



D. Rochman and A.J. Koning

Thermal capture cross sections for evaluated libraries: selection, validation and recommendation

Technical meeting on thermal capture and capture gammas, IAEA,
May 23-27, 2022

- Introduction/context
- (n_{th}, g) sources
- Rule(s)
- Implementation and validation
- Remarks
- Conclusion

- Evaluated thermal capture cross sections are of prime importance for reactor applications:
 - Fission products, actinides, structural & light element activation
 - Incore, out of core
- Evaluated $(n_{th,g})$ are related to I_g
- Evaluated $(n_{th,g})$ are obtained from resonance parameters
- Evaluation of $(n_{th,g})$ is not based on resonance parameters
 - Using differential measurement (EXFOR)
 - Using integral information (reactor simulations): xs related to other quantities (e.g. FY)
- Some $(n_{th,g})$ are not measured but required for application (spent fuel, or astrophysics). What can be done ?

$(n_{th,g})$ sources (not ordered)

1. EXFOR
2. Holden 1991-1993 (compilation) + RIPL
3. k_0 neutron activation
4. Atlas of Neutron resonances BNL-325, 2006, 2018
5. EAF-2010
6. JUKO Research reports 2000-2017
7. Sukhoruchkin 2009 and 2015
8. Standard
9. Systematics
10. ENDF/B
11. JEF(F)
12. JENDL
13. and more



Atlas of Neutron Resonances
Resonance Parameters and Thermal Cross Sections
Z=1-100
S.F. Mughabghab

Mughabghab Atlas 2018
Mughabghab Atlas 2006



LANDOLT-BÖRNSTEIN
Neutron Resonance Parameters
Supplement to Volume 124
Springer

Sukhoruchkin 2015

Neutron Activation Analysis: <http://www.kayzero.com/k0naa>



ELSEVIER

Available online at www.sciencedirect.com



ScienceDirect

Nuclear Data Sheets 110 (2009) 3107-3214

Nuclear Data Sheets

www.elsevier.com/locate/nucds

RIPL – Reference Input Parameter Library for Calculation of Nuclear Reactions and Nuclear Data Evaluations

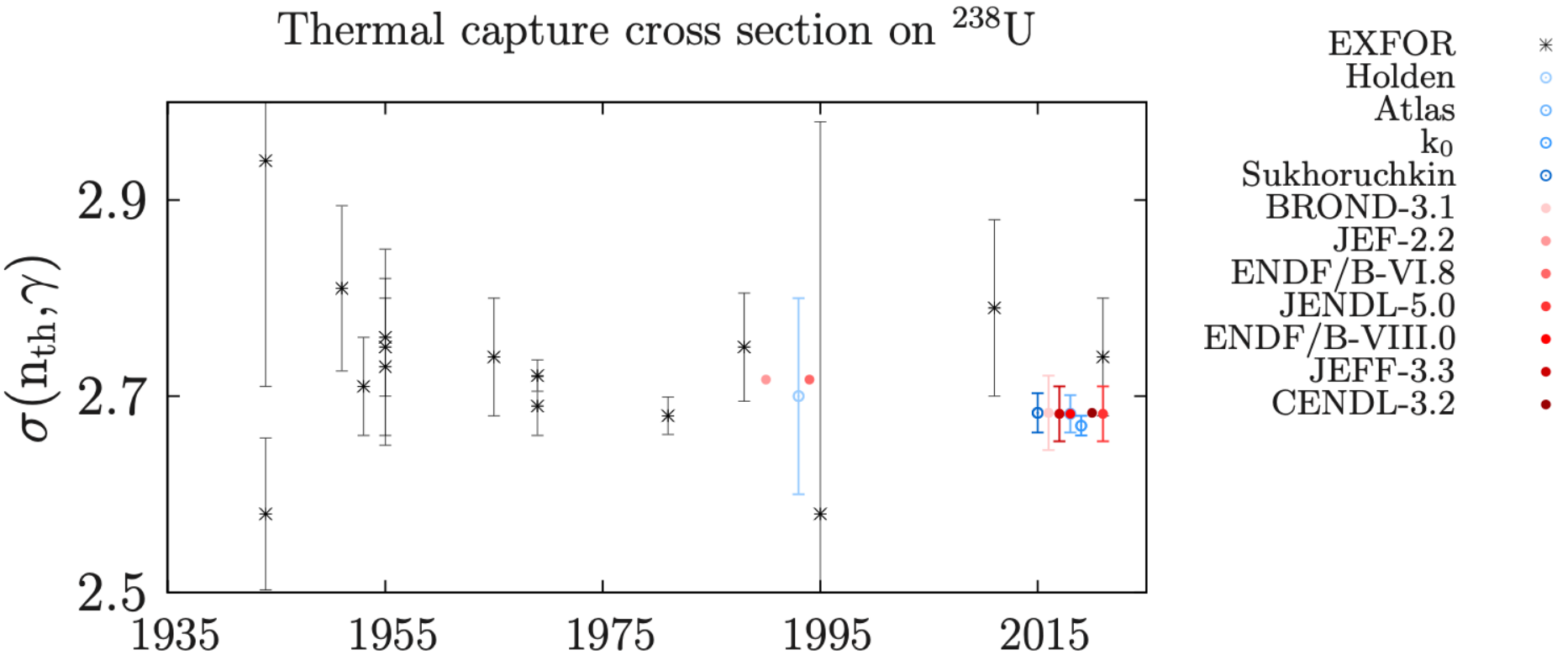
R. Capote,^{1*} M. Herman,^{1,2} P. Obložinský,^{1,2} P.G. Young,³ S. Goriely,⁴ T. Belgya,⁵ A.V. Ignatyuk,⁶ A.J. Koning,⁷ S. Hilaire,⁸ V.A. Plujko,⁹ M. Avrigeanu,¹⁰ O. Bersillon,⁸ M.B. Chadwick,³ T. Fukahori,¹¹ Zhigang Ge,¹² Yinlu Han,¹² S. Kailas,¹³ J. Kopecky,¹⁴ V.M. Maslov,¹⁵ G. Reffo,¹⁶ M. Sin,¹⁷ E.Sh. Soukhovitskii,¹⁸ P. Talou⁹

RIPL: Kopecky compilation
(Includes Holden tables)

EXFOR

(n_{th,g}) sources: Example 1

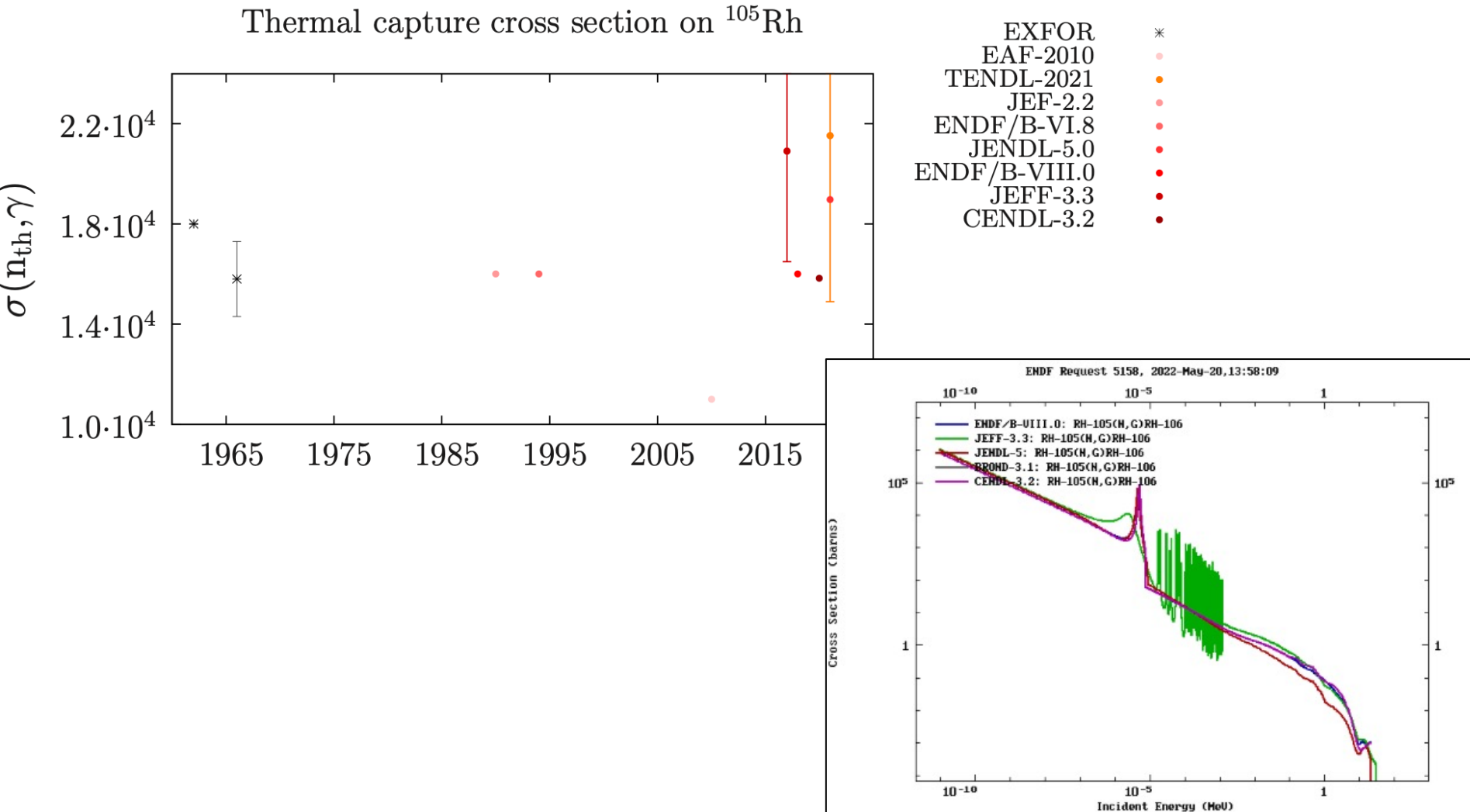
- Evaluated values (red) and compilations (blue) do not come from EXFOR alone



(n_{th,g}) sources: Example 2

- Evaluated values (red) do not come from EXFOR alone

Thermal capture cross section on ¹⁰⁵Rh



- Rules of selection applied for TENDL and some JEFF fission products
- Traceability
- Scripts based

New webpage under construction at IAEA

TALYS

TALYS-Related Software and Databases
TALYS and the TALYS-related packages are open source software and datasets (GPL License) for the simulation of nuclear reactions.

TALYS
Arjan Koning, Stéphane Hilalre, Stéphane Goriely
Nuclear reaction model code.

Download TALYS-1.95
Download previous versions
Read Tutorial

Created at NRG, CERN, IAEA

EXFORTABLES
Arjan Koning
Experimental nuclear reaction database based on EXFOR.

Download EXFORTABLES-1.0
Read Tutorial

RESONANCETABLES
Arjan Koning, Dimitri Rochman
Database for thermal cross sections, MACS and average resonance parameters.

Download RESONANCETABLES-1.0
Read Tutorial

Created at IAEA, PSI

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TASMAN, TEFAL, Libraries, ENDFTABLES, ISOTOPIA and Tools for TALYS (“T6”) to follow

RESONANCETABLES-1.0

Database for thermal cross sections, MACS and average resonance parameters

Arjan Koning and Dimitri Rochman

Draft version

Available on new website soon, together with other TALYS-related software

<https://www-nds.iaea.org/talys/>

Example: Table with all possible options for thermal capture cross sections



62 151 0	1.51400E+04	3.00000E+02	Mugh18	Sm151
ripl	1.52000E+04	3.00000E+02	JUKO	
mugh06	1.51700E+04	3.00000E+02	Mugh06	
mugh18	1.51400E+04	3.00000E+02	Mugh18	
sukhoruchkin	1.51700E+04	2.99000E+02	Sukhoruchkin	
exfor	1.20000E+04	0.00000E+00	Melaika_1955_12086003	
exfor	1.51700E+04	3.00000E+02	Mughabghab_2006_V10022001	
exfor	1.50200E+04	5.25000E+02	Marrone_2006_22893007	
cendl3.1	1.51239E+04	CE=	9.98937E-01	
endfb8.0	1.51379E+04	CE=	9.99861E-01	
jeff3.3	1.51294E+04	CE=	9.99300E-01	
jendl4.0	1.51598E+04	CE=	1.00131E+00	
tendl.2019	1.51334E+04	CE=	9.99564E-01	
62 152 0	2.06000E+02	3.00000E+00	Mugh18	Sm152
ripl	2.06000E+02	1.50000E+01	JUKO	
mugh06	2.06000E+02	6.00000E+00	Mugh06	
mugh18	2.06000E+02	3.00000E+00	Mugh18	
sukhoruchkin	2.06000E+02	6.00000E+00	Sukhoruchkin	
exfor	1.38000E+02	2.76000E+01	Seren_1947_11447096	
exfor	2.00000E+02	6.00000E+00	Pattenden_1958_21325012	
exfor	2.24000E+02	7.00000E+00	Tattersall_1960_20638048	
exfor	2.15000E+02	1.00000E+01	Fehr_1960_12023006	
exfor	2.09000E+02	9.00000E+00	Cabell_1962_20627002	
exfor	2.09100E+02	2.07000E+01	Bernabei_1962_12099002	
exfor	2.06000E+02	6.00000E+00	Mughabghab_2006_V10022041	
exfor	2.04800E+02	6.32832E+00	Karadag_2007_22964002	
exfor	2.38930E+02	1.91100E+01	Nyarko_2010_31698006	
exfor	2.07300E+02	9.40000E+00	Agbemava_2011_31717003	
exfor	2.07000E+02	1.00000E+00	FarinaArbocco_2013_23266134	
exfor	2.12000E+02	8.00000E+00	Nguyen_2017_30843002	
cendl3.1	2.06640E+02	CE=	1.00311E+00	
endfb8.0	2.05946E+02	CE=	9.99738E-01	
jeff3.3	2.05974E+02	CE=	9.99874E-01	
jendl4.0	2.05842E+02	CE=	9.99233E-01	
tendl.2019	2.05971E+02	CE=	9.99859E-01	

Library values obtained
from point wise files
(PREPRO's RECENT module)

Order of adoption for thermal cross sections, resonance integrals and MACS

1. Kayzero database
2. Mughabghab 2018 Atlas
3. Sukhoruchkin 2015 Atlas
4. Mughabghab 2006 Atlas
5. RIPL or Kopecky database
6. EXFOR (the most recent value)

For MACS, the order of adoption is

1. Mughabghab 2018 Atlas
2. Sukhoruchkin 2015 Atlas
3. KADONIS database
4. EXFOR (the most recent value)


Final database: if 1 doesn't exist, we take 2,
if it doesn't exist, we take 3, etc.

Implementation and validation

- Different tools were developed, followed by (extensive) testing

TALYS-based evaluated nuclear data library

Home Reference & us Citations Feedback TALYS



New webpage under construction at IAEA

TALYS

TALYS-Related Software and Databases

TALYS and the TALYS-related packages are open source software and datasets (OSF License) for the simulation of nuclear reactions.

TALYS
Arjan Koning, Stéphane Hilborn, Stéphane Gorjode

EXFORTABLES
Experimental nuclear reactions

Download EXFORTABLES-1.0
Read Tutorial

We believe that our great goal can be achieved with systematism and reproducibility. We are so outside the box, that the box is a point!

How to...
Sub-li...

TASMAN, TEFAL, Libraries, ENDF/TABLES, ISOTOPIA

NEA
NUCLEAR ENERGY AGENCY

Related sites

JEFF-3.3

The Joint Evaluated Fission and Fusion File (JEFF) is an evaluated library produced via an international collaboration of NEA Data Bank participating countries.

JEFF-3.3 NDEC

Downloads

- Neutrons
- Decay Data
- Fission Yields
- Alpha
- Deuteron
- DPA
- Gamma
- He-3

Available online at www.sciencedirect.com

ScienceDirect

Nuclear Data Sheets

Nuclear Data Sheets 163 (2020) 163–190

www.elsevier.com/locate/nds

ELSEVIER

Check for updates

A Statistical Analysis of Evaluated Neutron Resonances with TARES for JEFF-3.3, JENDL-4.0, ENDF/B-VIII.0 and TENDL-2019

Implementation and validation

- Different tools were developed, followed by (extensive) testing

TABLE I. (Color online) Average $\overline{C/E}$ and standard deviation (*Std*) for each library and different quantities. In the case of the fission cross section, ^{238}U is excluded from the JEFF-3.3 library as thermal fission cross section is zero. Both contributions from MF2 and MF3 are considered. In this table as in the following ones, the term *E* refers to Ref. [30].

Thermal capture cross section				
Library	Isotopes	$\overline{C/E}$	<i>Std</i>	Outlier
TARES&TENDL-2019	374	1.000	0.050	3
JEFF-3.3	308	1.045	0.454	4
JENDL-4.0	244	1.021	0.194	12
ENDF/B-VIII.0	282	1.216	3.871	1
Thermal elastic cross section				
TARES&TENDL-2019	226	0.991	0.075	15
JEFF-3.3	208	1.056	1.398	2
JENDL-4.0	193	1.087	1.398	1
ENDF/B-VIII.0	199	1.138	1.421	1
Thermal fission cross section				
TARES&TENDL-2019	45	1.003	0.521	2
JEFF-3.3	34	2.310	7.923	0
JENDL-4.0	33	0.971	0.371	4
ENDF/B-VIII.0	33	2.057	6.142	0

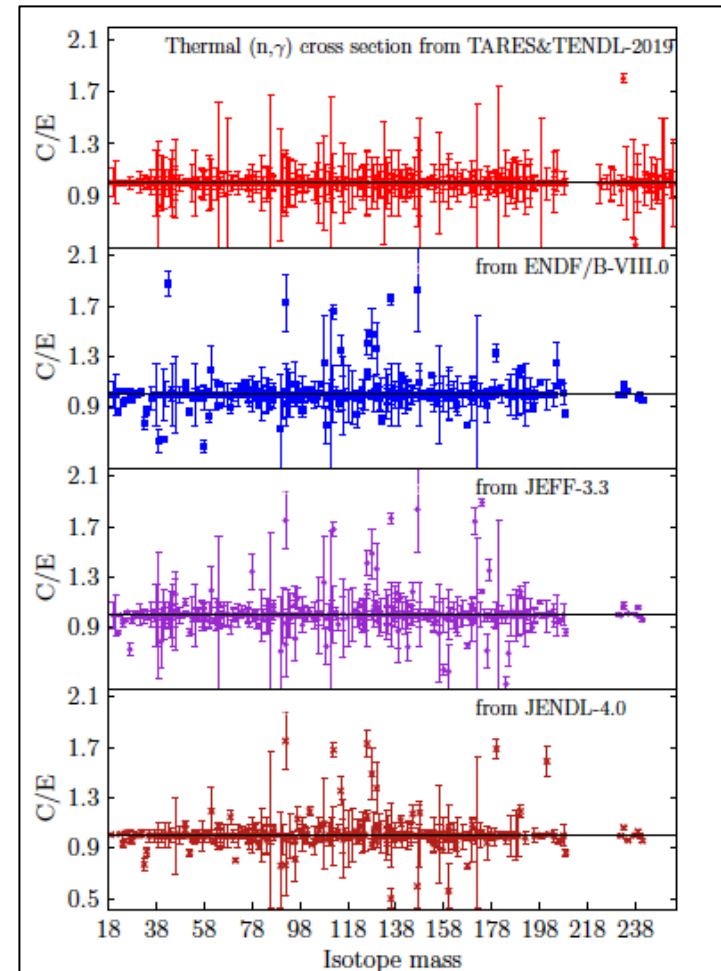
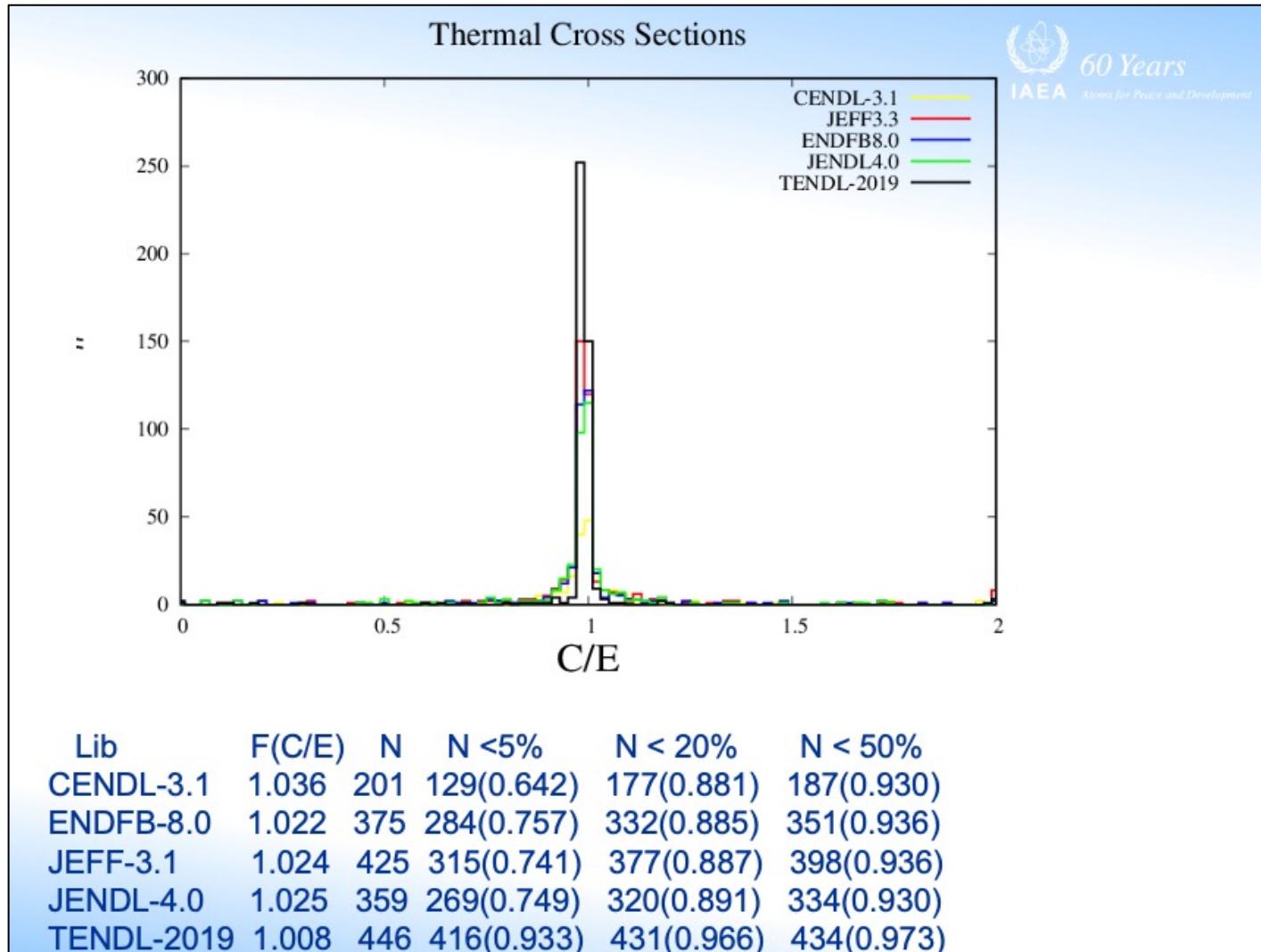


FIG. 8. (Color online) C/E values for the thermal capture cross sections for the various considered libraries. The “E” is from Ref. [30], see Table I for details. The uncertainties are experimental. For other reactions, see Appendix B.

Implementation and validation

- Different tools were developed, followed by (extensive) testing



- K_0 generally good, with some exceptions

Table 3.1: List of isotopes presenting the largest differences between the recommended values and the k_0 neutron activation. For ^{70}Zn , ^{76}Se and ^{190}Os the uncertainty on the resonance integral for the k_0 database is assigned to 10 %, as no value is originally provided.

Isotope	cross section		resonance integral	
	This work	Refs. [7, 29]	This work	Refs. [7, 29]
^{110}Pd	0.30 ± 0.03 b.	0.0137 ± 0.0012 b.	2.18 ± 0.04 b.	0.163 ± 0.023 b.
^{68}Zn	1.07 ± 0.19 b.	0.0701 ± 0.0020 b.	3.06 ± 0.55 b.	0.224 ± 0.007 b.
^{70}Zn	92.3 ± 18.8 mb.	21 ± 1 mb.	0.94 ± 0.16 b.	0.166 ± 0.017 b.
^{76}Se	85.0 ± 8.7 b.	18.9 ± 0.6 b.	39.0 ± 3.4 b.	14.6 ± 1.5 b.
^{190}Os	10.95 ± 1.12 b.	3.48 ± 0.25 b.	39.5 ± 1.9 b.	7.06 ± 0.8 b.

[7] Frans de Corte. “The k_0 -standardization method: a move to the optimization of neutron activation analysis”. PhD thesis. University of Gent, Belgium, 1987 (cited on page 17).

[29] A. Trkov and V. Radulovic. “Nuclear reactions and physical models for neutron activation analysis”. In: *Jour. of Radioanal. Nucl. Chem.* 304 (2015), pages 763–778 (cited on pages 17, 18).

Remarks on (n_{th},g) not measured

- Short-lived fission products, or isotopes for astrophysics.
- What to do ?
 - Rely on systematics (HFR) for resonance parameters,

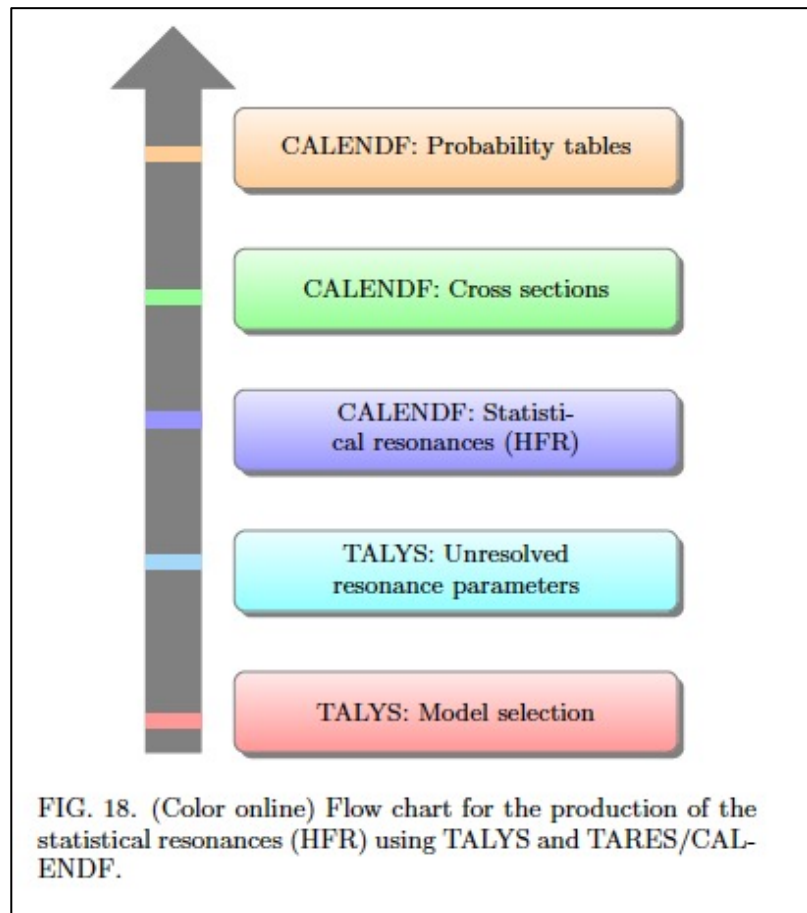
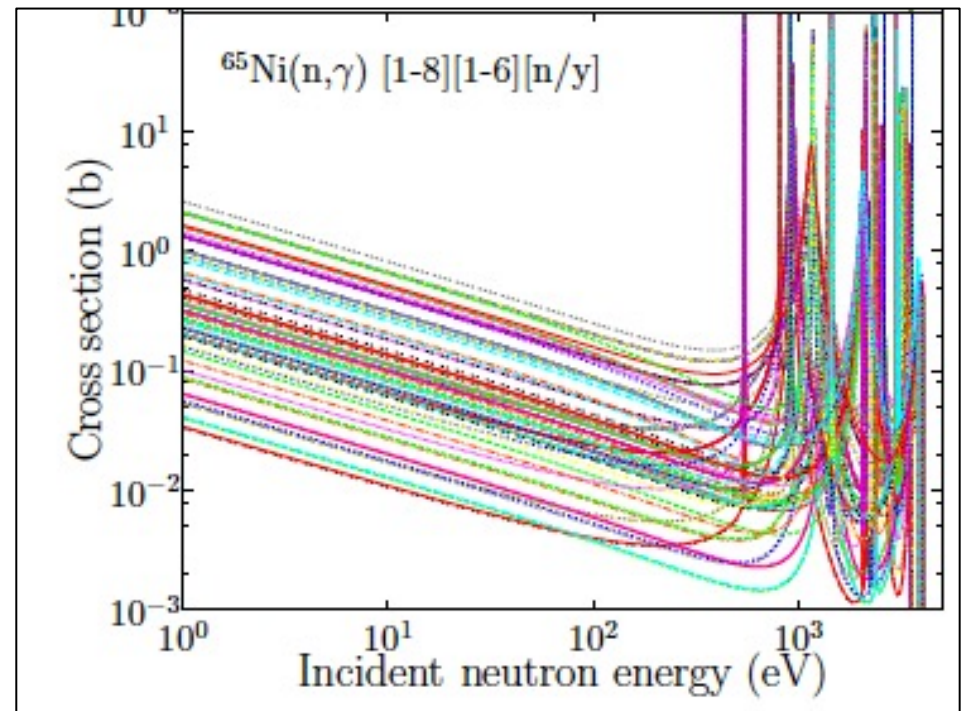


FIG. 18. (Color online) Flow chart for the production of the statistical resonances (HFR) using TALYS and TARES/CALENDF.




Remarks on (n_{th},g) not measured

- Short-lived fission products, or isotopes for astrophysics.
- What to do ?
 - or simply on model variations for average values (thermal, MACS)
 - (work performed with A. Koning, S. Goriely and D. Rochman)

TALYS-based evaluated nuclear data library

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“ We believe that our great goal can be achieved with systematism and reproducibility. We are so outside the box, that the box is a point”

How to reference

Sub-library files

1. Neutron
2. Proton
3. Deuteron (updated)
4. Triton
5. He3
6. Alpha
7. Gamma
8. Fission yields
9. Thermal scattering
10. For astrophysics

TENDL-2021: (release date: December 30, 2021)

Last update: February 23, 2022

TENDL is a nuclear data library which provides the output of the TALYS nuclear model code system for direct use in both basic physics and applications. The 11th version is **TENDL-2021**, which is based on both default and adjusted TALYS calculations and data from other sources (previous releases can be found here: [2008](#), [2009](#), [2010](#), [2011](#), [2012](#), [2013](#), [2014](#), [2015](#), [2017](#) and [2019](#)).

Up to 2014, TENDL was produced at NRG Petten. Since 2015, TENDL is mainly developed at PSI and the IAEA (Nuclear Data Section). Still, many people contribute to TENDL with the testing and processing of the files.

TENDL contains evaluations for seven types of incident particles, for all isotopes living longer than 1 second: Z=1 ¹H to Z=115 ²⁹¹Mc (about 2800 isotopes), up to 200 MeV, with

TENDL-stars 2021

(n,g) cross sections, reaction rates and MACS for astrophysics

Description

Below are various links to 8892 isotopes for astrophysics applications (capture cross sections, capture rates and MACS).

Different reaction "model sets" were used: "a model set" represents a combination of 7 TALYS models:

1. Gamma strength function (values 8 or 9): either Gogny D1M HFB+QRPA, or SMLO
2. Level density (values 1, 2 or 5): Constant temperature + Fermi gas model, or Back-shifted Fermi gas model, or Microscopic level densities (Skyrme force) from Hilaire's combinatorial tables
3. JLM microscopic optical model potential or KD optical model (values y or n)
4. Gamma strength function for M1 (values 3 or 8): Hartree-Fock BCS tables or Gogny D1M HFB+QRPA
5. Collective enhancement (values y or n): yes or no
6. Width fluctuation (values 0, 1 or 2): Moldauer model, or Hofmann-Richert-Tepel-Weidenmueller model
7. Mass model (values 0, 1, 2 or 3): Duflo-Zuker formula, Moeller table, Goriely HFB-Skyrme table, or HFB-Gogny D1M table (except for known masses, where the experimental value is used)

Each of the set models is named with 7 values, such as "91n3n12" (default TALYS model), or 81n8n12. These values correspond to the ones in the TALYS manual. Quantities are defined such as:

- 1) XS (n,g) cross sections: capture cross sections from 0.01 eV to 20 MeV (from the TALYS file xs000000.tot)
- 2) $\langle \sigma_{v} \rangle$: reaction rates for all temperatures taking into account the thermal population of the target (from the TALYS file astrorate.g)
- 3) G(T): normalized partition function (from the TALYS file astrorate.tot)
- 4) MACS: Maxwellian Averaged (n,g) Cross Sections

Available files

	XS (n,g) cross sections with	$\langle \sigma_{v} \rangle$ rates with	G(T) with	MACS 30 keV
1. 270 stable isotopes:	12 or 288 model sets	12 or 288 model sets	12 or 288 model sets	12 or 288 model sets
2. 3256 isotopes with known mass	12 or 288 model sets	12 or 288 model sets	12 model sets	12 model sets
3. 5366 other isotopes	12 or 288 model sets	12 or 288 model sets	12 model sets	12 model sets

Last update: Wed May 19, 2022

Remarks on (n_{th},g) not measured

```

1 # File for Tc079 reaction (n,g) PSI, Switzerland, 03-30-2022 11:27:59 PM
2 # Reaction calculated with TALYS 1.96
3 #
4 #
5 #
6 # name of model sets with 7 digits, such as 85y8n01
7 #
8 # |||||massmodel (0,1,2, or 3) (0: Duflo-Zuker formula, 1: Moeller table, 2: Goriely HFB-Skyrme table,
9 # |||||widthmode (0,1 or 2) (0: no width fluctuation, 1: Moldauer model, 2: Hofmann-Richert-Tepel-Weidenmueller mo
10 # |||colenhance (y, n) (y: (yes) explicit collective enhancement of the level density)
11 # |||strengthM1 (3, 8) (3: Hartree-Fock BCS tables, 8:Gogny D1M HFB+QRPA
12 # ||jlmomp (y, n) (y: (yes) JLM microscopic optical model potential)
13 # |ldmodel (1, 2, 5) (1: Constant temperature + Fermi gas model, 2: Back-shifted Fermi gas model,
14 # |strength (8, 9) (8: Gogny D1M HFB+QRPA, 9: SML0
15 #
16 # Default TALYS model: 91n3n12
17 #
18 # Number of model sets: 288
19 #
20 # energy (MeV) Mean Std Median MAD min max 91n3n12 ( 1) 85y8n00 ( 2) 85y8n01 ( 3) 85y8n02 ( 4) 85y8n03 ( 5)
21 1.00000E-11 6.008e+06 5.170e+06 4.512e+06 3.538e+06 8.238e+05 1.640e+07 8.739e+06 9.226e+06 8.739e+06 1.640e+07
22 2.53000E-08 1.194e+05 1.028e+05 8.972e+04 7.036e+04 1.638e+04 3.261e+05 1.737e+05 2.340e+05 1.834e+05 1.737e+05
23 2.00000E-07 4.249e+04 3.657e+04 3.190e+04 2.501e+04 5.825e+03 1.160e+05 6.180e+04 8.323e+04 6.524e+04 6.180e+04
24 1.00000E-06 3.440e-05 5.947e-05 5.766e-06 3.917e-06 5.283e-07 1.946e-04 3.102e-06 3.223e-06 8.354e-06 5.108e-06
25 1.00000E-05 1.108e-05 1.871e-05 2.186e-06 1.520e-06 2.085e-07 6.152e-05 1.102e-06 1.373e-06 3.322e-06 2.033e-06
26 1.00000E-04 4.138e-06 5.661e-06 1.574e-06 1.069e-06 1.973e-07 1.942e-05 7.319e-07 1.554e-06 3.204e-06 1.965e-06
27 2.00000E-04 3.427e-06 3.867e-06 1.889e-06 1.249e-06 2.427e-07 1.372e-05 8.189e-07 1.979e-06 3.958e-06 2.428e-06
28 5.00000E-04 3.119e-06 2.469e-06 2.248e-06 1.328e-06 3.493e-07 8.662e-06 1.090e-06 2.921e-06 5.713e-06 3.507e-06
29 1.00000E-03 3.327e-06 2.331e-06 2.881e-06 1.670e-06 4.763e-07 9.227e-06 1.444e-06 4.033e-06 7.822e-06 4.802e-06
30 2.00000E-03 3.939e-06 3.064e-06 3.645e-06 1.932e-06 6.532e-07 1.265e-05 1.973e-06 5.631e-06 1.088e-05 6.677e-06
31 5.00000E-03 5.492e-06 5.034e-06 3.070e-06 2.046e-06 4.329e-07 1.957e-05 3.047e-06 8.818e-06 1.699e-05 1.043e-05
32 1.00000E-02 7.396e-06 7.274e-06 4.288e-06 3.258e-06 3.033e-07 2.737e-05 4.558e-06 1.239e-05 2.385e-05 1.465e-05
33 2.00000E-02 1.012e-05 1.033e-05 5.971e-06 5.104e-06 2.114e-07 3.813e-05 5.932e-06 1.733e-05 3.336e-05 2.051e-05
34 3.00000E-02 1.215e-05 1.257e-05 7.205e-06 6.290e-06 1.706e-07 4.604e-05 7.161e-06 2.098e-05 4.039e-05 2.486e-05
35 5.00000E-02 1.517e-05 1.585e-05 9.011e-06 8.314e-06 1.296e-07 5.764e-05 8.966e-06 2.637e-05 5.081e-05 3.132e-05
36 1.00000E-01 1.964e-05 2.067e-05 1.160e-05 1.113e-05 8.842e-08 7.436e-05 1.158e-05 3.437e-05 6.634e-05 4.107e-05
37 2.00000E-01 2.251e-05 2.375e-05 1.317e-05 1.289e-05 6.040e-08 8.412e-05 1.313e-05 3.968e-05 7.684e-05 4.801e-05
38 5.00000E-01 1.712e-05 1.805e-05 9.936e-06 9.789e-06 3.800e-08 6.499e-05 9.578e-06 3.054e-05 5.982e-05 3.862e-05
39 8.00000E-01 1.096e-05 1.152e-05 6.237e-06 6.124e-06 3.030e-08 4.218e-05 5.873e-06 1.960e-05 3.892e-05 2.611e-05
40 1.00000E+00 5.994e-02 1.364e-02 5.690e-02 9.890e-03 3.770e-02 9.118e-02 5.021e-02 9.111e-02 6.864e-02 5.678e-02
41 1.20000E+00 1.150e-02 7.182e-03 1.427e-02 3.070e-03 8.911e-08 2.149e-02 2.148e-02 5.344e-02 5.344e-02 1.548e-02
42 1.60000E+00 3.778e-02 1.439e-02 4.065e-02 1.021e-02 1.245e-02 6.052e-02 4.459e-02 6.049e-02 2.111e-02 5.532e-02
43 1.80000E+00 2.457e-02 2.761e-02 1.413e-02 9.925e-03 2.194e-08 8.462e-02 2.937e-08 1.984e-02 8.462e-02 9.715e-08
44 2.00000E+00 5.281e-02 1.102e-02 5.307e-02 9.235e-03 3.444e-02 7.549e-02 4.456e-02 7.548e-02 6.301e-02 5.623e-02
45 2.50000E+00 5.049e-02 9.442e-03 5.141e-02 8.050e-03 3.438e-02 6.872e-02 4.375e-02 6.872e-02 5.951e-02 5.848e-02
46 3.00000E+00 4.650e-02 7.474e-03 4.529e-02 6.570e-03 3.349e-02 6.010e-02 4.211e-02 6.010e-02 5.326e-02 4.558e-02
47 3.50000E+00 4.155e-02 5.647e-03 4.038e-02 4.680e-03 3.185e-02 5.114e-02 3.973e-02 5.114e-02 4.586e-02 4.826e-02
48 4.00000E+00 3.659e-02 4.282e-03 3.611e-02 3.275e-03 2.932e-02 4.516e-02 3.707e-02 4.311e-02 3.878e-02 4.148e-02
49 5.00000E+00 3.129e-02 5.311e-03 3.113e-02 3.985e-03 2.267e-02 4.206e-02 4.202e-02 3.117e-02 3.133e-02 4.007e-02
50 6.00000E+00 2.477e-02 4.381e-03 2.363e-02 3.145e-03 1.747e-02 3.518e-02 3.518e-02 2.365e-02 2.327e-02 2.985e-02
51 7.00000E+00 2.006e-02 3.688e-03 1.945e-02 2.550e-03 1.403e-02 2.905e-02 2.899e-02 1.878e-02 1.795e-02 2.295e-02
52 8.00000E+00 1.649e-02 3.154e-03 1.634e-02 2.205e-03 1.149e-02 2.413e-02 2.404e-02 1.523e-02 1.414e-02 1.816e-02
53 1.00000E+01 6.177e-03 6.431e-03 4.397e-03 4.397e-03 2.713e-10 1.759e-02 1.722e-02 5.266e-08 9.482e-03 1.225e-02
54 1.20000E+01 5.312e-03 5.483e-03 4.161e-03 4.161e-03 2.465e-10 1.384e-02 1.314e-02 3.649e-08 9.213e-03 8.959e-03
55 1.40000E+01 2.127e-03 3.779e-03 0.000e+00 1.375e-08 1.251e-10 1.103e-02 1.468e-08 2.660e-08 7.121e-03 2.864e-08
56 1.60000E+01 6.749e-03 1.458e-03 6.202e-03 1.329e-03 4.472e-03 9.422e-03 9.039e-03 5.762e-03 6.016e-03 5.888e-03
57 1.80000E+01 6.063e-03 1.314e-03 5.557e-03 1.188e-03 4.001e-03 8.454e-03 8.131e-03 5.171e-03 5.399e-03 5.296e-03
58 2.00000E+01 5.418e-03 1.178e-03 4.960e-03 1.057e-03 3.576e-03 7.534e-03 7.344e-03 4.596e-03 4.811e-03 4.783e-03

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Conclusion

- There is a need for international recommendations on $(n_{th,g})$, RIPL-like,
- Need of clear rules, traceability,
- Need of accessibility in a computer format (and eventually reports/papers)
- $(n_{th,el})$, I_g , RP are also important,
- $(n_{th,g})$ for 8892 isotopes can be needed.

Wir schaffen Wissen – heute für morgen

