

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

Web:redcullen1.net/HOMEPAGE.NEW

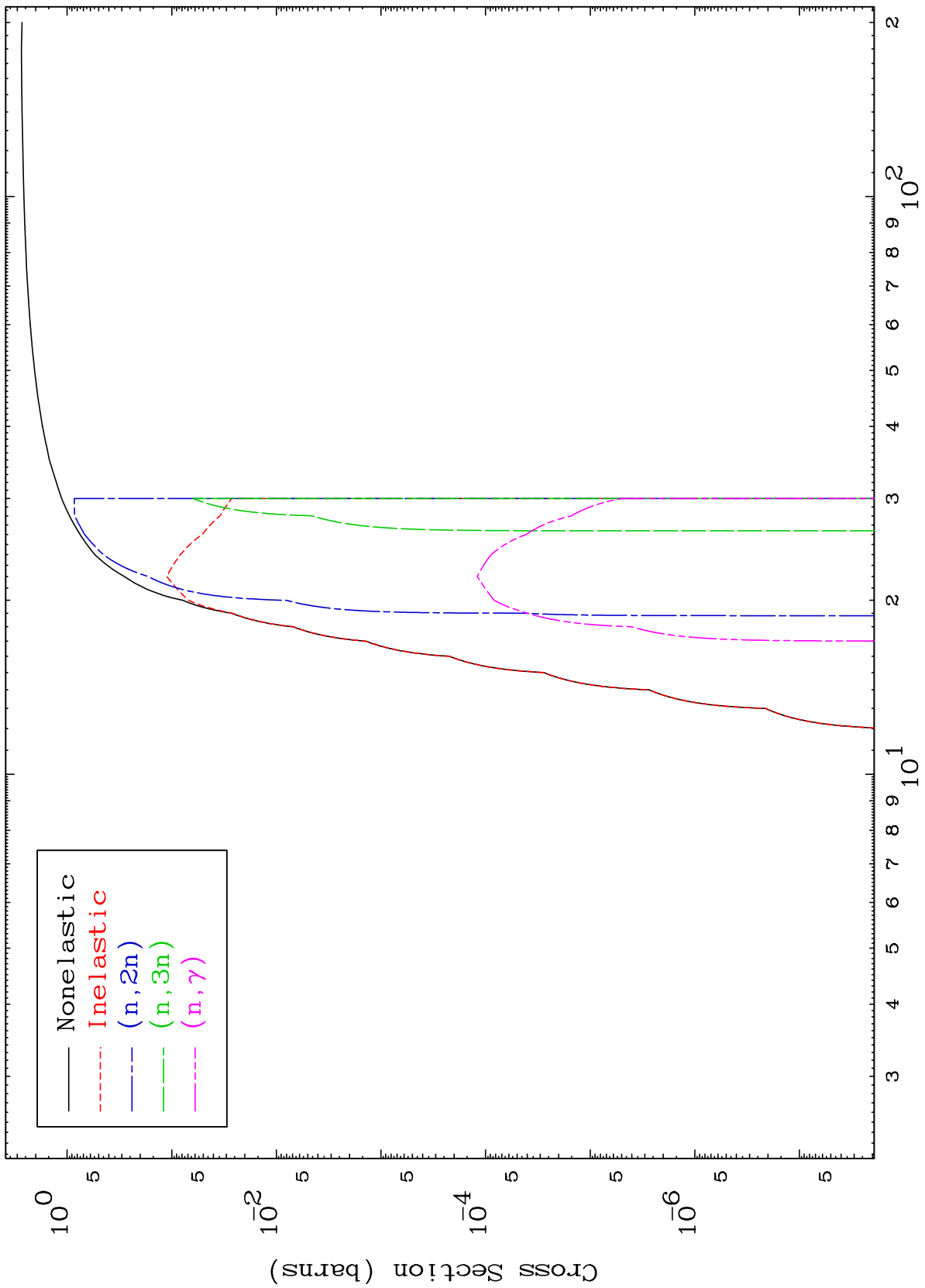
Press Mouse Button to Start

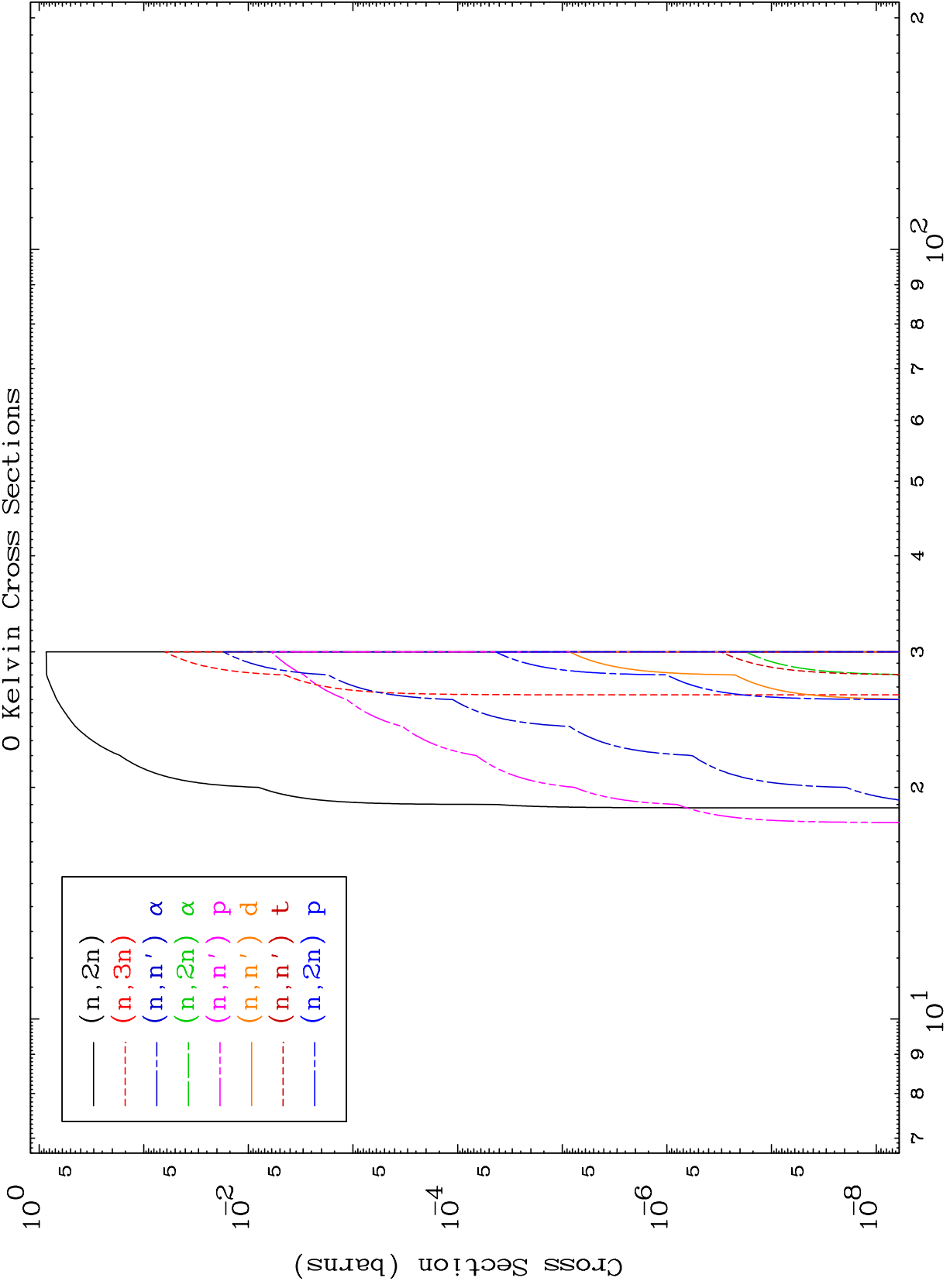
MAT 7916

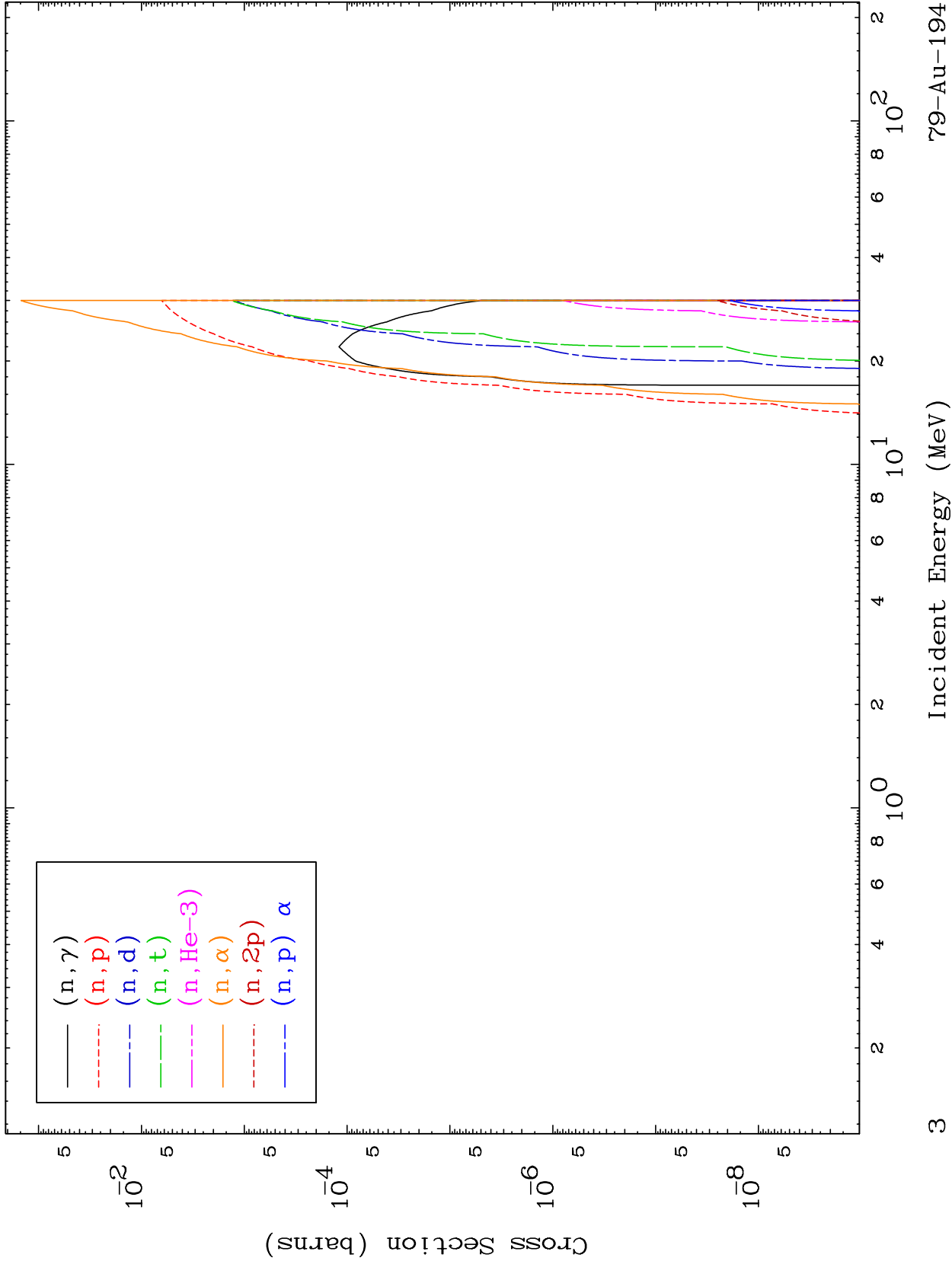
0 Kelvin

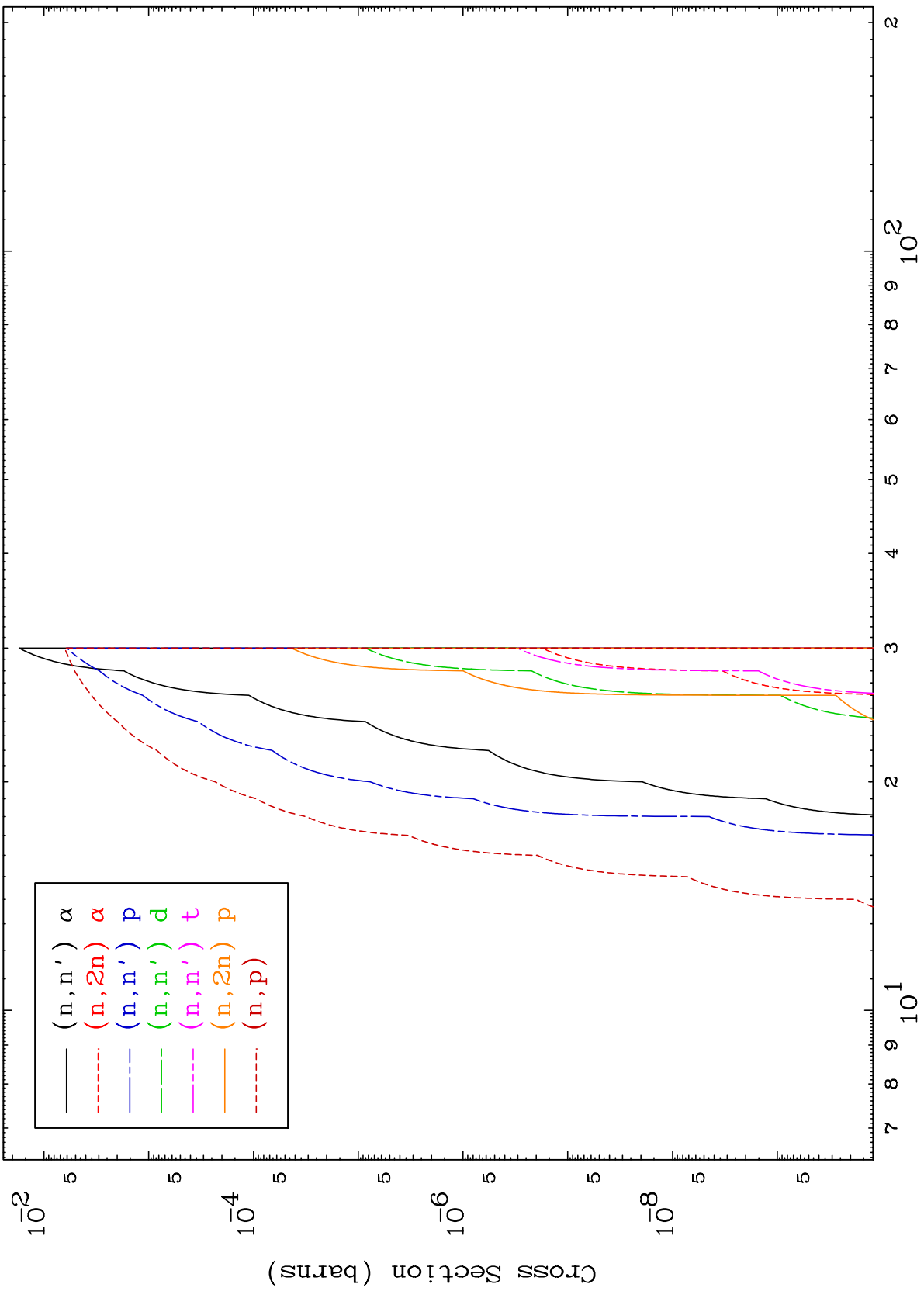
Major Cross Sections

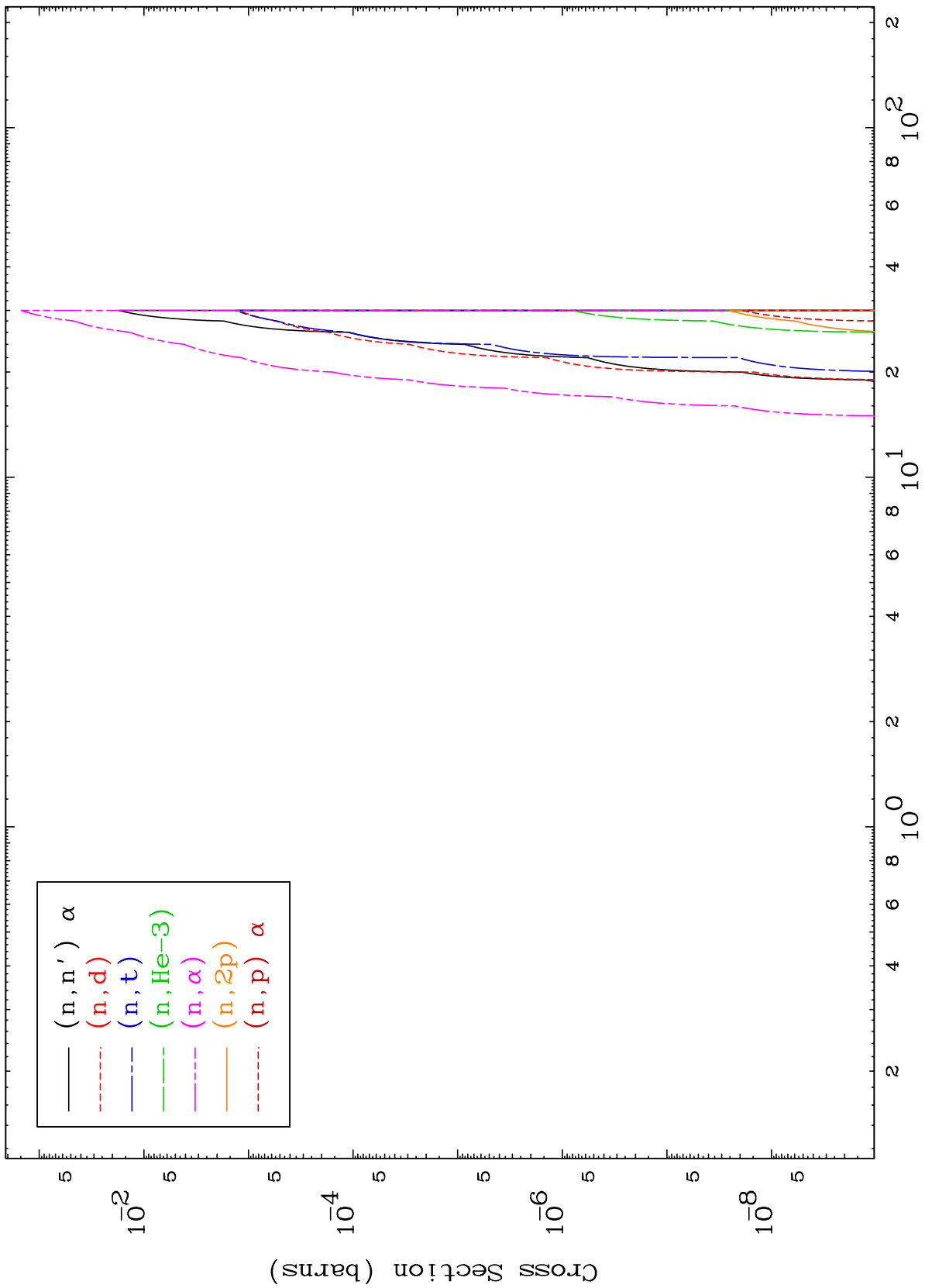
79-Au-194







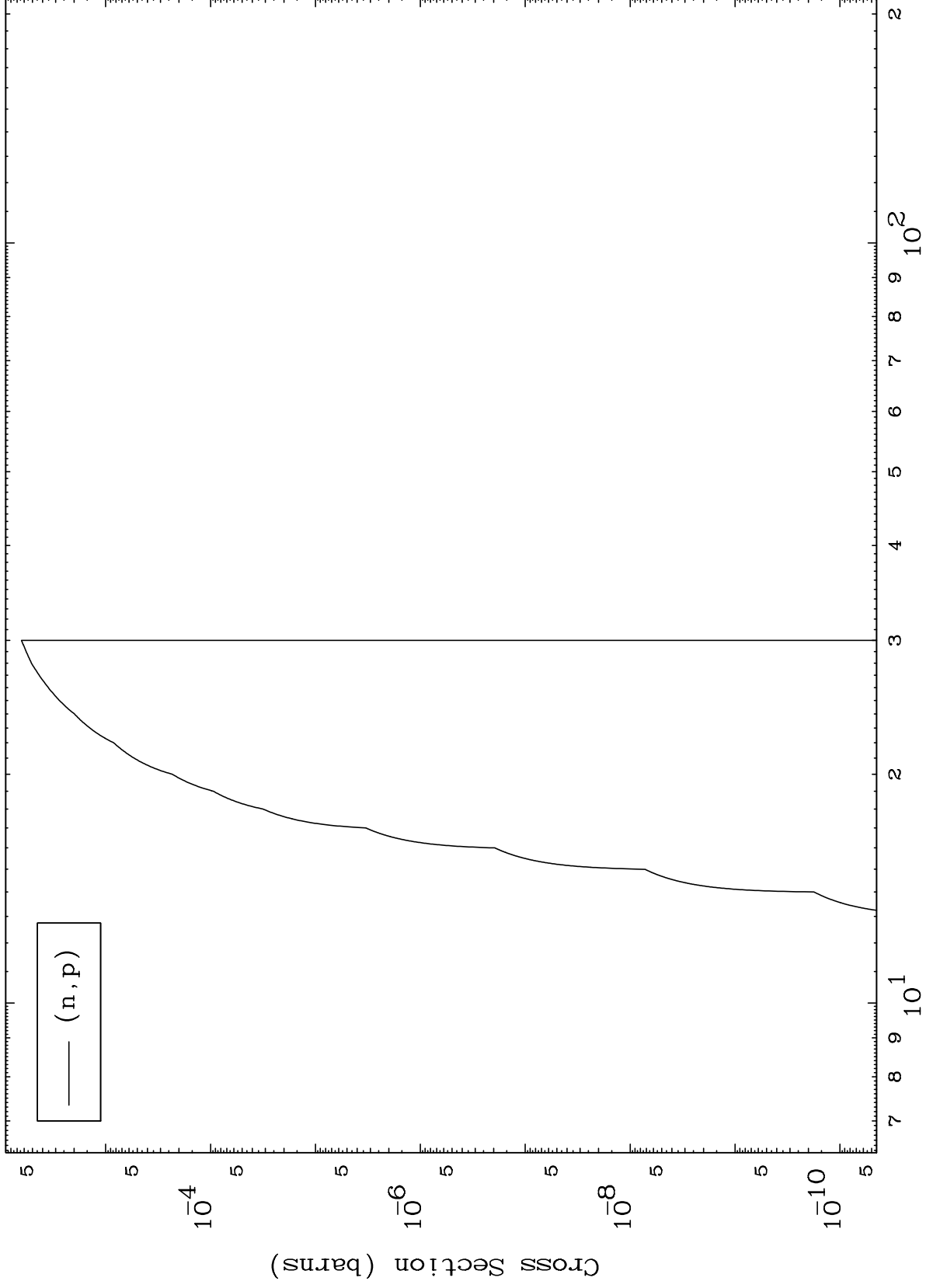




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(α, p) Levels
0 Kelvin Cross Sections

79-Au-194



6

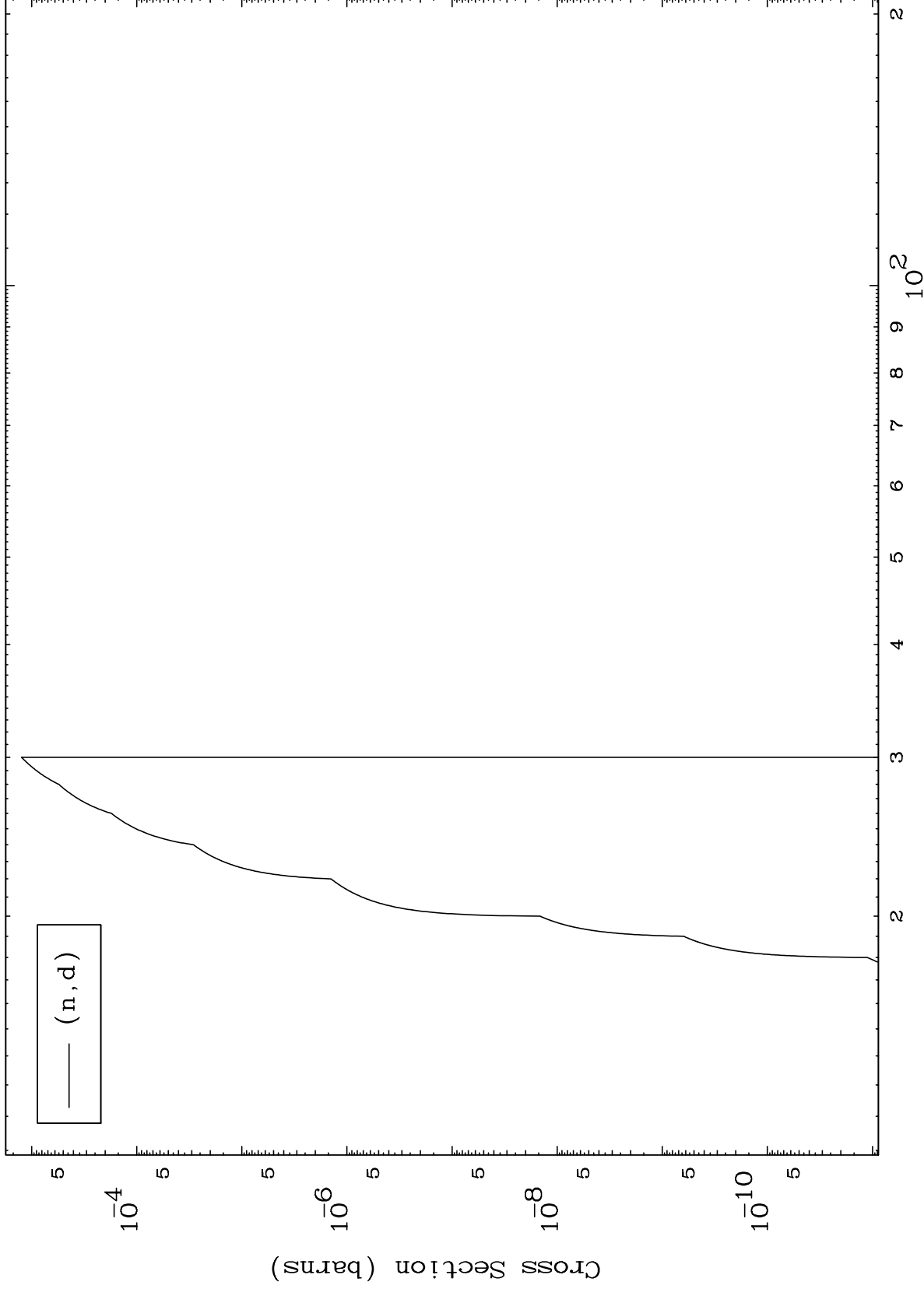
Incident Energy (MeV)

79-Au-194

MAT 7916

(α, d) Levels
0 Kelvin Cross Sections

79-Au-194

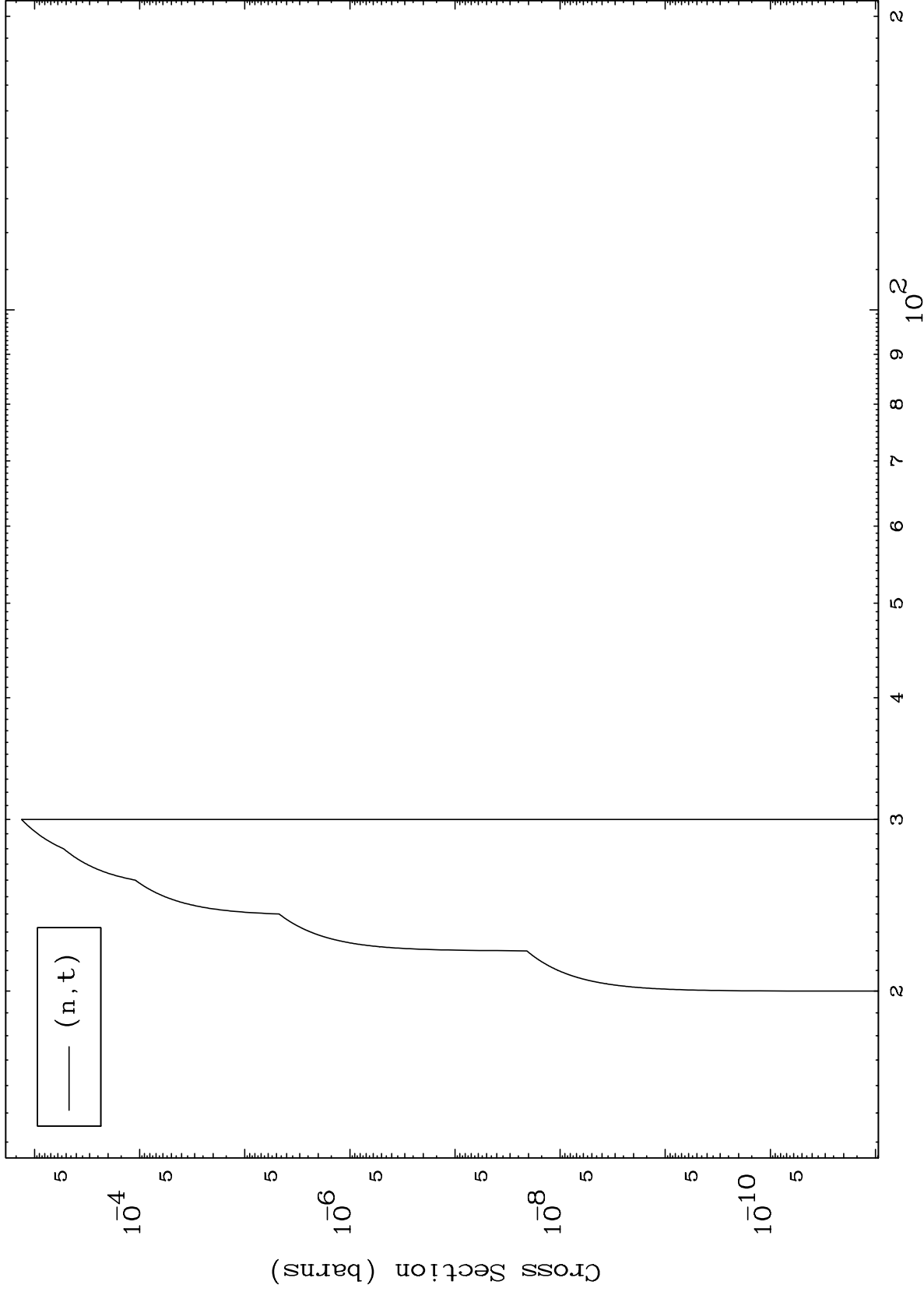


7

Incident Energy (MeV)

79-Au-194

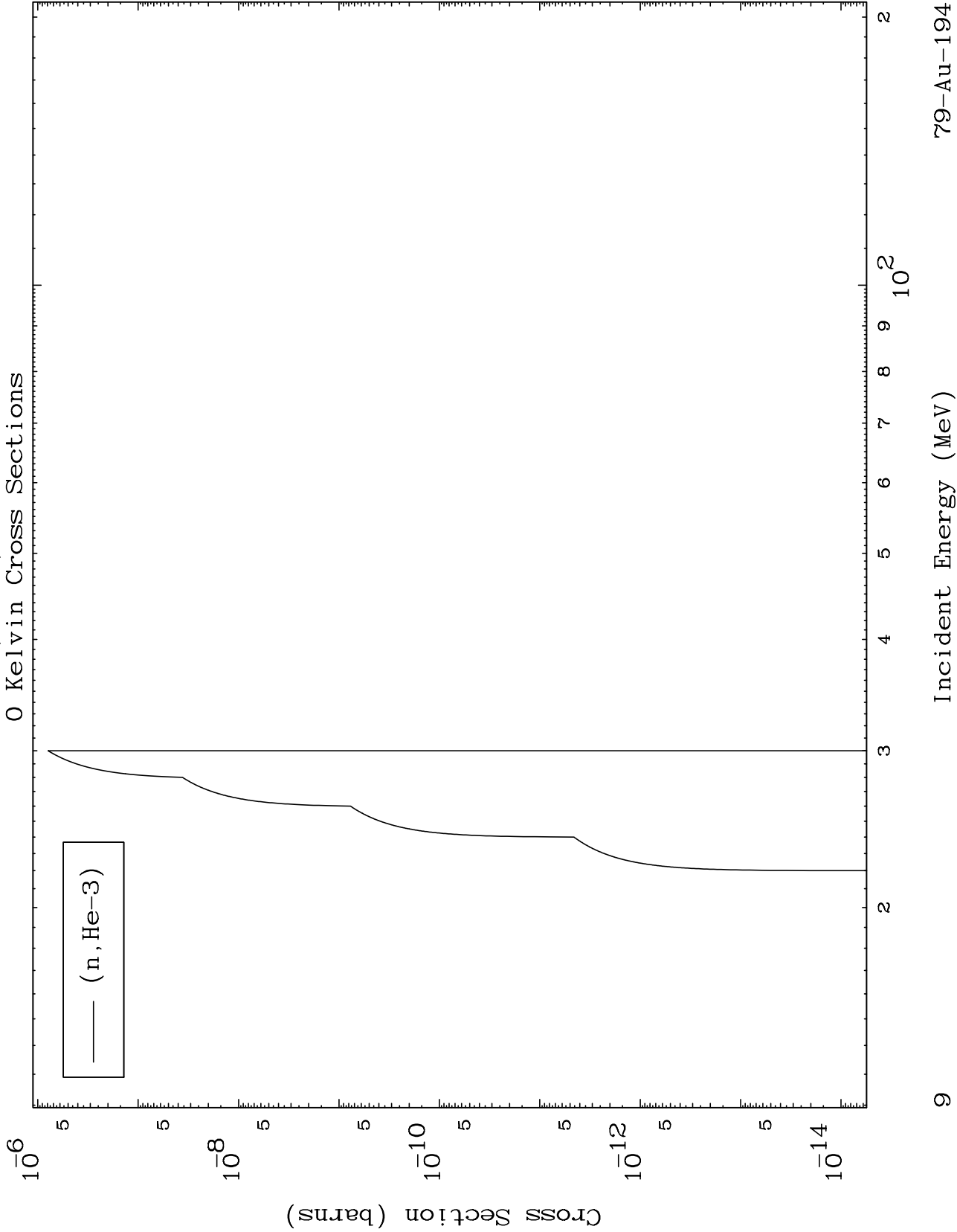
(α, t) Levels
0 Kelvin Cross Sections



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($\alpha, \text{He}3$) Levels

79-Au-194

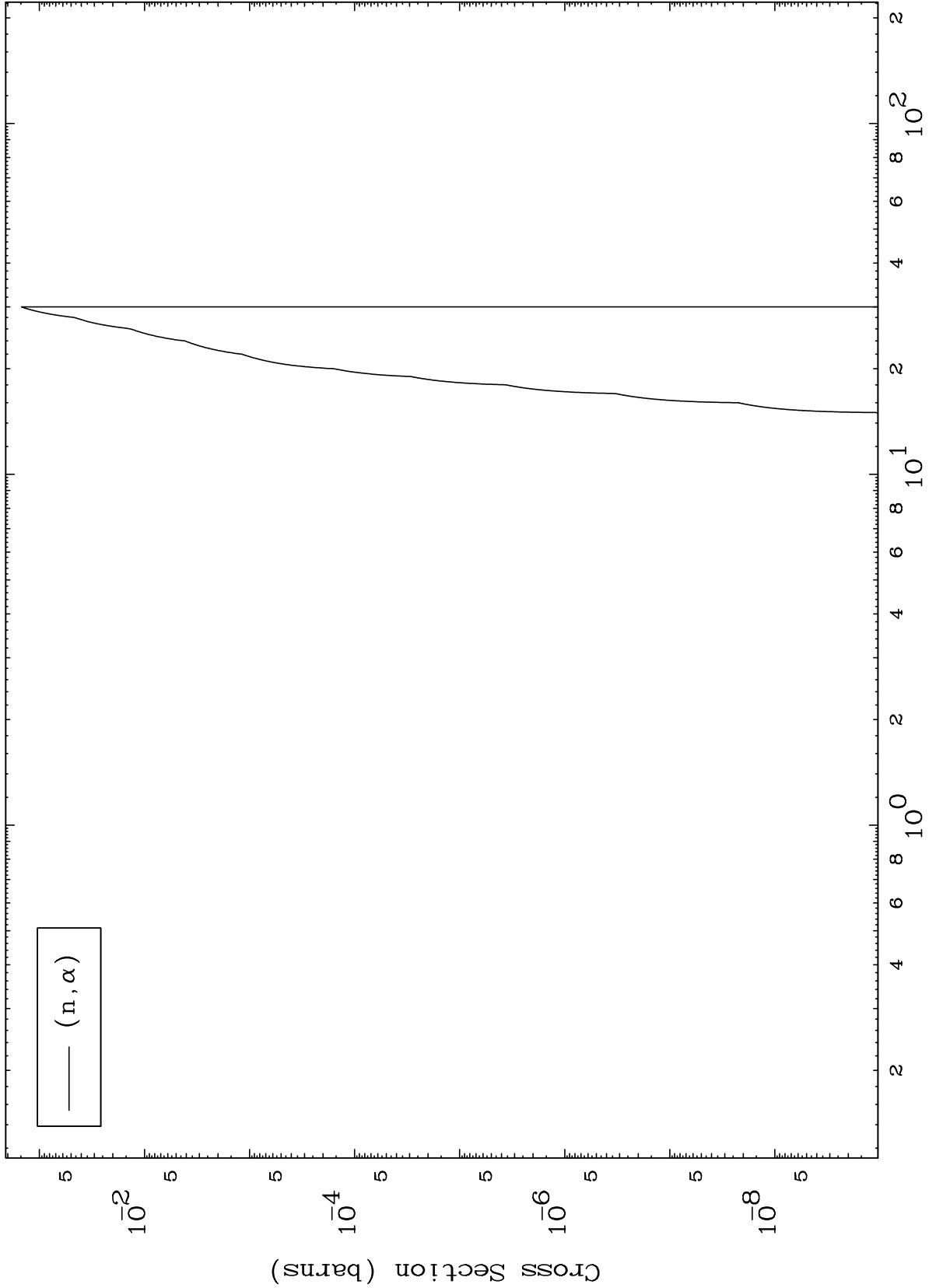


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(α, α) Levels

79-Au-194

0 Kelvin Cross Sections



10

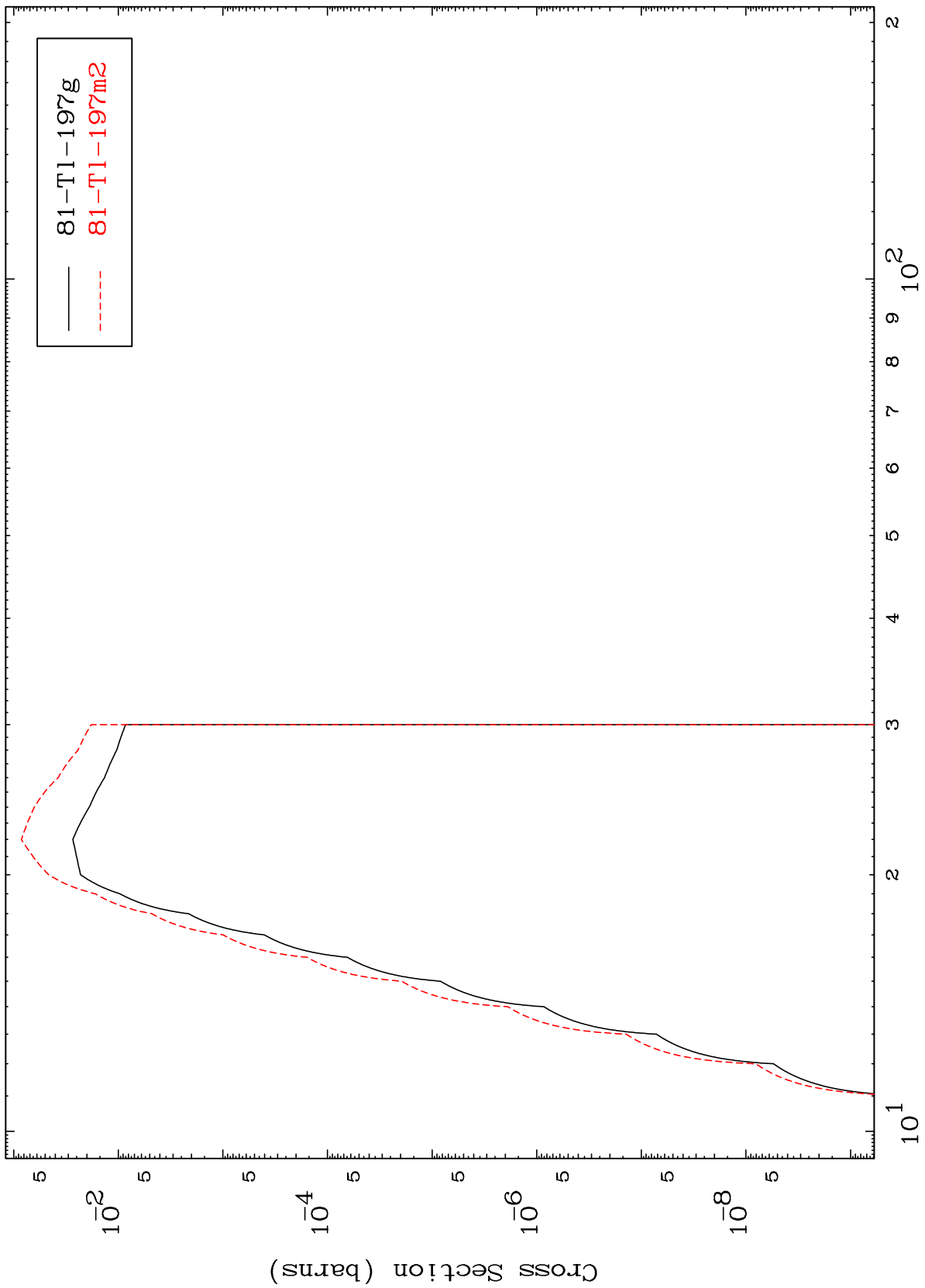
Incident Energy (MeV)

79-Au-194

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79-Au-194

Inelastic
Radionuclide Production Cross Section



79-Au-194

Incident Energy (MeV)

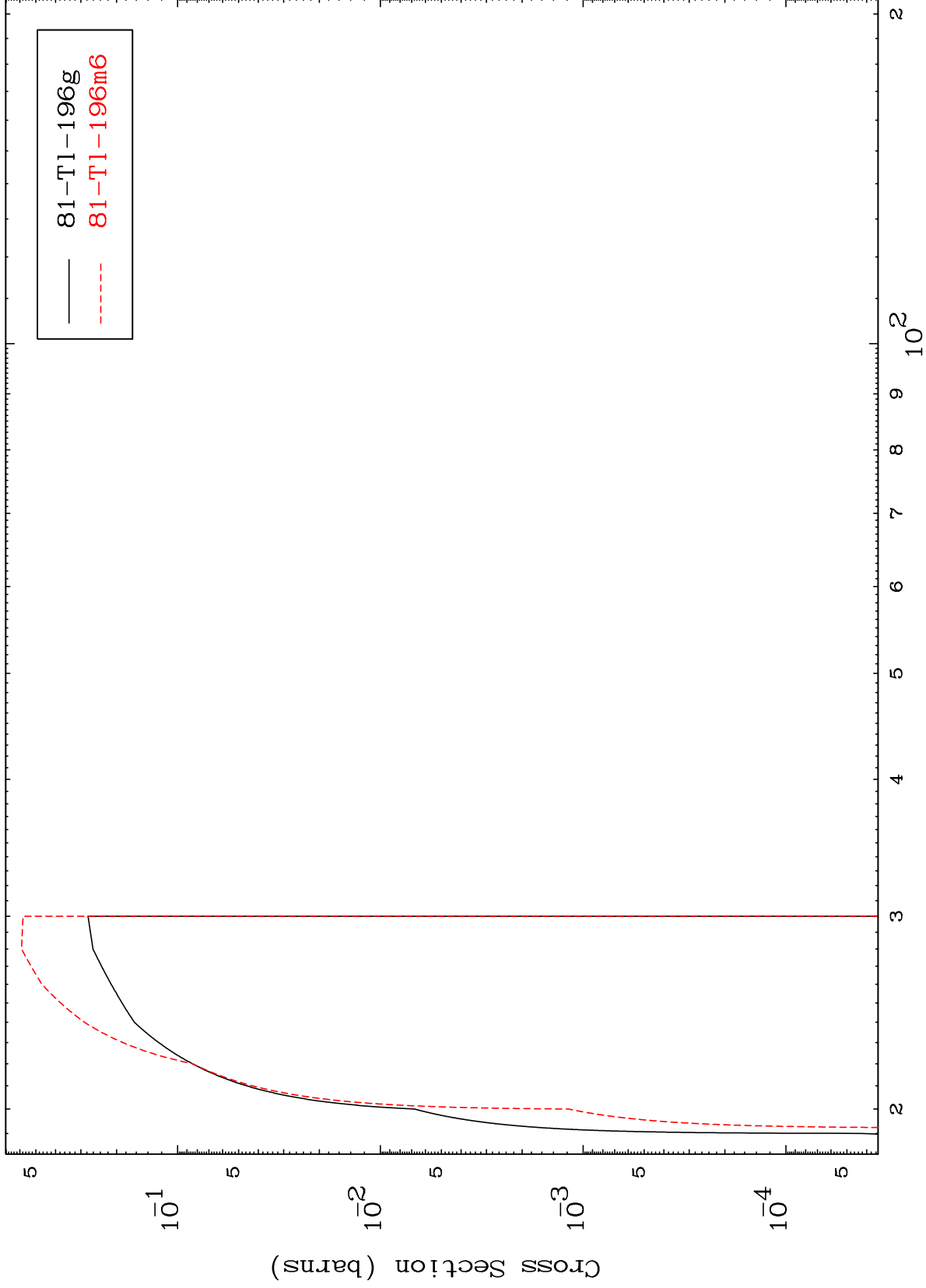
11

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(n,2n)

79-Au-194

Radionuclide Production Cross Section



12

Incident Energy (MeV)

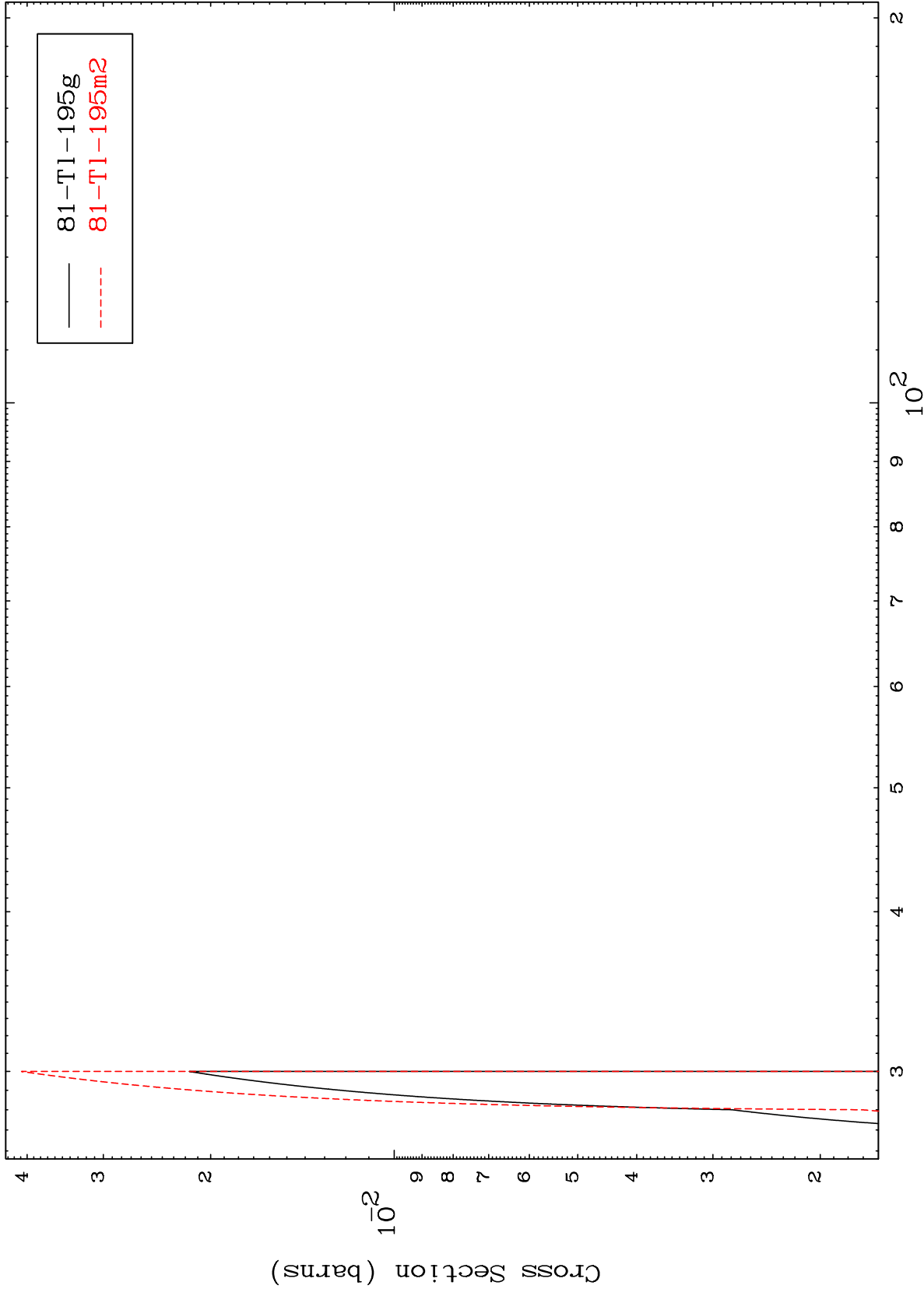
79-Au-194

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(n,3n)

79-Au-194

Radionuclide Production Cross Section



81-Tl-195g
81-Tl-195m2

13

Incident Energy (MeV)

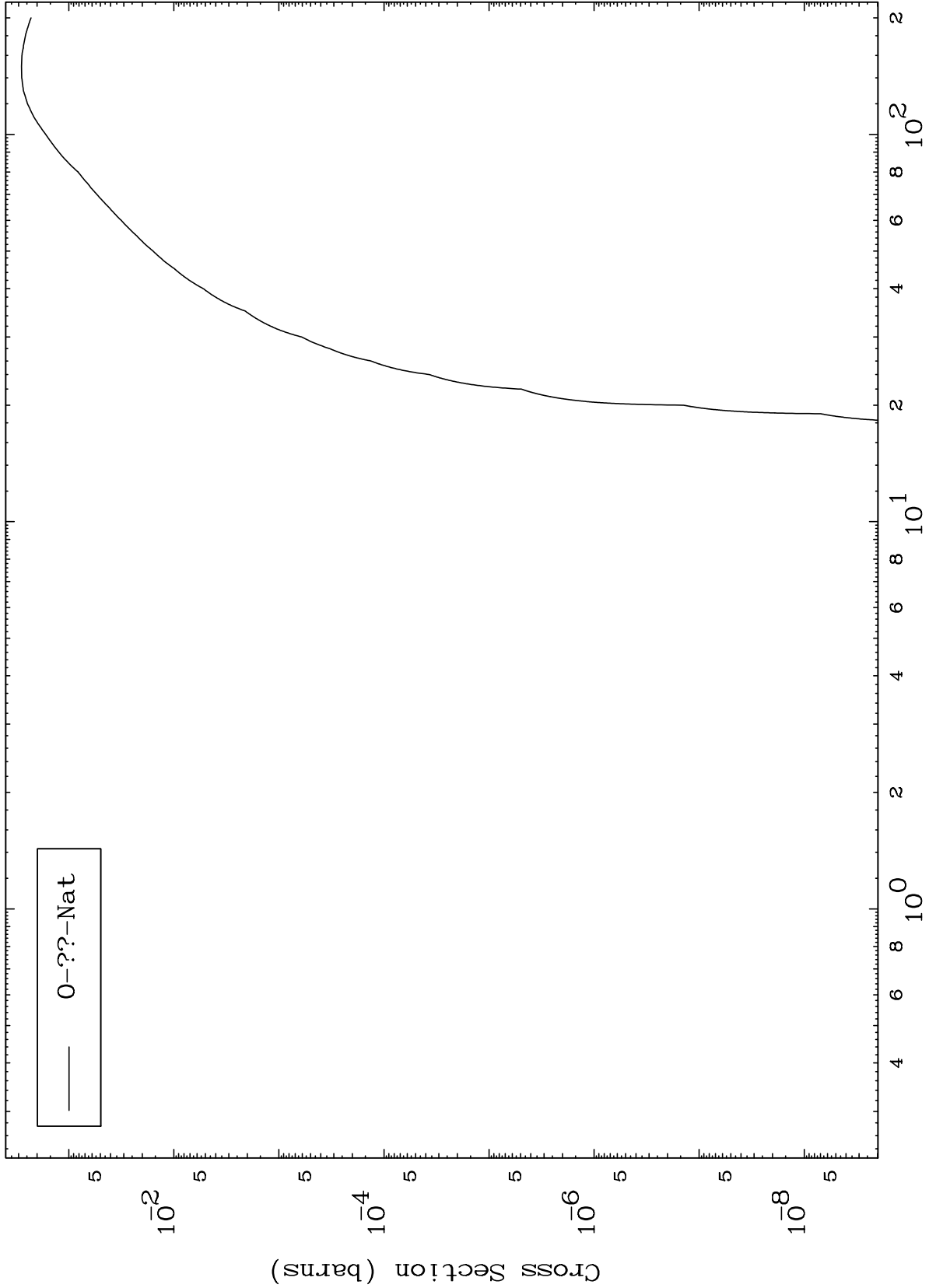
79-Au-194

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Fission

79-Au-194

Radionuclide Production Cross Section



14

Incident Energy (MeV)

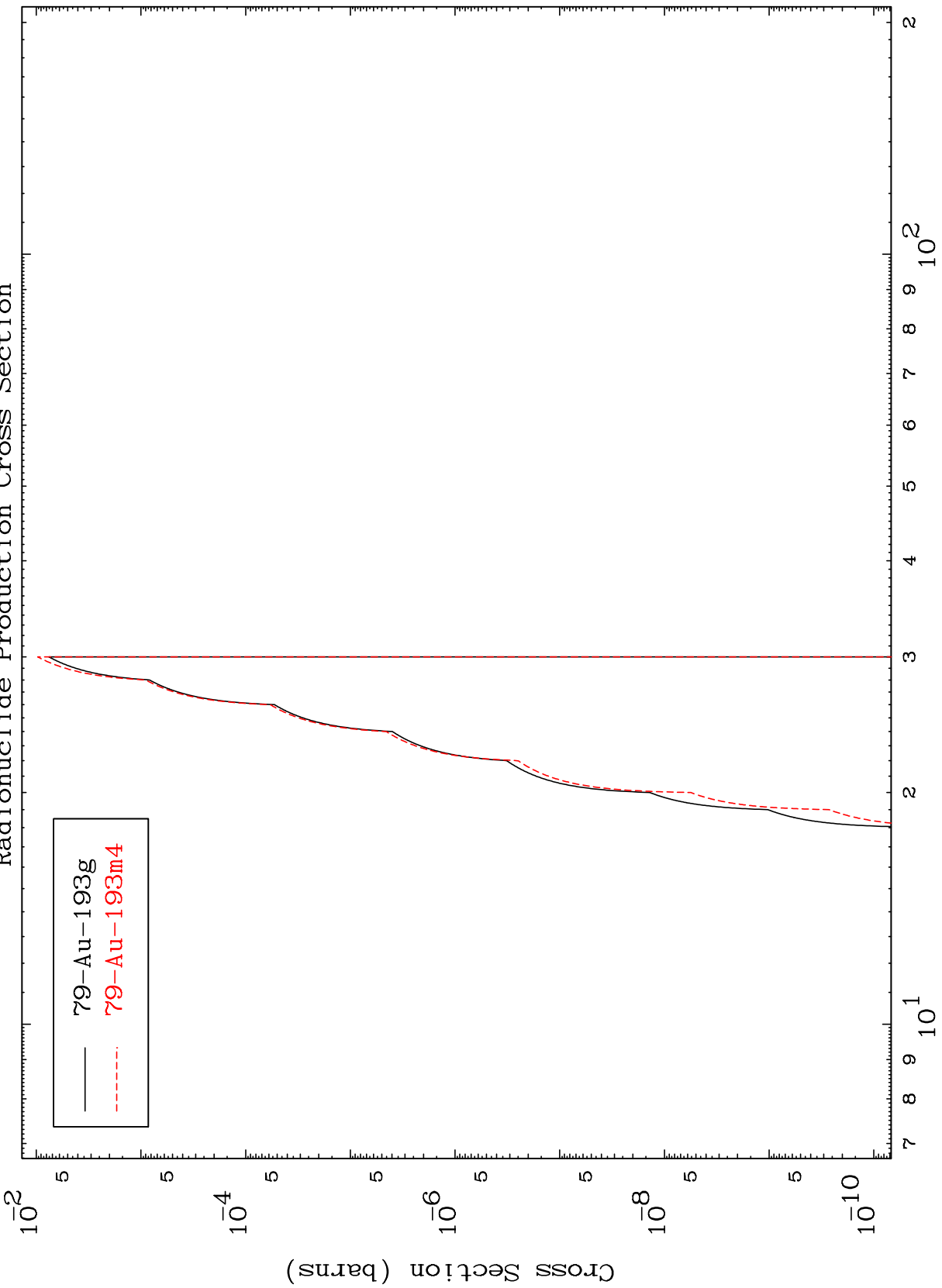
79-Au-194

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$(n, n') \alpha$

79-Au-194

Radionuclide Production Cross Section



79-Au-193g
79-Au-193m4

15

Incident Energy (MeV)

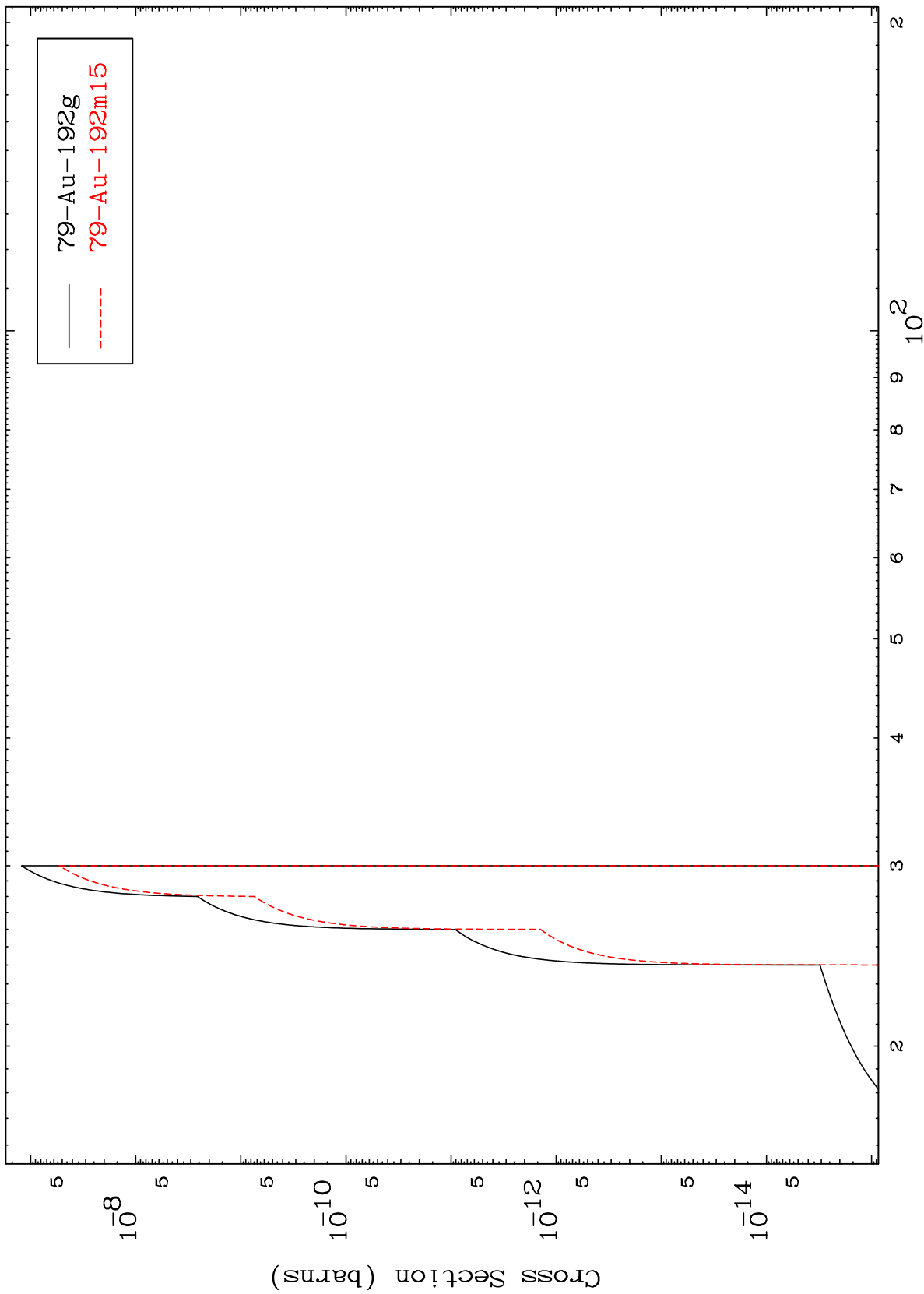
79-Au-194

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$(n,2n) \alpha$

$^{79}\text{Au-194}$

Radionuclide Production Cross Section



16

Incident Energy (MeV)

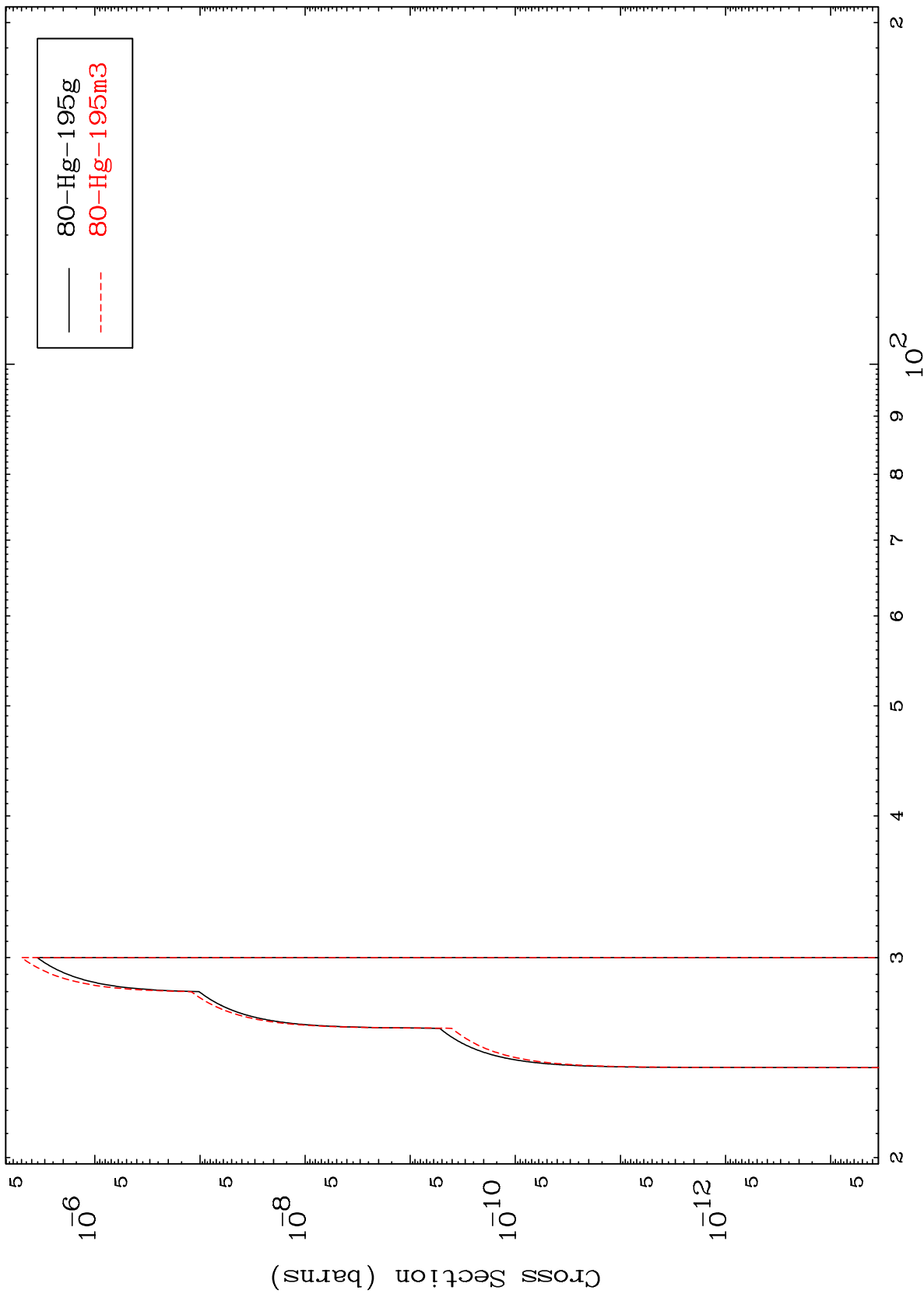
$^{79}\text{Au-194}$

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(n,n') d

79-Au-194

Radionuclide Production Cross Section

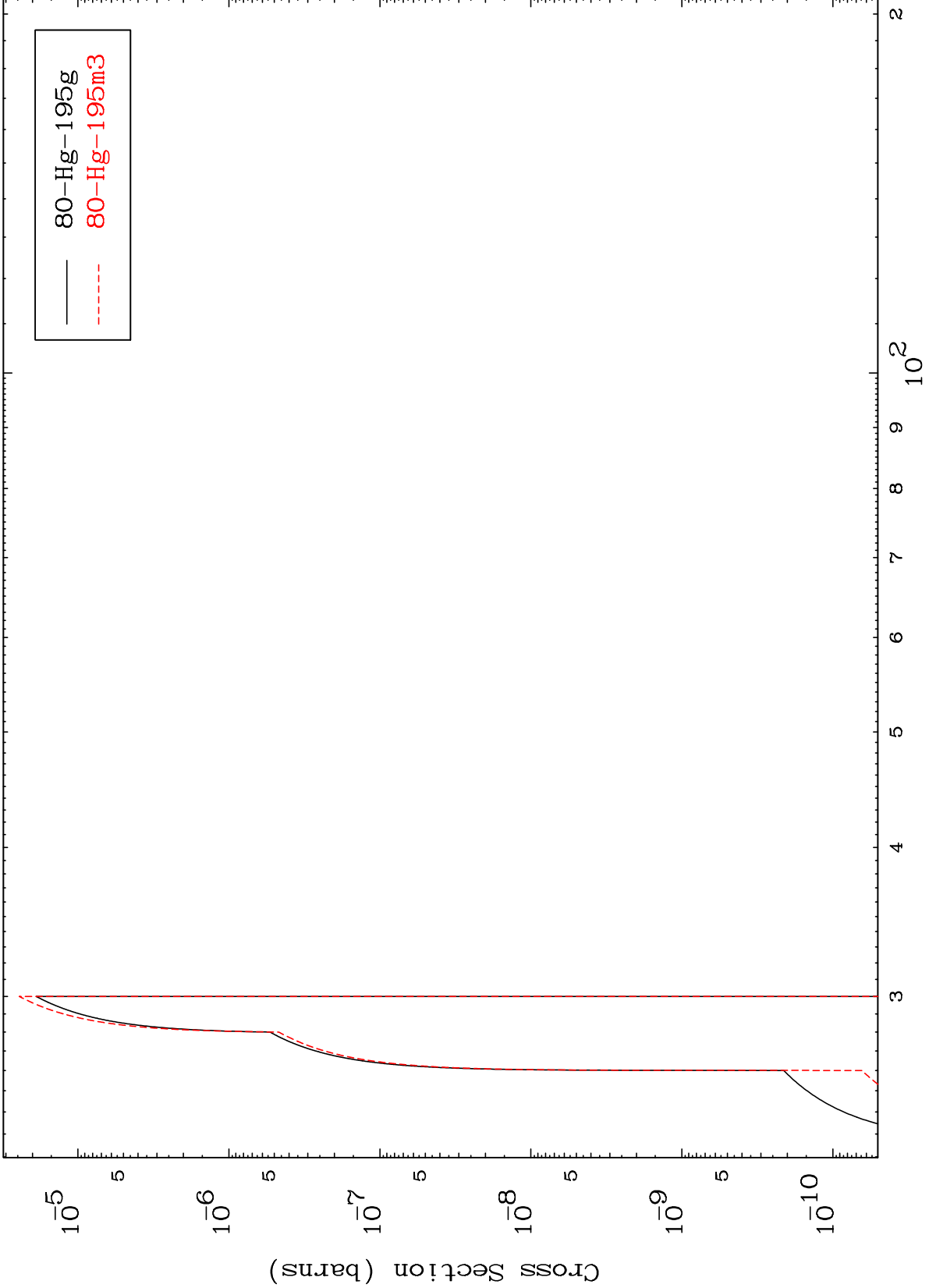


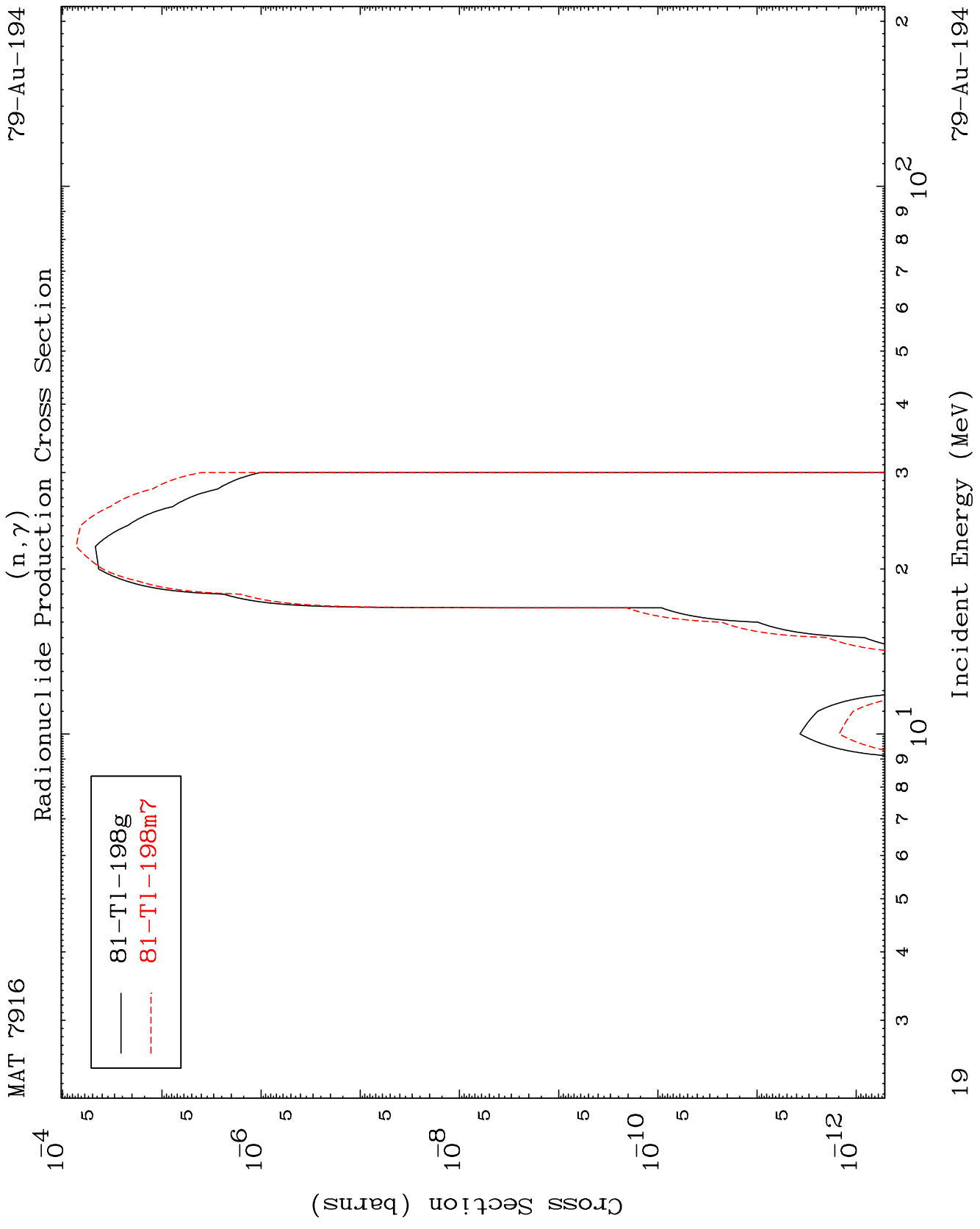
17

Incident Energy (MeV)

79-Au-194

Radionuclide Production Cross Section

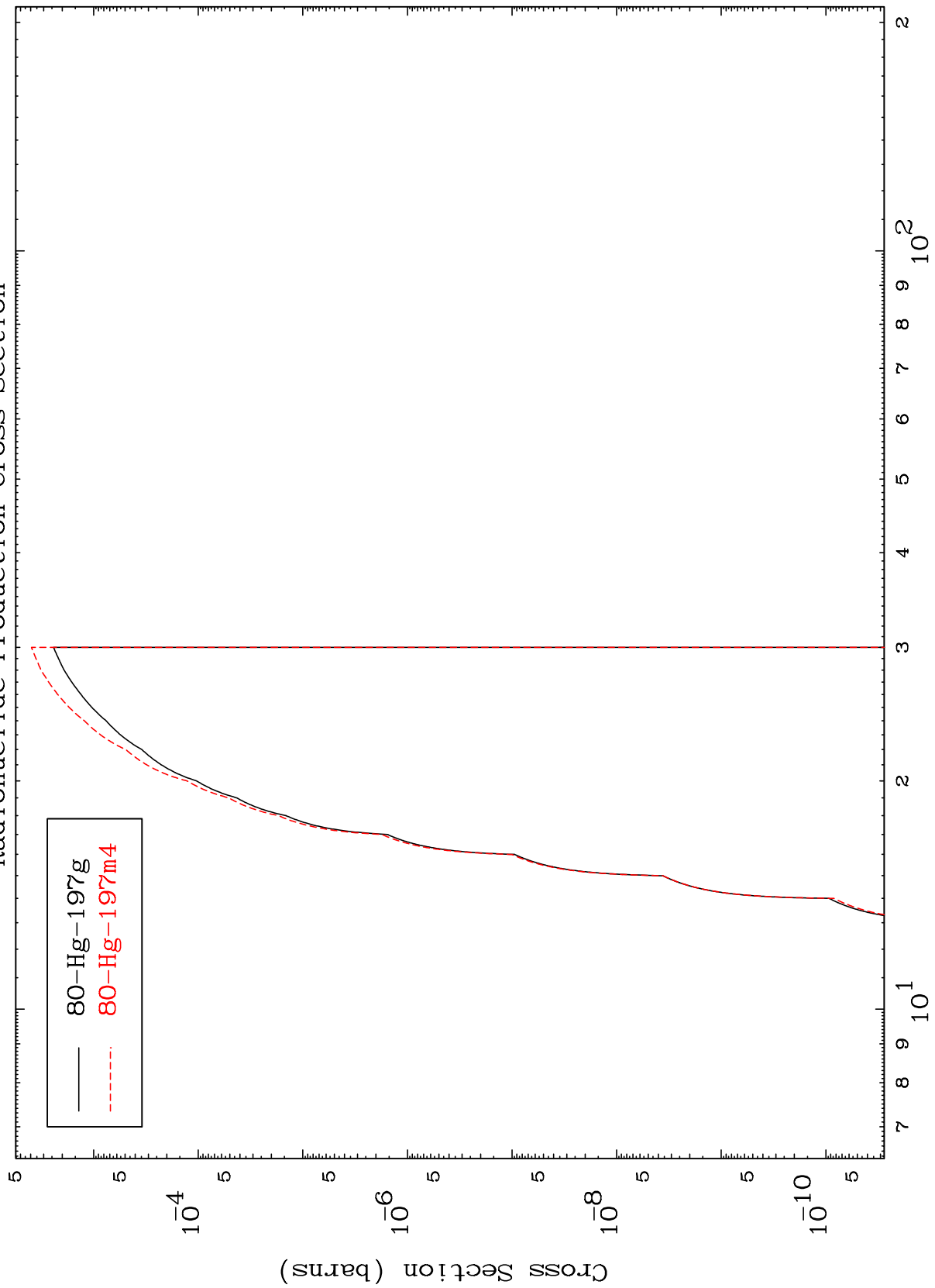




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79-Au-194

(n,p)
Radionuclide Production Cross Section

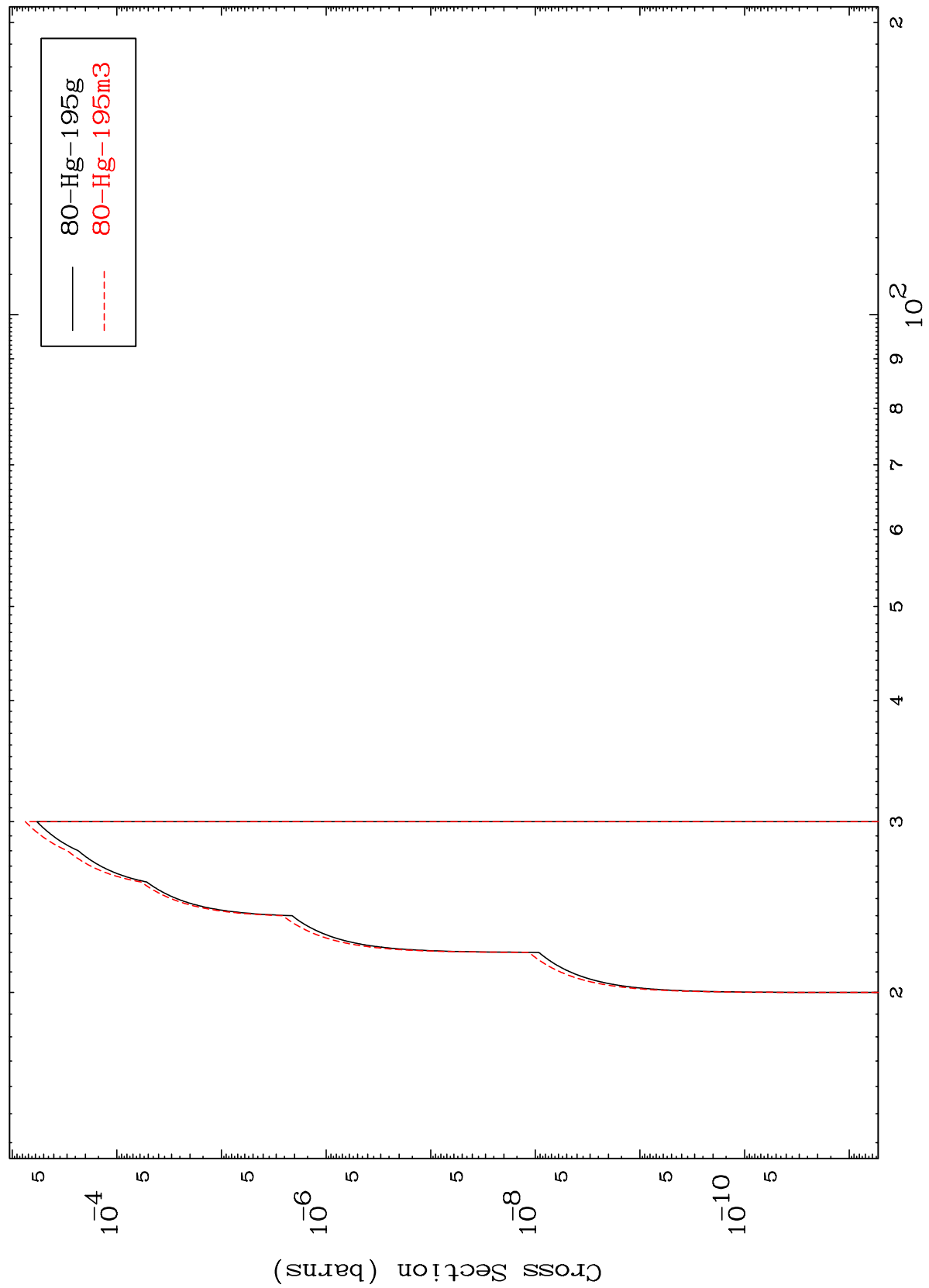


20

Incident Energy (MeV)

79-Au-194

(n, t)
Radionuclide Production Cross Section

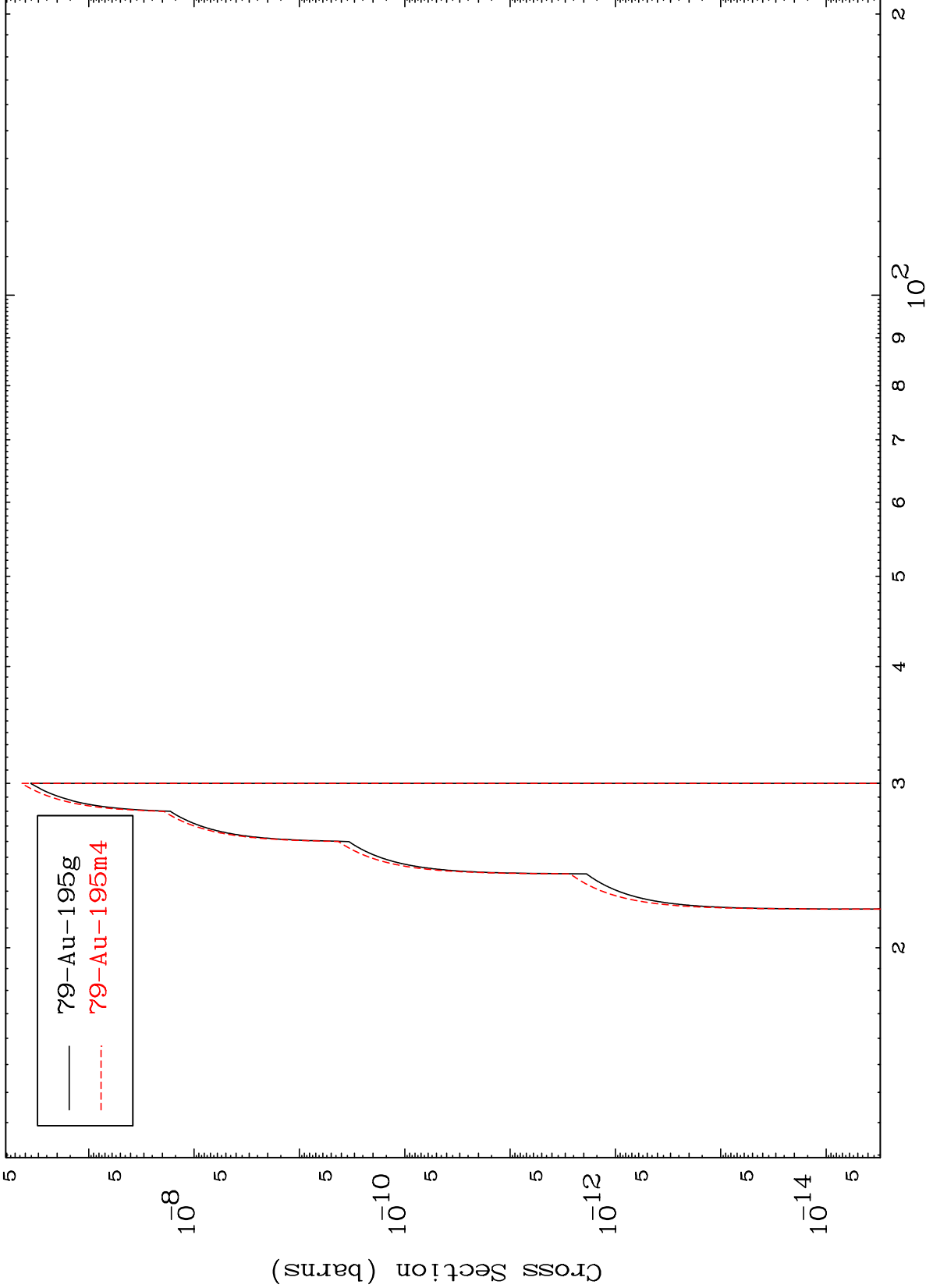


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(n,He-3)

⁷⁹Au-194

Radionuclide Production Cross Section



22

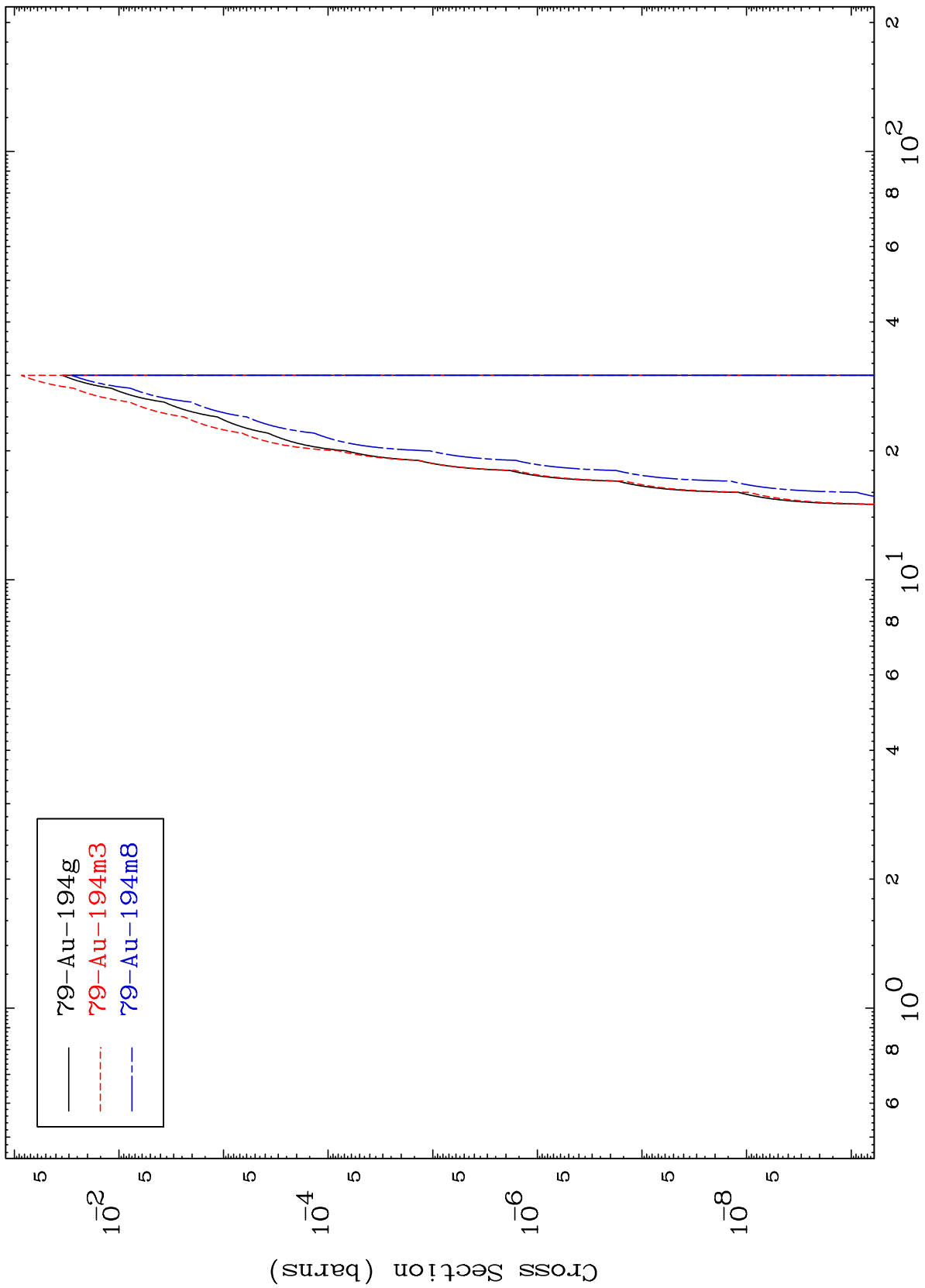
Incident Energy (MeV)

⁷⁹Au-194

MAT 7916

79-Au-194

Radionuclide Production Cross Section
(n, α)



23

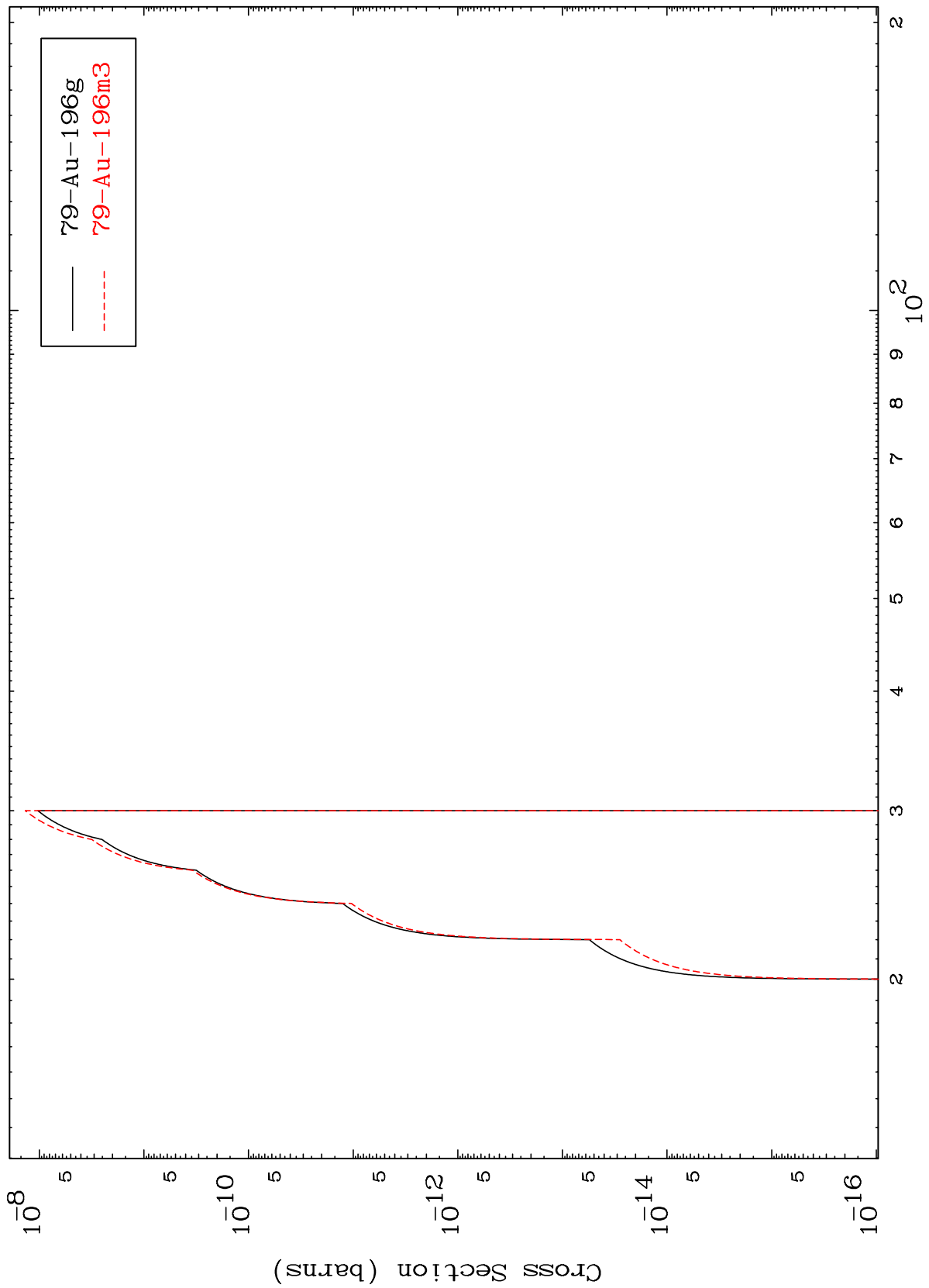
79-Au-194

Incident Energy (MeV)

MAT 7916

79-Au-194

(n,2p)
Radionuclide Production Cross Section

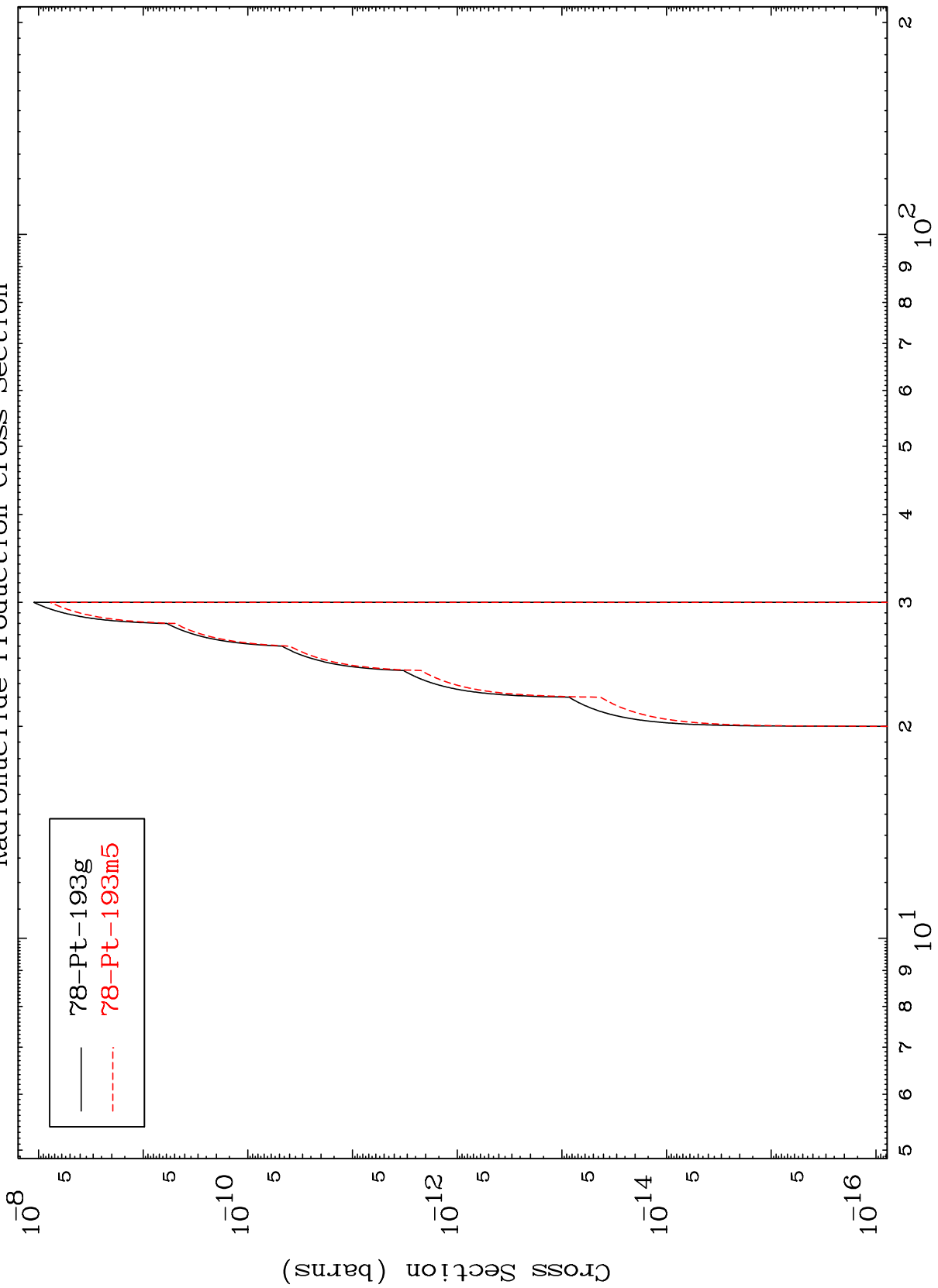


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$(n,p) \alpha$

$^{79}\text{Au-194}$

Radionuclide Production Cross Section



25

Incident Energy (MeV)

$^{79}\text{Au-194}$