

Program EVALPLOT  
(Version 2021-1)

by

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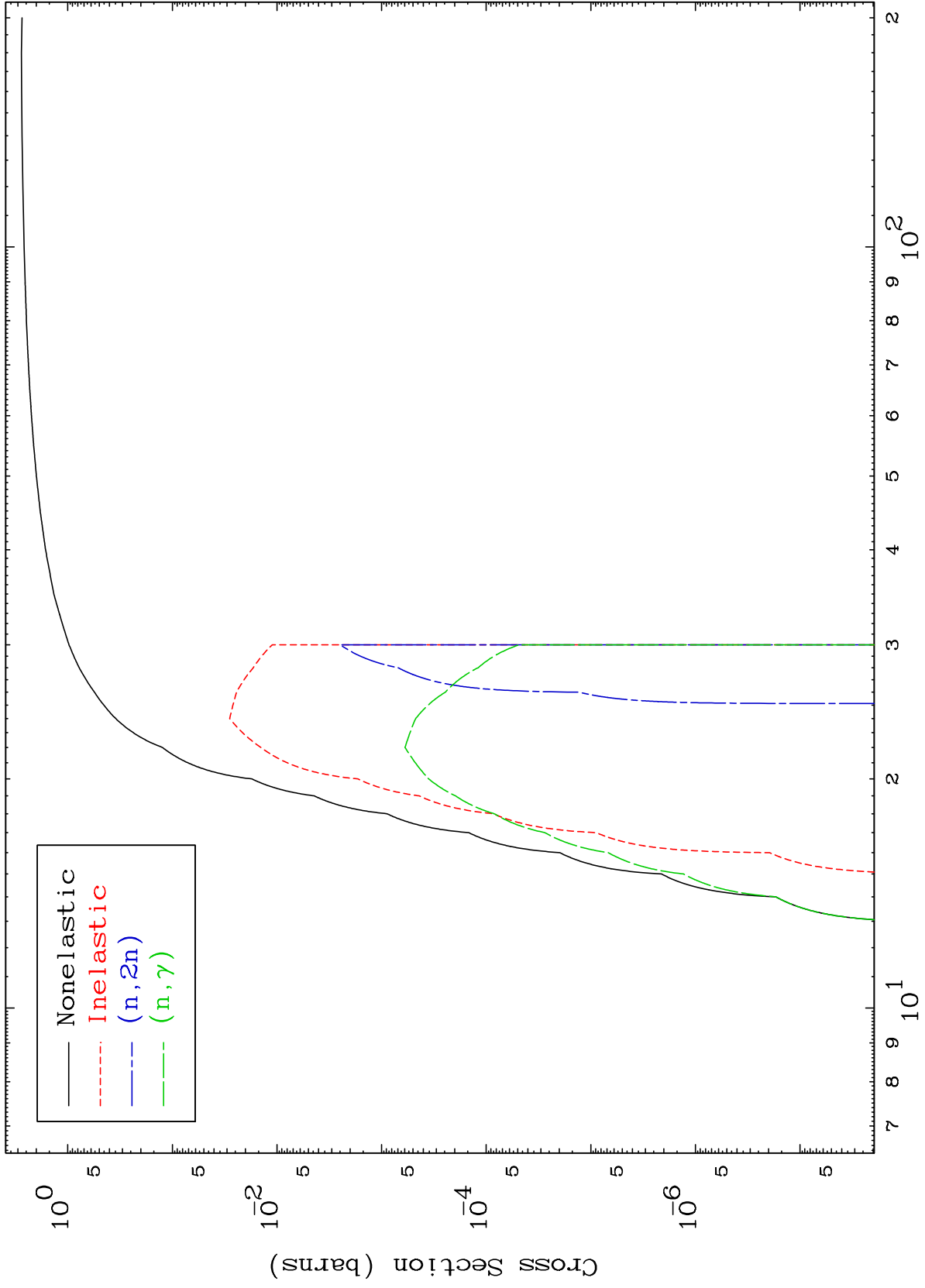
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 8523

0 Kelvin Cross Sections

85-At-202m



1

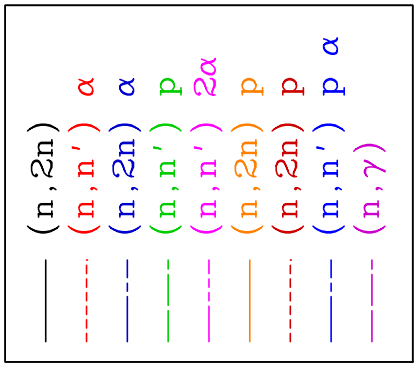
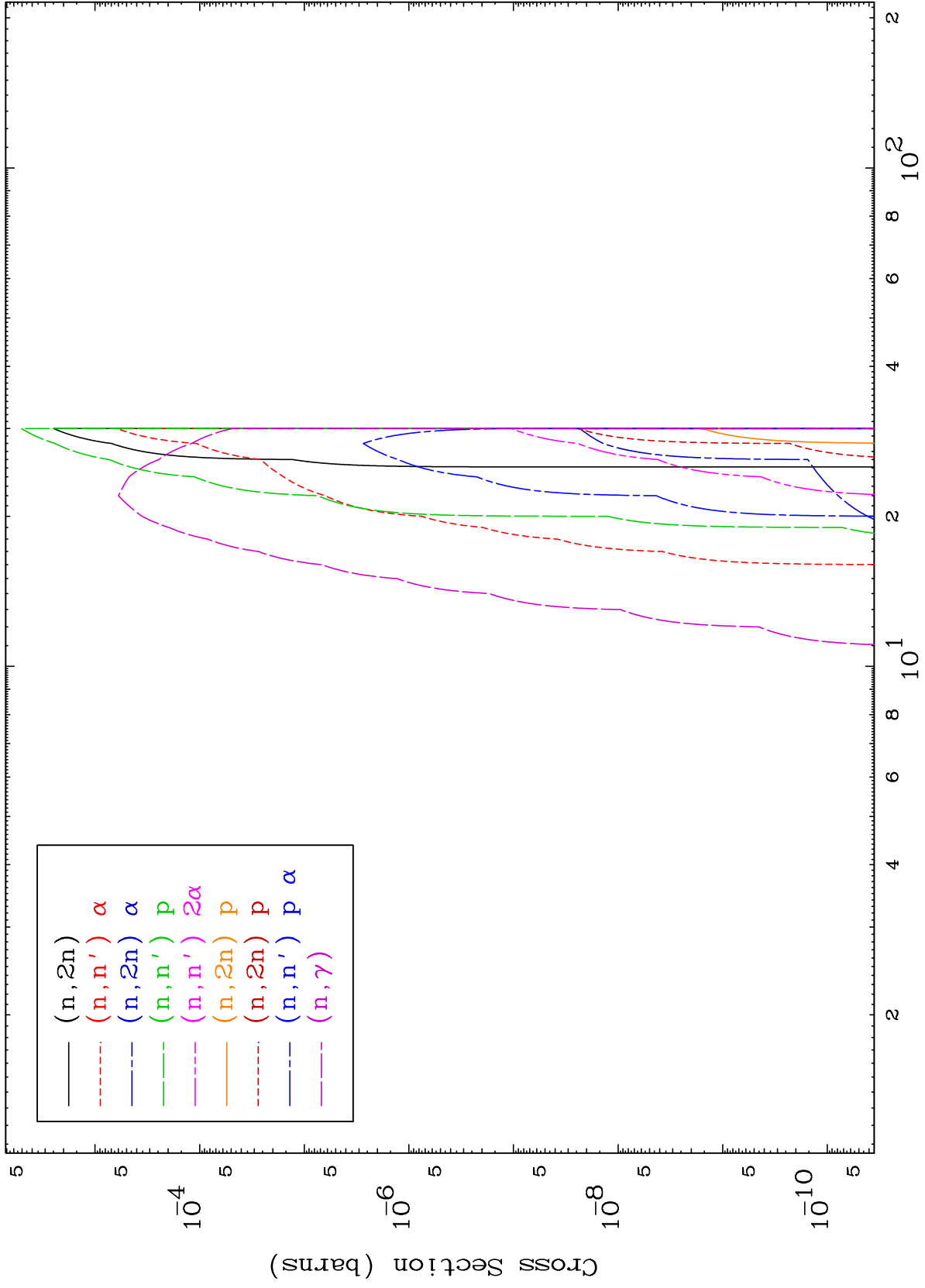
Incident Energy (MeV)

85-At-202m

MAT 8523

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

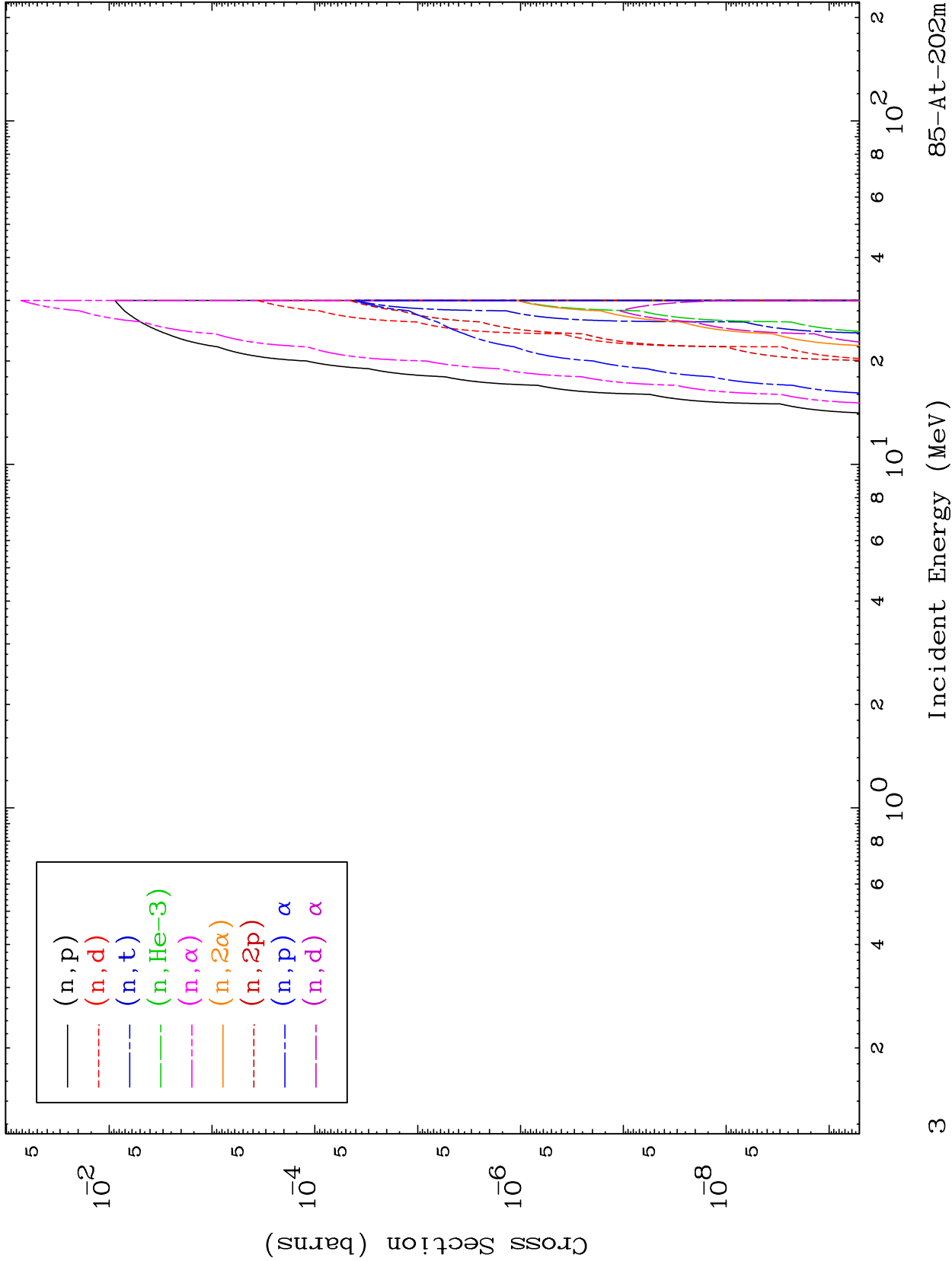
85-At-202m



MAT 85223

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

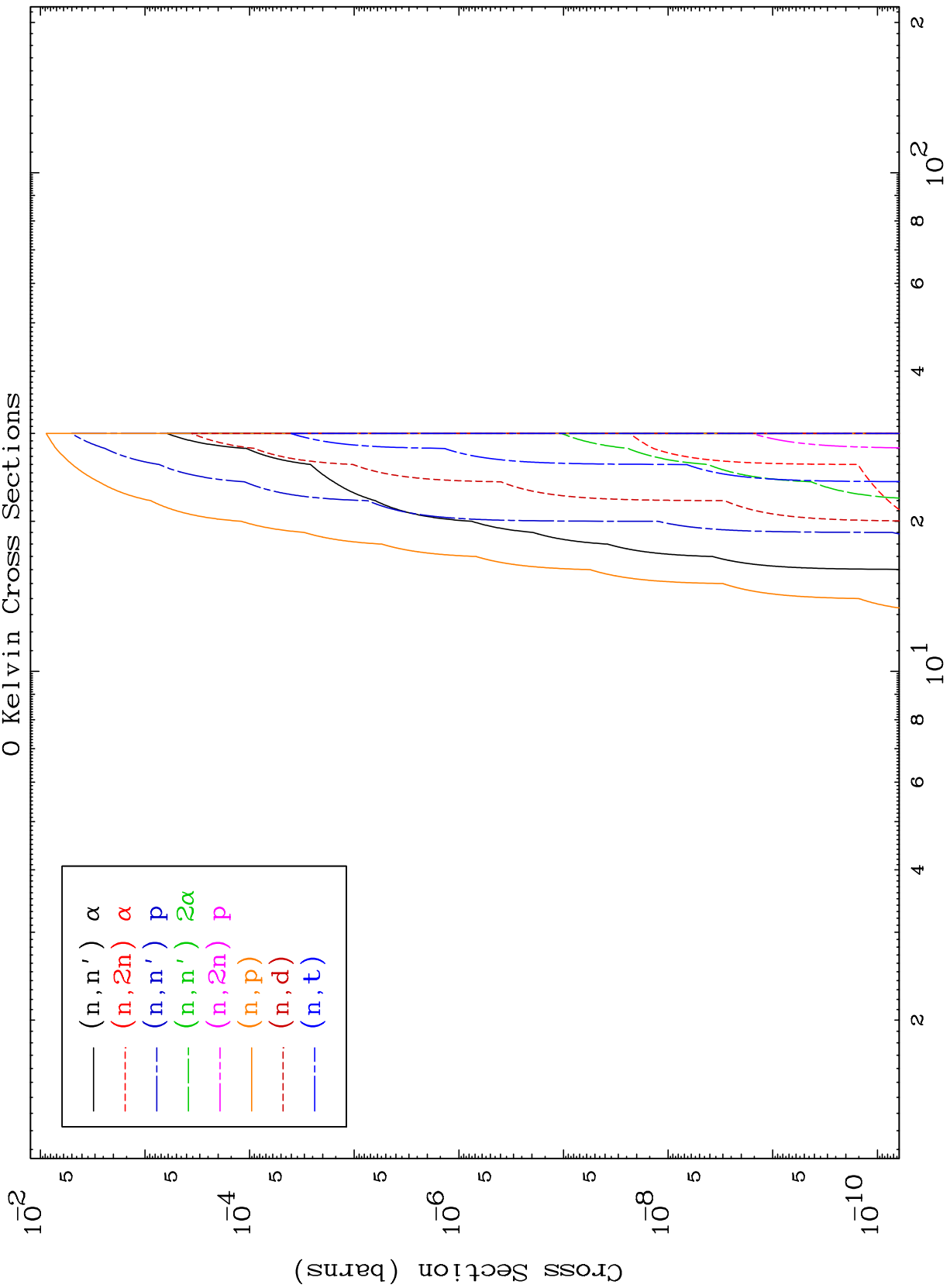
85-At-202m



MAT 8523

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

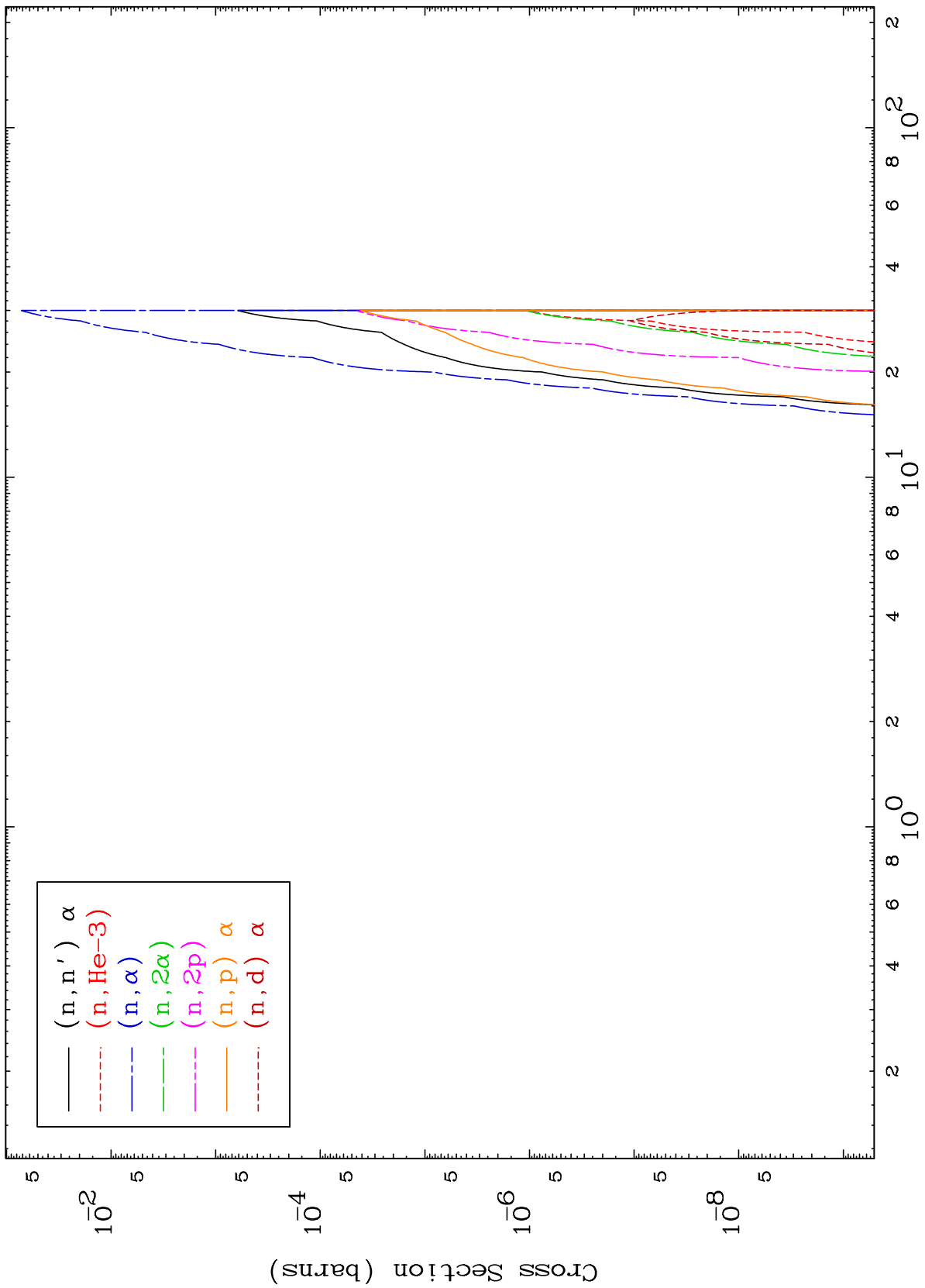
85-At-202m



MAT 8523

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

85-At-202m



5

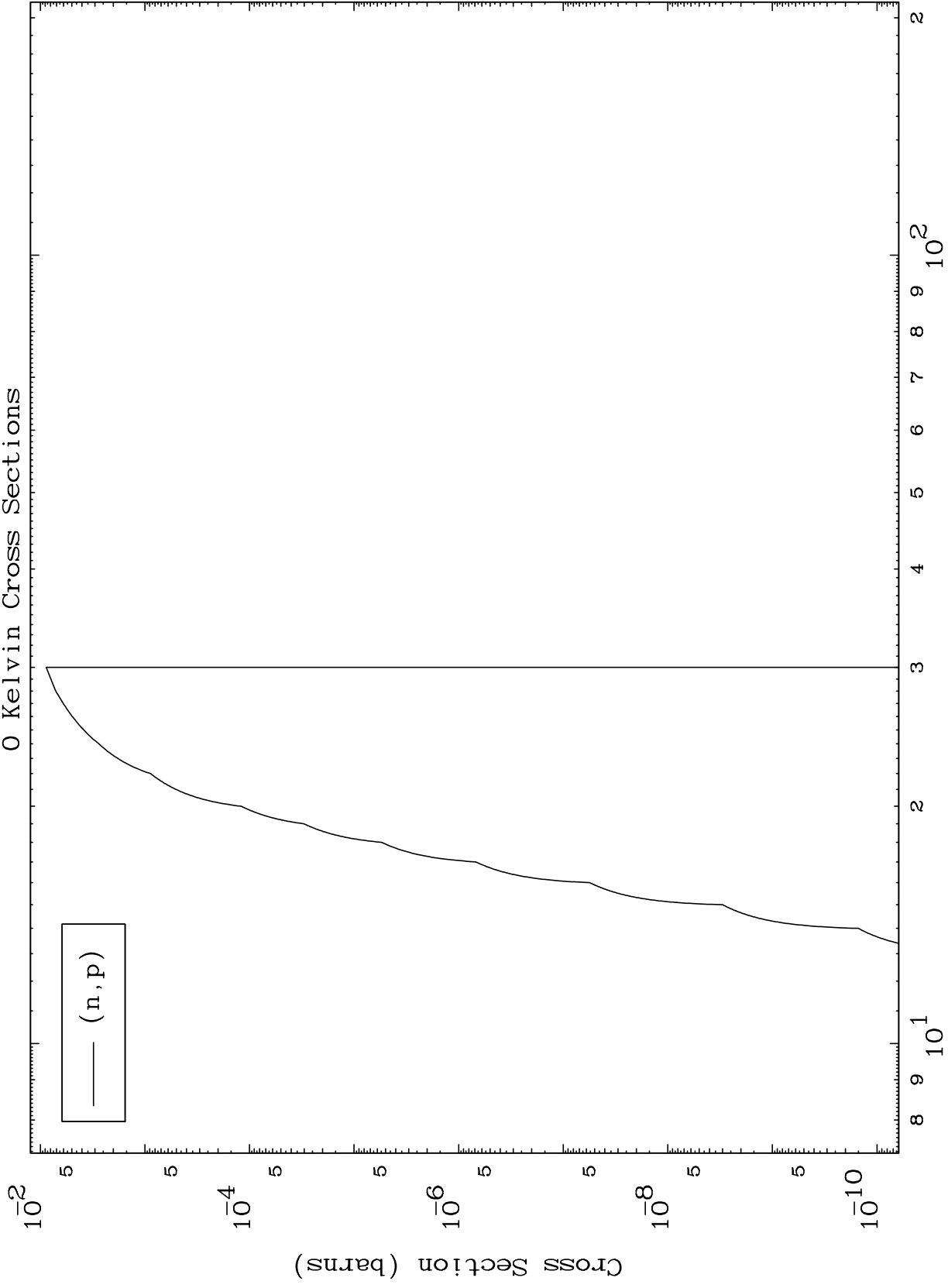
Incident Energy (MeV)

85-At-202m

MAT 8523

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

85-At-202m



Incident Energy (MeV)

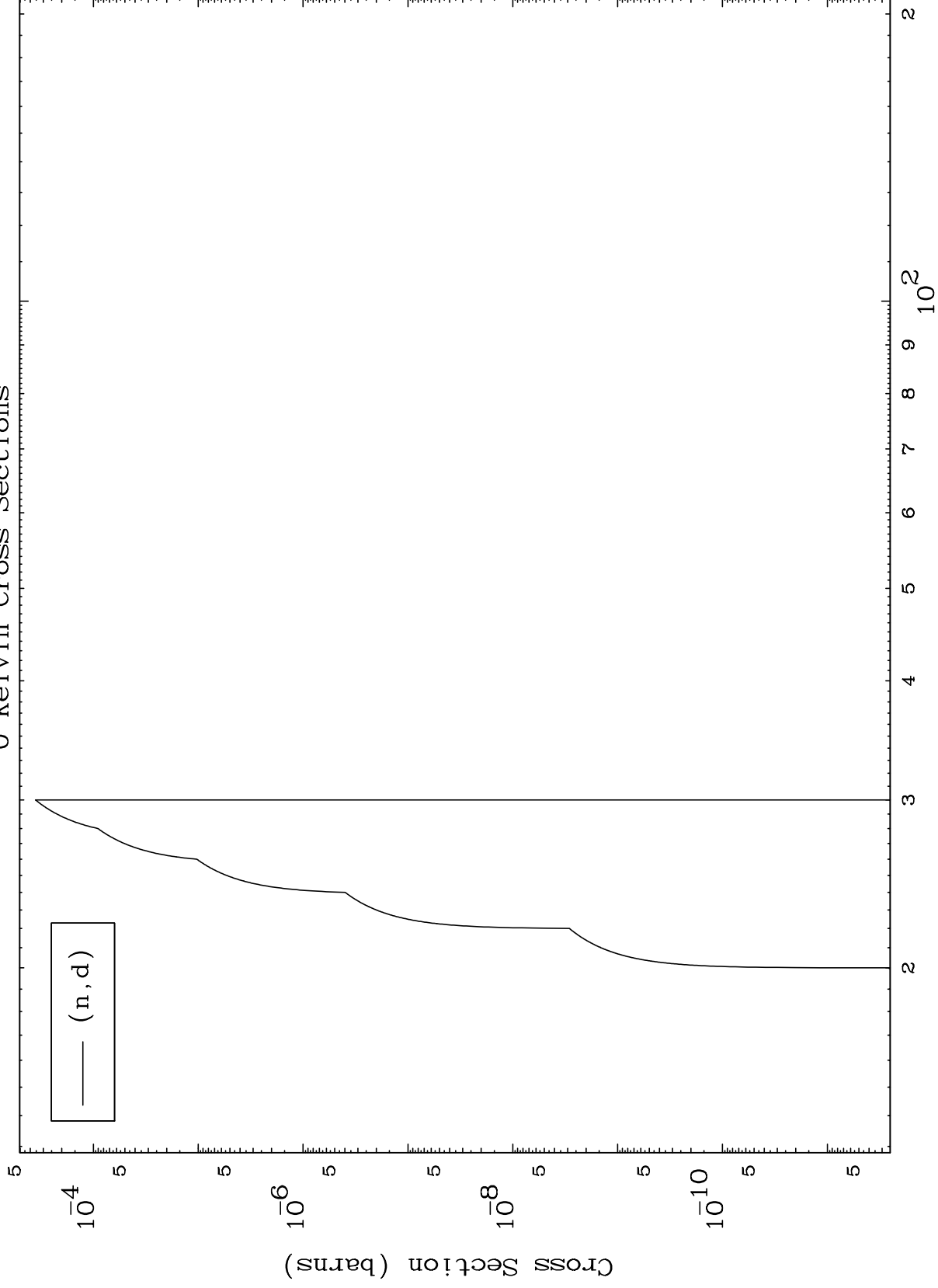
85-At-202m

6

MAT 8523

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

85-At-202m



7

Incident Energy (MeV)

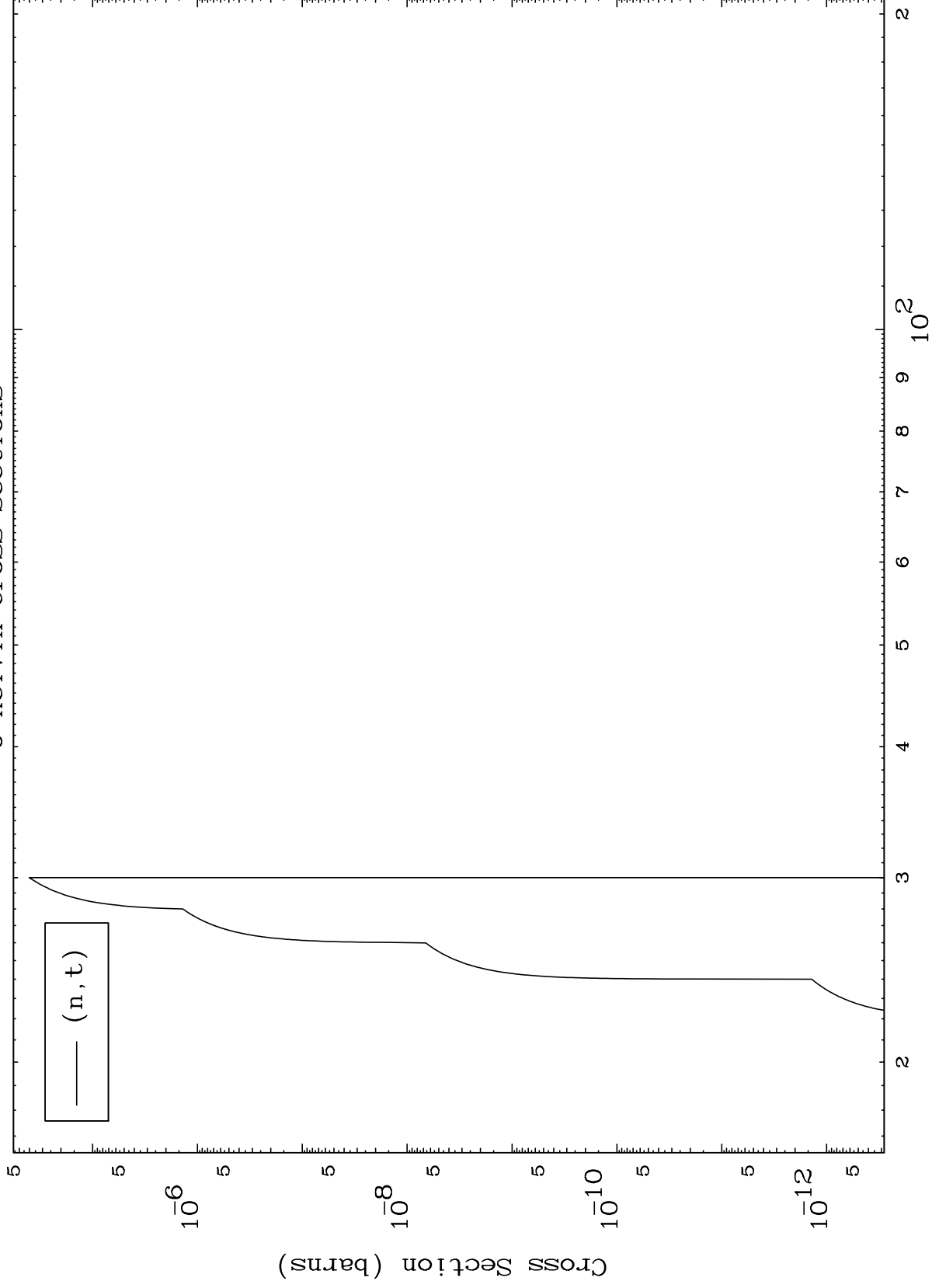
85-At-202m



MAT 8523

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

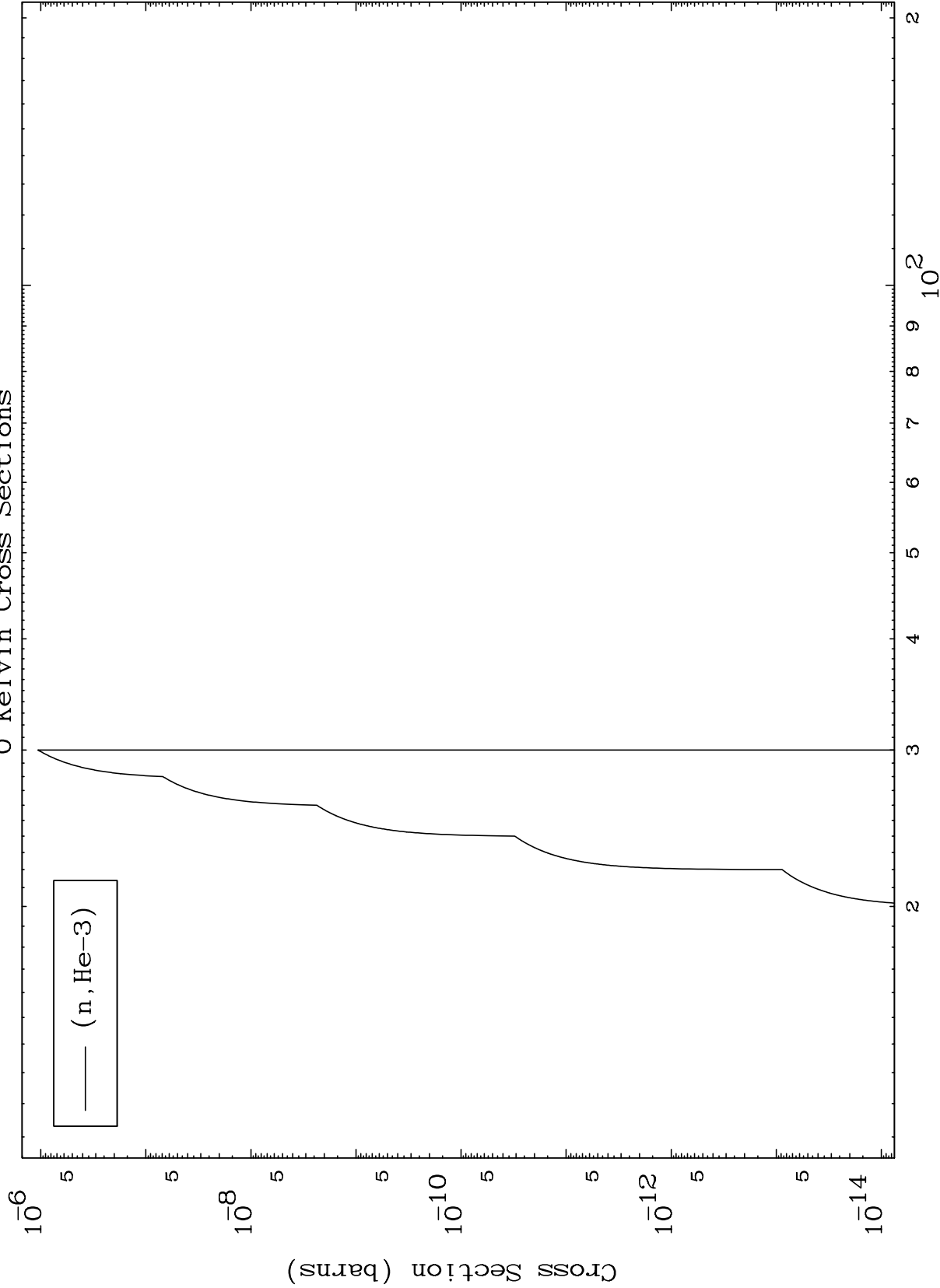
85-At-202m



8

Incident Energy (MeV)

85-At-202m

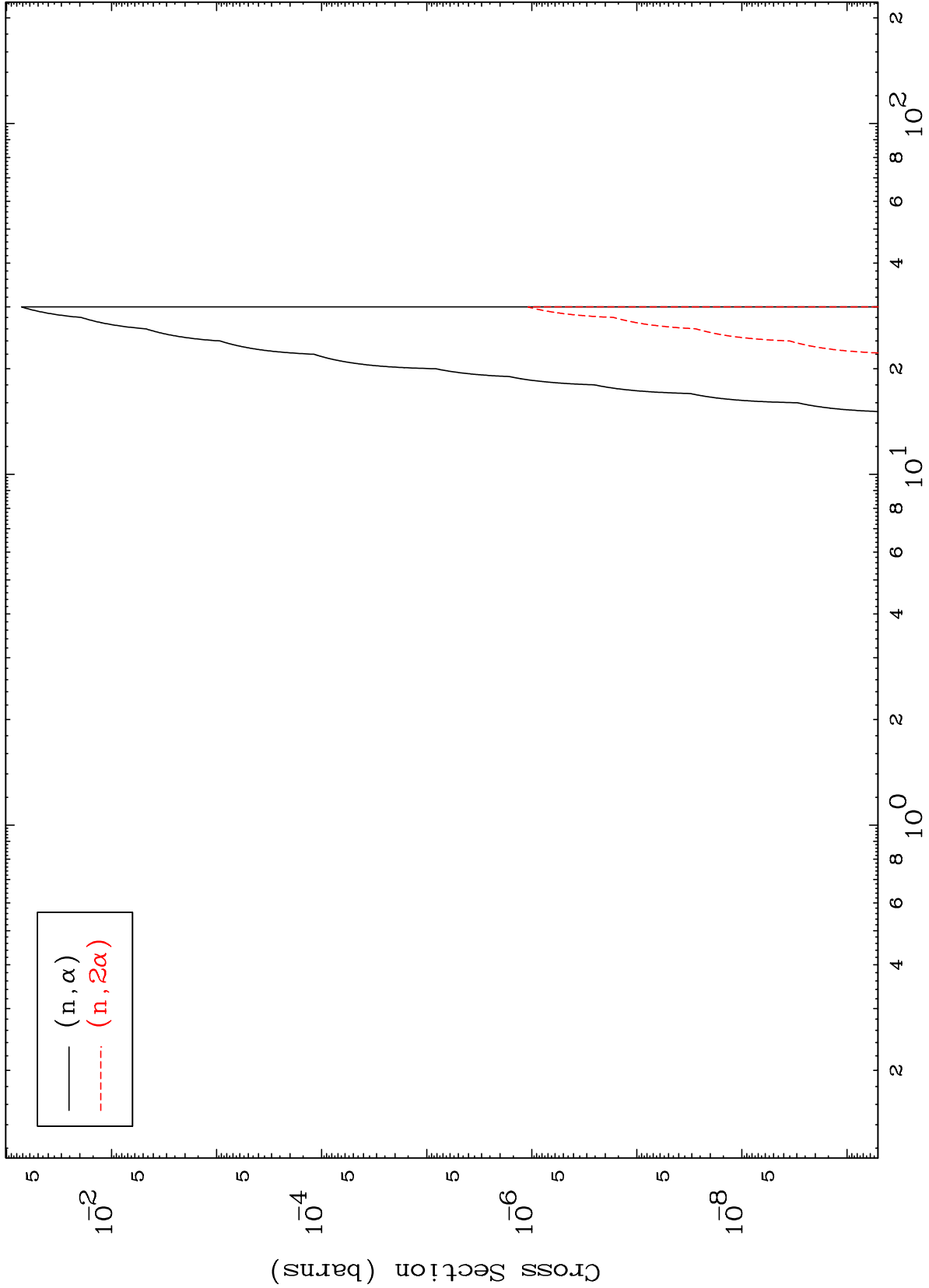


MAT 8523

( $\alpha, \alpha$ ) Levels

85-At-202m

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

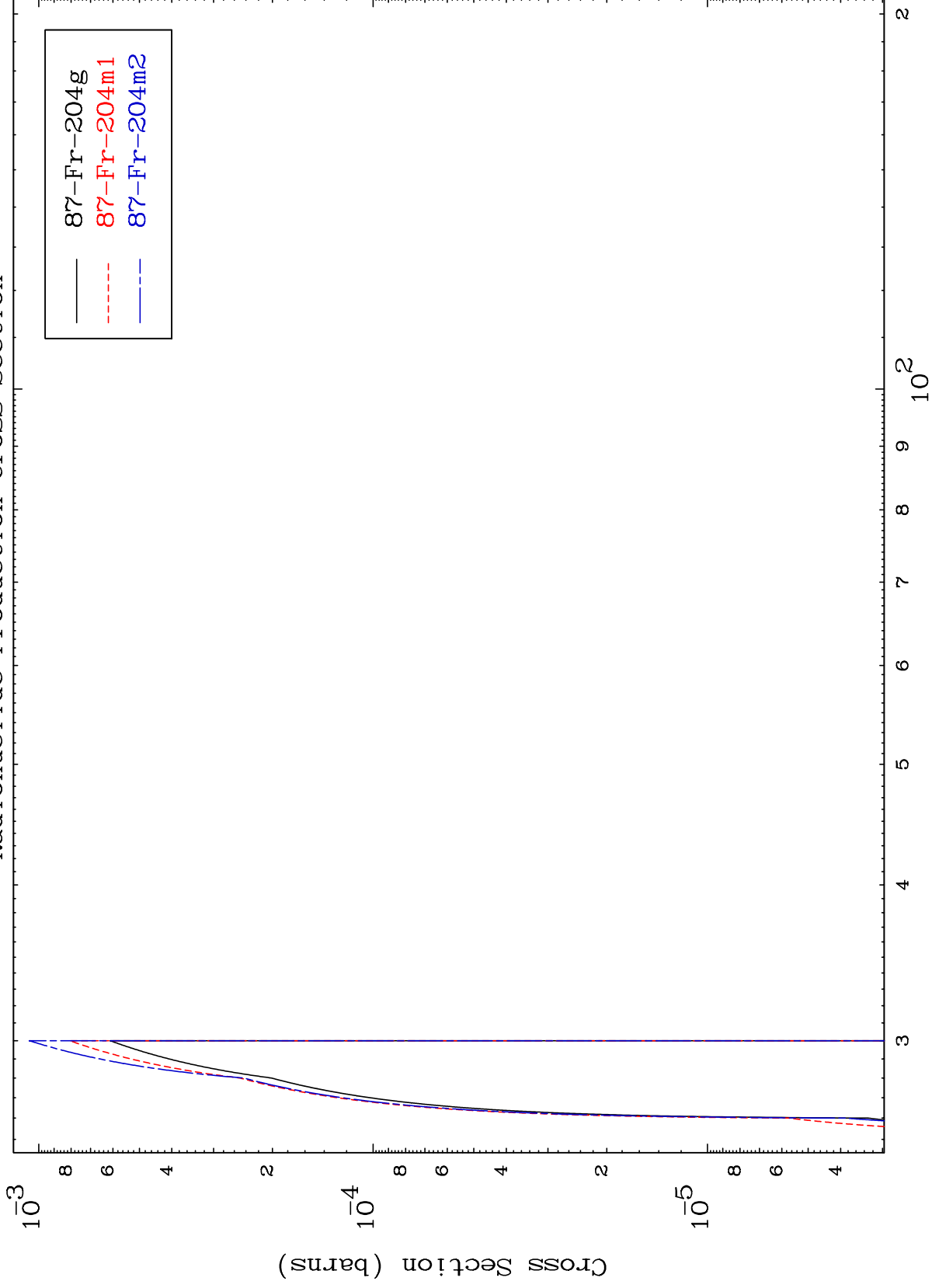
Incident Energy (MeV)

85-At-202m

MAT 8523

85-At-202m

(n,2n)  
Radionuclide Production Cross Section



11

85-At-202m

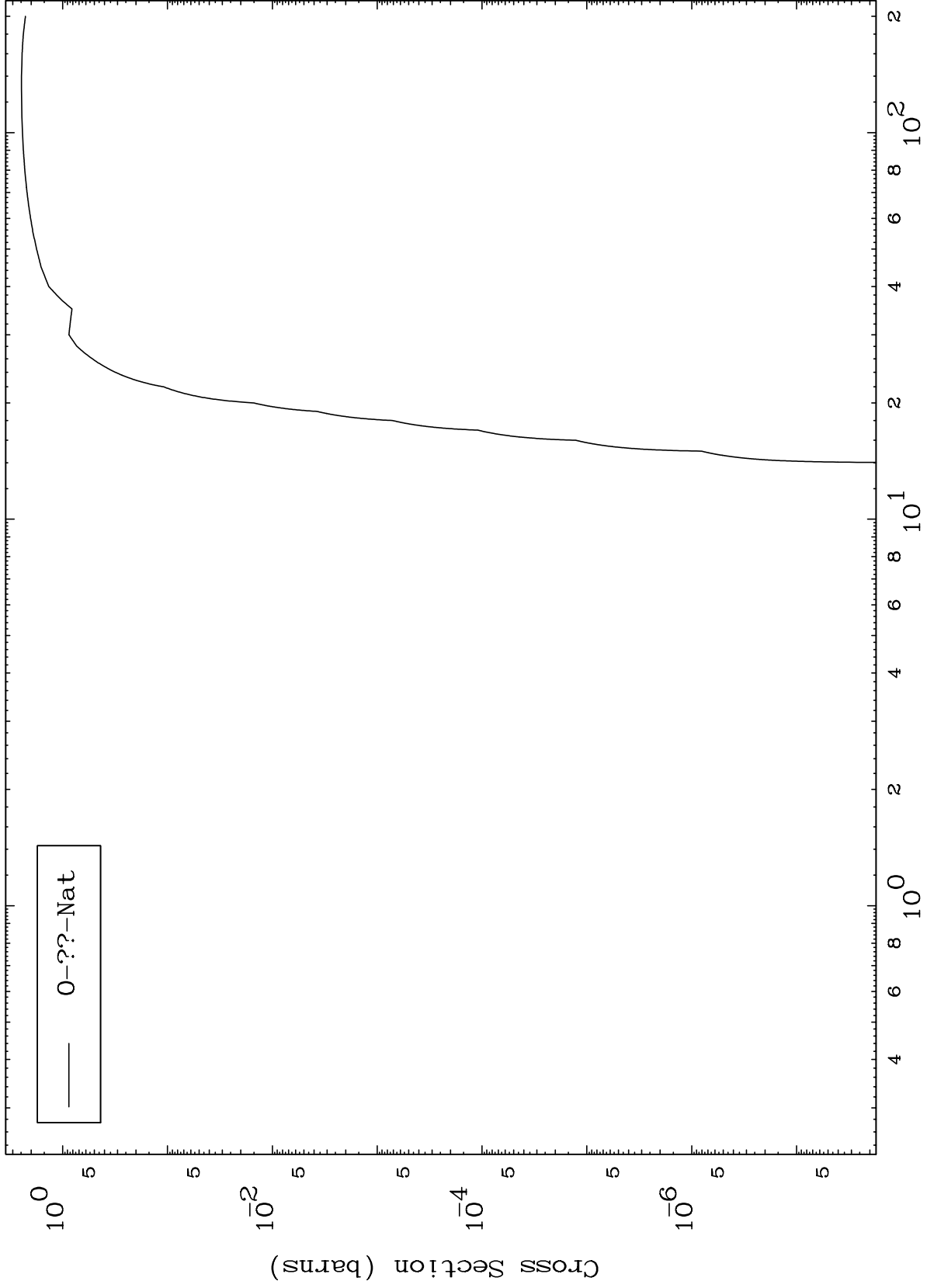
Incident Energy (MeV)

MAT 8523

Fission

85-At-202m

Radionuclide Production Cross Section



12

Incident Energy (MeV)

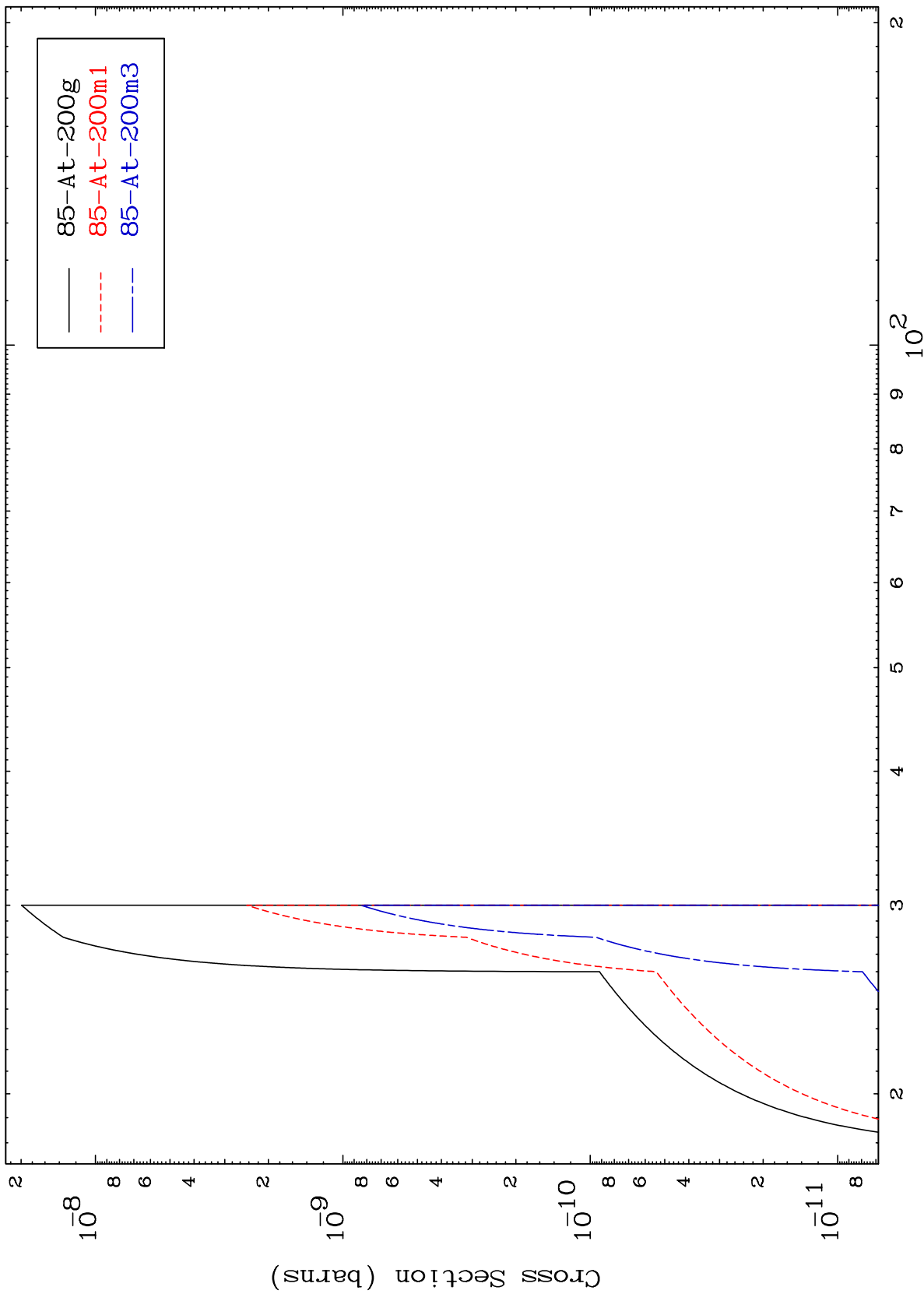
85-At-202m

MAT 8523

$(n,2n) \alpha$

85-At-202m

Radionuclide Production Cross Section

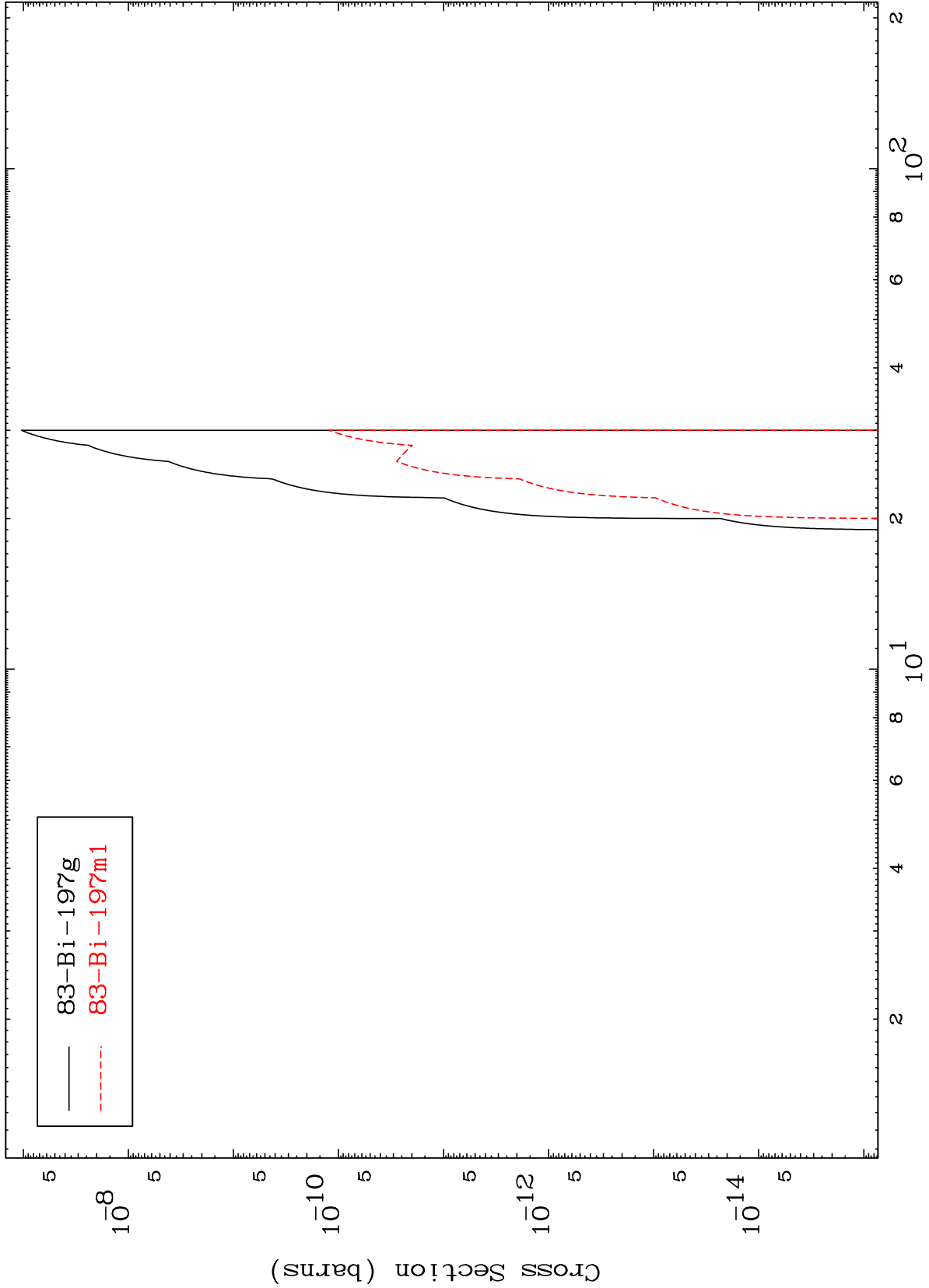


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(n,n') 2 $\alpha$

85-At-202m

Radionuclide Production Cross Section

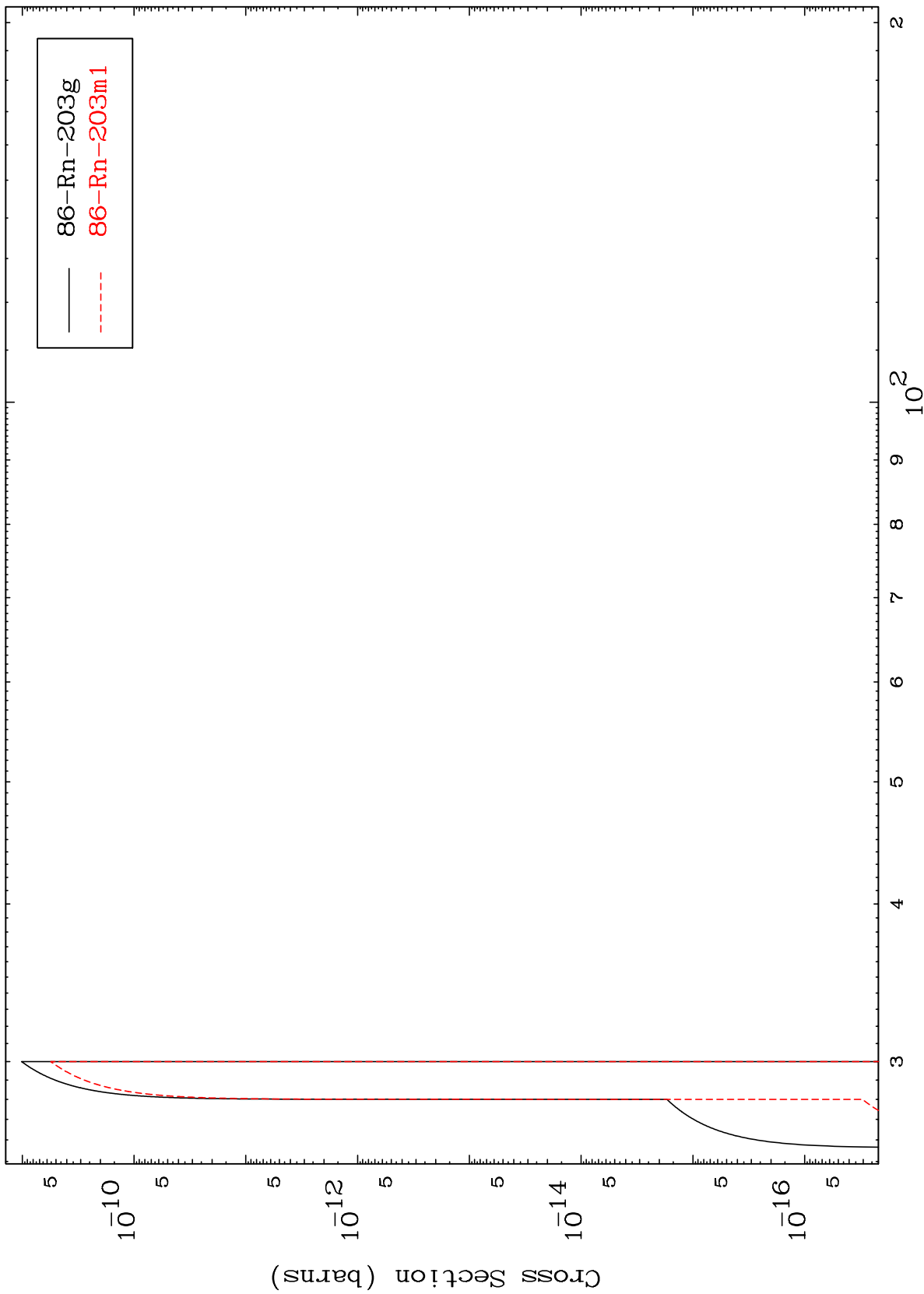


MAT 8523

(n,2n) p

85-At-202m

Radionuclide Production Cross Section



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Incident Energy (MeV)

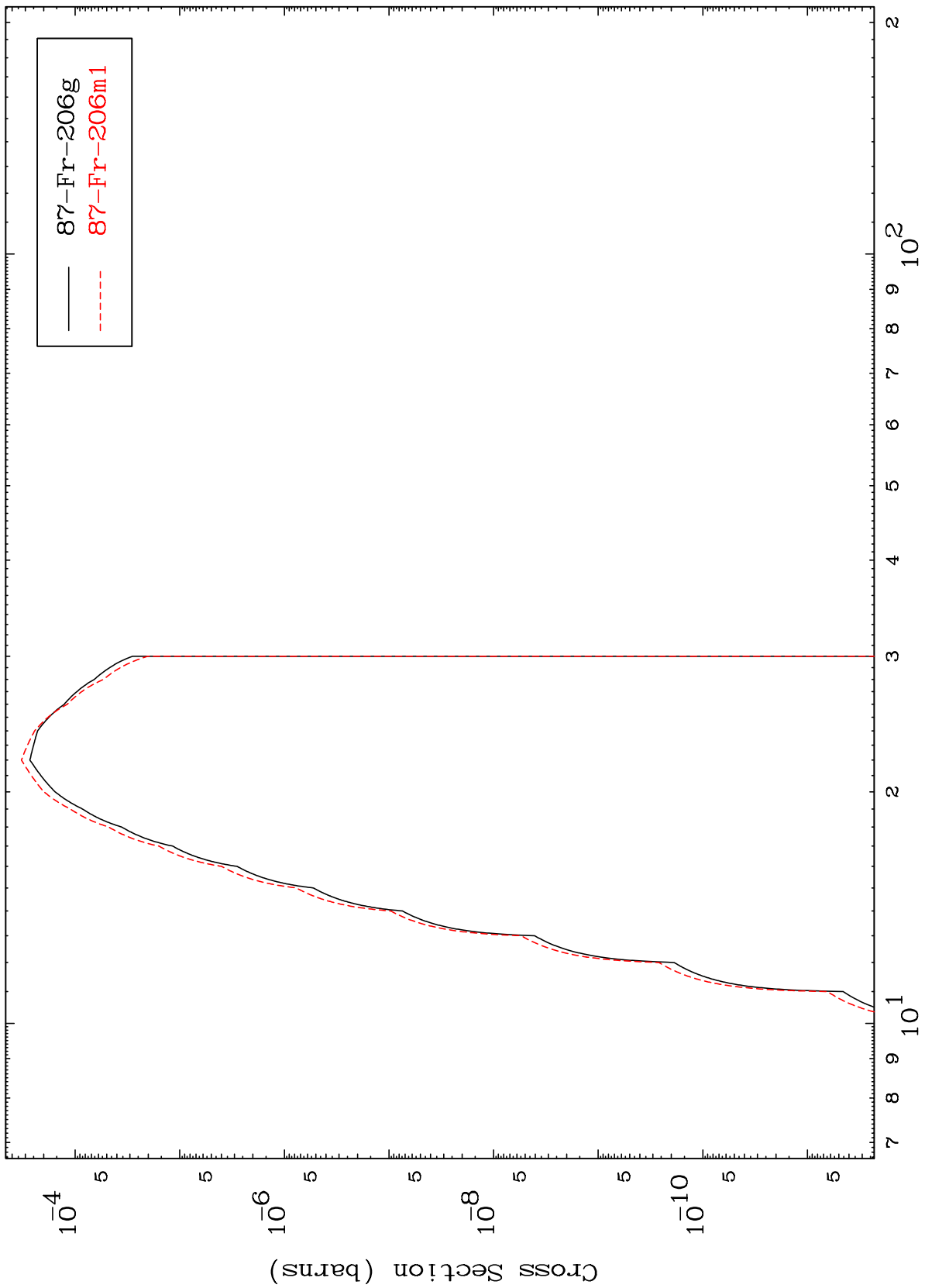
85-At-202m



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85-At-202m

(n,  $\gamma$ )  
Radionuclide Production Cross Section



16

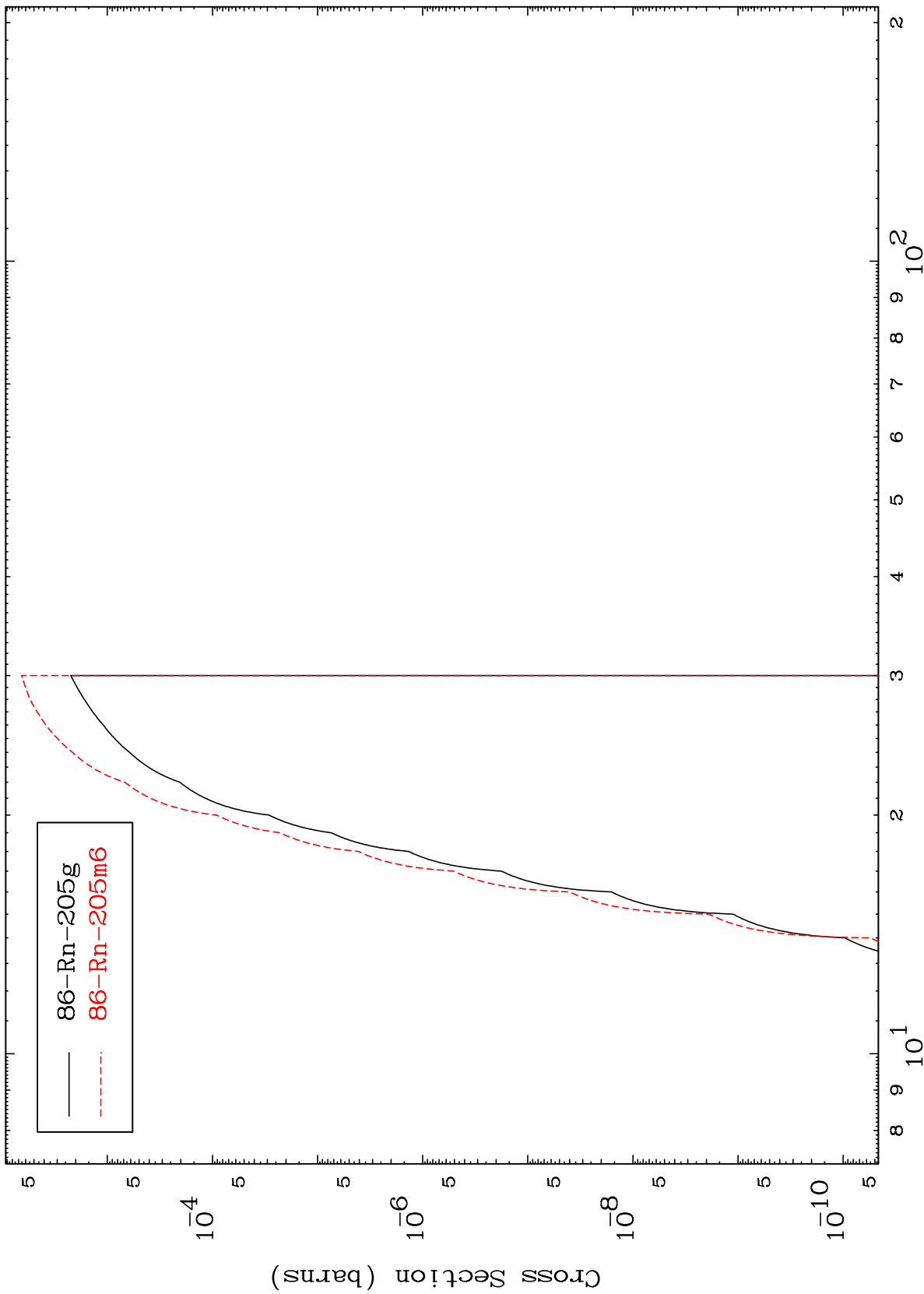
Incident Energy (MeV)

85-At-202m

MAT 8523

85-At-202m

(n,p)  
Radionuclide Production Cross Section



17

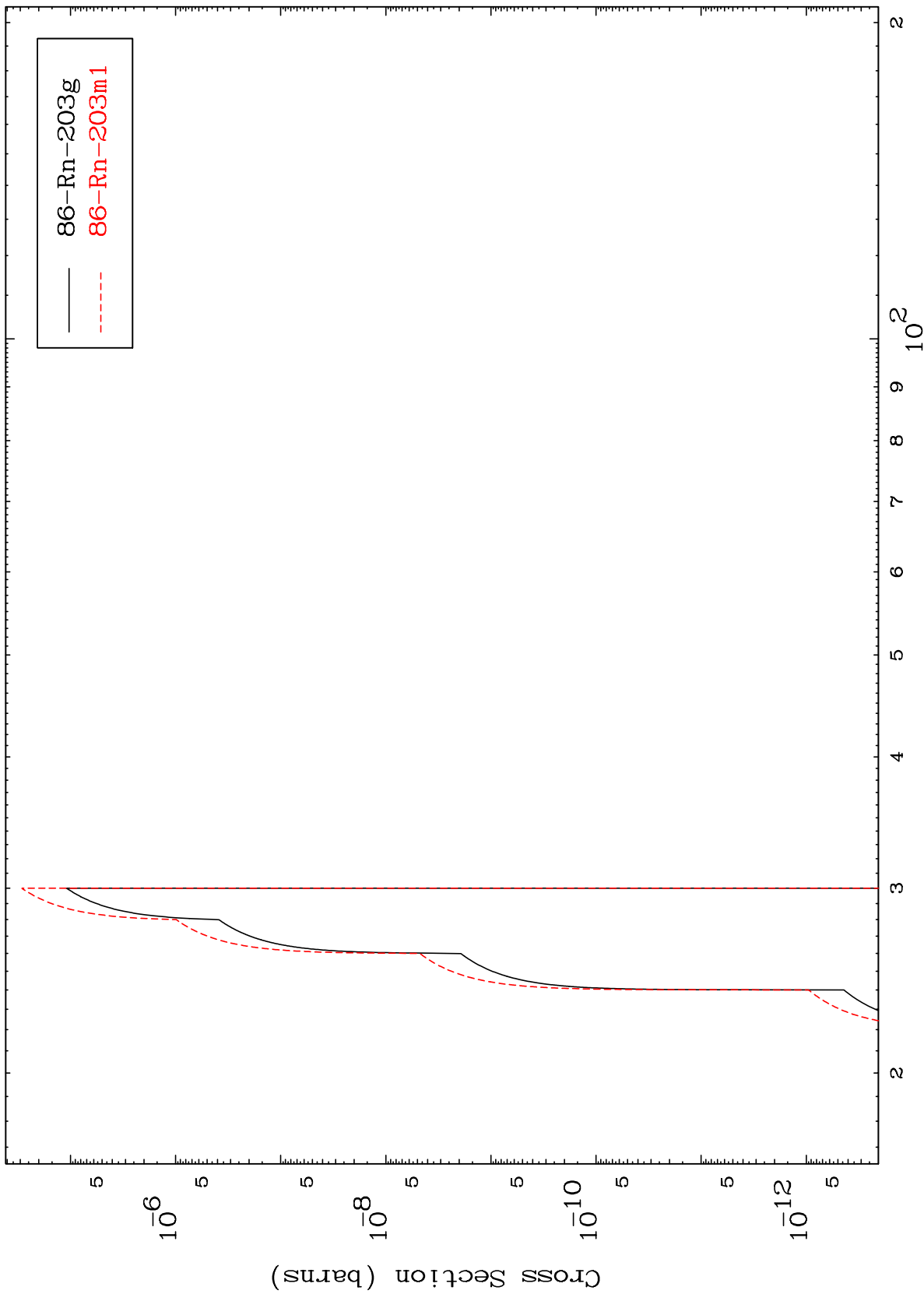
Incident Energy (MeV)

85-At-202m

MAT 8523

85-At-202m

(n,t)  
Radionuclide Production Cross Section



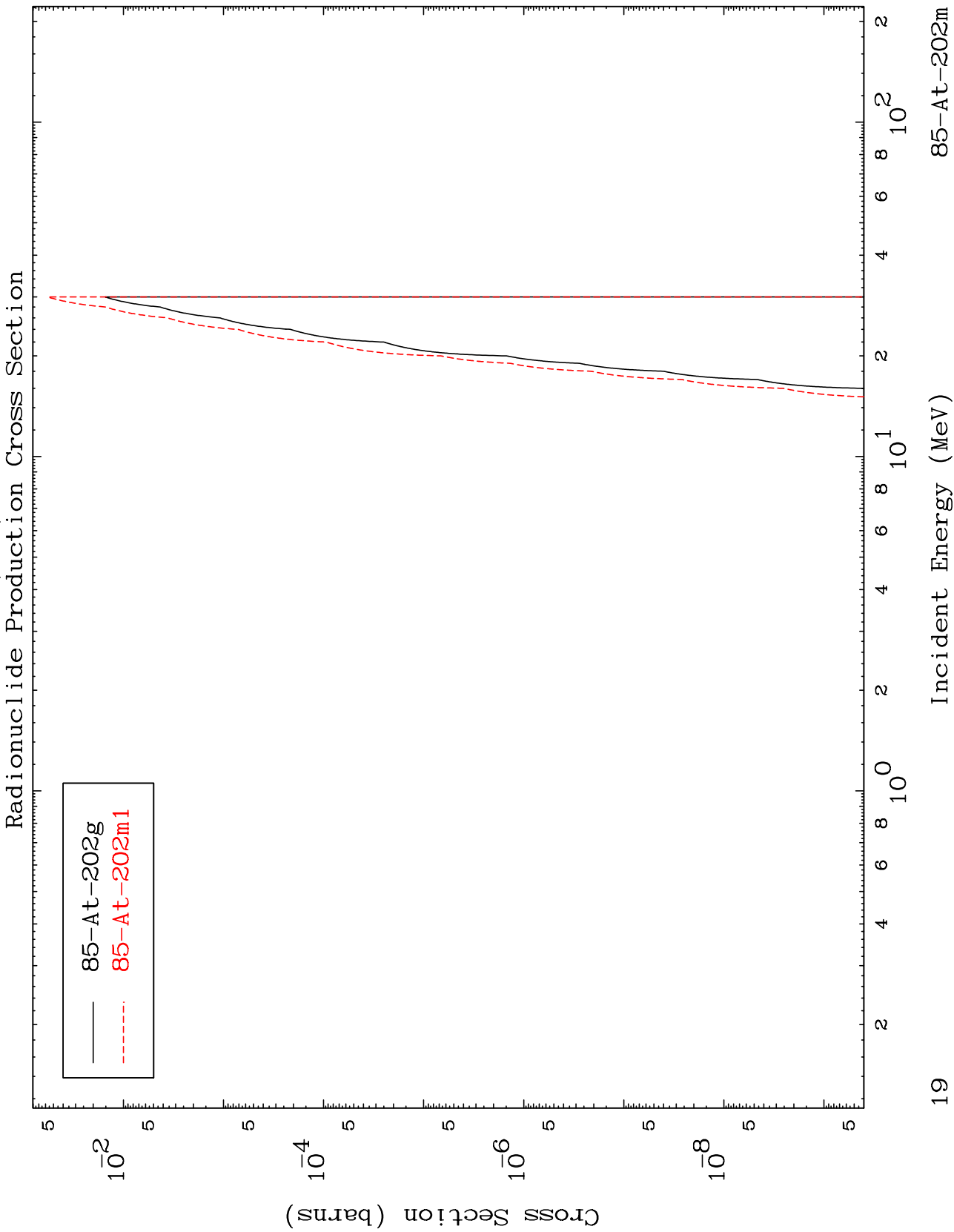
18

Incident Energy (MeV)

85-At-202m

MAT 8523

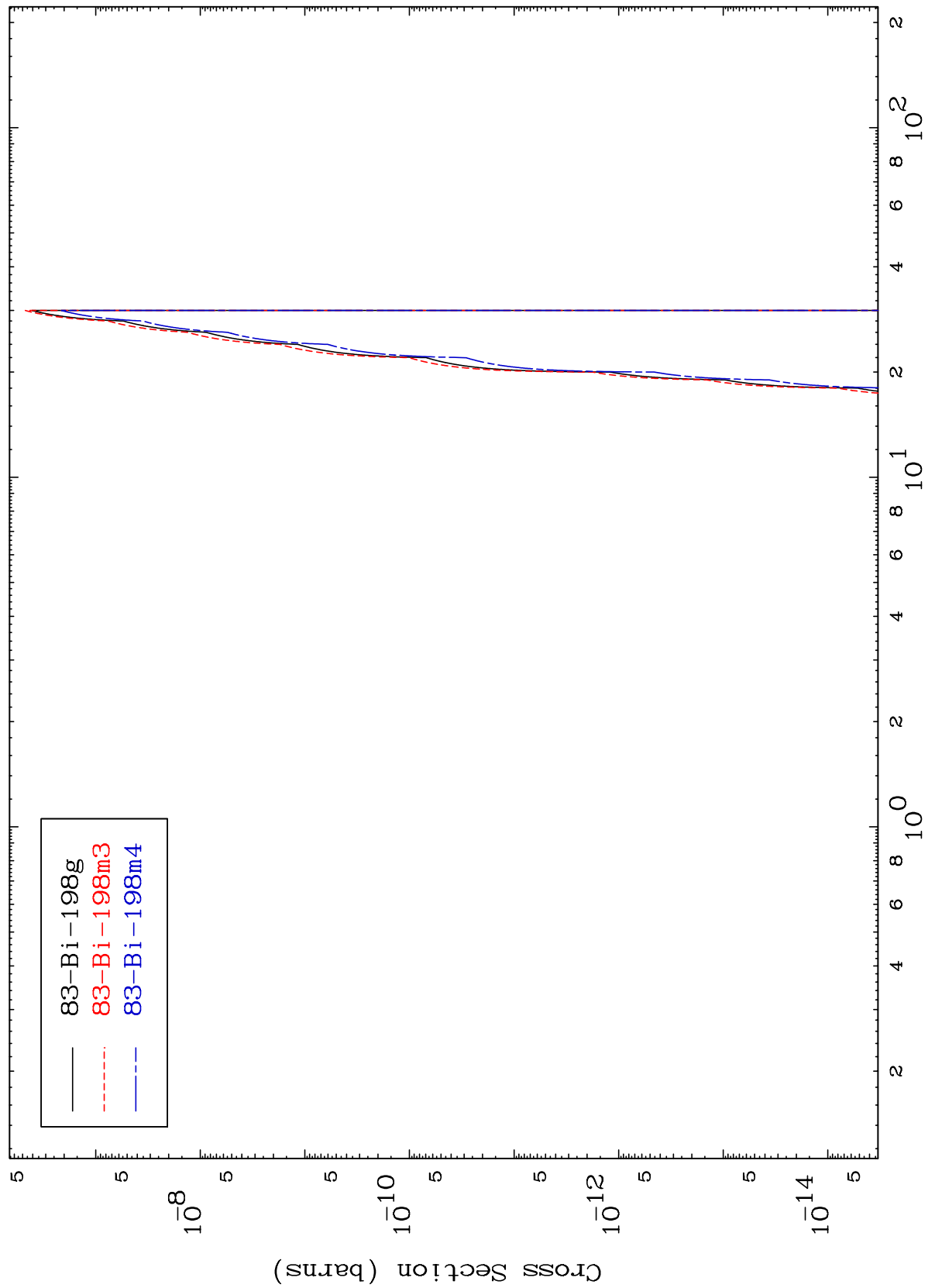
85-At-202m



MAT 8523

85-At-202m

Radionuclide Production Cross Section  
(n,2 $\alpha$ )



20

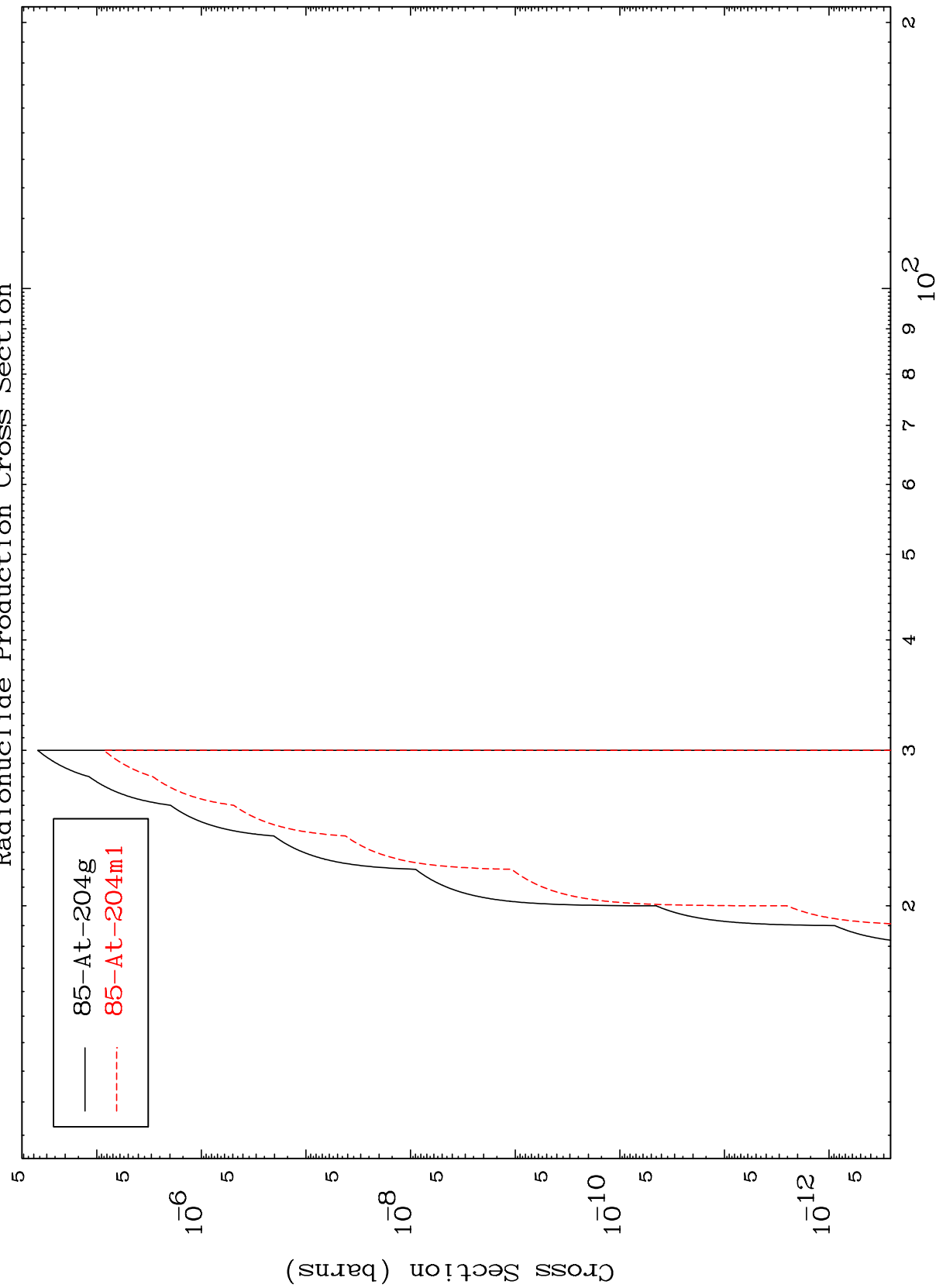
Incident Energy (MeV)

85-At-202m

MAT 8523

85-At-202m

(n,2p)  
Radionuclide Production Cross Section



Incident Energy (MeV)

85-At-202m

21

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(n,p)  $\alpha$

85-At-202m

Radionuclide Production Cross Section

