

Program EVALPLOT
(Version 2021-1)

by

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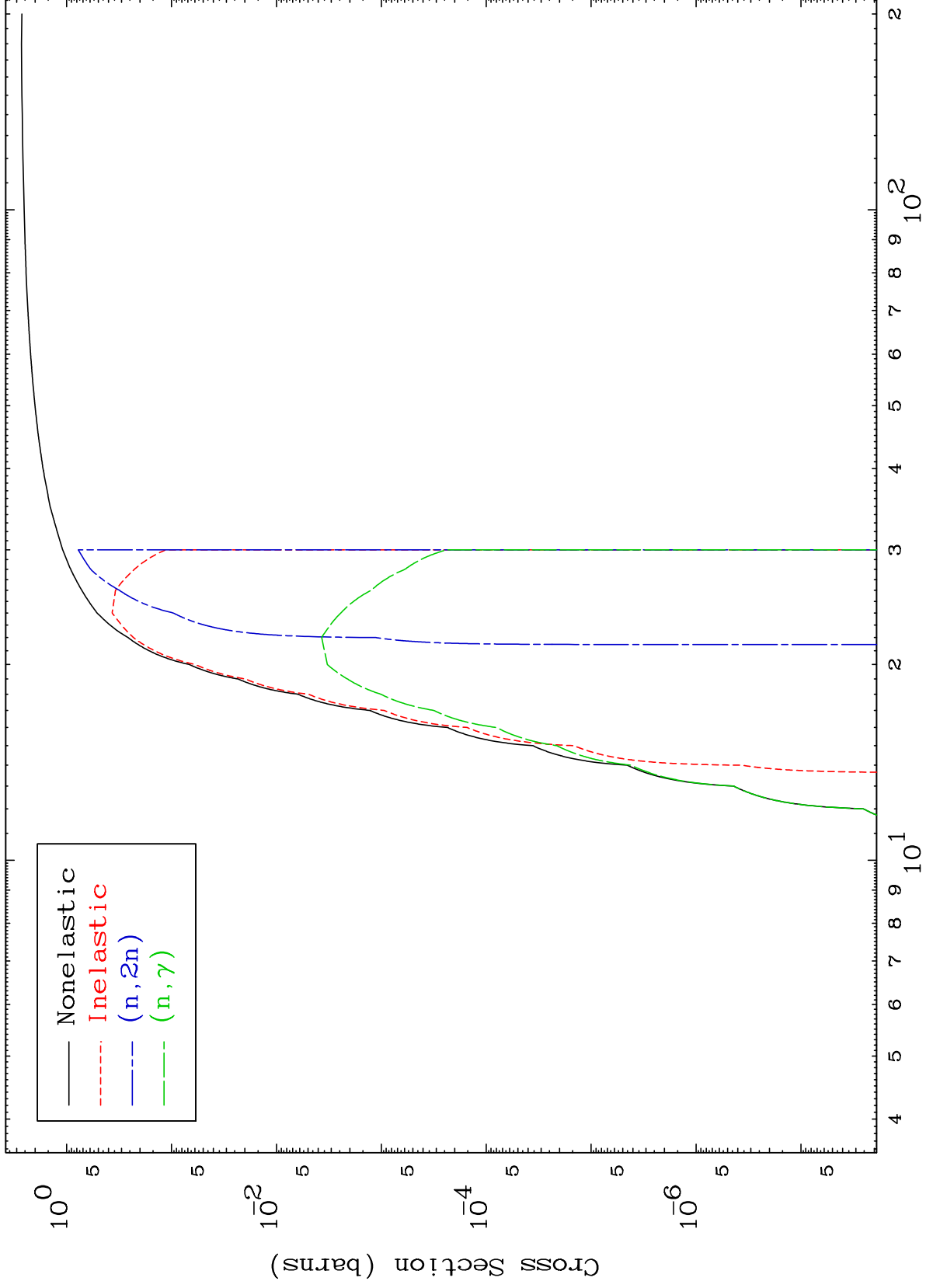
Press Mouse Button to Start

MAT 7901

0 Kelvin Cross Sections

α Major

79-Au-189



1

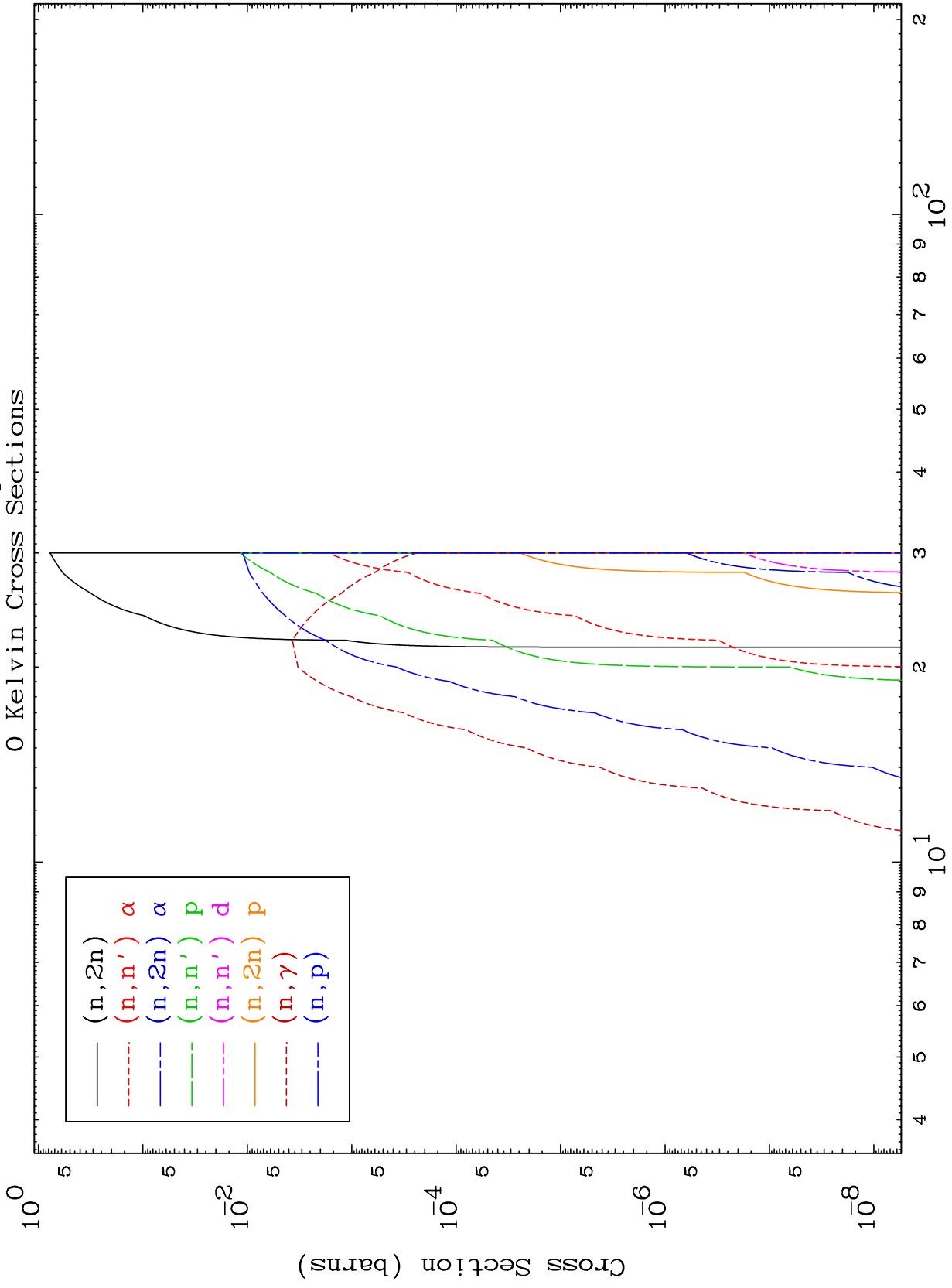
Incident Energy (MeV)

79-Au-189

MAT 7901

α Neutron Absorption
0 Kelvin Cross Sections

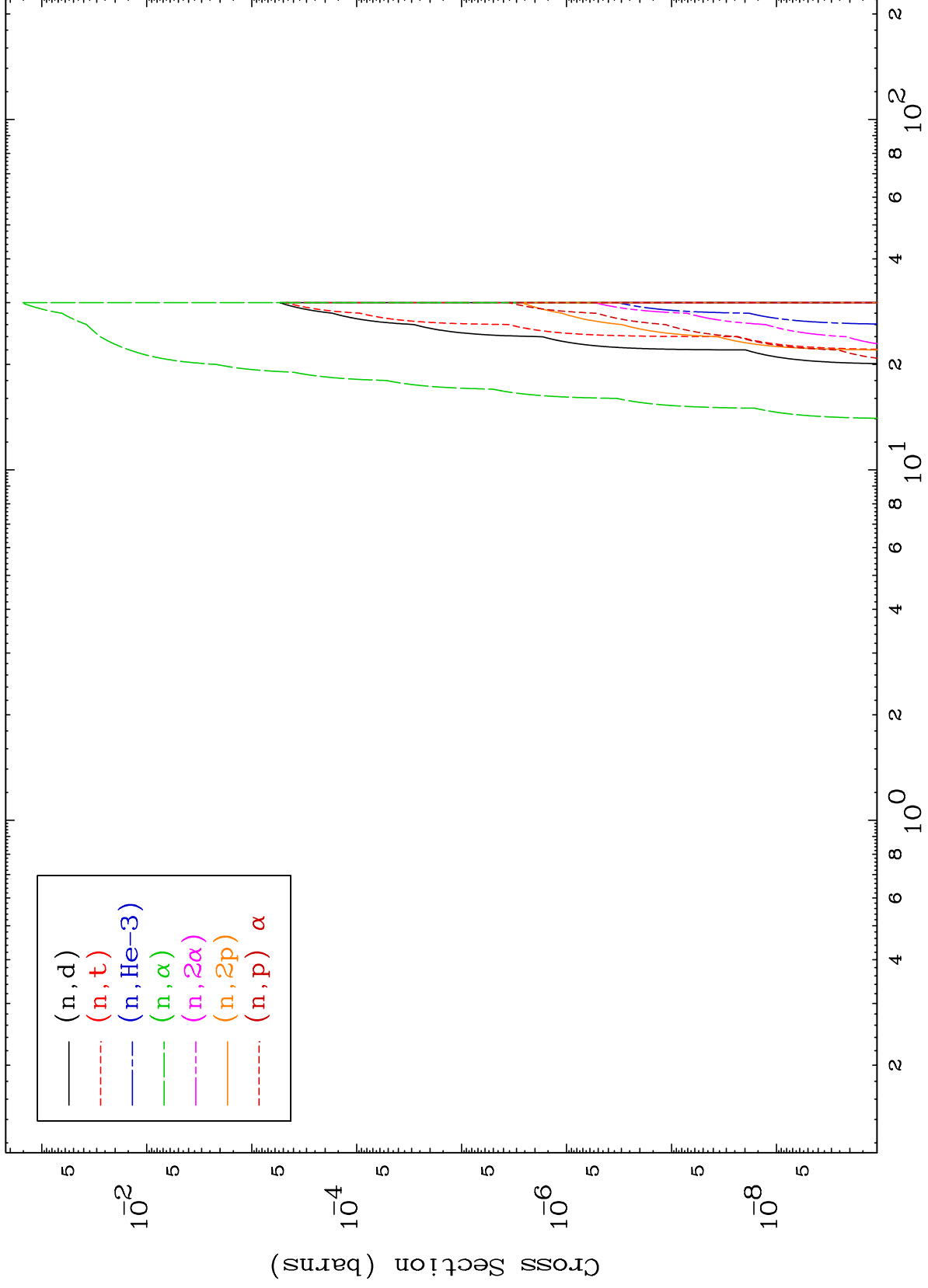
79-Au-189

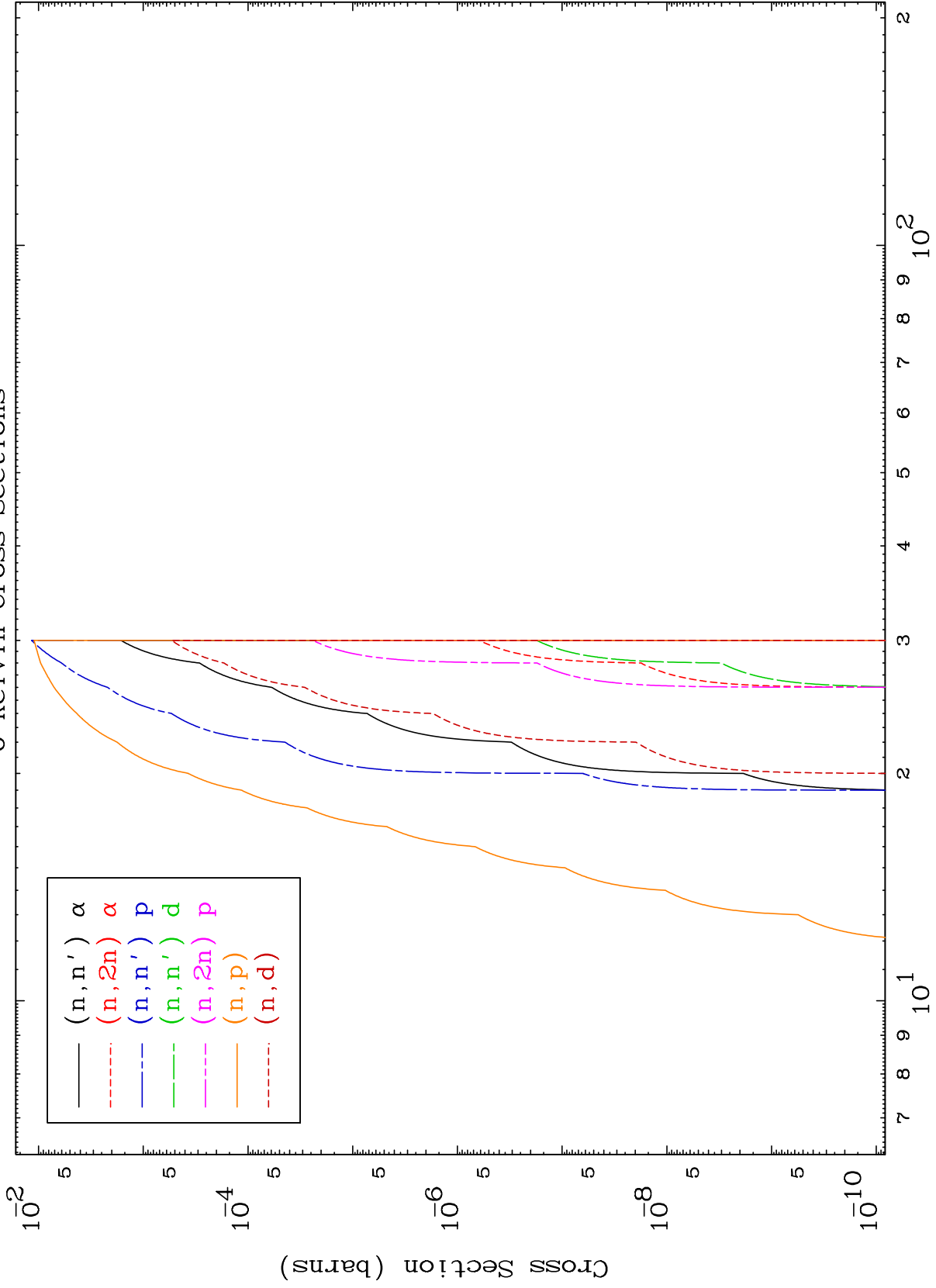


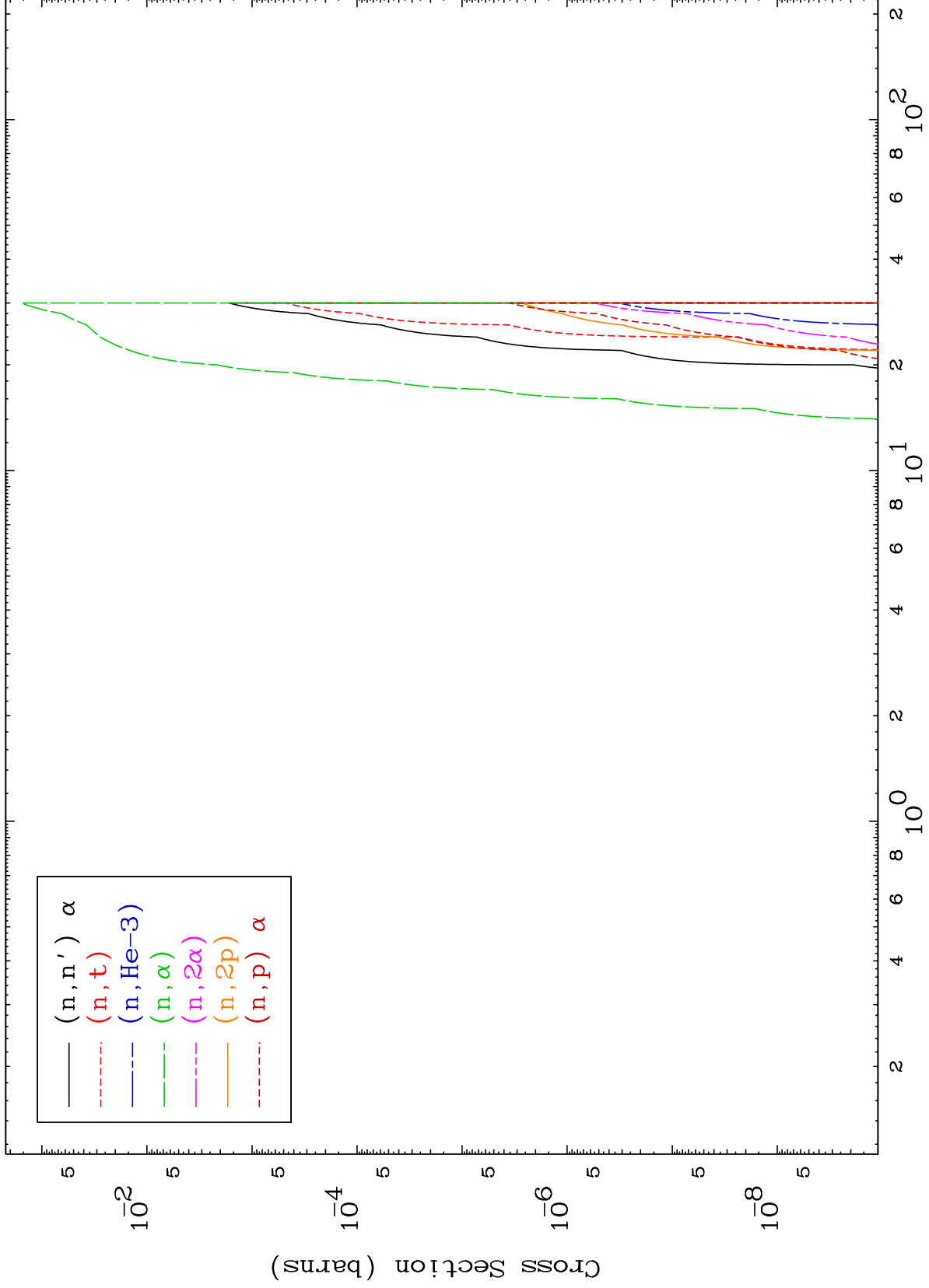
2

Incident Energy (MeV)

79-Au-189



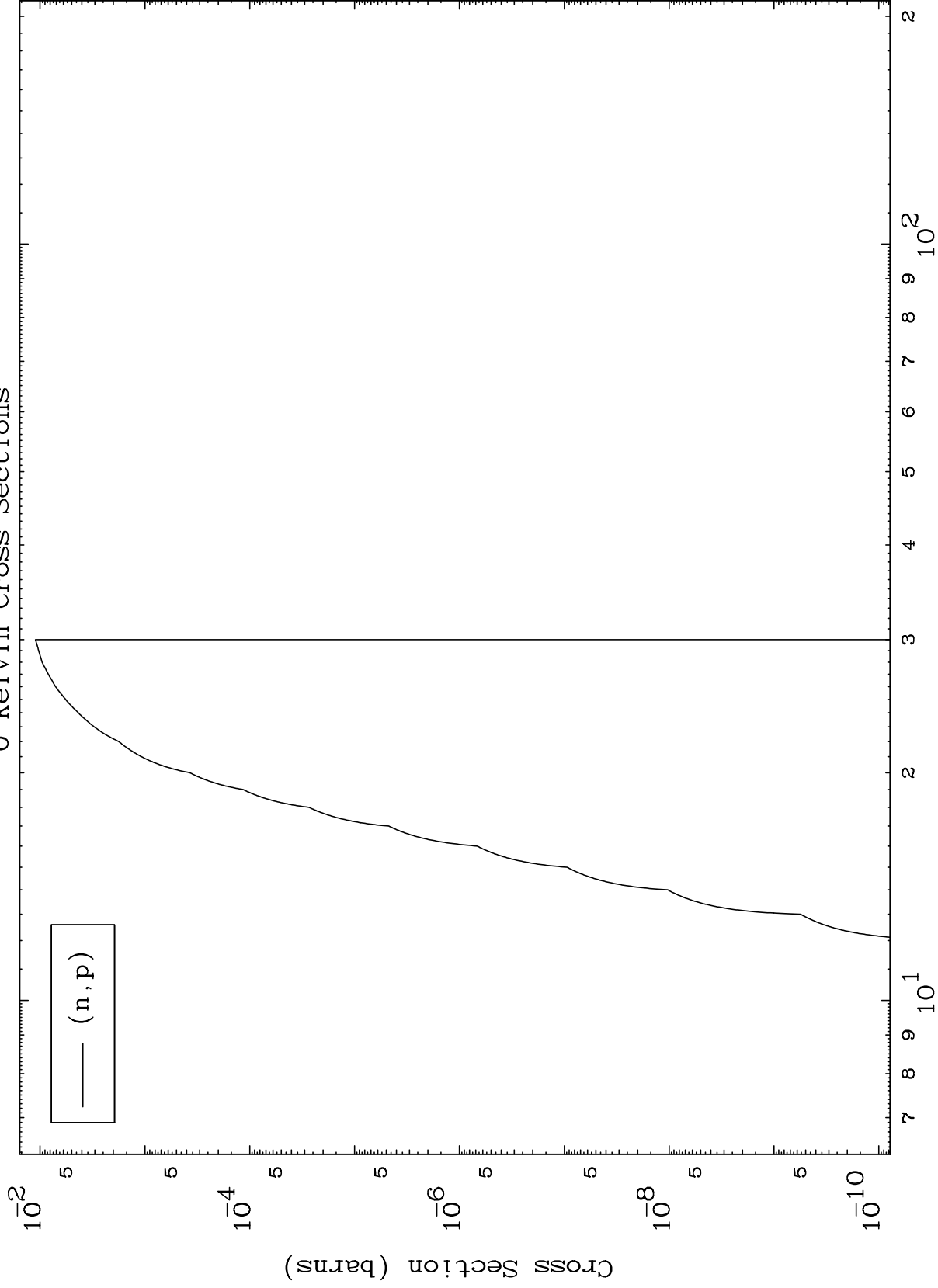




MAT 7901

(α, p) Levels
0 Kelvin Cross Sections

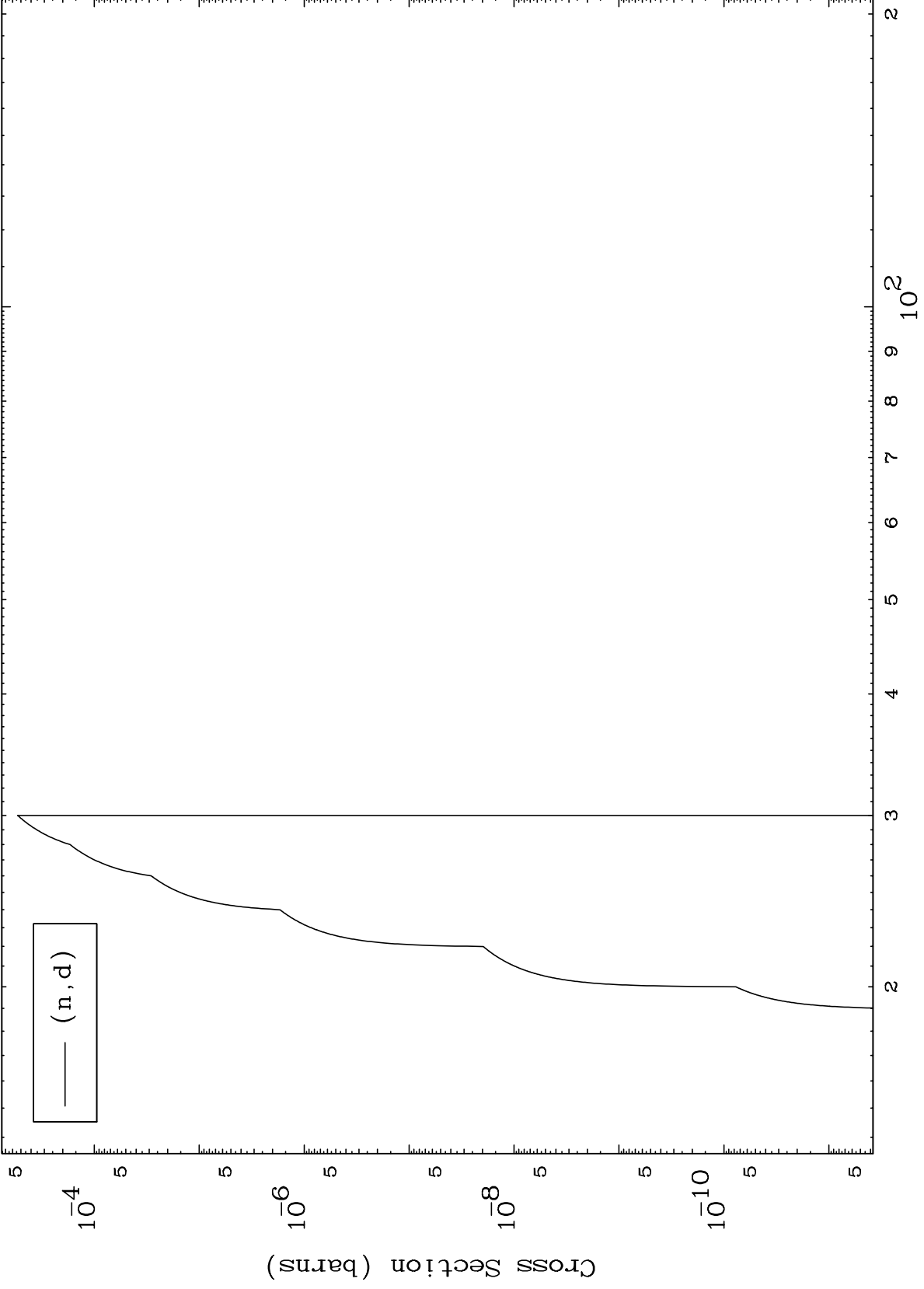
79-Au-189



6

Incident Energy (MeV)

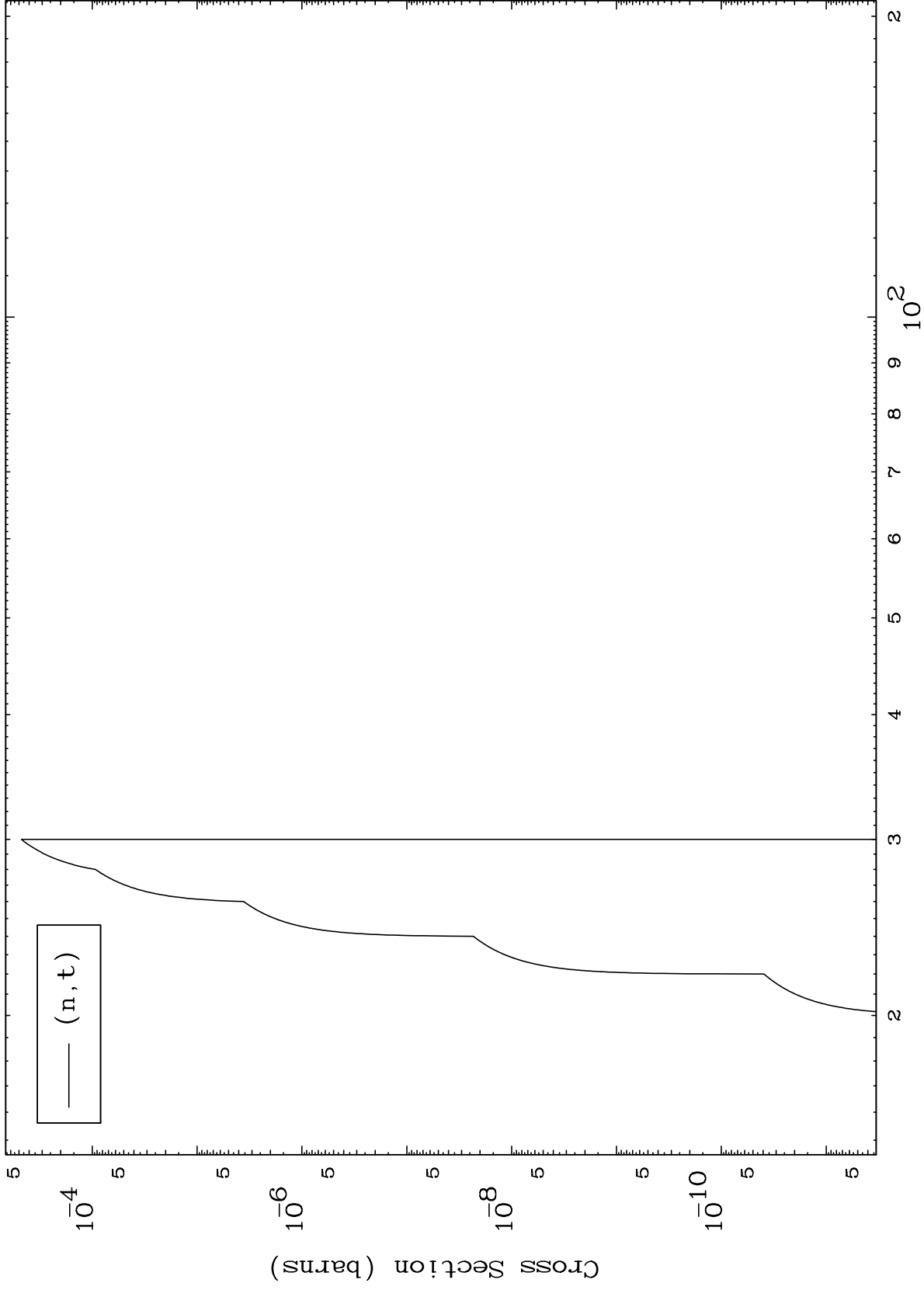
79-Au-189



MAT 7901

(α, t) Levels
0 Kelvin Cross Sections

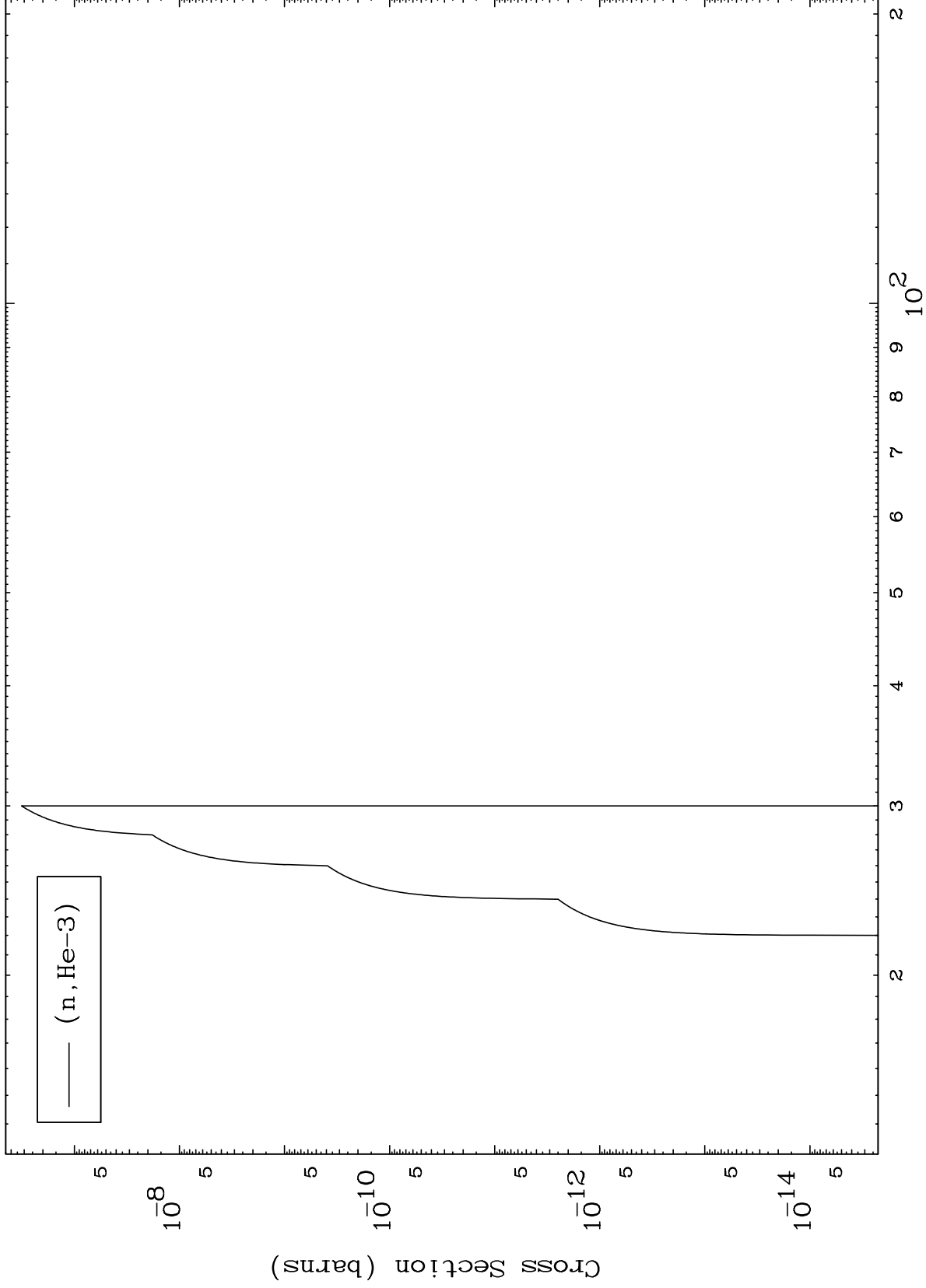
79-Au-189



8

Incident Energy (MeV)

79-Au-189

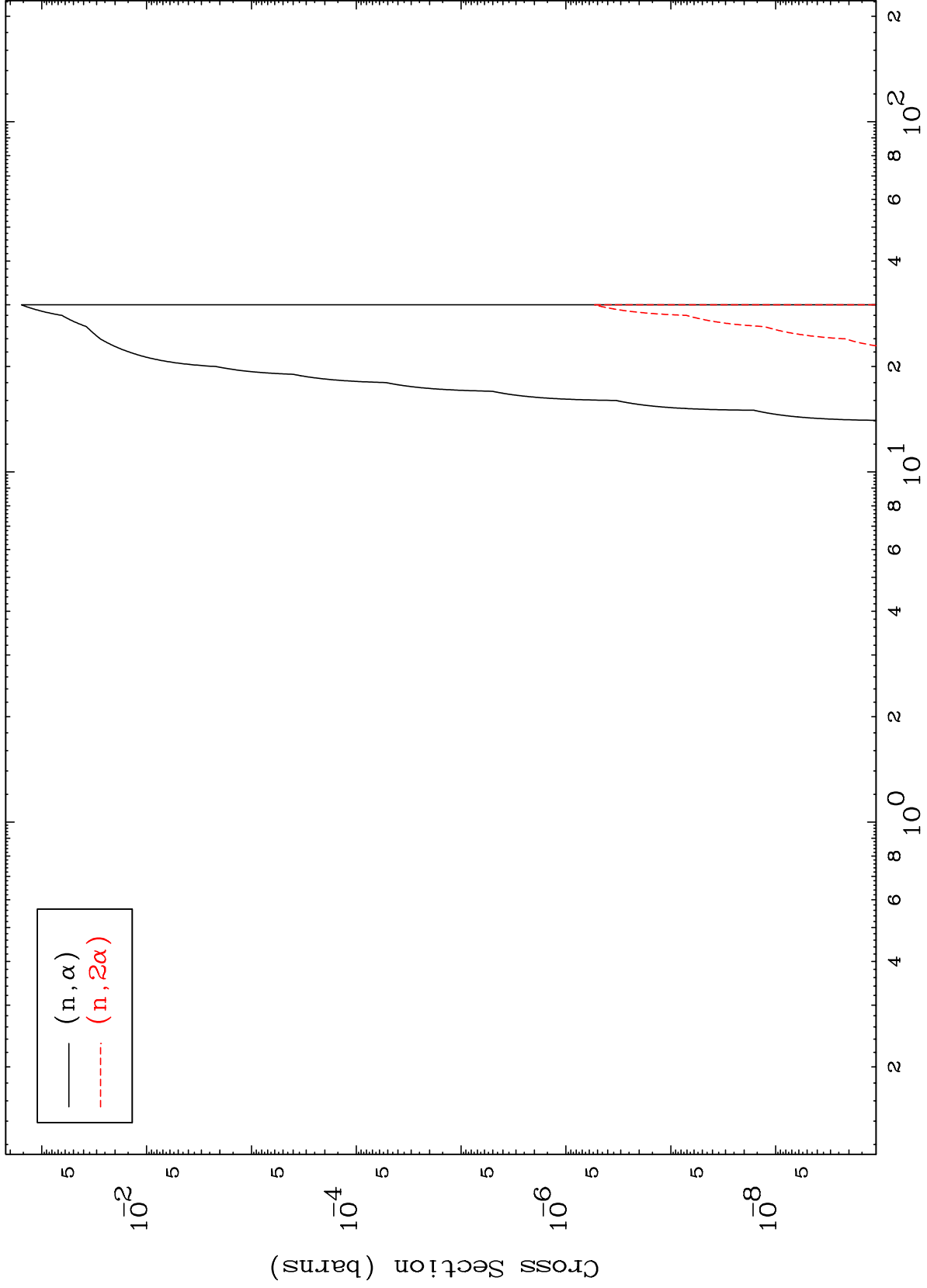


MAT 7901

(α, α) Levels

79-Au-189

0 Kelvin Cross Sections

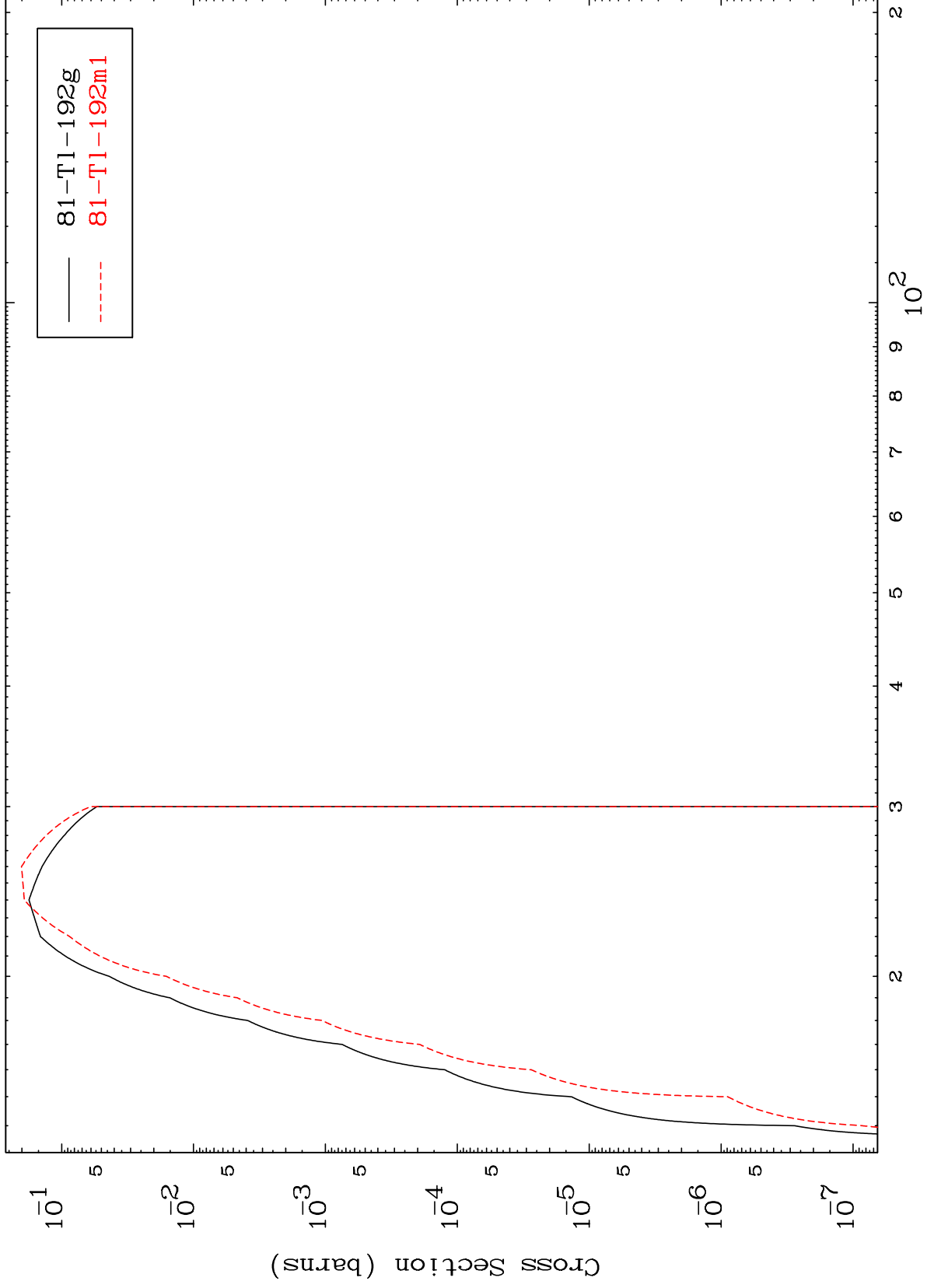


10

Incident Energy (MeV)

79-Au-189

Inelastic
Radionuclide Production Cross Section

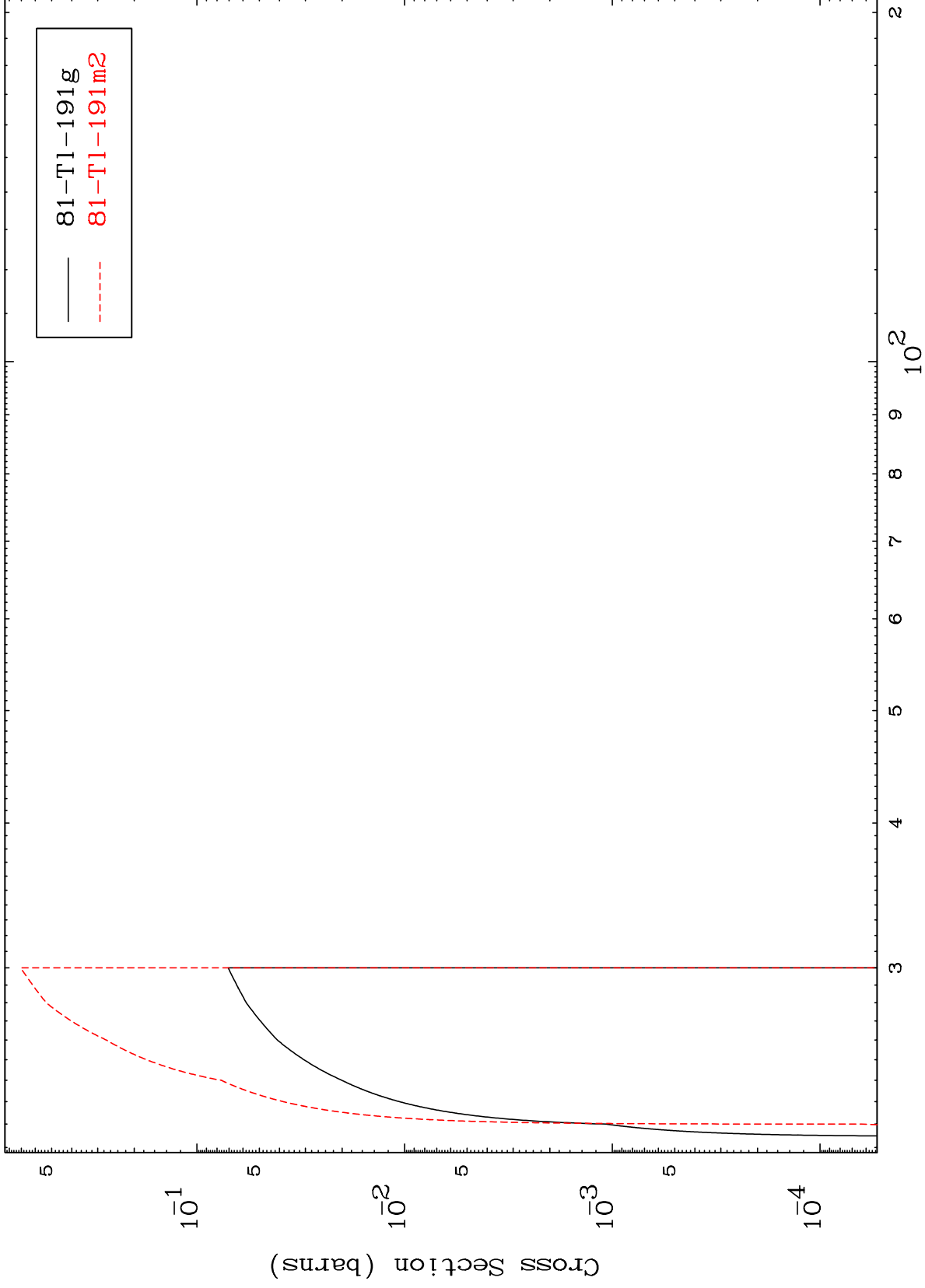


MAT 7901

(n,2n)

79-Au-189

Radionuclide Production Cross Section



12

Incident Energy (MeV)

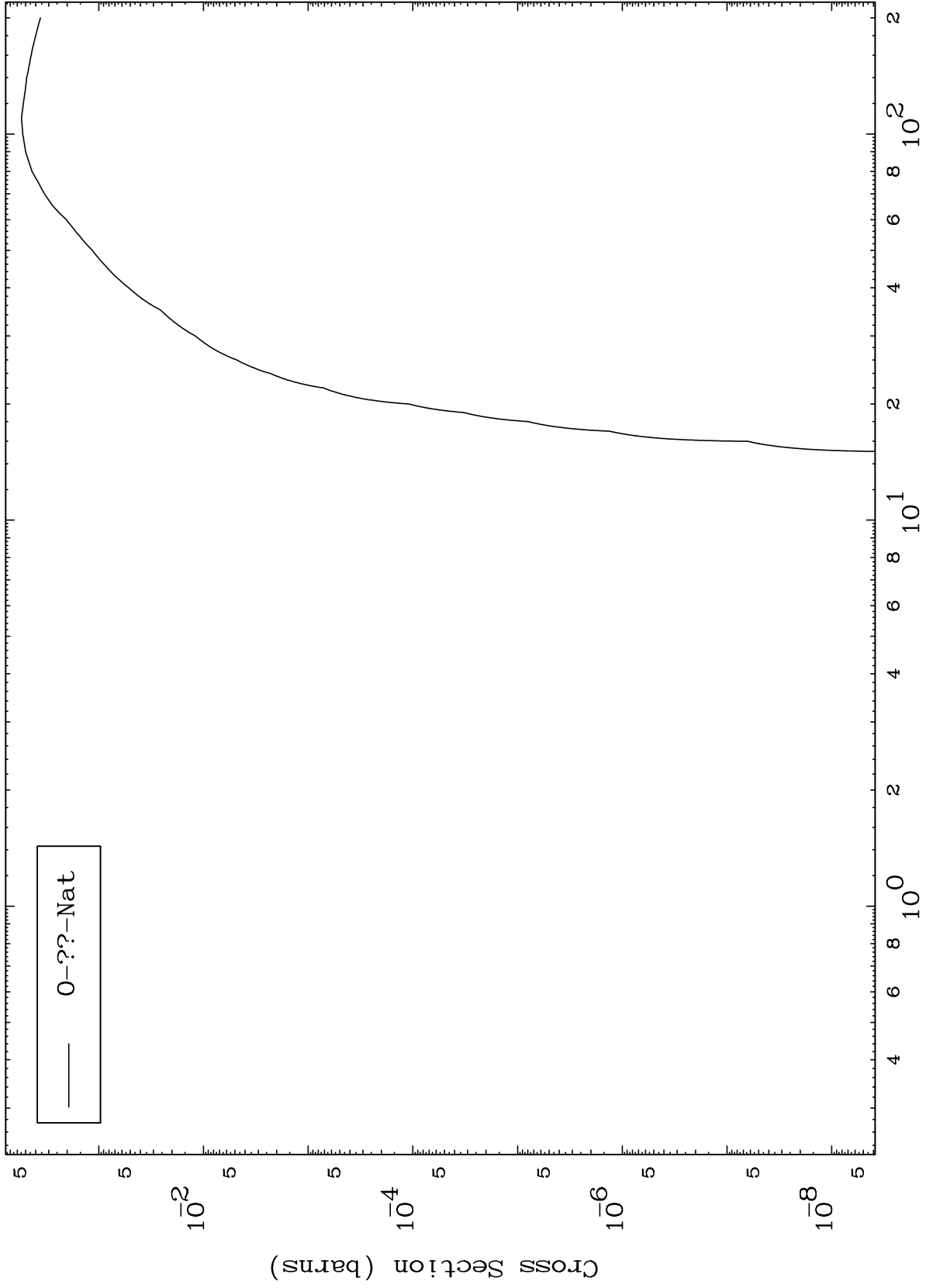
79-Au-189

MAT 7901

Fission

⁷⁹Au-189

Radionuclide Production Cross Section

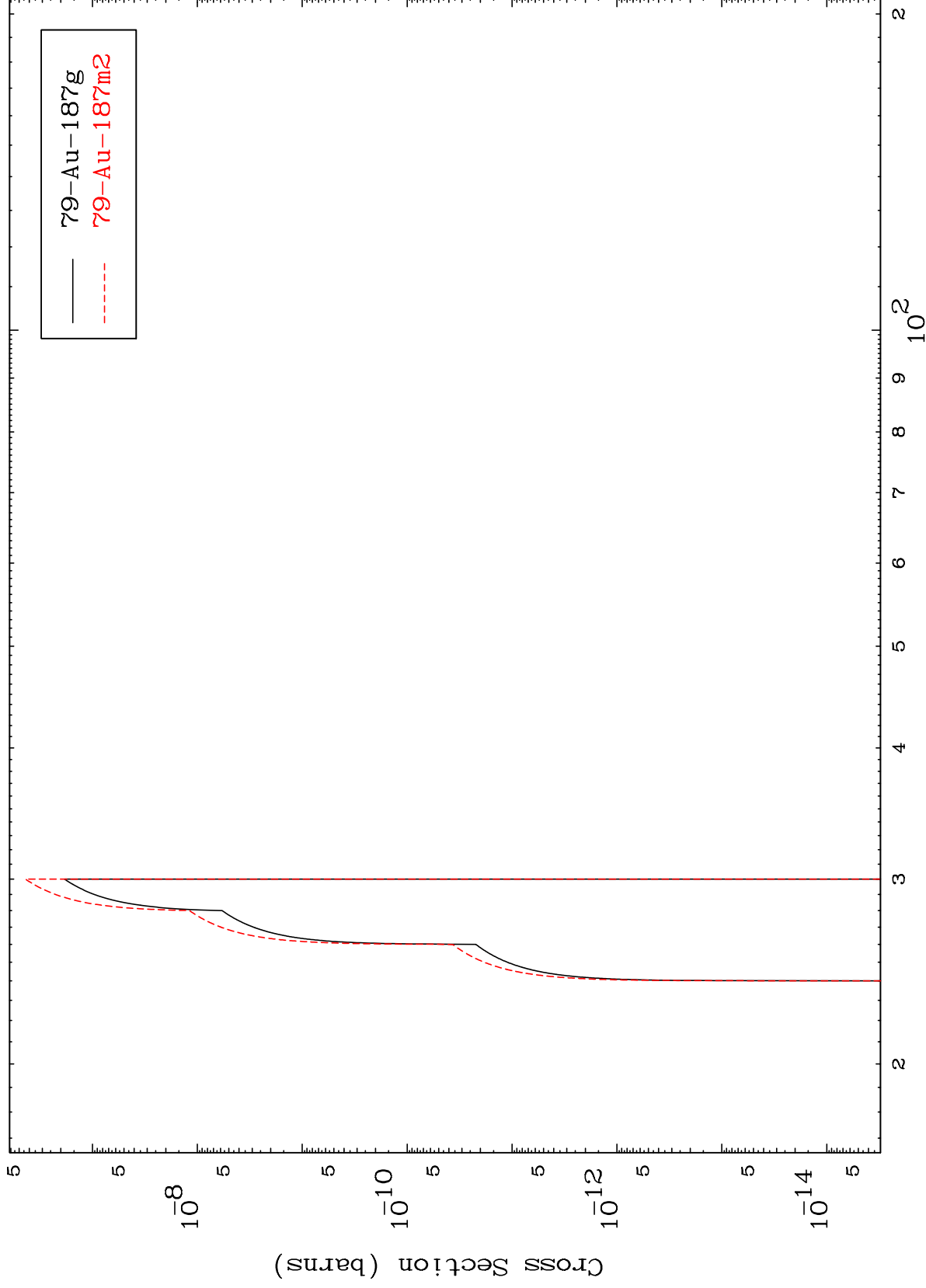


MAT 7901

$(n,2n) \alpha$

$^{79}\text{Au-189}$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

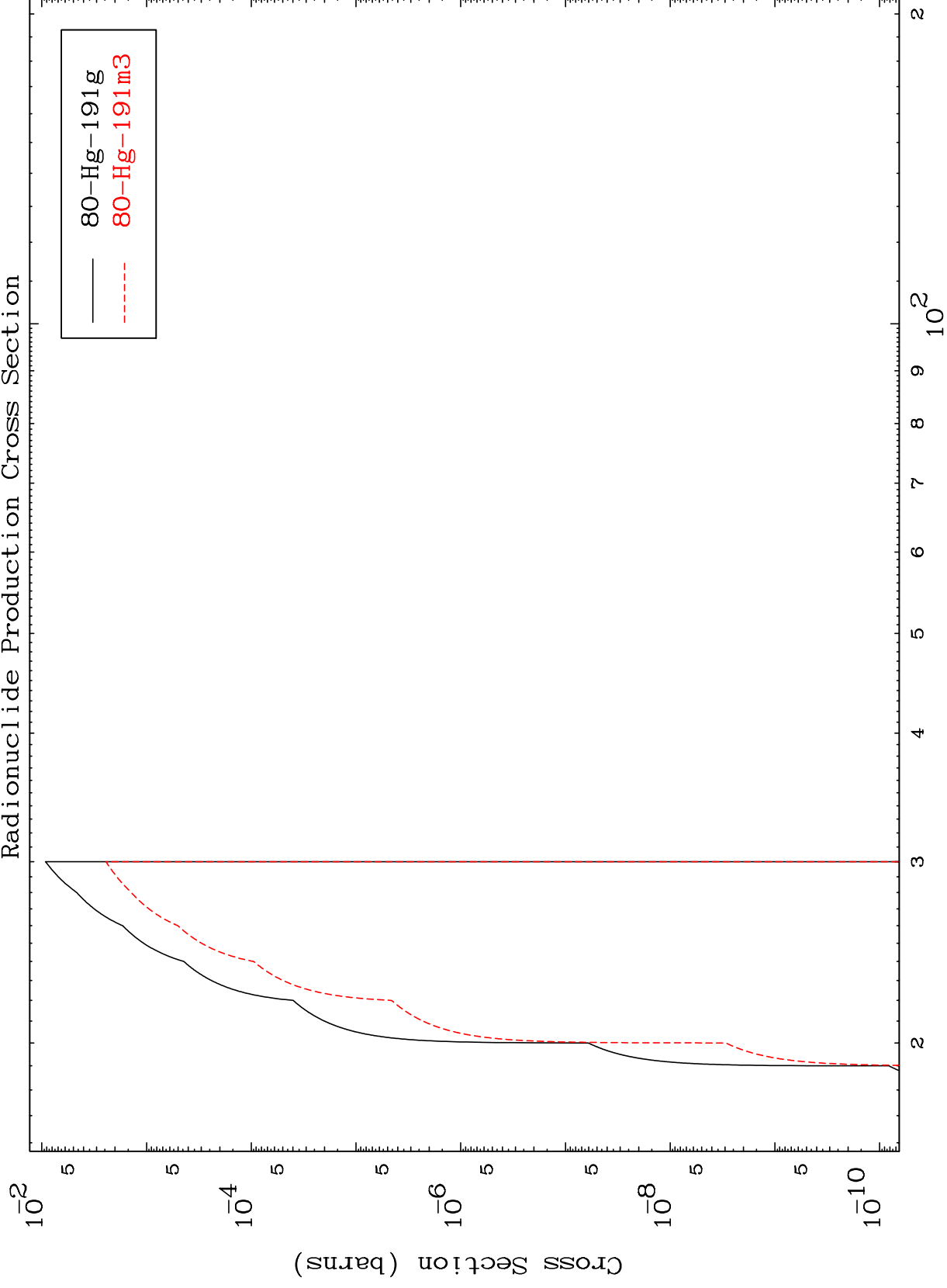
$^{79}\text{Au-189}$

MAT 7901

(n, n') p

79-Au-189

Radionuclide Production Cross Section

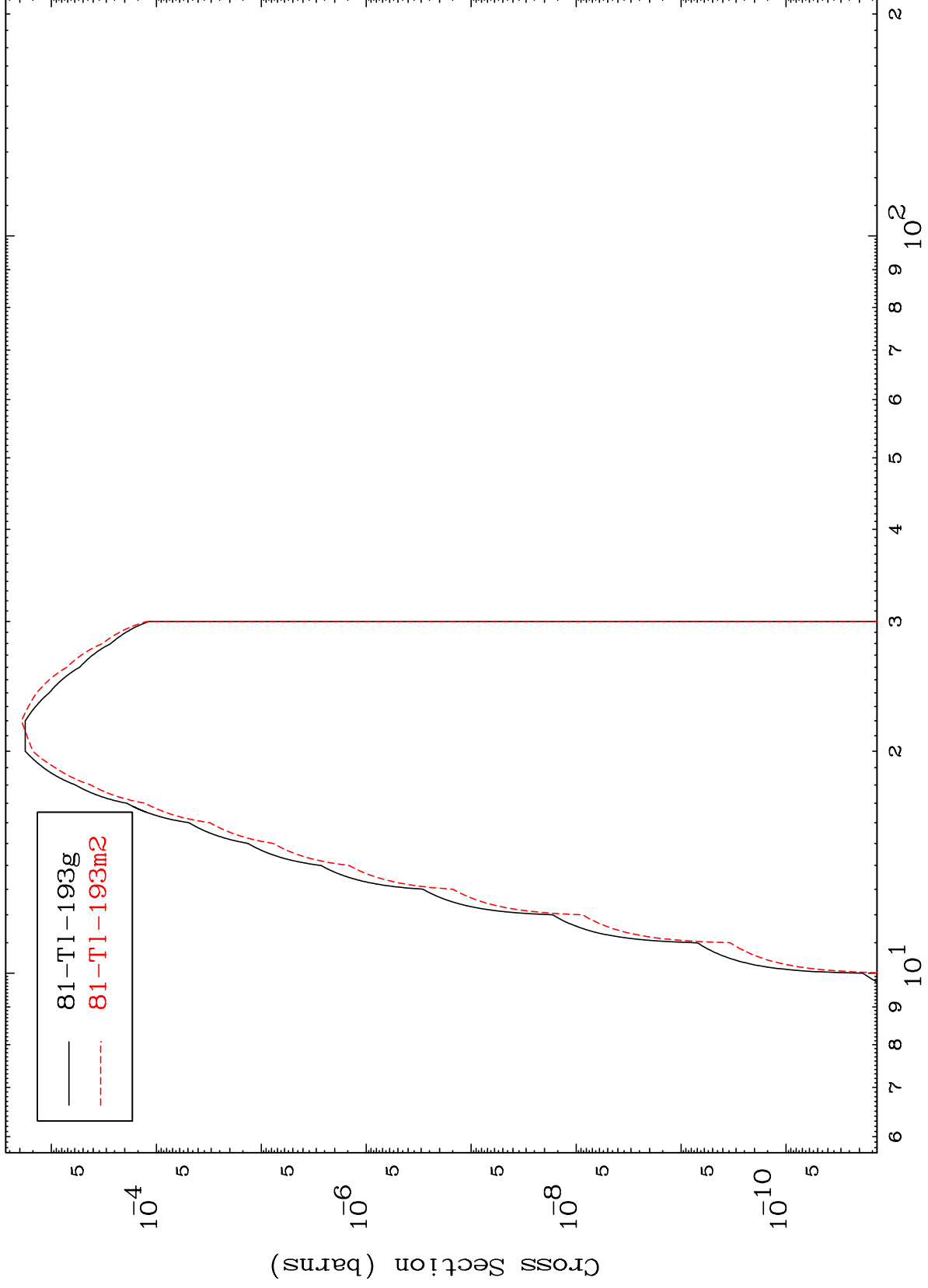


15

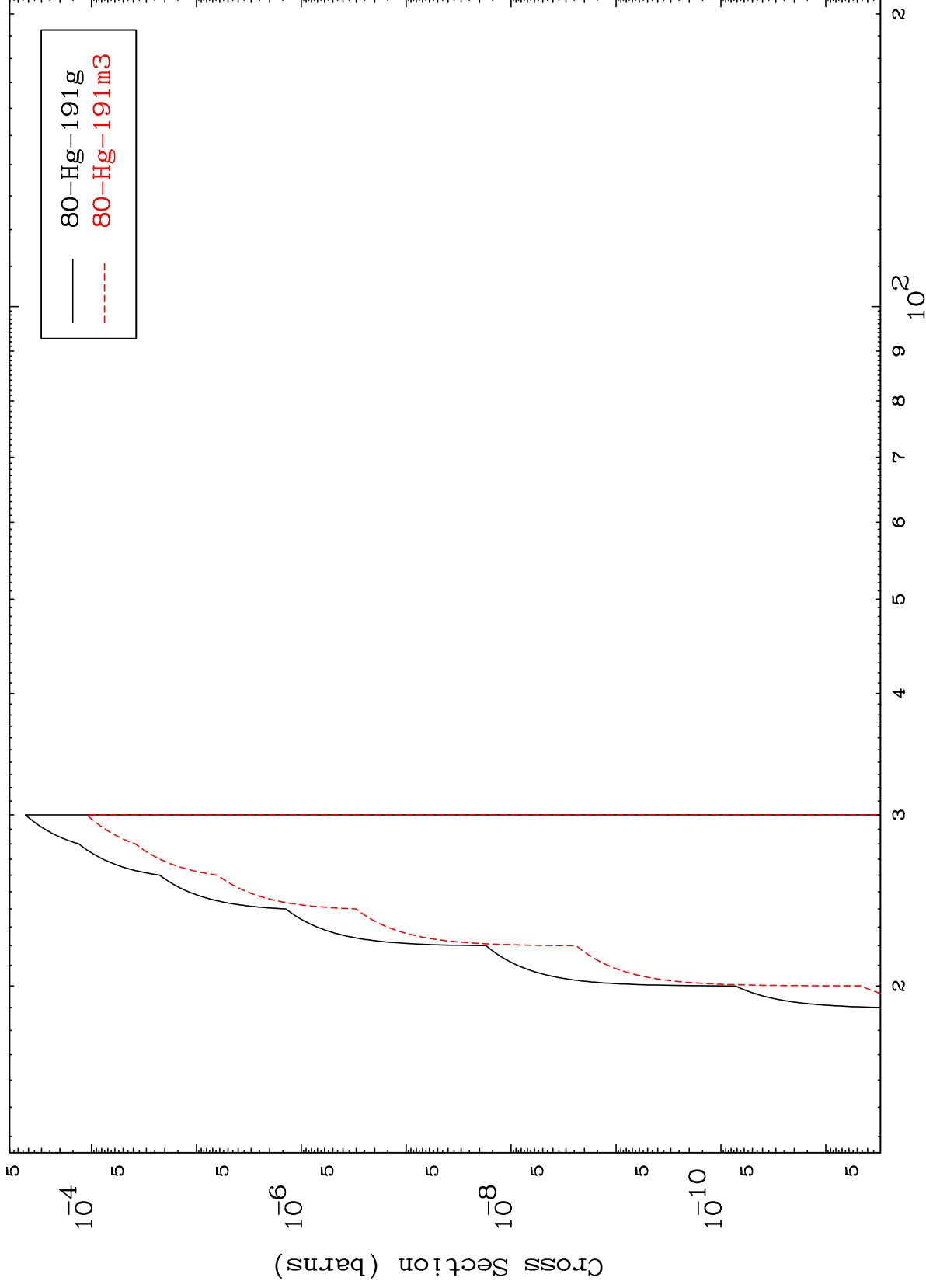
Incident Energy (MeV)

79-Au-189

(n,γ)
Radionuclide Production Cross Section



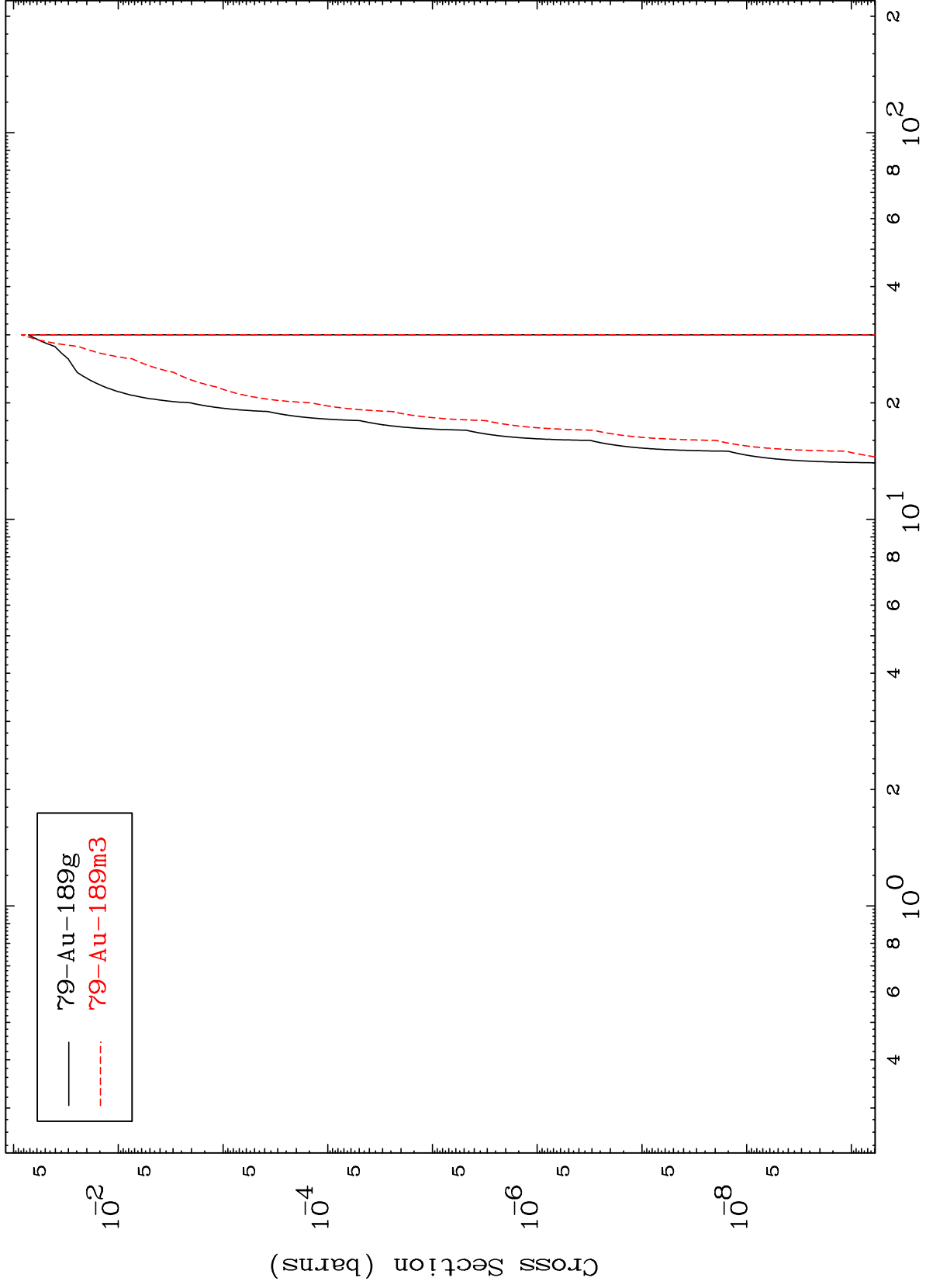
Radionuclide Production Cross Section



MAT 7901

⁷⁹Au-189

Radionuclide Production Cross Section



18

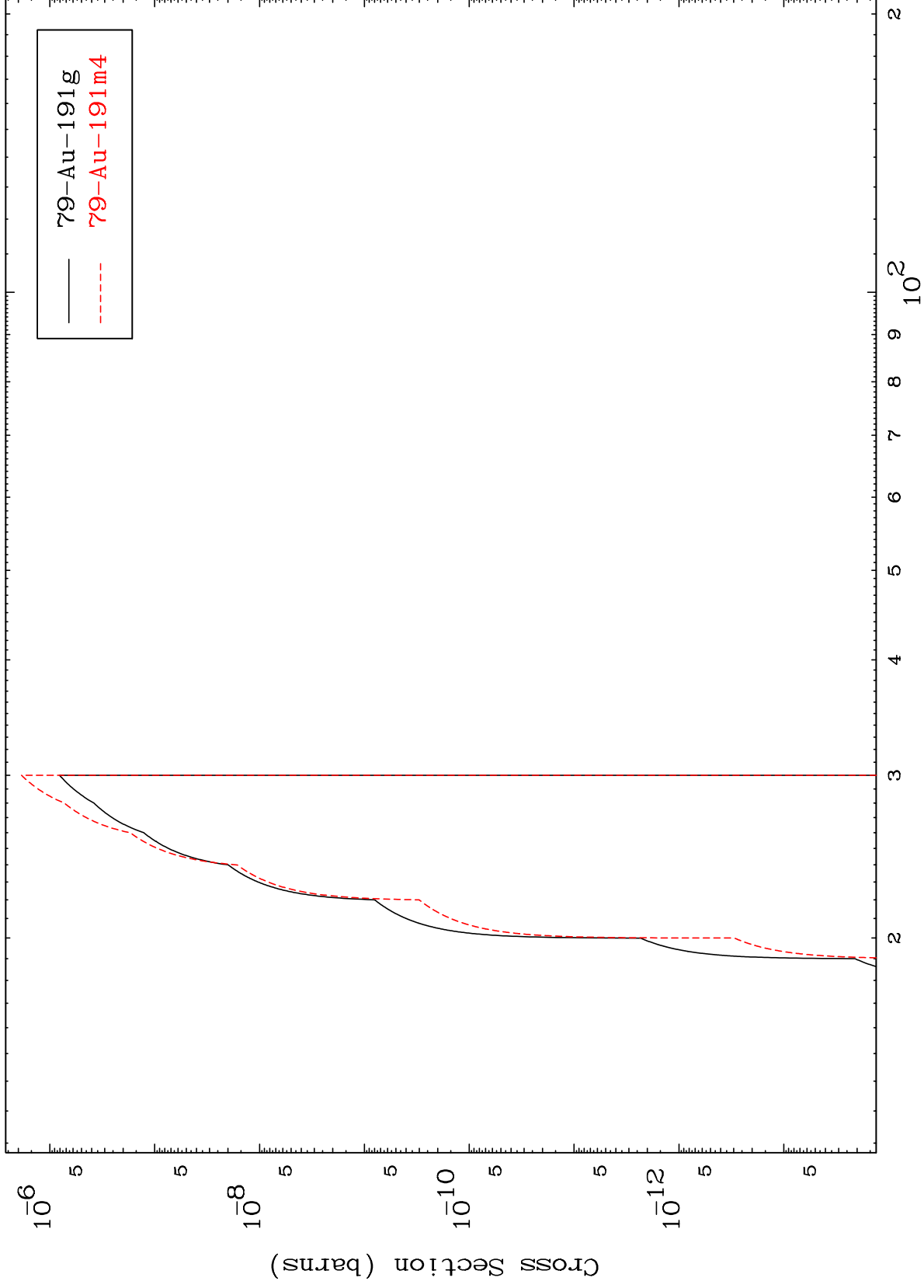
⁷⁹Au-189

Incident Energy (MeV)

MAT 7901

79-Au-189

(n,2p)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

79-Au-189