



WIR SCHAFFEN WISSEN – HEUTE FÜR MORGEN

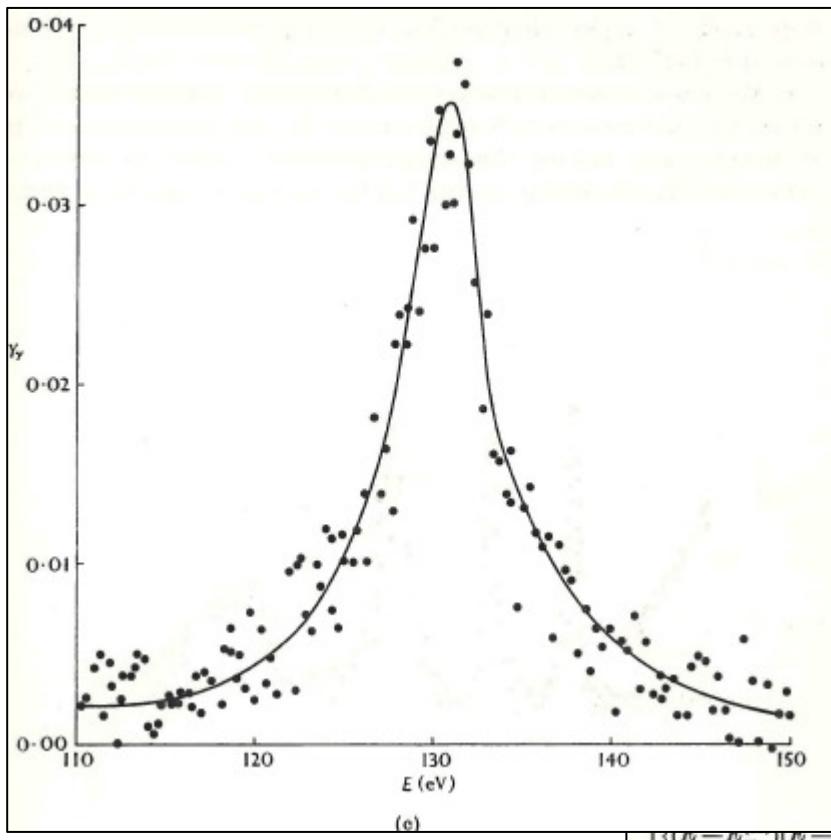
D. Rochman

Resonance parameters and related nuclear reactions

Joint ICTP-IAEA workshop, Trieste, Italy, October 16-20th, 2023

Summary

- Resonance energy range
 - Fitting
 - Resolved resonance range
 - Unresolved resonance range

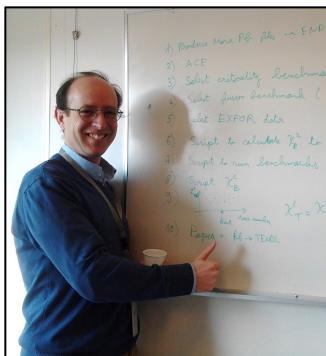
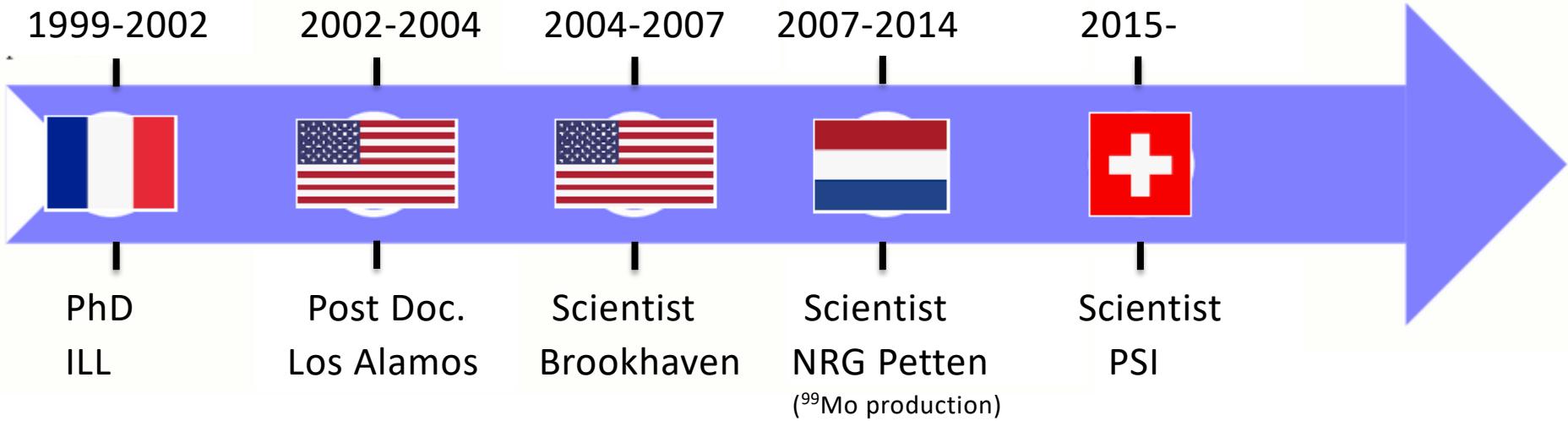


$$\begin{aligned}
& \sum'_{l's'} \left(\mathcal{P}_c^2 \mathcal{P}_{c'}^2 |Q_{cc'}|^2 \right. \\
& \left| \frac{G_{m(c)}^2 G_{m(c')}^2}{(E - E_m^{(H)})^2 + (\frac{1}{2} \Gamma_m^{(H)})^2} \right. \\
& \left. \frac{\mathcal{P}_c^2 \mathcal{P}_{c'}^2}{mc \mathcal{P}_{mc} \mathcal{P}_{mc'}} \cdot \frac{G_{m(c)} G_{m(c')}}{(E - E_m^{(H)})^2 + (\frac{1}{2} \Gamma_m^{(H)})^2} \times \right. \\
& \left. \{(E - E_m^{(H)}) \sin \theta_{m(cc')} + \frac{1}{2} \Gamma_m^{(H)} \cos \theta_{m(cc')} \} \right. \\
& \left. \frac{\mathcal{P}_c^2 \mathcal{P}_{c'}^2}{mc \mathcal{P}_{mc} \mathcal{P}_{m'c'} \mathcal{P}_{m'c'}} \times \right. \\
& \left. \frac{G_{m(c)} G_{m(c')} G_{m'(c)} G_{m'(c')}}{\{(E - E_m^{(H)})^2 + (\frac{1}{2} \Gamma_m^{(H)})^2\} \{(E - E_{m'}^{(H)})^2 + (\frac{1}{2} \Gamma_{m'}^{(H)})^2\}} \right. \\
& \left. E_{m'}^{(H)} + \frac{1}{2} \Gamma_m^{(H)} \Gamma_{m'}^{(H)} \right) \cos(\theta_{m'(cc')} - \theta_{m(cc')}) \\
& - E_m^{(H)} - \frac{1}{2} \Gamma_m^{(H)} (E - E_{m'}^{(H)}) \} \sin(\theta_{m'(cc')} - \theta_{m(cc')}) \Bigg)
\end{aligned}$$



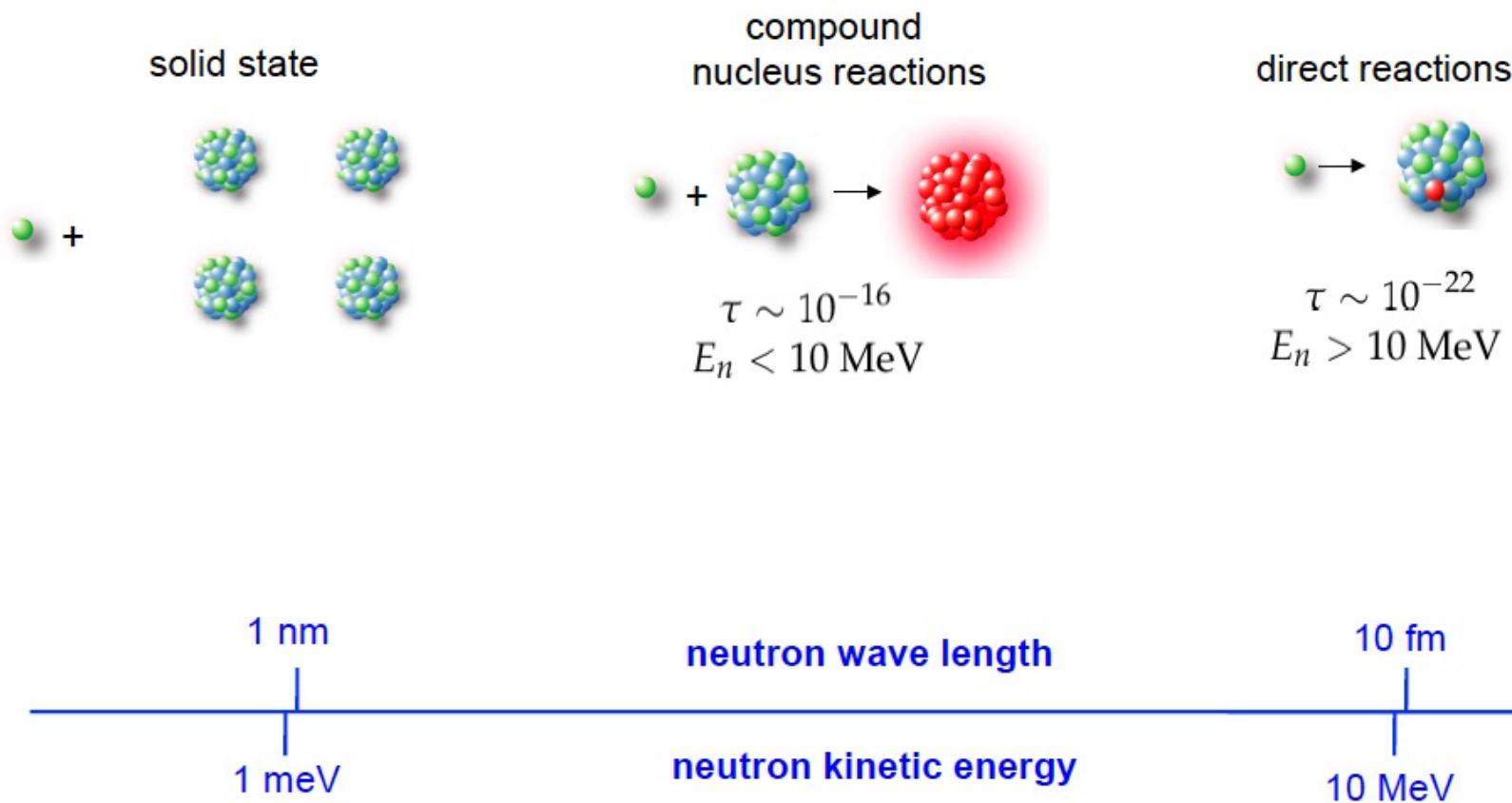
A short CV

- 50 years old, married, two kids, one dog



- PhD on fission yield measurements
- Postdoc on cross section measurements
- Scientist:
 - nuclear data evaluation
 - reactor calculations
 - spent nuclear fuel characterization

Neutron induced reactions

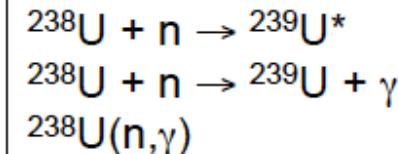
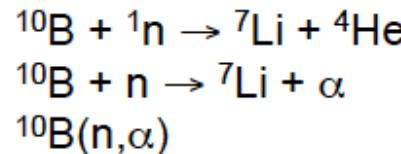


Neutron induced reactions

Reaction:

- $X + a \rightarrow Y + b$
- $X(a,b)Y$
- $X(a,b)$

Examples of equivalent notations:



Reaction cross section σ , expressed in barns, $1 \text{ b} = 10^{-28} \text{ m}^2$

Neutron induced nuclear reactions:

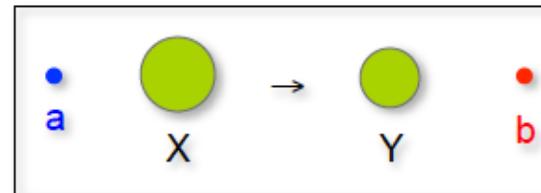
- elastic scattering (n,n)
- inelastic scattering (n,n')
- capture (n,γ)
- fission (n,f)
- particle emission (n,α) , (n,p) , (n,xn)

Total cross section σ_{tot} : sum of all reactions

Neutron induced reactions

Reaction:

- $X + a \rightarrow Y + b$
- $X(a,b)Y$



Cross section:

function of the kinetic energy of the particle **a** $\sigma(E_a) = \int \int \frac{d^2\sigma(E_a, E_b, \Omega)}{dE_b d\Omega} dE_b d\Omega$

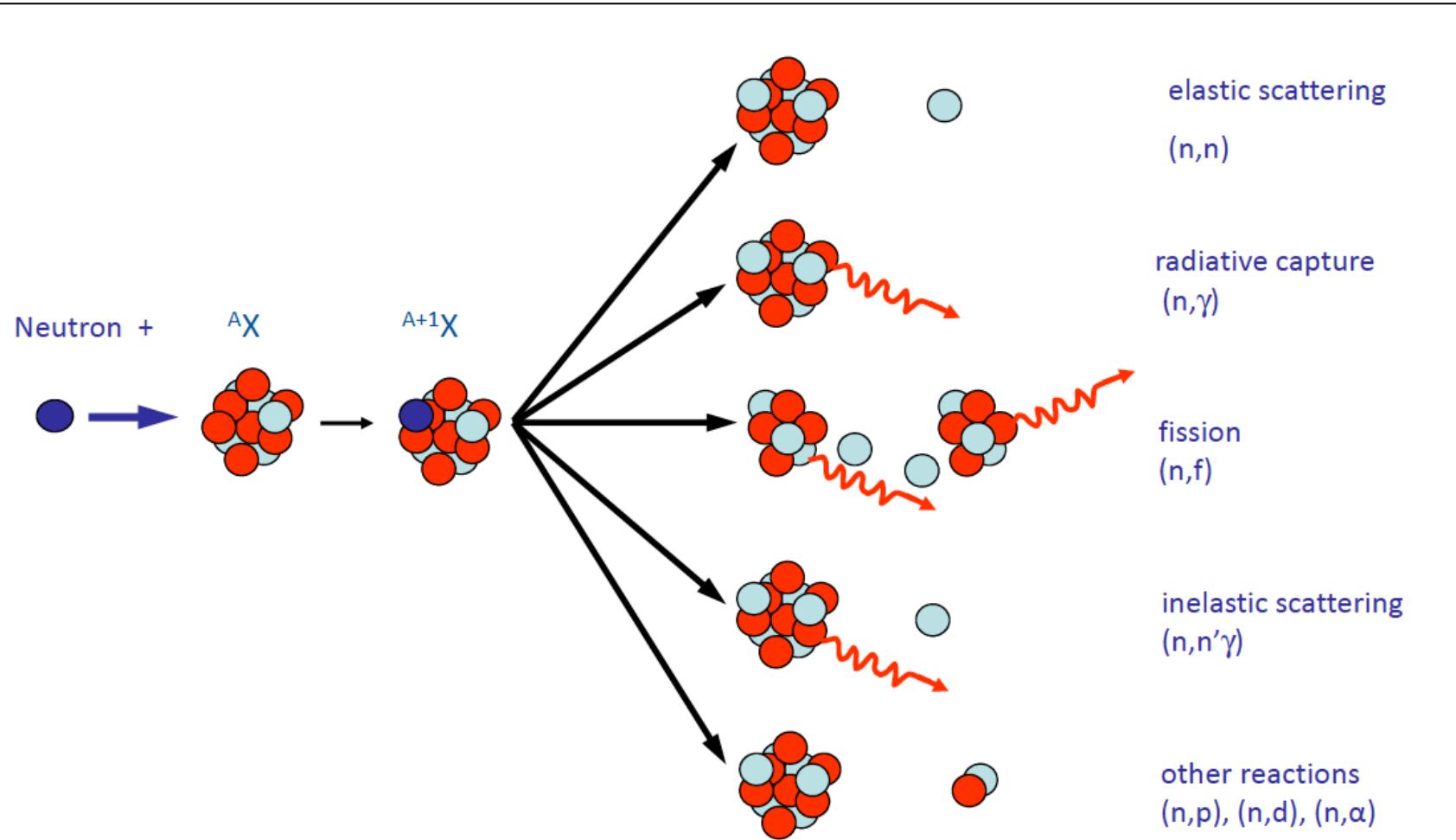
Differential cross section:

function of the kinetic energy of the particle **a** $\frac{d\sigma(E_a, E_b)}{dE_b}$
and function of the kinetic energy or the angle $\frac{d\sigma(E_a, \Omega)}{d\Omega}$
of the particle **b**

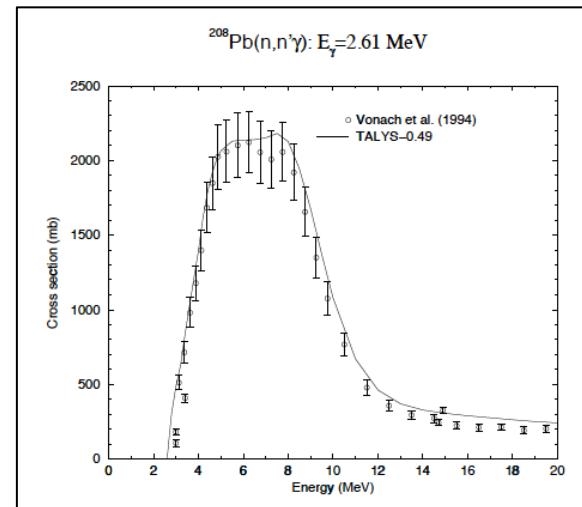
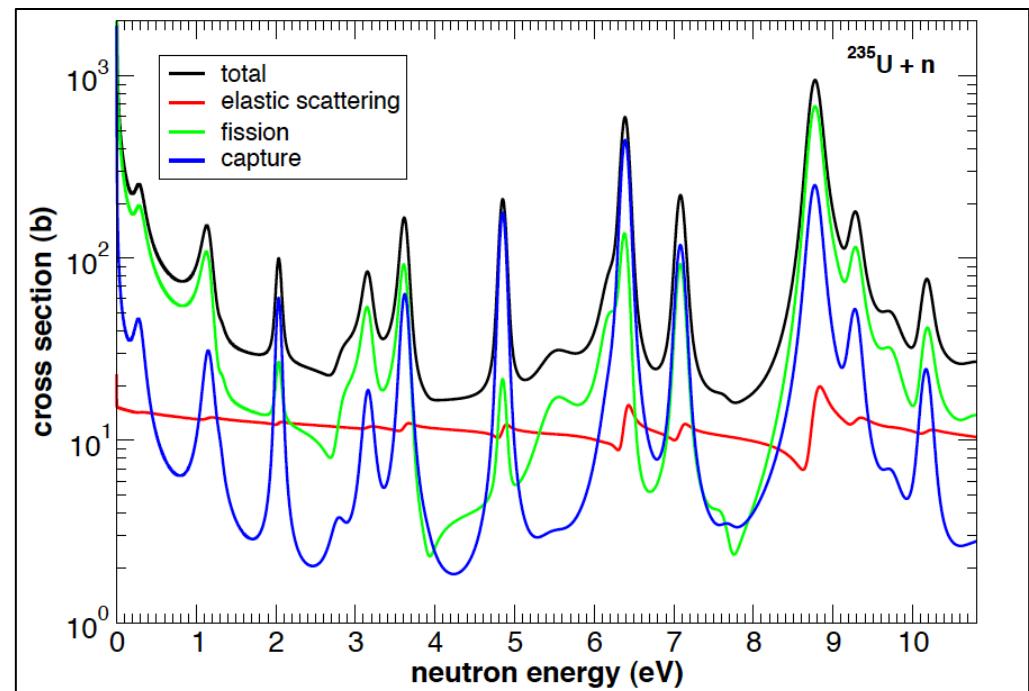
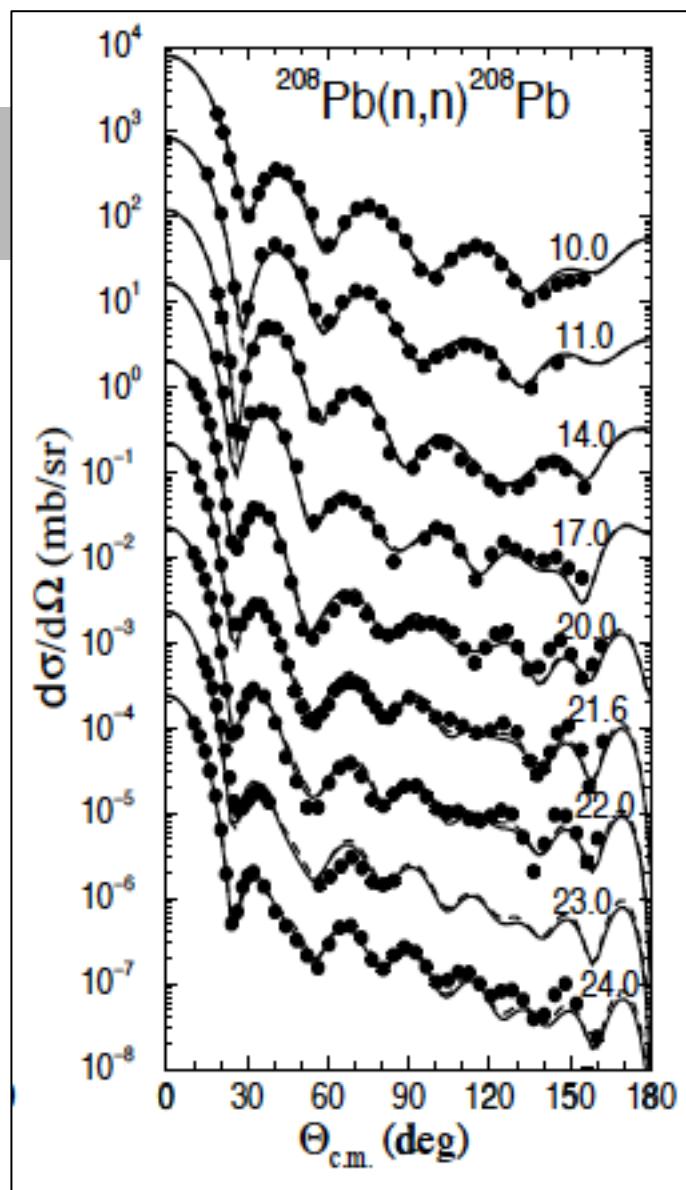
Double differential cross section:

function of the kinetic energy of the particle **a** $\frac{d^2\sigma(E_a, E_b, \Omega)}{dE_b d\Omega}$
and function of the kinetic energy and the angle
of the particle **b**

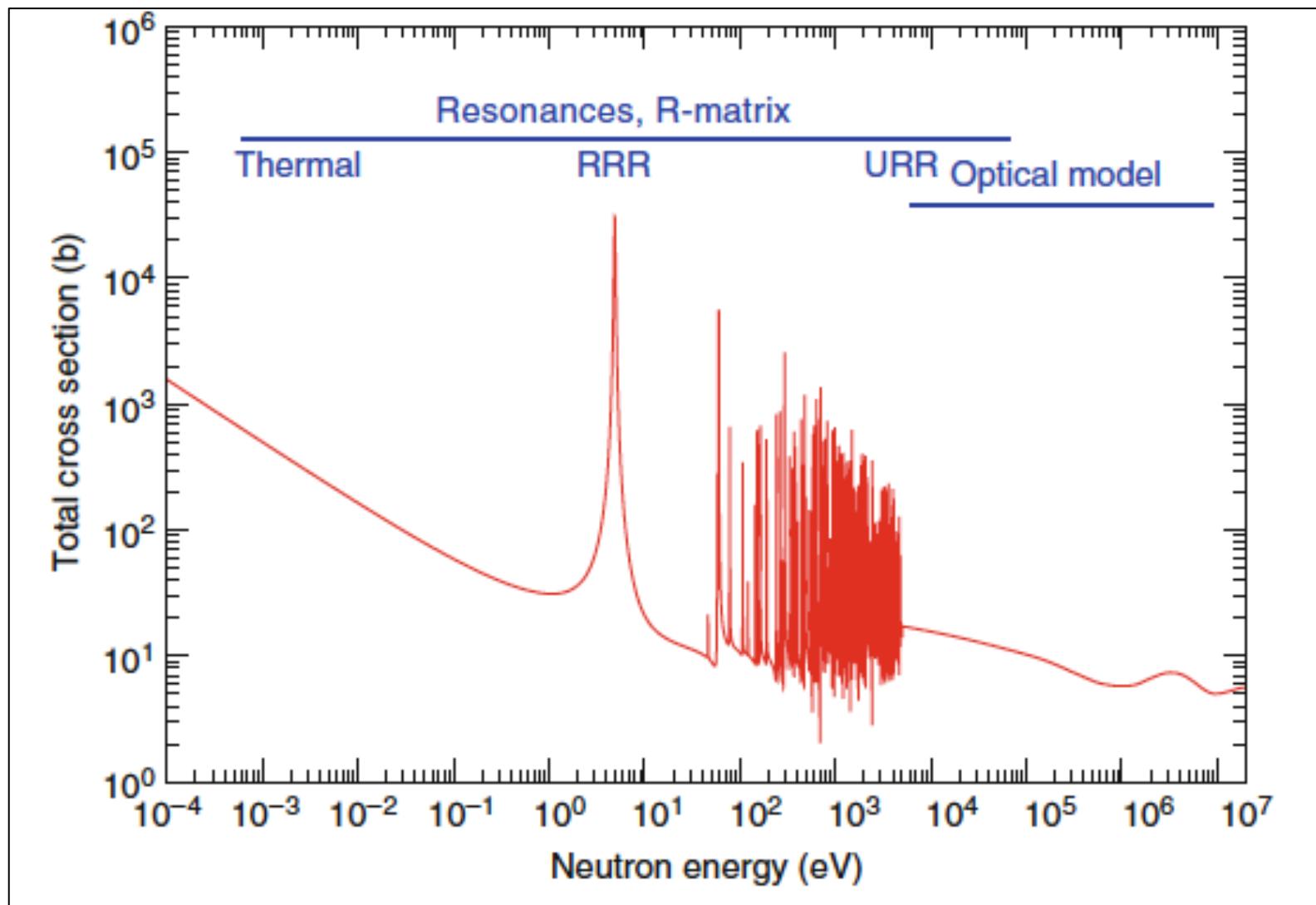
Neutron induced reactions



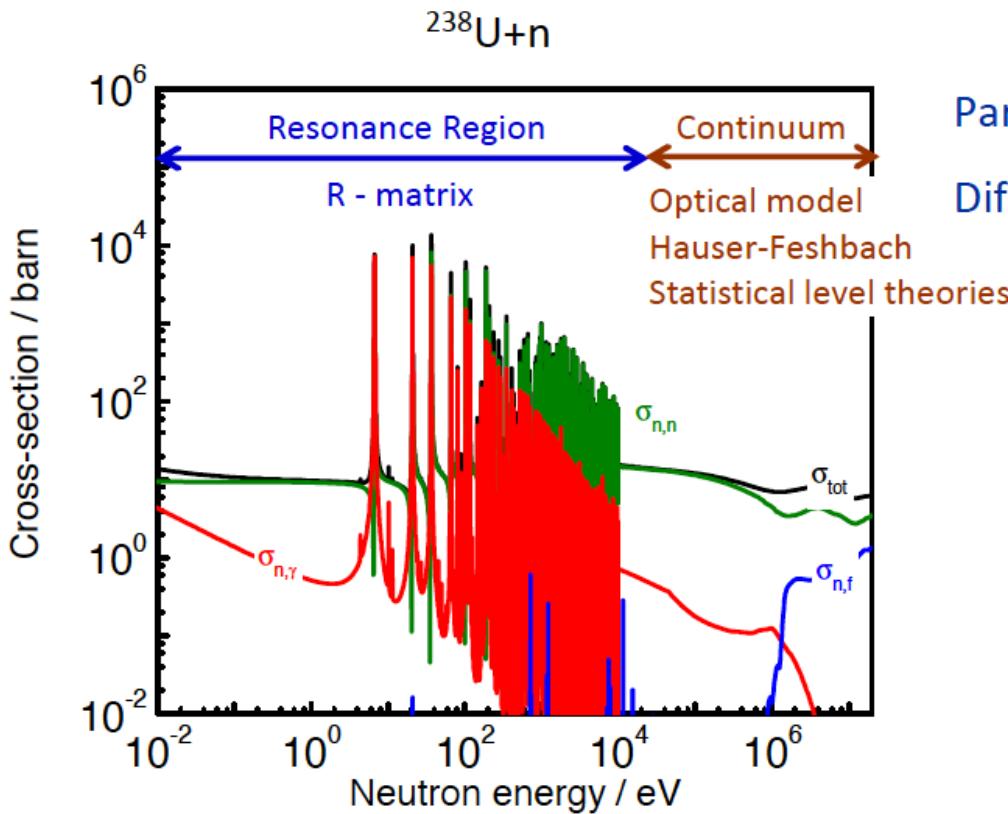
Neutron induced reactions: examples



Neutron induced reactions: separation



Neutron induced reactions: separation



Parameterised by nuclear reaction model

Different regions : different theories

- Resonance region
 - R-matrix theory
- Continuum
 - Optical model
 - Hauser-Feshbach
 - Statistical level theories
 - ...

Compound nucleus: spin and parity

$n + X$
entrance channel

\rightarrow
(1)

C^*
compound nucleus

\rightarrow
(2)

$Y + r$
exit channel

\vec{I} spin target nucleus

π_o parity target nucleus

\vec{i} spin incoming neutron ($i=1/2$)

$\vec{\ell}$ orbital angular momentum

$(\ell=0,1,2 \rightarrow s,p,d-\text{wave})$

E_n incident neutron energy

$$E^* = S_n + \frac{A}{A+1} E_n$$

$$\vec{J} = \vec{I} + \vec{i} + \vec{\ell}$$

$$\vec{J} = \vec{\ell} + \vec{s}$$

$$|\ell - s| \leq J \leq |\ell + s|$$

$$\vec{s} = \vec{I} + \vec{i}$$

$$|I - i| \leq s \leq |I + i|$$

$$\pi = (-1)^\ell \pi_o$$

Conservation of :

- Energy
- Total angular momentum
- Parity

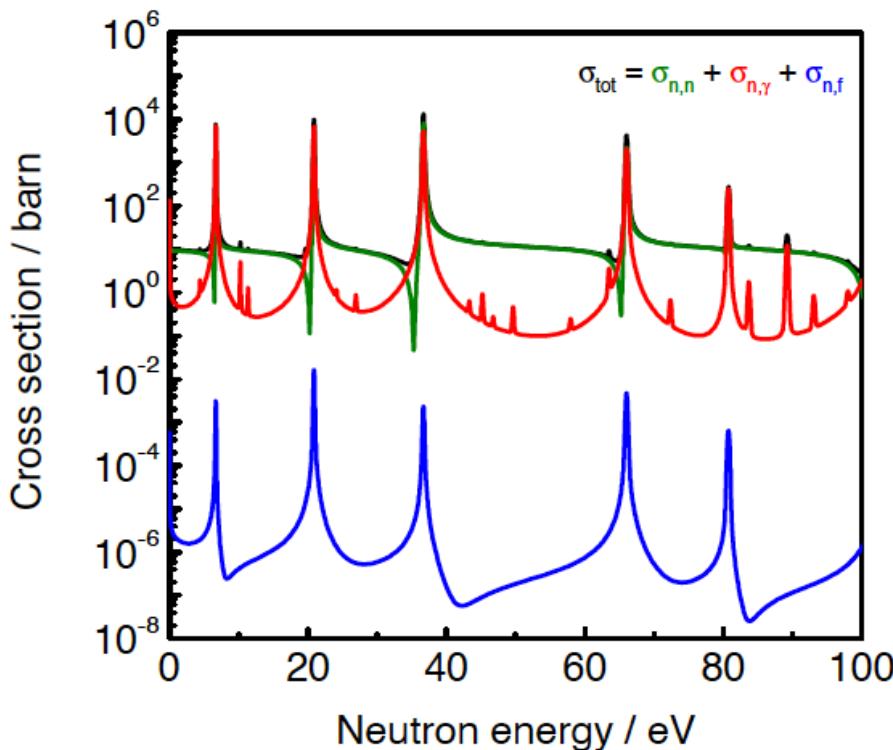
Compound nucleus: spin and parity

$$\begin{array}{ccc}
 n + X & \rightarrow & C^* \\
 \text{entrance channel} & & \text{compound nucleus} \\
 \text{Incident neutron: } i = 1/2 & &
 \end{array}$$

$$\begin{aligned}
 \vec{j} &= \vec{l} + \vec{i} + \vec{\ell} \\
 \vec{s} &= \vec{l} + \vec{i} \quad |l - i| \leq s \leq |l + i| \\
 \vec{j} &= \vec{\ell} + \vec{s} \quad |\ell - s| \leq j \leq |\ell + s|
 \end{aligned}$$

Target nucleus	Orbital angular momentum	Channel spin	Compound nucleus Spin & Parity		Spin factor
			$\pi = (-1)^\ell \pi_o$	j^π $ \ell - s \leq j \leq \ell + s $	
$ ^{1\pi}_o$	ℓ	s $ l - i \leq s \leq l + i $			$g = \frac{2J+1}{2(2l+1)}$
0^+	0 (s-wave)	$1/2$	+	$1/2^+$	1
	1 (p-wave)	$1/2$	-	$1/2^-, 3/2^-$	1, 2
	2 (d-wave)	$1/2$	+	$3/2^+, 5/2^+$	2, 3
$1/2^+$	0 (s-wave)	0	+	0^+	$1/4$
		1		1^+	$3/4$
	1 (p-wave)	0	-	1^-	$3/4$
		1		$0^-, 1^-, 2^-$	$1/4 \quad 3/4 \quad 5/4$
	2 (d-wave)	0	+	2^+	$5/4$
		1		$1^+, 2^+, 3^+$	$3/4 \quad 5/4 \quad 7/4$

Thermal and resonance region



R-matrix theory

Lane and Thomas, Rev. Mod. Phys. 30 (1958) 257

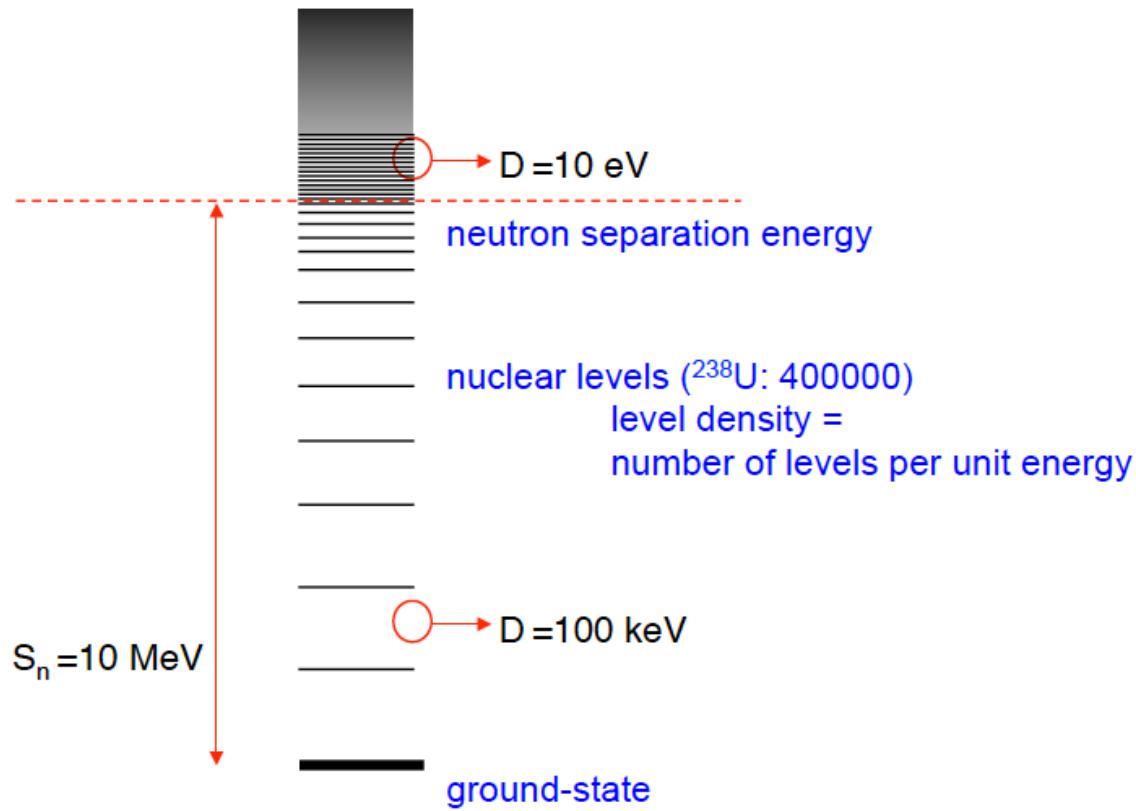
Model parameters

R and $(E_R, J^\pi, \Gamma_n, \Gamma_\gamma \dots)_j$

E_R	resonance energy
$\Gamma_n, \Gamma_\gamma, \Gamma_f$	partial widths
Γ	total width
$(\Gamma = \Gamma_n + \Gamma_\gamma + \Gamma_f \dots)$	
R	scattering radius

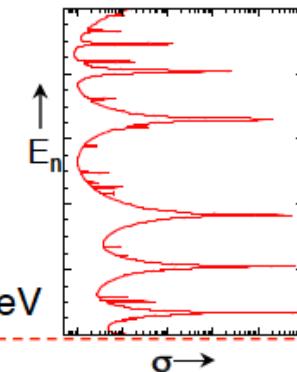
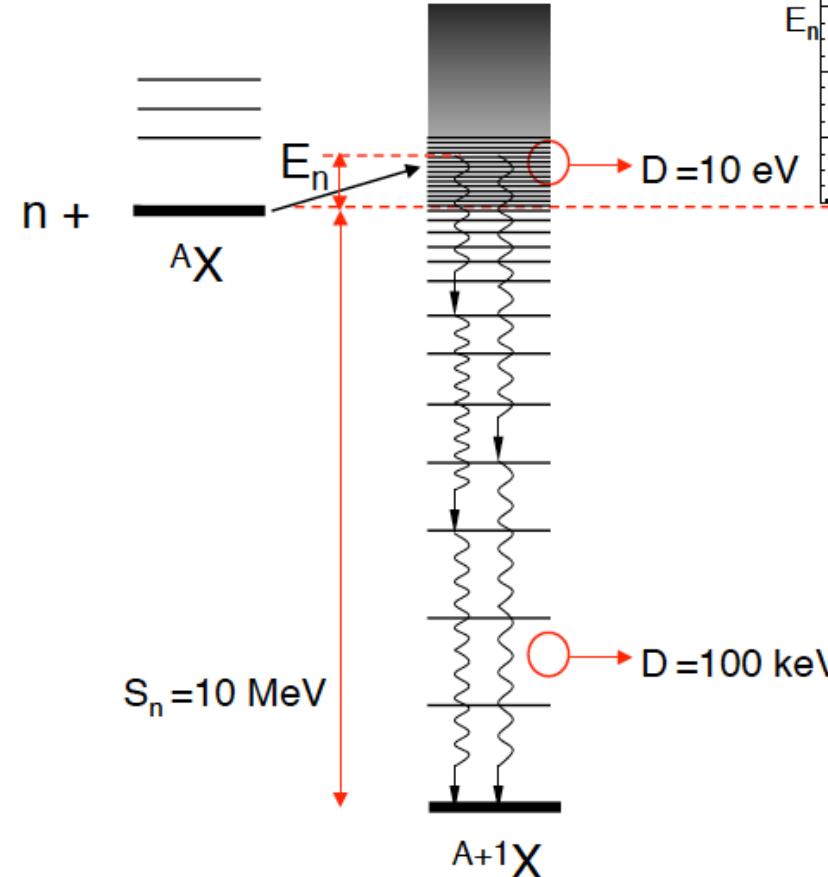
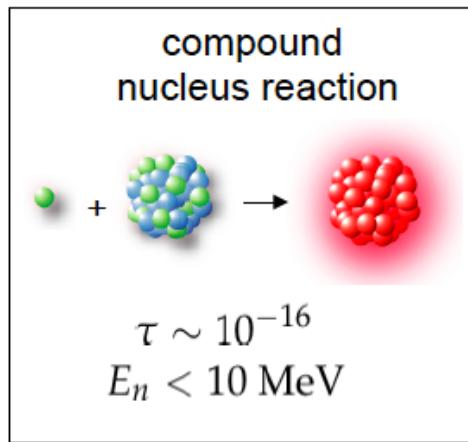
Thermal and resonance region

Nuclear levels

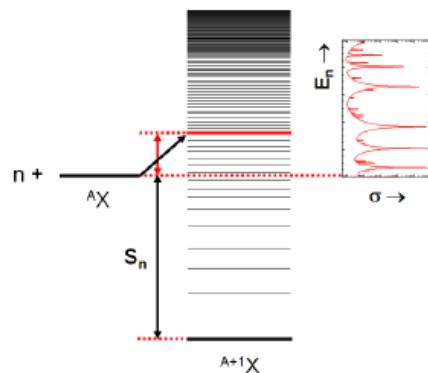
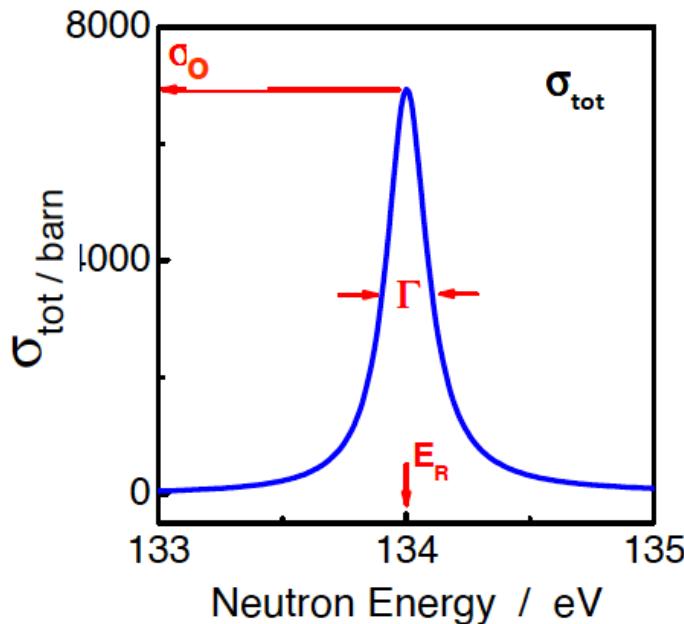


Thermal and resonance region

Compound neutron-nucleus reactions



Thermal and resonance region



The resonant structure can be described by a Breit-Wigner shape :

$$\sigma_{\text{tot}} \sim \frac{\Gamma}{(E_n - E_R)^2 + (\Gamma/2)^2}$$

with

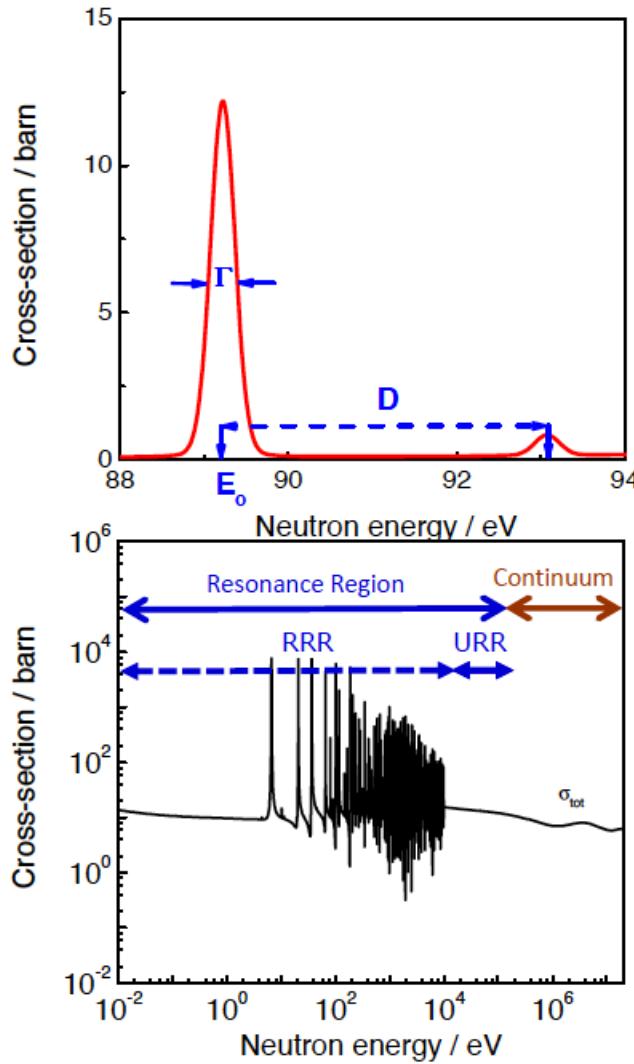
Γ total width (FWHM)

E_R resonance energy

Heisenberg uncertainty principle

$$\Delta E \Delta t = \frac{\hbar}{2} \quad \Gamma \Delta t = \frac{\hbar}{2}$$

Thermal and resonance region



D : level distance
 Γ : resonance width
 Δ_R : instrument resolution

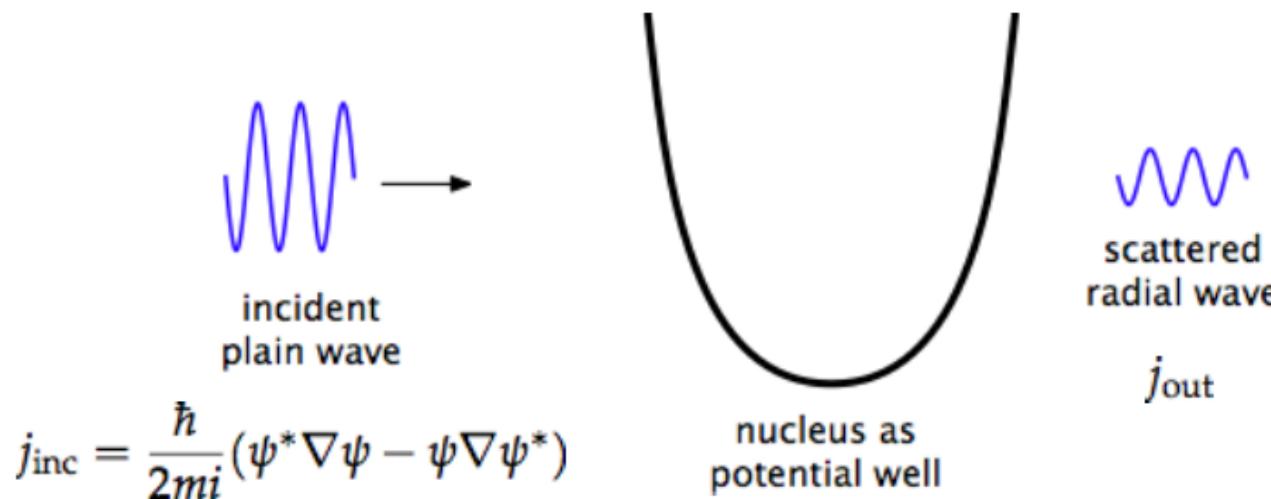
- Resonance Region : $D > \Gamma$

Resolved Resonance Region : $\Delta_R < D$

Unresolved Resonance Region : $\Delta_R > D$

- Continuum Region : $D < \Gamma$

Thermal and resonance region



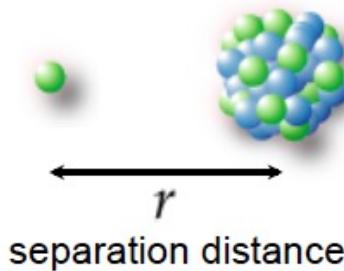
Conservation of probability density: $\sigma(\Omega) = \frac{r^2 j_{\text{out}}(r, \Omega)}{j_{\text{inc}}}$

Solve Schrödinger equation of system to get cross sections.
 Shape of wave functions of in- and outgoing particles are known,
 potential is unknown. Two approaches:

- calculate potential (optical model calculations, smooth cross section)
- use eigenstates (R-matrix, resonances)

Thermal and resonance region

Find the wave functions



$r > a_c$ external region

$r < a_c$ internal region

$r = a_c$ match value and derivate of Ψ

$$\left[\frac{d^2}{dr^2} - \frac{\ell(\ell + 1)}{r^2} - \frac{2m_c}{\hbar^2}(V - E) \right] rR(r) = 0$$

External region: **easy**, solve Schrödinger equation

central force, separate radial and angular parts.

$$\psi(r, \theta, \phi) = R(r)\Theta(\theta)\Phi(\phi)$$

solution: solve Schrödinger equation of relative motion:

- Coulomb functions
- special case of neutron particles (neutrons): fonctions de Bessel

Internal region: **very difficult**, Schrödinger equation cannot be solved directly

solution: expand the wave function as a linear combination of its eigenstates.

using the **R-matrix**:

$$R_{cc'} = \sum_{\lambda} \frac{\gamma_{\lambda c} \gamma_{\lambda c'}}{E_{\lambda} - E}$$

Thermal and resonance region

- Approximation of the R-matrix:
 - Single level Breit Wigner
 - Multi level Breit Wigner (& Multi Niveau Breit Wigner)
 - Reich Moore
 - R-Matrix limited
- Applied to (n,el), (n,g), (n,f)

For an isolated, neutron resonance, we write the Breit-Wigner equation for capture reactions in the form

$$\sigma_\gamma = \sigma_0 \frac{\Gamma_\gamma}{\Gamma} \left(\frac{E_0}{E} \right)^{1/2} \frac{1}{1 + y^2},$$

where

$$y = \frac{2}{\Gamma} (E - E_0),$$

$$\sigma_0 = 4\pi \lambda_0^2 \frac{g\Gamma_n}{\Gamma} = \frac{2.608 \times 10^6}{E_0(\text{eV})} \left(\frac{A+1}{A} \right)^2 \frac{g\Gamma_n}{\Gamma}.$$

Thermal and resonance region

- For resonances far from the resonance range, the capture cross section at 0.0253 eV is

$$\sigma_\gamma^0 = 4.099 \times 10^6 \left(\frac{A+1}{A} \right)^2 \sum_j^N \frac{g \Gamma_{nj}^0 \Gamma_{\gamma j}}{E_{0j}^2}.$$

- Γ_n^0 is the reduce width:

– For $I=0, V_I=1$

$$\Gamma_{nj}^\ell = \sqrt{\frac{1 \text{ eV}}{E_0}} \frac{\Gamma_{nj}}{V_\ell}. \quad \Gamma_n^1 = \frac{\Gamma_n}{\sqrt{E_0}} \left(1 + \frac{11369}{E(\text{keV}) A^{2/3}} \right)$$

- orbital momentum of incoming neutron relative to nucleus: ℓ
- Resonance spin and parity:

$$\mathbf{J} = \mathbf{I} + \mathbf{1/2} + \ell$$

$$\pi = \pi_i \times (-1)^\ell$$

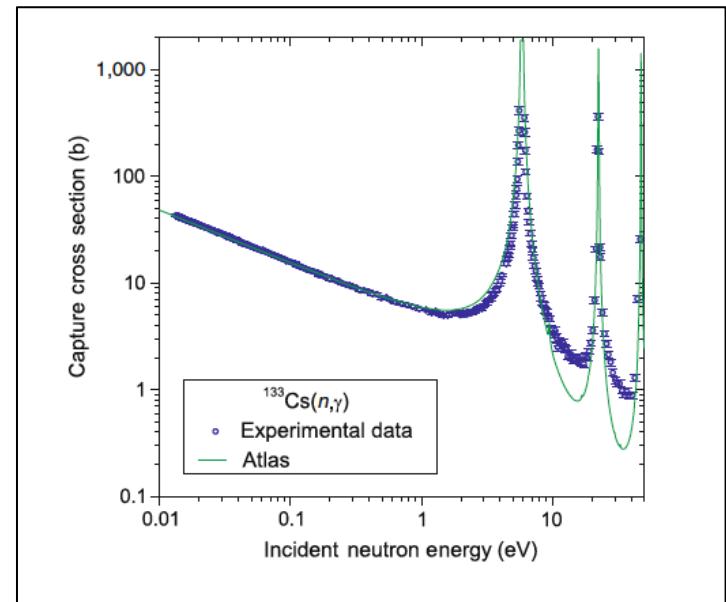
- partial waves:

s-wave $\ell = 0$

p-wave $\ell = 1$

d-wave $\ell = 2$

f-wave $\ell = 3$

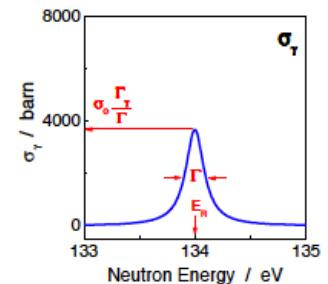


Thermal and resonance region

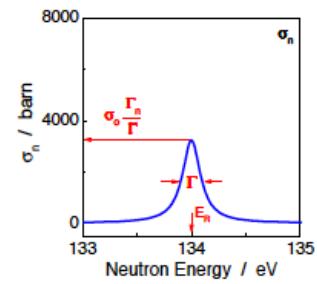
- Example for SLBW, $l=0$

$$(E_R, \Gamma_n, \Gamma_\gamma, J^\pi, R_{\ell=0})$$

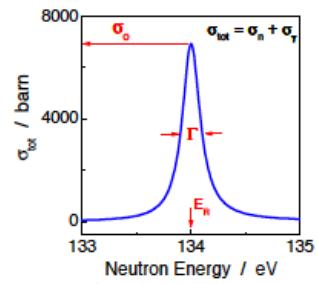
- (n, γ) $\sigma_\gamma(E_n) = g \frac{\pi}{k_n^2} \frac{\Gamma_n \Gamma_\gamma}{(E_n - E_R)^2 + (\Gamma/2)^2}$



- (n, n) $\sigma_n(E_n) = g \frac{\pi}{k_n^2} \frac{\Gamma_n \Gamma_n}{(E_n - E_R)^2 + (\Gamma/2)^2} + g \frac{4\pi}{k_n} \frac{\Gamma_n (E_n - E_R) R}{(E_n - E_R)^2 + (\Gamma/2)^2} + 4\pi R^2$



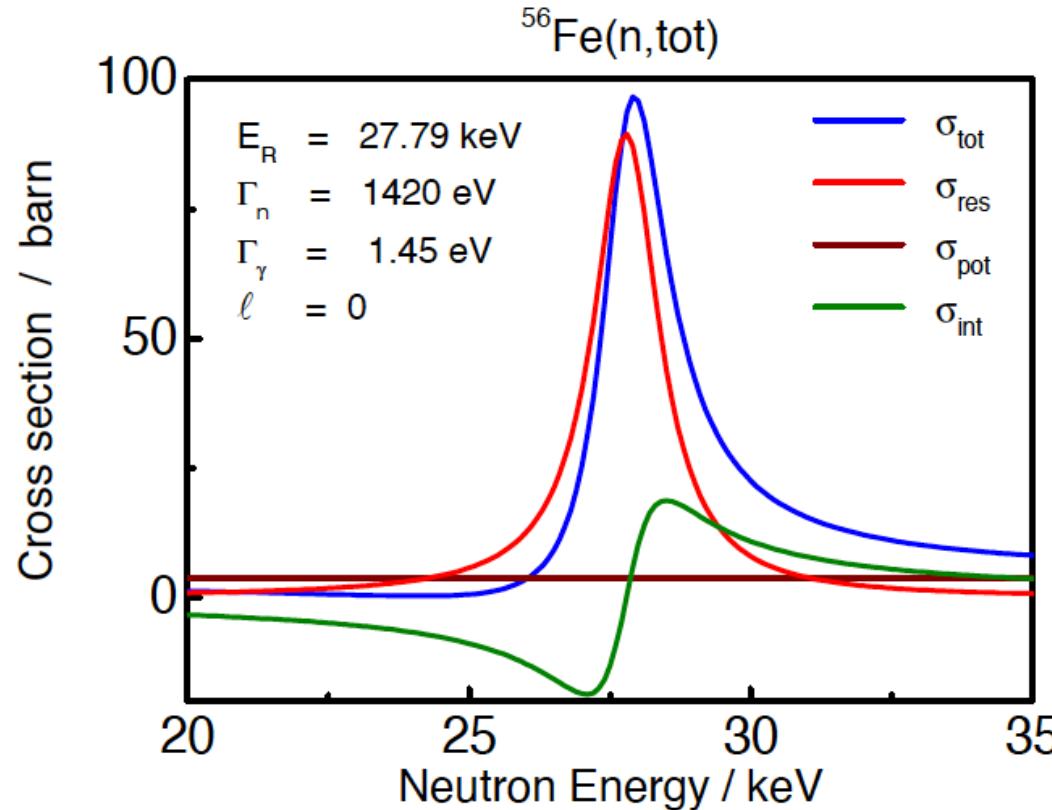
- (n, tot) $\sigma_{\text{tot}}(E_n) = g \frac{\pi}{k_n^2} \frac{\Gamma_n \Gamma}{(E_n - E_R)^2 + (\Gamma/2)^2} + g \frac{4\pi}{k_n} \frac{\Gamma_n (E_n - E_R) R}{(E_n - E_R)^2 + (\Gamma/2)^2} + 4\pi R^2$



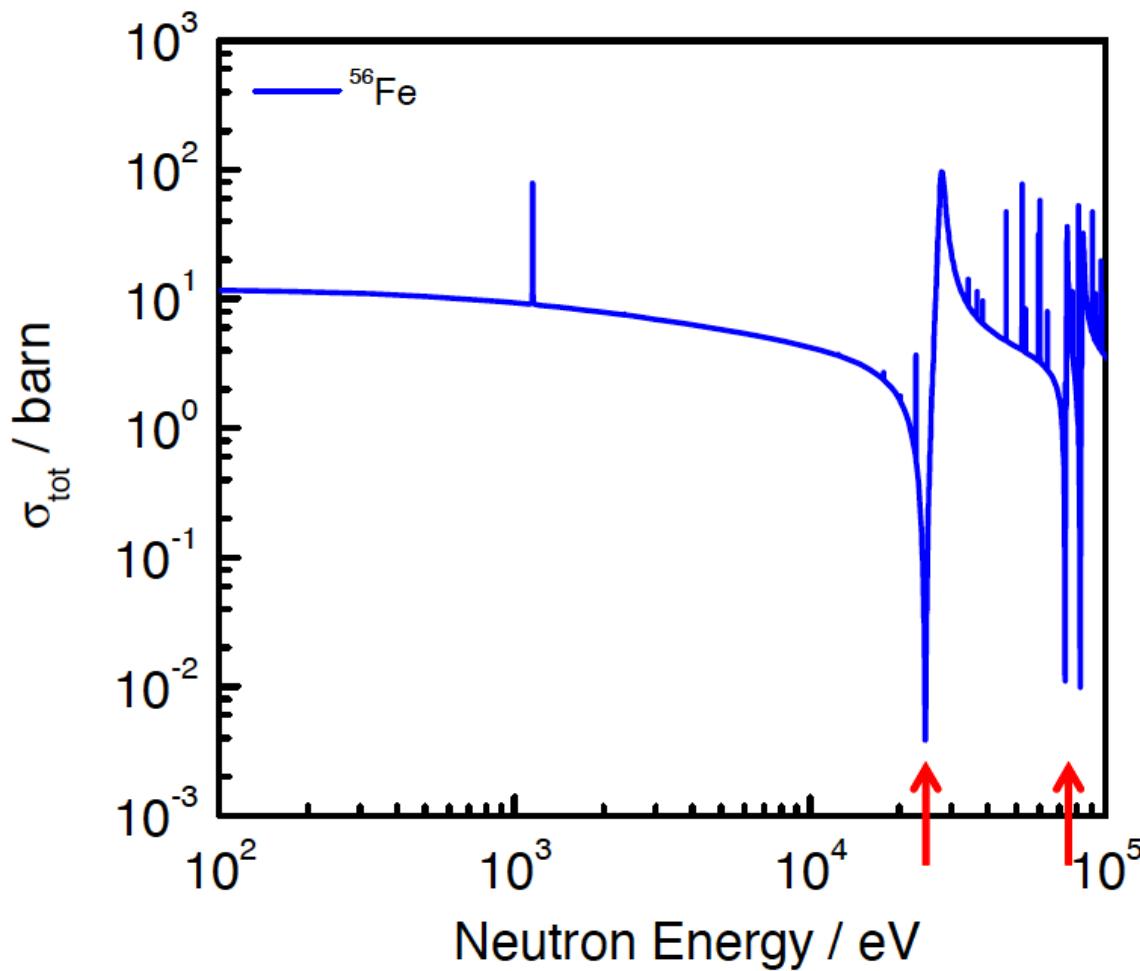
Thermal and resonance region: interference

- Example for SLBW, $l=0$

$$\sigma_{\text{tot}}(E_n) = g \frac{\pi}{k_n^2} \frac{\Gamma_n \Gamma}{(E_n - E_R)^2 + (\Gamma/2)^2} + \boxed{g \frac{4\pi}{k_n} \frac{\Gamma_n (E_n - E_R) R}{(E_n - E_R)^2 + (\Gamma/2)^2}} + 4\pi R^2$$



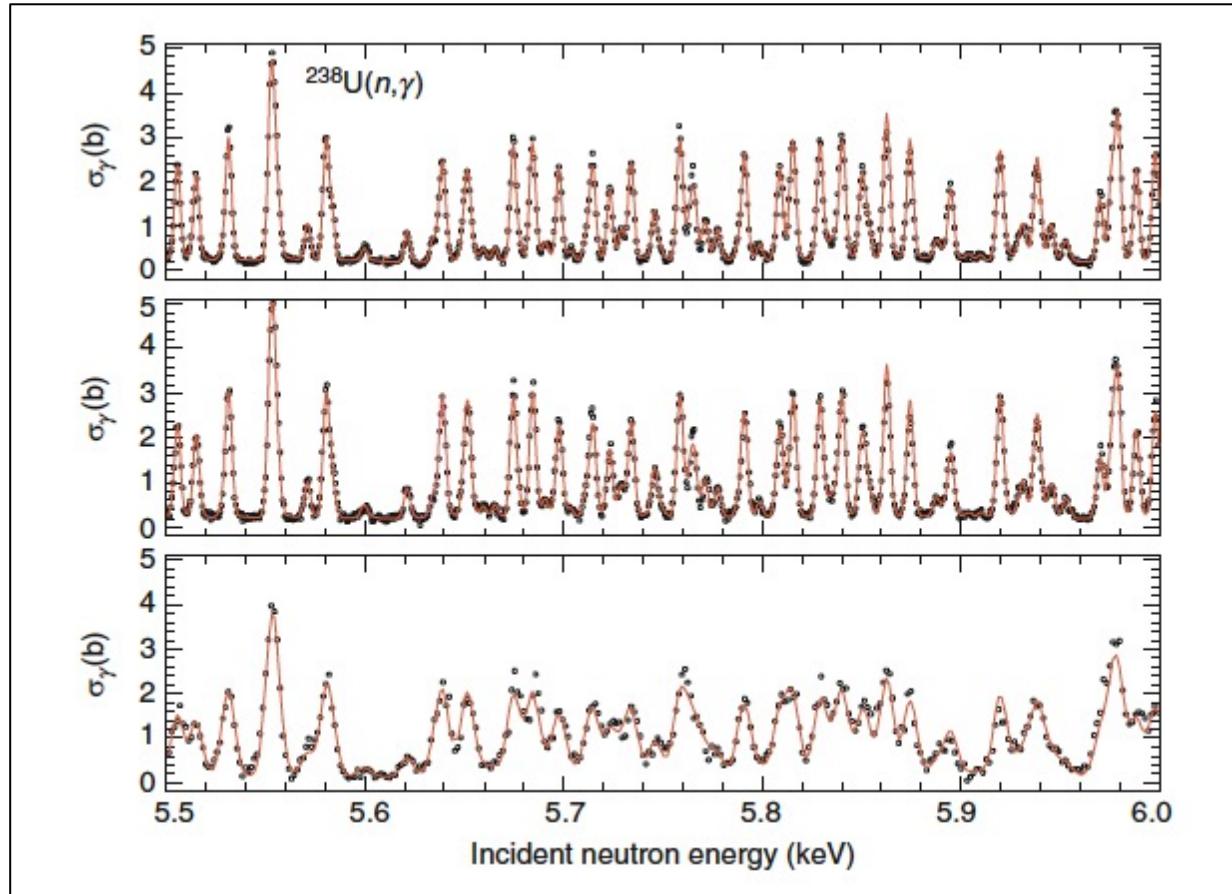
Thermal and resonance region: interference



- Iron filtered beams
- Shielding problems

Thermal and resonance region

- Specific codes are used to “fit” observed resonances: SAMMY, REFIT



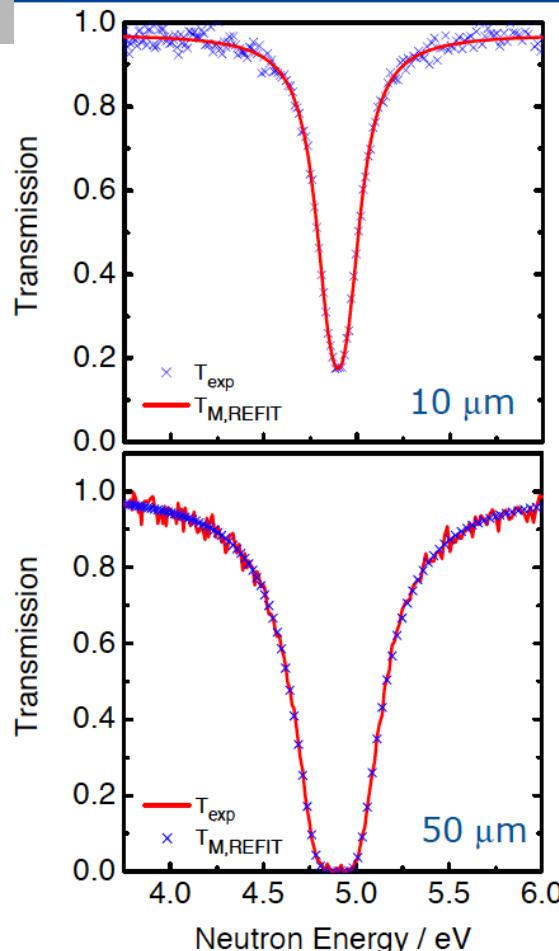
- And extract resonance parameters

Thermal and resonance region

- Specific codes are used to “fit” observed resonances: SAMMY, REFIT

4.9 eV resonance for $^{197}\text{Au} + \text{n}$

$^{197}\text{Au} : I^\pi = 3/2^+$



Resonance shape analysis with REFIT

$\ell = 0$ from shape, spin $J = 2$ from fit with capture data

$\Delta_D \sim 80 \text{ meV}$ and $\Delta_R \sim 5 \text{ meV}$ ($L = 50 \text{ m}$)

$$\Gamma_n = (15.06 \pm 0.08) \text{ meV}$$

$$\Gamma_\gamma = (121.7 \pm 1.3) \text{ meV}$$



$$\Gamma_n = (14.66 \pm 0.30) \text{ meV}$$

$$\Gamma_\gamma = (124.8 \pm 3.7) \text{ meV}$$

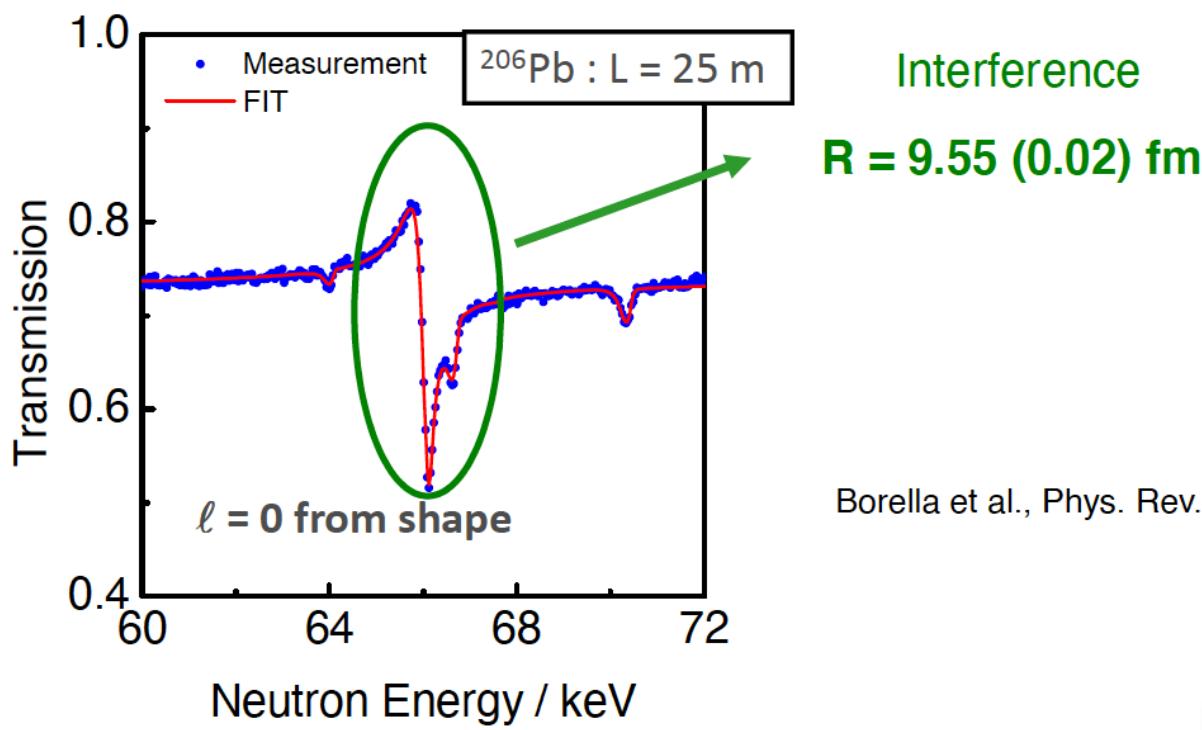
Uncertainties only due to counting statistics

Thermal and resonance region

- Specific codes are used to “fit” observed resonances: SAMMY, REFIT

Determination of scattering radius

$$\sigma_{\text{tot}}(E_n) = g \frac{\pi}{k_n^2} \frac{\Gamma_n \Gamma}{(E_n - E_R)^2 + (\Gamma/2)^2} + g \frac{4\pi}{k_n} \frac{\Gamma_n (E_n - E_R) R}{(E_n - E_R)^2 + (\Gamma/2)^2} + 4\pi R^2$$



Thermal and resonance region

- Specific codes are used to “fit” observed resonances: SAMMY, REFIT

Determination of statistical factor

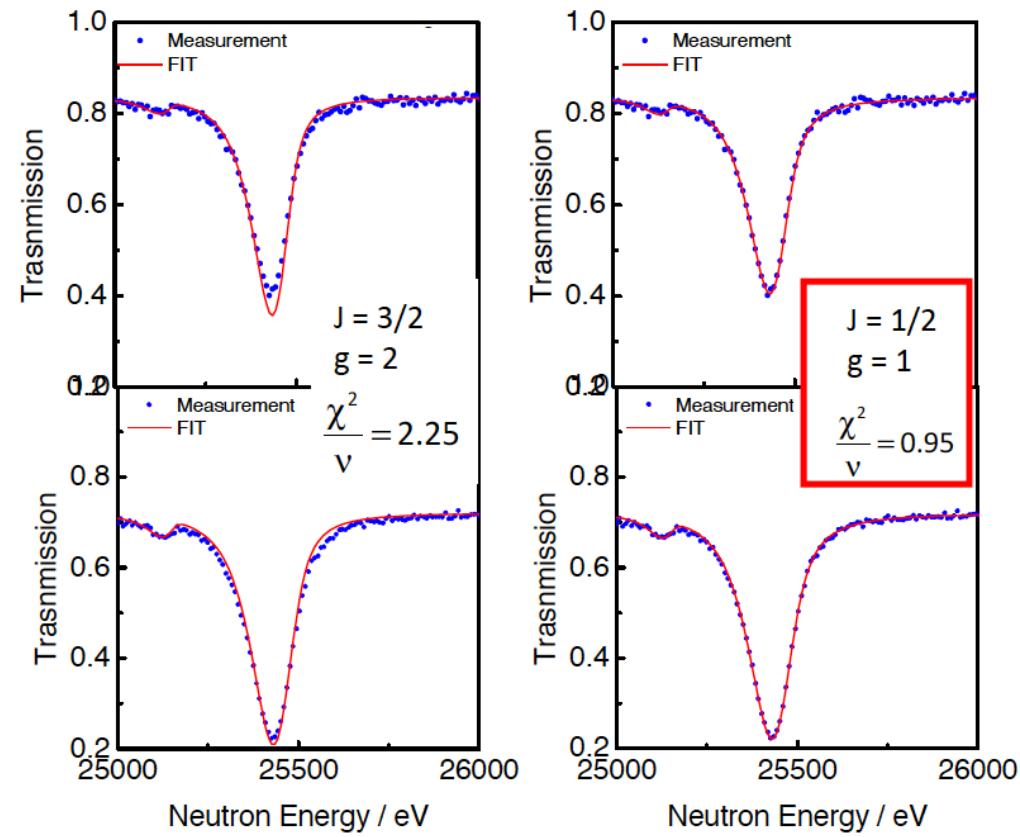
$^{206}\text{Pb} (I^\pi = 0^+) + n$

$\ell = 1$ from shape

$$g = \frac{2J+1}{2(2\ell+1)}$$

$\Rightarrow J = 1/2$

$\Rightarrow g = 1$

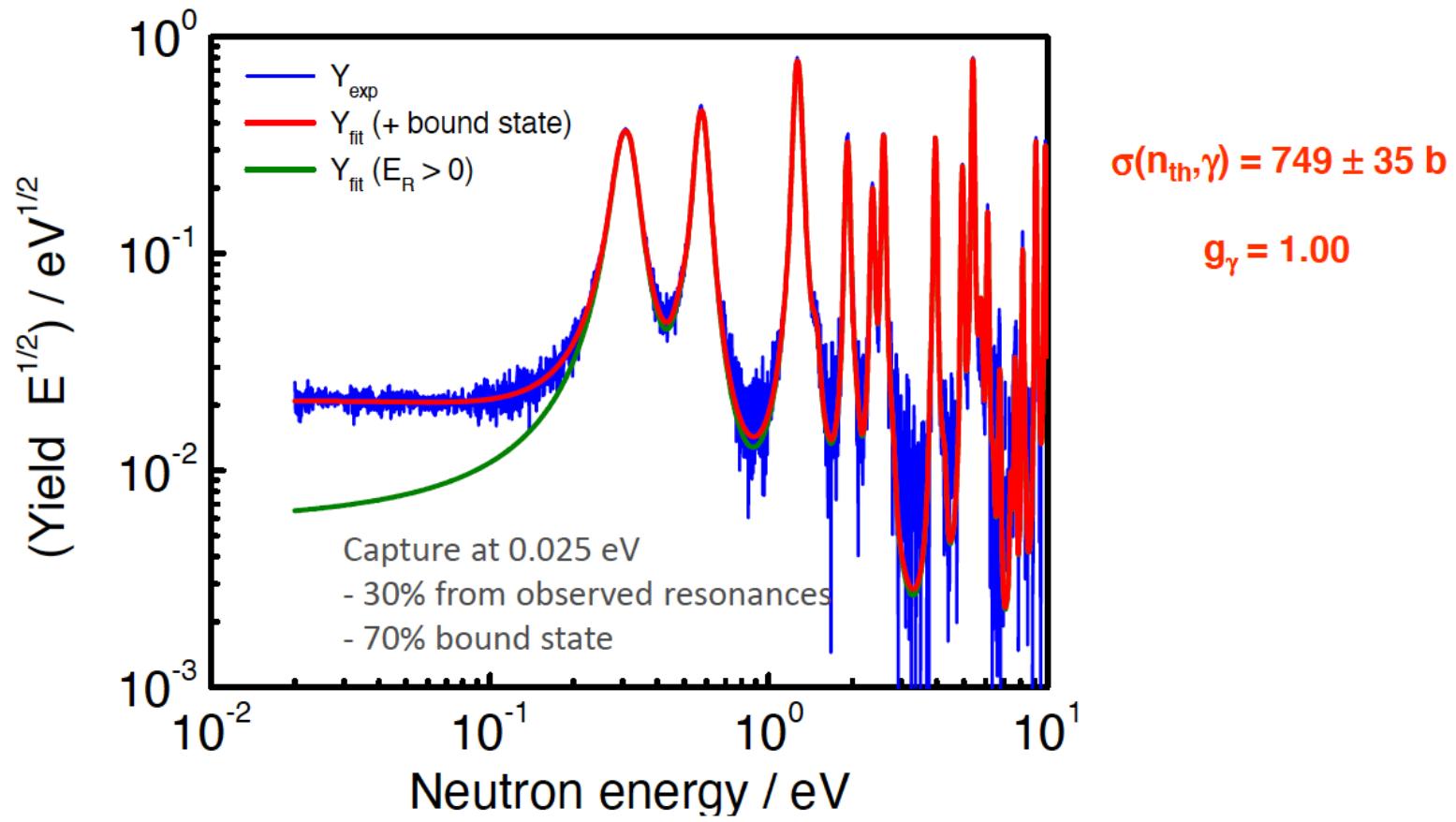


Borella et al., Phys. Rev. C 76 (2007) 014605

Thermal and resonance region

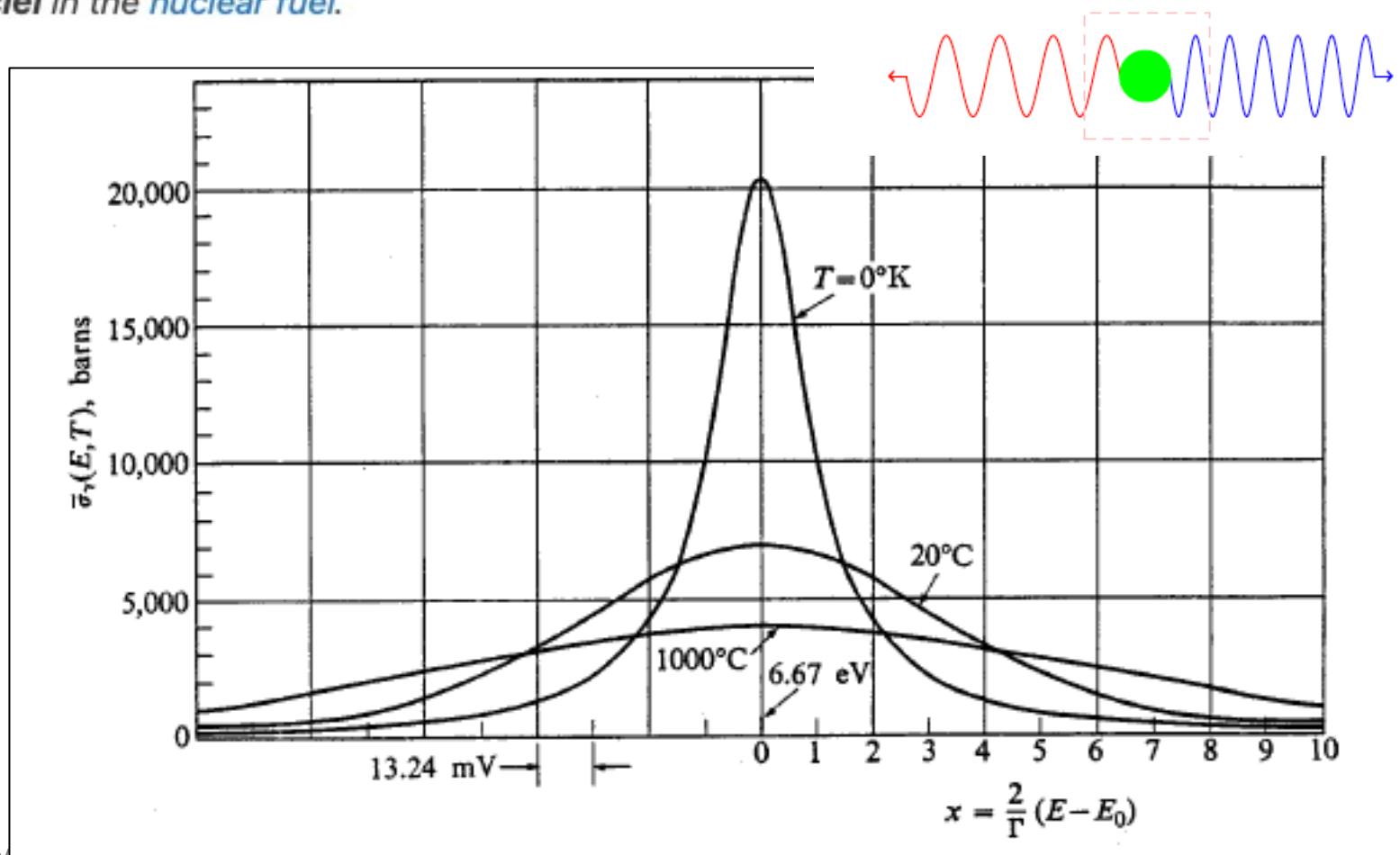
- Specific codes are used to “fit” observed resonances: SAMMY, REFIT

$^{241}\text{Am}(n,\gamma)$: results of RSA with REFIT



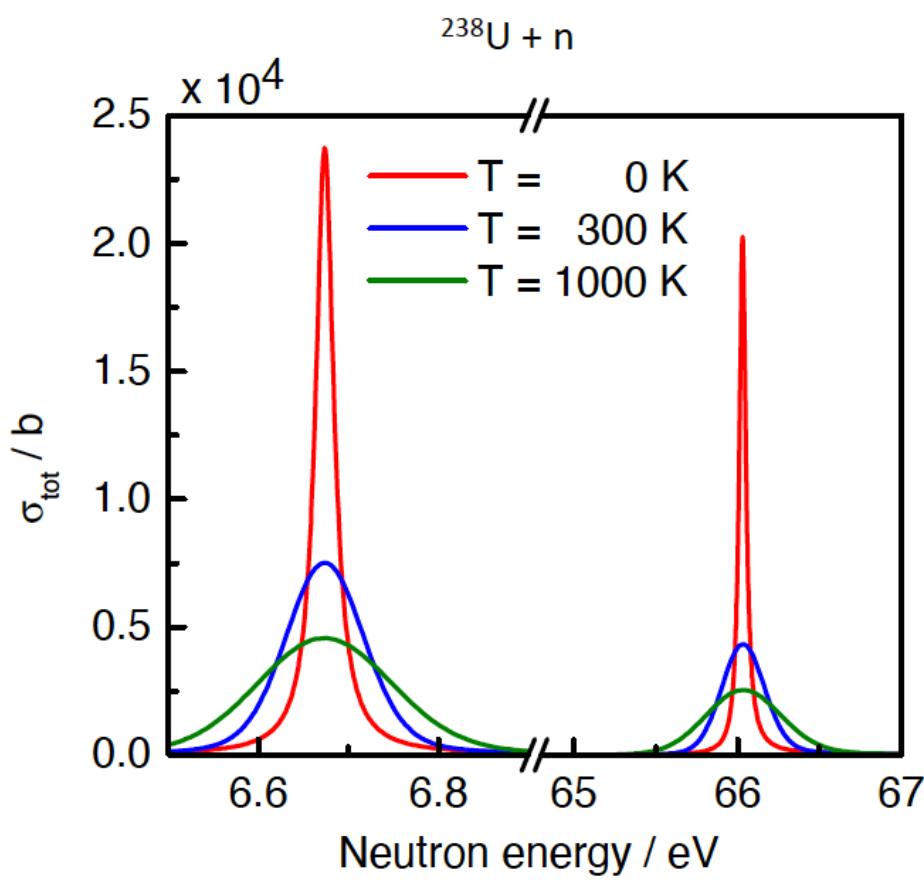
Resonance region: Doppler broadening

In general, **Doppler broadening** is the broadening of spectral lines due to the **Doppler effect** caused by a distribution of kinetic energies of molecules or atoms. In reactor physics, a particular case of this phenomenon is the **thermal Doppler broadening of the resonance capture cross-sections** of the **fertile material** (e.g., ^{238}U or ^{240}Pu) caused by the **thermal motion of target nuclei** in the **nuclear fuel**.



Resonance region: Doppler broadening

- The reaction rate is maintained



$$\bar{\sigma}(v, T) = \frac{1}{v} \int |v - V| \sigma(|v - V|) P(V) dV$$

Free gas model

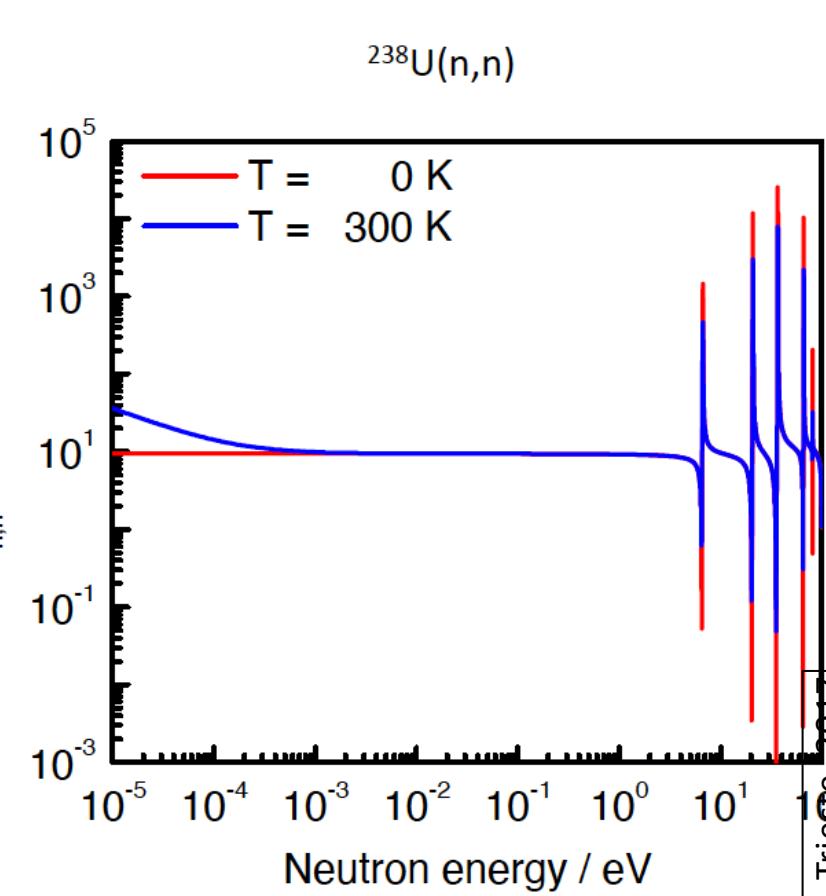
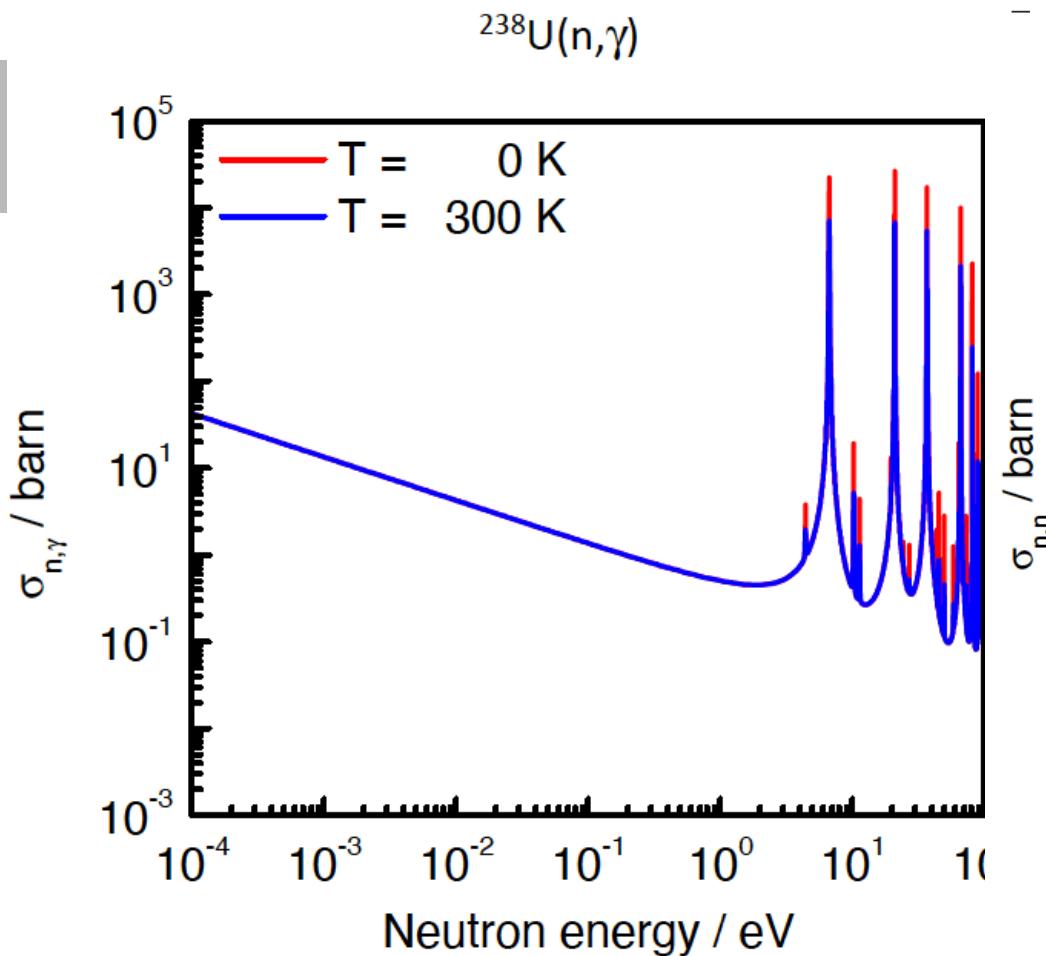
$$\bar{\sigma}(E, T) = \int S(E, E') \sigma(E', T = 0) dE'$$

$$S(E, E') = \frac{1}{\Delta_D \sqrt{\pi}} e^{-\left(\frac{E' - E}{\Delta_D}\right)^2} \sqrt{\frac{E'}{E}}$$

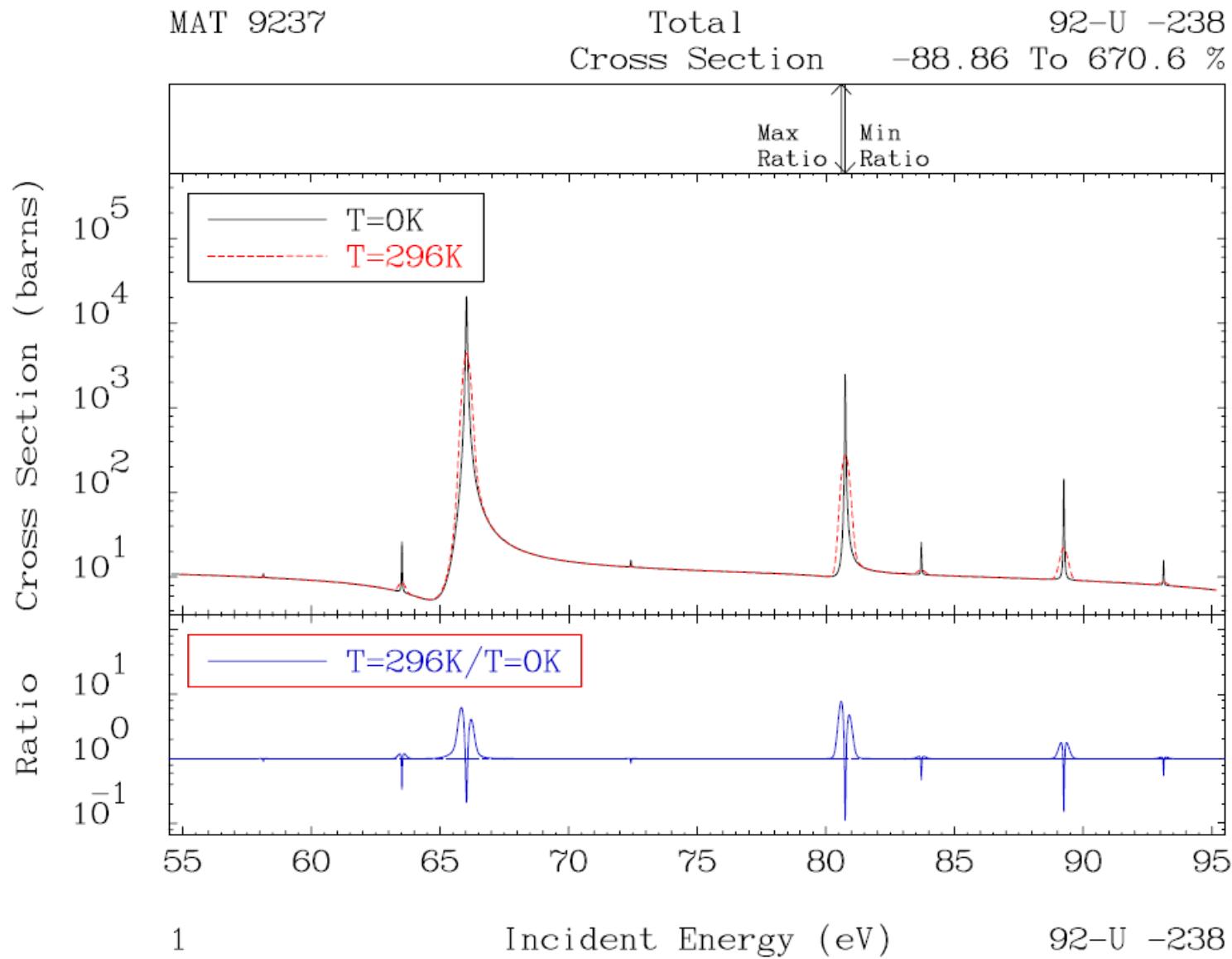
$$\Delta_D = \sqrt{\frac{4 E k_B T}{m_X / m_n}}$$

$$\text{FWHM} = 2 \sqrt{\ln 2} \Delta_D$$

Resonance region: Doppler broadening

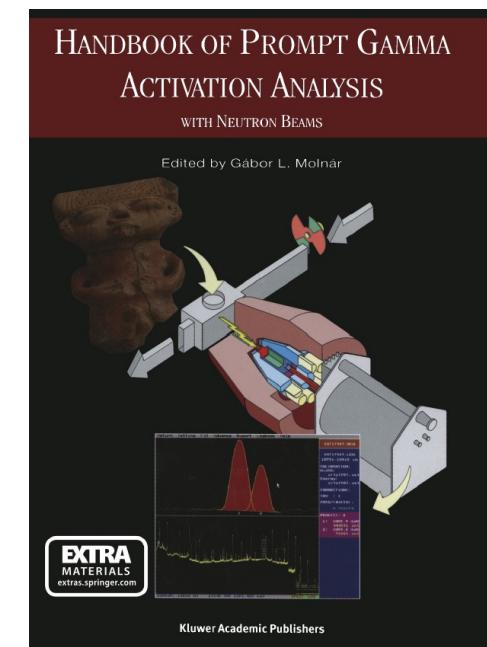
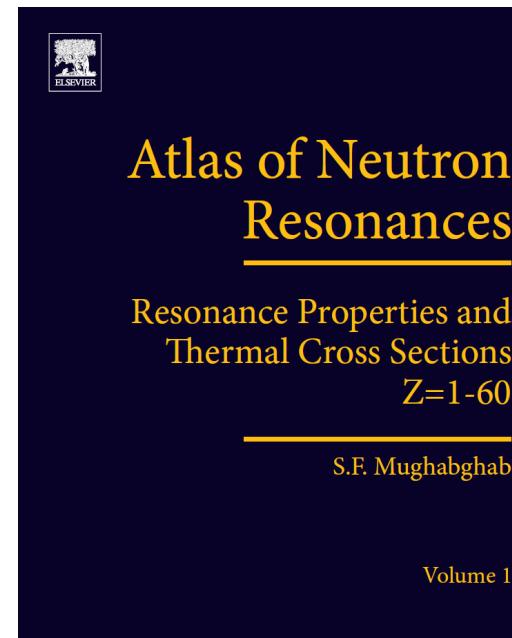


Resonance region: Doppler broadening



Thermal and resonance region

- Additionally, recommended resonance parameters can be found, such as in the “Atlas of Neutron Resonances”



IAEA.org
International Atomic Energy Agency

Neutron Activation Analysis Portal

provided by the IAEA Nuclear Data Section

Nuclear Data

The IAEA makes available a large amount of nuclear data relevant to users of the k_0 method. Some of these have been produced through Coordinated Research Projects, or Data Development initiatives, within the IAEA Nuclear Data Section, whereas others have been adopted from external partners. This webpage brings together these data sources for the convenience of users.

Compilation of k_0 and related nuclear data

An electronic database is available from the IUPAC website, which has been [updated recently](#). The database can be downloaded and used locally to access k_0 -values and associated nuclear data. This database was also recommended by an IAEA-NDS CRP.

Prompt Gamma-rays from Neutron Activation Analysis

The PGAA Coordinated Research Project webpages provide documentation and access to data. The data can be conveniently viewed through a specially designed interface.

Evaluated Gamma-ray Activation File

The EGAF database was produced during the above Coordinated Research Project. The data are made available in the ENDF6 format.

Atlas of Neutron Capture Cross Sections

The ATLAS database provides convenient access to cross section plots and data tables for the capture cross sections of 739 targets from H-1 to Cm-248.

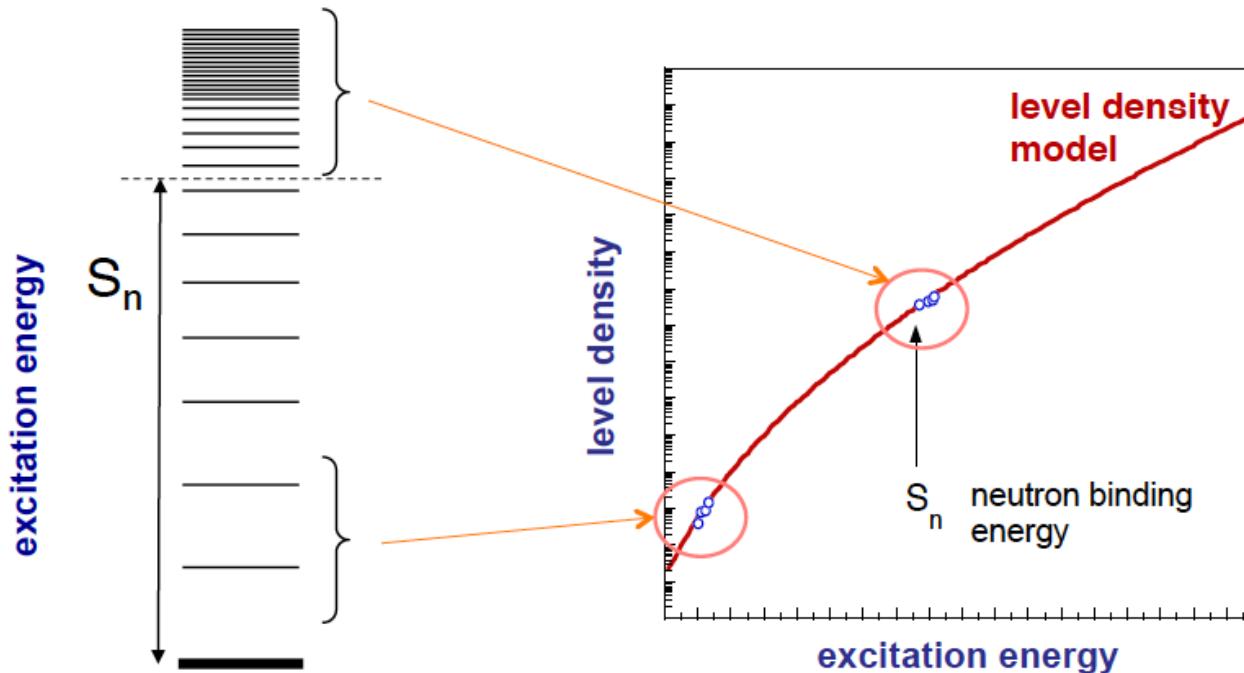
Thermal and resonance region

- Additionally, recommended resonance parameters can be found, such as in the “Atlas of Neutron Resonances”

$^{66}_{30}\text{Zn}$		$^{66}_{30}\text{Zn}$					
THERMAL CROSS SECTIONS							
$\sigma_\gamma = 0.62 \pm 0.06 \text{ b}$ $\sigma_s = 4.4 \pm 0.3 \text{ mb}$ $\sigma_a < 0.020 \text{ mb}$ $b_{coh} = 5.98 \pm 0.05 \text{ fm}$ $g_\gamma = 1.004$ $R' = 6.21 \pm 0.30 \text{ fm}$							
RESONANCE PROPERTIES							
$D_0 = 4.7 \pm 0.4 \text{ keV}$ $D_1 = 0.84 \pm 0.05 \text{ keV}$ $S_0 = 1.9 \pm 0.2$ $S_1 = 0.70 \pm 0.07$ $\langle \Gamma_{\gamma 0} \rangle = 0.400 \pm 0.020 \text{ eV}$ $\langle \Gamma_{\gamma 1} \rangle = 0.190 \pm 0.060 \text{ eV}$ $S_{\gamma 0} = 0.85 \pm 0.09$ $S_{\gamma 1} = 2.26 \pm 0.73$ $I_{\gamma}^{\prime c} = 0.92 \pm 0.06 \text{ b}$ $\sigma_{\gamma}^{\prime c}(5 \text{ keV}) = 87.2 \pm 9.5$ $\sigma_{\gamma}^{\prime c}(30 \text{ keV}) = 36.7 \pm 4.0 \text{ mb}$ $\sigma_{\gamma}^{\prime c}(30 \text{ keV}) = 37.1 \pm 4.0 \text{ mb}$							
RESONANCE PARAMETERS							
$I^\pi = 0^+$		%Abn = 27.90	Sn = $7052.33 \pm 0.22 \text{ keV}$				
$\sigma_\gamma(+)$ = 0.227 b		$\sigma_\gamma(B) = 0.393 \text{ b}$	$\sigma_\gamma(D) = 0.050 \text{ b}$				
E_0 (keV)	J	I	$g\Gamma_n$ (eV)	Γ_γ (eV)	$g\Gamma_n^0$ (meV)	$g\Gamma_n^{-1}$ (meV)	$g\Gamma_n\Gamma_\gamma/\Gamma$ (meV)
-4.108	1/2	0		(0.4)	3980		
0.3235 ± 0.0009			0.011 ± 0.001	0.1665 ± 0.0200	0.61 ± 0.06		
3.2300 ± 0.0015	1		0.01			40	
3.5580 ± 0.0015	(1/2)	1	0.006 ± 0.002	(0.17)		20 ± 7	5.79 ± 0.18
4.046 ± 0.002	1/2	0	5.2 ± 0.5	0.198 ± 0.002	82 ± 8		194.6 ± 1.8
4.385 ± 0.002	(1/2)	1	0.07	0.156 ± 0.006		200	49.27 ± 0.64
5.204 ± 0.002	(1/2)	1	0.0021 ± 0.0002	(0.17)		3.9 ± 0.4	2.04 ± 0.19
5.3430 ± 0.0025	(1/2)	1	0.15	0.206 ± 0.005		270	85.10 ± 0.77

Resonance region: level density

- Used both in the resonance range and fast neutron range



low-lying levels:
Count levels, all J^π

neutron resonances:
Count levels, selected J^π ,
extract D_0

- All level density models reproduce the low-lying levels and D_0 at S_n

Resonance region: level spacing

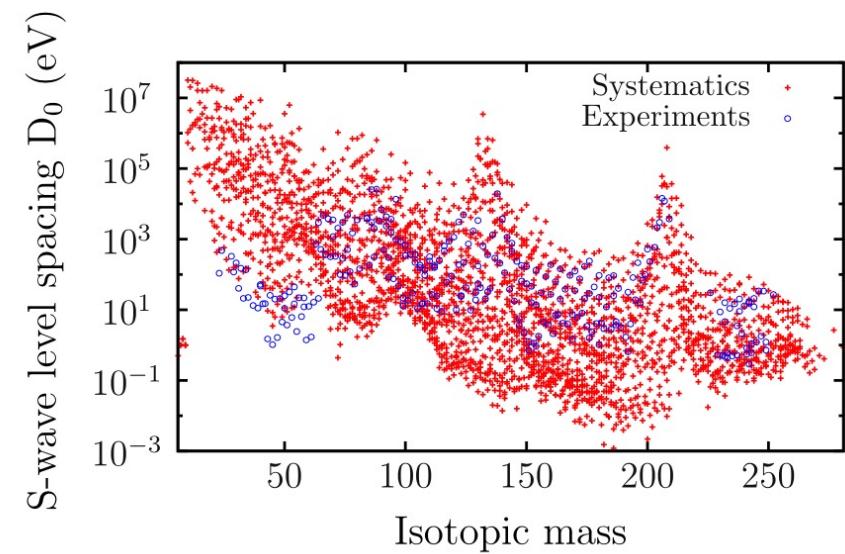
- Used both in the resonance range and fast neutron range

- The level spacing D_0 at the neutron binding energy is a crucial input parameter for calibrating level density models.
Level density: $\rho = 1/D$.

- D_0 is the spacing between levels excited by neutrons on nuclei bringing in zero orbital momentum (s-wave resonances).

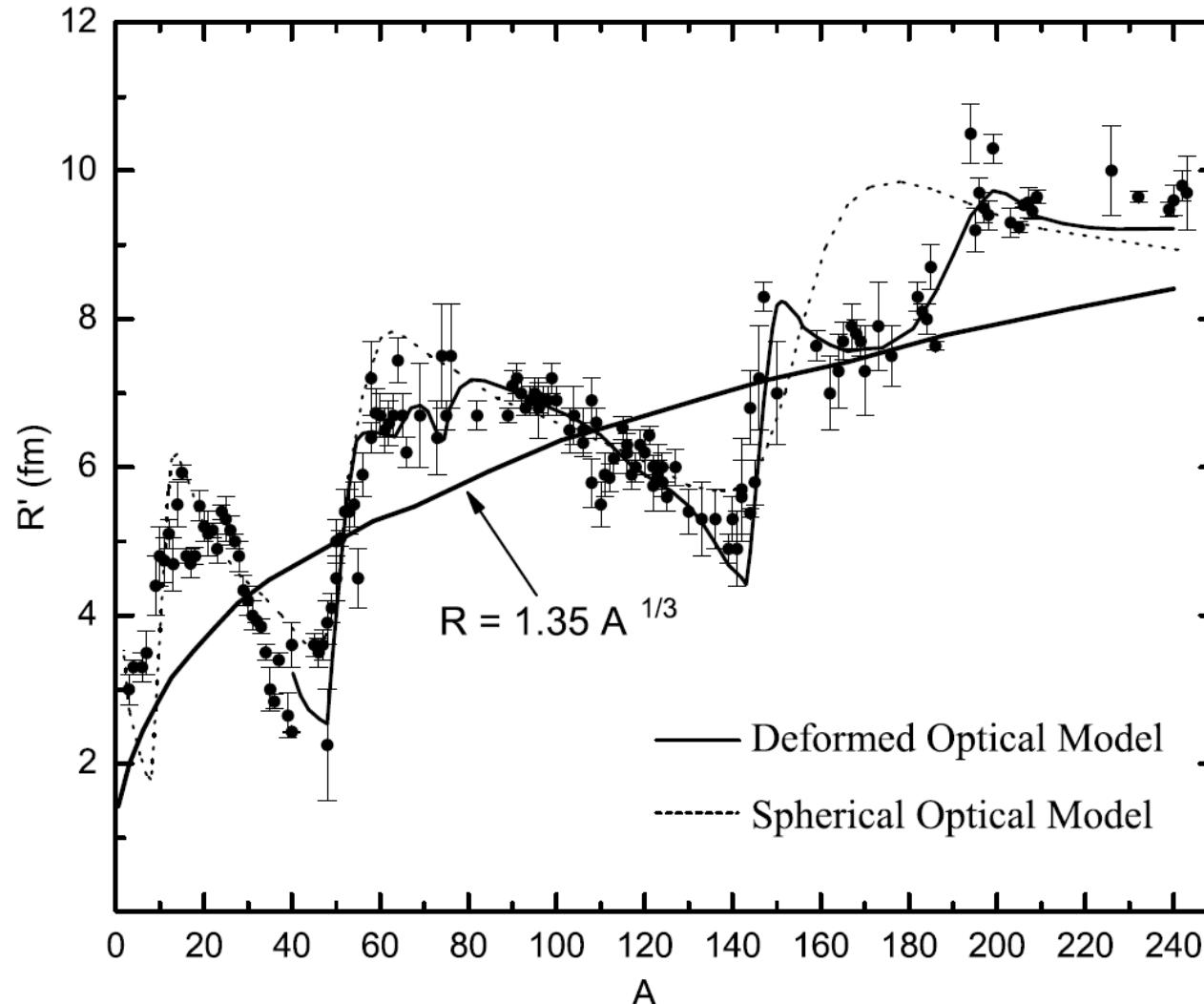
- Spacings from higher orbital momentum are equally important, but in general much more affected by missing levels.

- Problems concerning the determination of D_0 :
 - spin and parity assignment of levels
 - corrections for missing levels (which are not observed experimentally)



Resonance region: scattering radius R

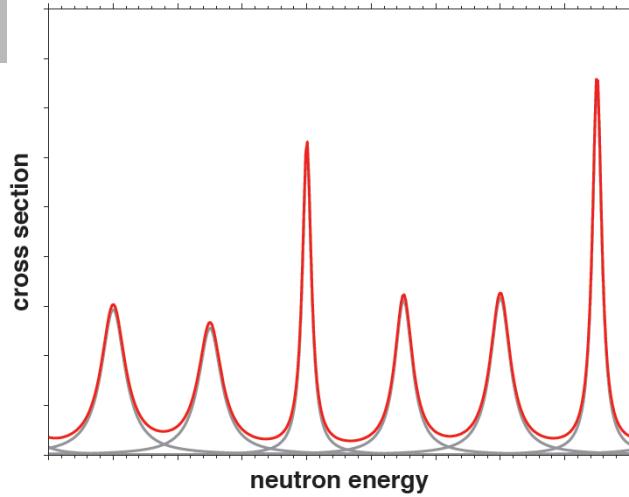
- Used for the potential scattering cross section ($1.35A^{1/3}$, A target mass)



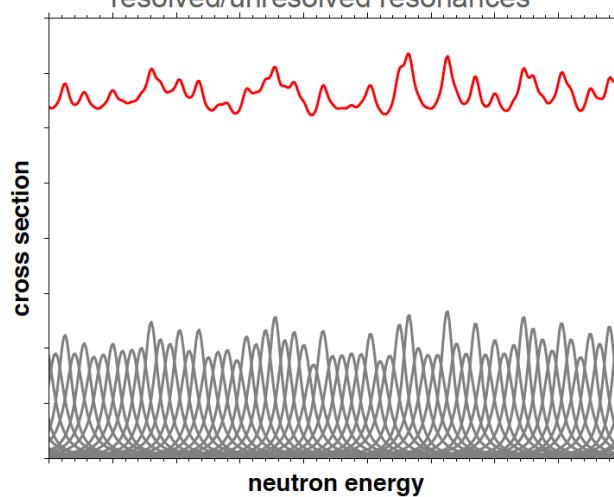
Limit Resonance – fast neutron range

- The limit between the resonance energy range and fast neutron range is called the **unresolved resonance range**

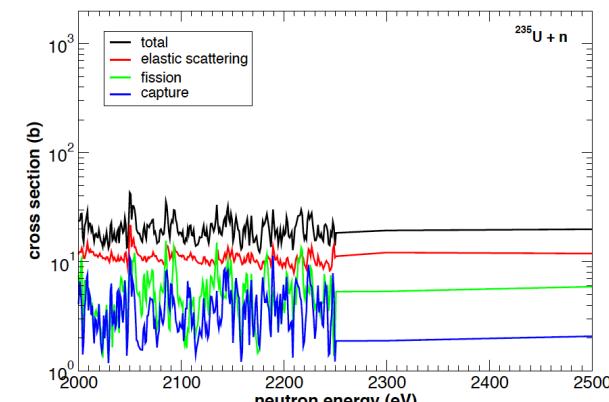
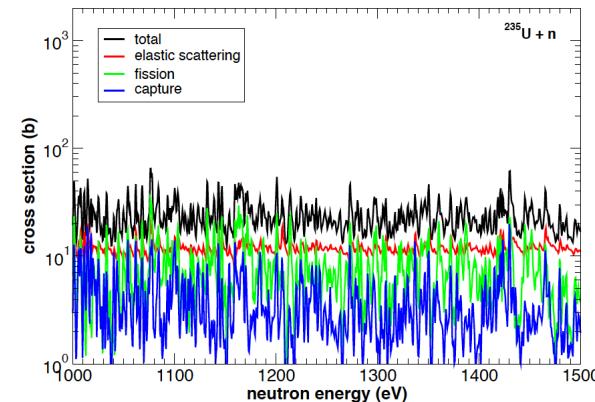
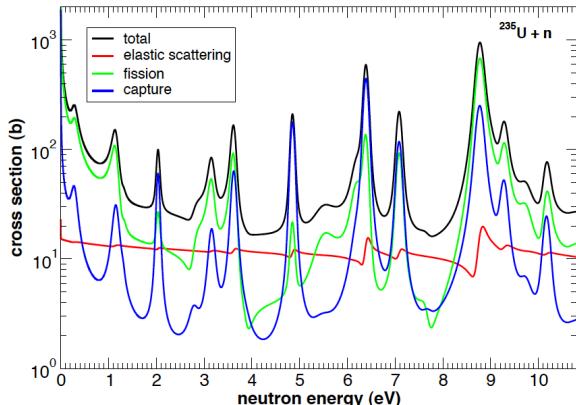
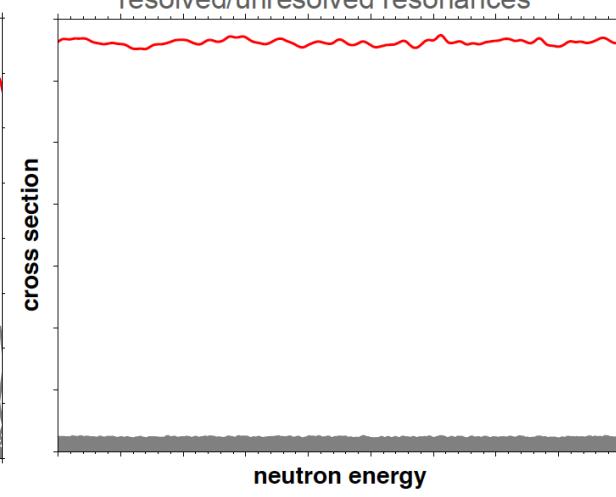
resolved/unresolved resonances



resolved/unresolved resonances

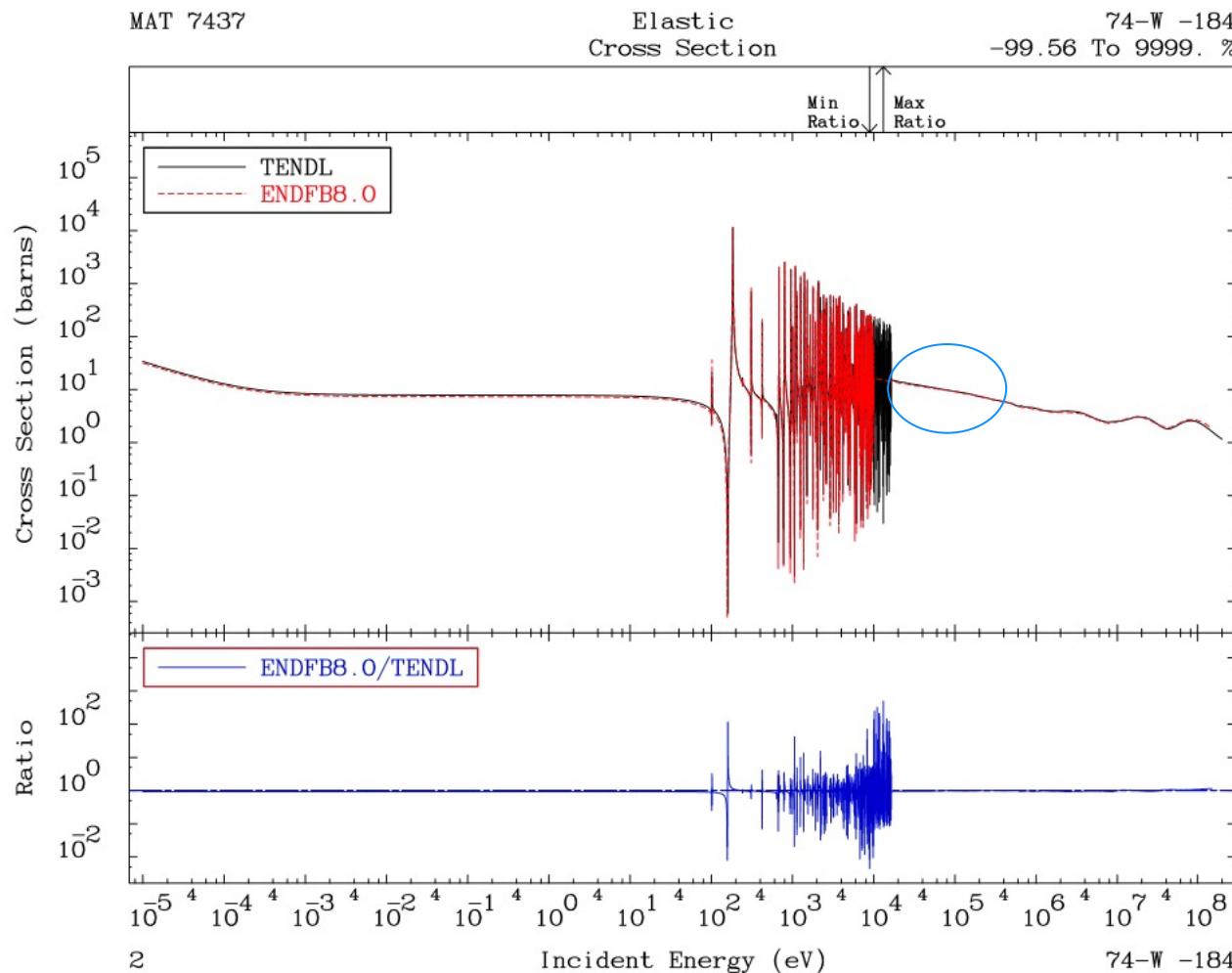


resolved/unresolved resonances



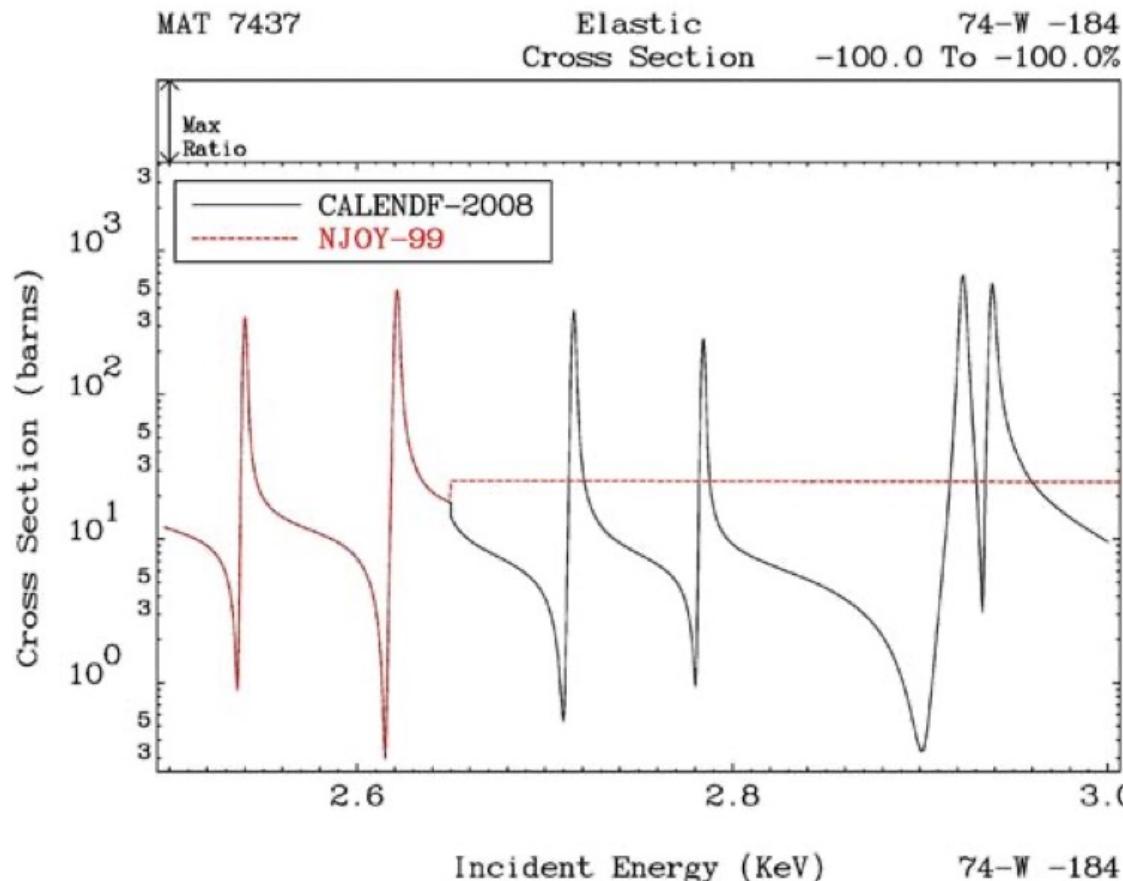
Unresolved resonance range

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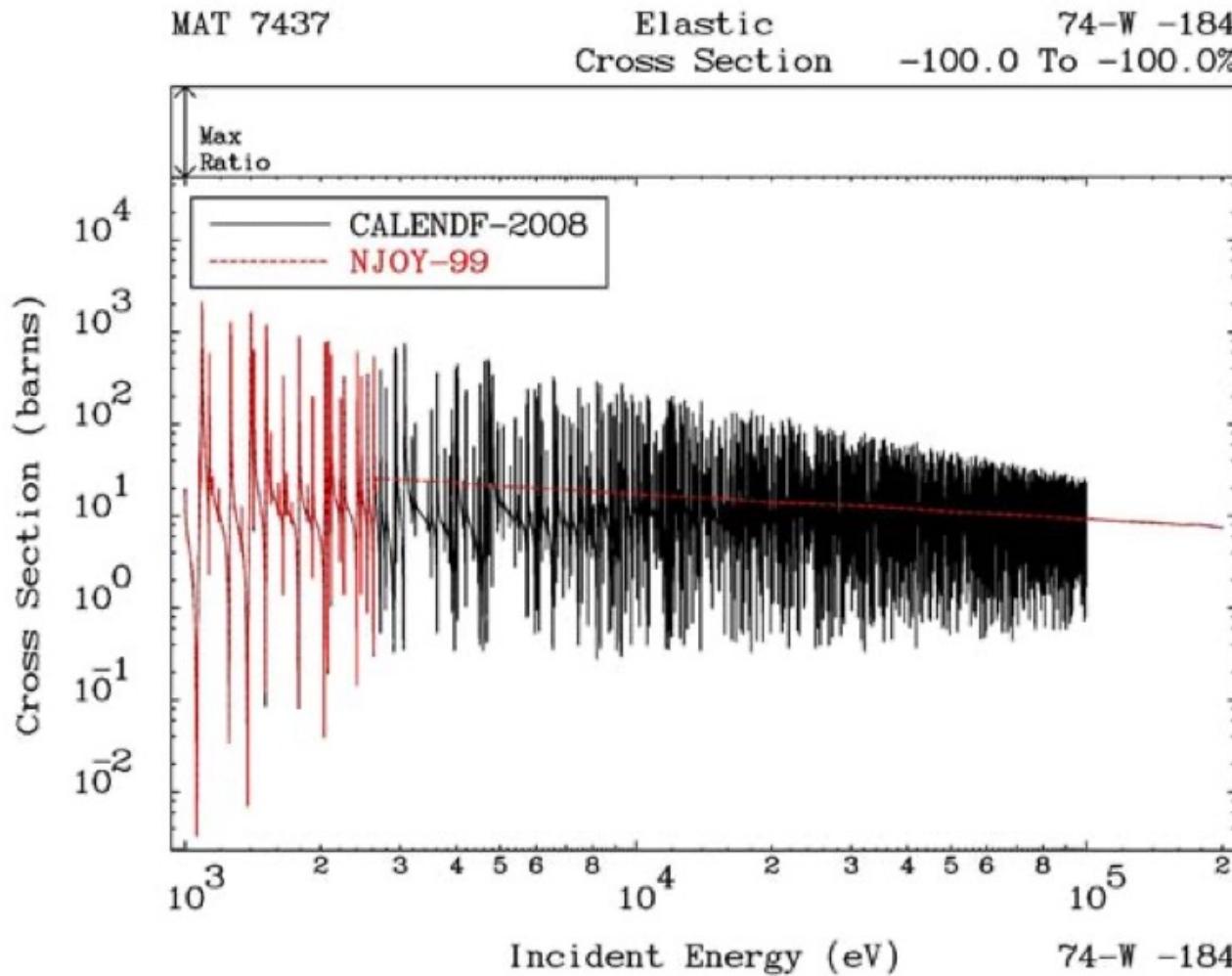
Unresolved resonance range

- The limit between the resonance energy range and fast neutron range is called the **unresolved resonance range**

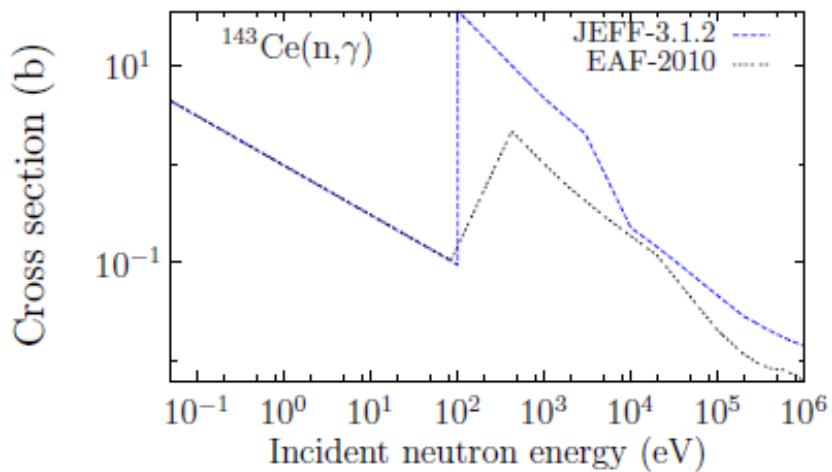
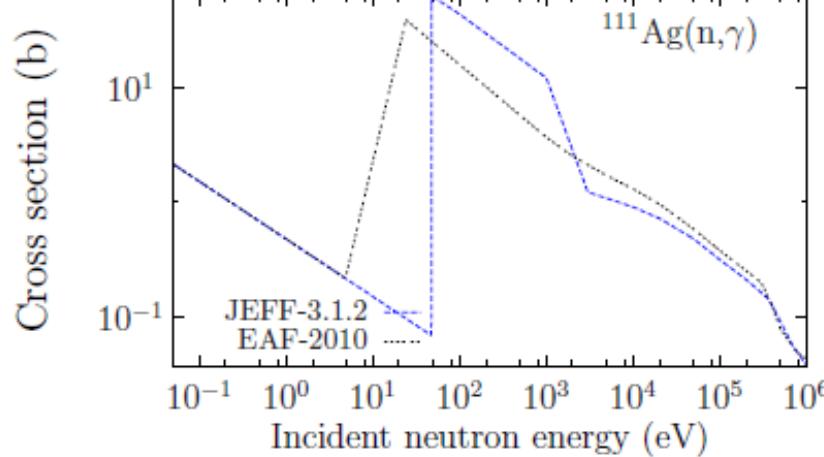
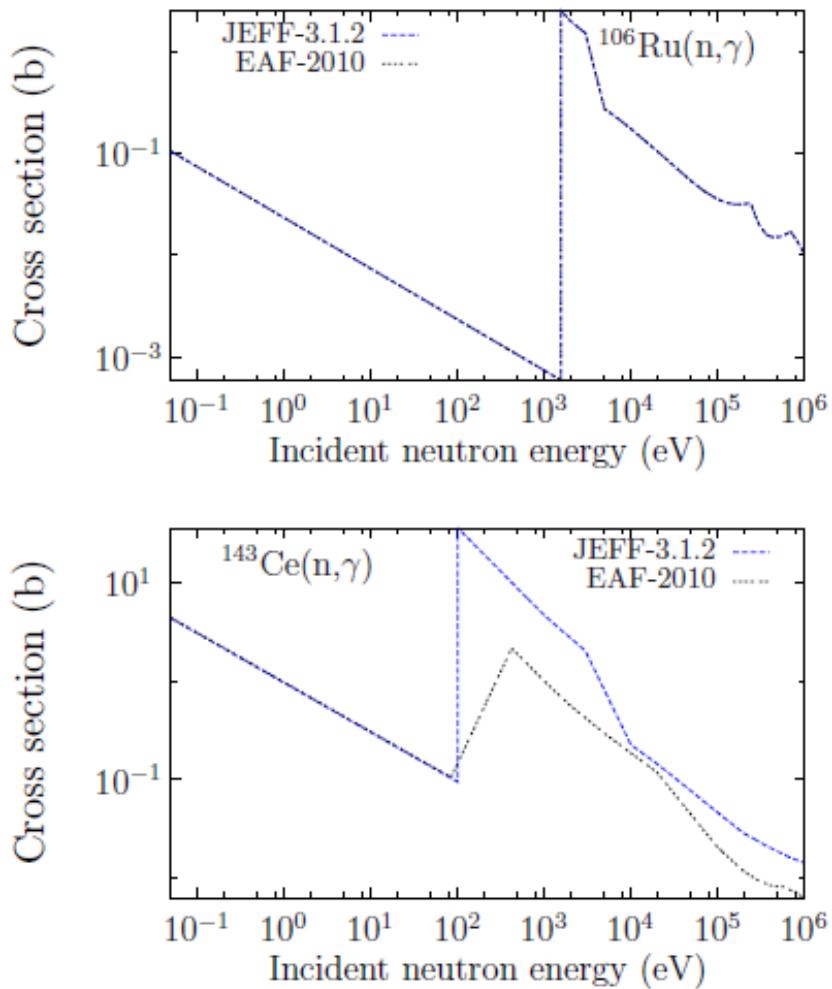
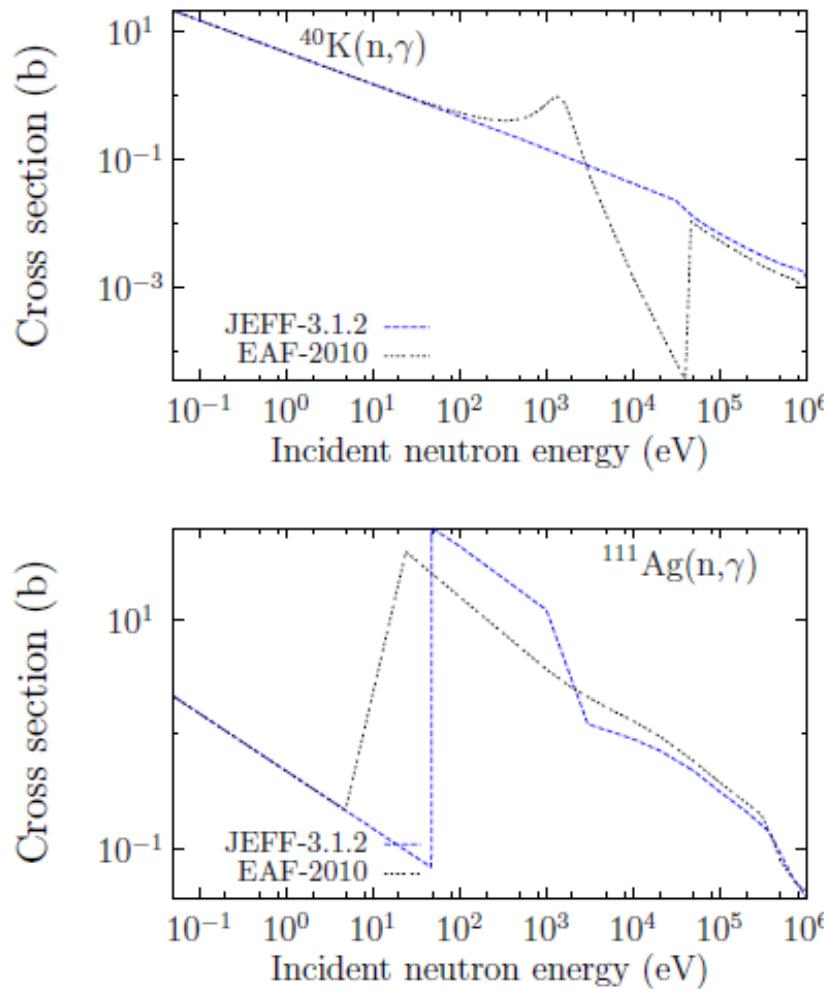


Unresolved resonance range (URR)

- The limit between the resonance energy range and fast neutron range is called the **unresolved resonance range**

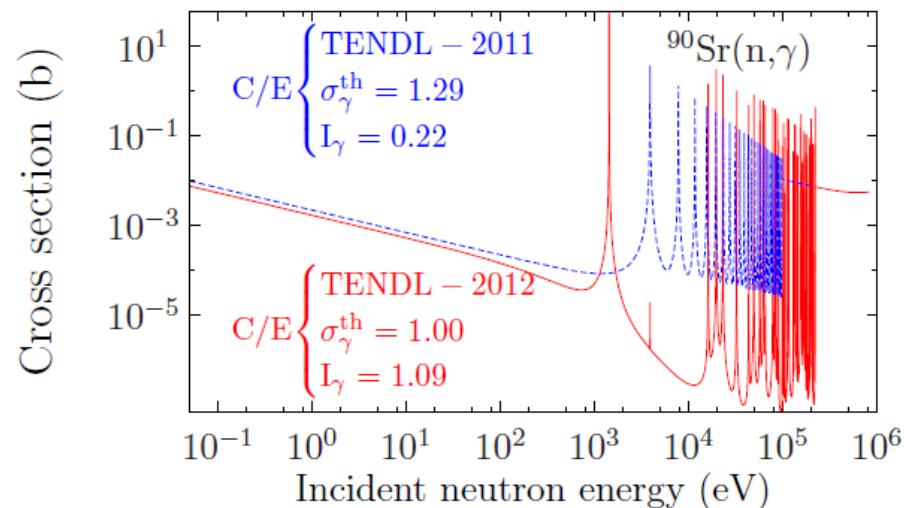
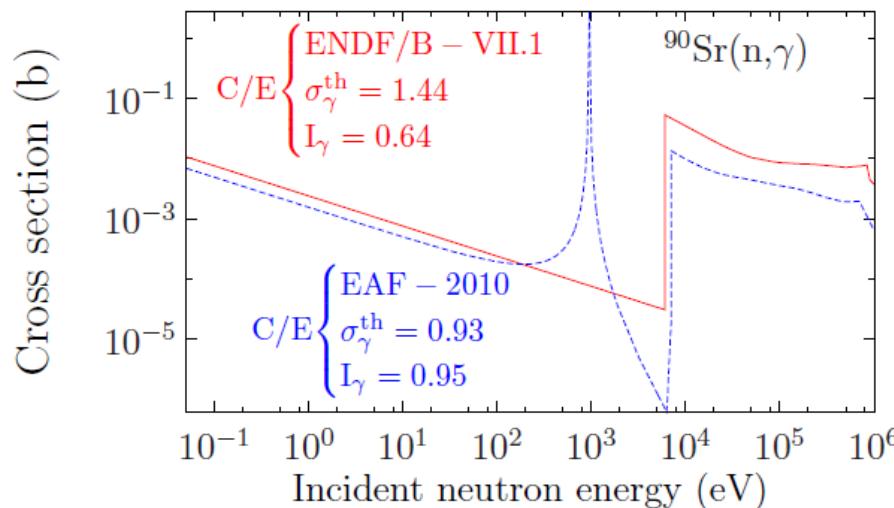


URR: create “statistical” resonances



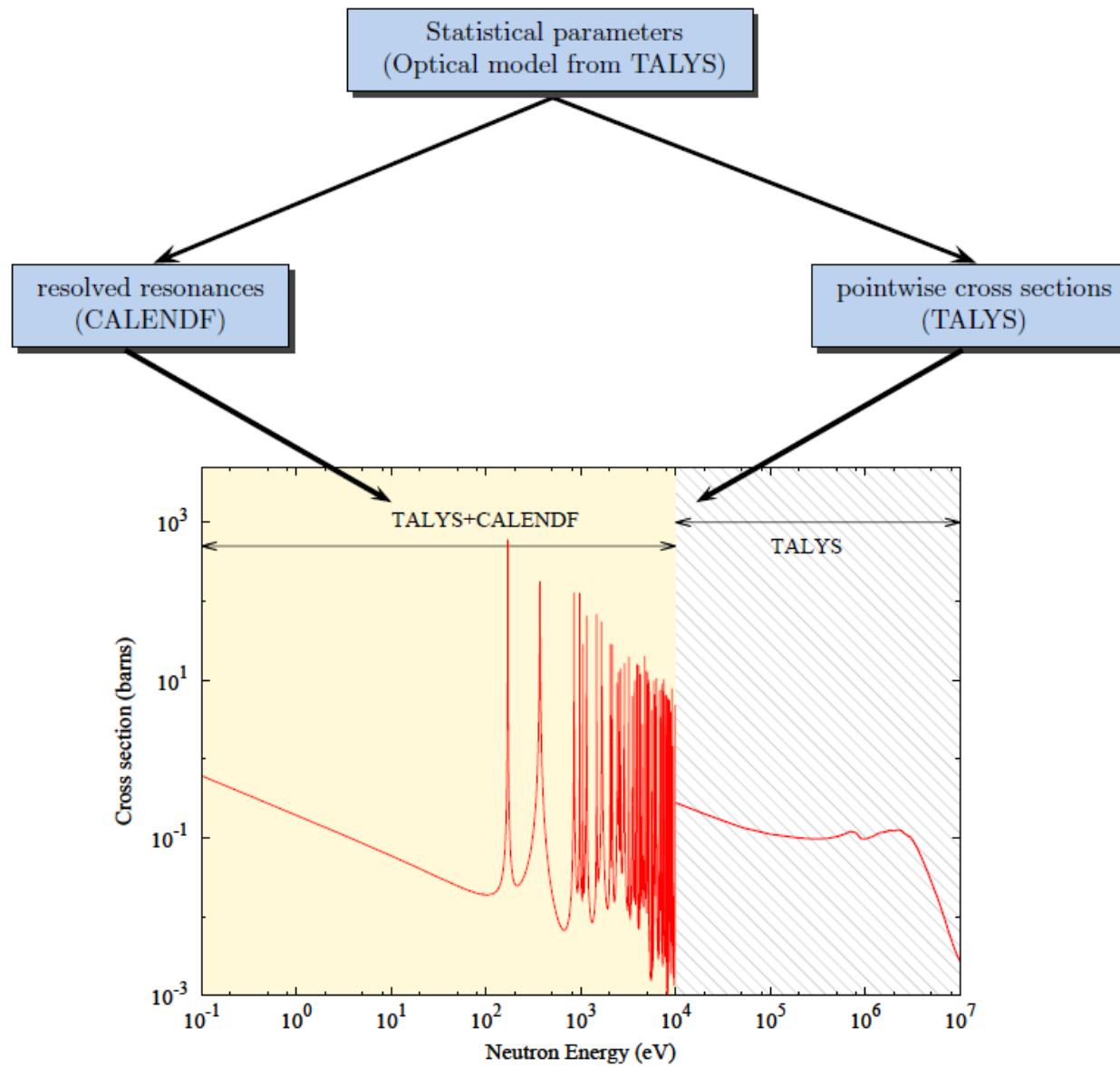
URR: create “statistical” resonances

Examples of different approaches for ^{90}Sr ($h_{1/2} = 28$ sec) in the low energy region.



Left: basic optical model calculation for ENDF/B-VII.1 and Single Resonance Approximation (SRA) for EAF-2010. Right: multi-SRA for TENDL-2011 and the present methodology for TENDL-2012.

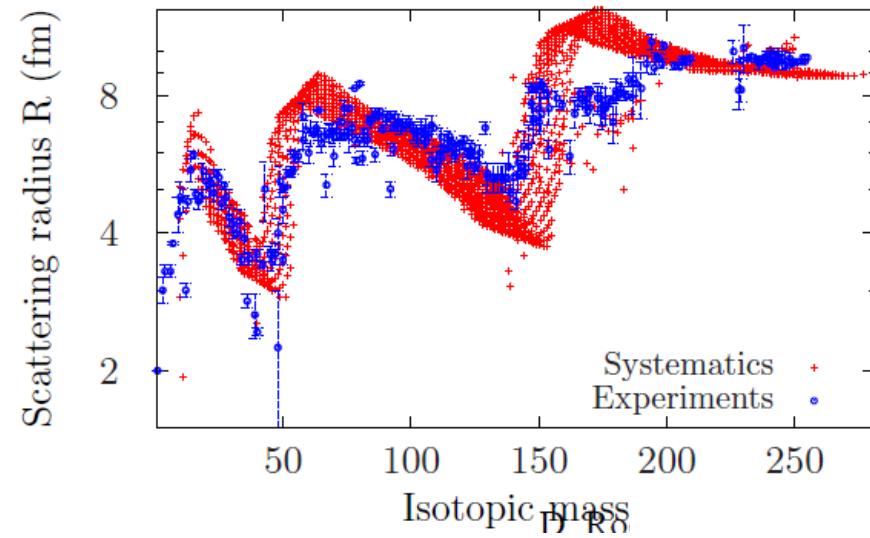
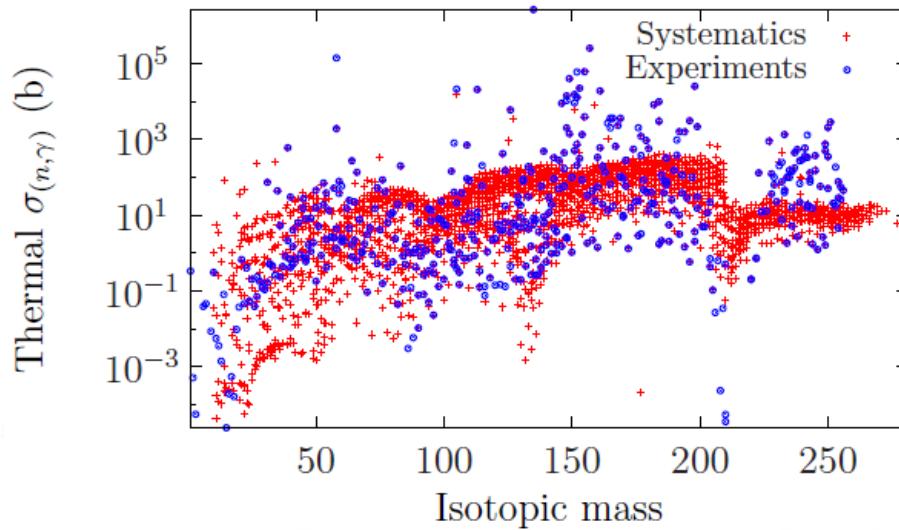
URR: create “statistical” resonances



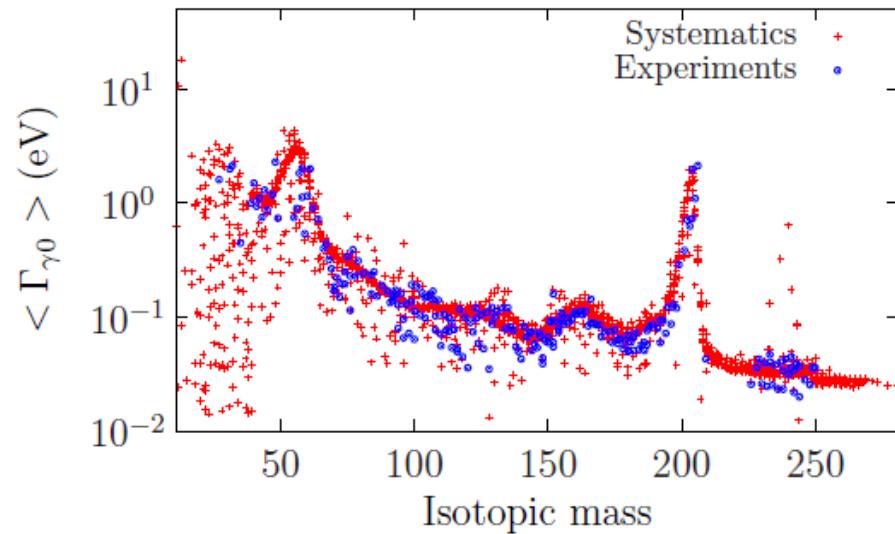
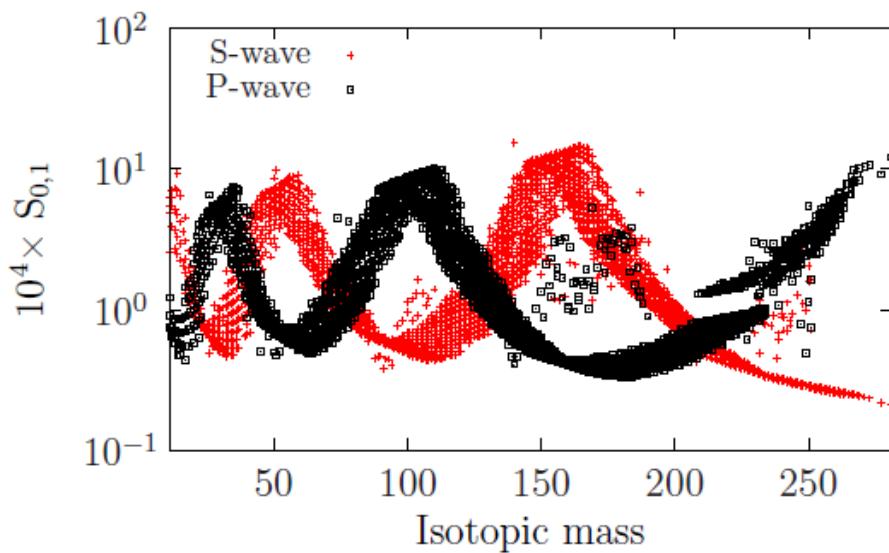
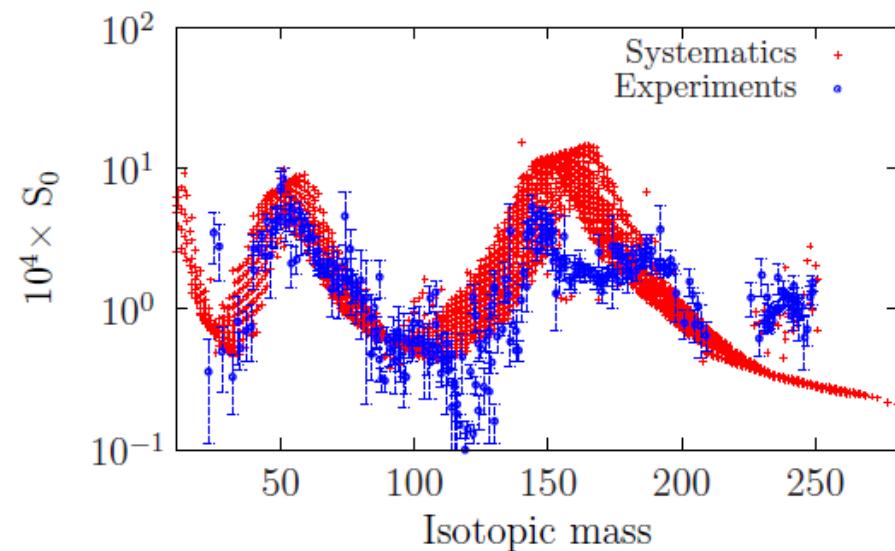
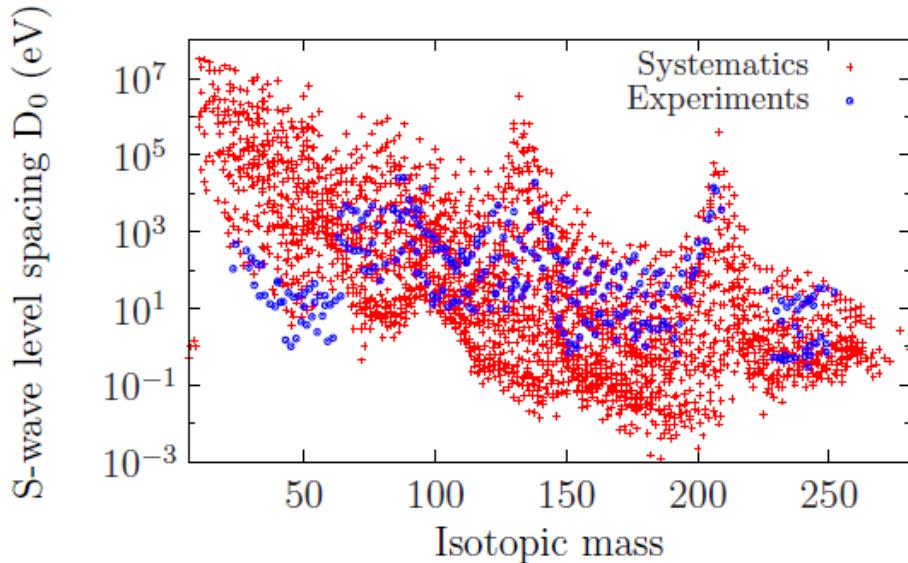
Statistical resonances

As a starting point energy-dependent statistical parameters as well as specific cross sections are needed in the whole energy range. These parameters are for each orbital angular momentum l and spin of the resonance state j :

- the scattering radius r ,
- the average level spacing D_0 ,
- the average reduced neutron width Γ_n^0 ,
- the average radiation width Γ_γ ,
- and if relevant the average fission width Γ_f .



Statistical resonances



Statistical resonances: 3 groups of isotopes

1. isotopes without any experimental reaction information (about 1600 isotopes). In this case, as no specific information can be used to adjust calculations, we fully rely on systematics, as defined in TALYS.
2. isotopes with scarce experimental data, such as thermal cross sections, resonance integrals, average cross sections at high energy (about 400 isotopes). Such isotopes are for instance ^{40}K , ^{54}Mn , ^{60}Co , ^{90}Sr , ^{105}Rh , ^{106}Ru , ^{109}Cd , ^{111}Ag , $^{138,143}\text{Ce}$ or ^{204}Hg .
3. isotopes with measured pointwise cross sections, resonances, integral measurements, and resolved resonance parameters (about 400 isotopes).

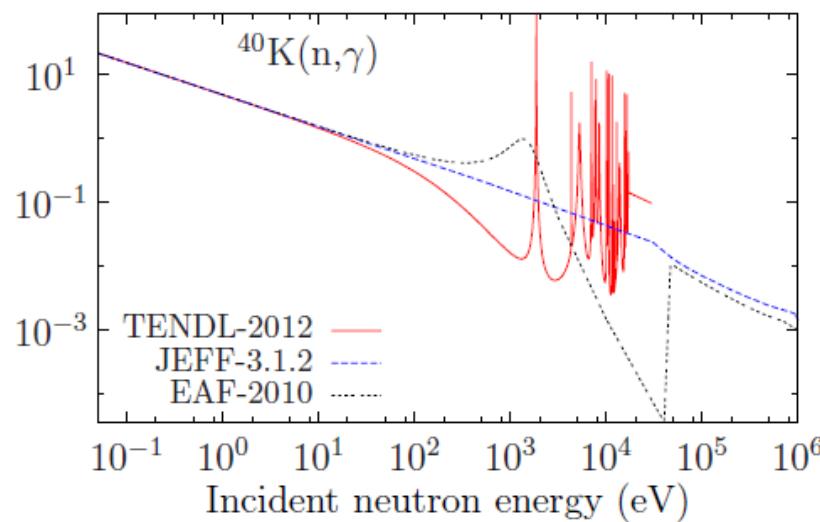
URR: Converting average parameters to statistical resonances

The idea is to generate random ladders of resonances using the statistical properties (as in the unresolved resonance range):

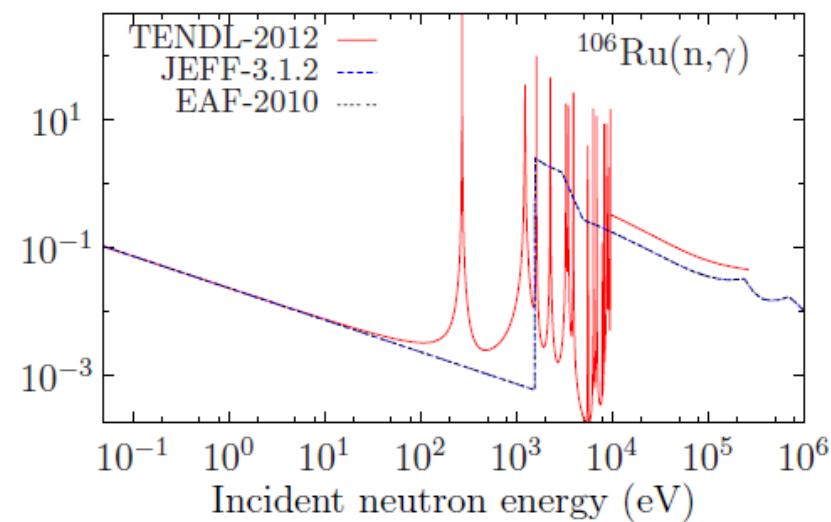
1. one ladder can be generated for an energy E by randomly selecting a starting resonance energy for one (l, j) sequence, and also randomly selecting a set of widths for that resonance using the appropriate average widths and χ^2 distribution functions.
2. We can then select the next higher resonance energy by sampling from the Wigner distribution for resonance spacings, and a new set of widths for that resonance can be chosen.
3. The process is continued until a long ladder of resonances for that (l, j) is obtained.
4. The process for the other (l, j) sequences is then repeated, each such sequence being uncorrelated in positions from the others.
5. for each (l, j) couples, a GOE random matrix (Gaussian Orthogonal Ensemble) is used to generate resonance energies (allowing to follow the Wigner law and to include correlations between two successive resonances).

URR: example for known thermal cross sections

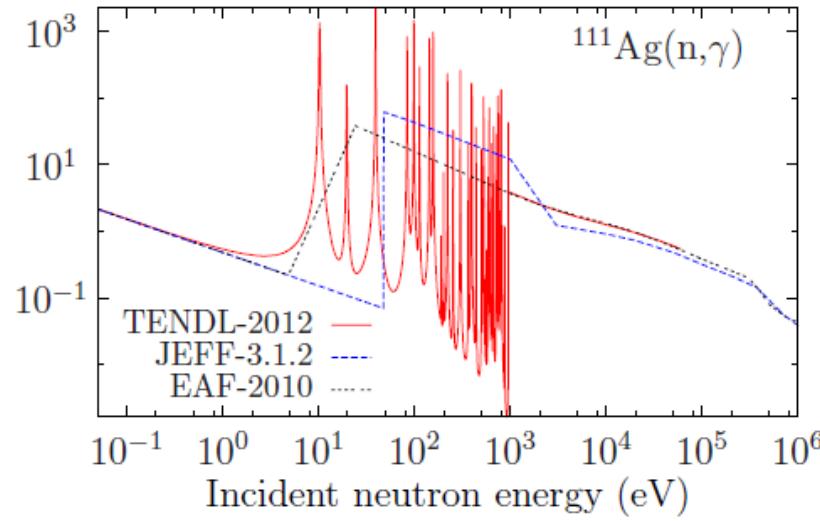
Cross section (b)



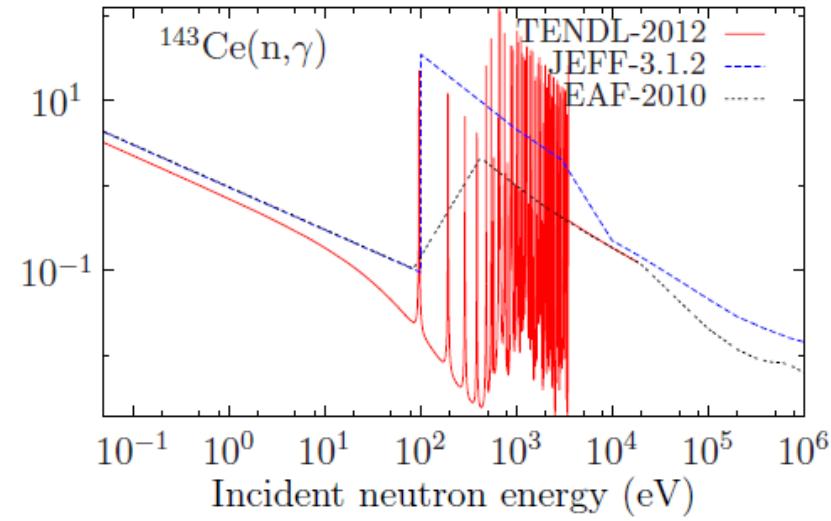
Cross section (b)



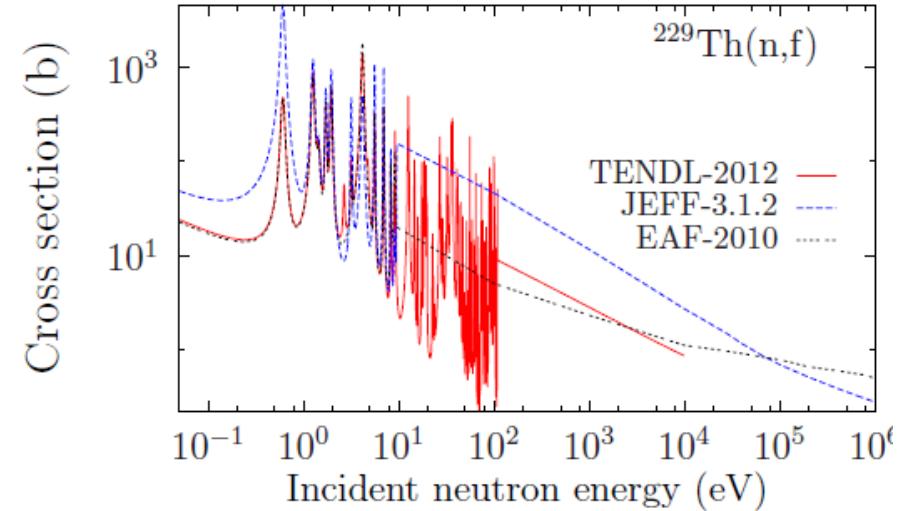
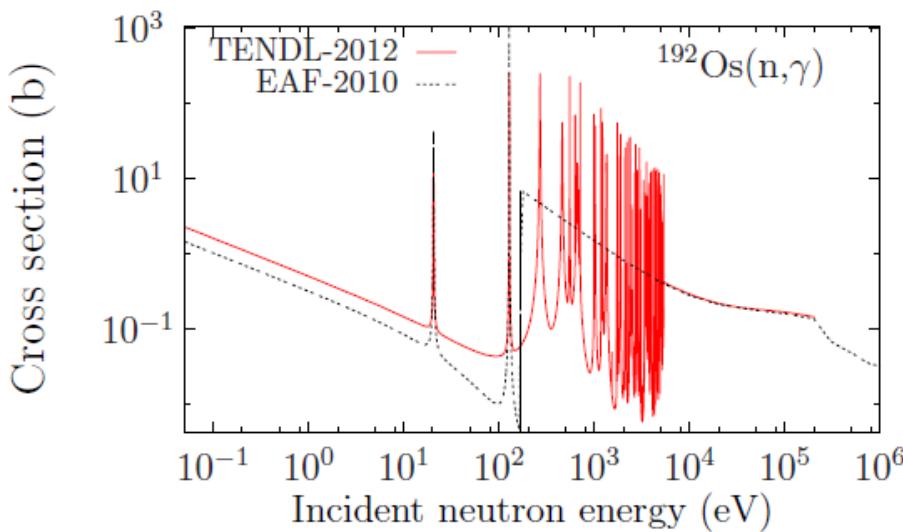
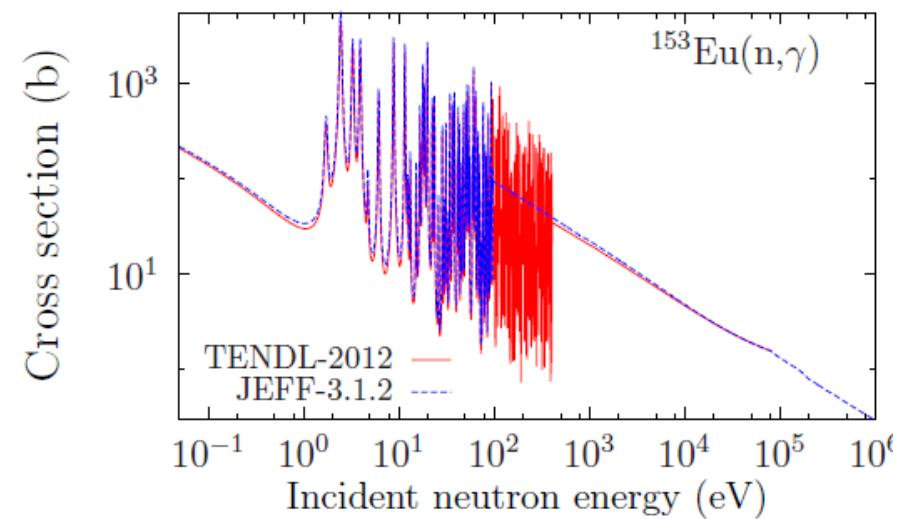
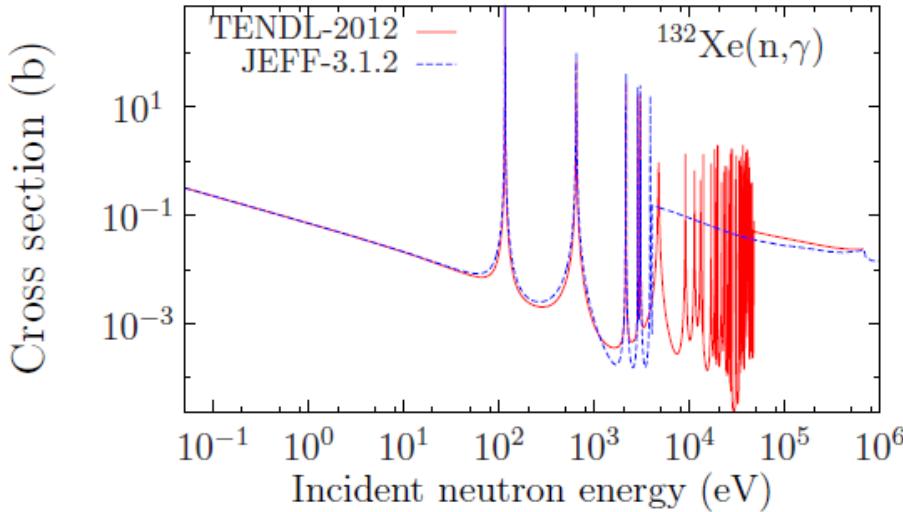
Cross section (b)



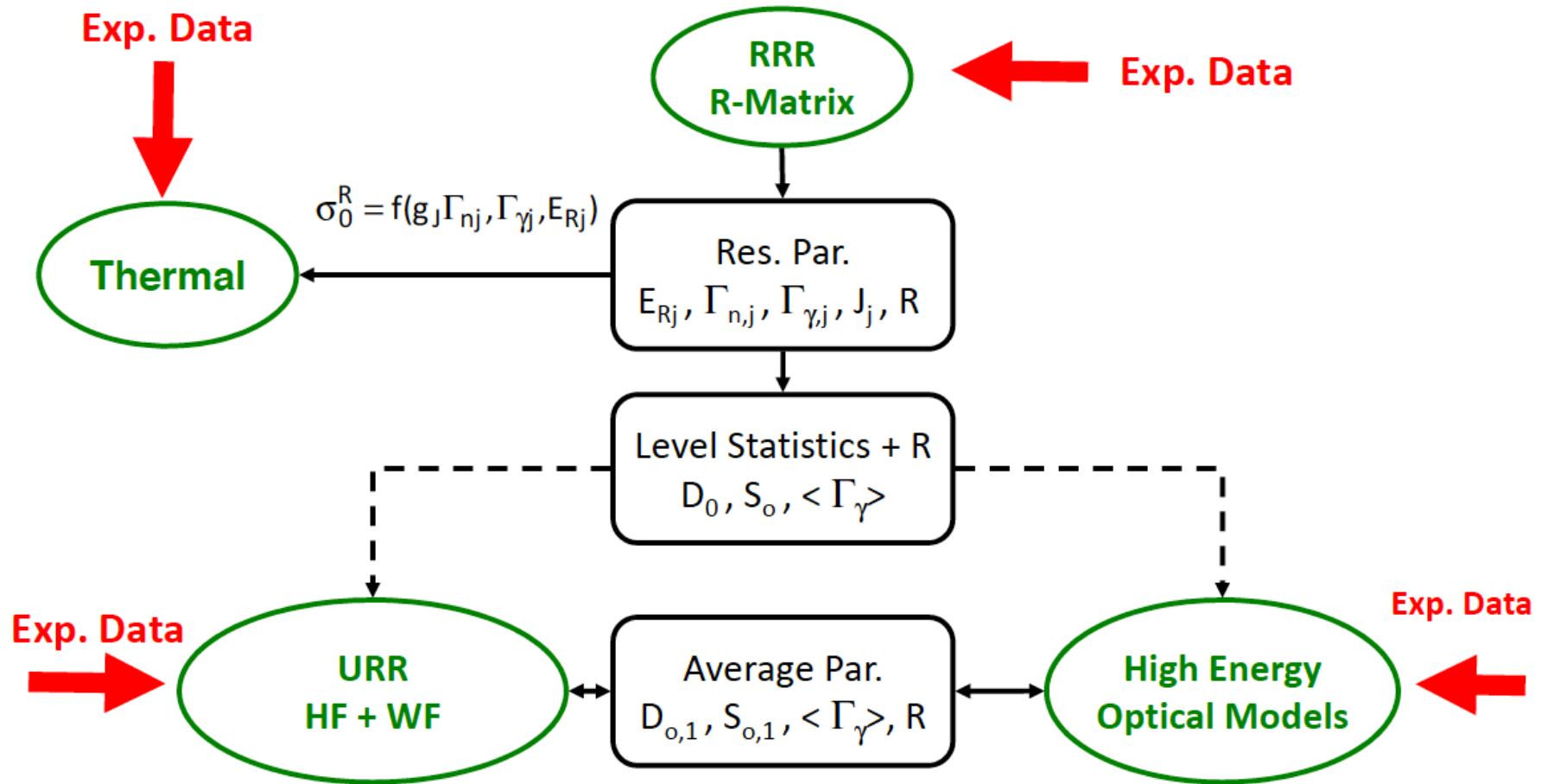
Cross section (b)



URR: example for known resonances

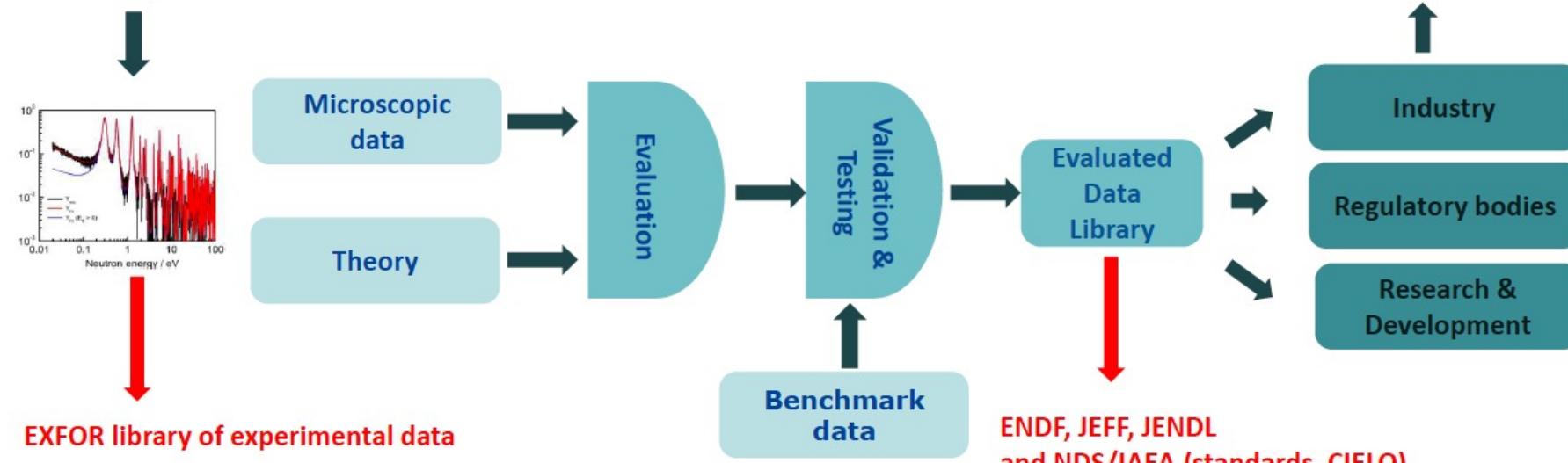
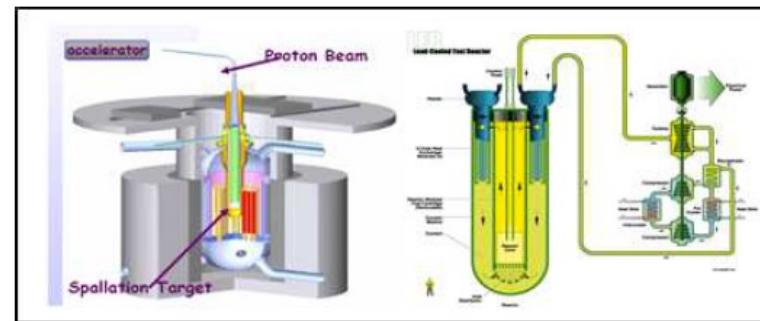
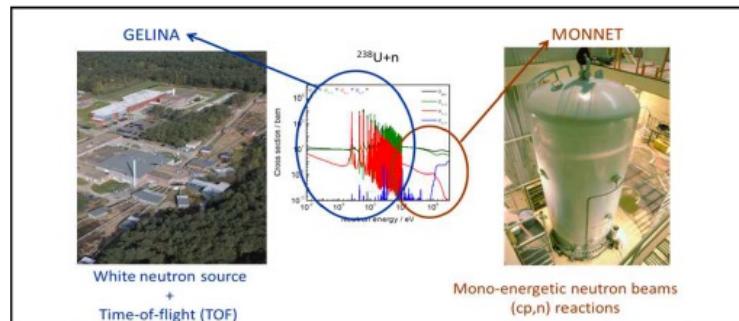


Conclusion



Conclusion

**NEA/OECD High Priority Request List
IAEA Coordinated Research Projects**



Wir schaffen Wissen – heute für morgen

