

Program EVALPLOT  
(Version 2018-1)

by

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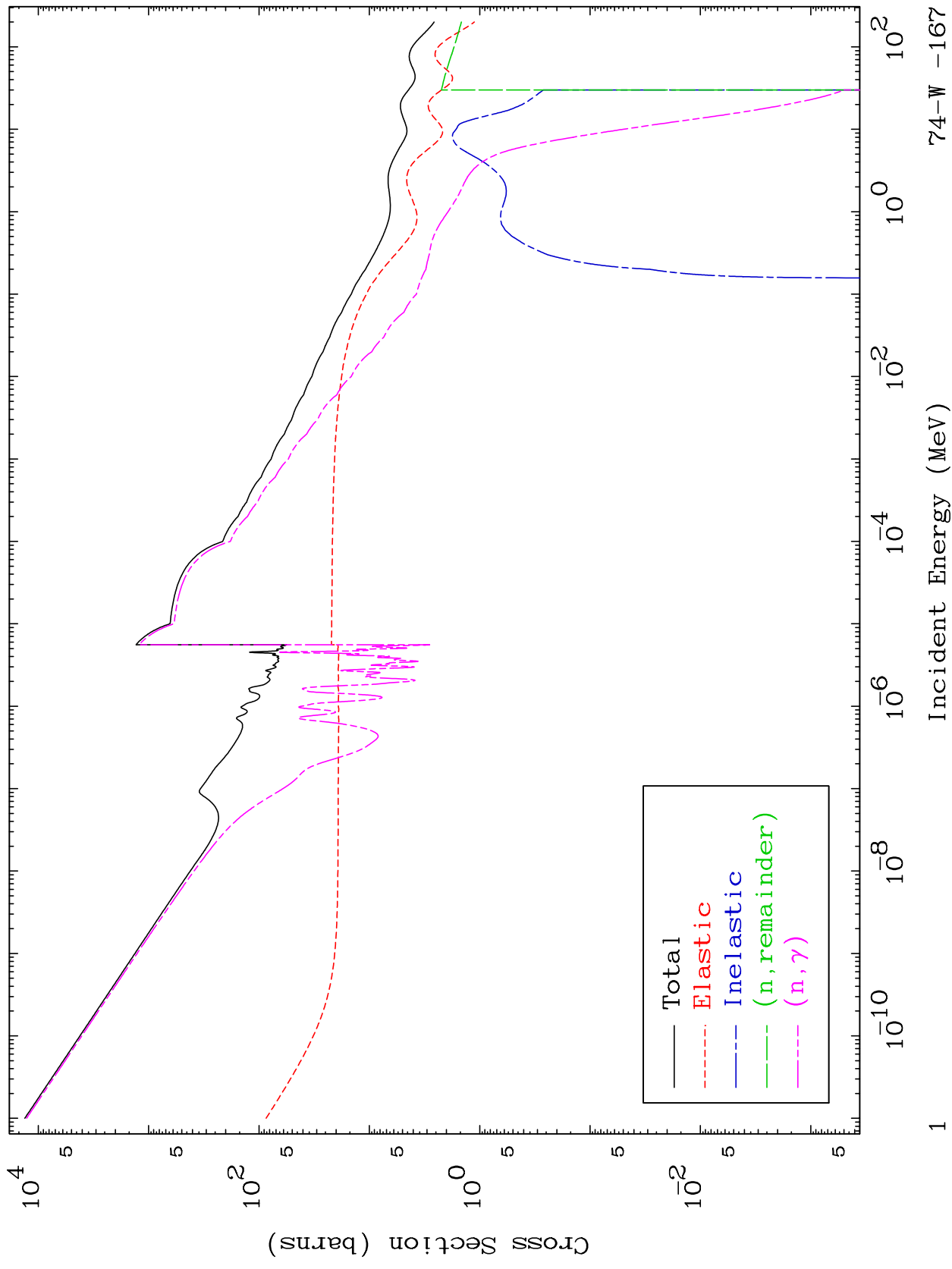
Press Mouse Button to Start

MAT 7386

Major

293 Kelvin Cross Sections

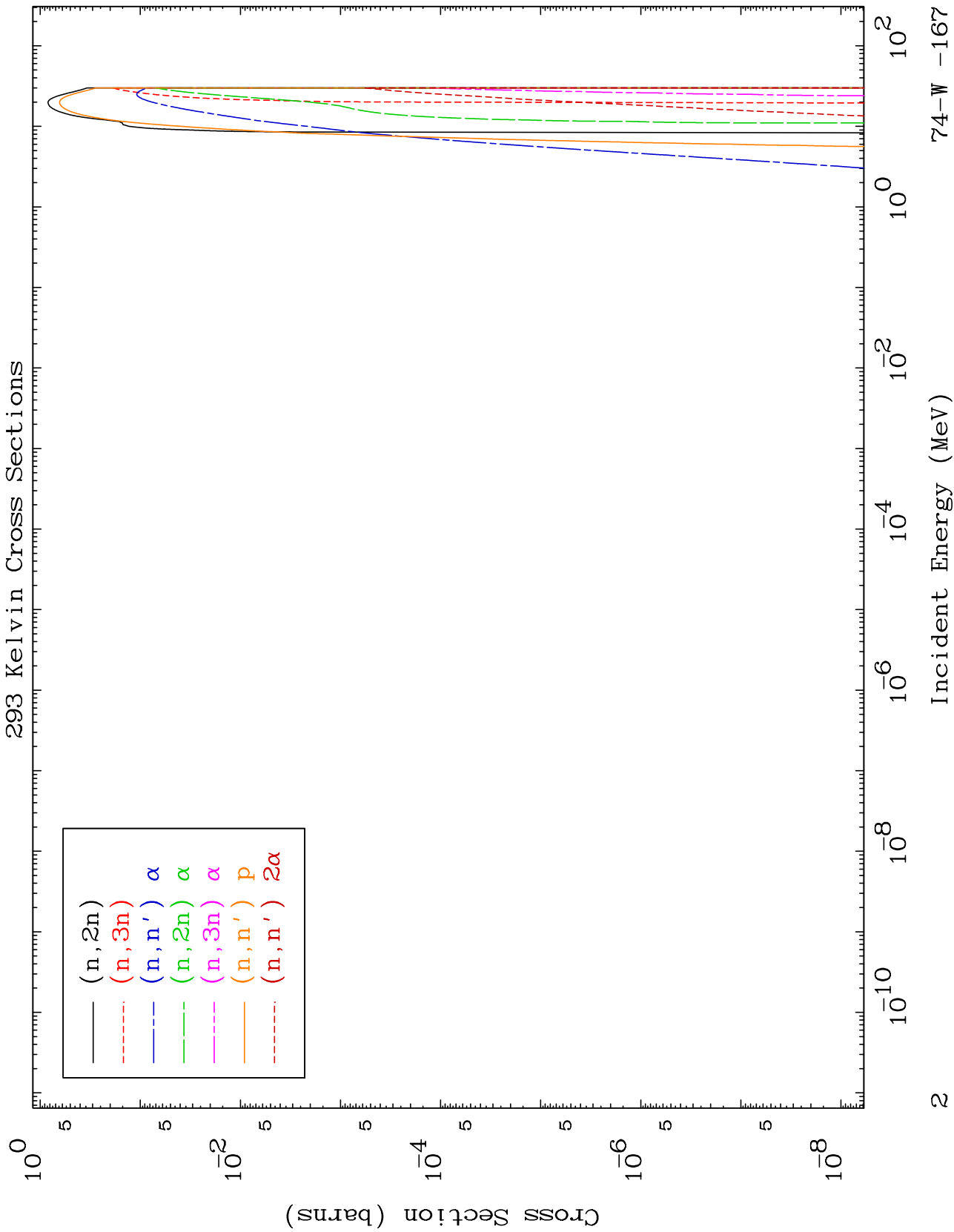
74-W -167



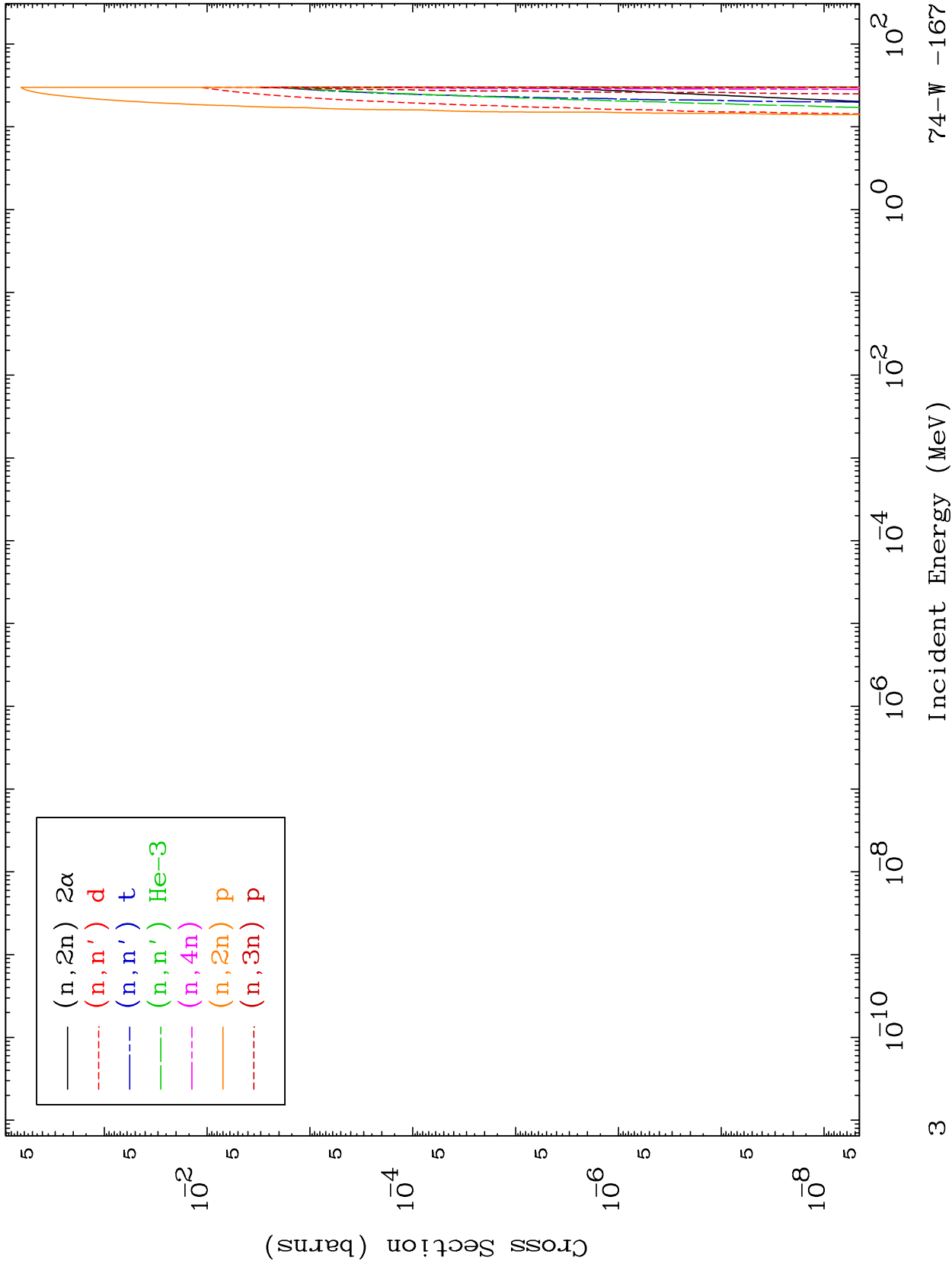
MAT 7386

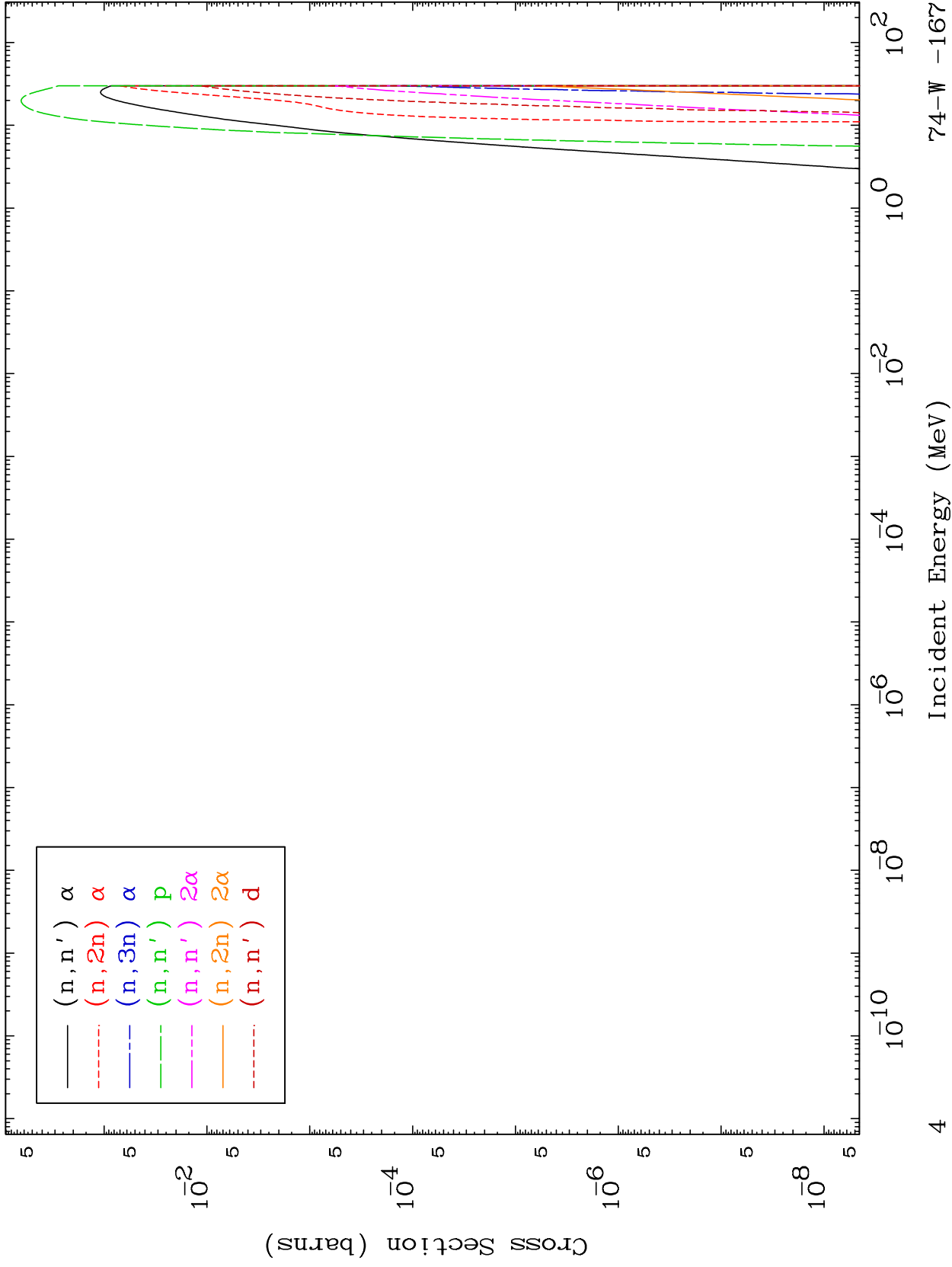
Neutron Production  
293 Kelvin Cross Sections

74-W -167

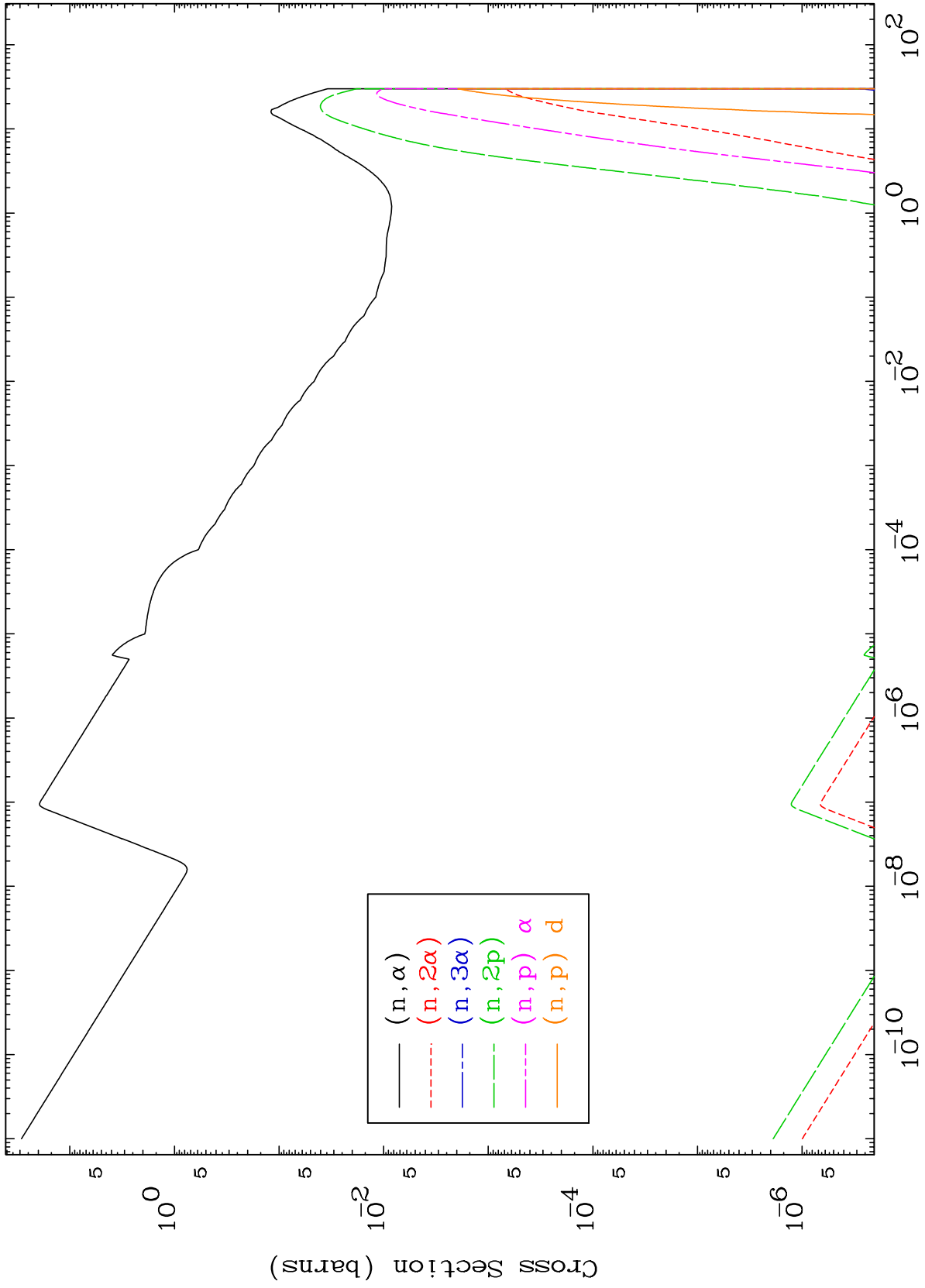


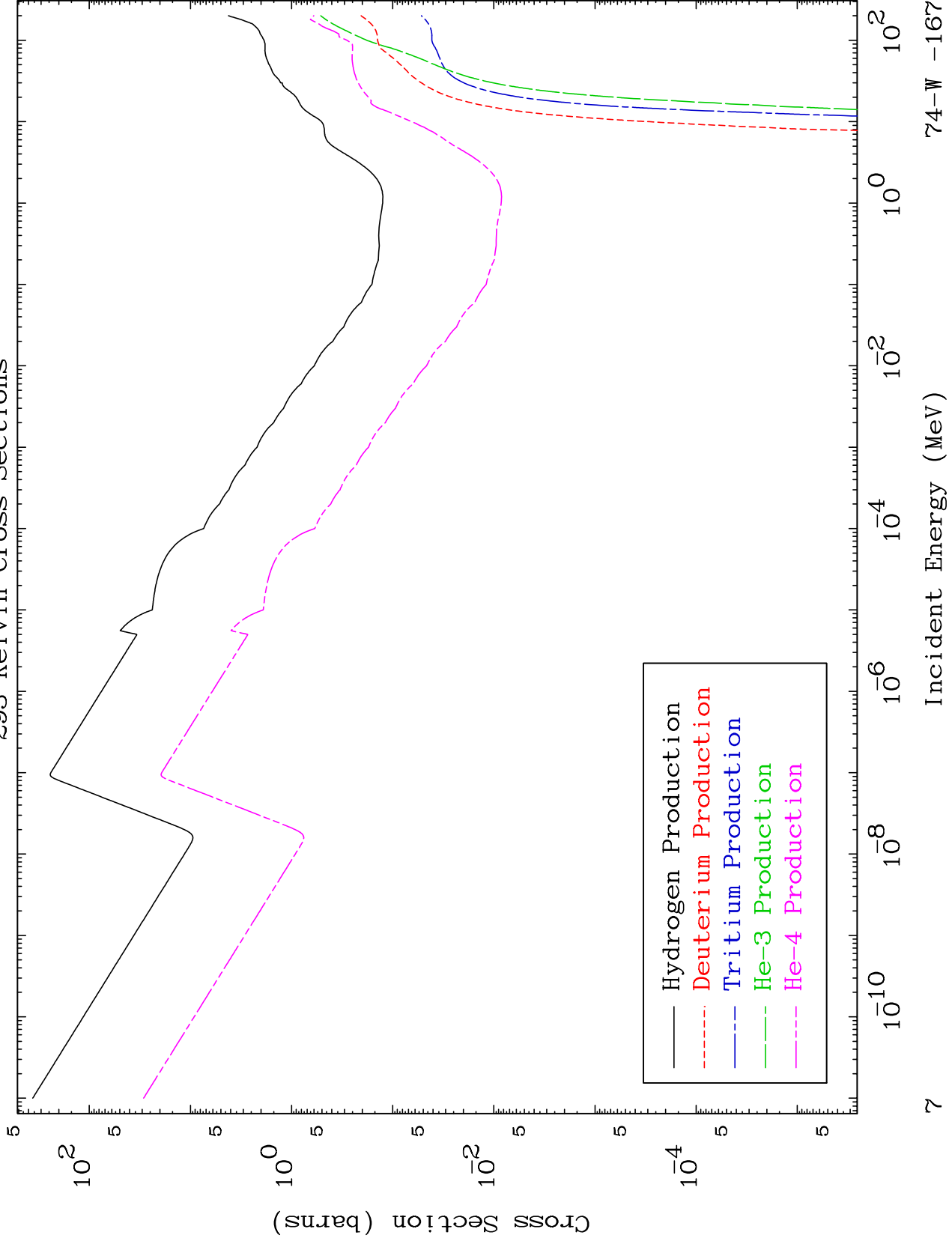
74-W -167



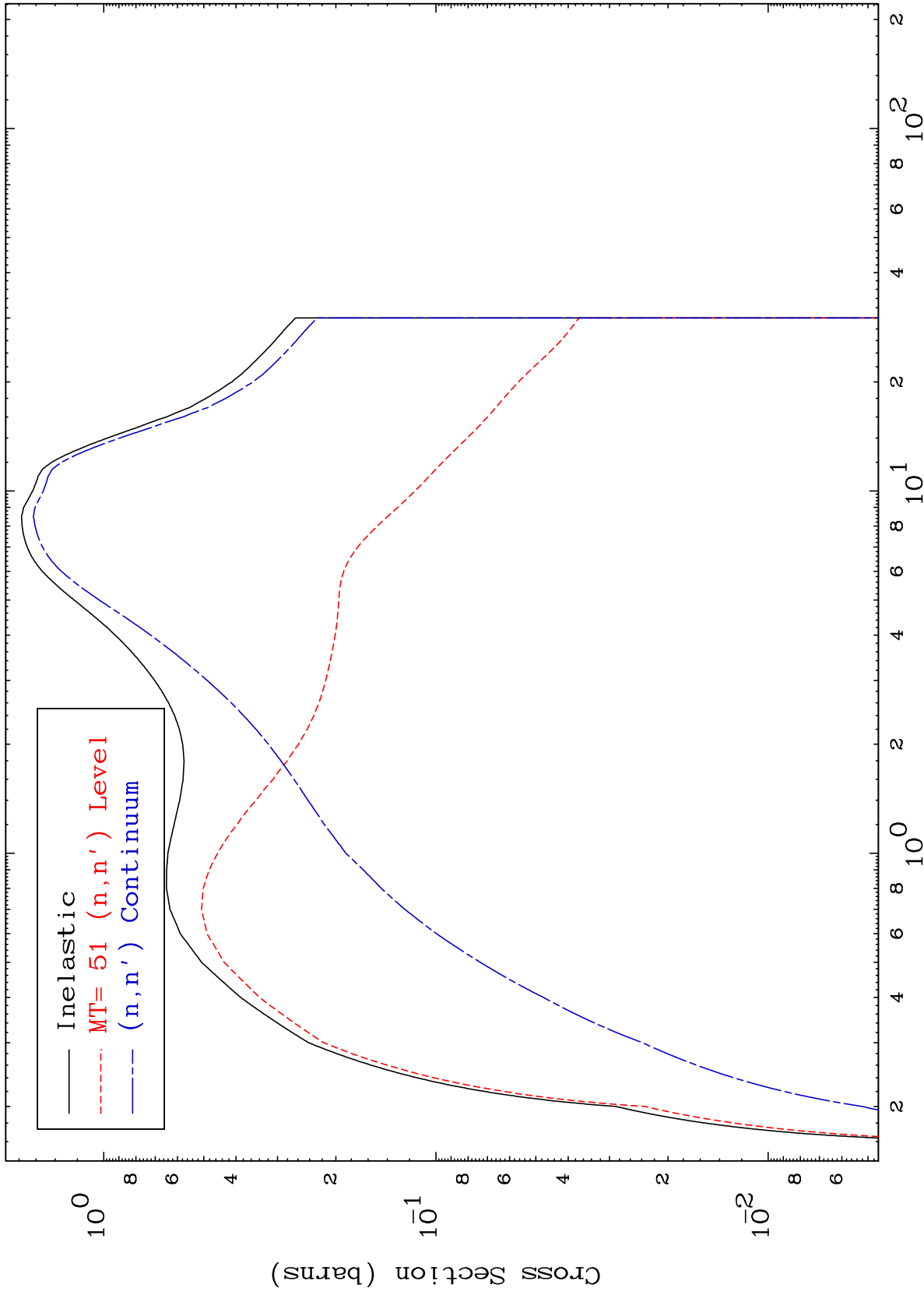








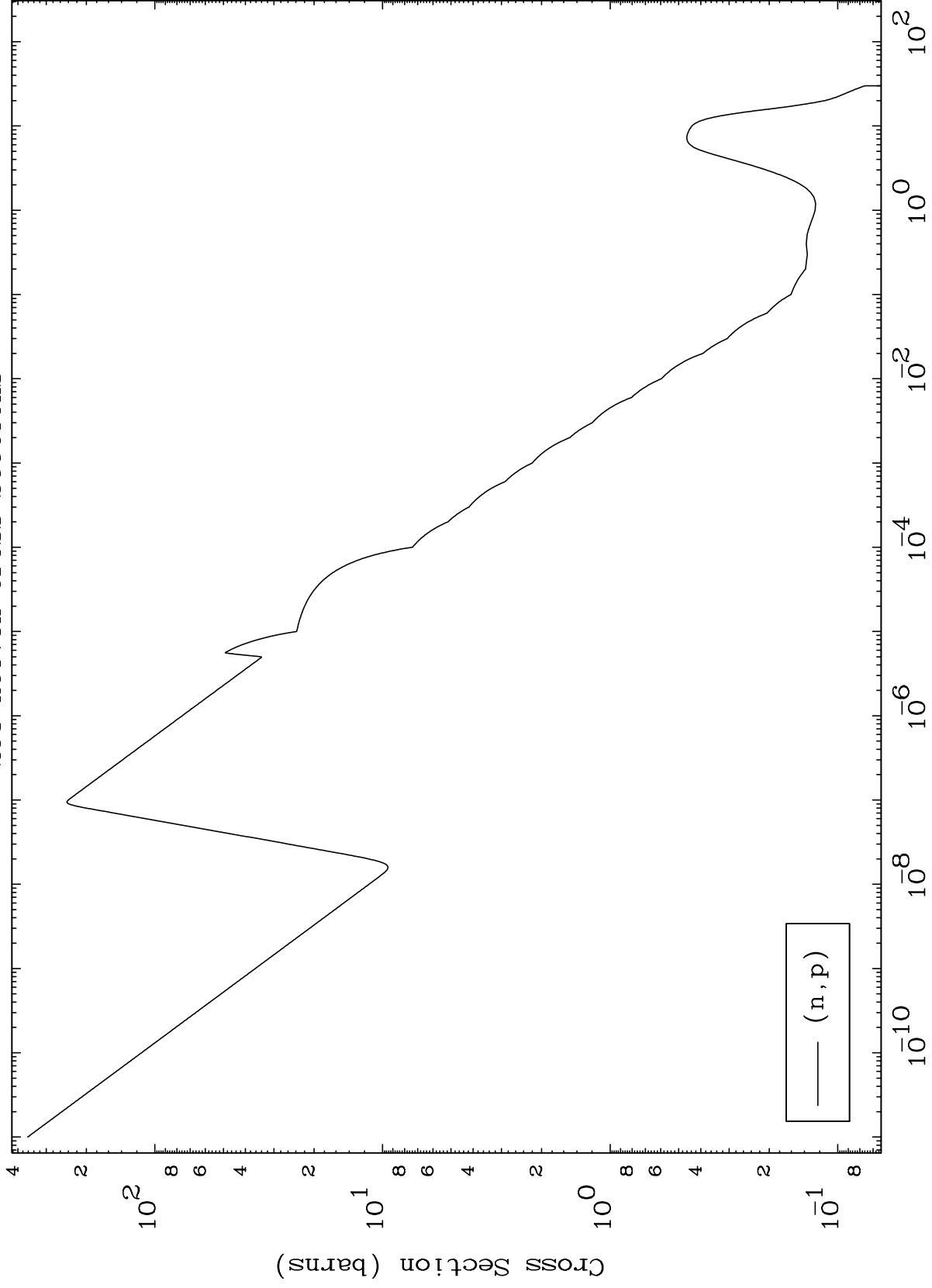




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(n,p) Levels  
293 Kelvin Cross Sections

74-W -167

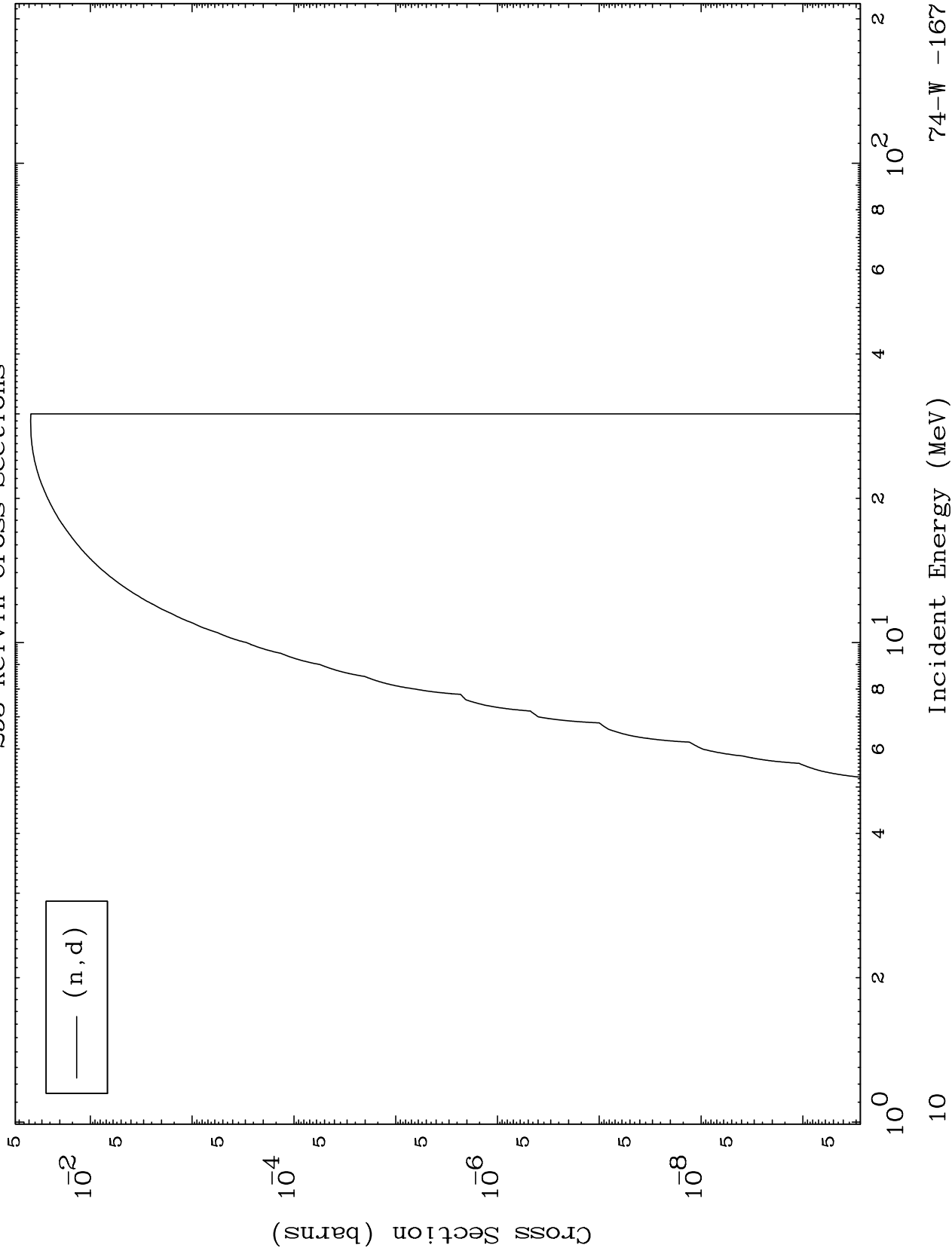


— (n,p)

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(n,d) Levels  
293 Kelvin Cross Sections

74-W -167

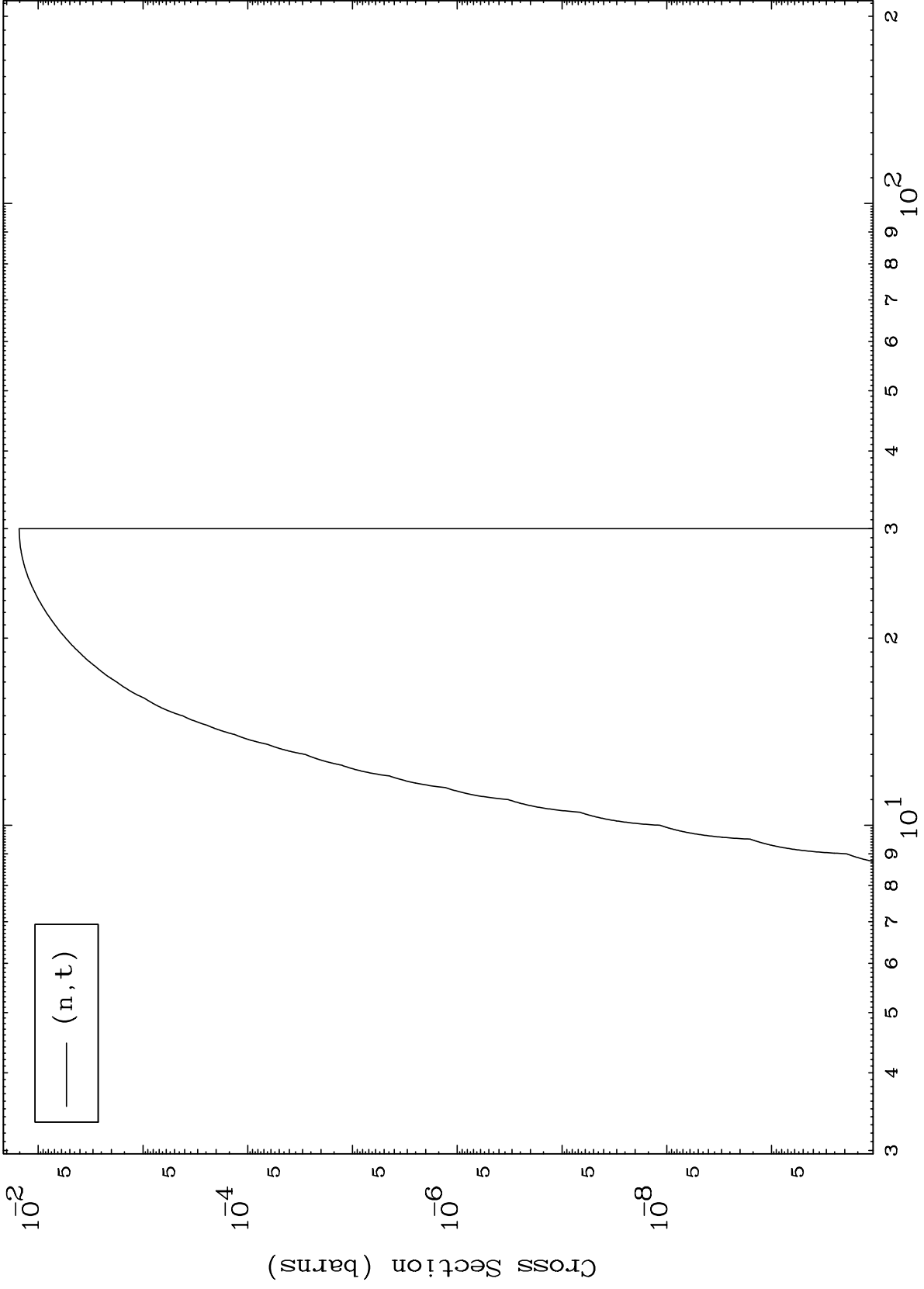


74-W -167

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(n,t) Levels  
293 Kelvin Cross Sections

74-W -167



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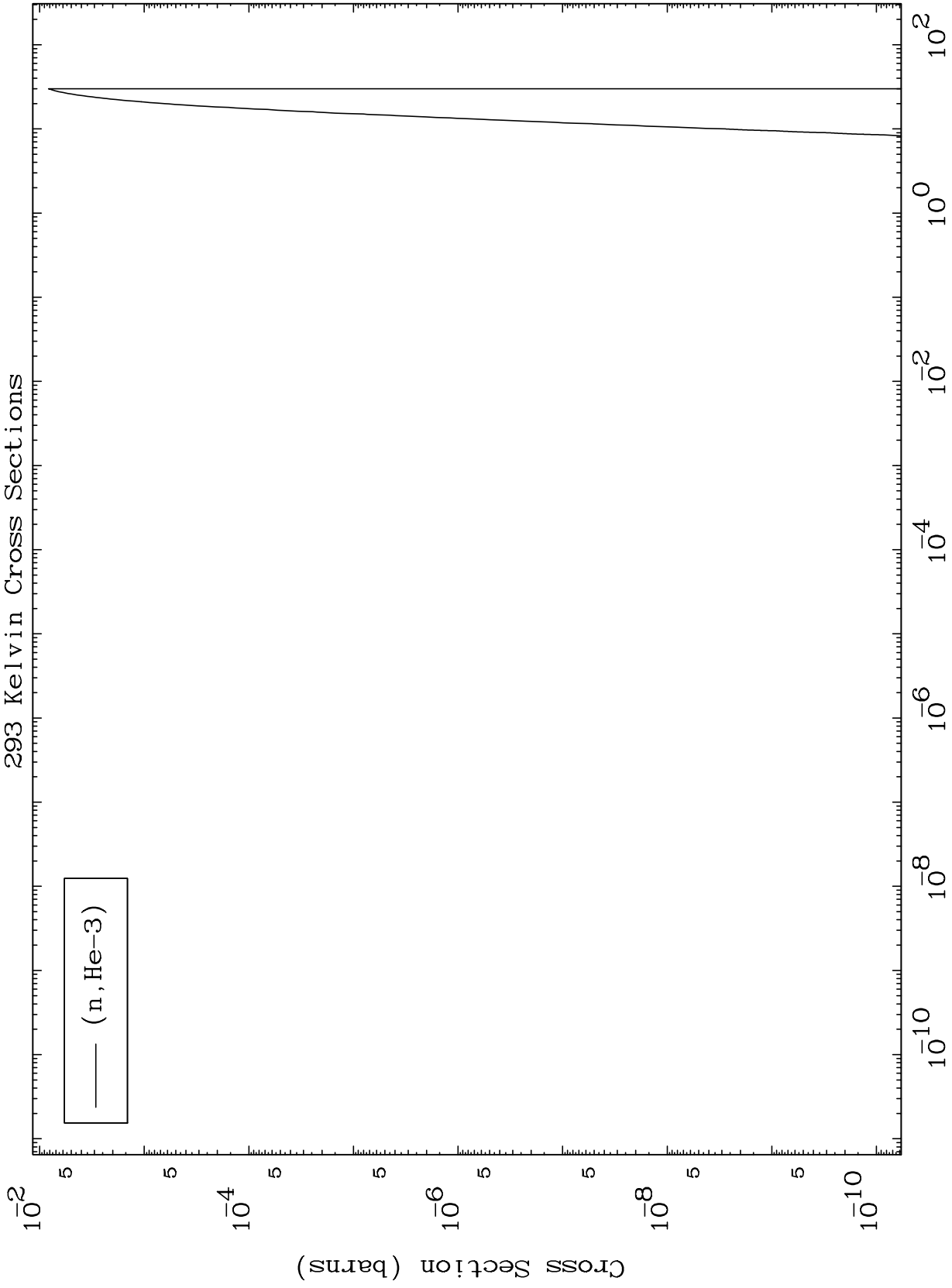
Incident Energy (MeV)

74-W -167

MAT 7386

(n,He3) Levels  
293 Kelvin Cross Sections

74-W -167



12

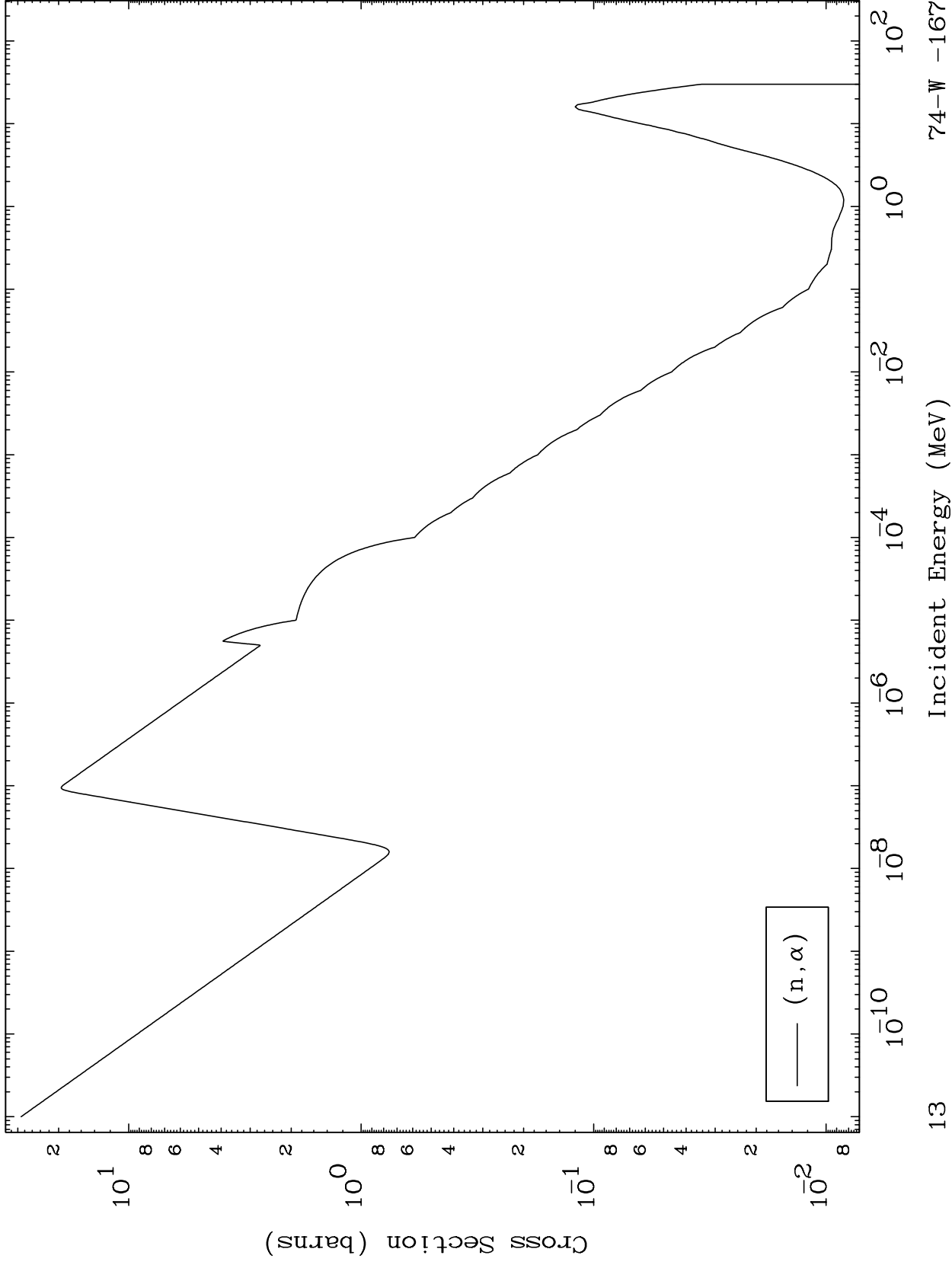
Incident Energy (MeV)

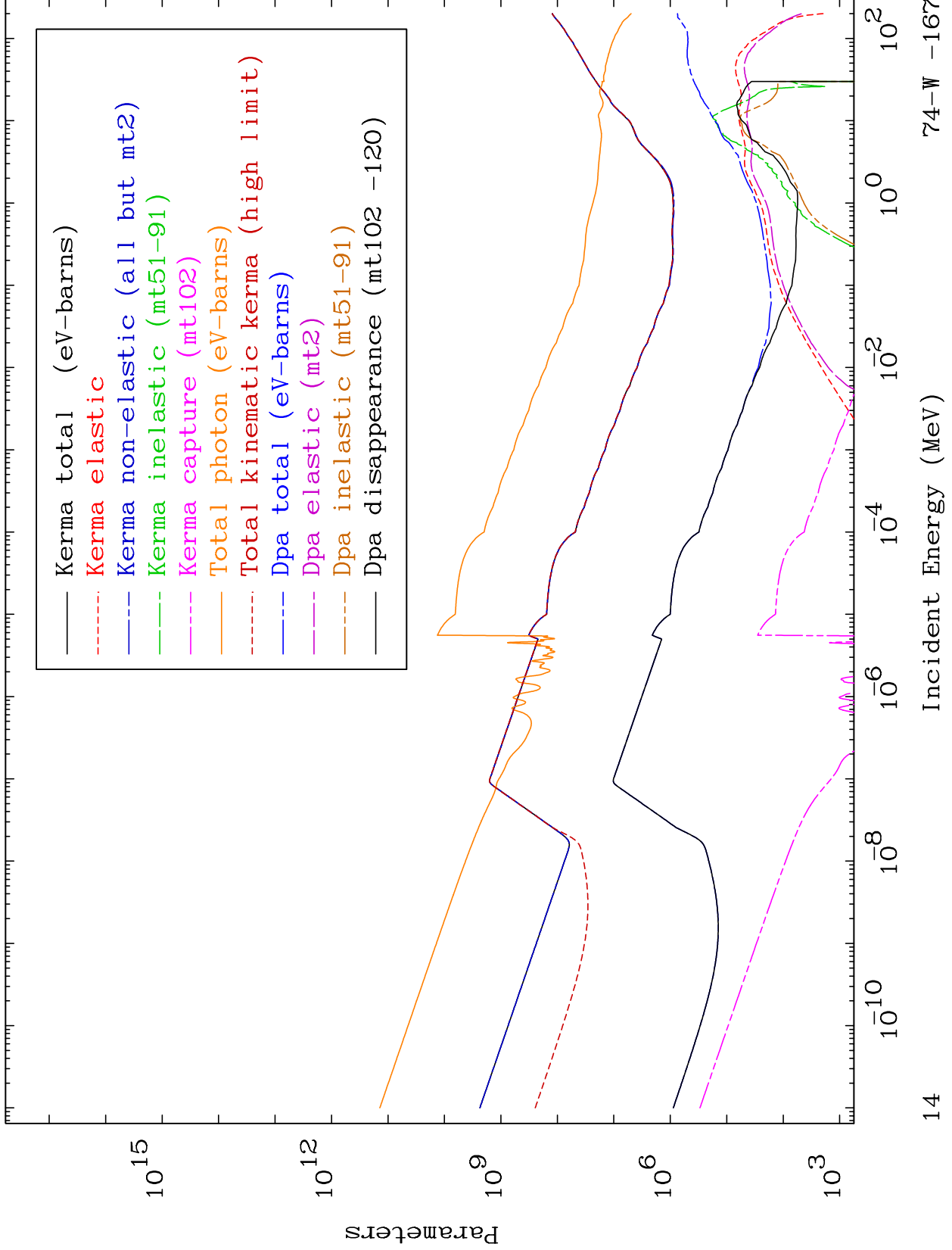
74-W -167

MAT 7386

(n,α) Levels  
293 Kelvin Cross Sections

74-W -167

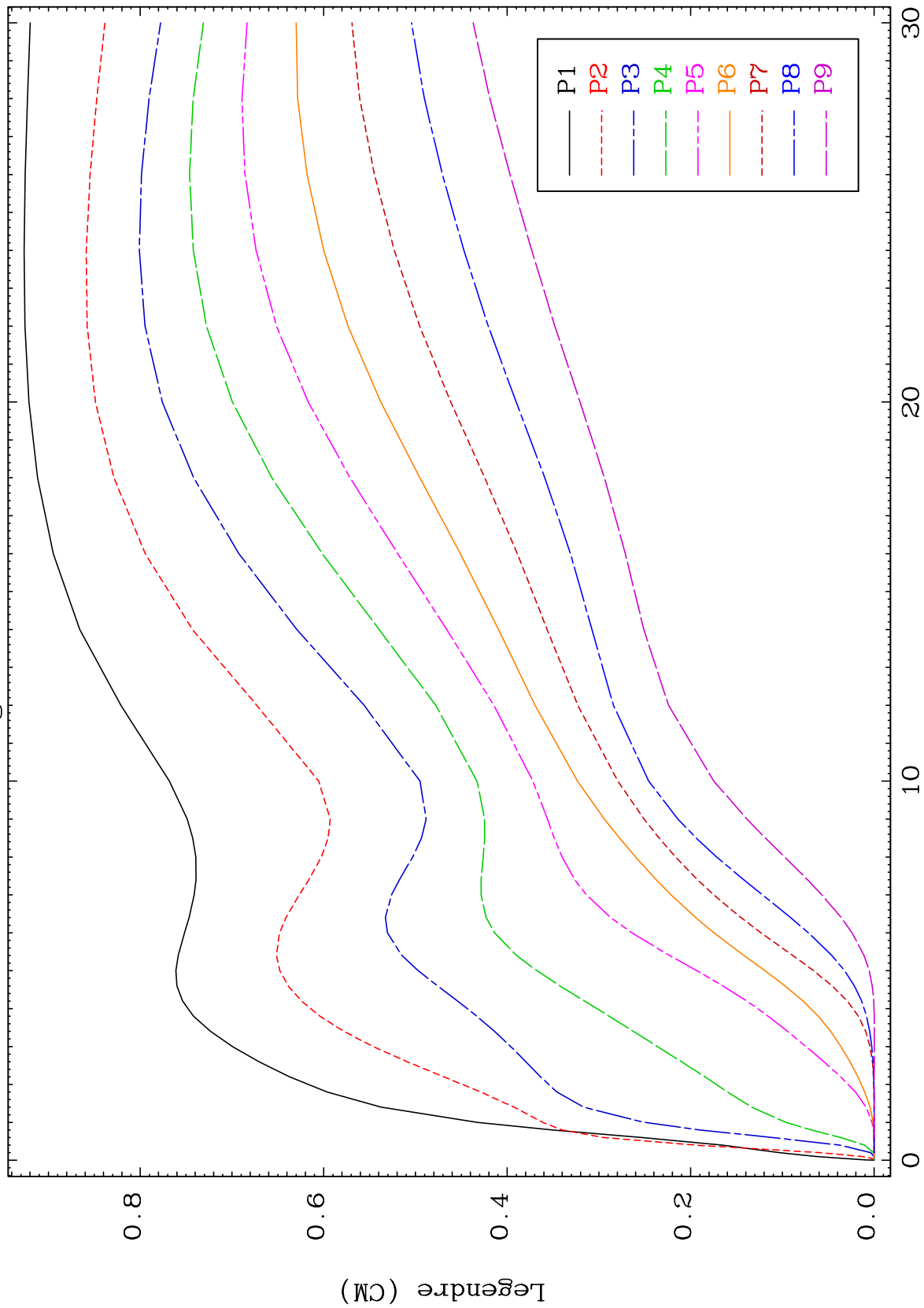




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Elastic Legendre Coefficients

74-W -167



15

Incident Energy (MeV)

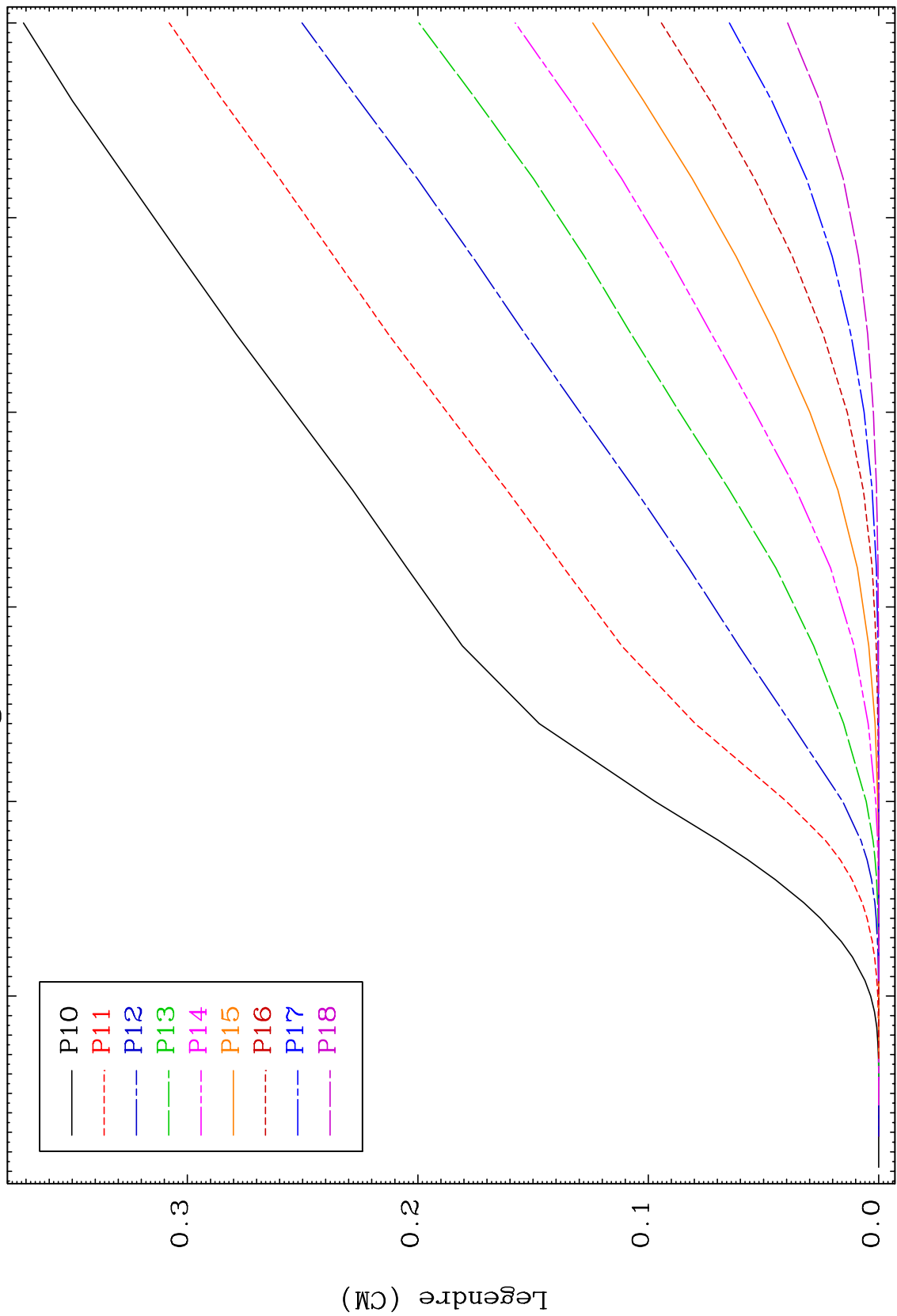
74-W -167



MAT 7386

Elastic Legendre Coefficients

74-W -167



16

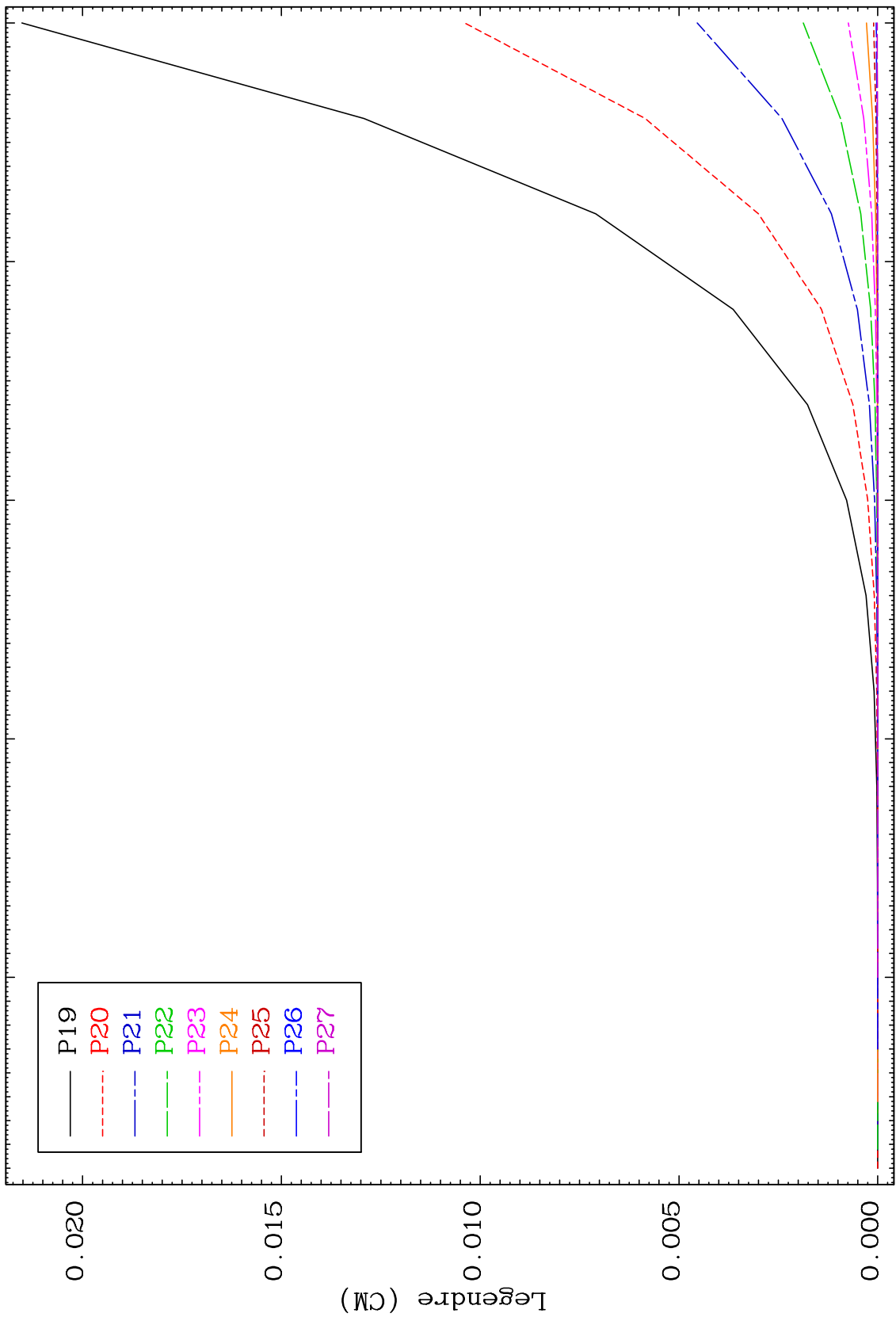
Incident Energy (MeV)

74-W -167

MAT 7386

Elastic Legendre Coefficients

74-W -167



17

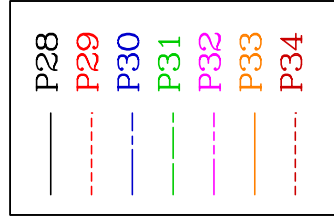
Incident Energy (MeV)

74-W -167

MAT 7386

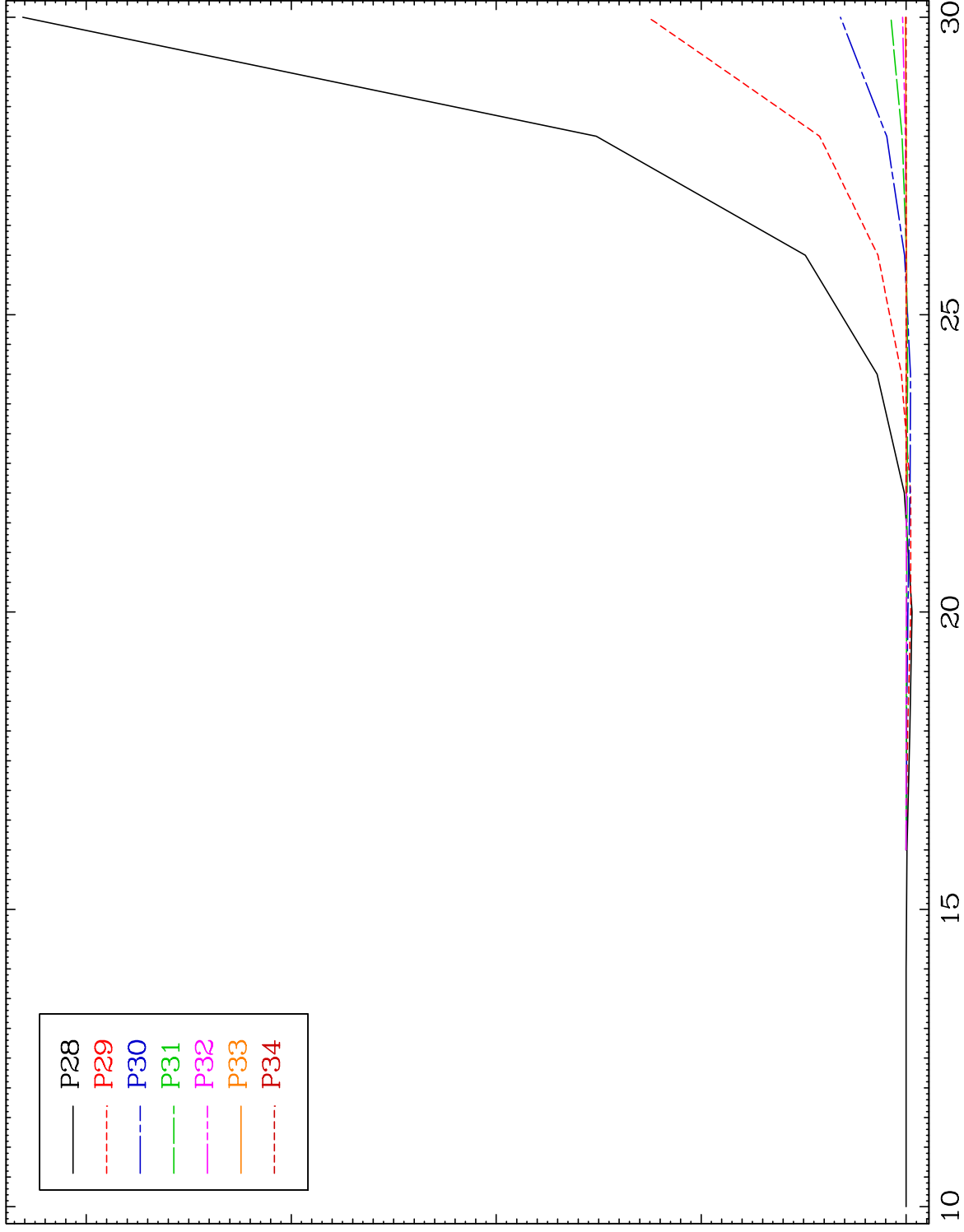
Elastic  
Legendre Coefficients

74-W -167



$\times 10^{-6}$

Legendre (CM)



10

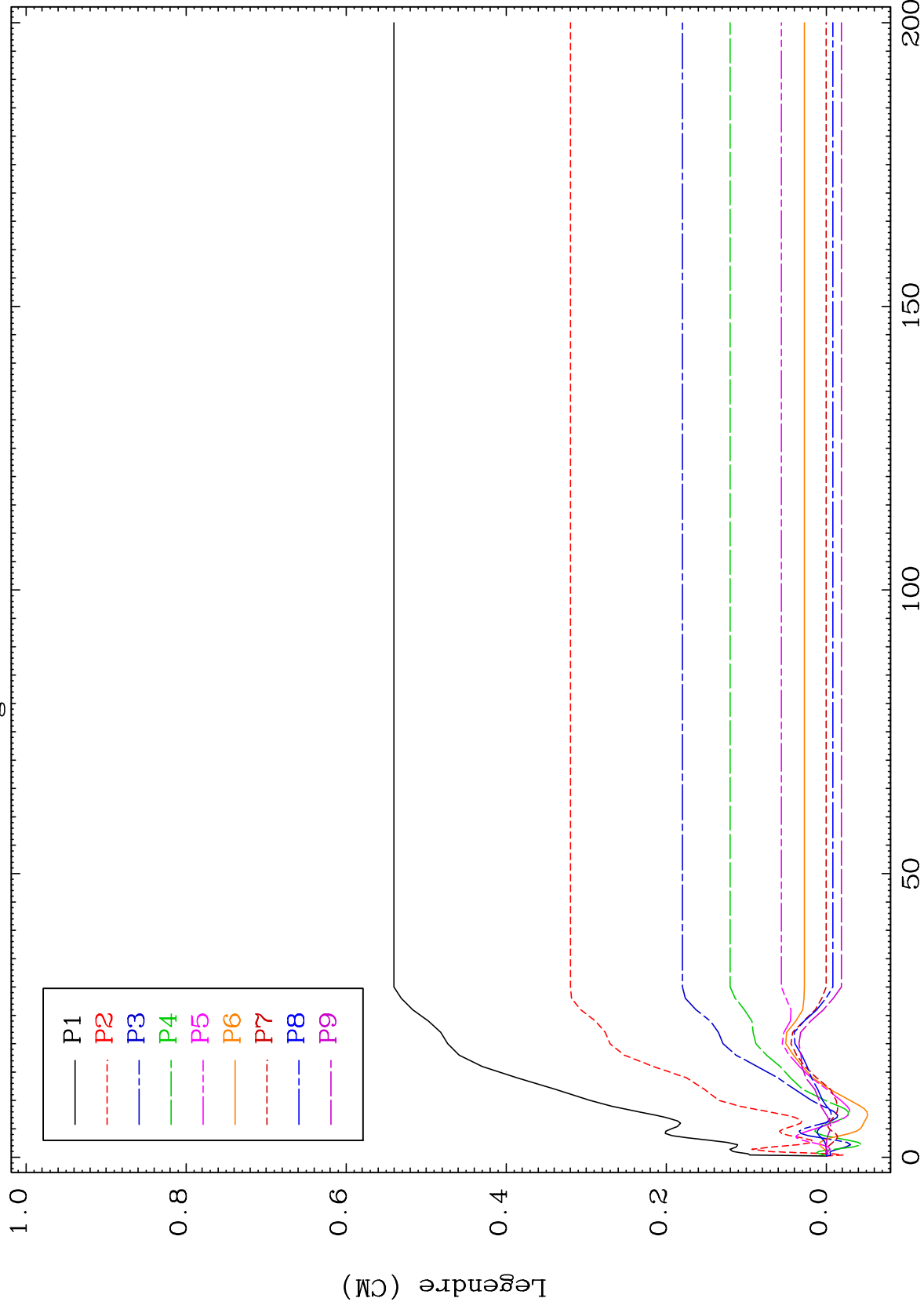
Incident Energy (MeV)

74-W -167

MAT 7386

MT= 51 (n,n') Level  
Legendre Coefficients

74-W -167



19

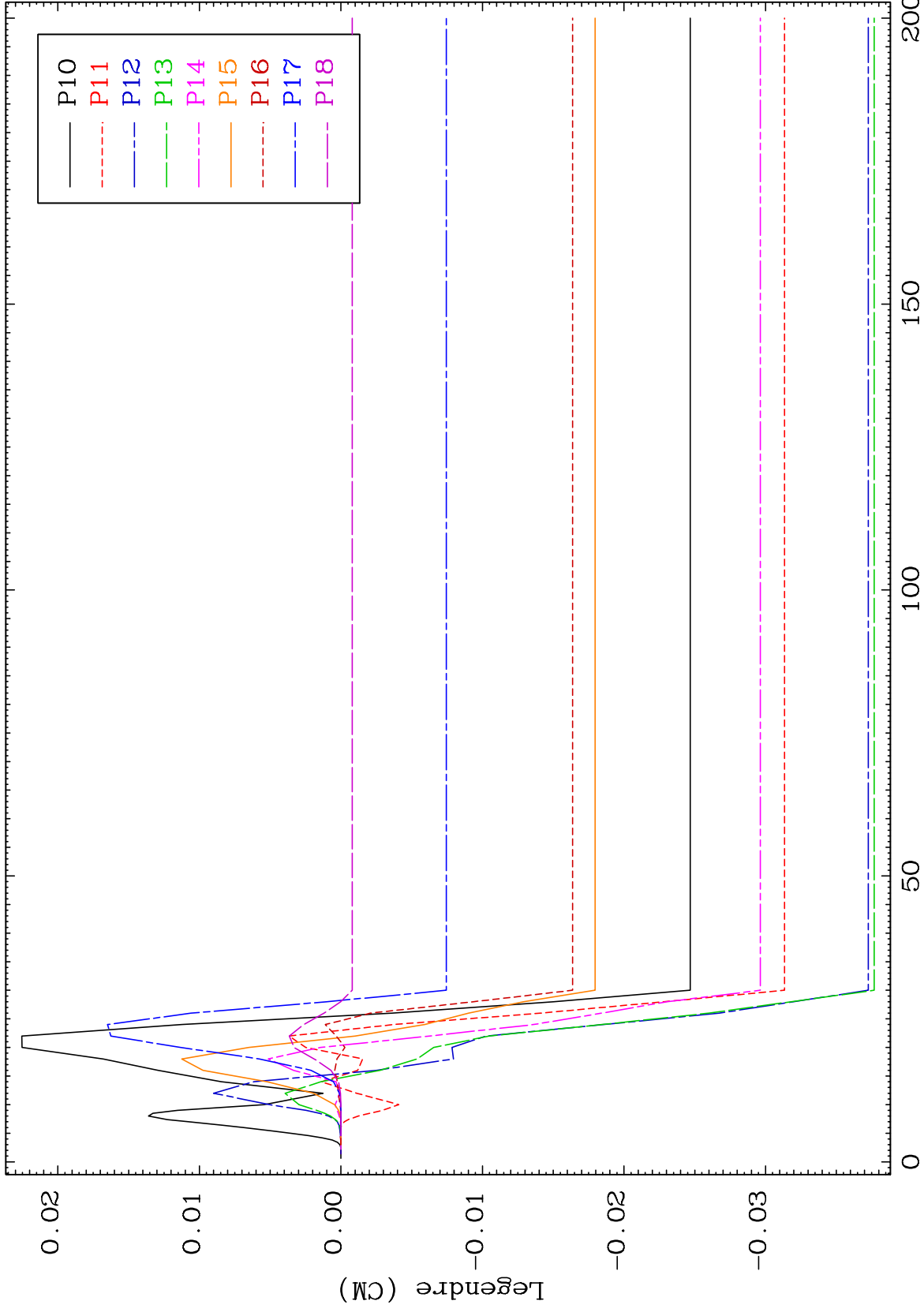
Incident Energy (MeV)

74-W -167

MAT 7386

MT= 51 (n,n') Level  
Legendre Coefficients

74-W -167



0

50

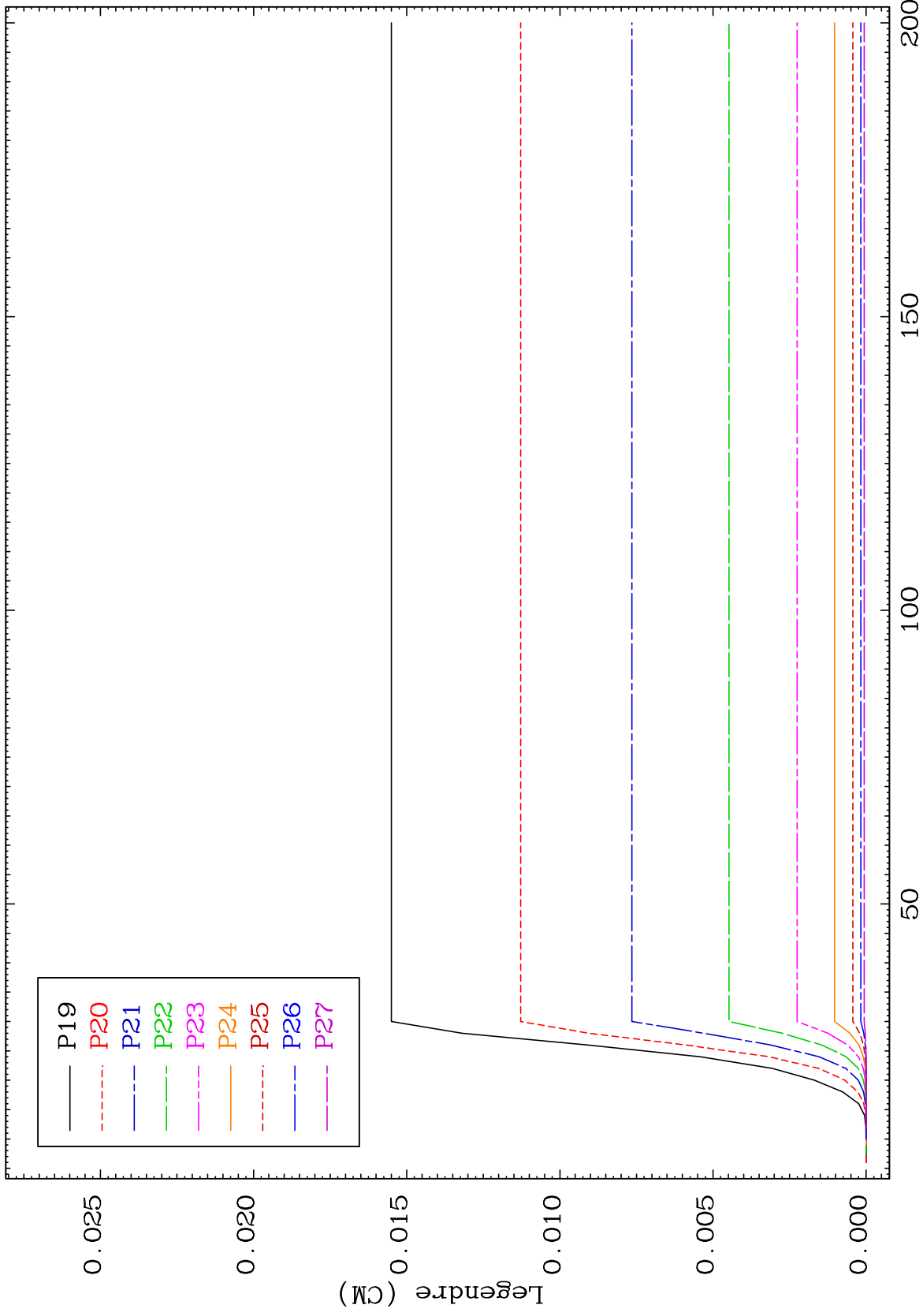
100

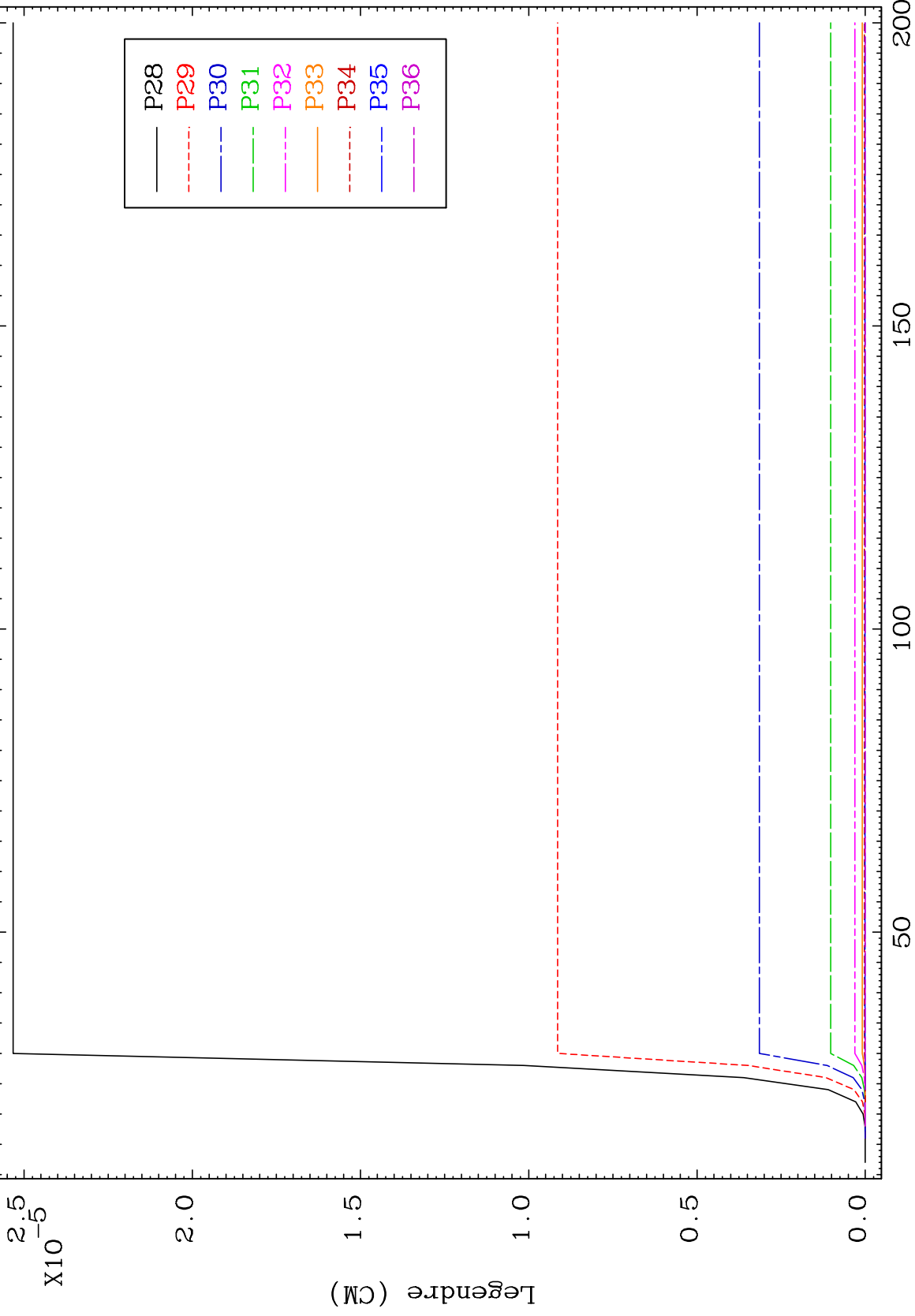
150

200

Incident Energy (MeV)

74-W -167



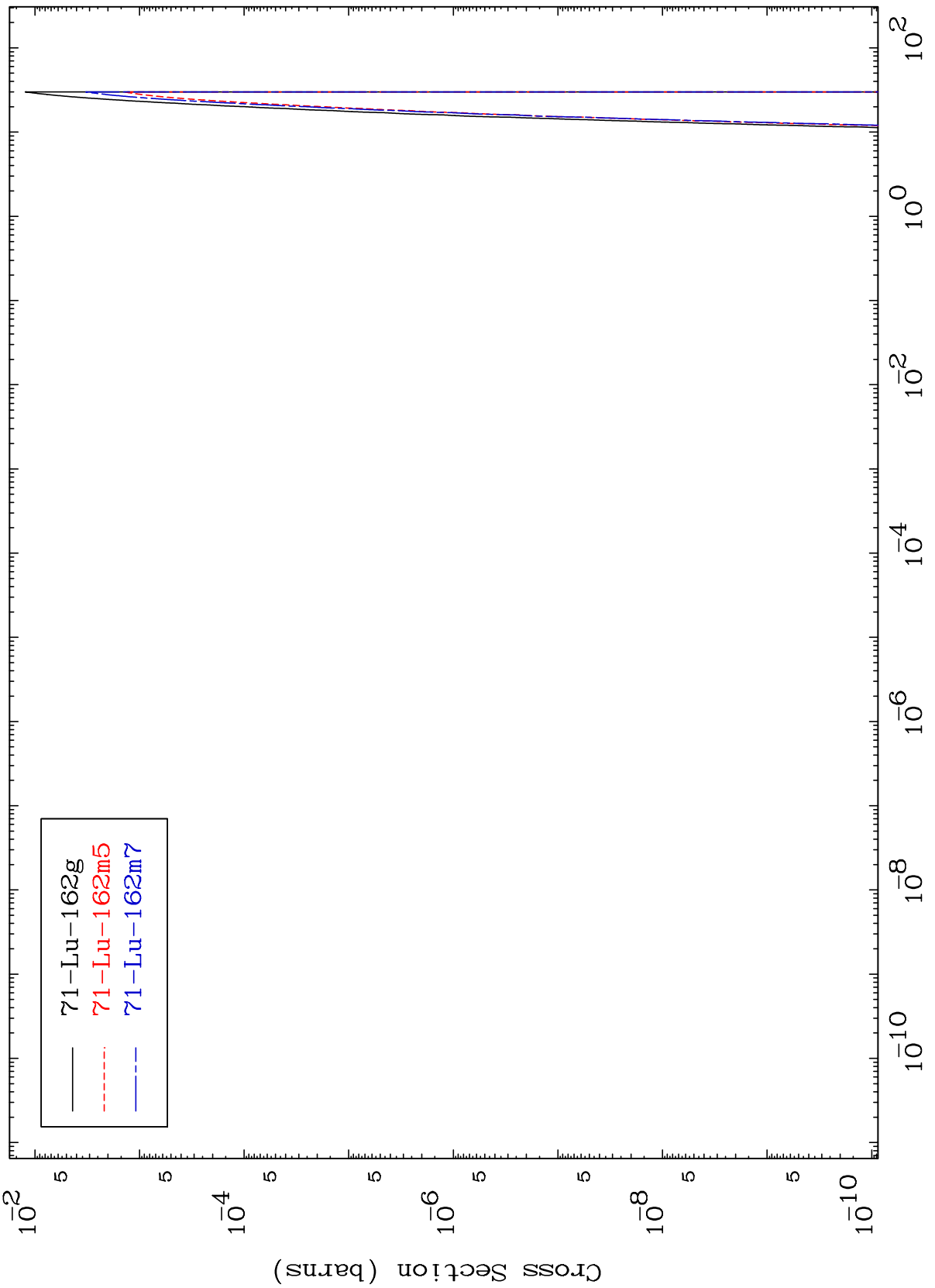


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(n,n') p  $\alpha$

74-W -167

Radionuclide Production Cross Section



Incident Energy (MeV)

74-W -167

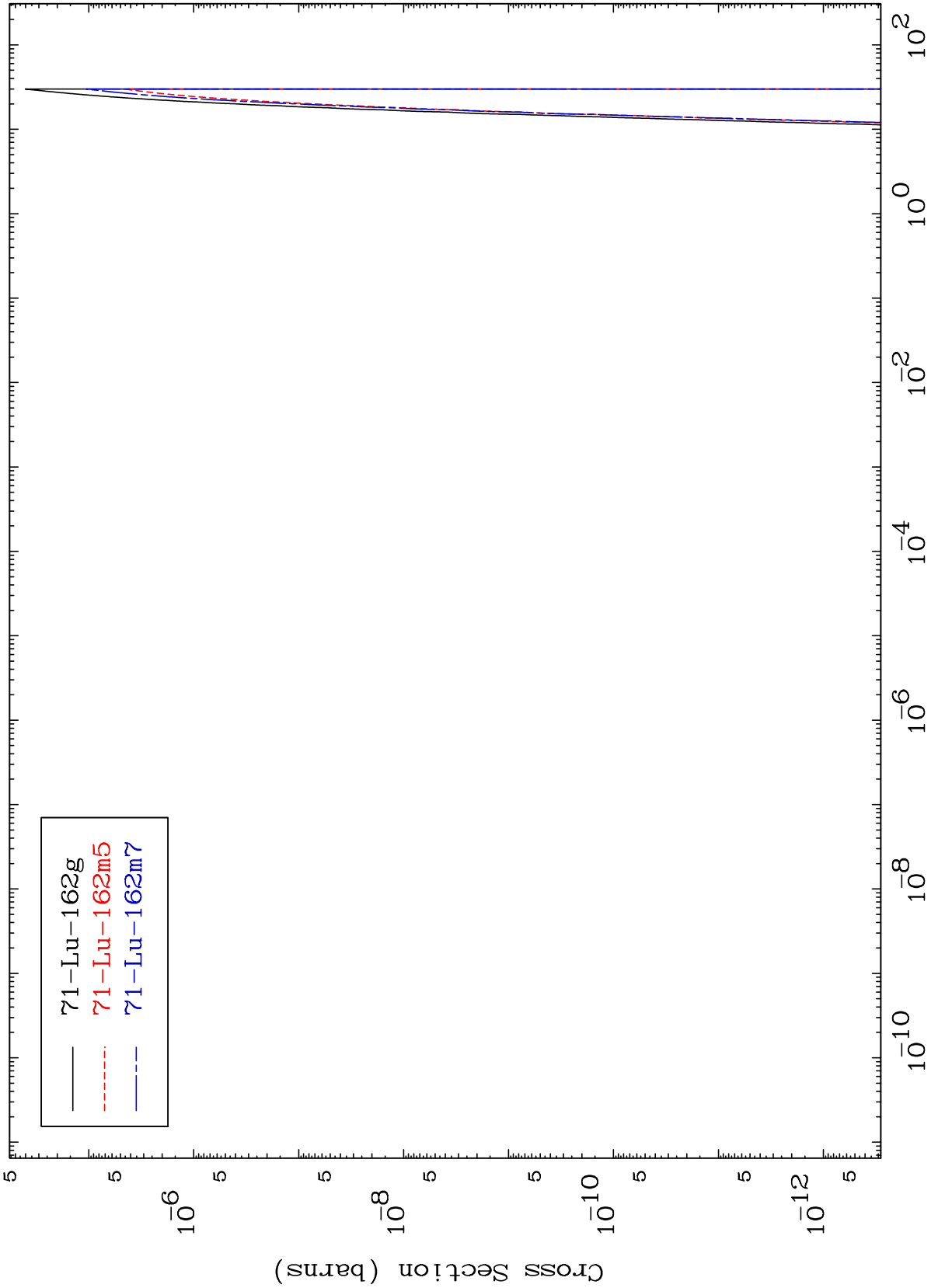


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(n,d)  $\alpha$

74-W -167

Radionuclide Production Cross Section



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Incident Energy (MeV)

74-W -167