

Program EVALPLOT
(Version 2018-1)

by

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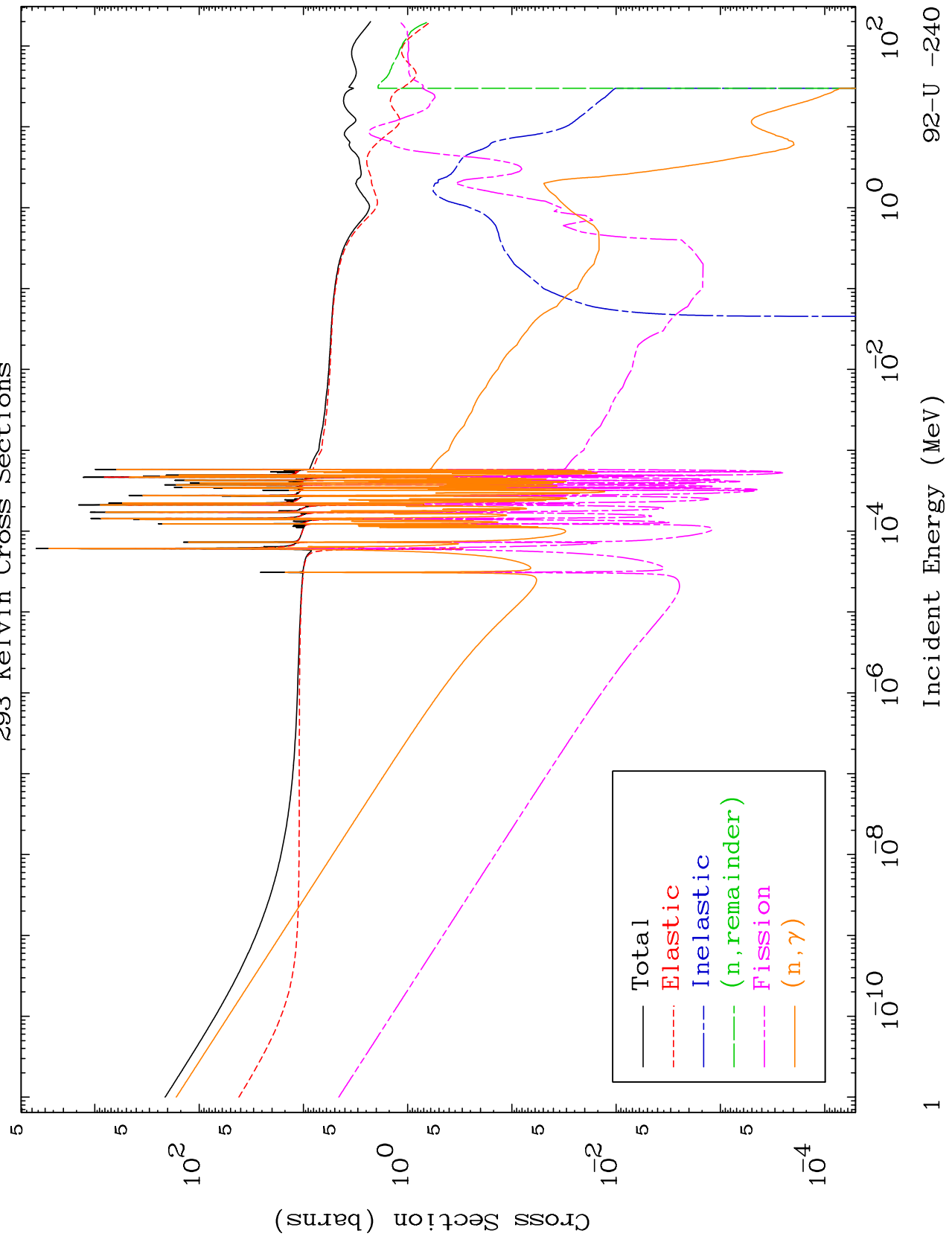
Press Mouse Button to Start

MAT 9243

Major

293 Kelvin Cross Sections

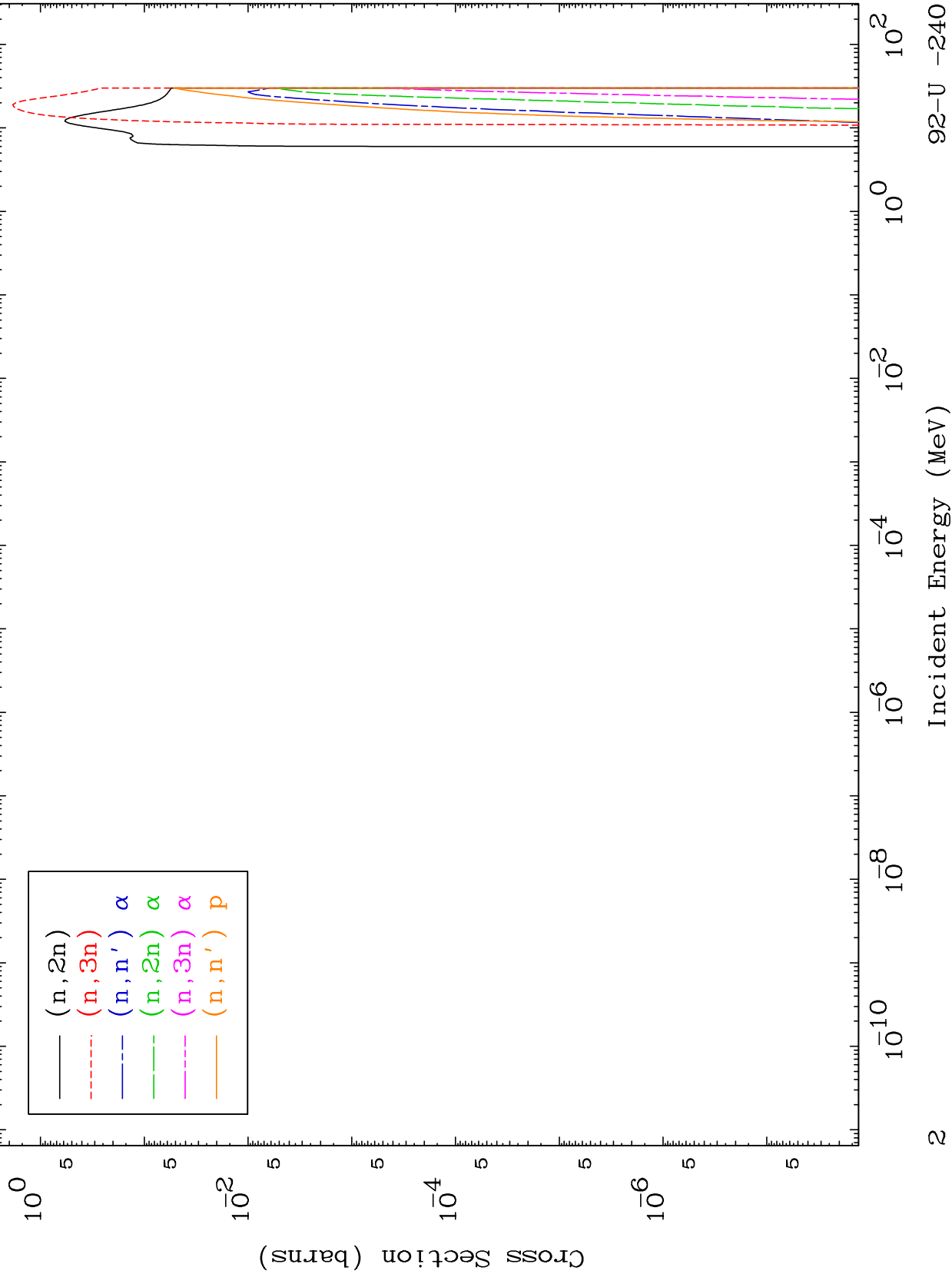
92-U -240



MAT 9243

Neutron Production
293 Kelvin Cross Sections

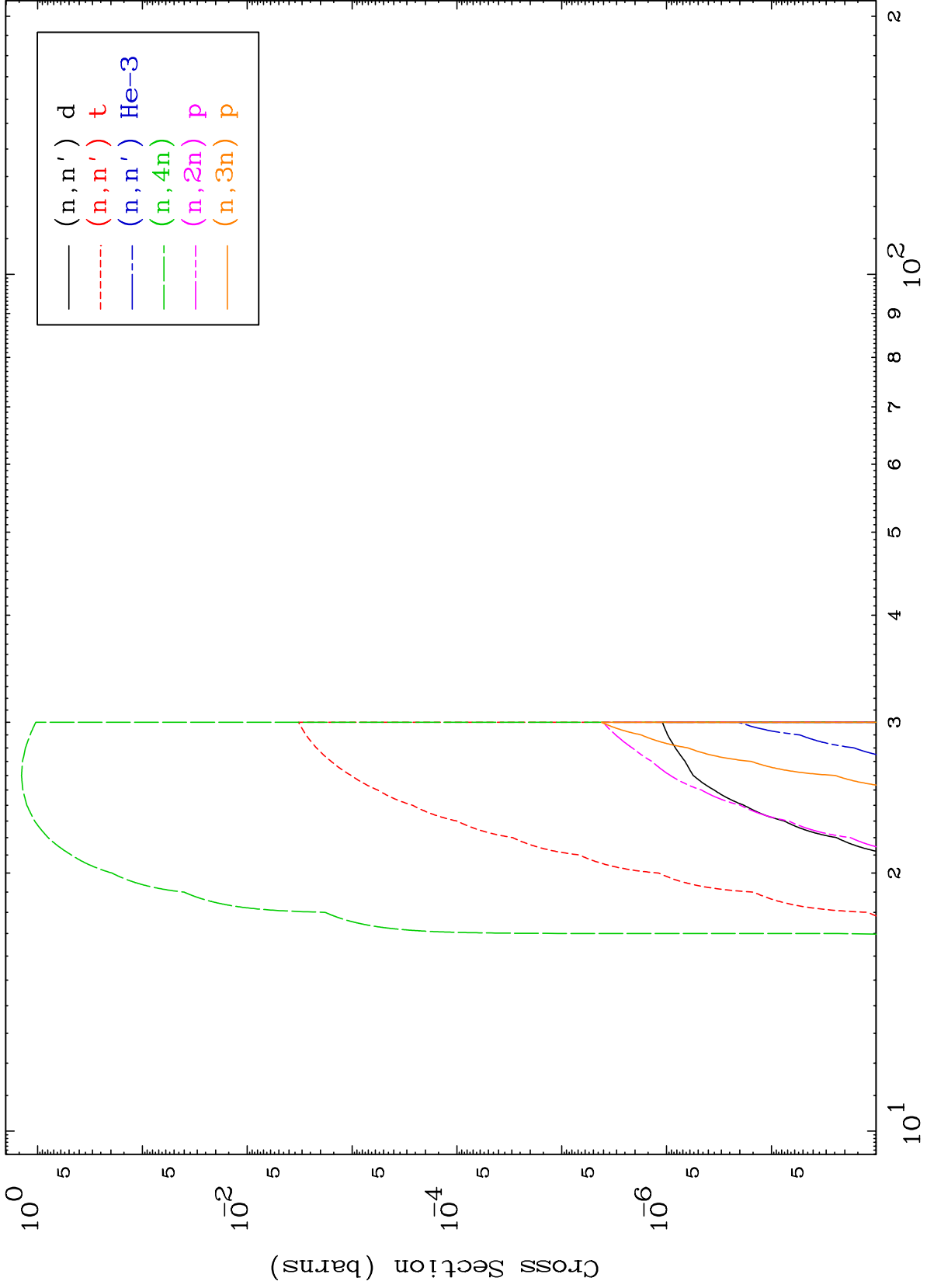
92-U -240



MAT 9243

Neutron Production
293 Kelvin Cross Sections

92-U -240



92-U -240

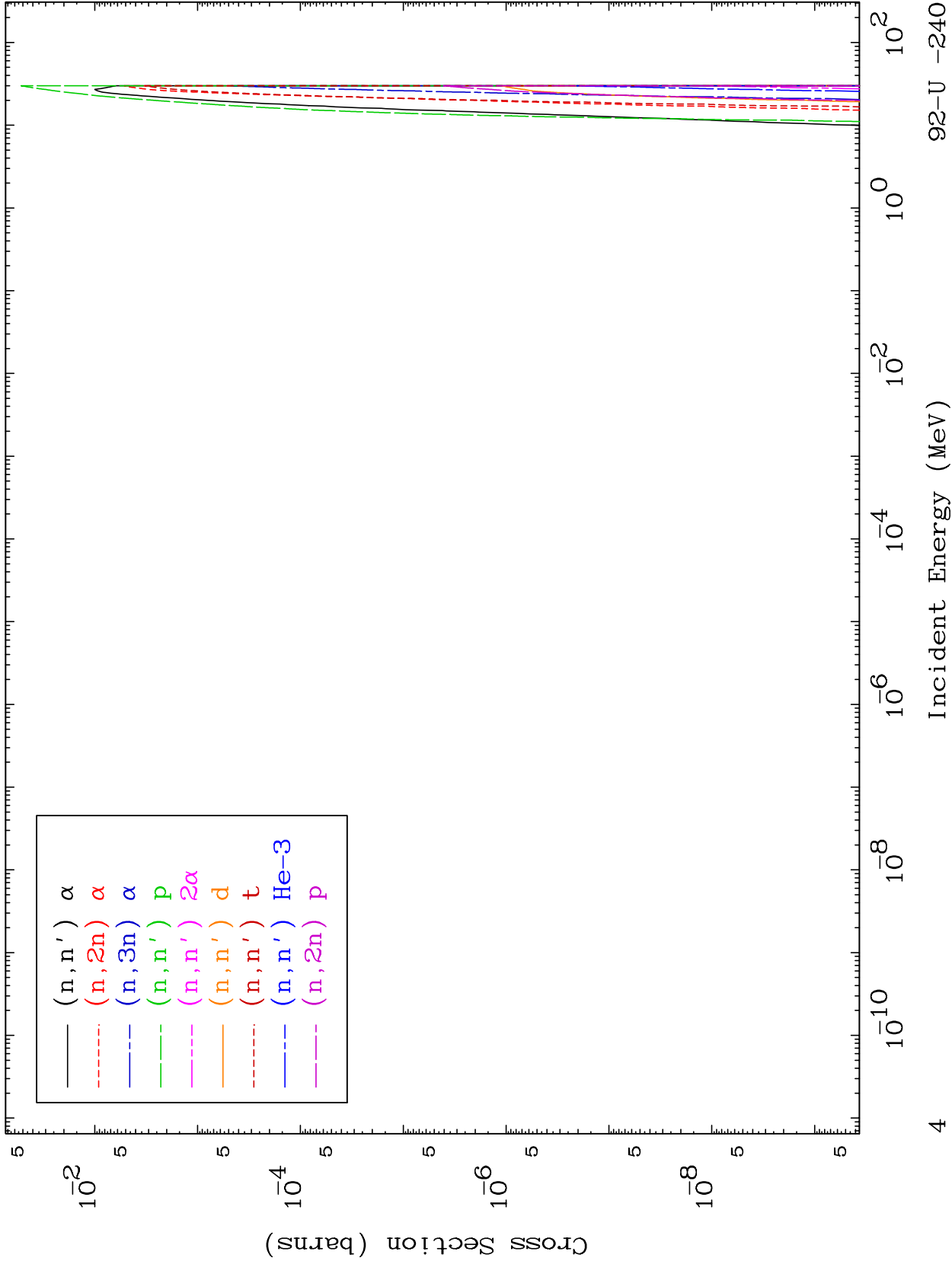
Incident Energy (MeV)

3

MAT 9243

Charged Particle
293 Kelvin Cross Sections

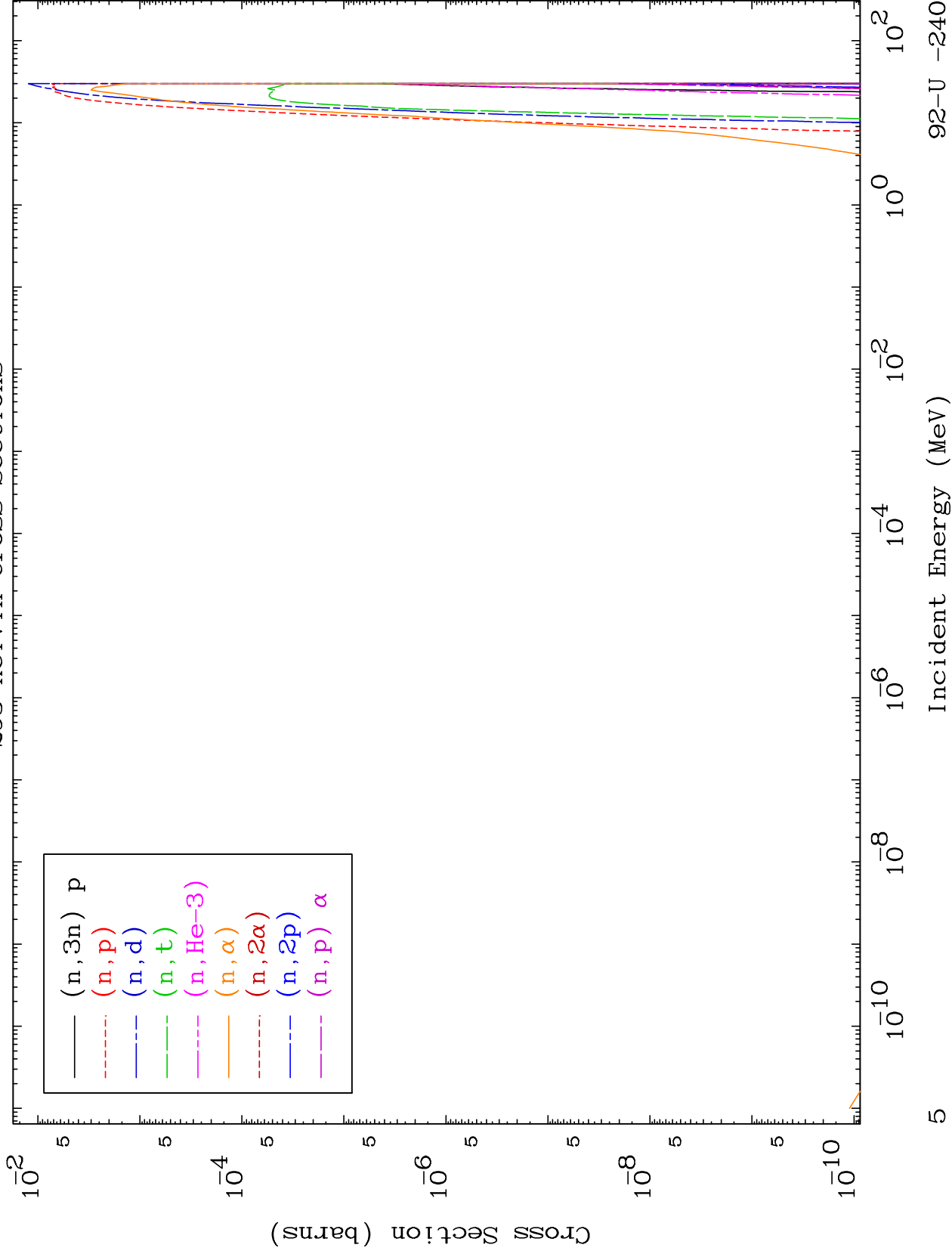
92-U -240



MAT 9243

Charged Particle
293 Kelvin Cross Sections

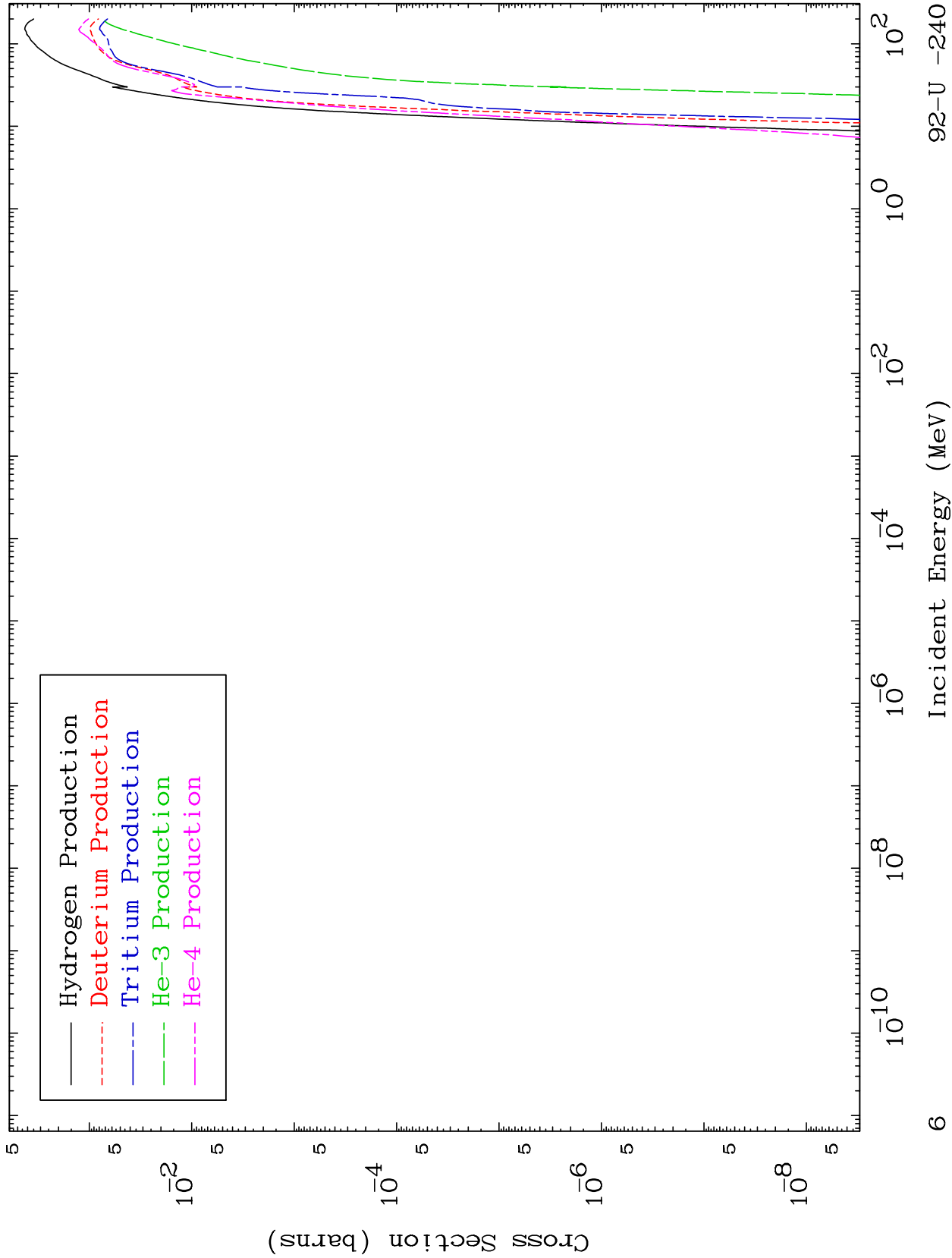
92-U -240



MAT 9243

Particle Production
293 Kelvin Cross Sections

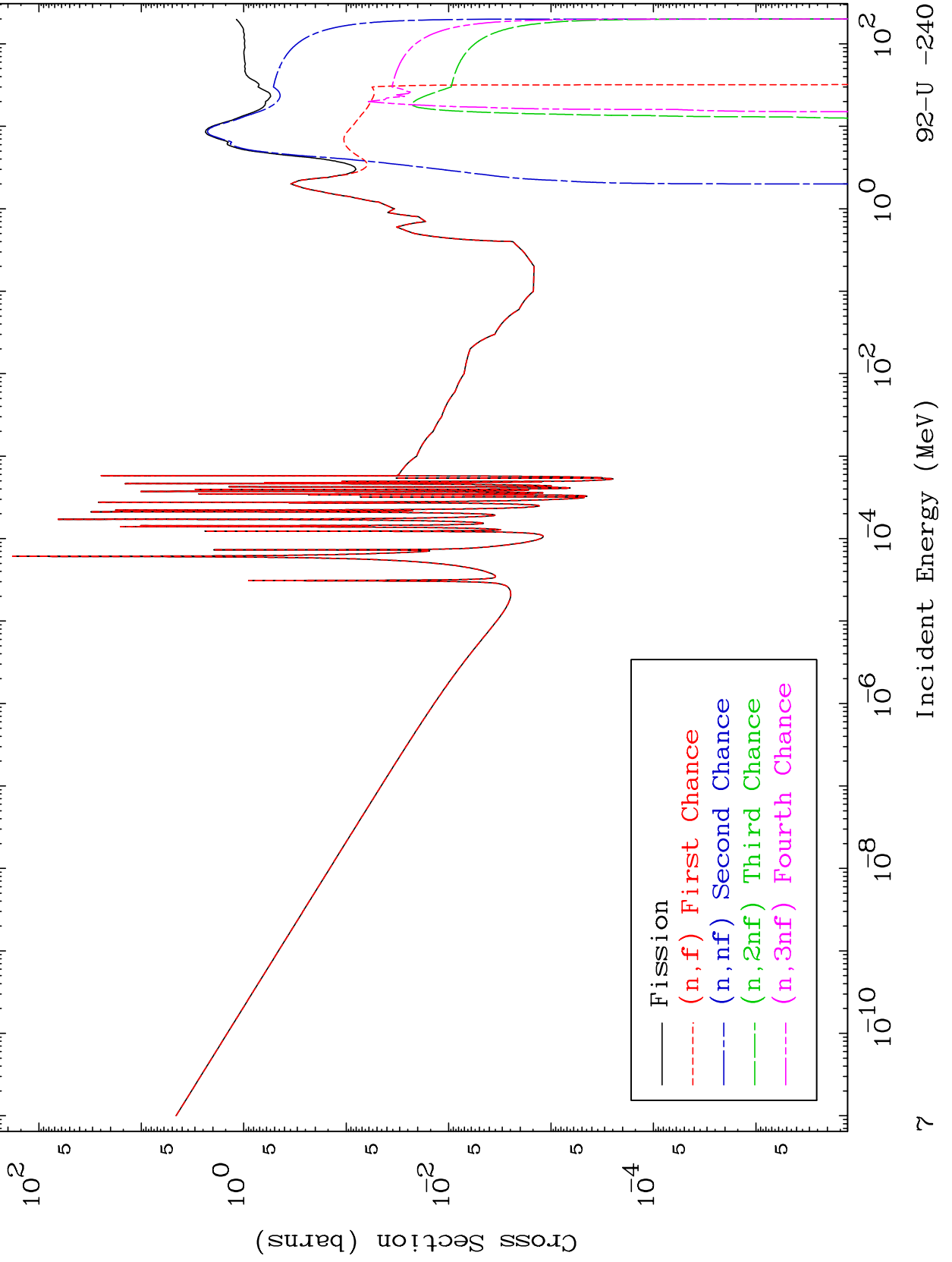
92-U -240



MAT 9243

Fission
293 Kelvin Cross Sections

92-U -240

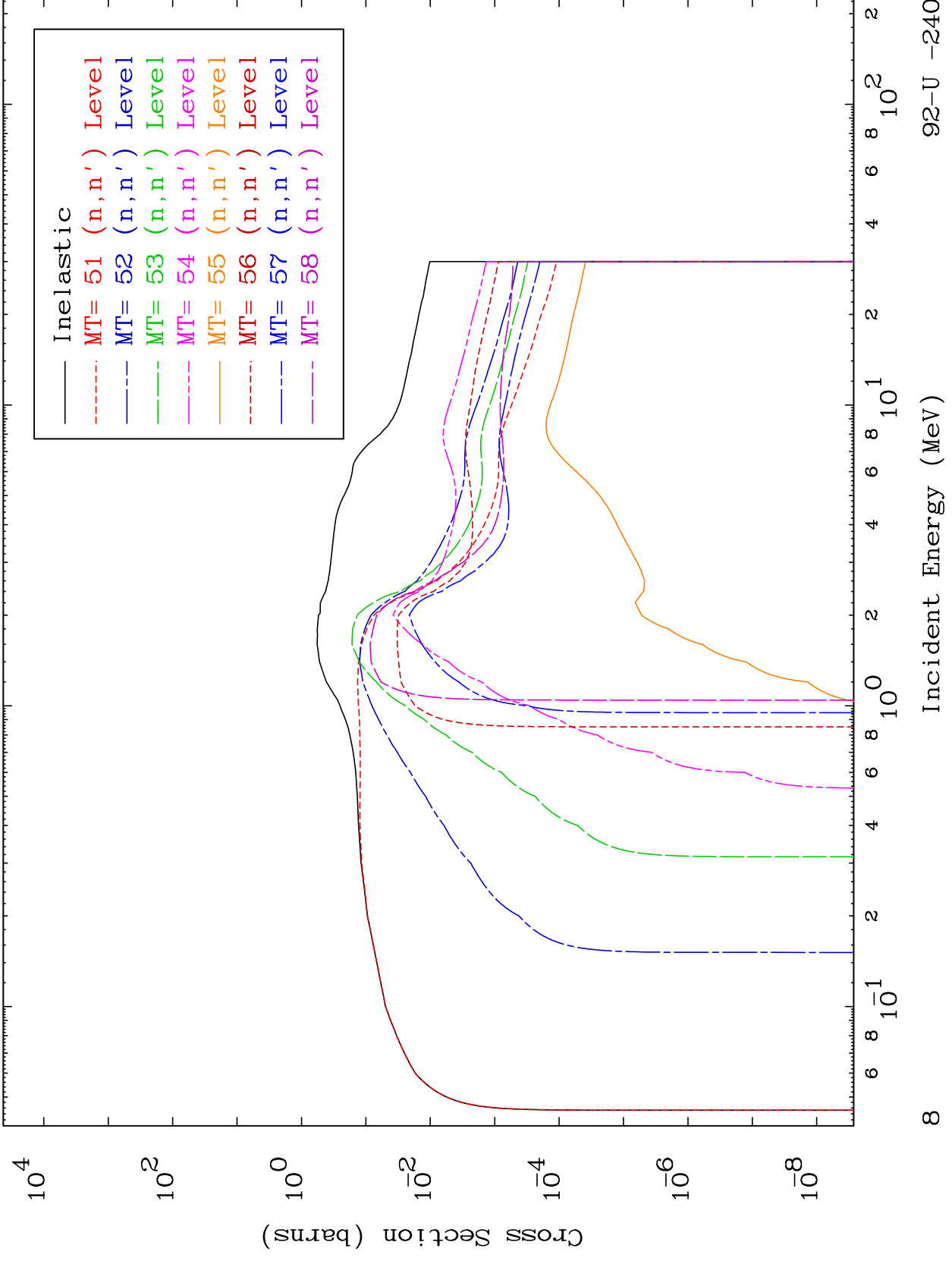


MAT 9243

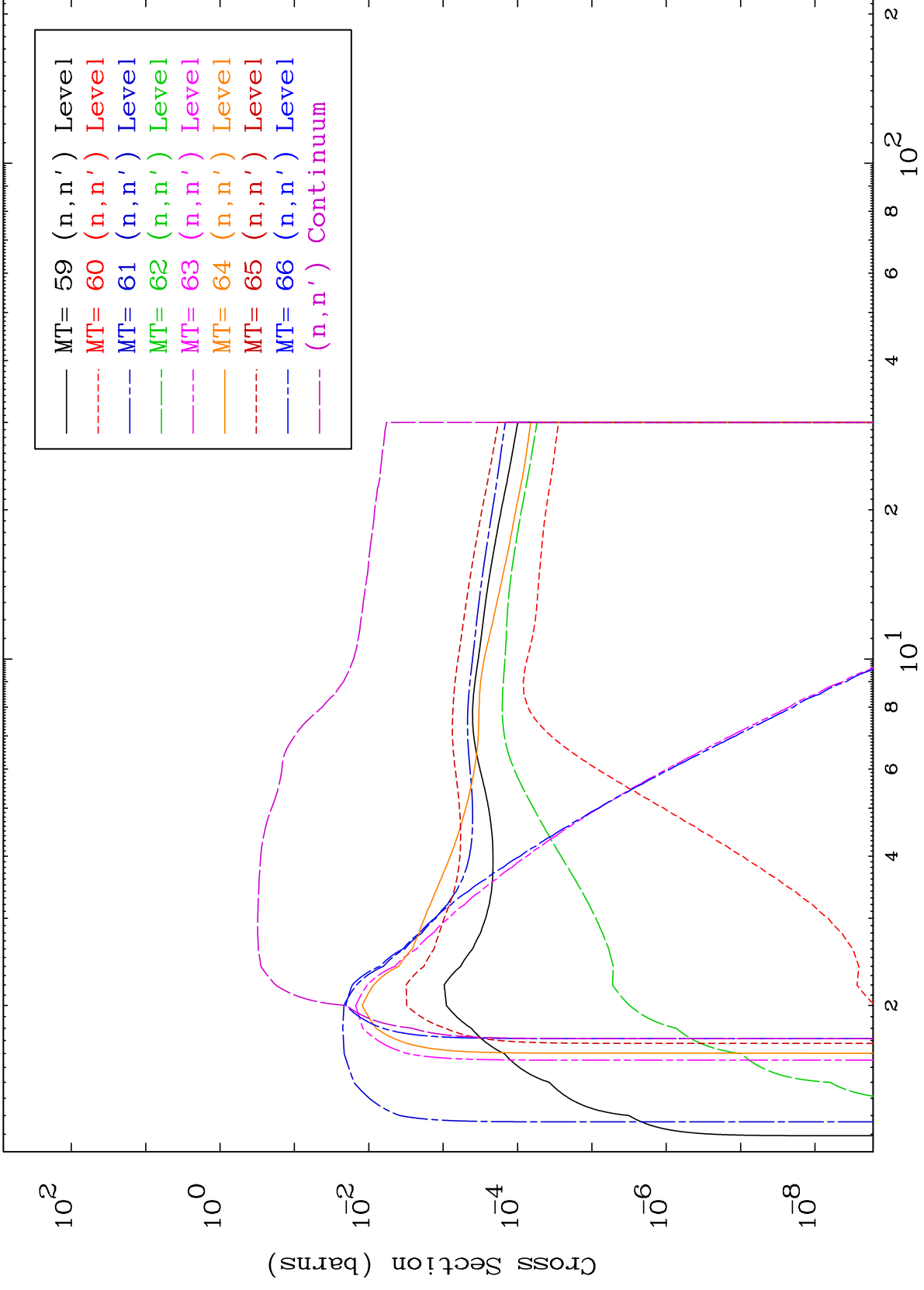
(n,n') Level

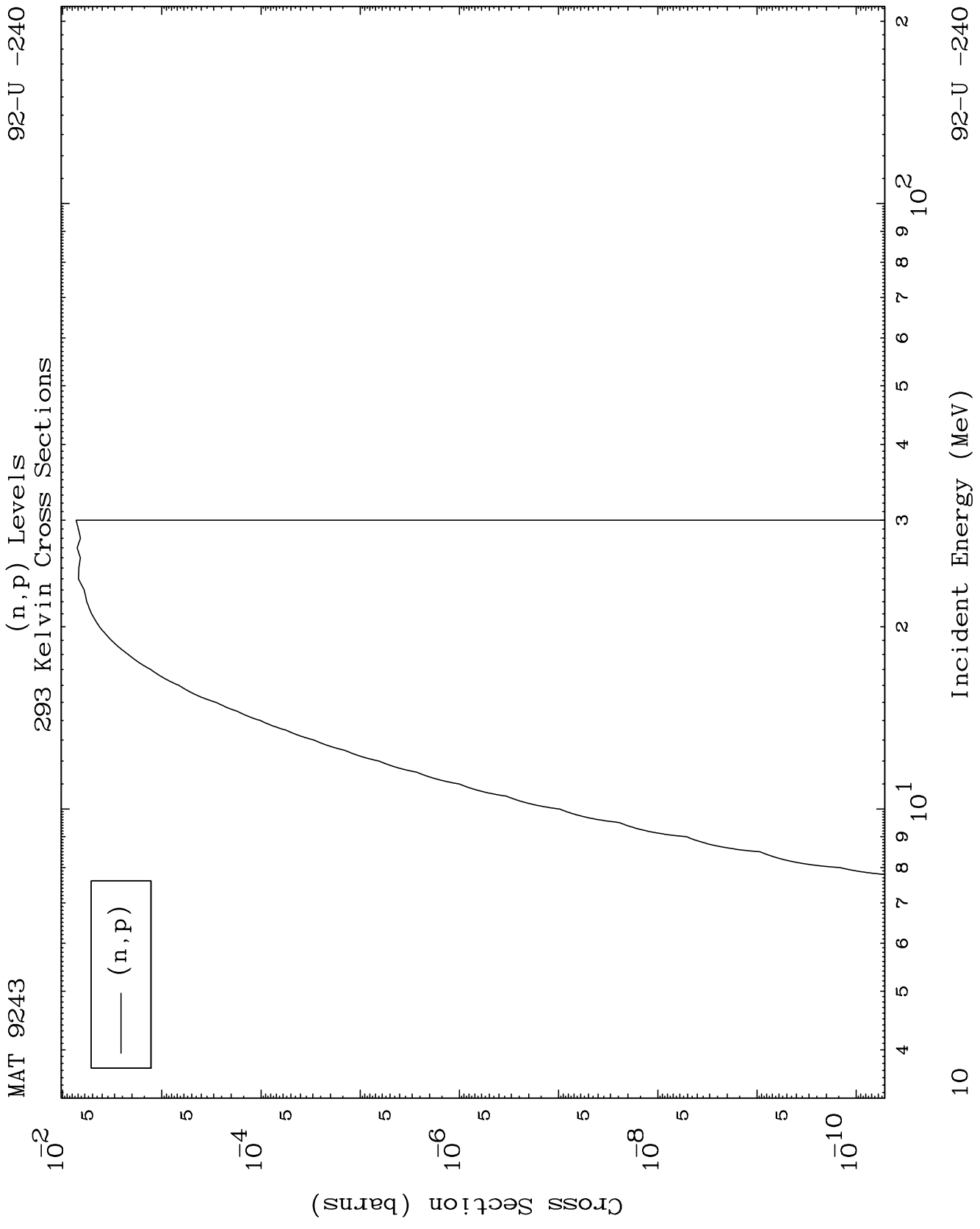
92-U -240

293 Kelvin Cross Sections



293 Kelvin Cross Sections

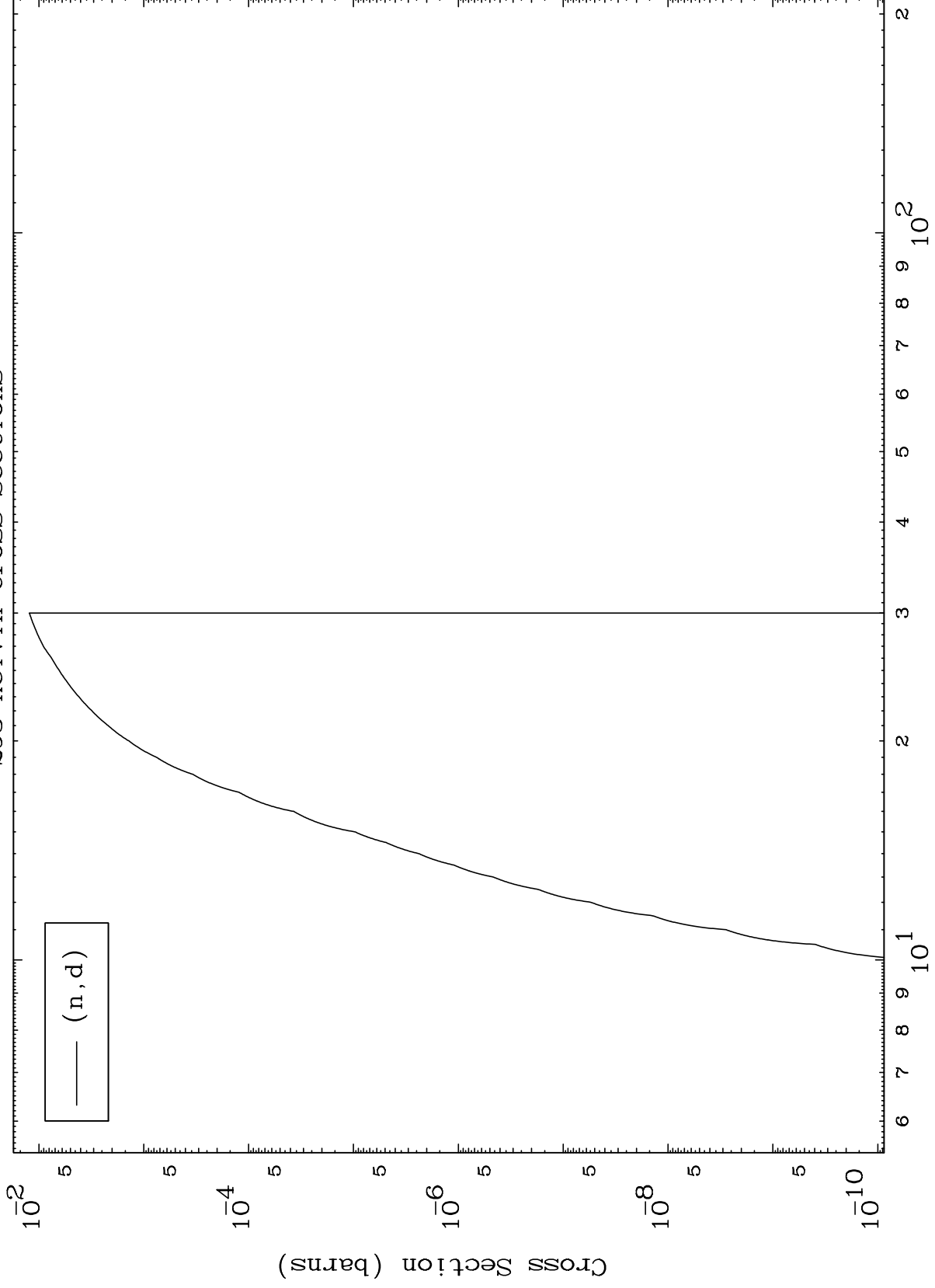




MAT 9243

(n,d) Levels
293 Kelvin Cross Sections

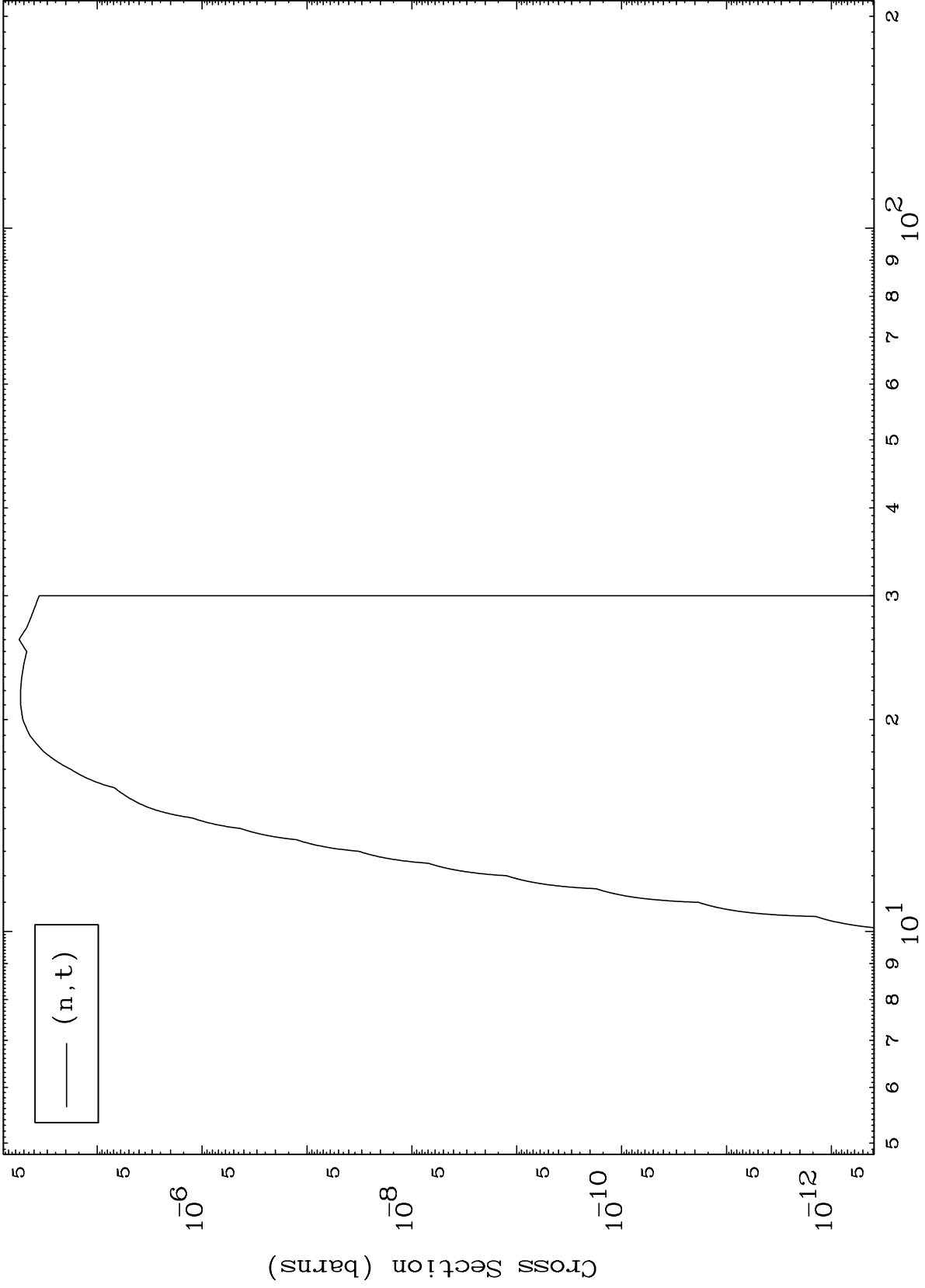
92-U -240



MAT 9243

(n,t) Levels
293 Kelvin Cross Sections

92-U -240



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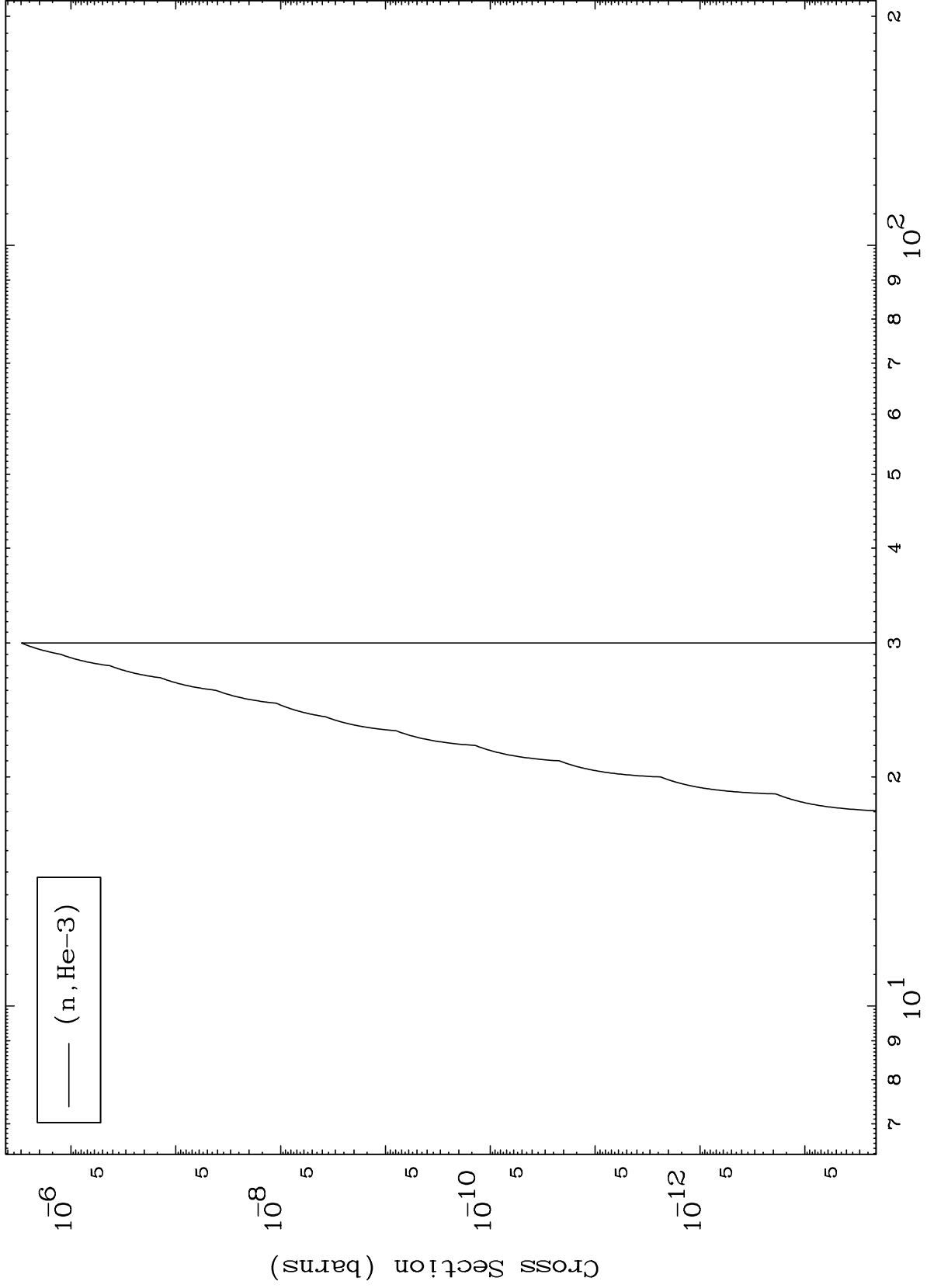
Incident Energy (MeV)

92-U -240

MAT 9243

(n,He3) Levels
293 Kelvin Cross Sections

92-U -240



13

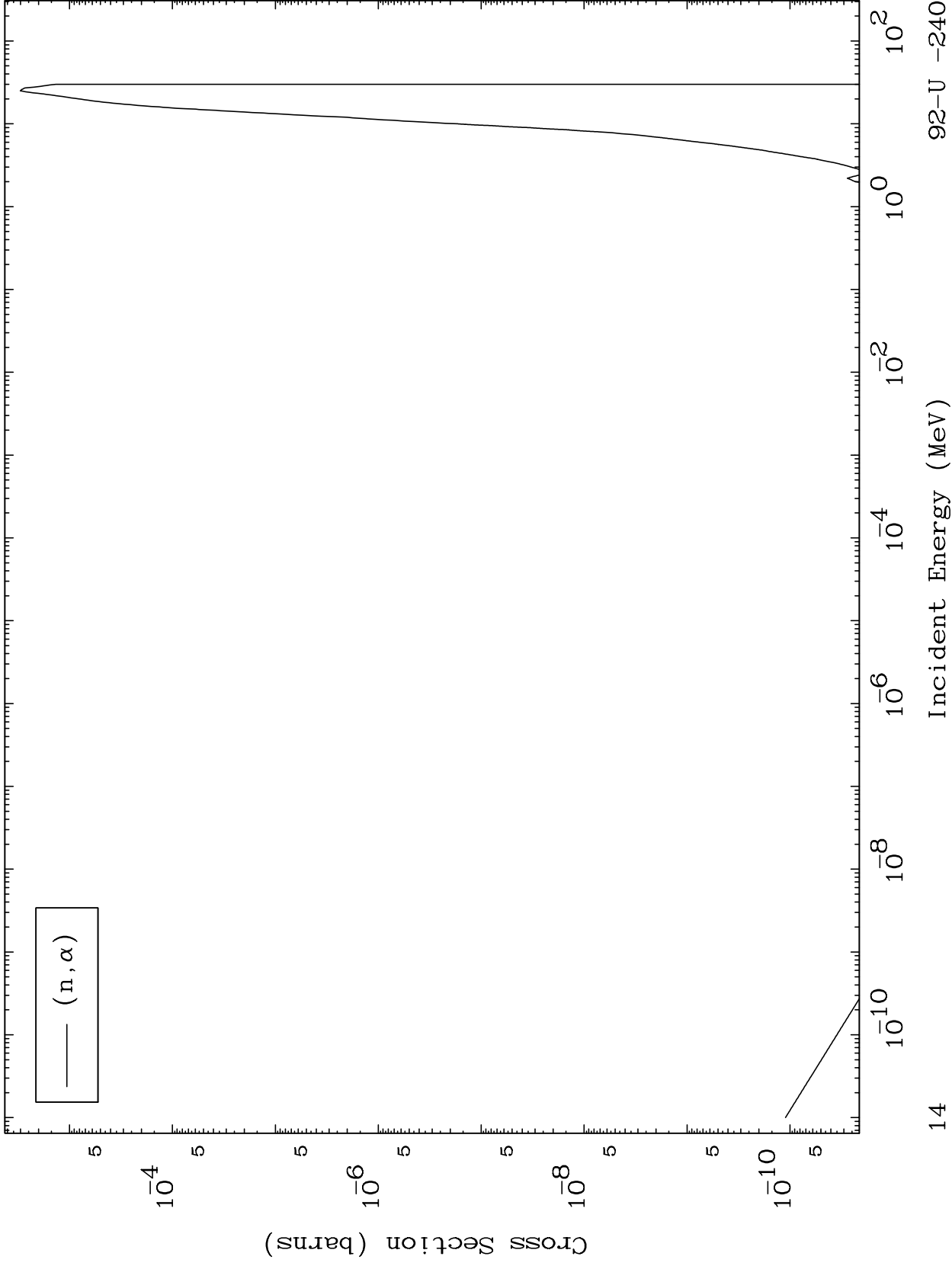
Incident Energy (MeV)

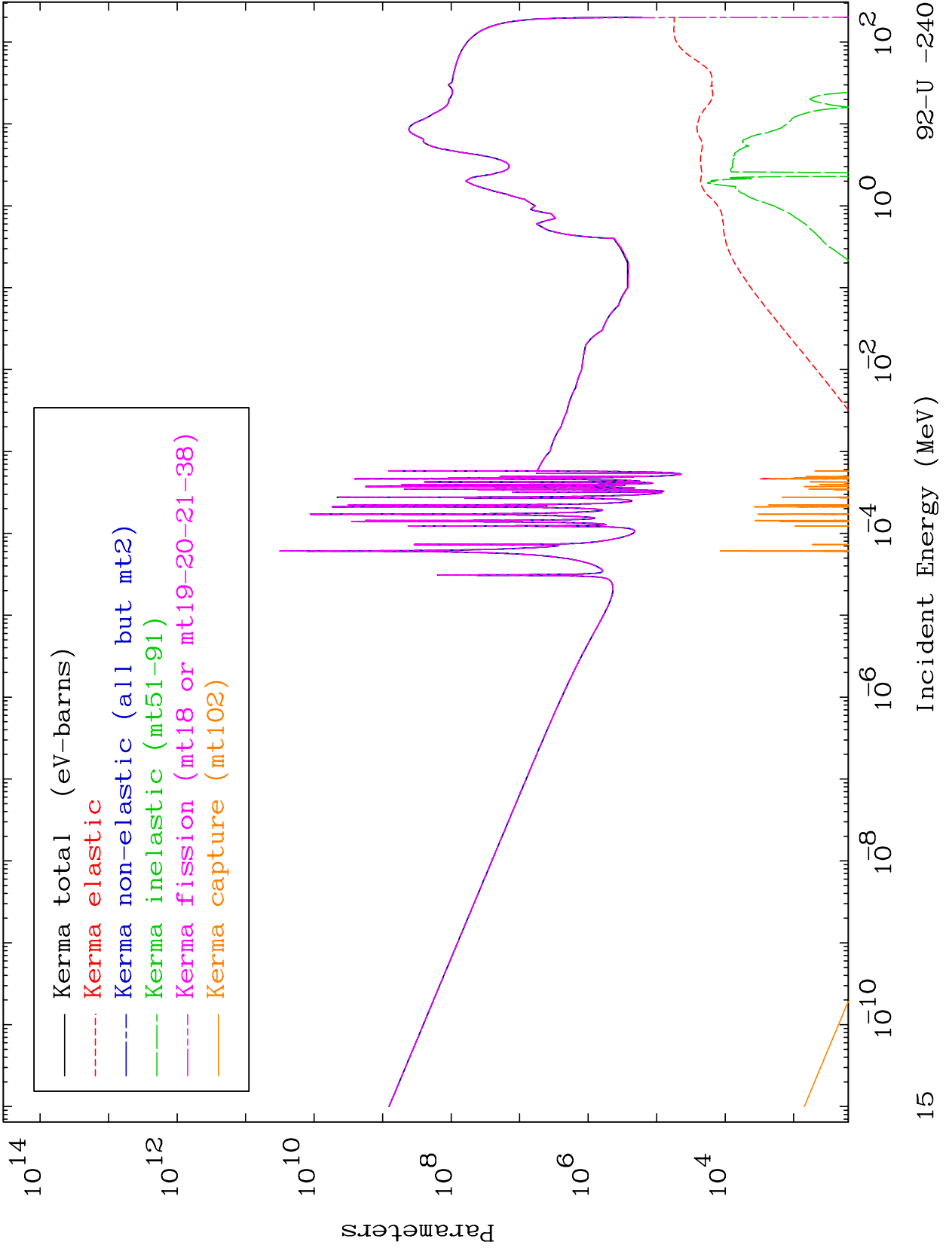
92-U -240

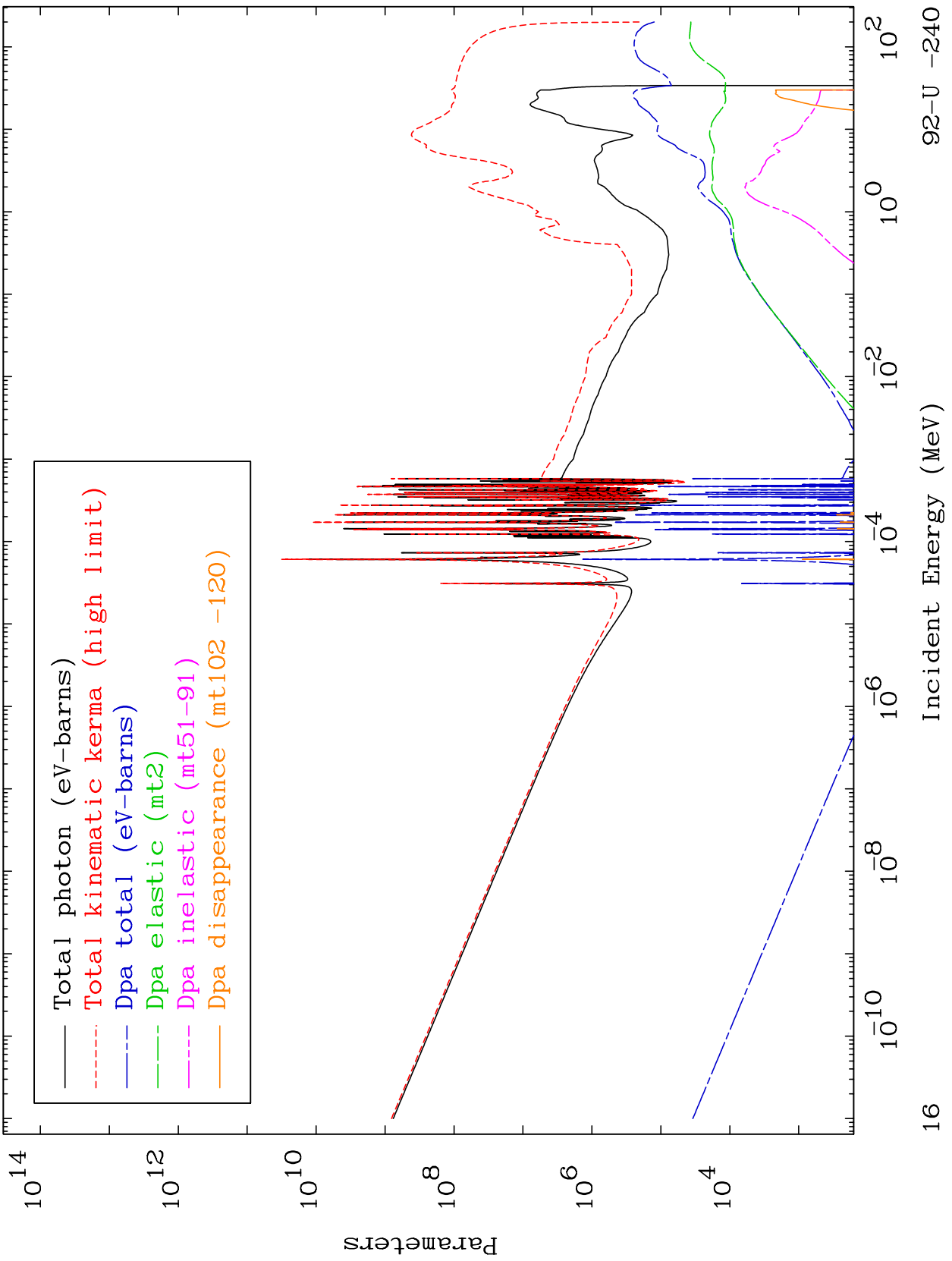
MAT 9243

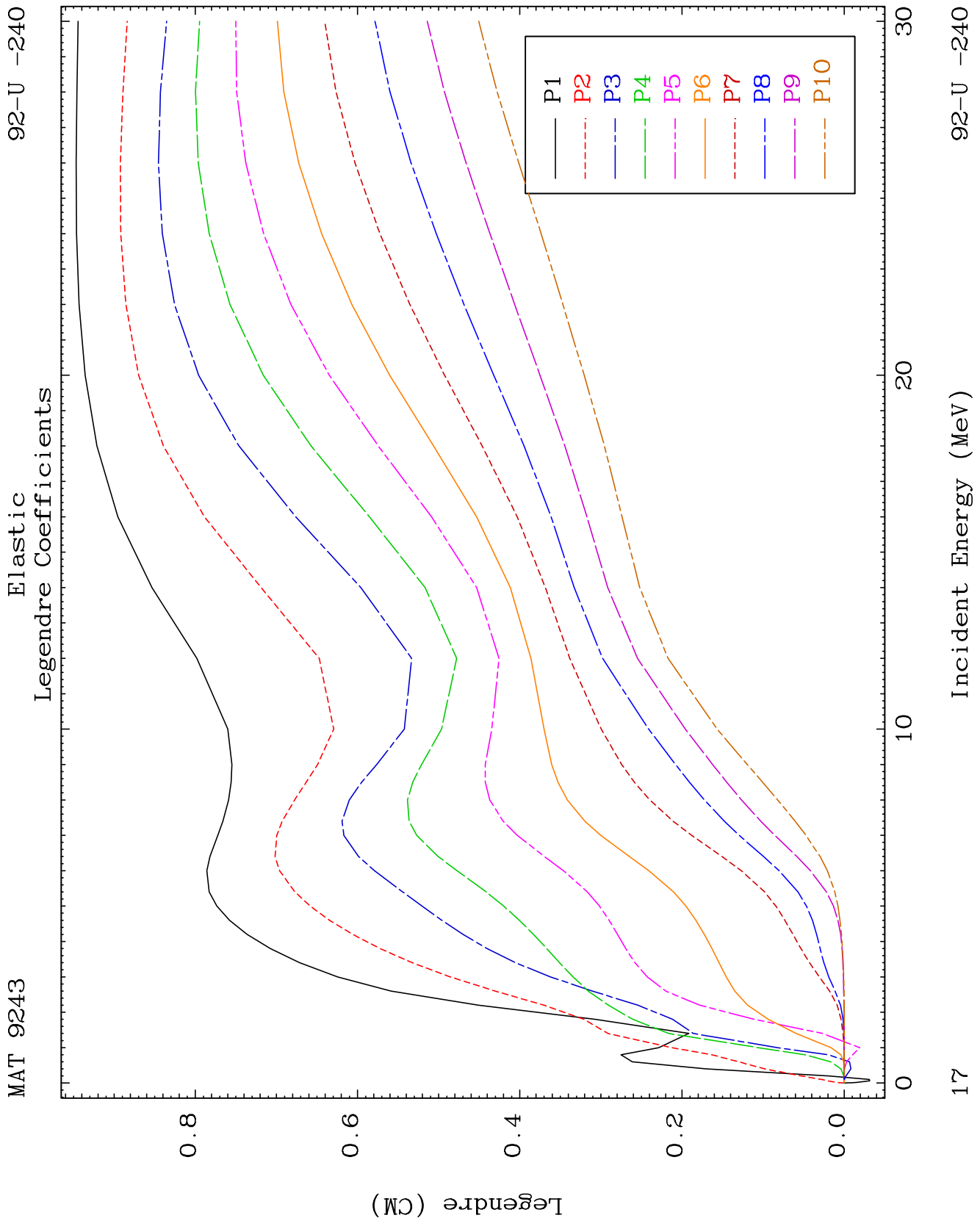
(n, α) Levels
293 Kelvin Cross Sections

92-U -240





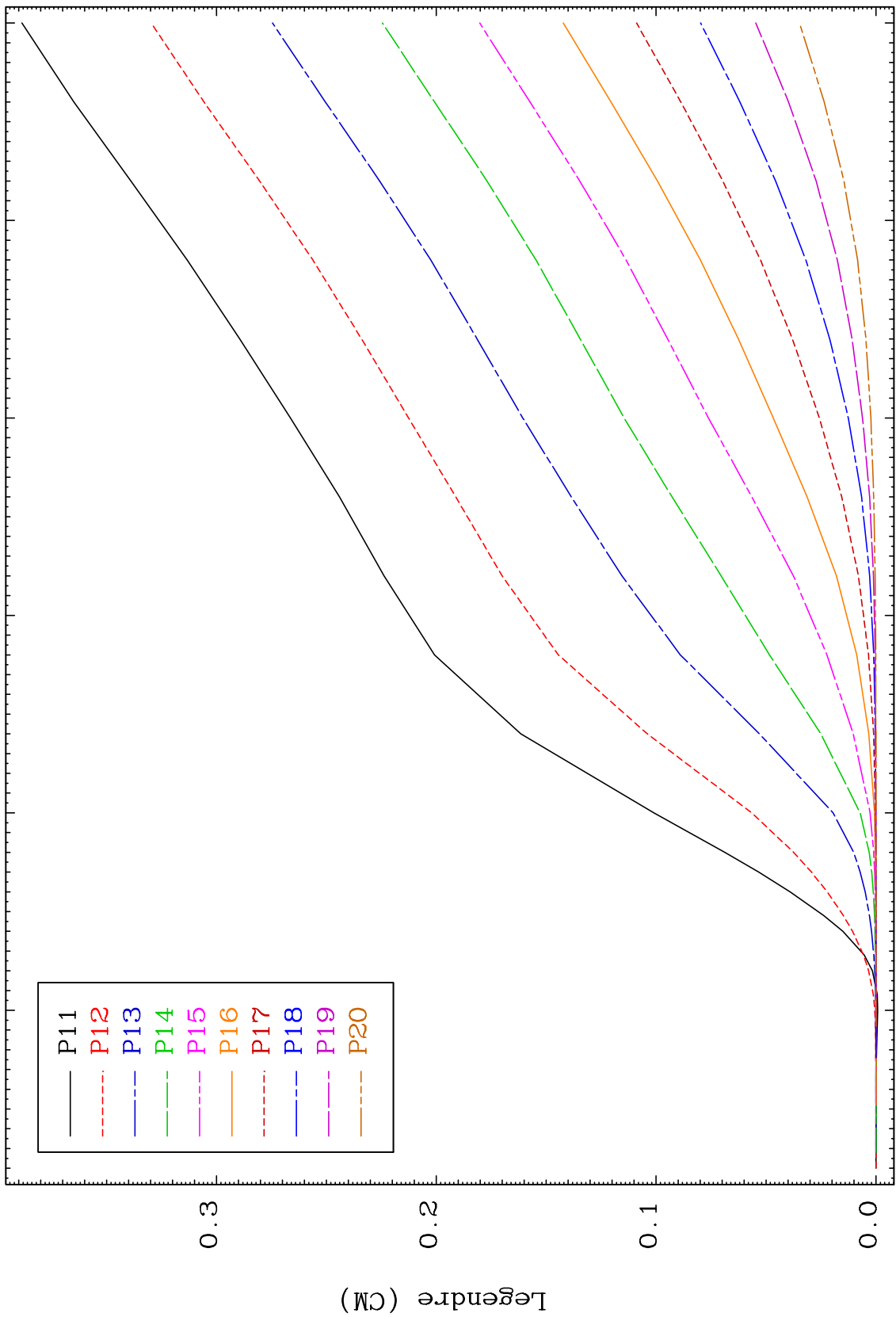




MAT 9243

Elastic Legendre Coefficients

92-U -240



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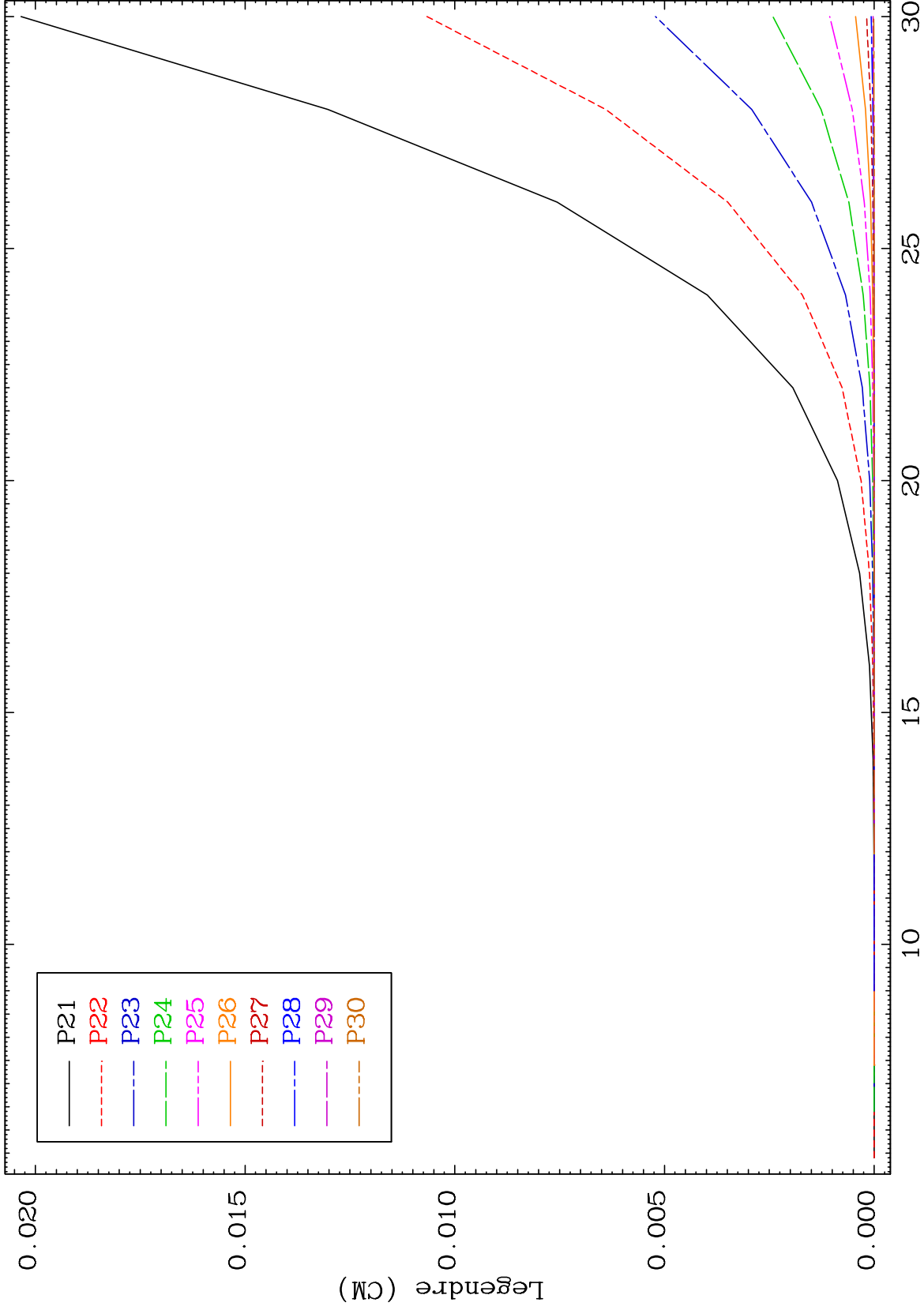
Incident Energy (MeV)

92-U -240

MAT 9243

Elastic
Legendre Coefficients

92-U -240



19

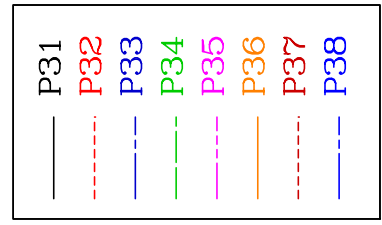
Incident Energy (MeV)

92-U -240

MAT 9243

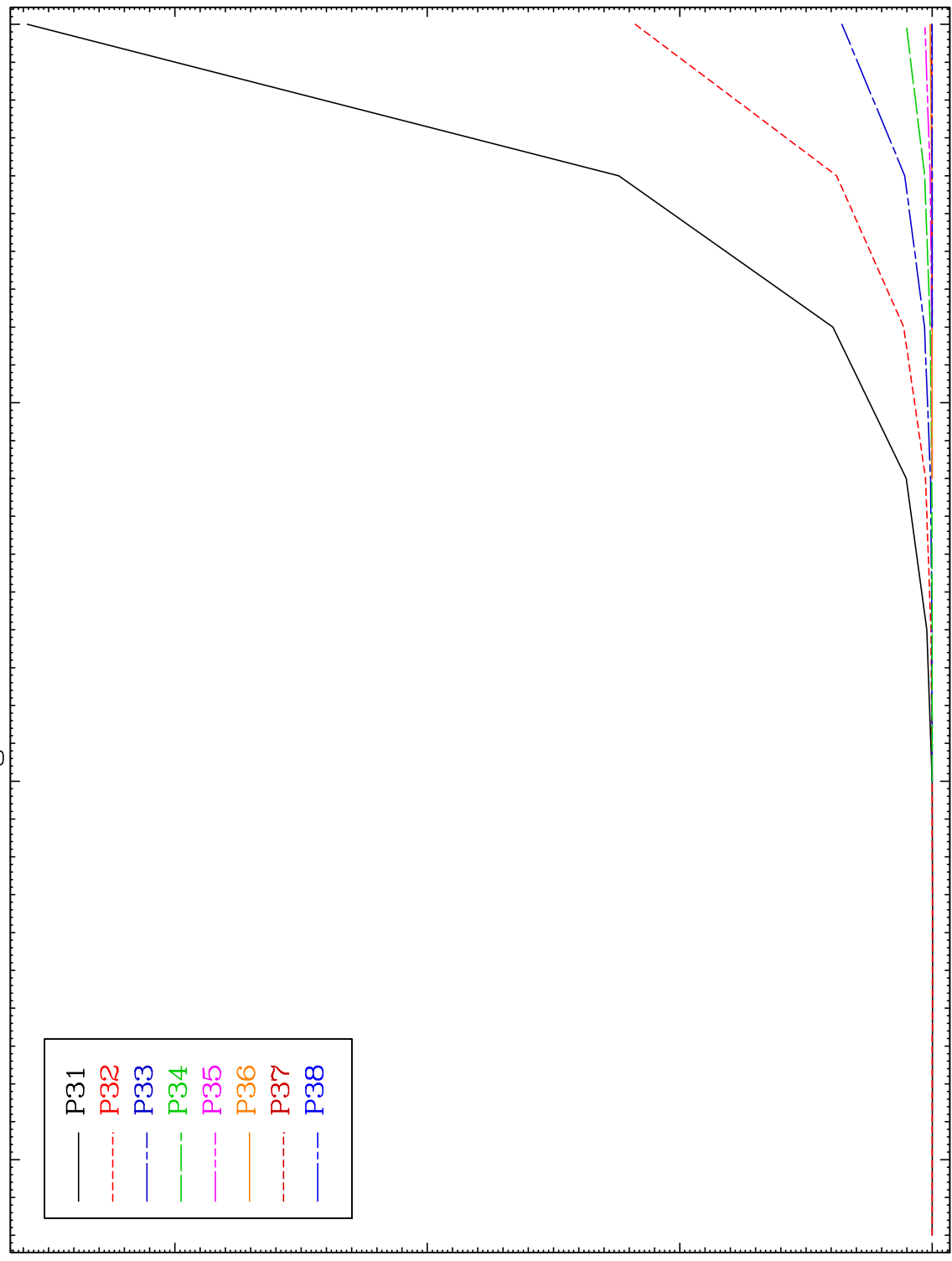
Elastic Legendre Coefficients

92-U -240



$\times 10^{-6}$

Legendre (CM)



20

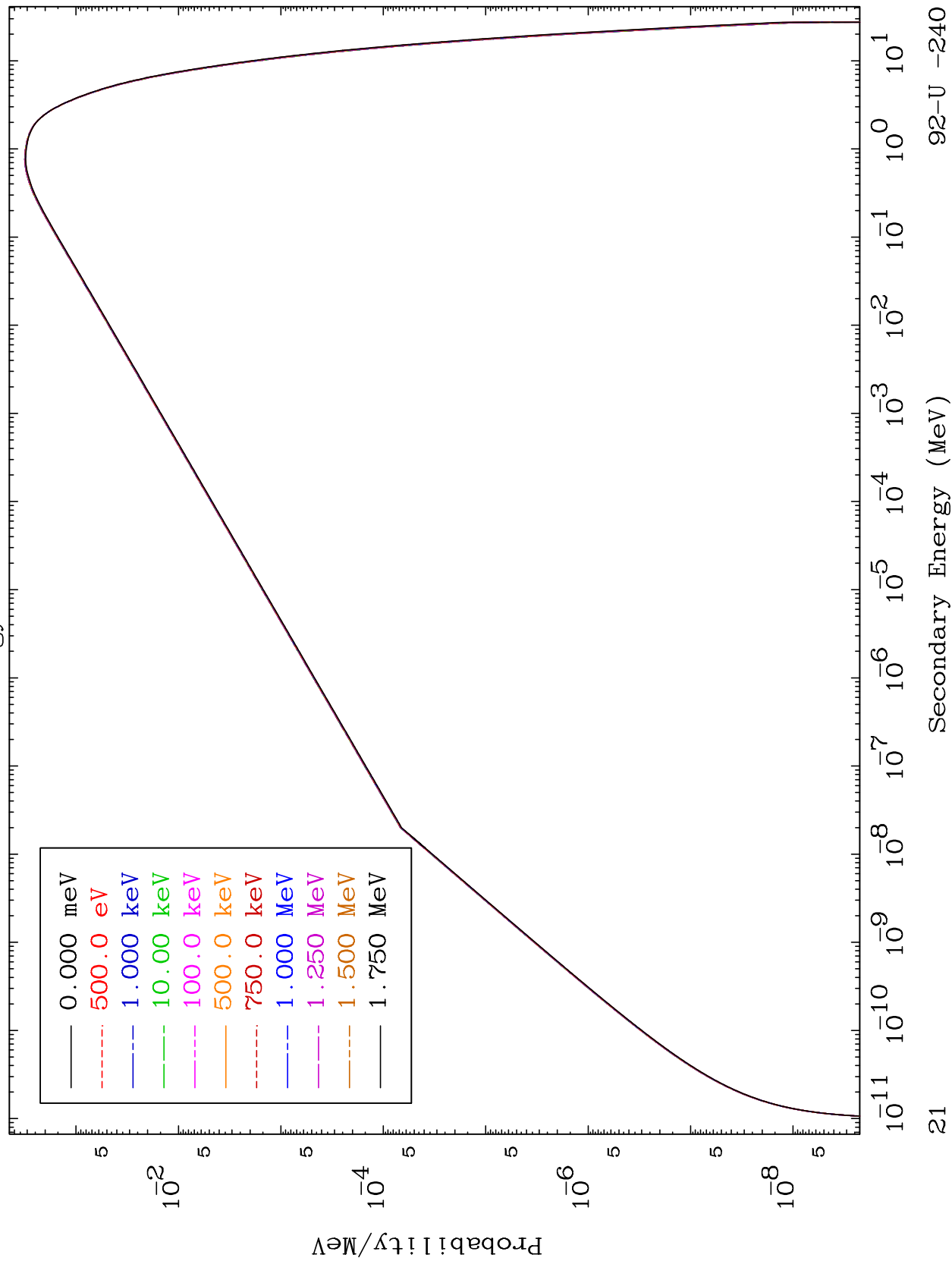
Incident Energy (MeV)

92-U -240

MAT 9243

Fission Energy Distribution

92-U -240

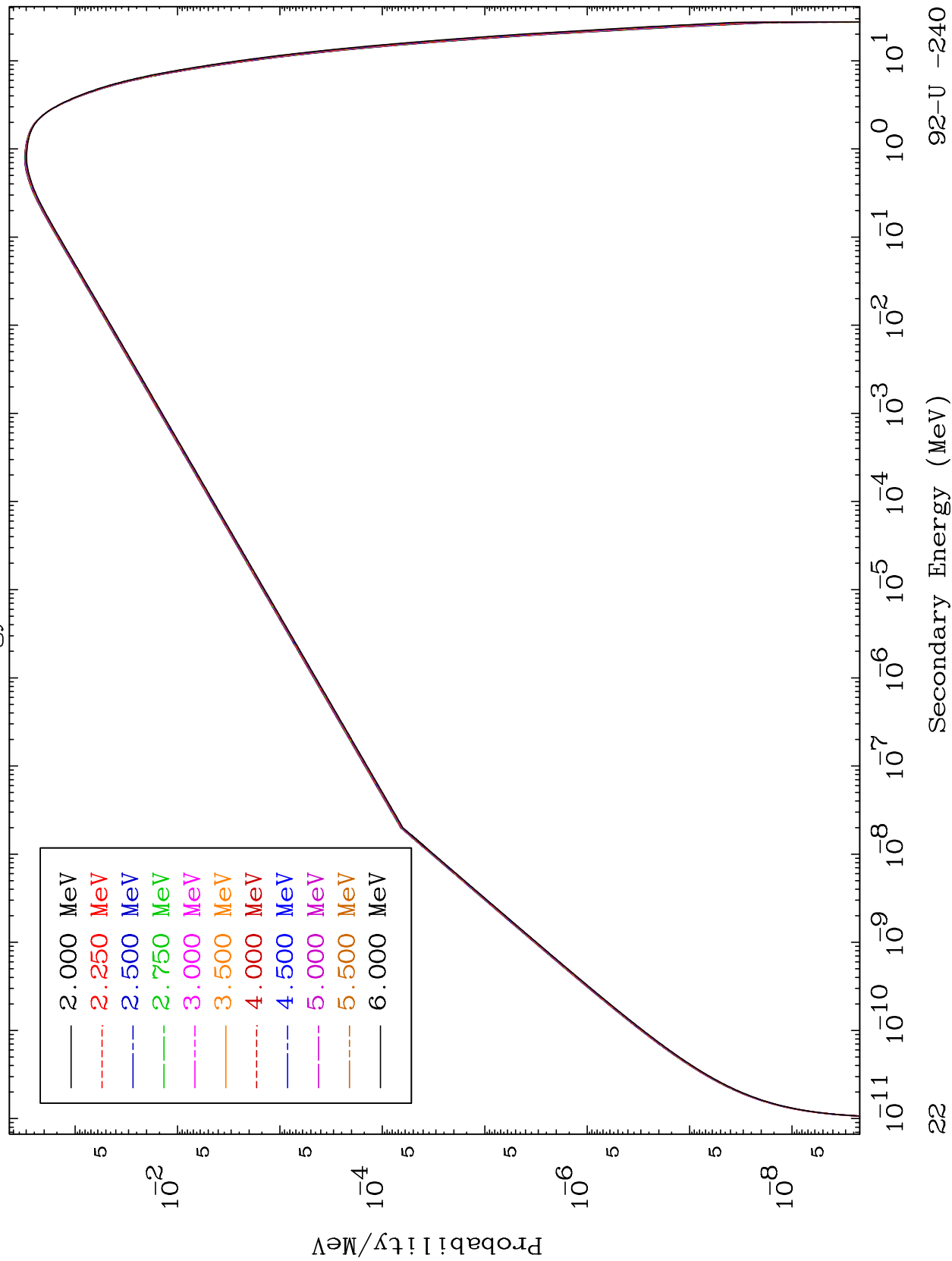


92-U -240

MAT 9243

Fission Energy Distribution

92-U -240



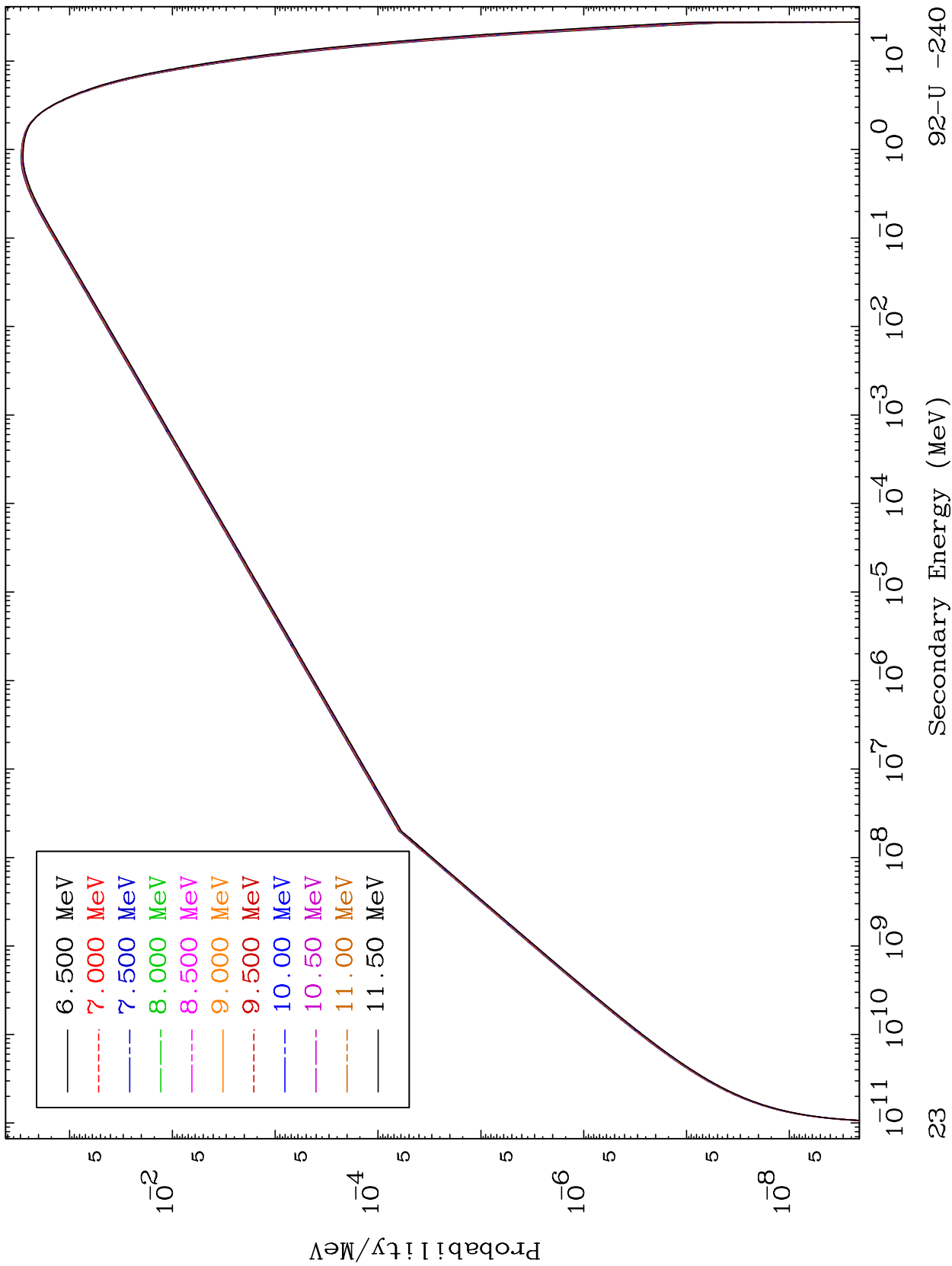
22

92-U -240

MAT 9243

Fission
Energy Distribution

92-U -240

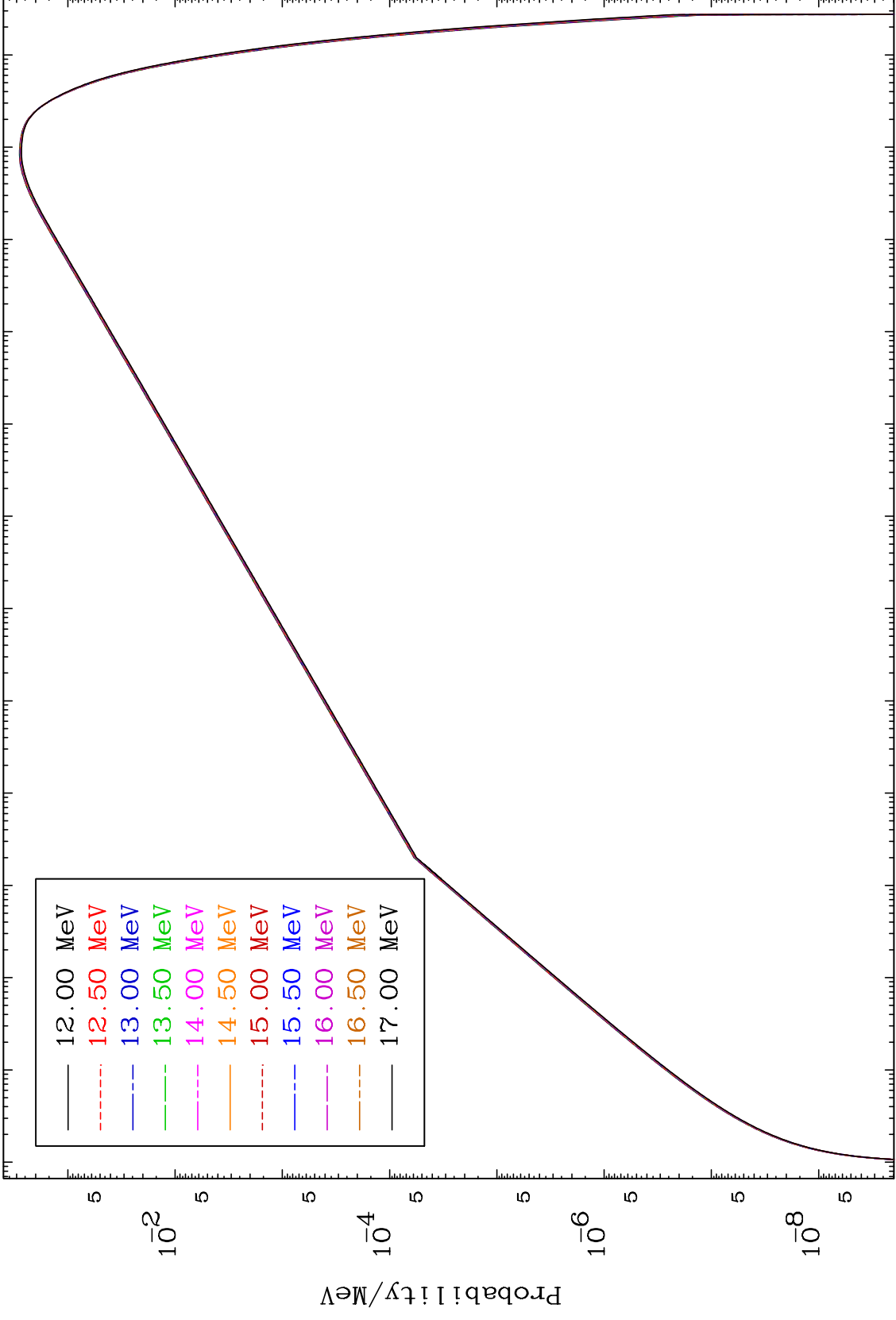


MAT 9243

Fission

92-U -240

Energy Distribution



10⁻¹¹ 10⁻¹⁰ 10⁻⁹ 10⁻⁸ 10⁻⁷ 10⁻⁶ 10⁻⁵ 10⁻⁴ 10⁻³ 10⁻² 10⁻¹ 10⁰ 10¹

92-U -240

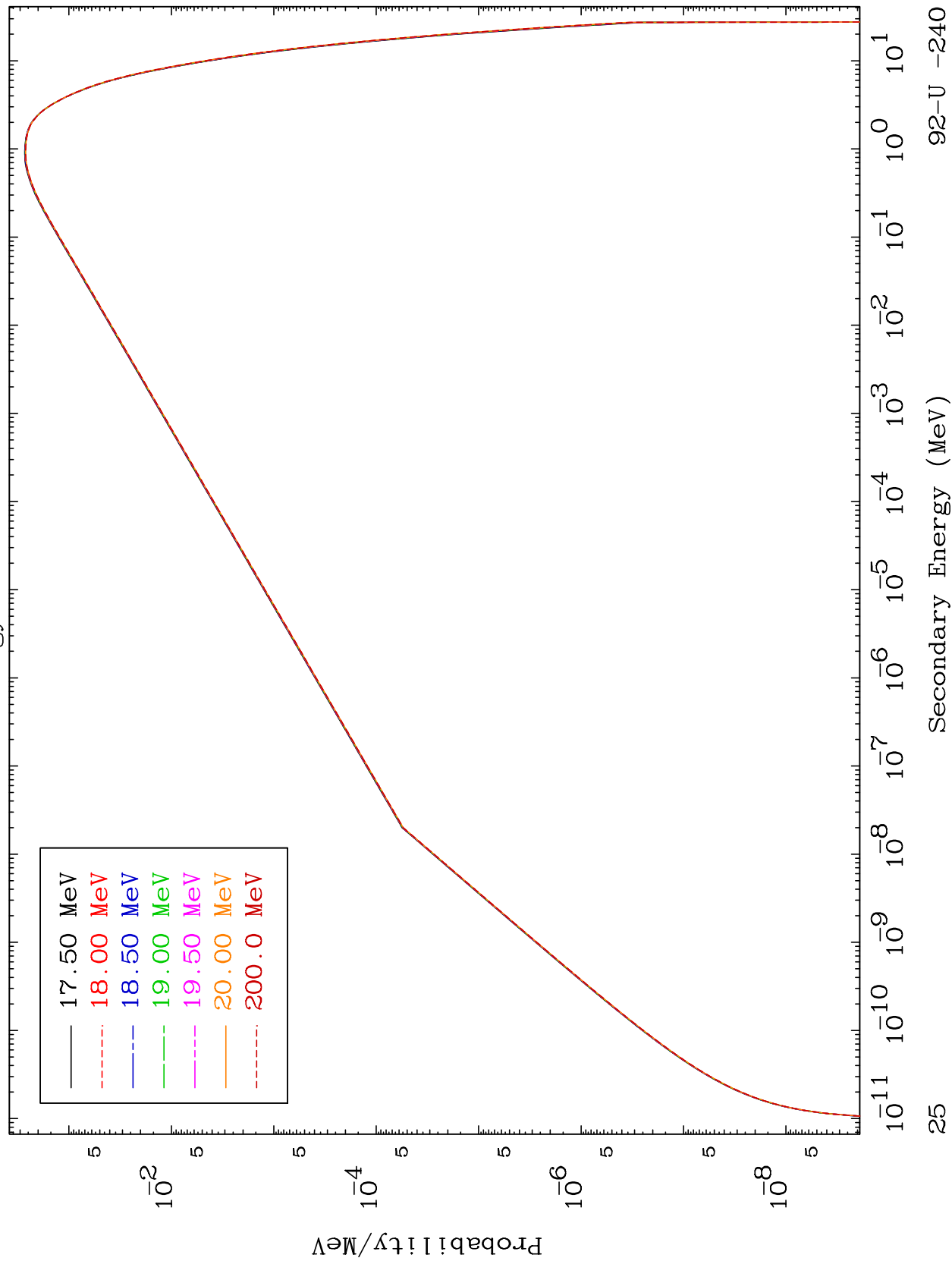
Secondary Energy (MeV)

24

MAT 9243

Fission Energy Distribution

92-U -240



25

92-U -240