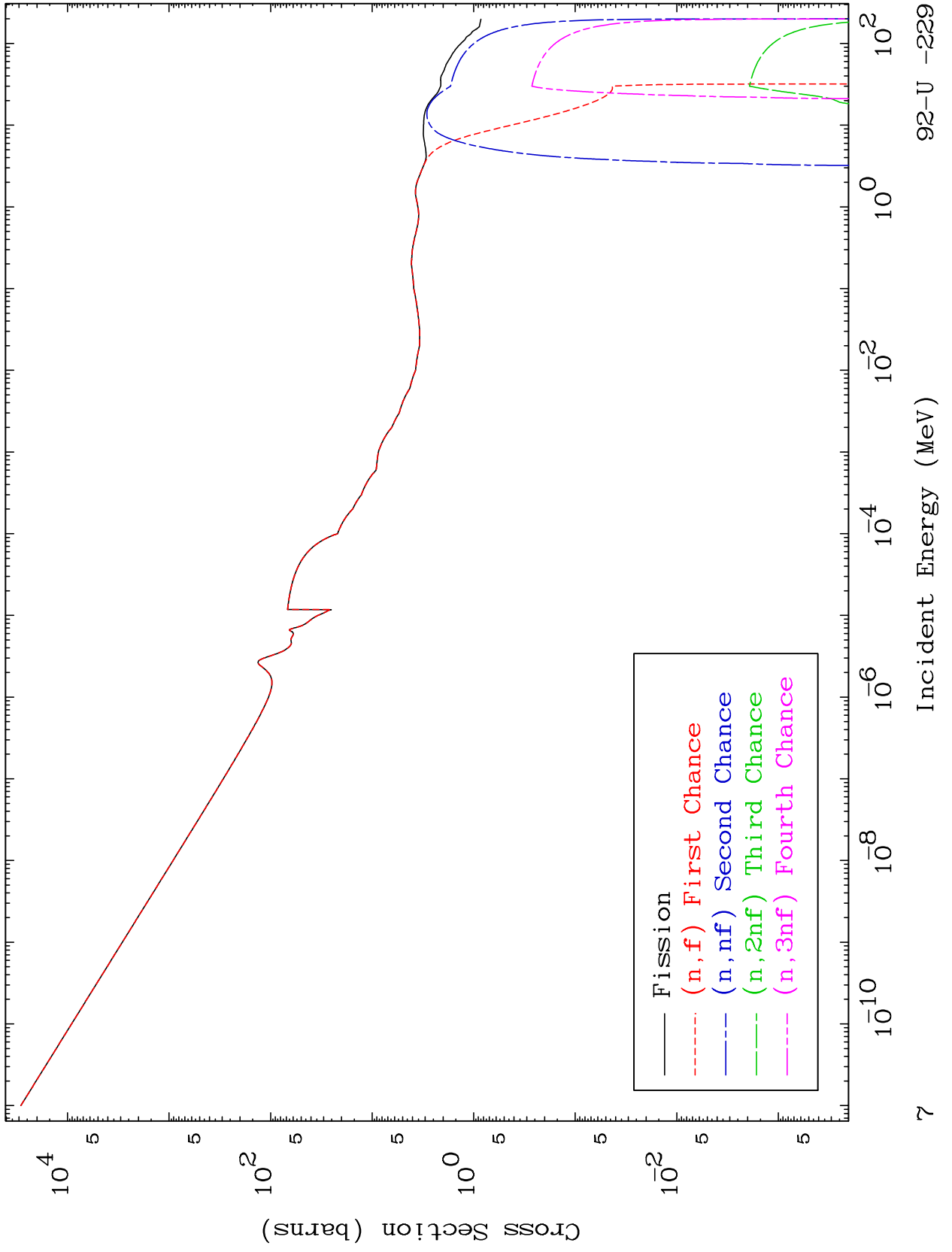
92-U -229



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Fission
293 Kelvin Cross Sections

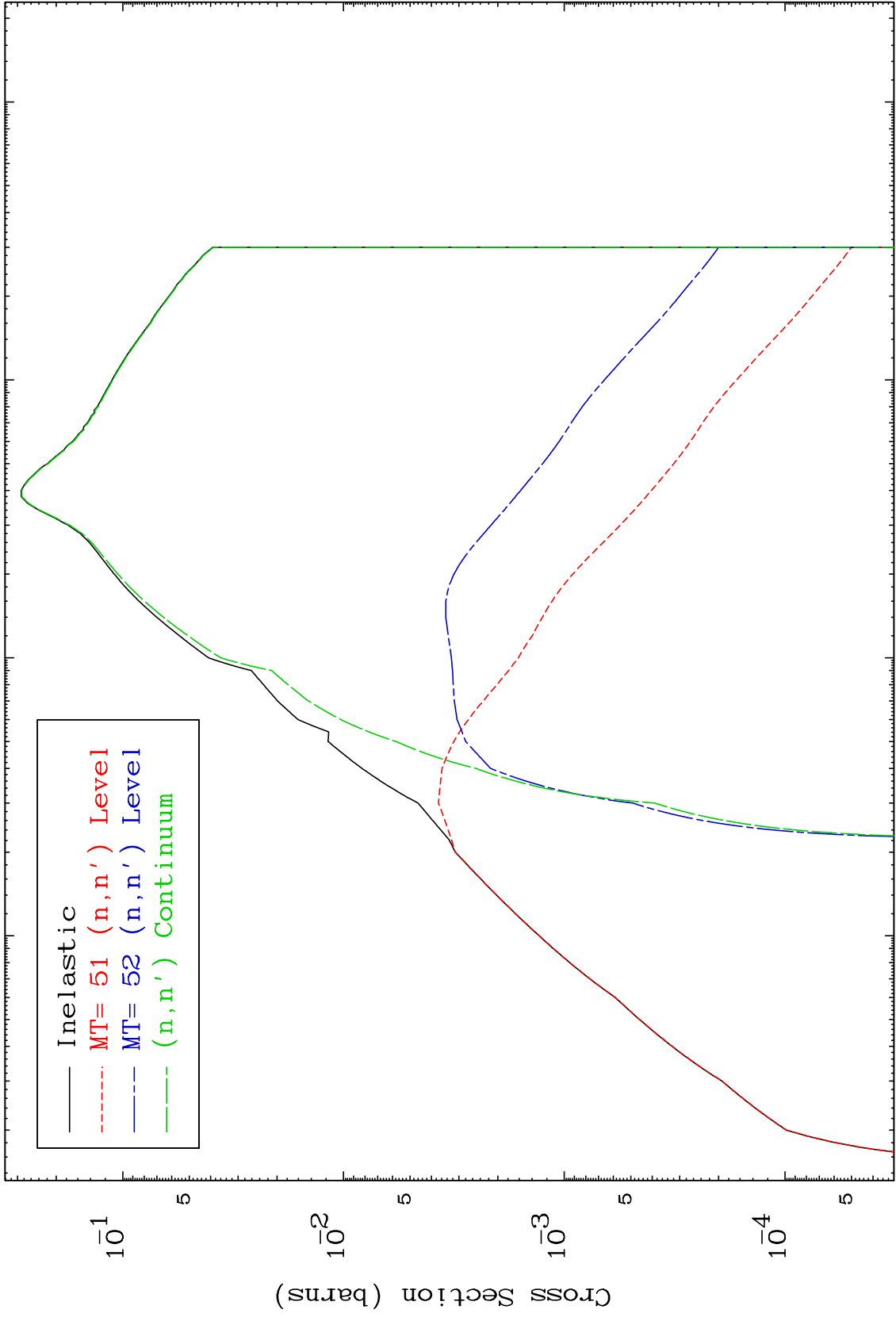
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MAT 9210

293 Kelvin Cross Sections

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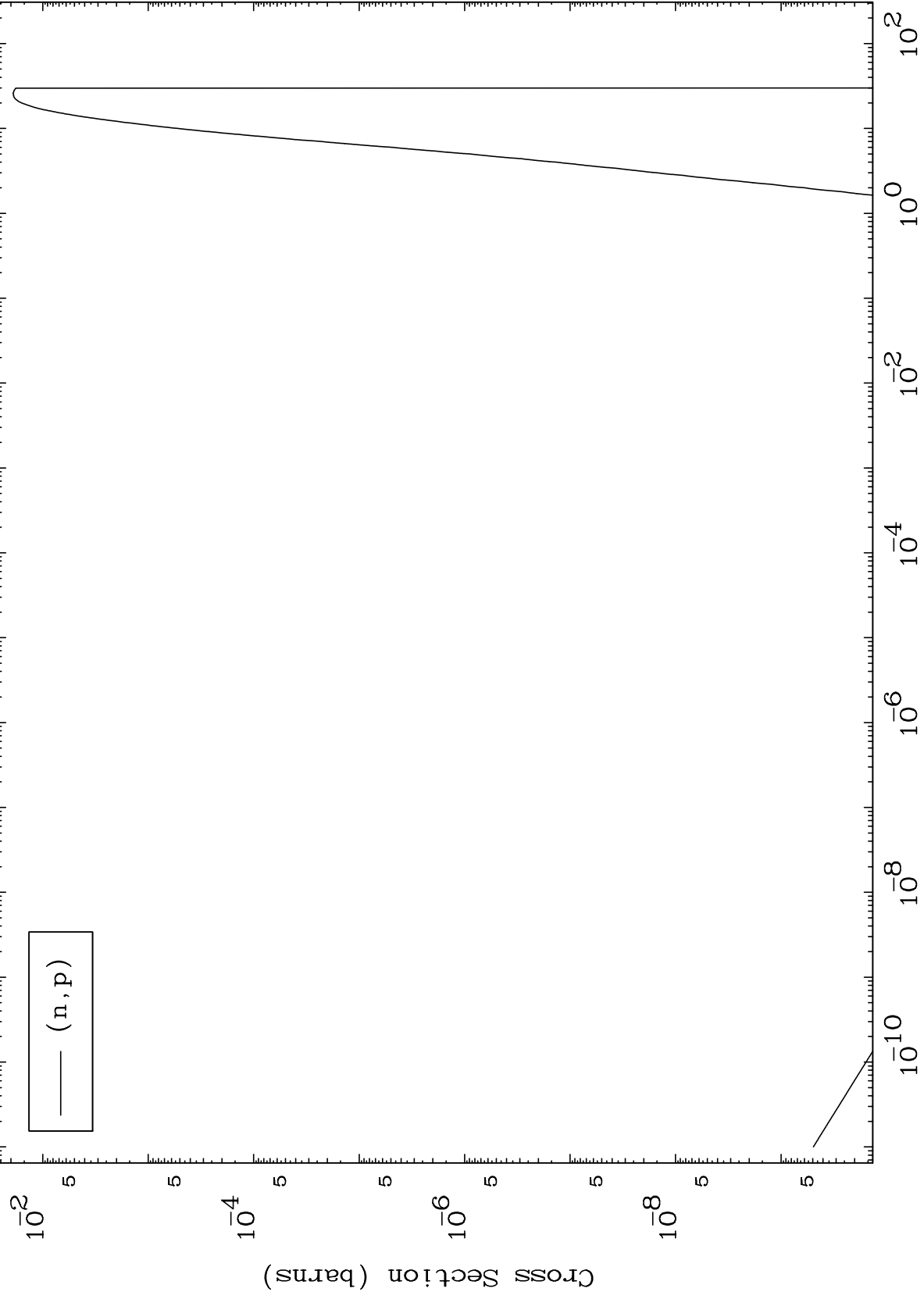
Incident Energy (MeV)

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(n,p) Levels
293 Kelvin Cross Sections

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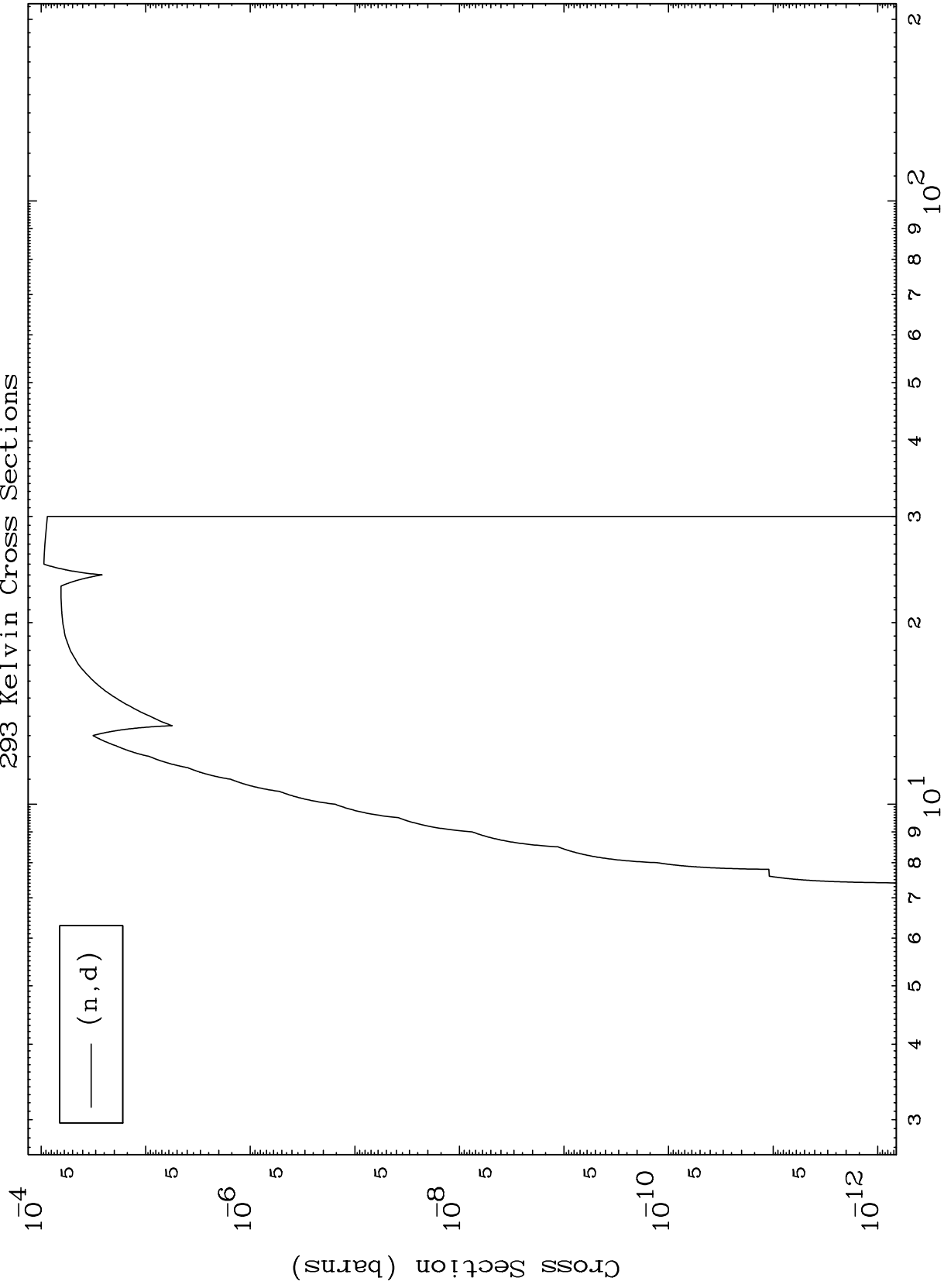
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Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

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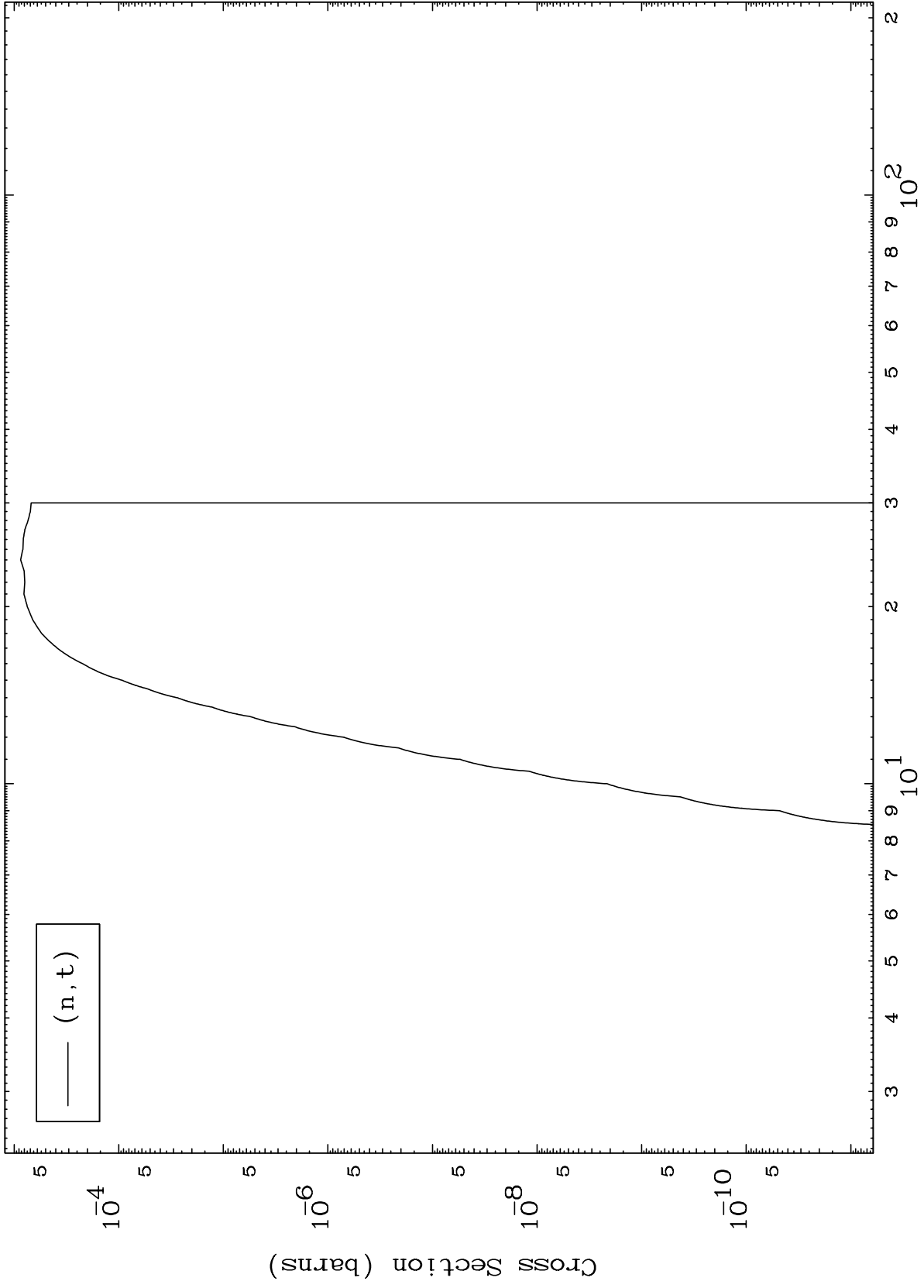
Incident Energy (MeV)

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(n,t) Levels
293 Kelvin Cross Sections

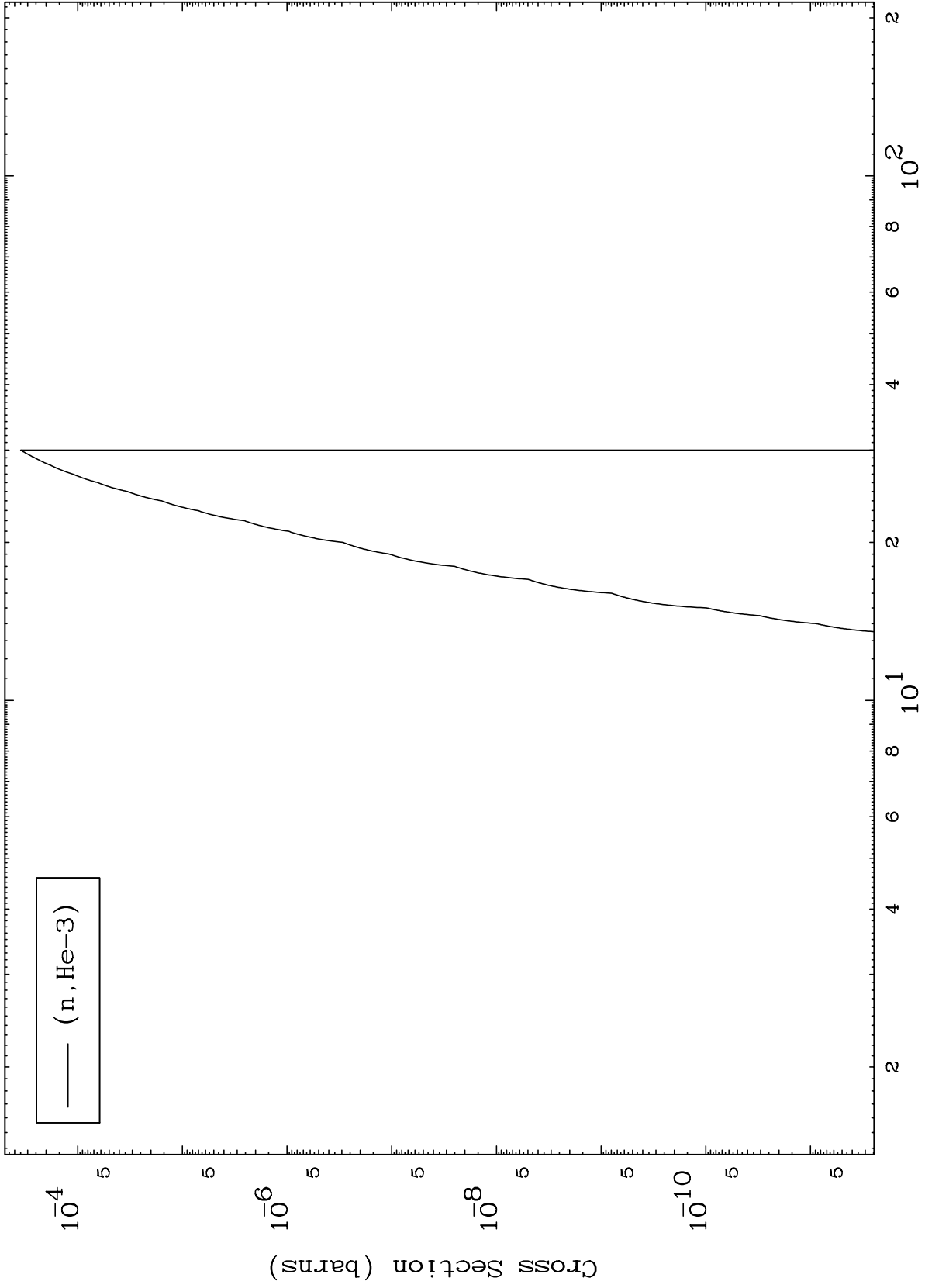
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(n,He3) Levels
293 Kelvin Cross Sections

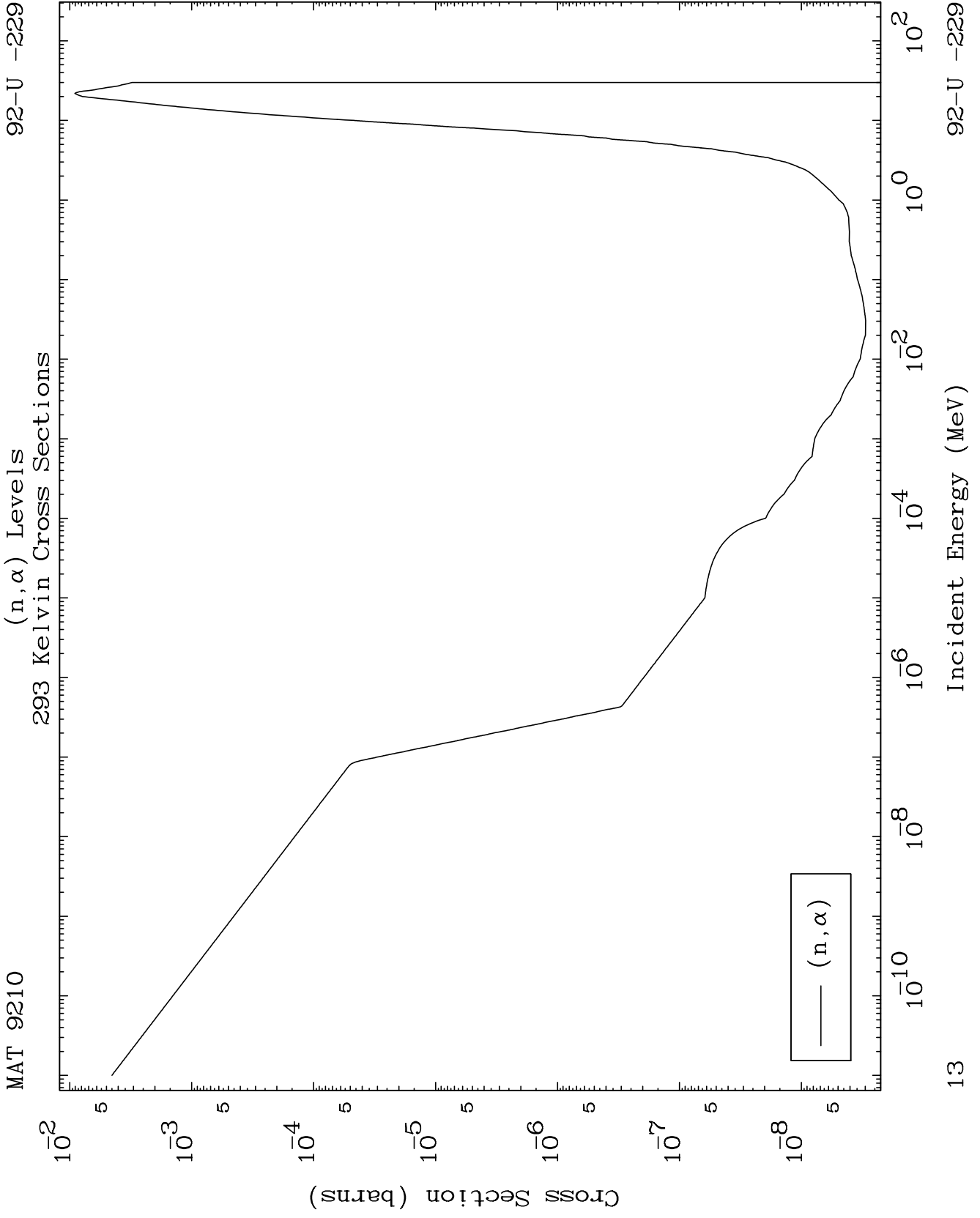
92-U -229

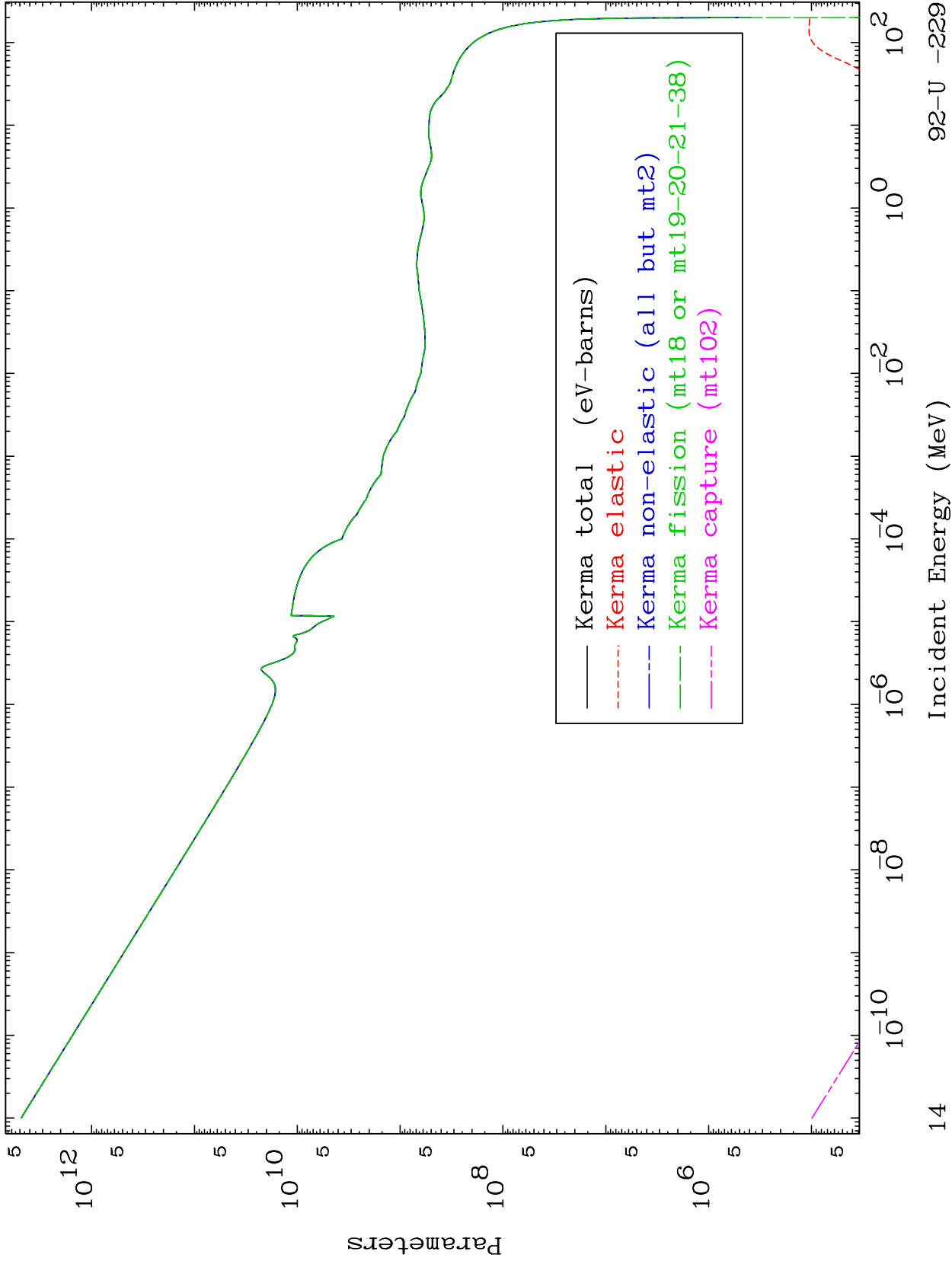


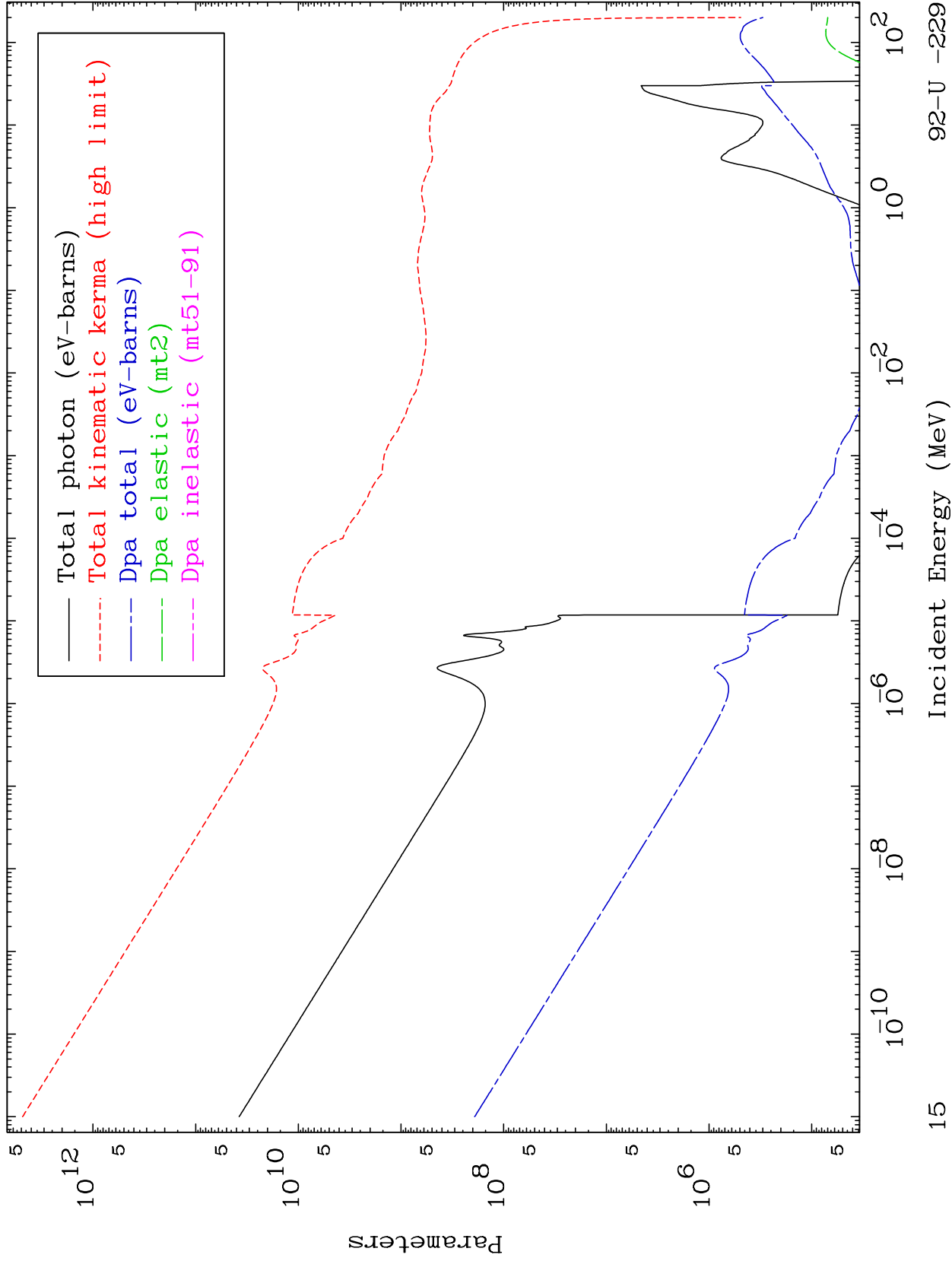
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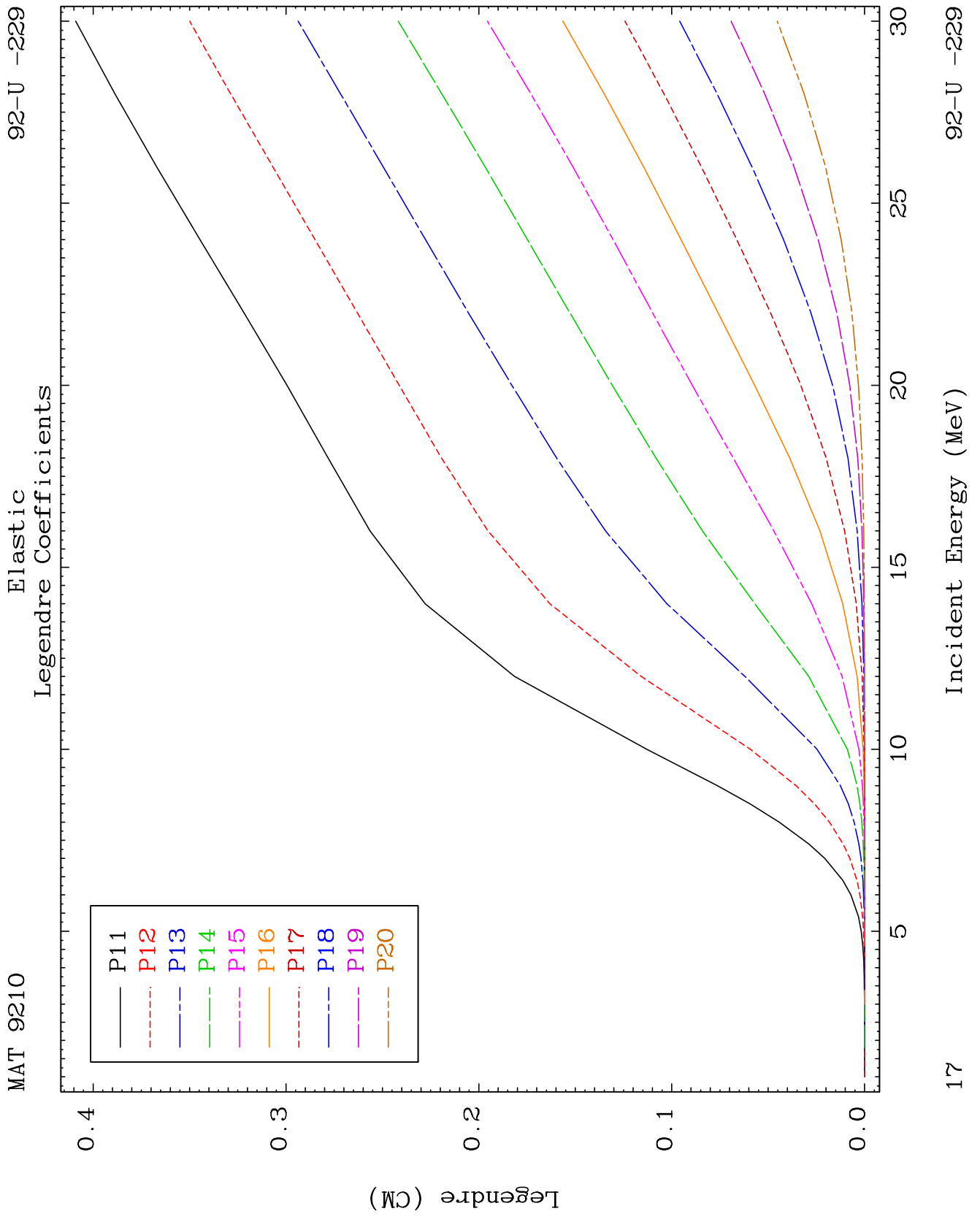
Incident Energy (MeV)

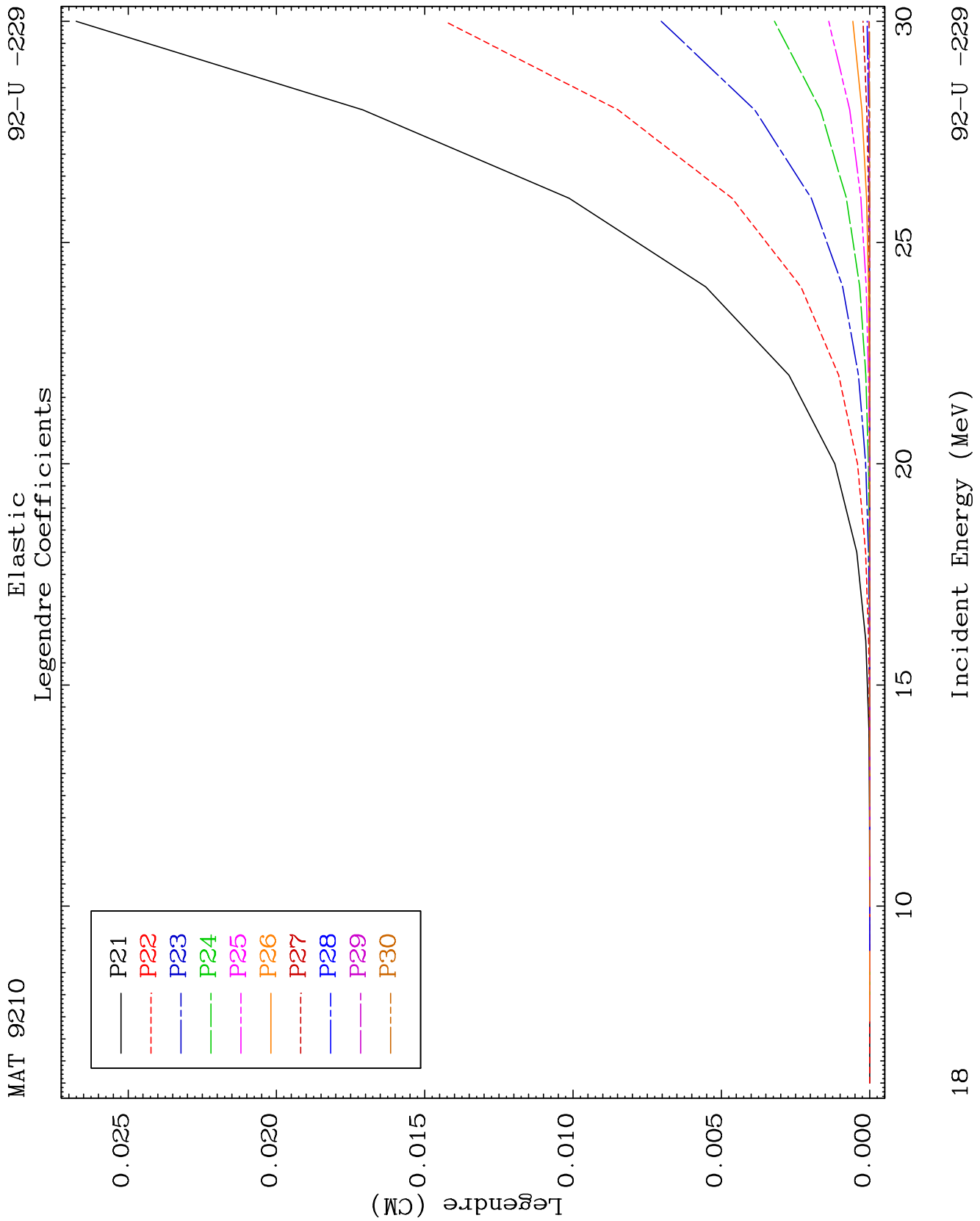
92-U -229







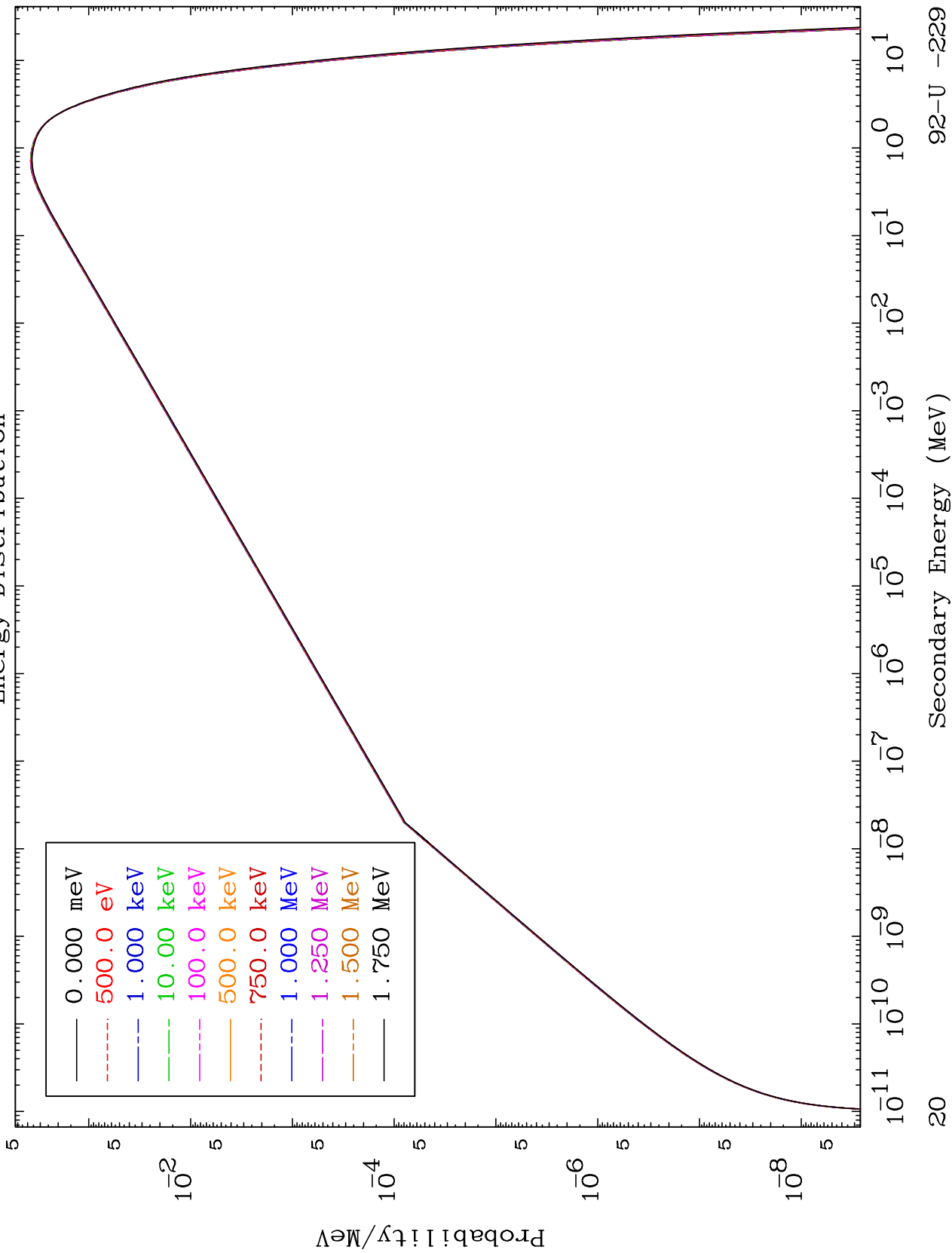




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Fission Energy Distribution

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92-U -229

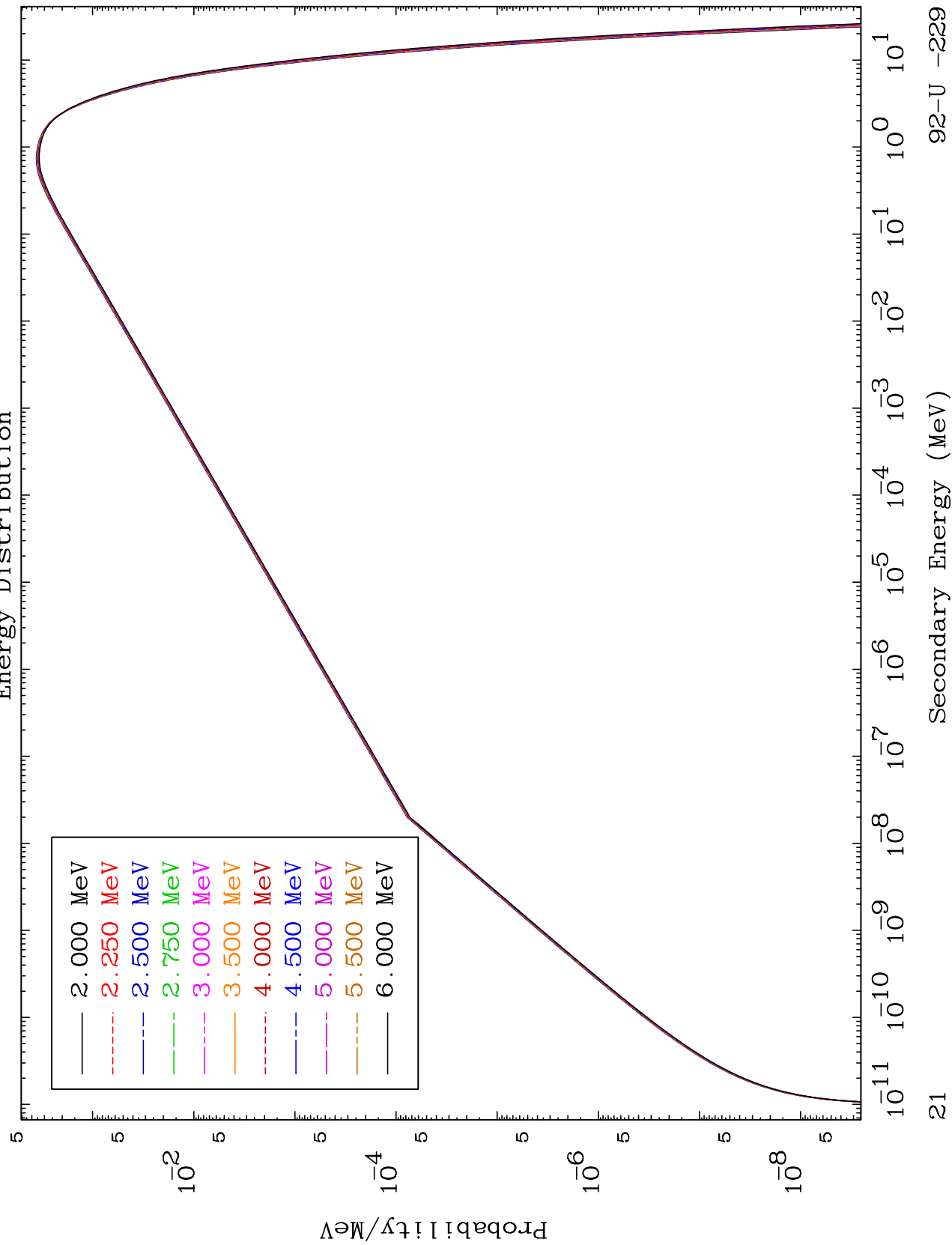
Secondary Energy (MeV)

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Fission Energy Distribution

92-U -229

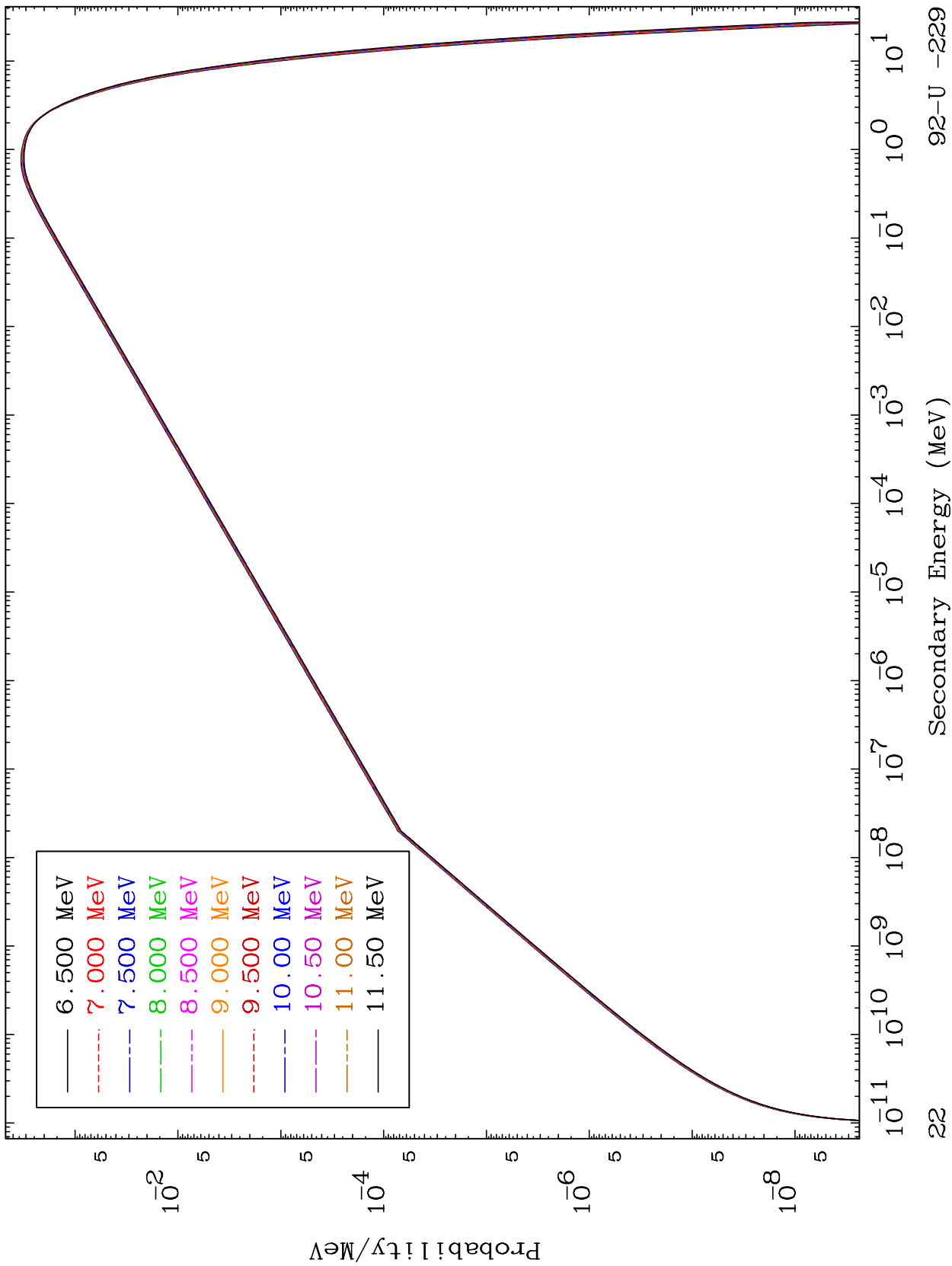


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Fission Energy Distribution

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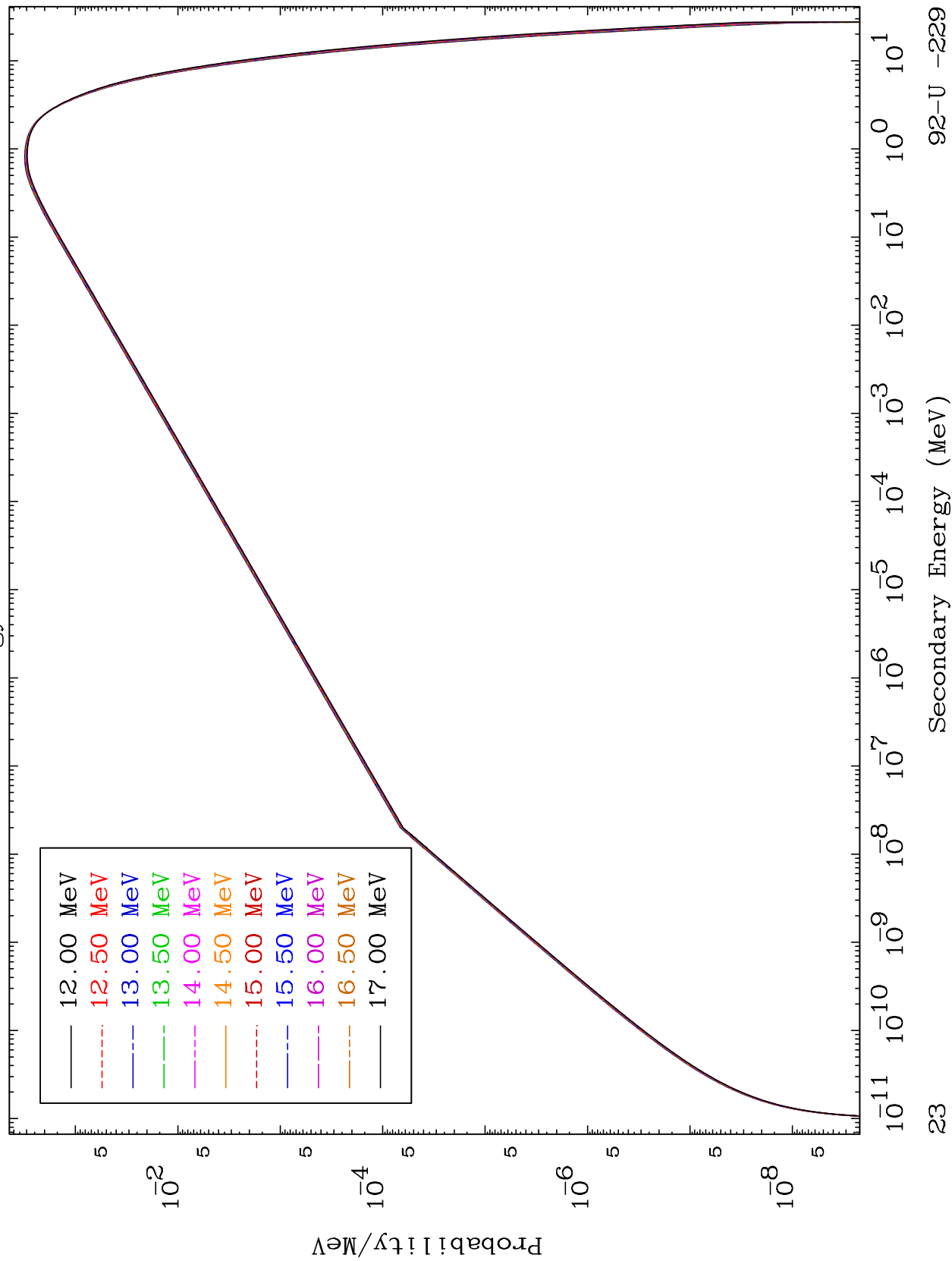
Secondary Energy (MeV)

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Fission Energy Distribution

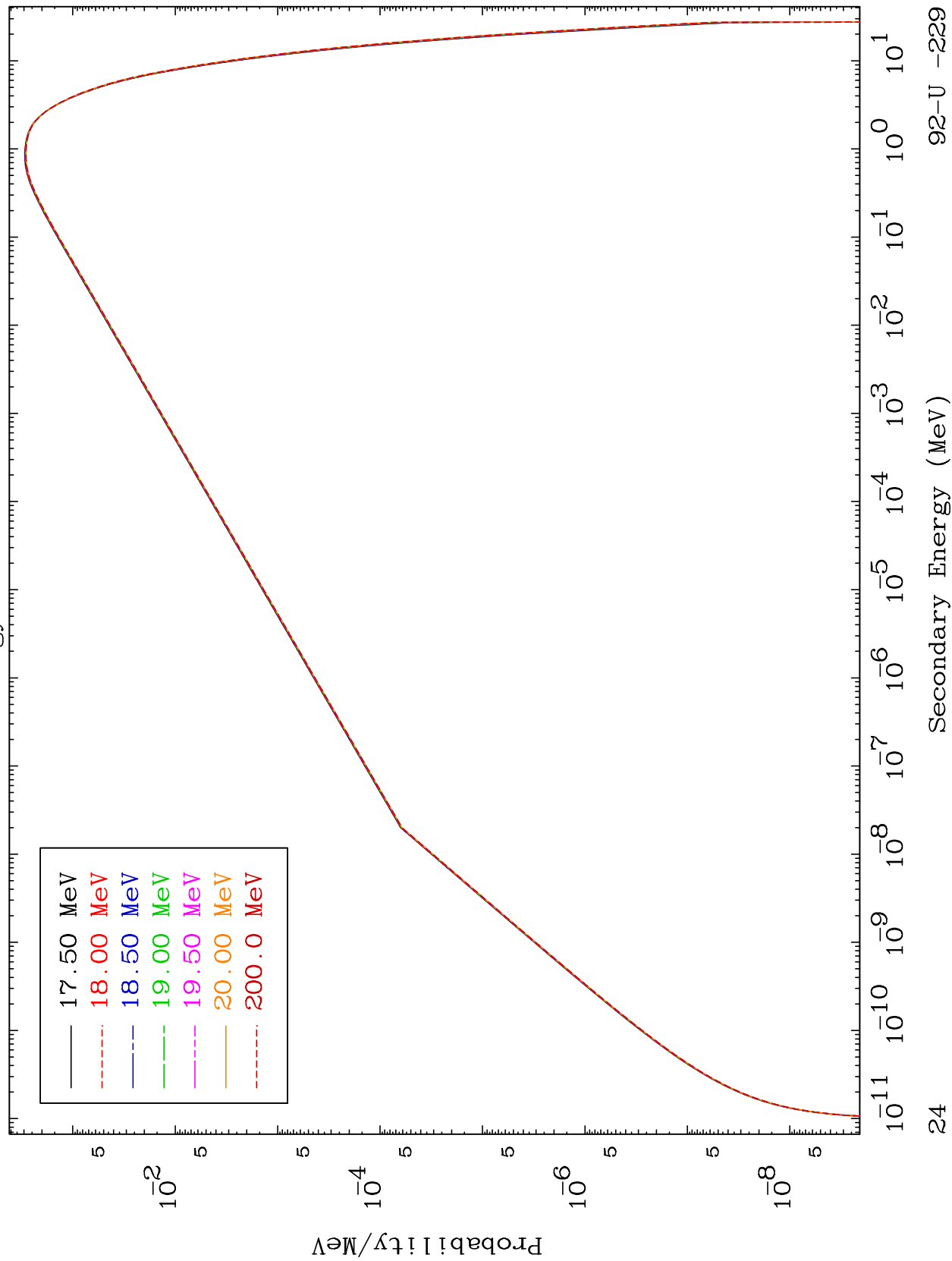
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Fission Energy Distribution

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Secondary Energy (MeV)

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