

Program EVALPLOT
(Version 2018-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

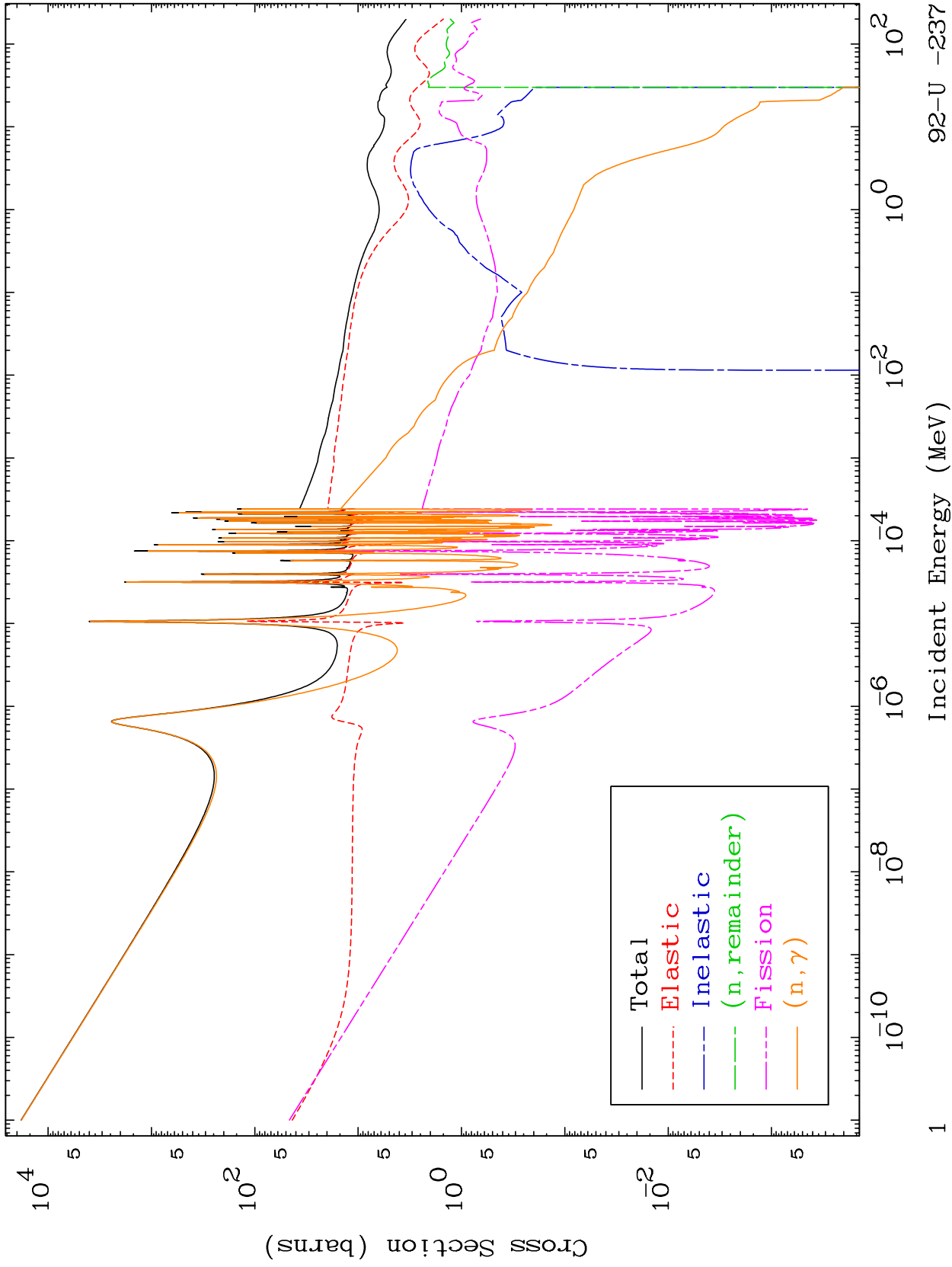
Press Mouse Button to Start

MAT 9234

Major

293 Kelvin Cross Sections

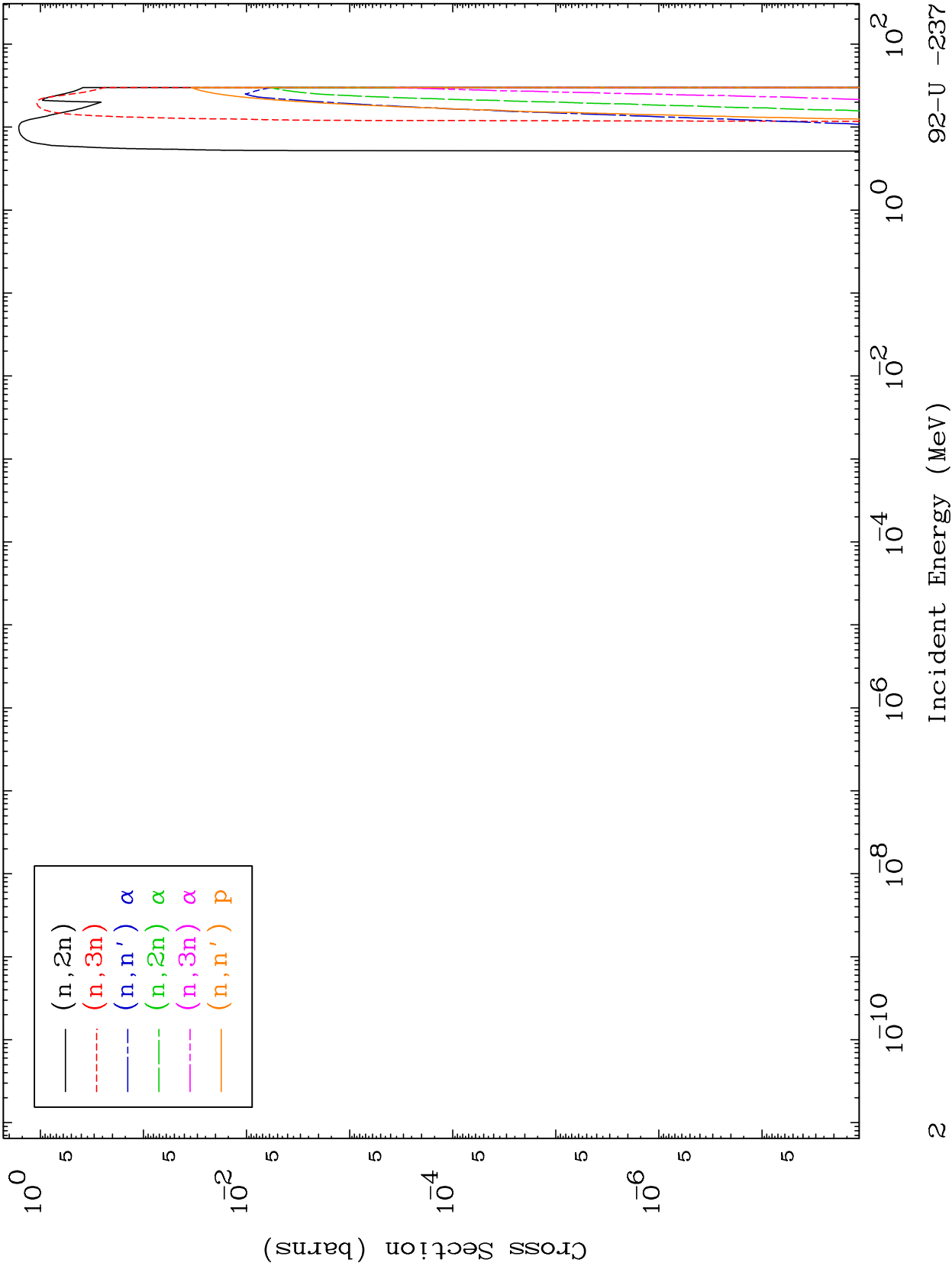
92-U -237



MAT 9234

Neutron Production
293 Kelvin Cross Sections

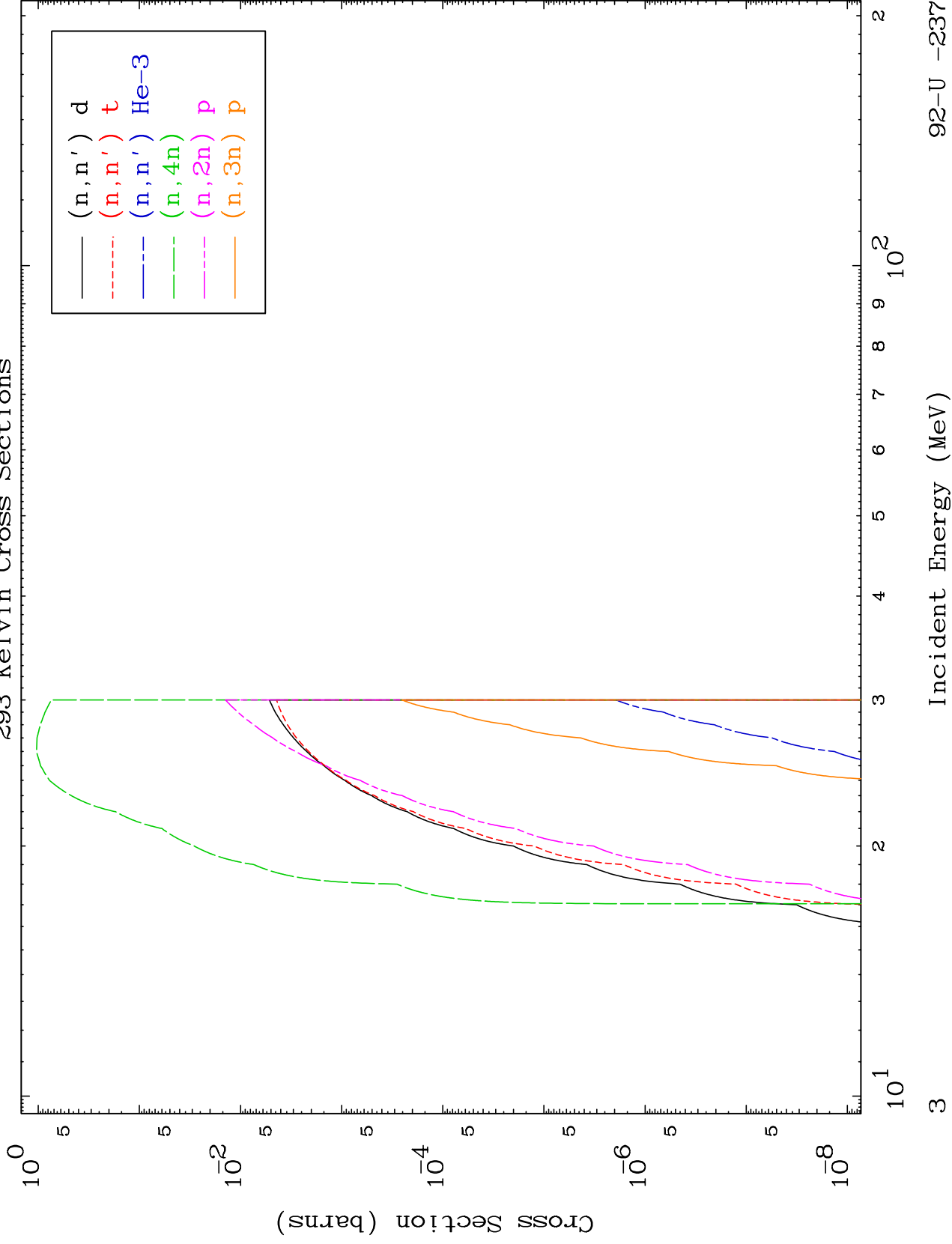
92-U -237



MAT 9234

Neutron Production
293 Kelvin Cross Sections

92-U -237

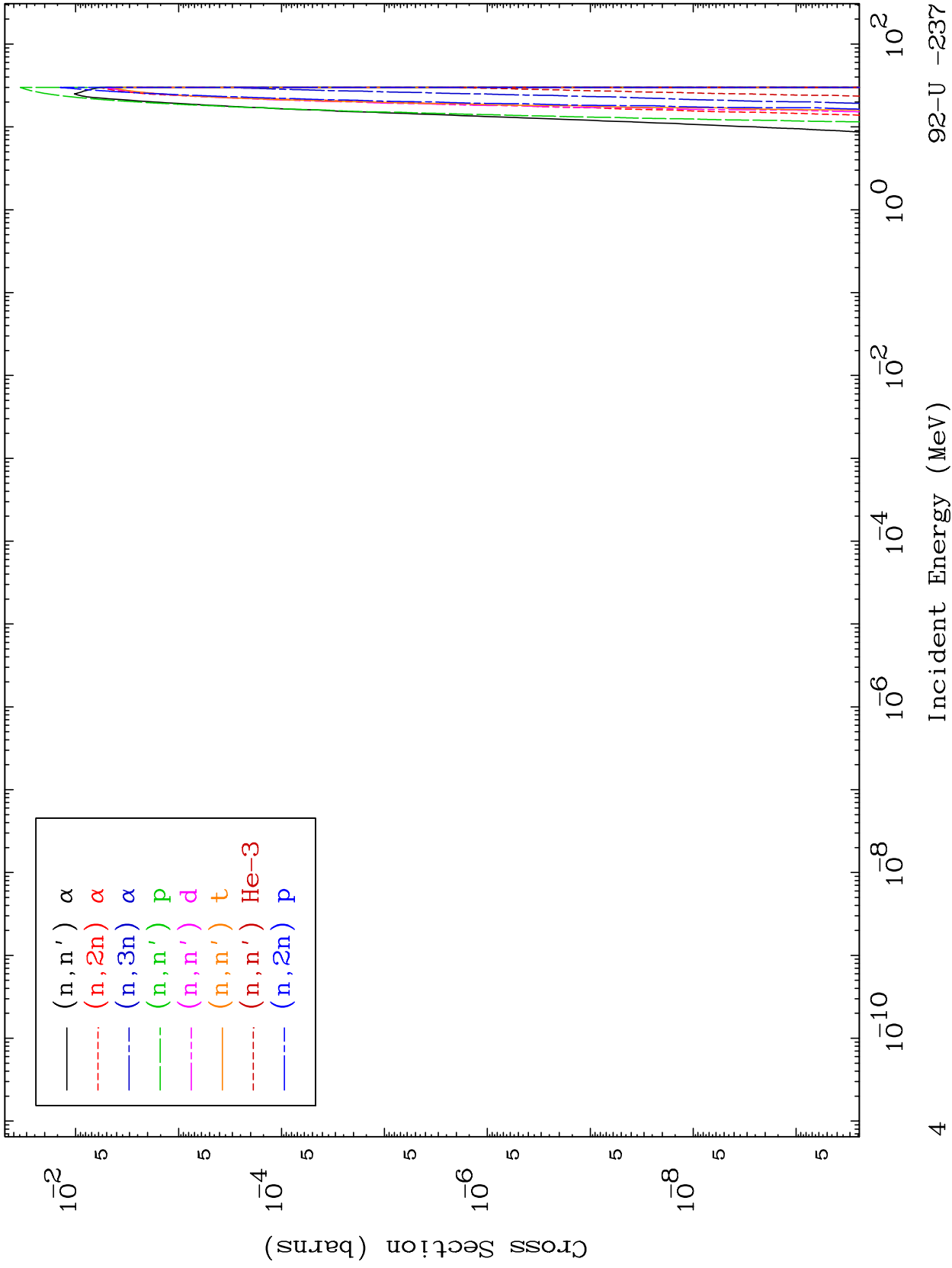


92-U -237

MAT 9234

Charged Particle
293 Kelvin Cross Sections

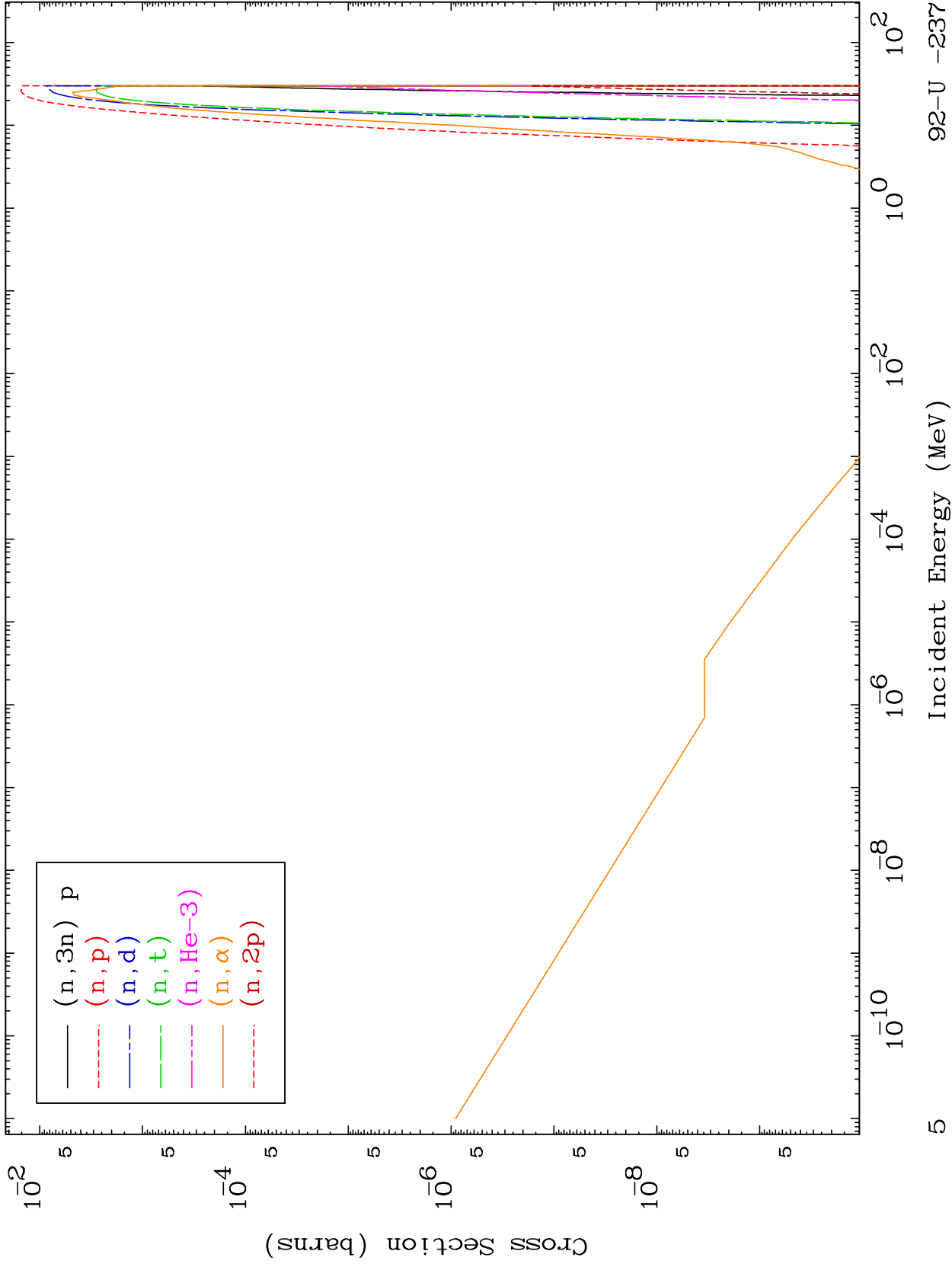
92-U -237



MAT 9234

Charged Particle
293 Kelvin Cross Sections

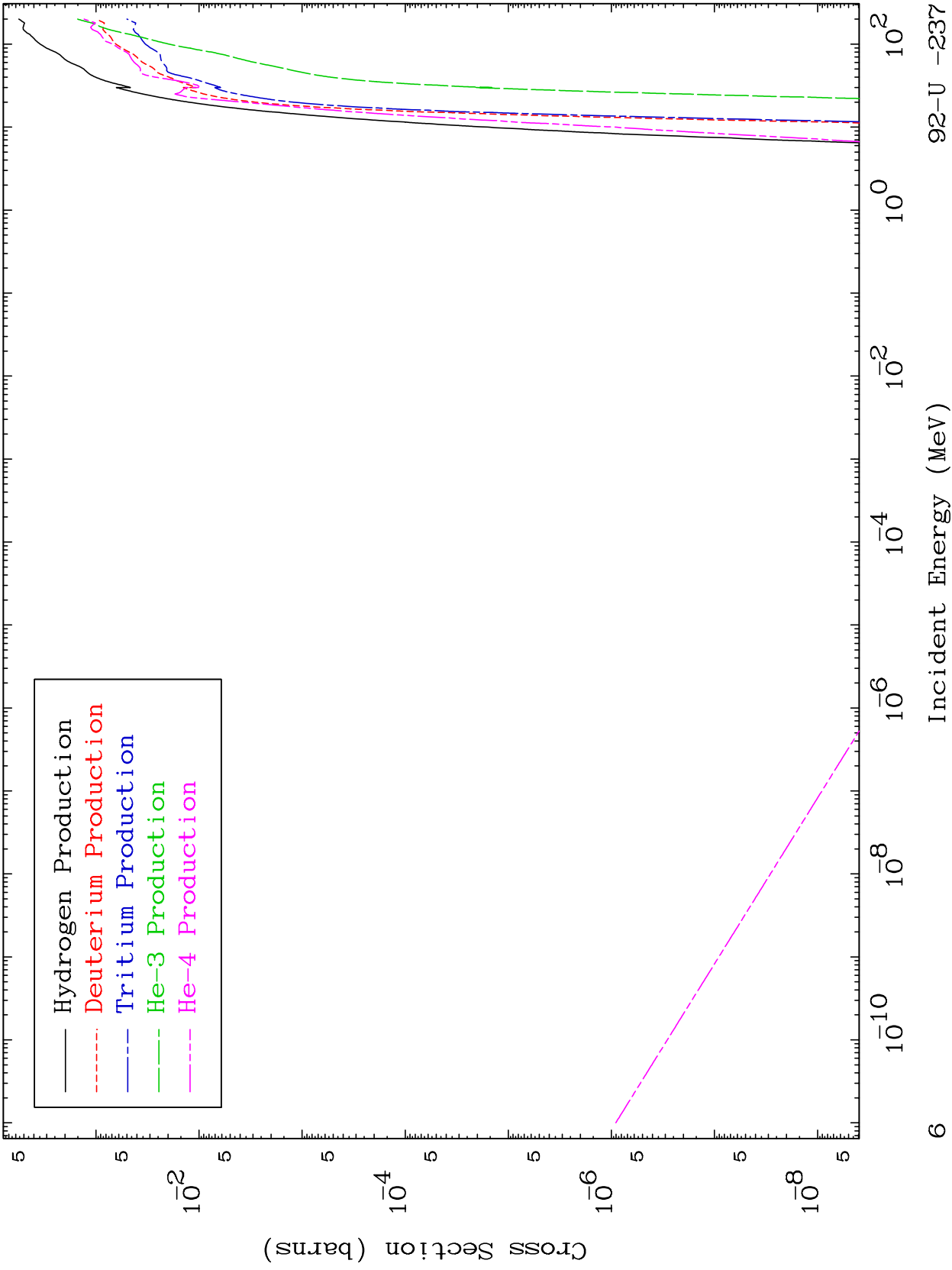
92-U -237



MAT 9234

Particle Production
293 Kelvin Cross Sections

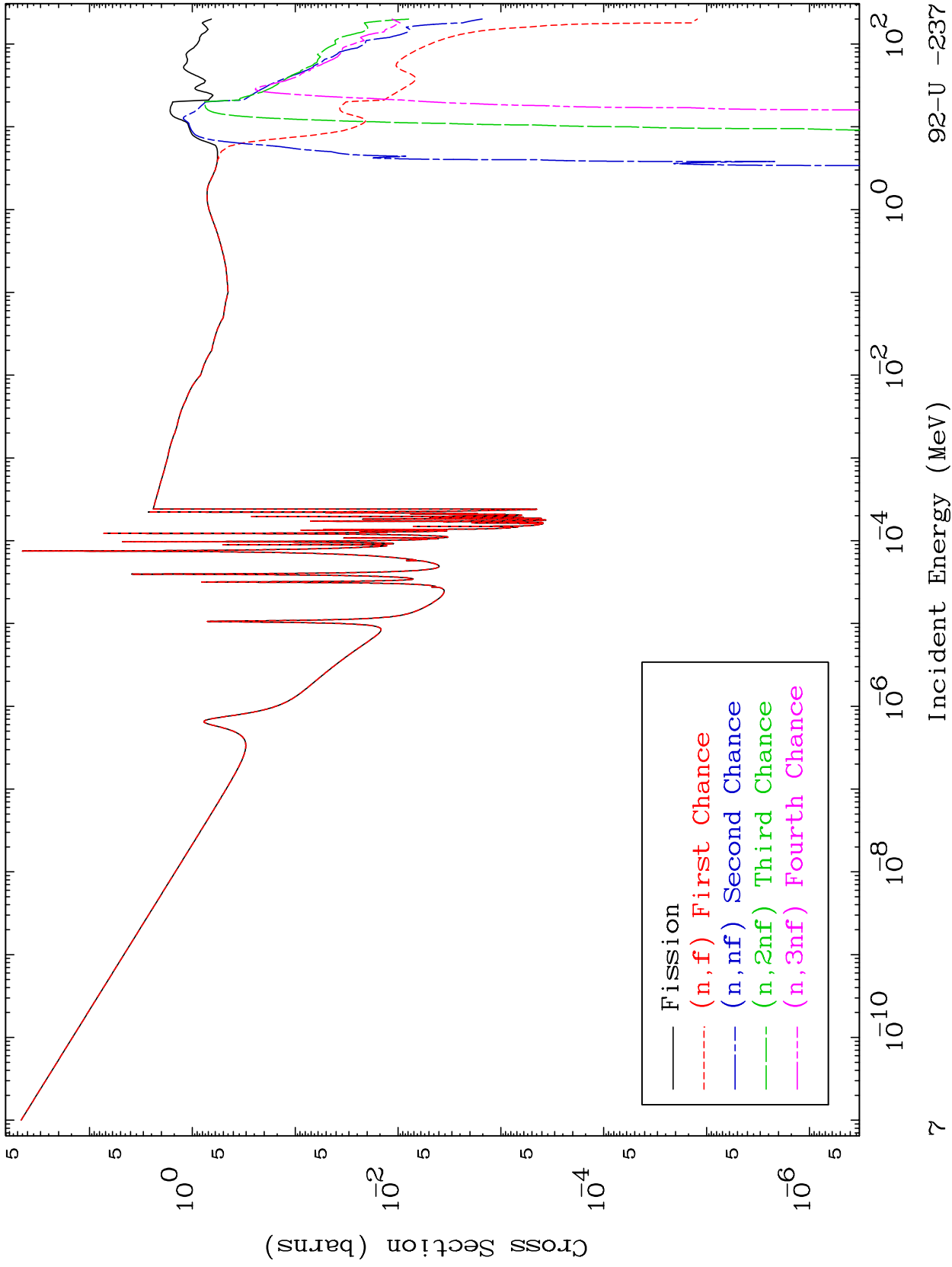
92-U -237



MAT 9234

Fission
293 Kelvin Cross Sections

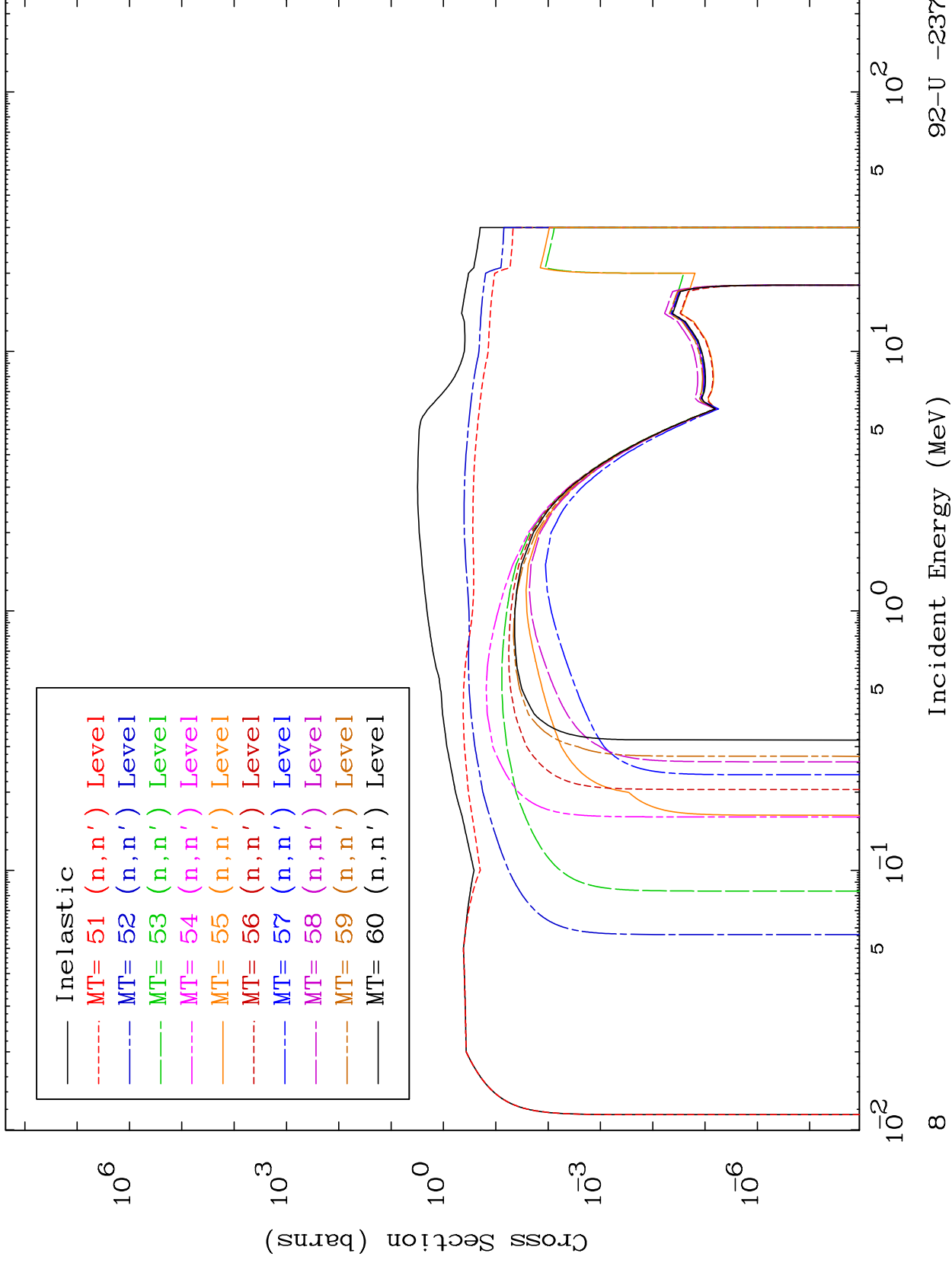
92-U -237



MAT 9234

(n,n') Level
293 Kelvin Cross Sections

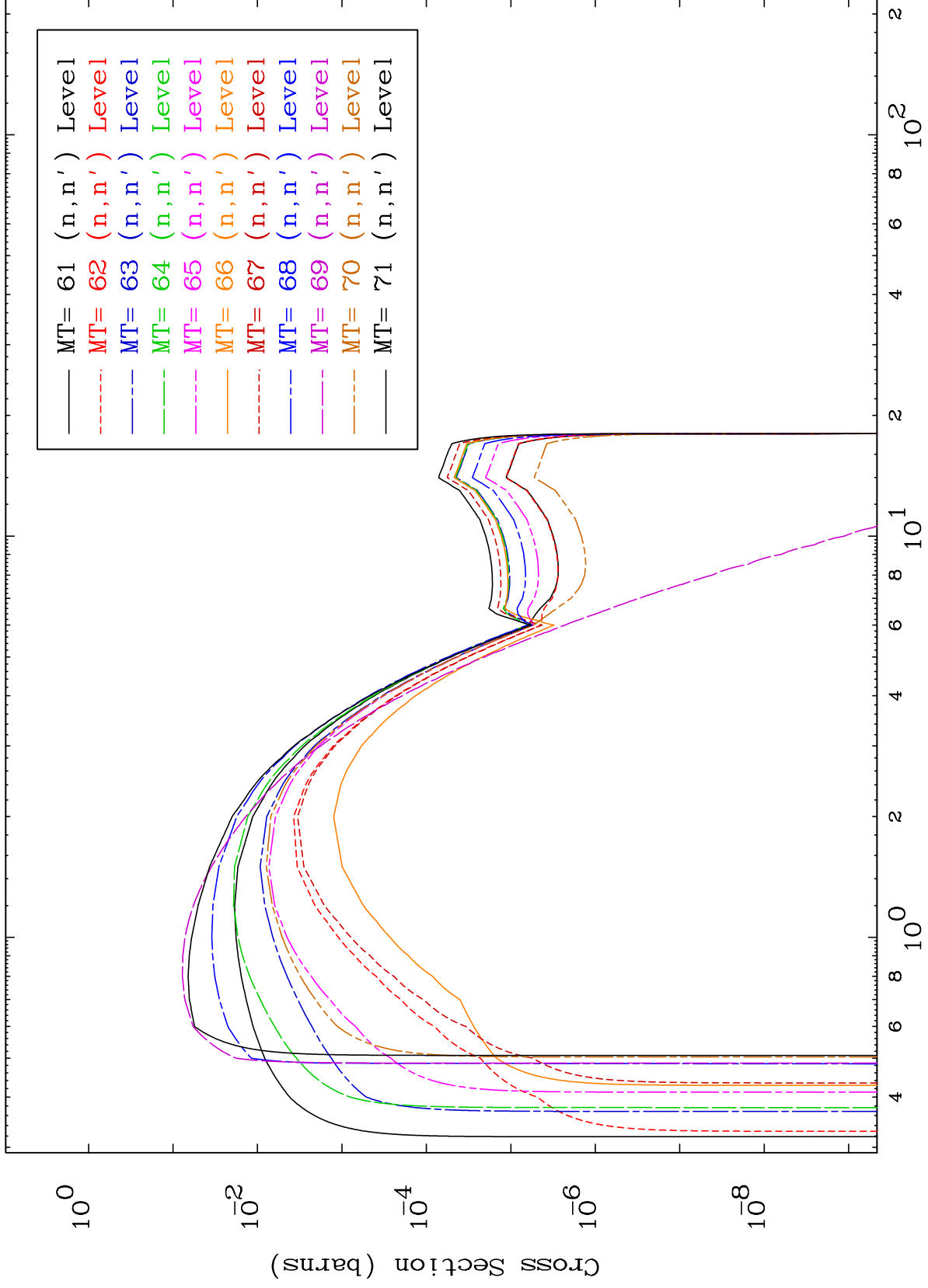
92-U -237



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293 Kelvin Cross Sections

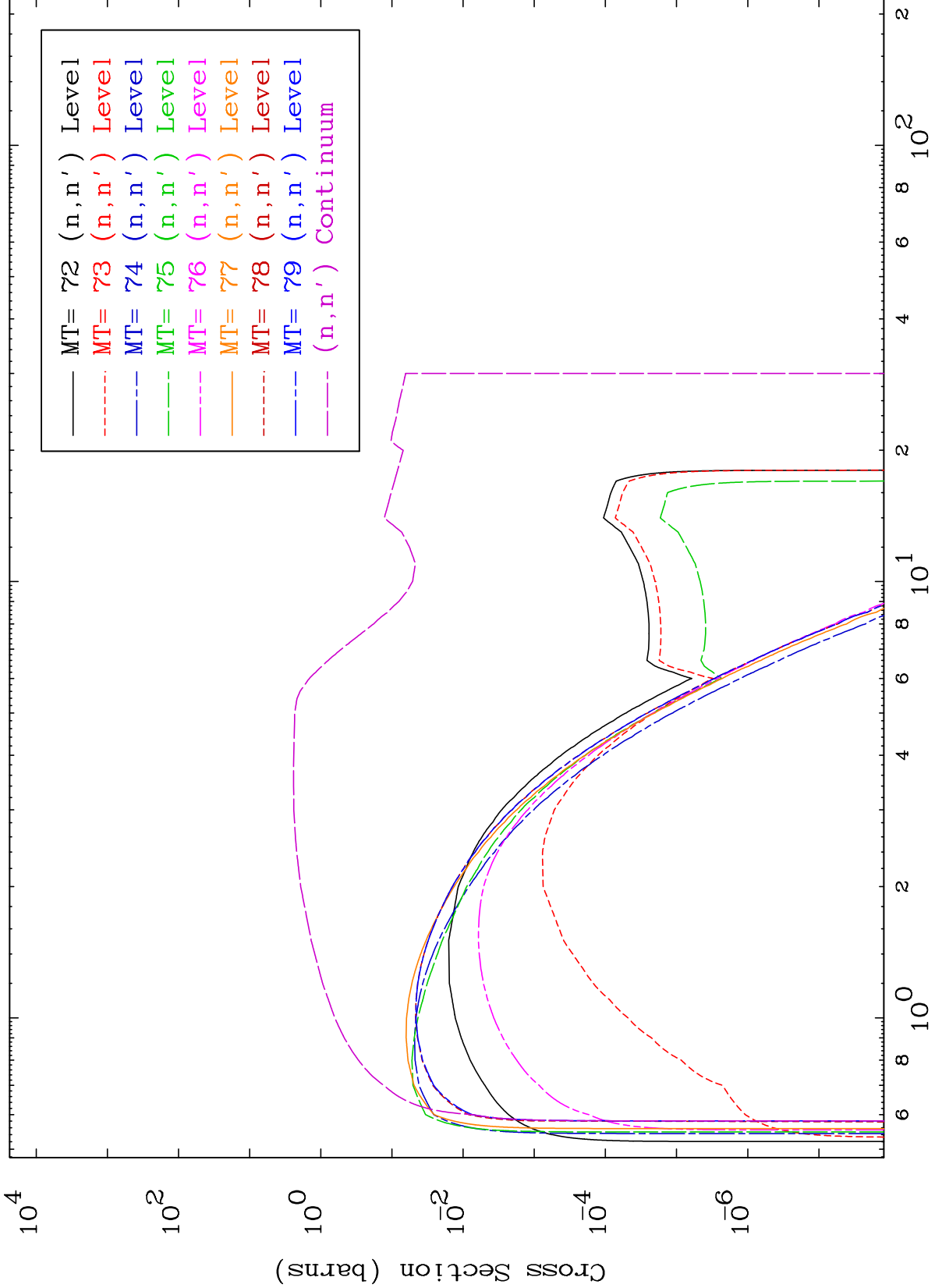


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(n,n') Level

92-U -237

293 Kelvin Cross Sections



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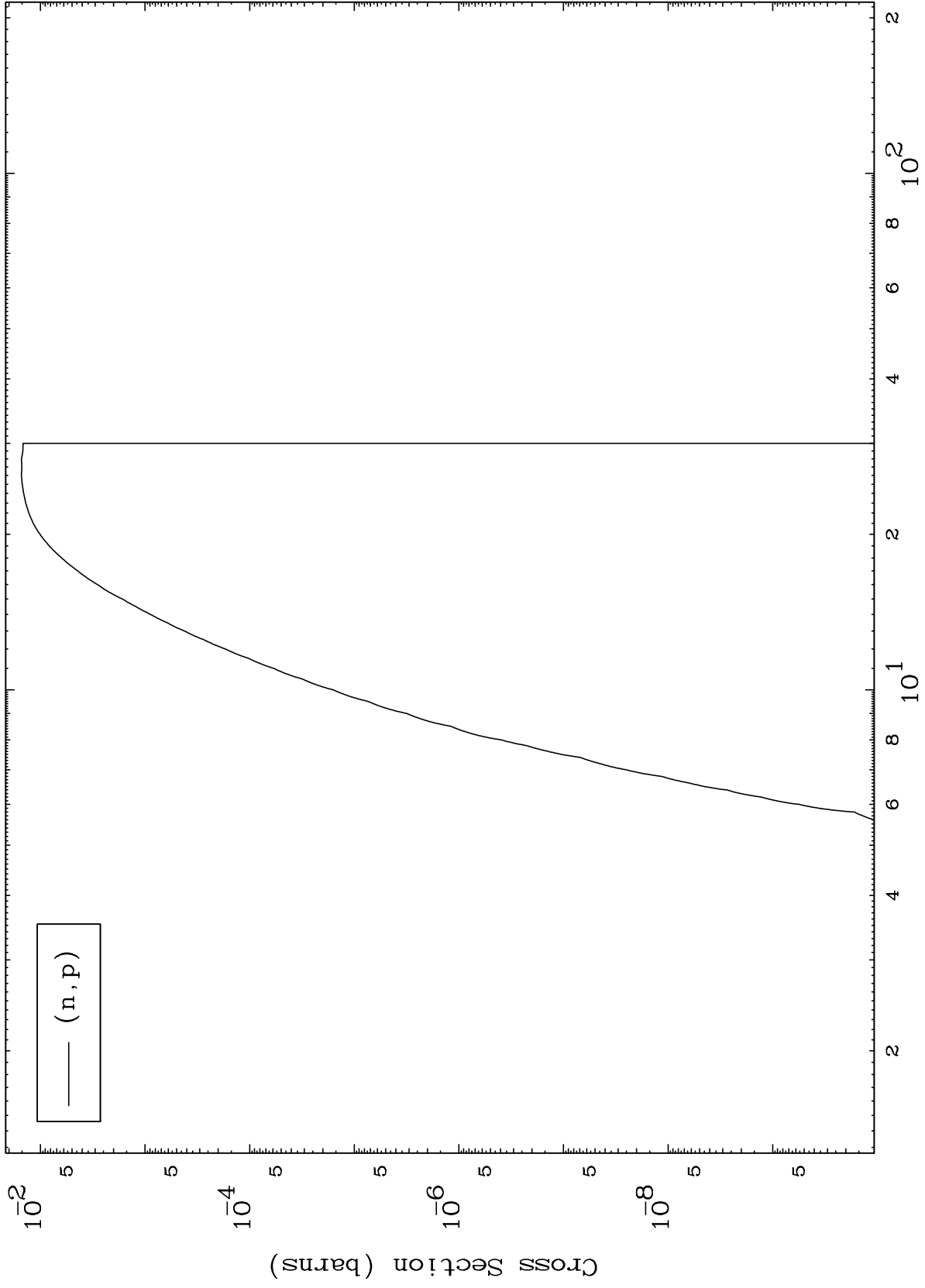
Incident Energy (MeV)

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MAT 9234

(n,p) Levels
293 Kelvin Cross Sections

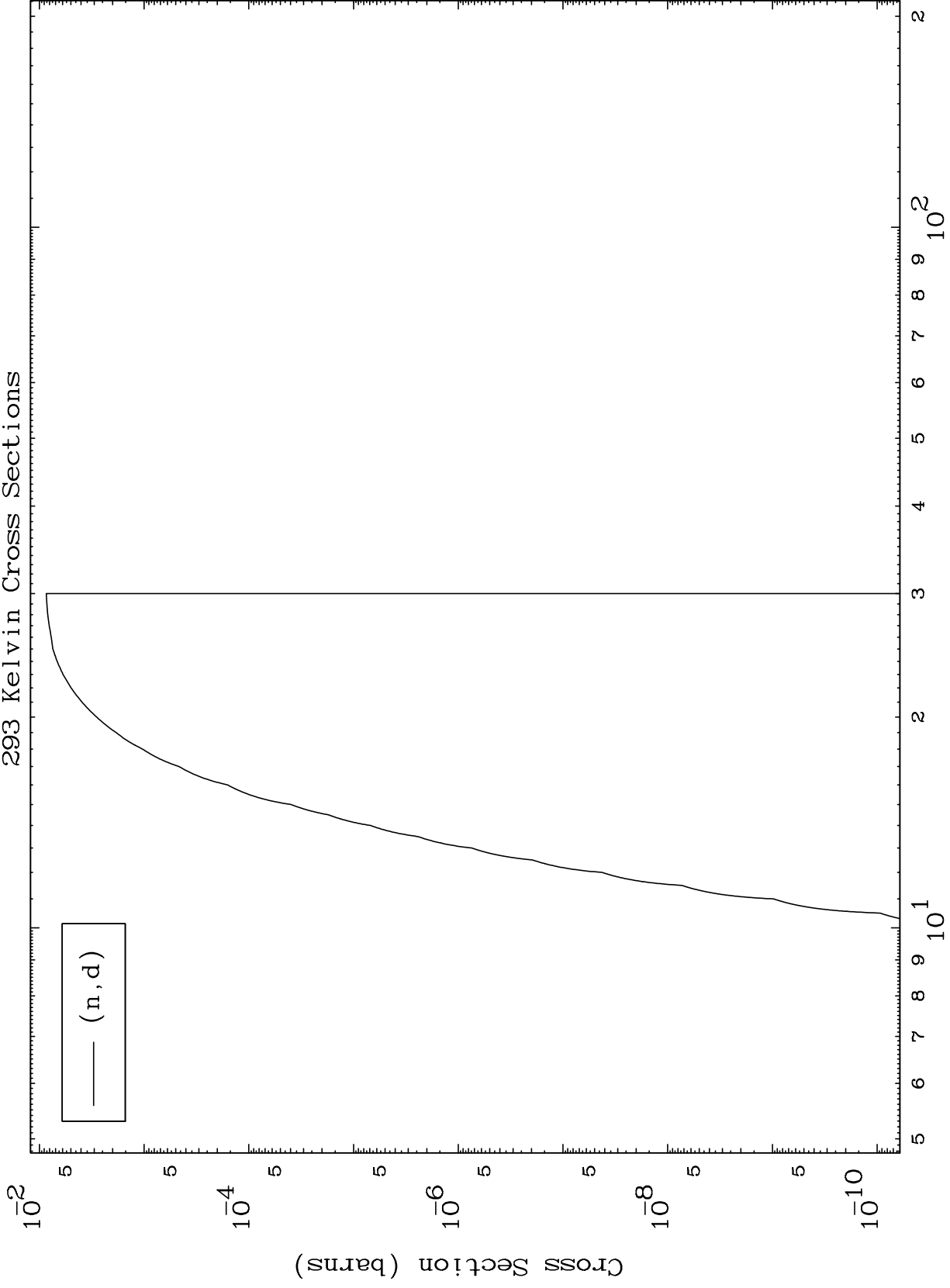
92-U -237



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(n,d) Levels
293 Kelvin Cross Sections

92-U -237



12

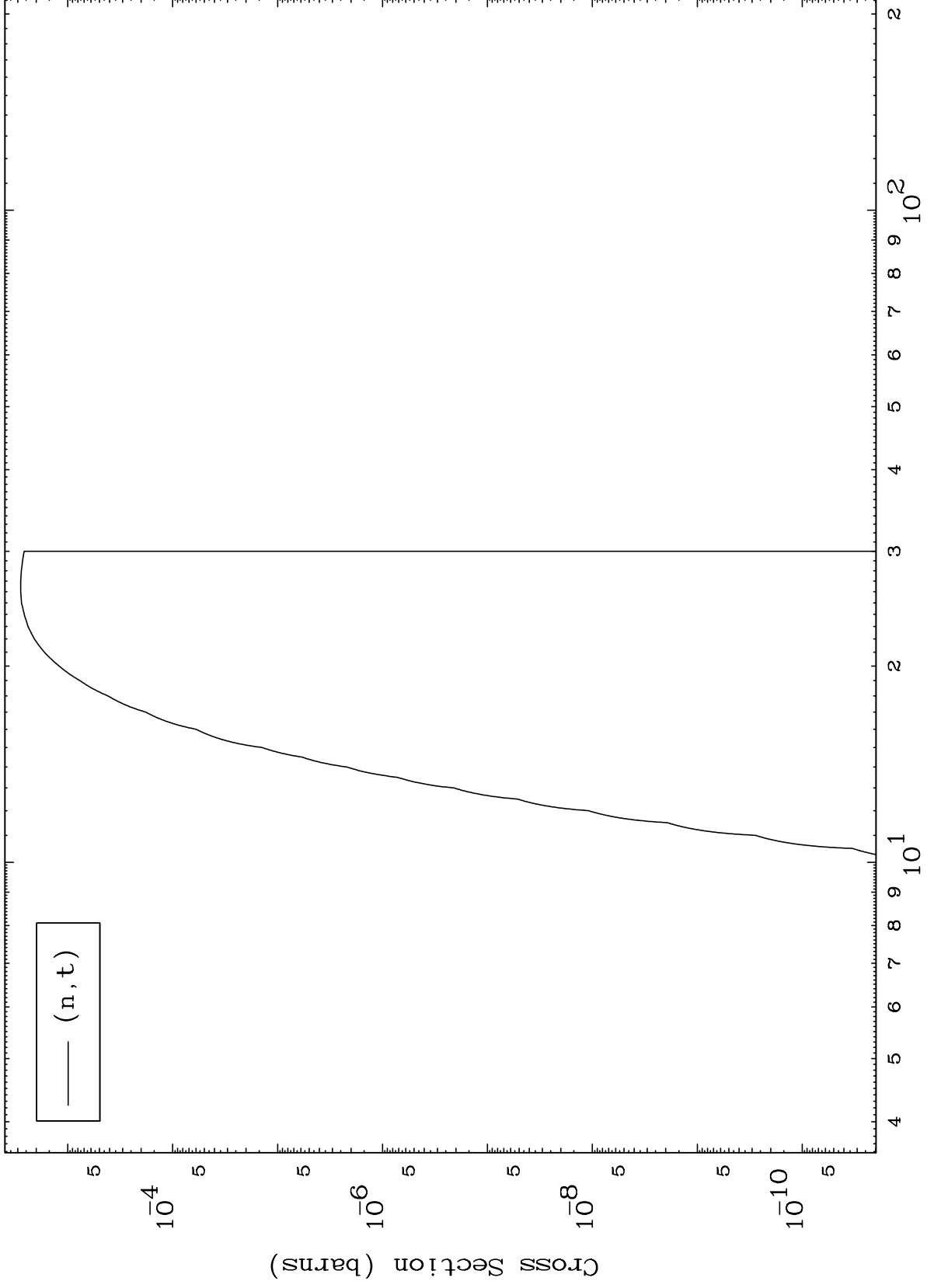
Incident Energy (MeV)

92-U -237

MAT 9234

(n,t) Levels
293 Kelvin Cross Sections

92-U -237



13

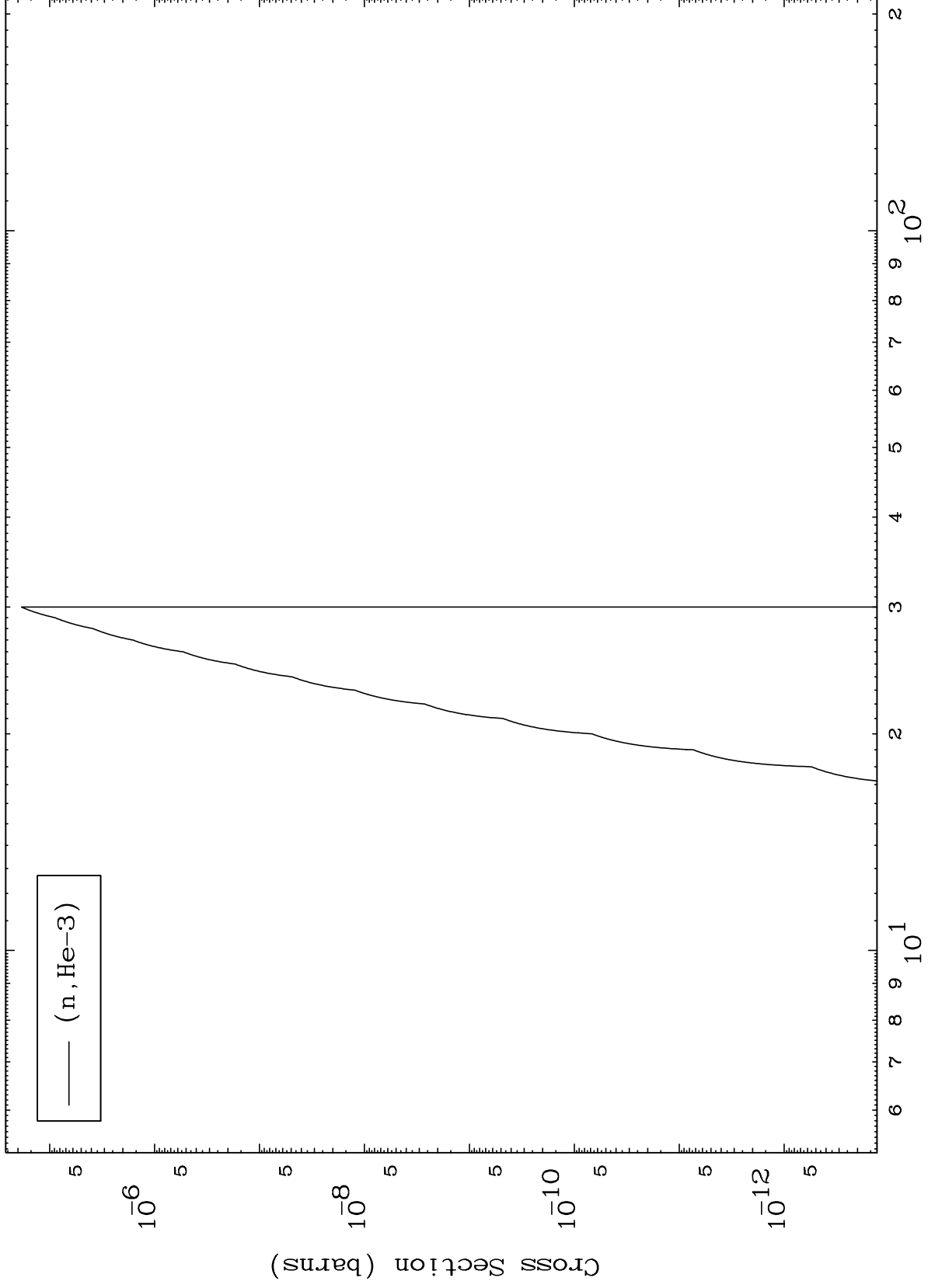
Incident Energy (MeV)

92-U -237

MAT 9234

(n,He3) Levels
293 Kelvin Cross Sections

92-U -237



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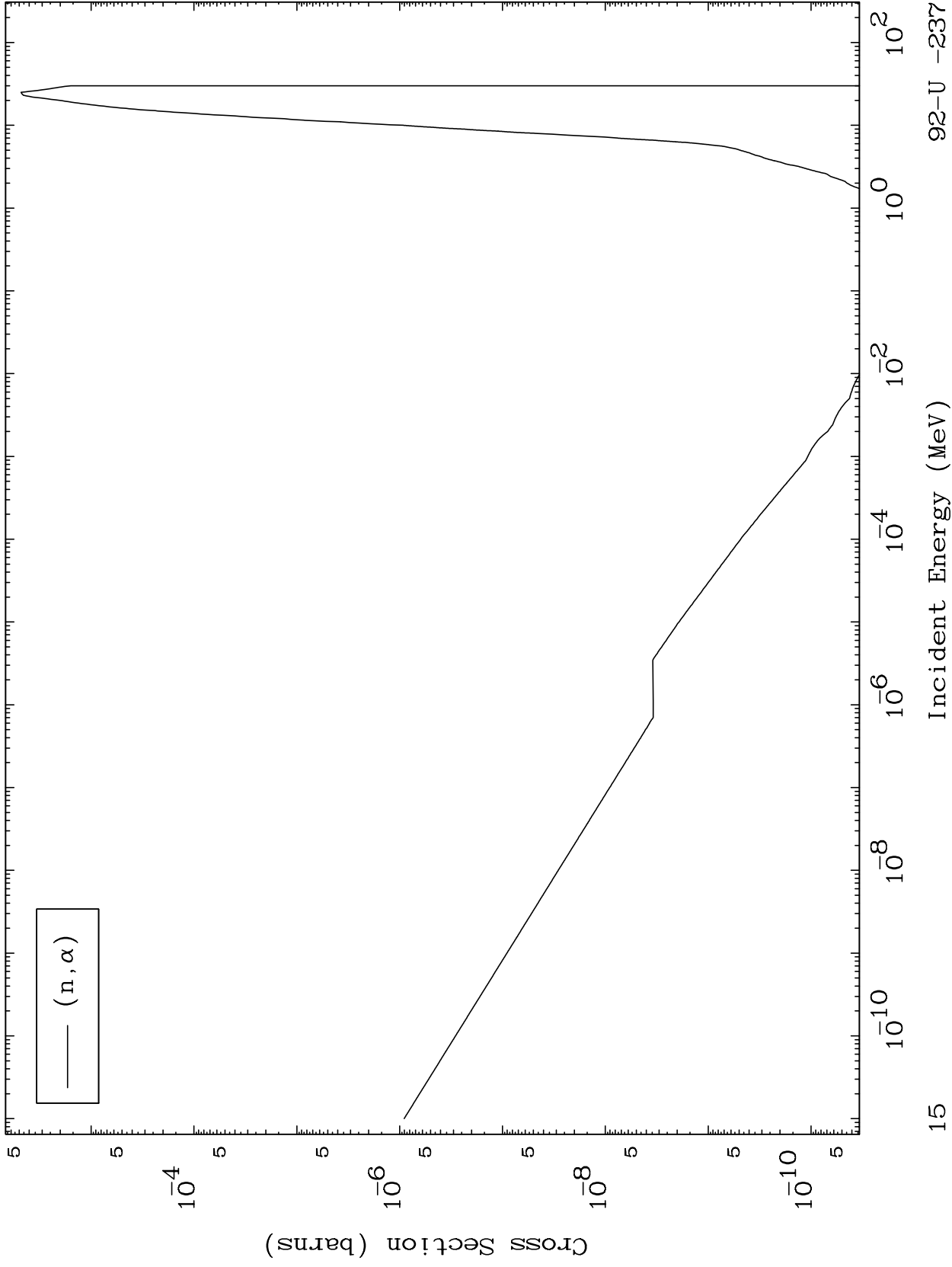
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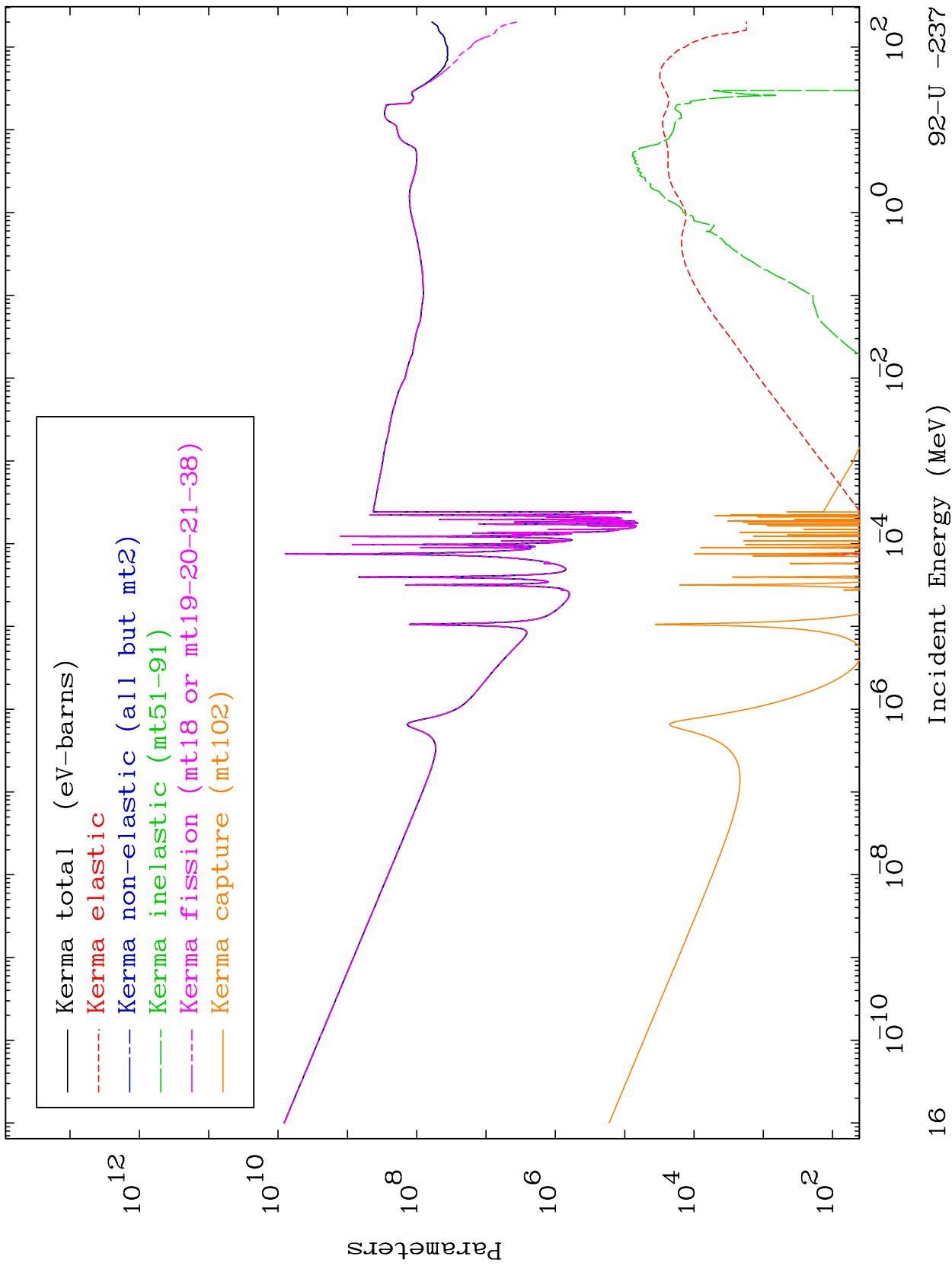
92-U -237

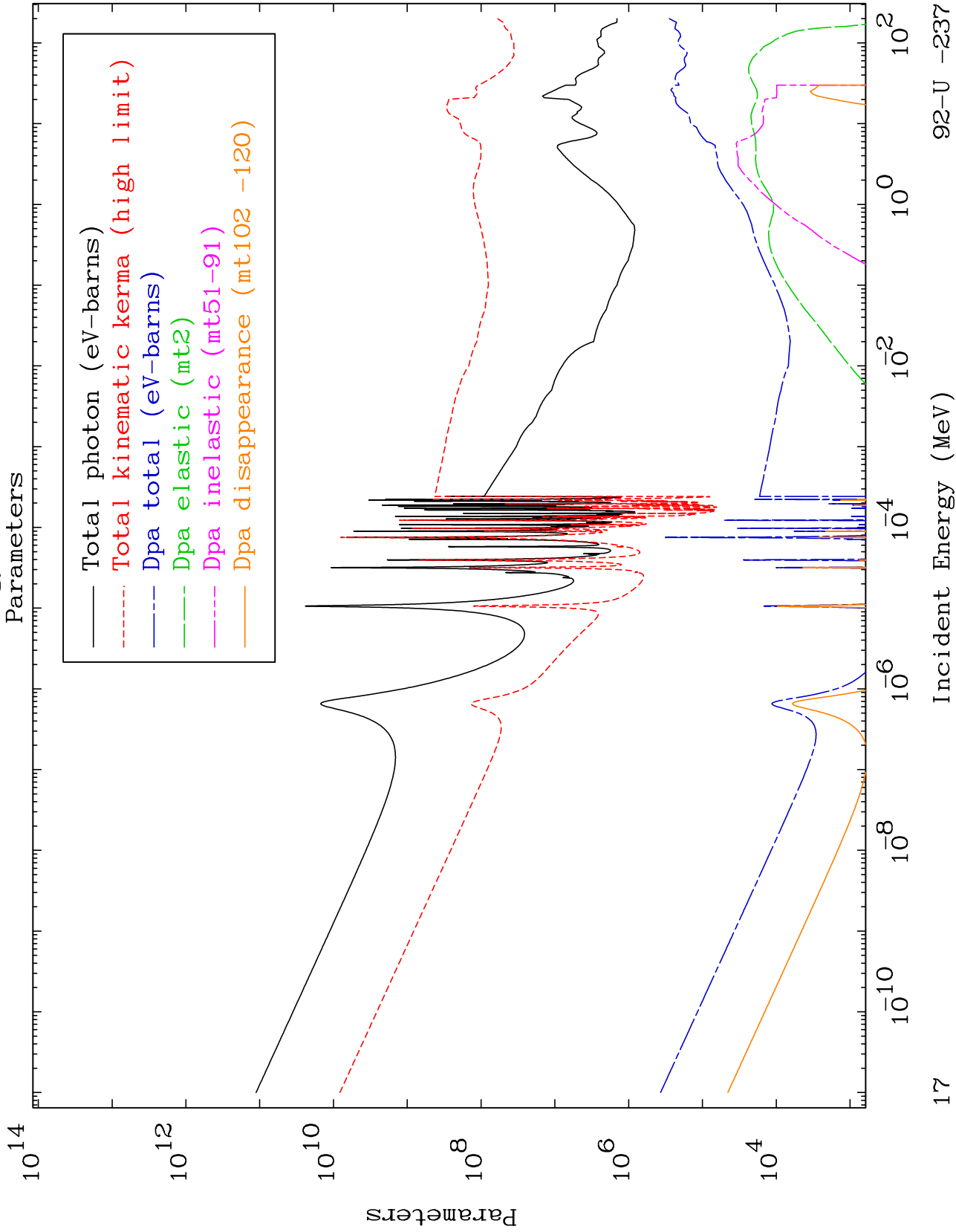
MAT 9234

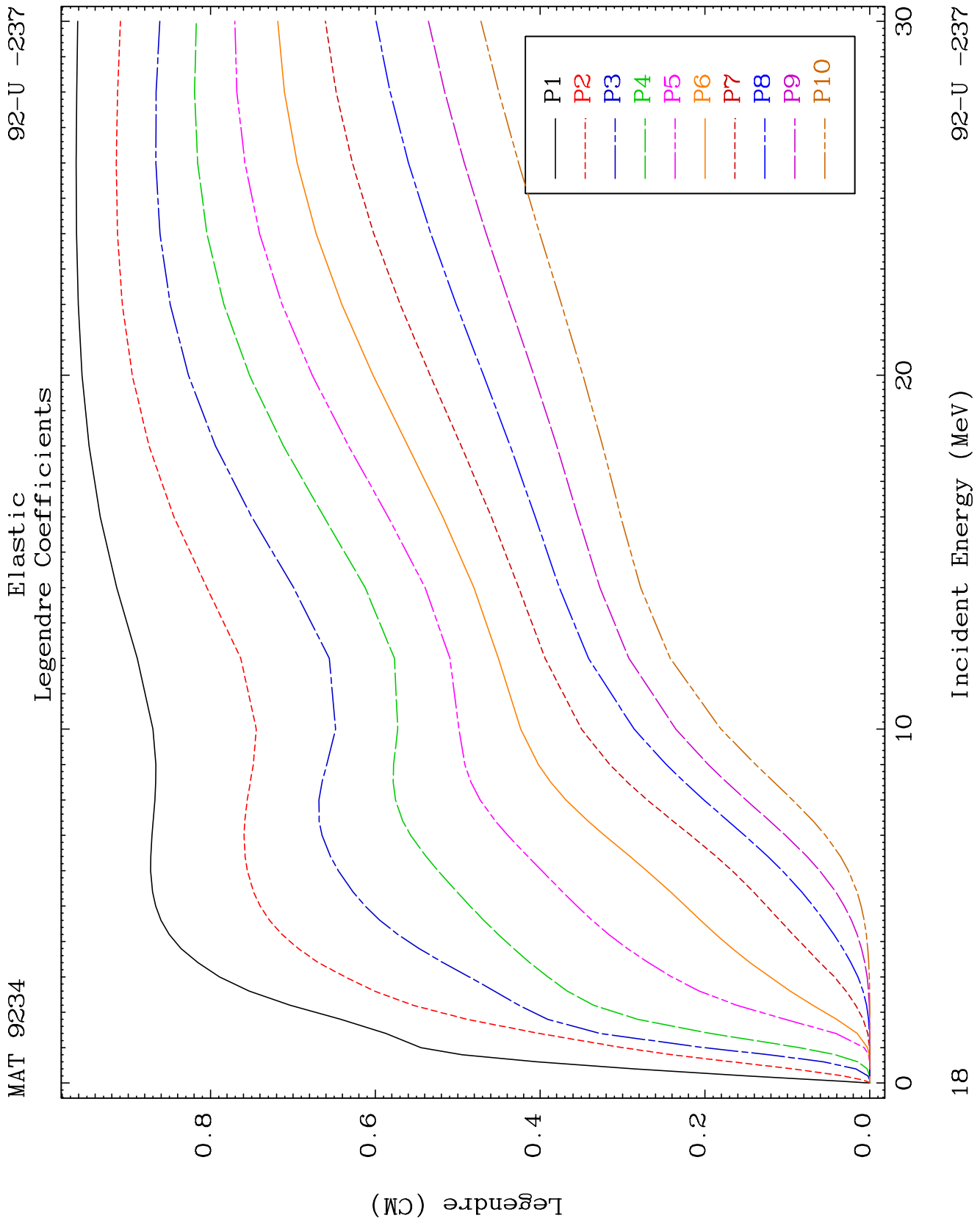
(n, α) Levels
293 Kelvin Cross Sections

92-U -237





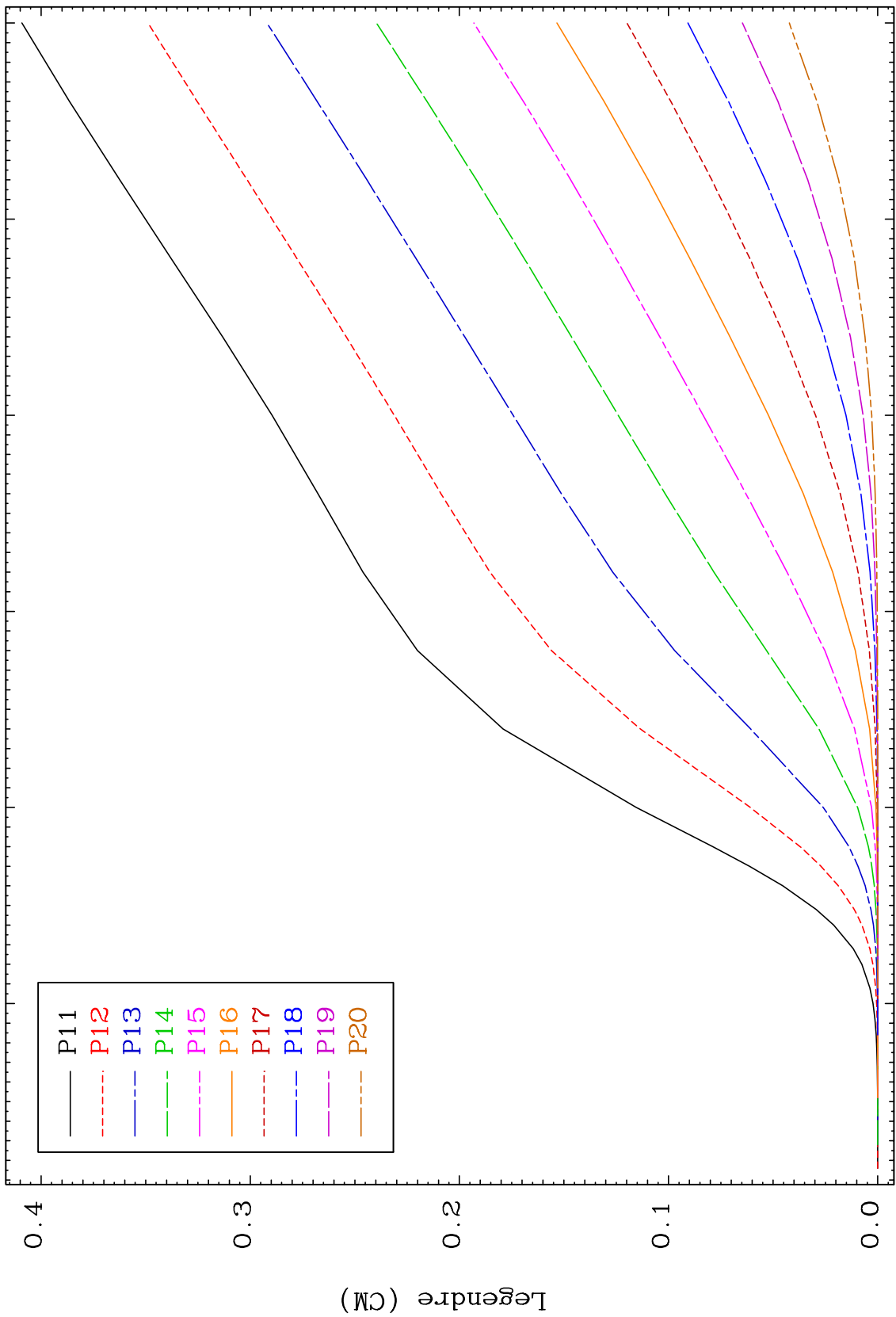




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Elastic Legendre Coefficients

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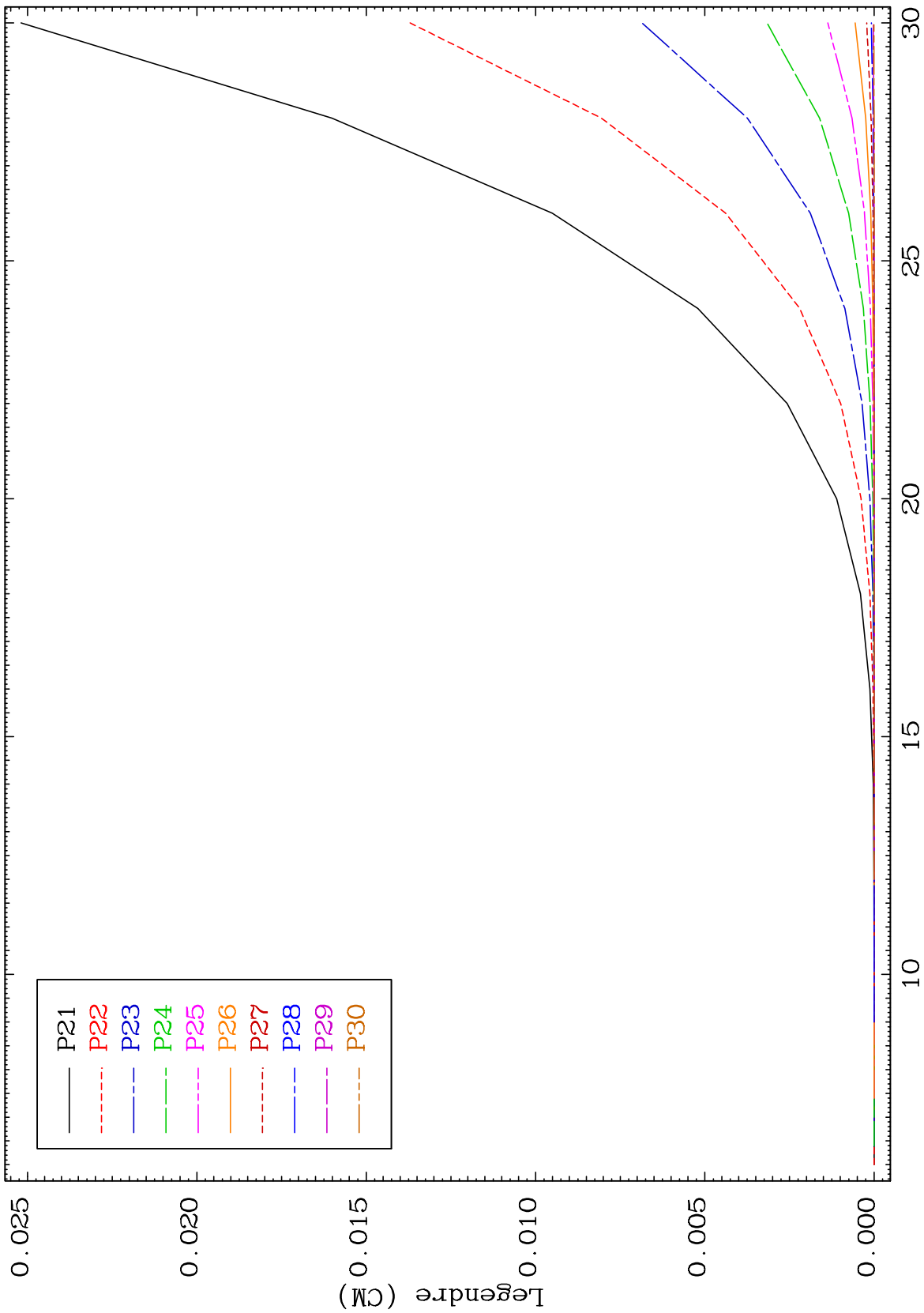
Incident Energy (MeV)

92-U -237

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Elastic
Legendre Coefficients

92-U -237



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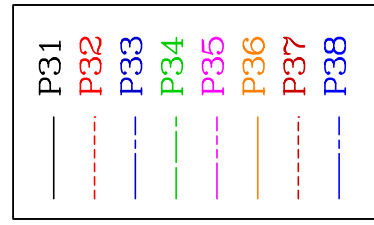
Incident Energy (MeV)

92-U -237

MAT 9234

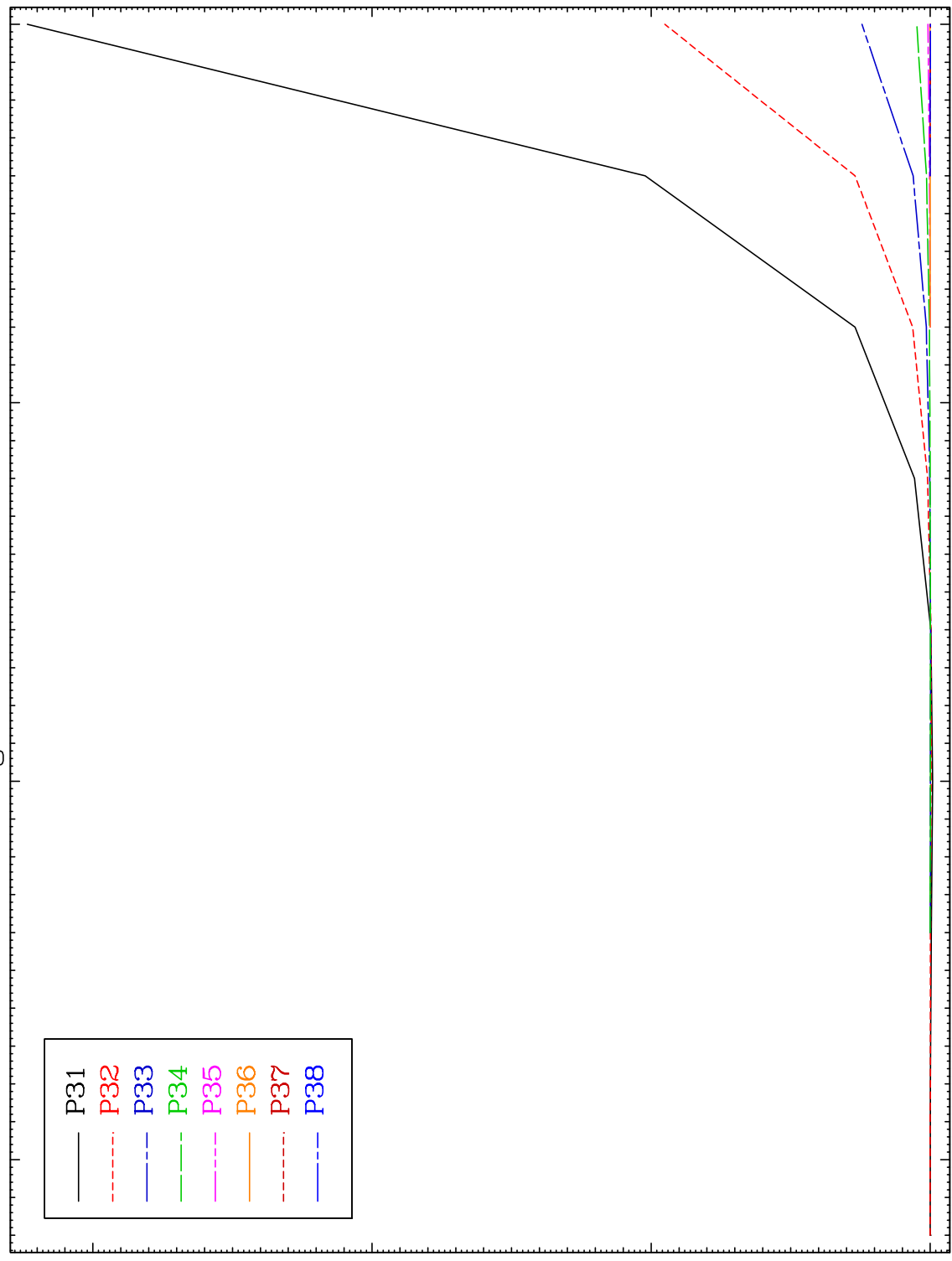
Elastic Legendre Coefficients

92-U -237



$\times 10^{-6}$

Legendre (CM)



21

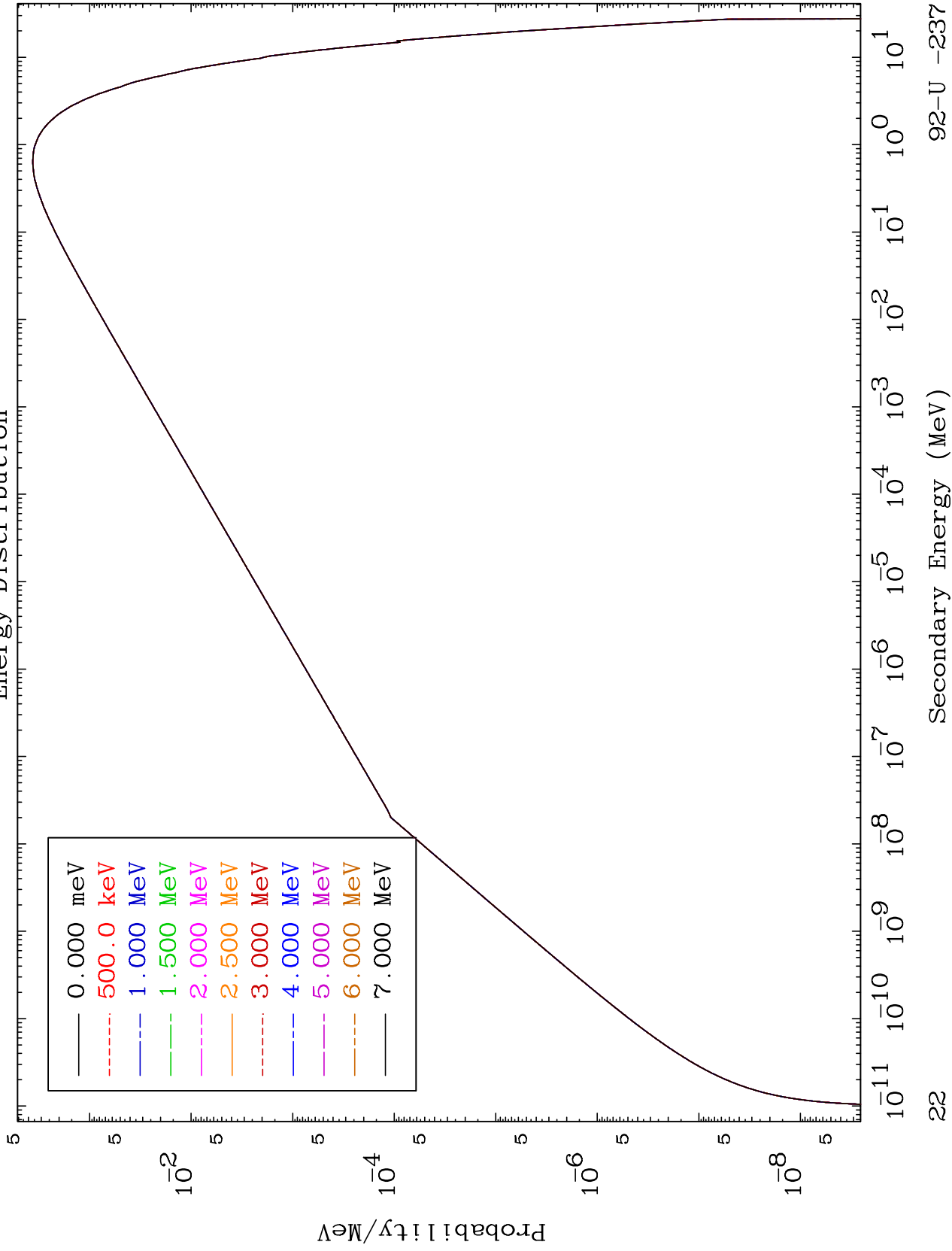
Incident Energy (MeV)

92-U -237

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Fission Energy Distribution

92-U -237



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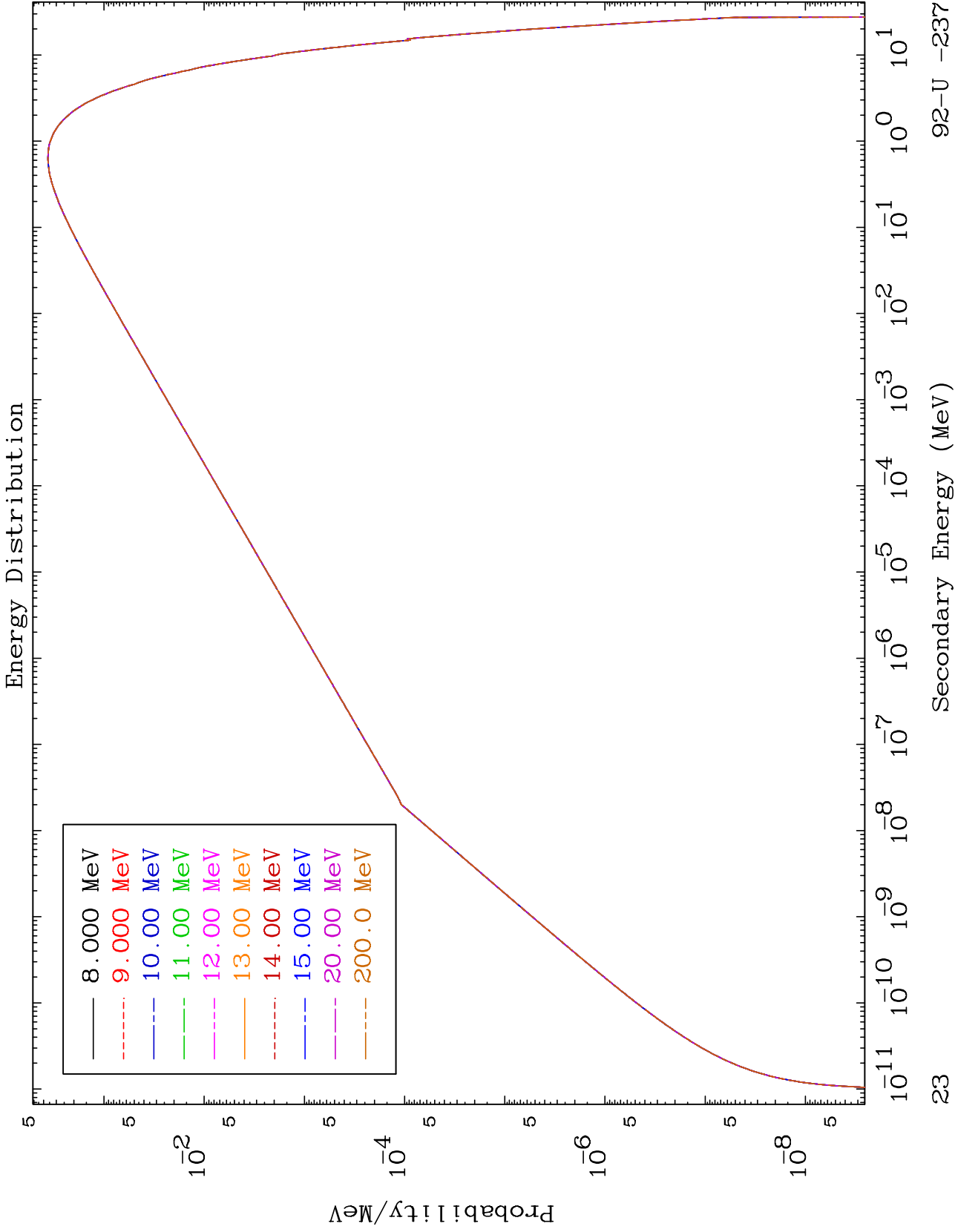
Secondary Energy (MeV)

22

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Fission

92-U -237



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Secondary Energy (MeV)

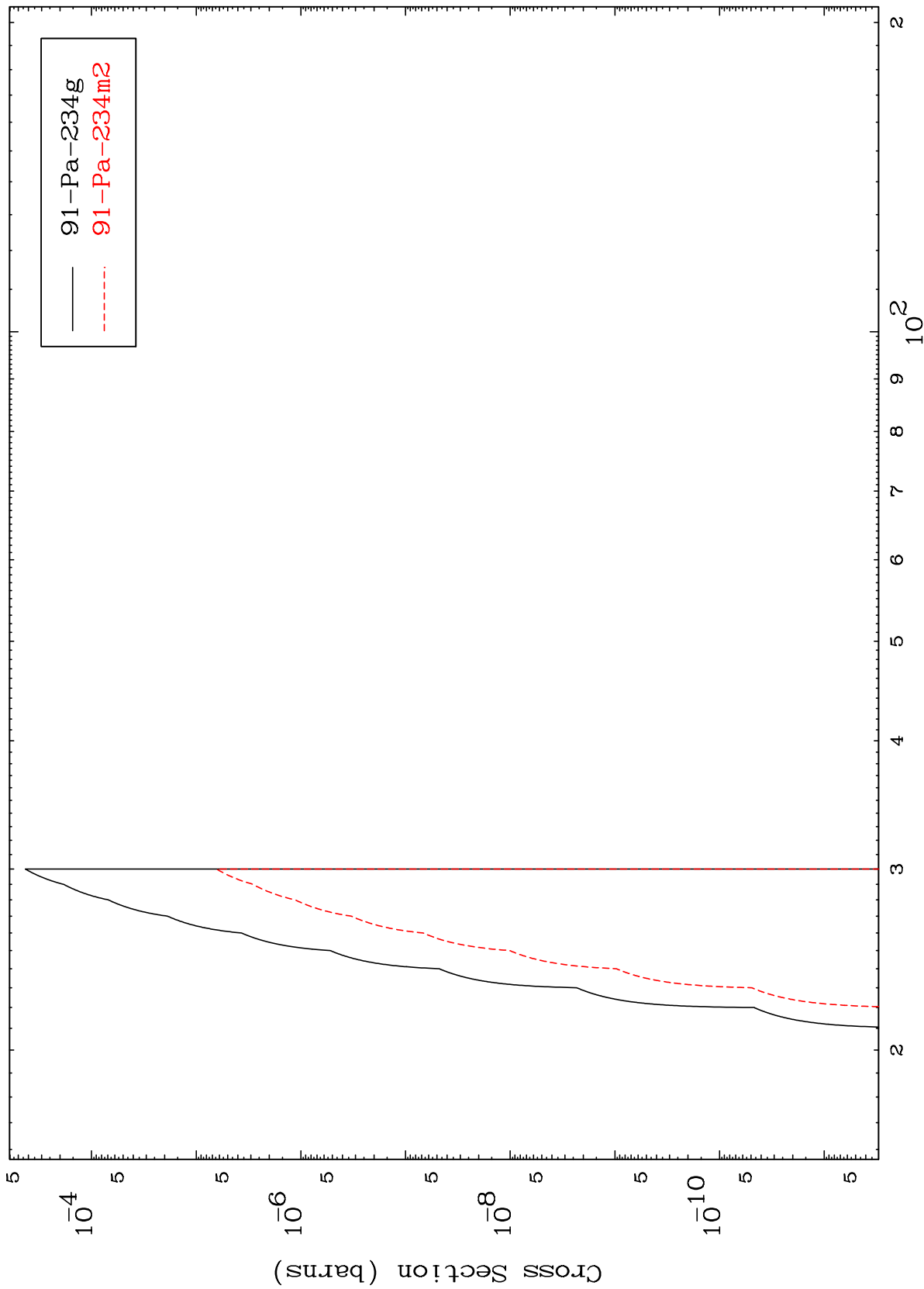
92-U -237

MAT 92334

(n,2n) d

92-U -237

Radionuclide Production Cross Section



24

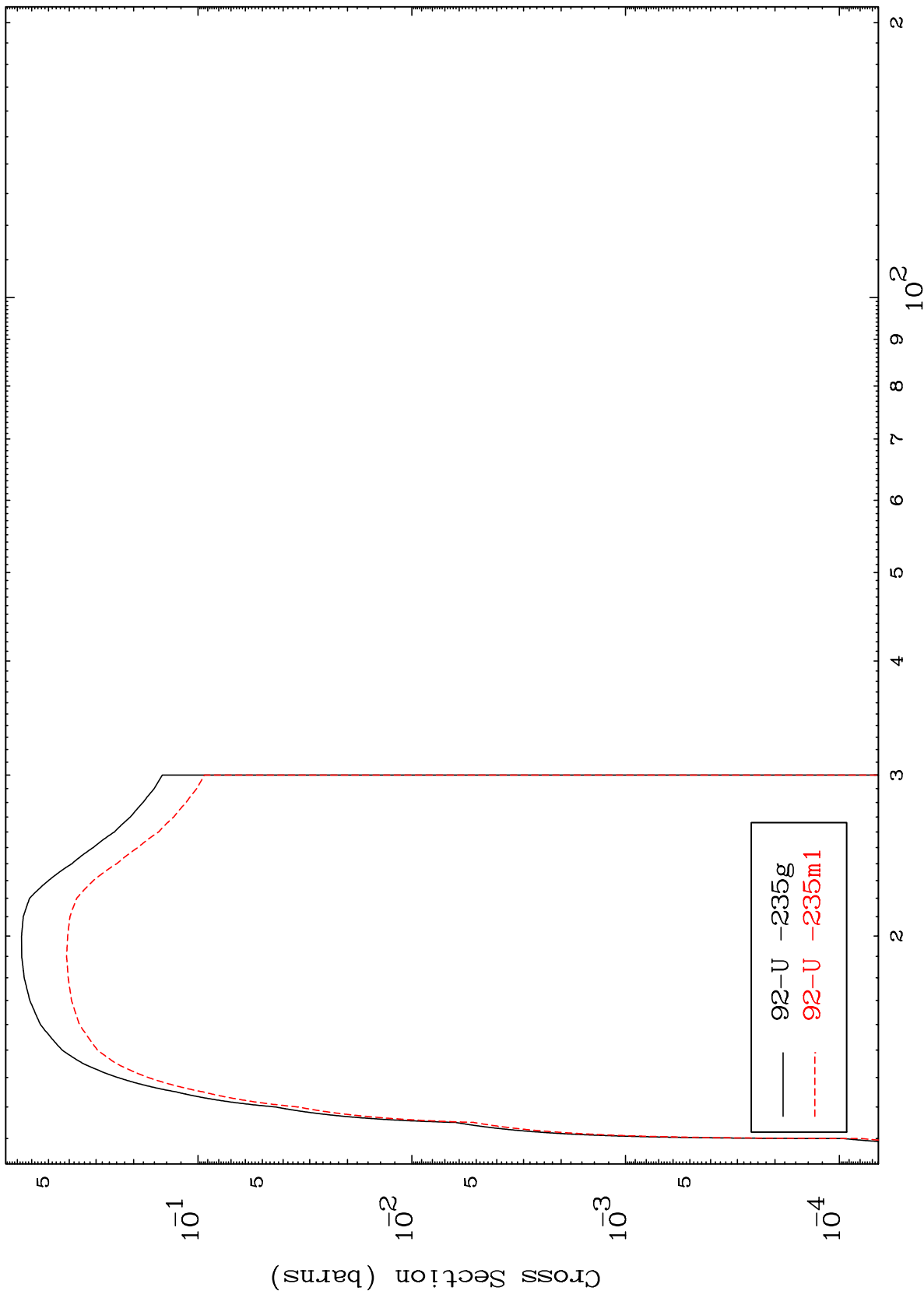
Incident Energy (MeV)

92-U -237

MAT 92334

92-U -237

Radionuclide Production Cross Section



92-U -237

Incident Energy (MeV)

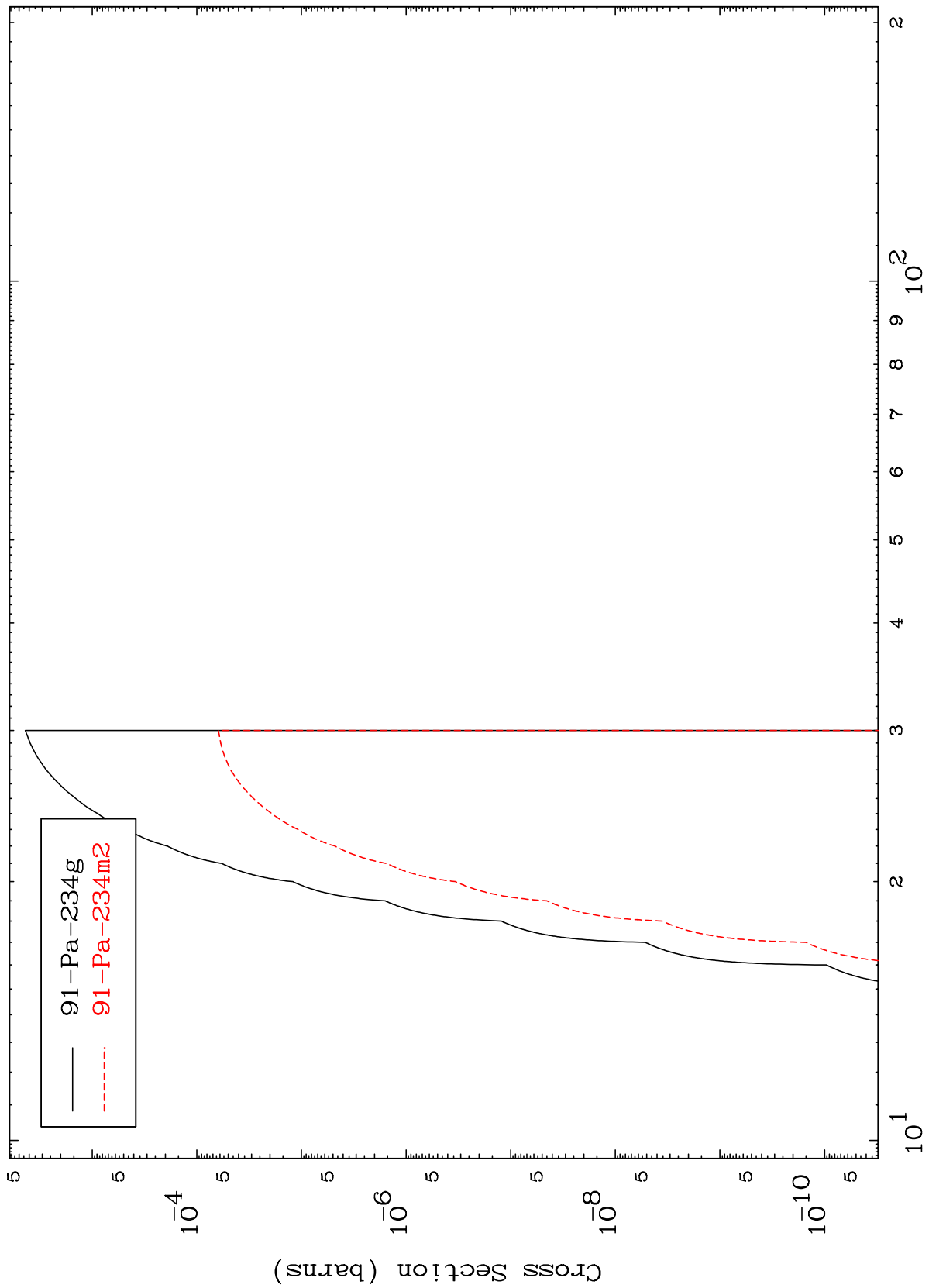
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MAT 9234

(n,n') t

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Radionuclide Production Cross Section



Incident Energy (MeV)

92-U -237

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Radionuclide Production Cross Section

