

Program EVALPLOT
(Version 2021-1)

by

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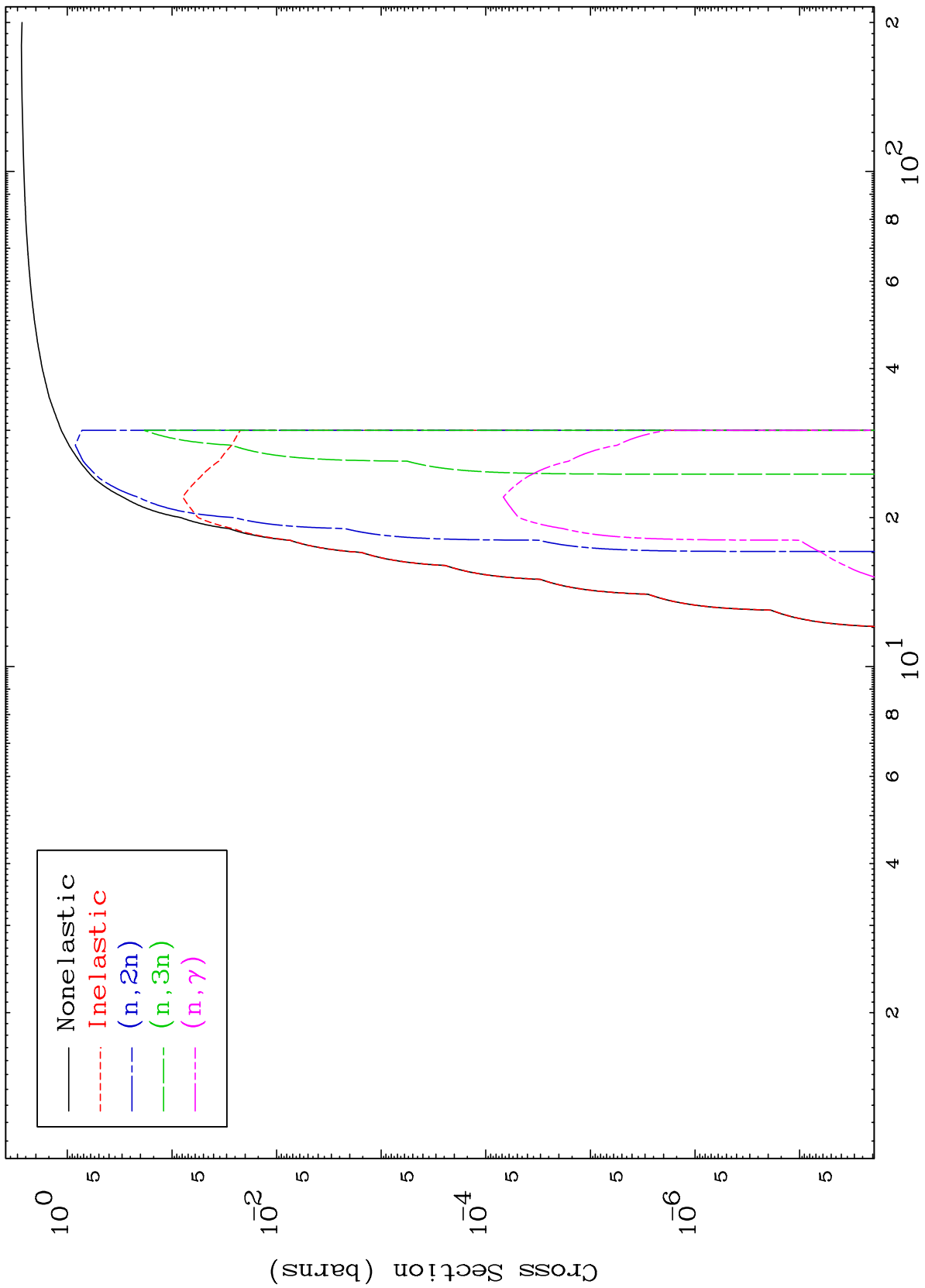
Press Mouse Button to Start

MAT 7924

α Major

79-Au-196n

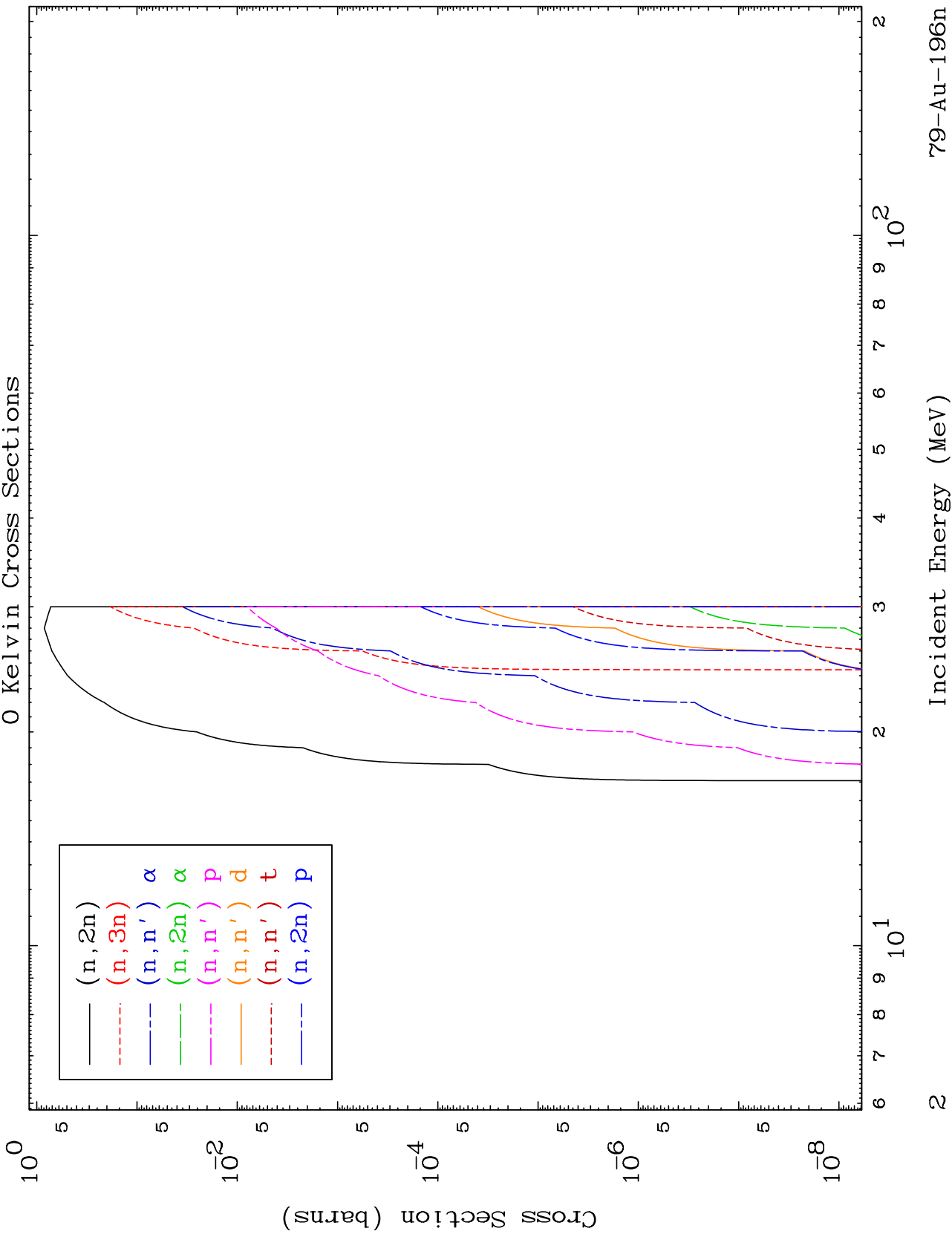
0 Kelvin Cross Sections



MAT 7924

α Neutron Absorption
0 Kelvin Cross Sections

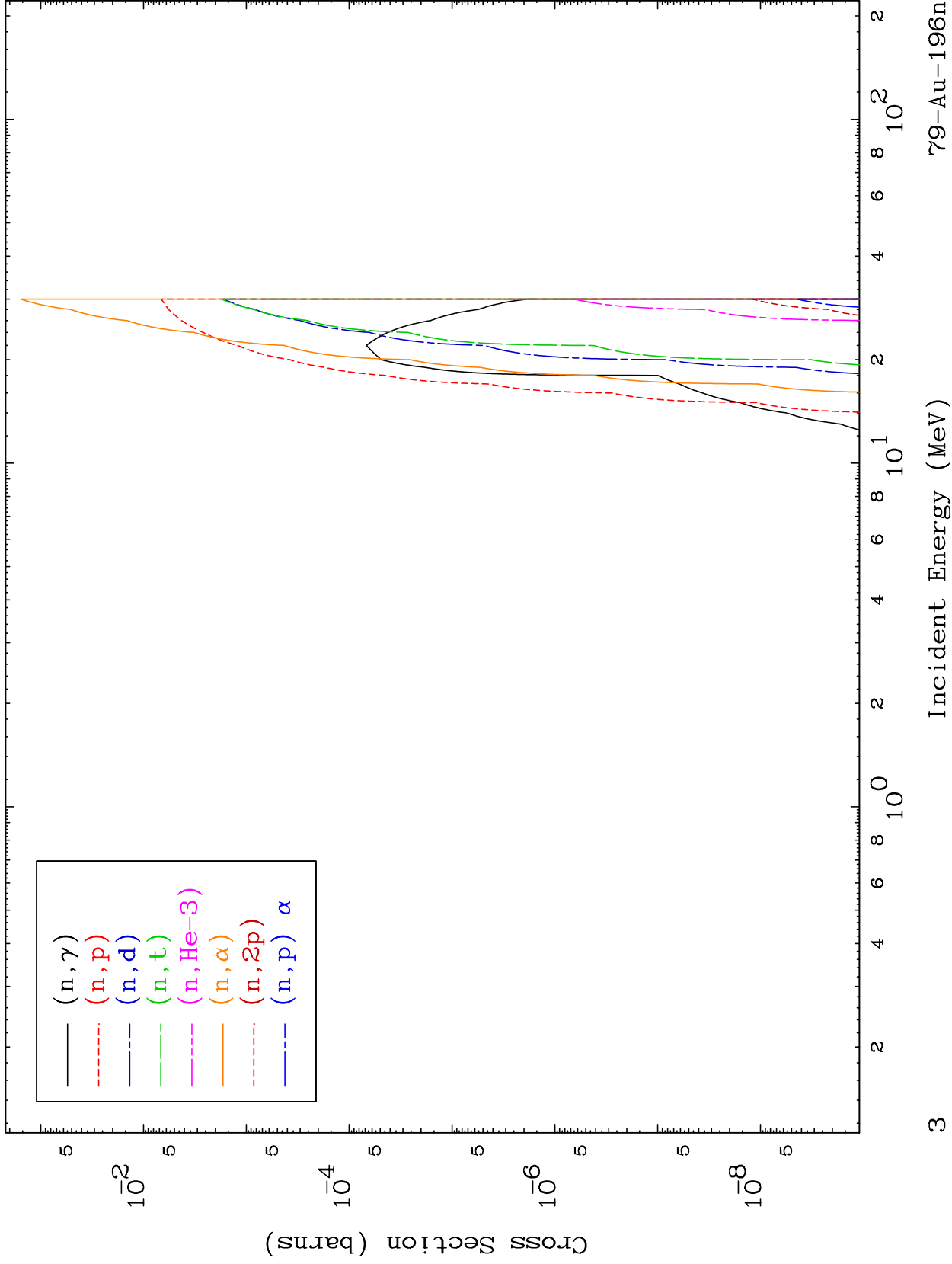
79-Au-196n

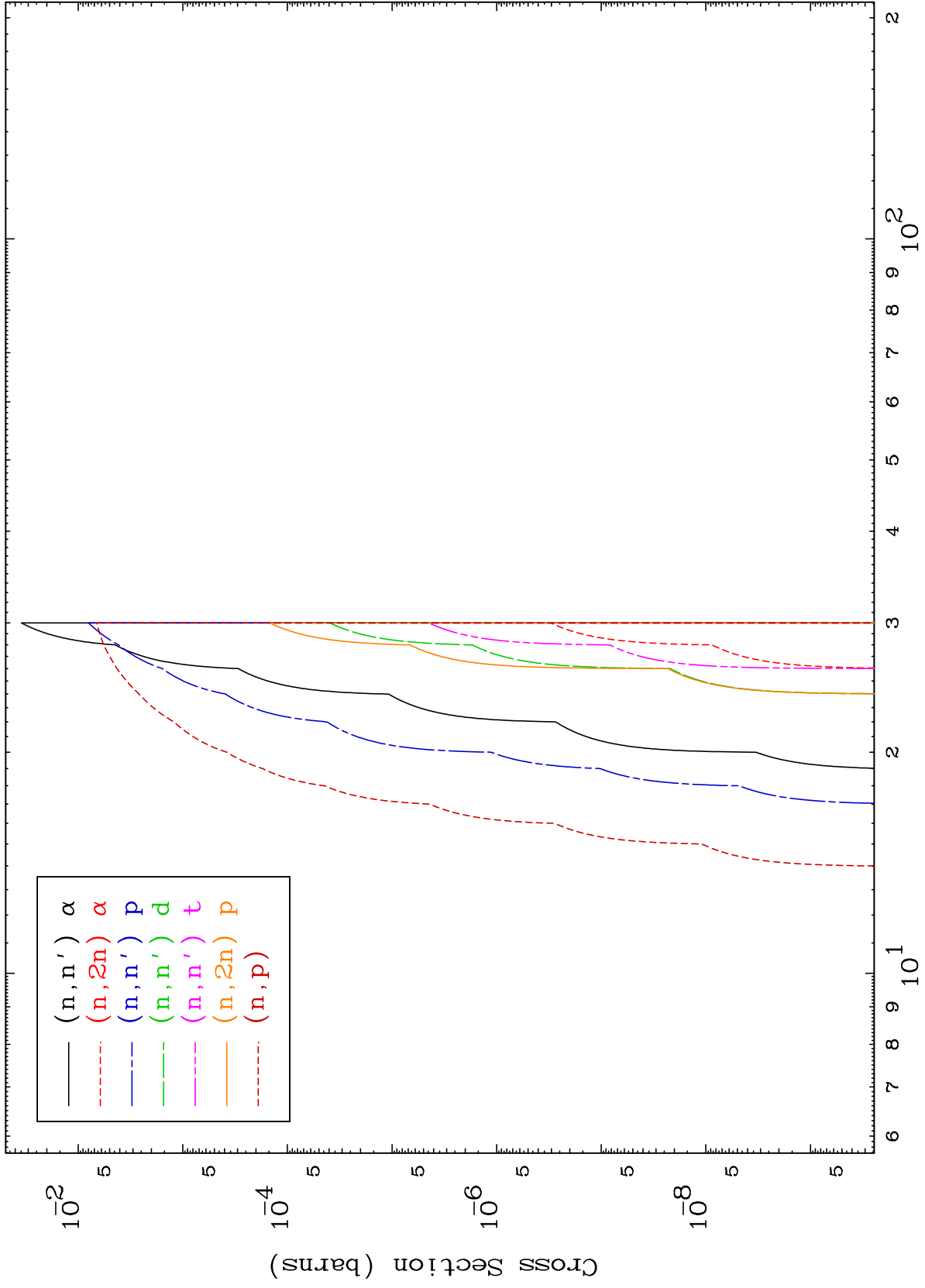


MAT 7924

α Neutron Absorption
0 Kelvin Cross Sections

79-Au-196n



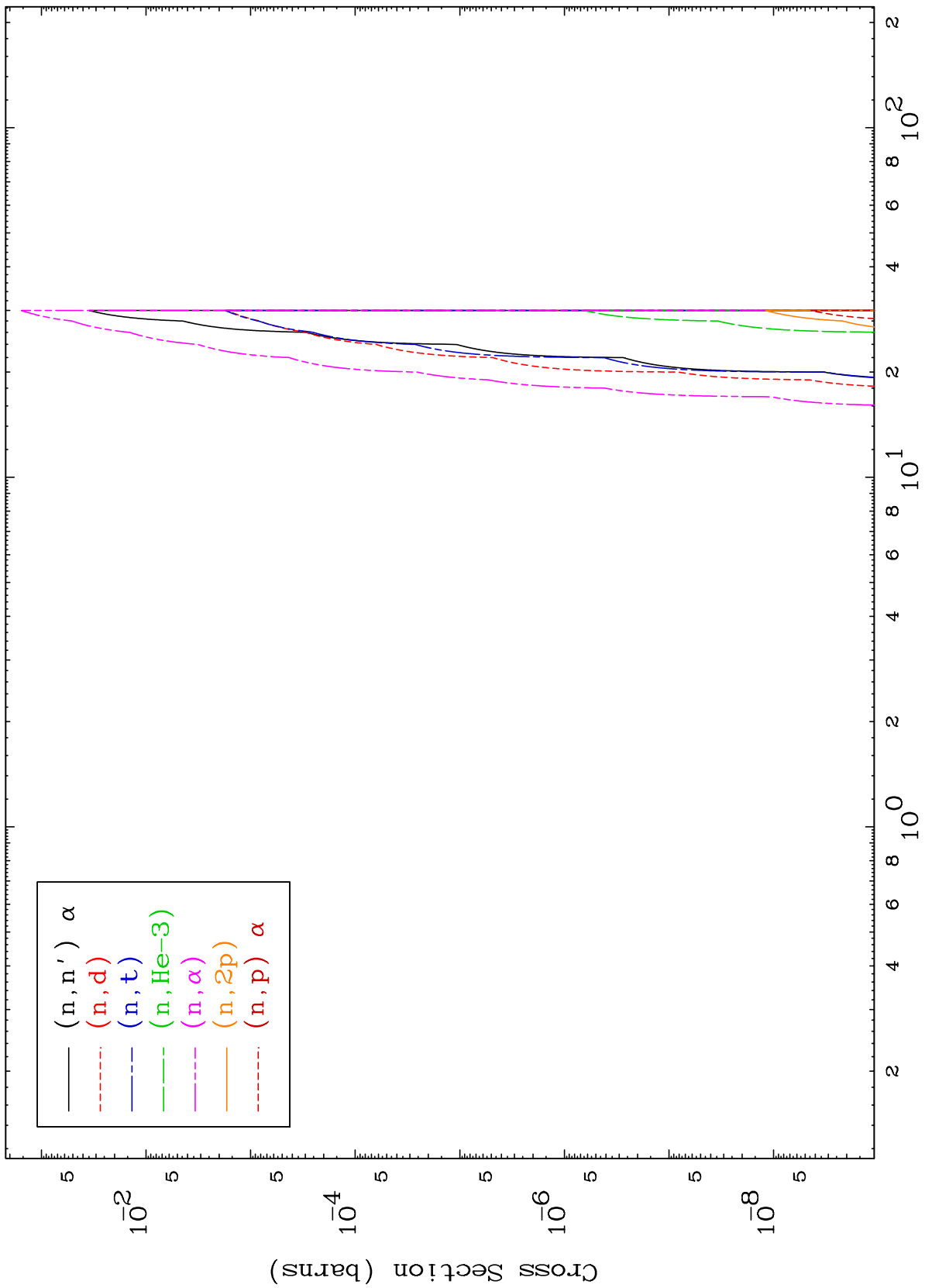


MAT 7924

α Charged Particle

79-Au-196n

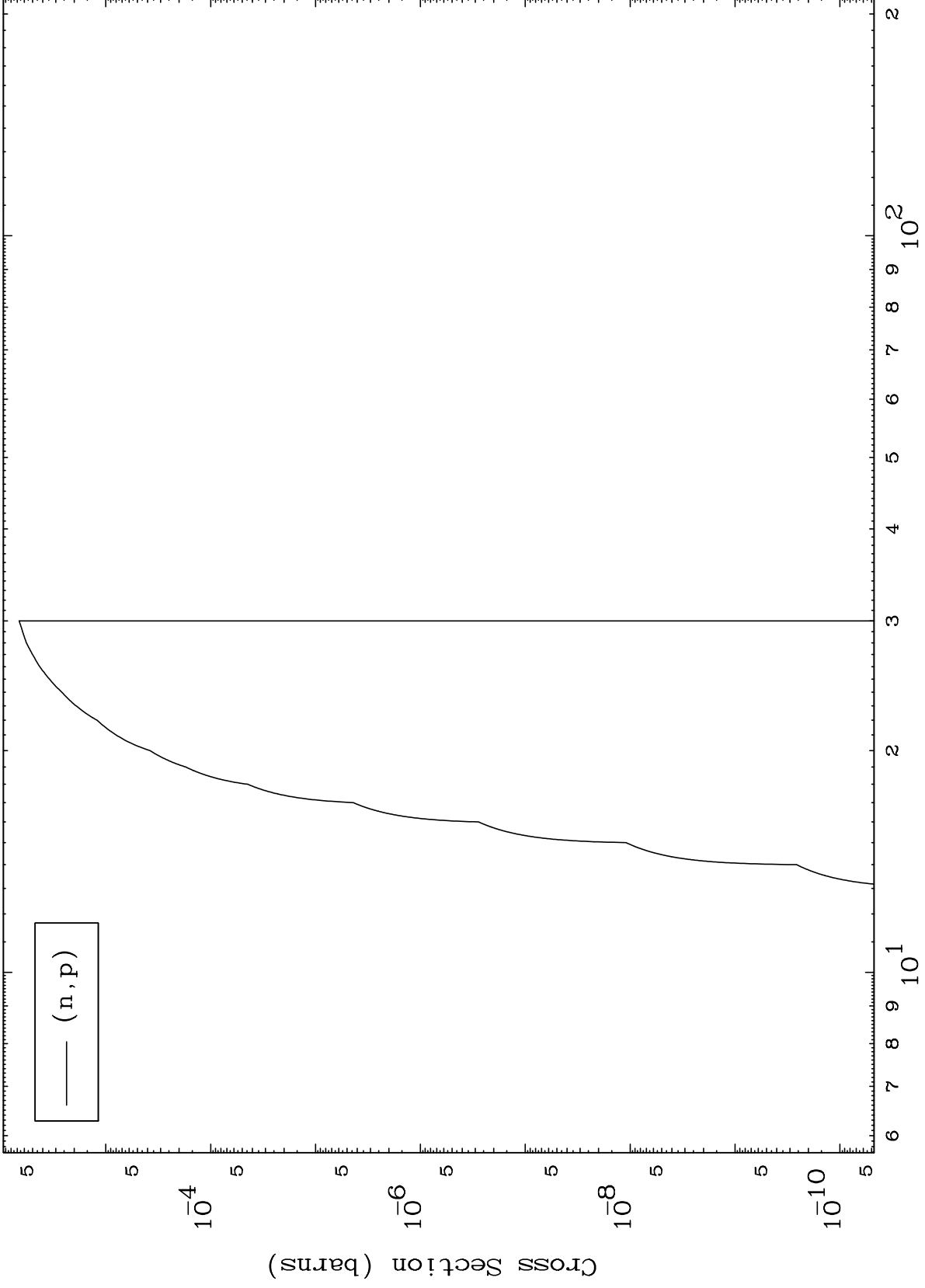
0 Kelvin Cross Sections



MAT 7924

(α, p) Levels
0 Kelvin Cross Sections

⁷⁹Au-196n



6

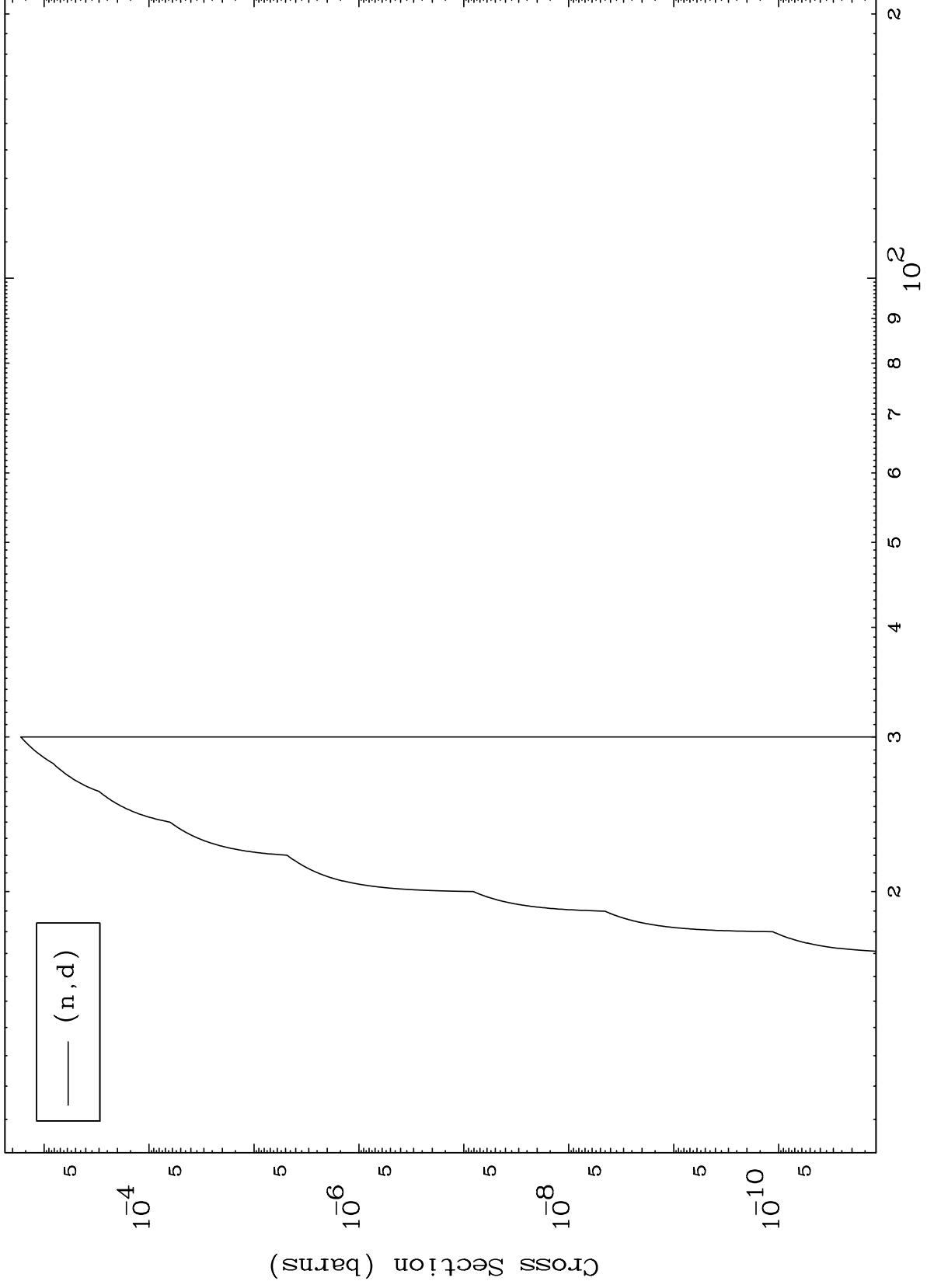
Incident Energy (MeV)

⁷⁹Au-196n

MAT 7924

(α, d) Levels
0 Kelvin Cross Sections

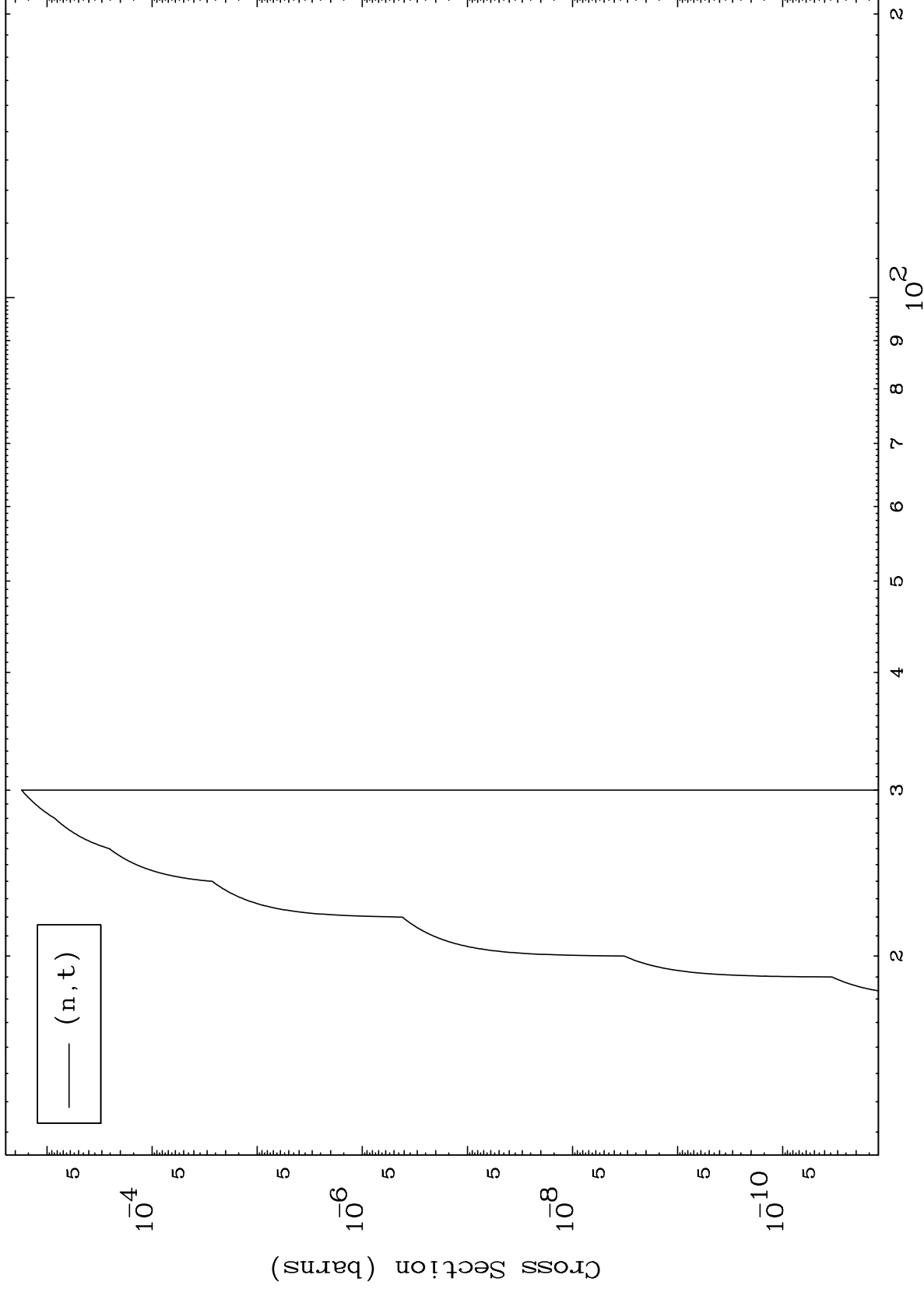
⁷⁹Au-196n

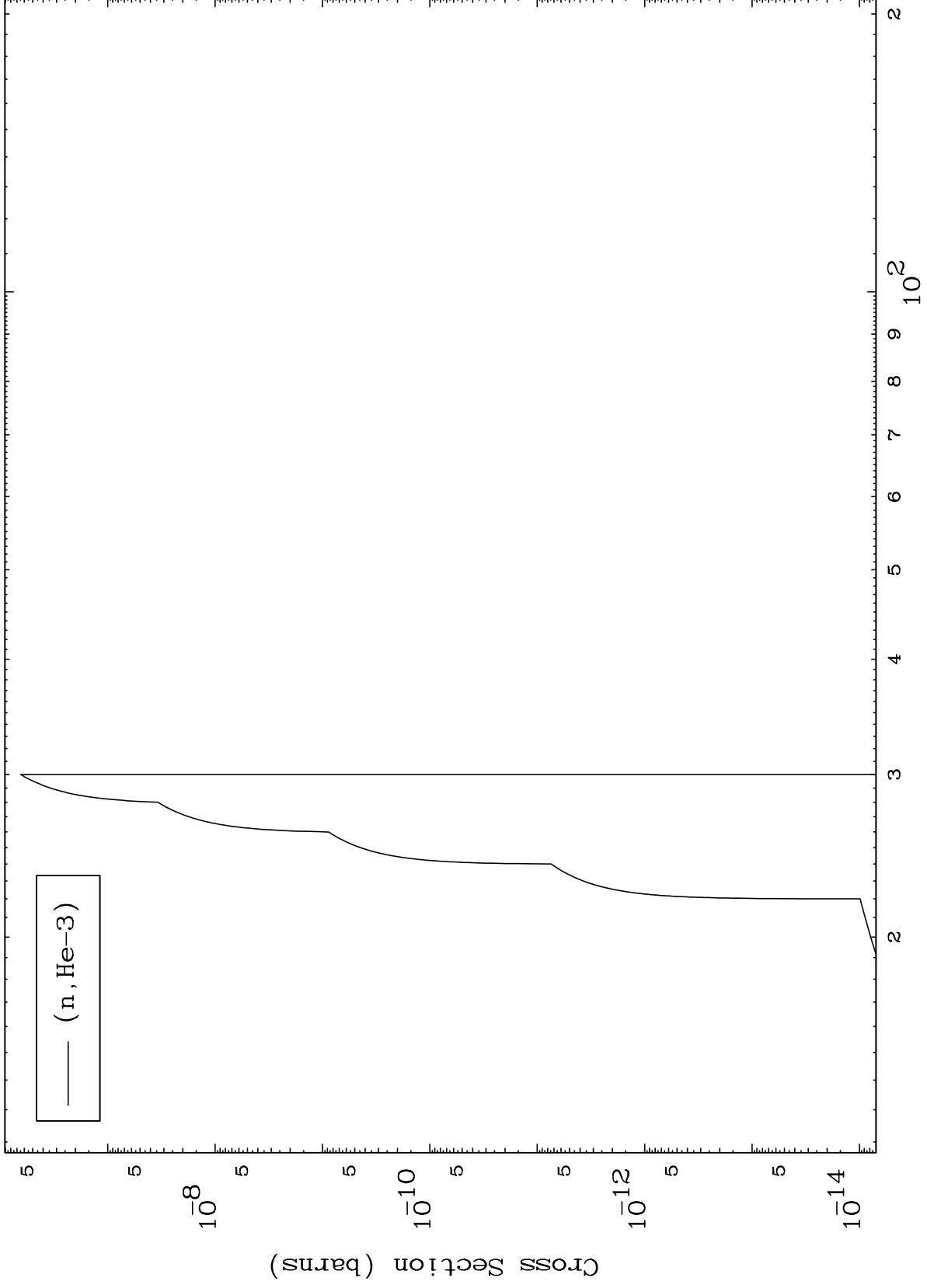


MAT 7924

(α, t) Levels
0 Kelvin Cross Sections

79-Au-196n



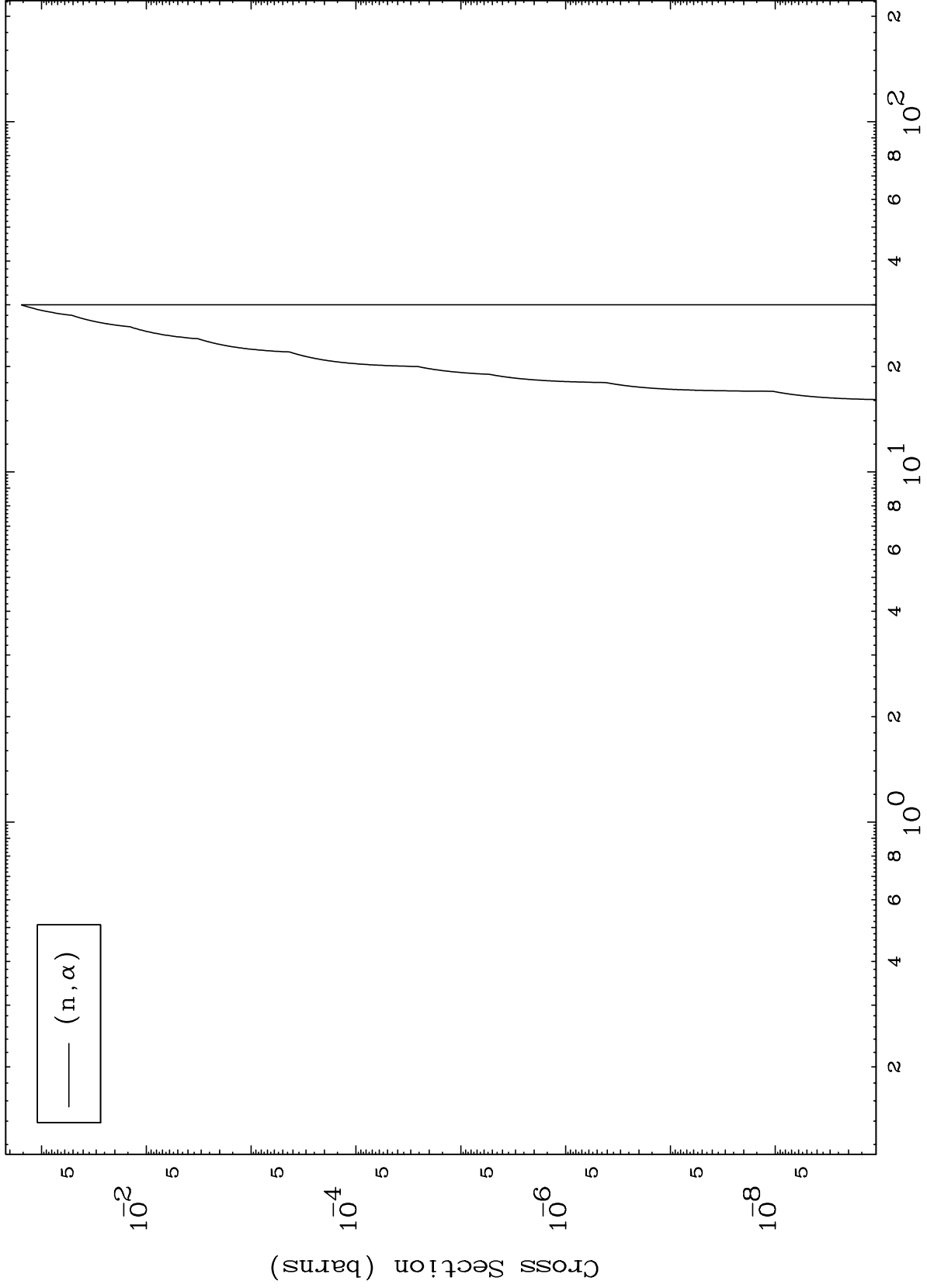


MAT 7924

(α, α) Levels

$^{79}\text{Au-196n}$

0 Kelvin Cross Sections



10

Incident Energy (MeV)

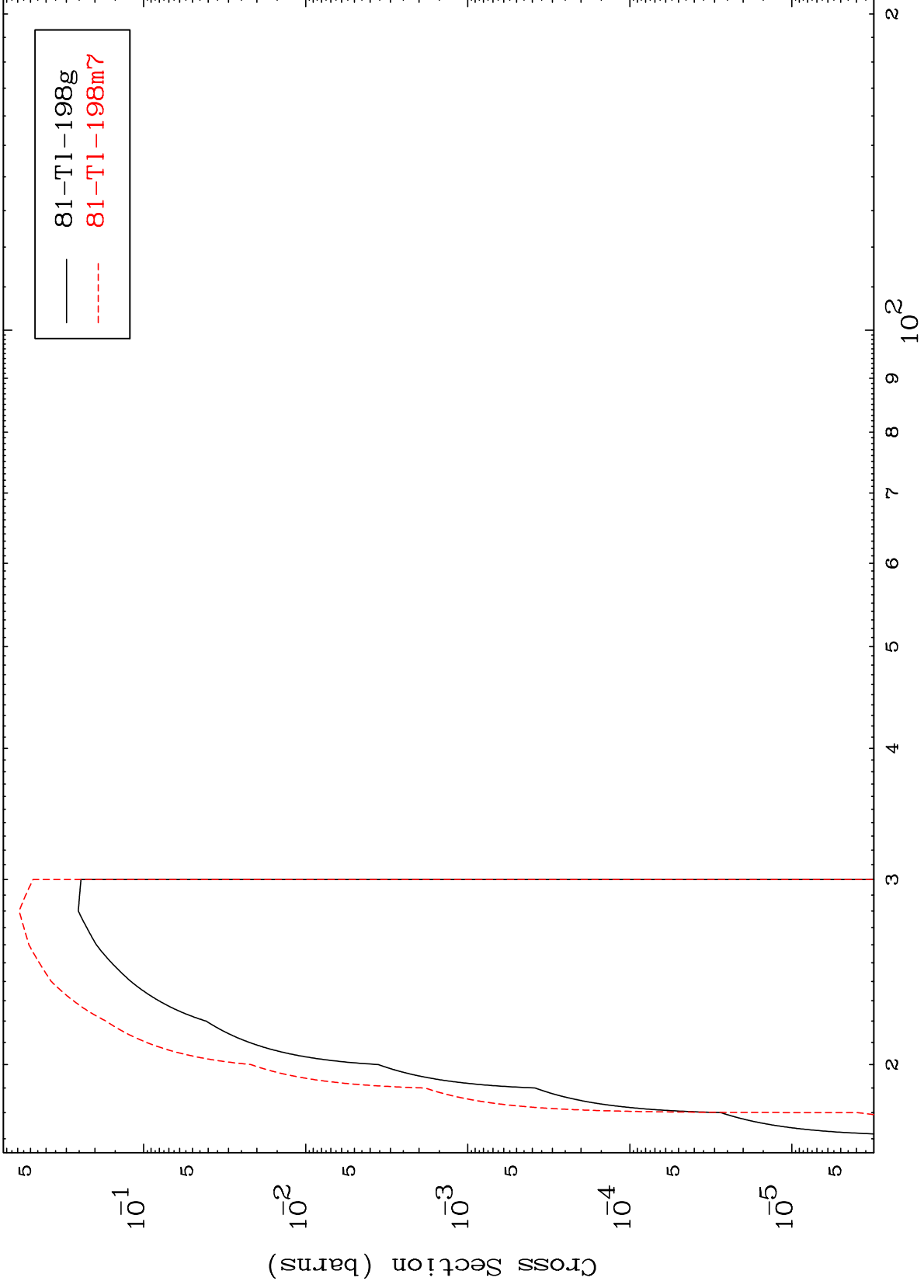
$^{79}\text{Au-196n}$

MAT 7924

(n,2n)

⁷⁹Au-196n

Radionuclide Production Cross Section



11

Incident Energy (MeV)

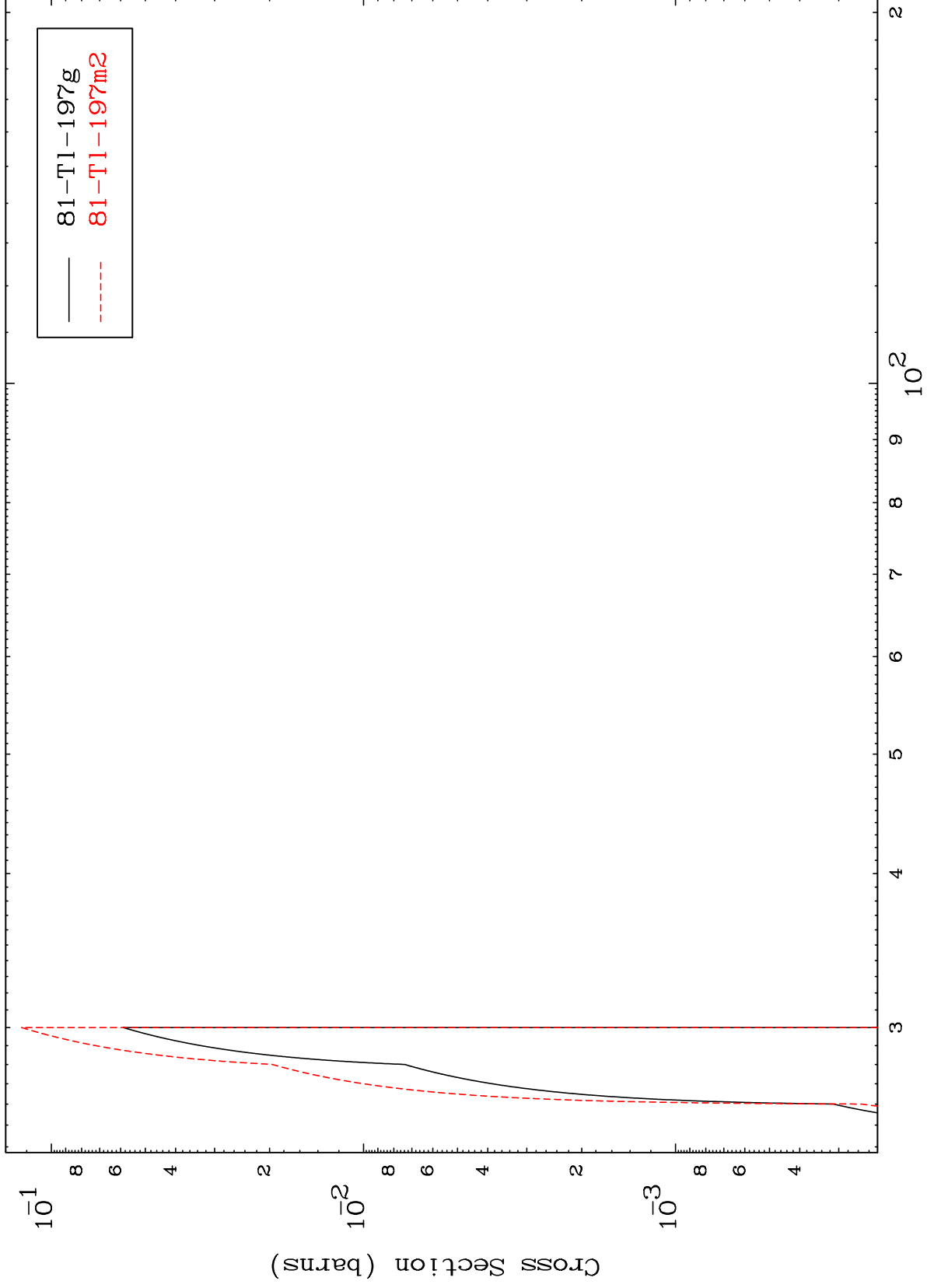
⁷⁹Au-196n

MAT 7924

(n,3n)

⁷⁹Au-196n

Radionuclide Production Cross Section



12

Incident Energy (MeV)

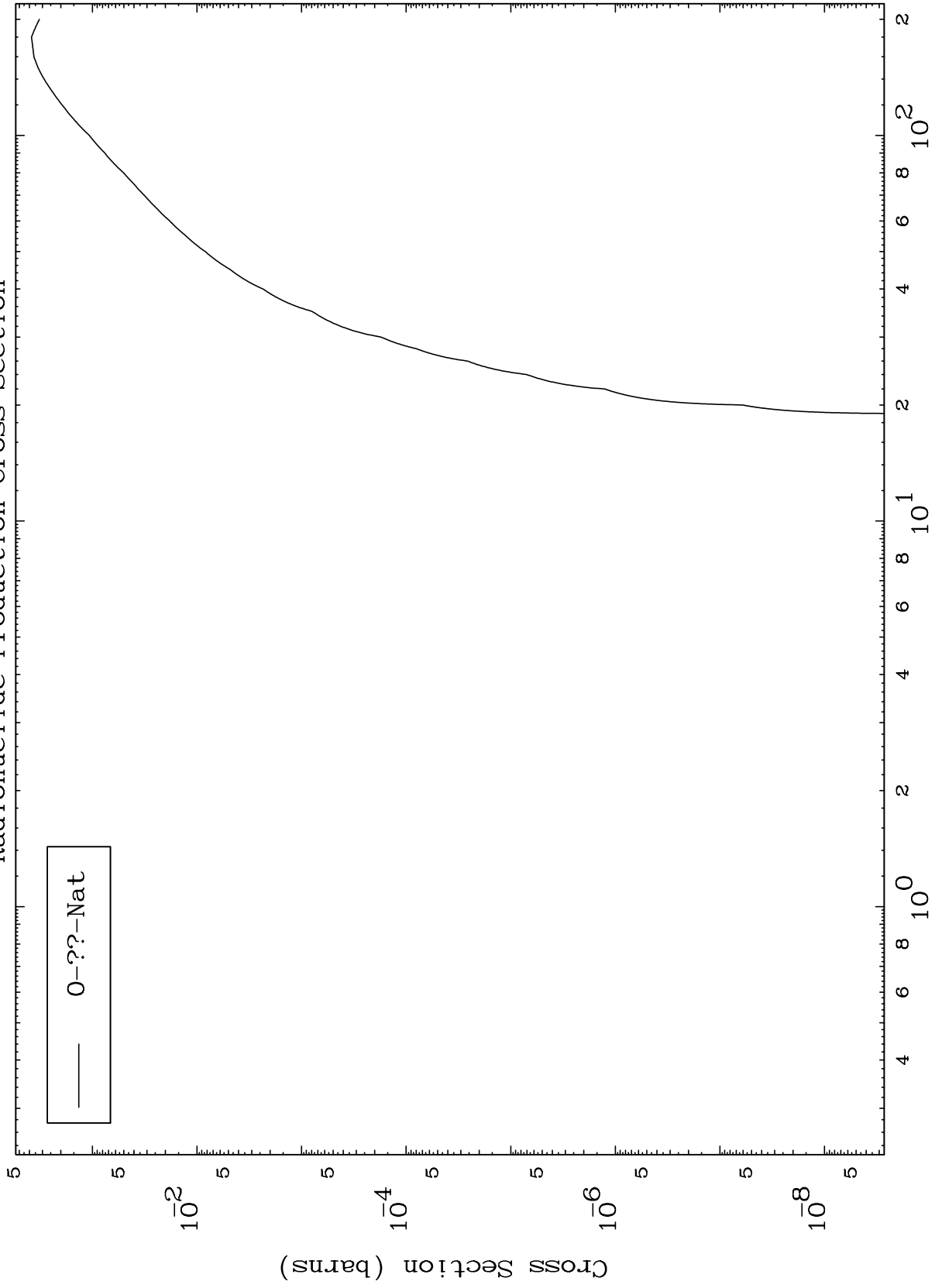
⁷⁹Au-196n

MAT 7924

Fission

⁷⁹Au-196n

Radionuclide Production Cross Section

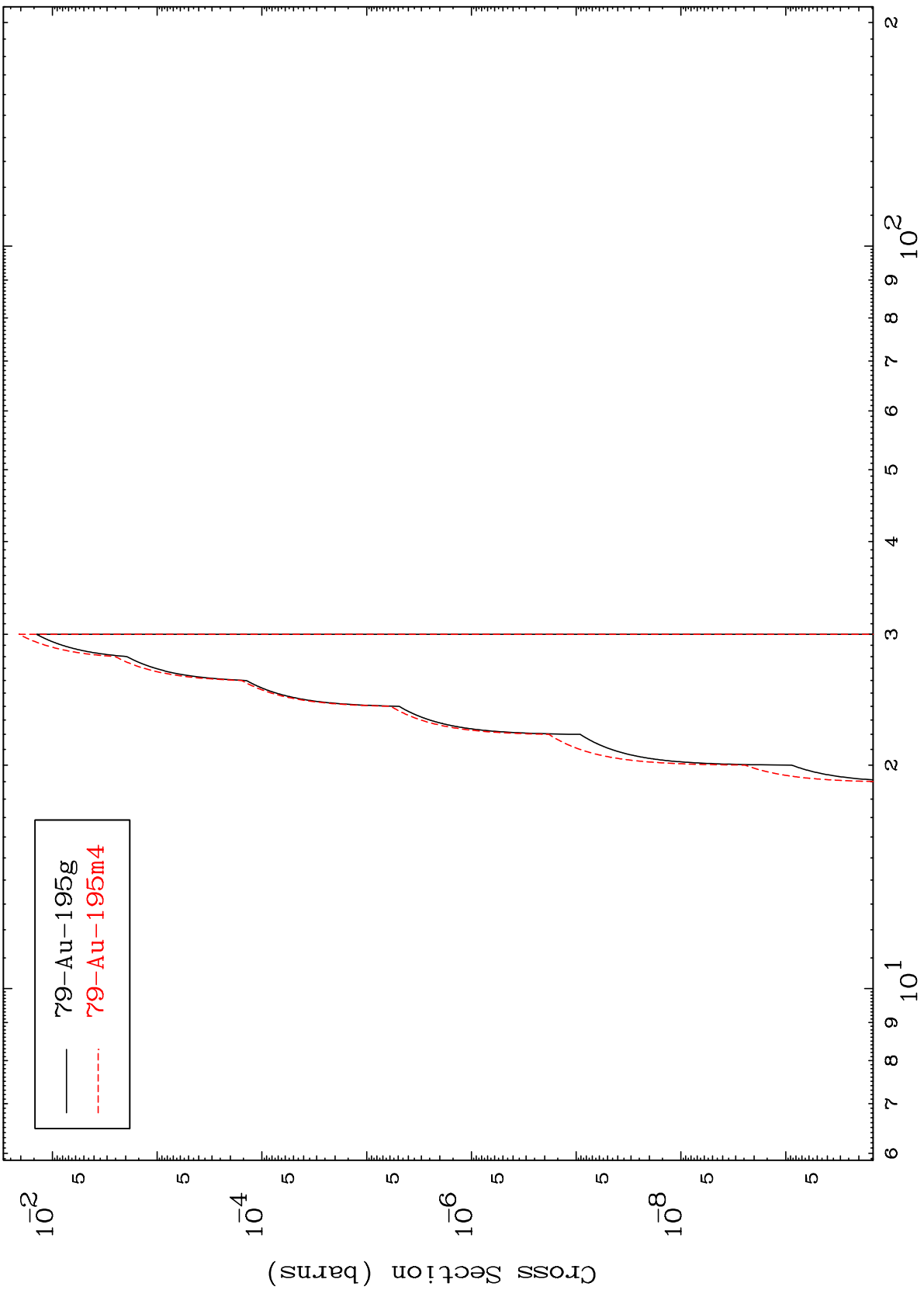


MAT 7924

$(n, n') \alpha$

$^{79}\text{Au}-196\text{n}$

Radionuclide Production Cross Section



Incident Energy (MeV)

$^{79}\text{Au}-196\text{n}$

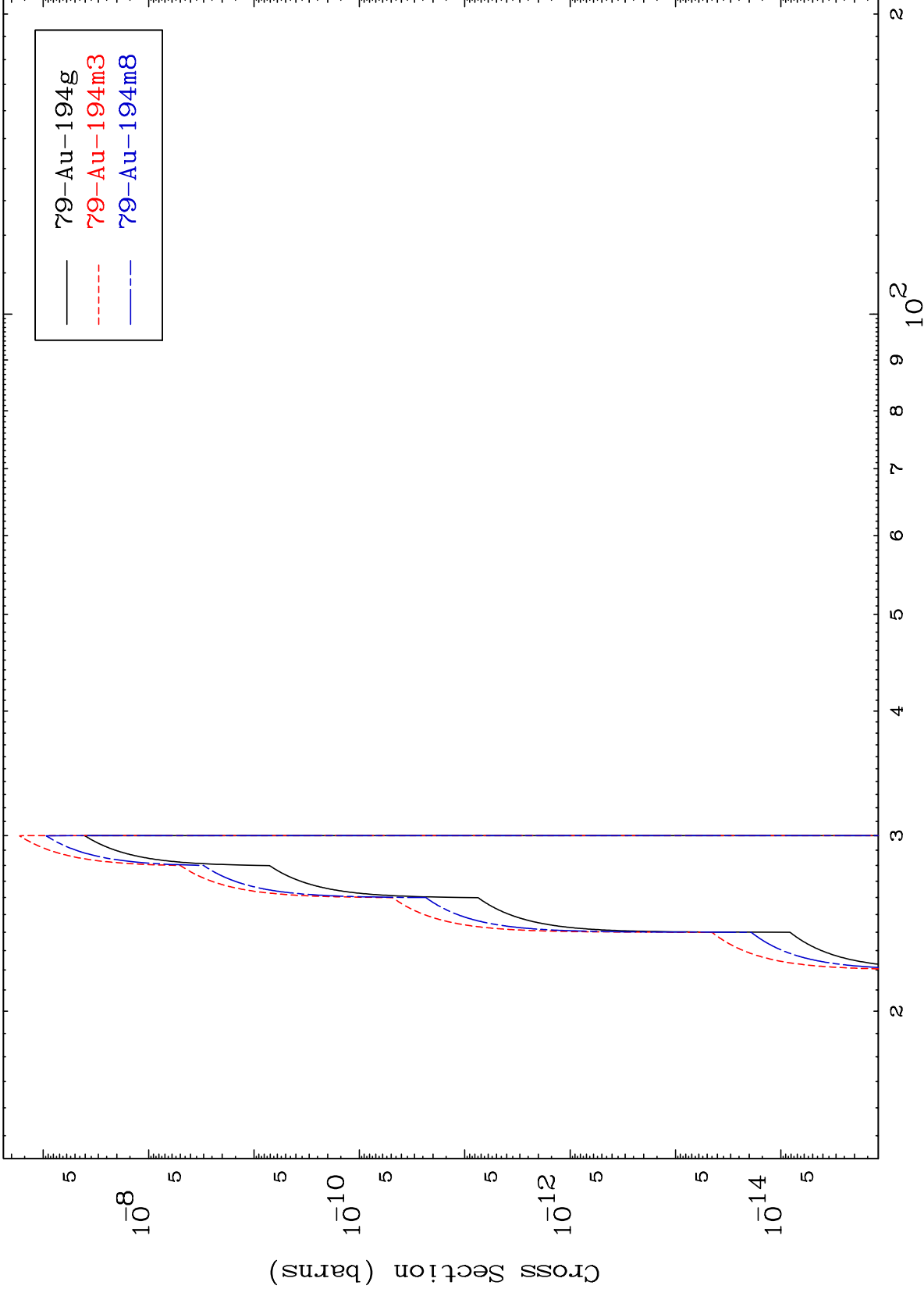
14

MAT 7924

$(n,2n) \alpha$

$^{79}\text{Au-196n}$

Radionuclide Production Cross Section



15

Incident Energy (MeV)

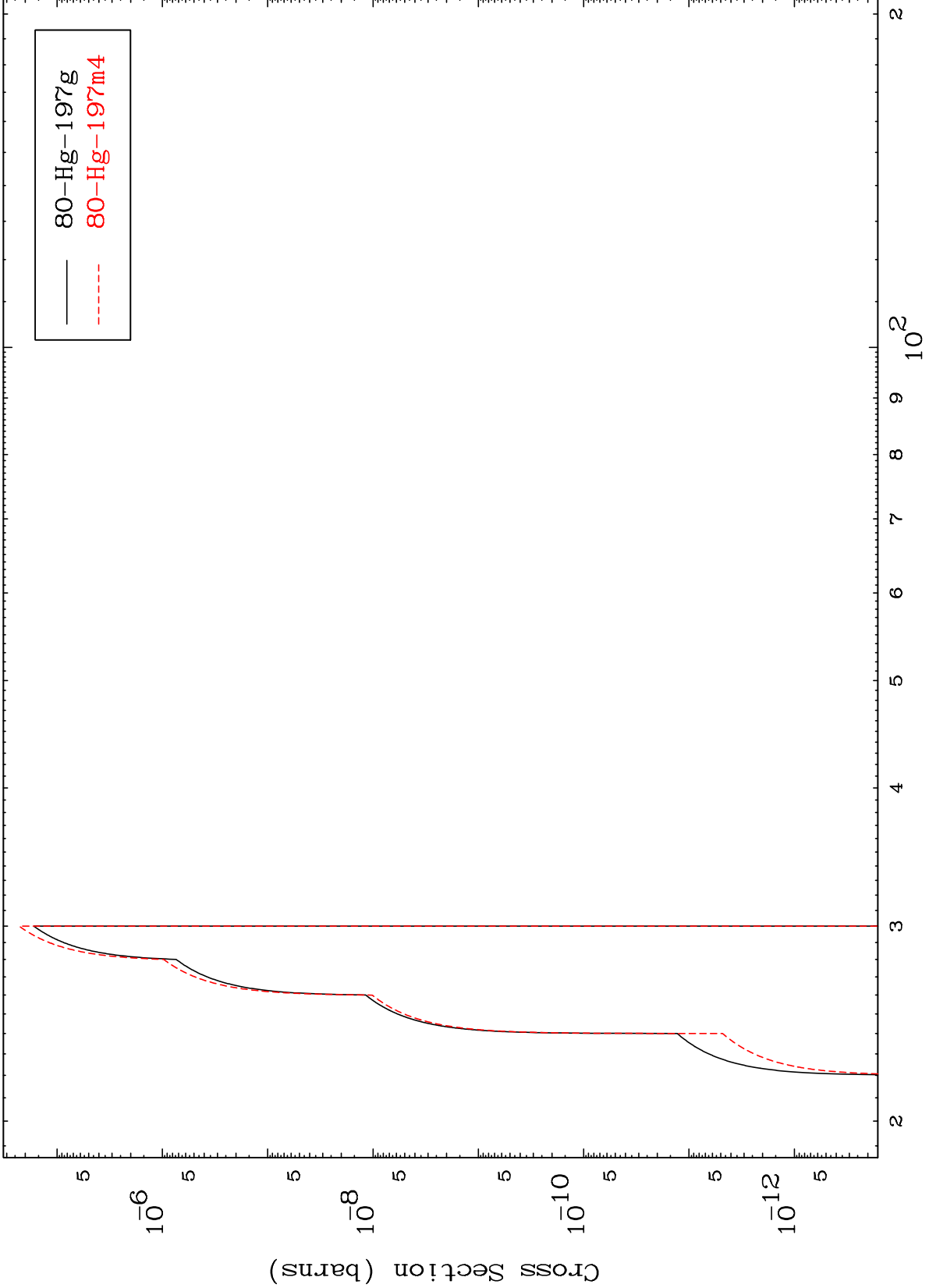
$^{79}\text{Au-196n}$

MAT 7924

(n,n') d

79-Au-196n

Radionuclide Production Cross Section



16

Incident Energy (MeV)

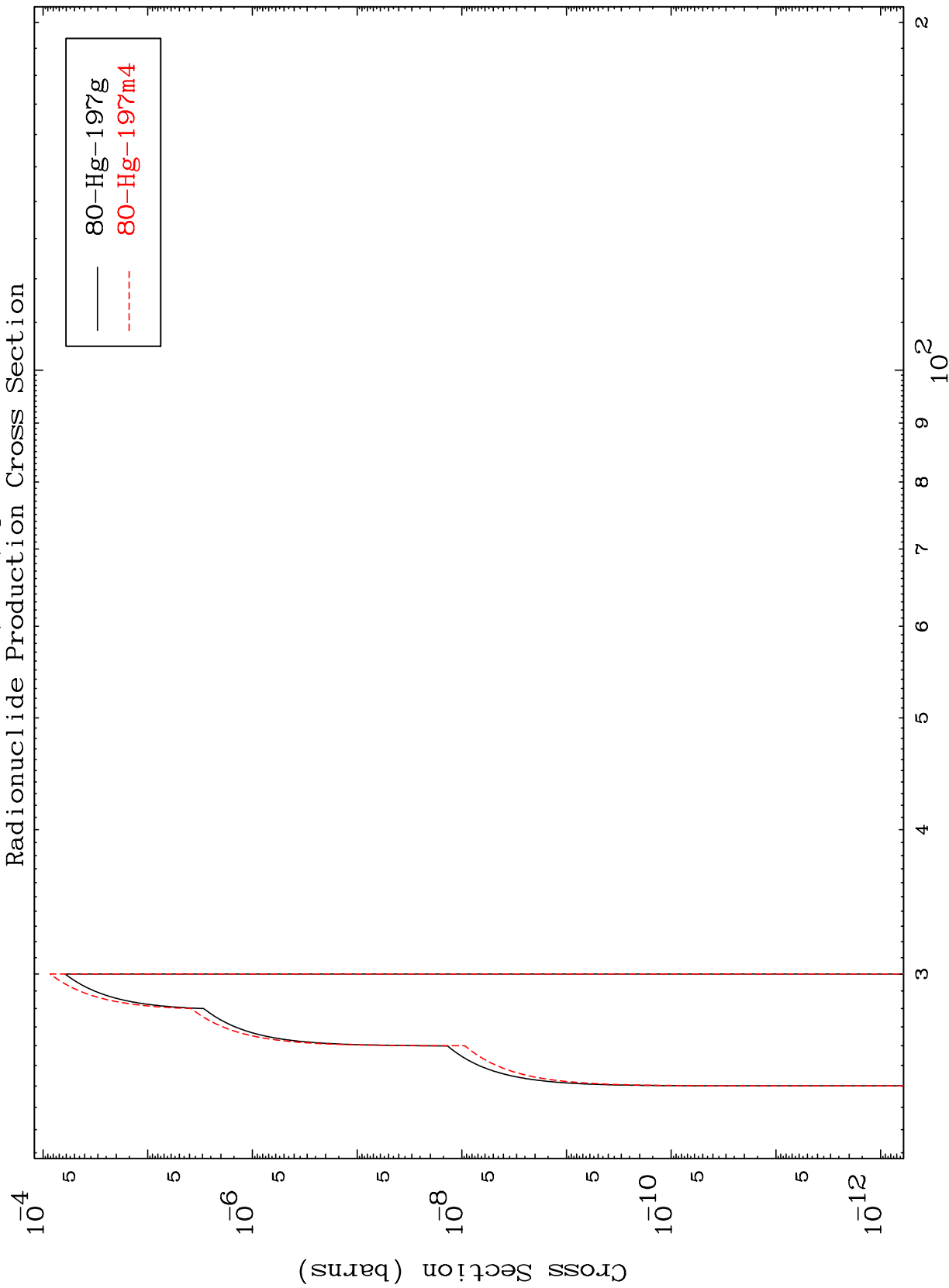
79-Au-196n

MAT 7924

79-Au-196n

(n,2n) p

Radionuclide Production Cross Section



17

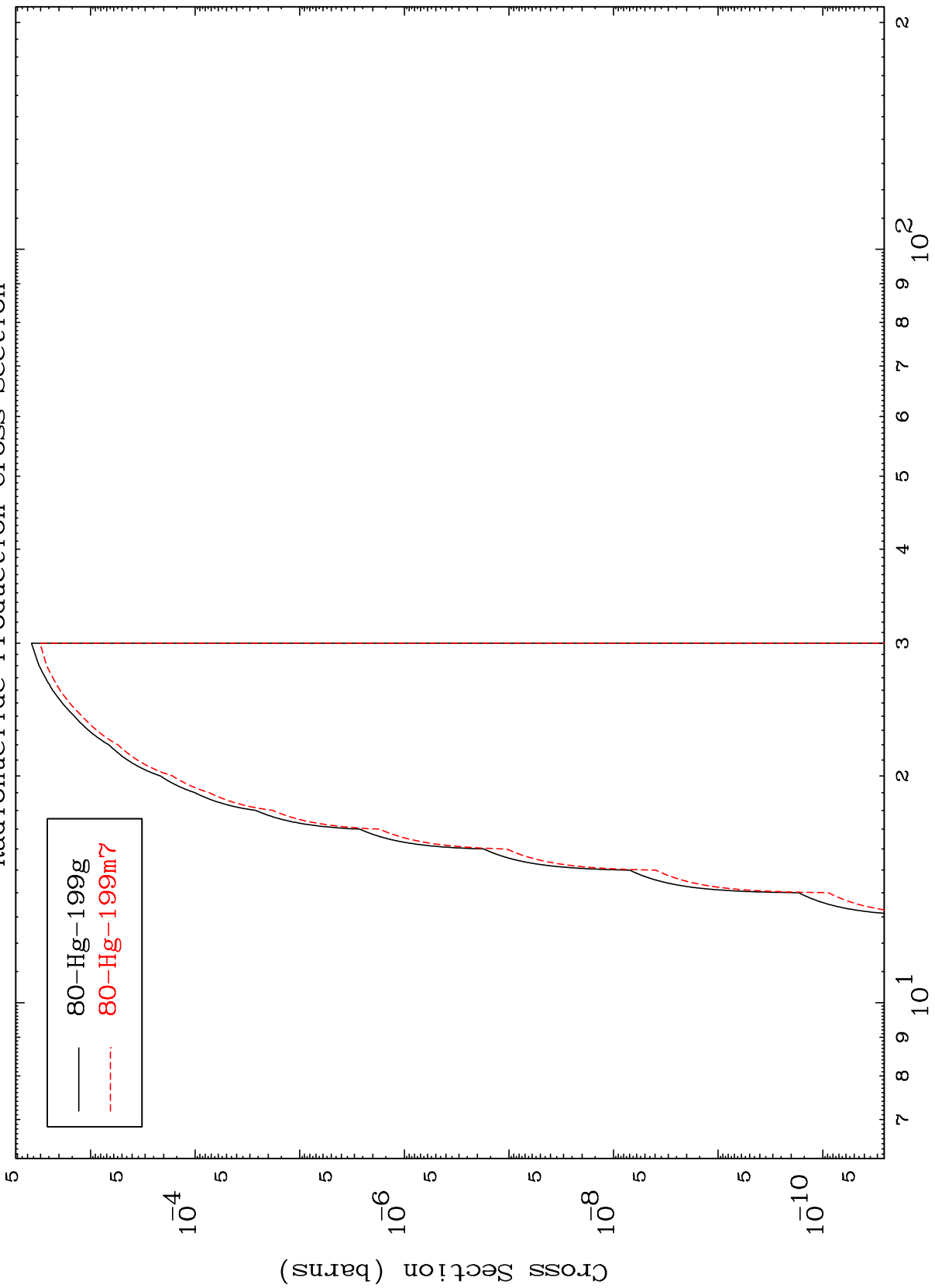
Incident Energy (MeV)

79-Au-196n

MAT 7924

79-Au-196n

(n,p)
Radionuclide Production Cross Section



18

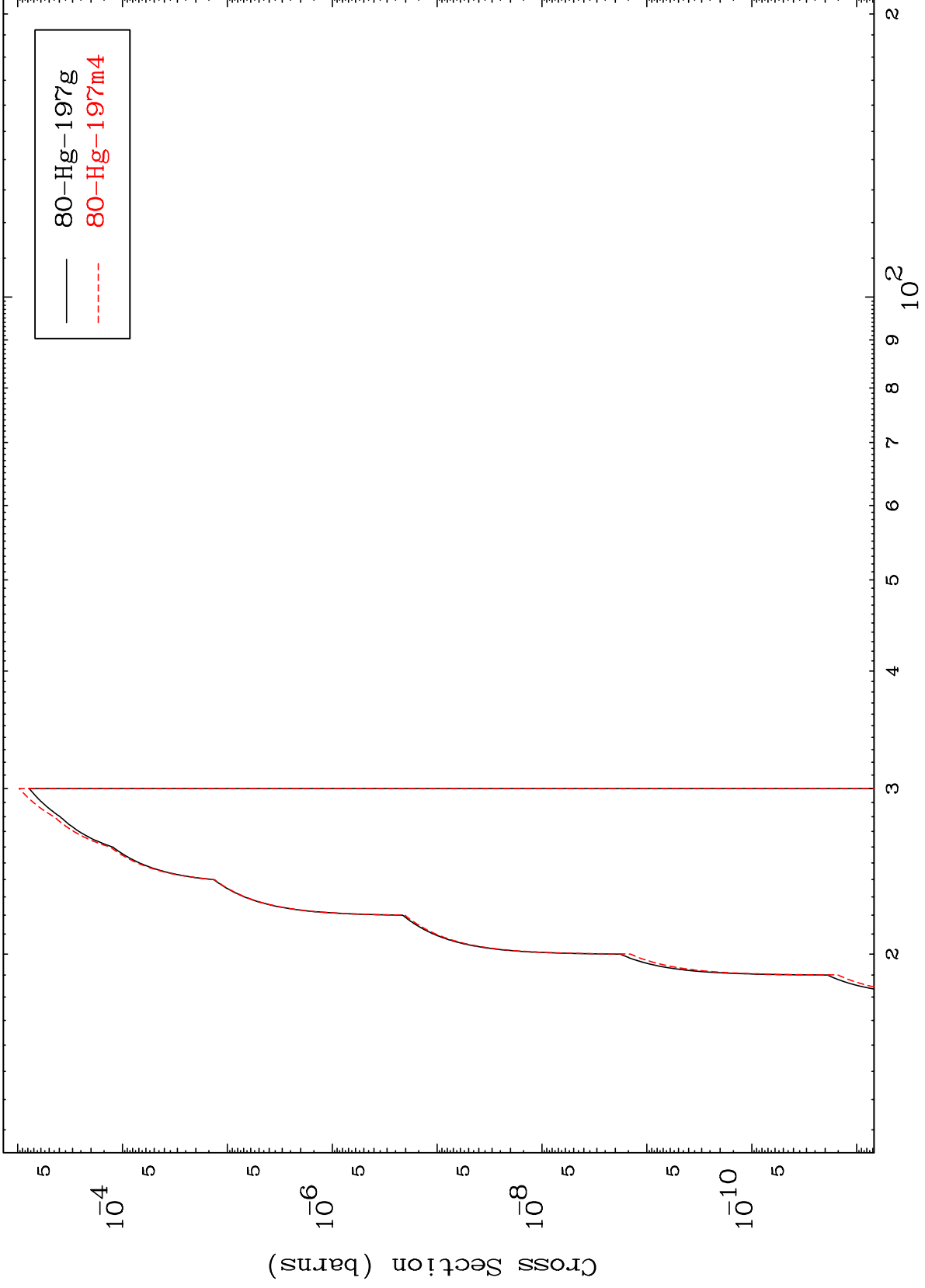
Incident Energy (MeV)

79-Au-196n

MAT 7924

79-Au-196n

(n, t)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

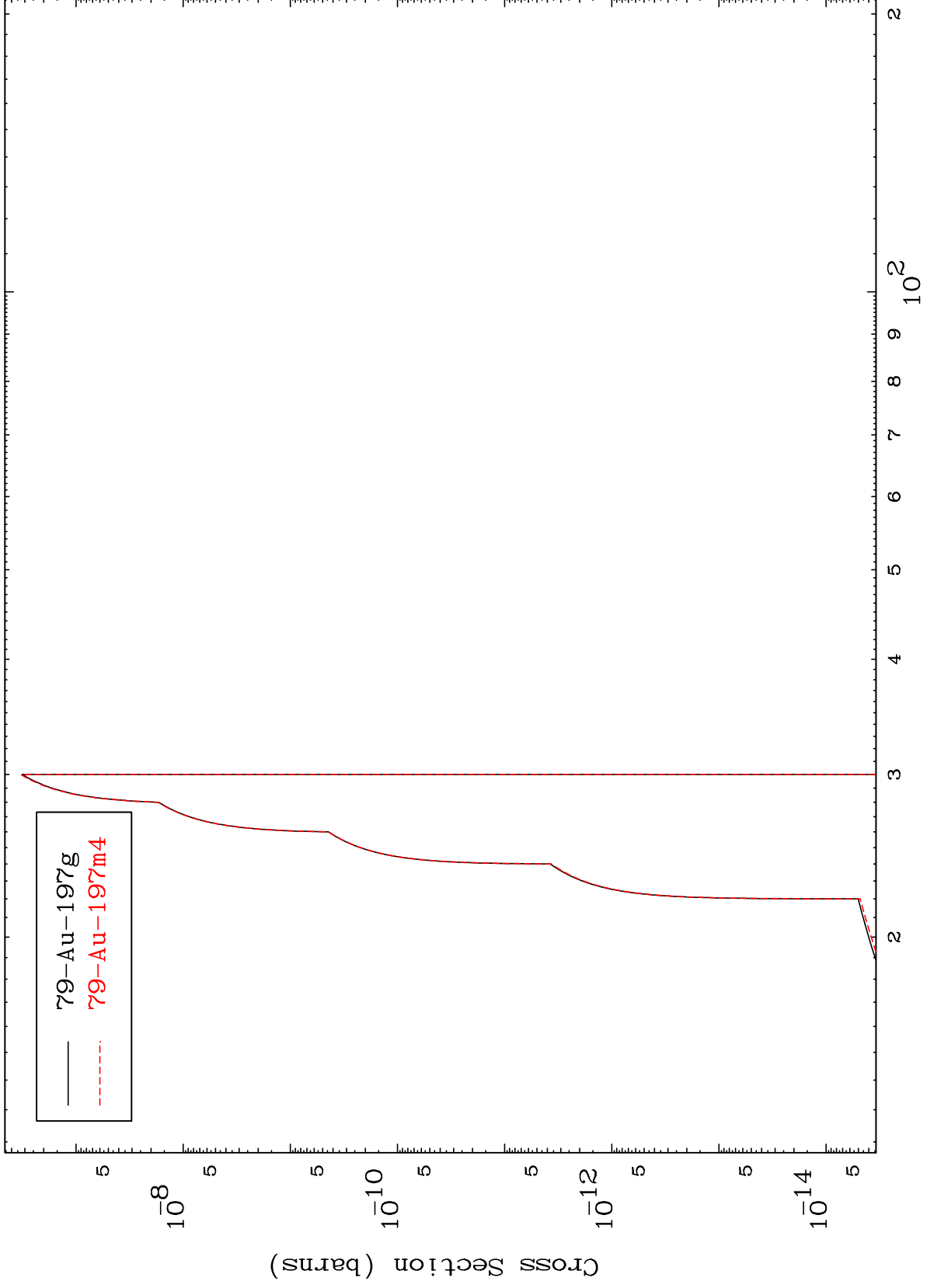
79-Au-196n

MAT 7924

(n,He-3)

79-Au-196n

Radionuclide Production Cross Section



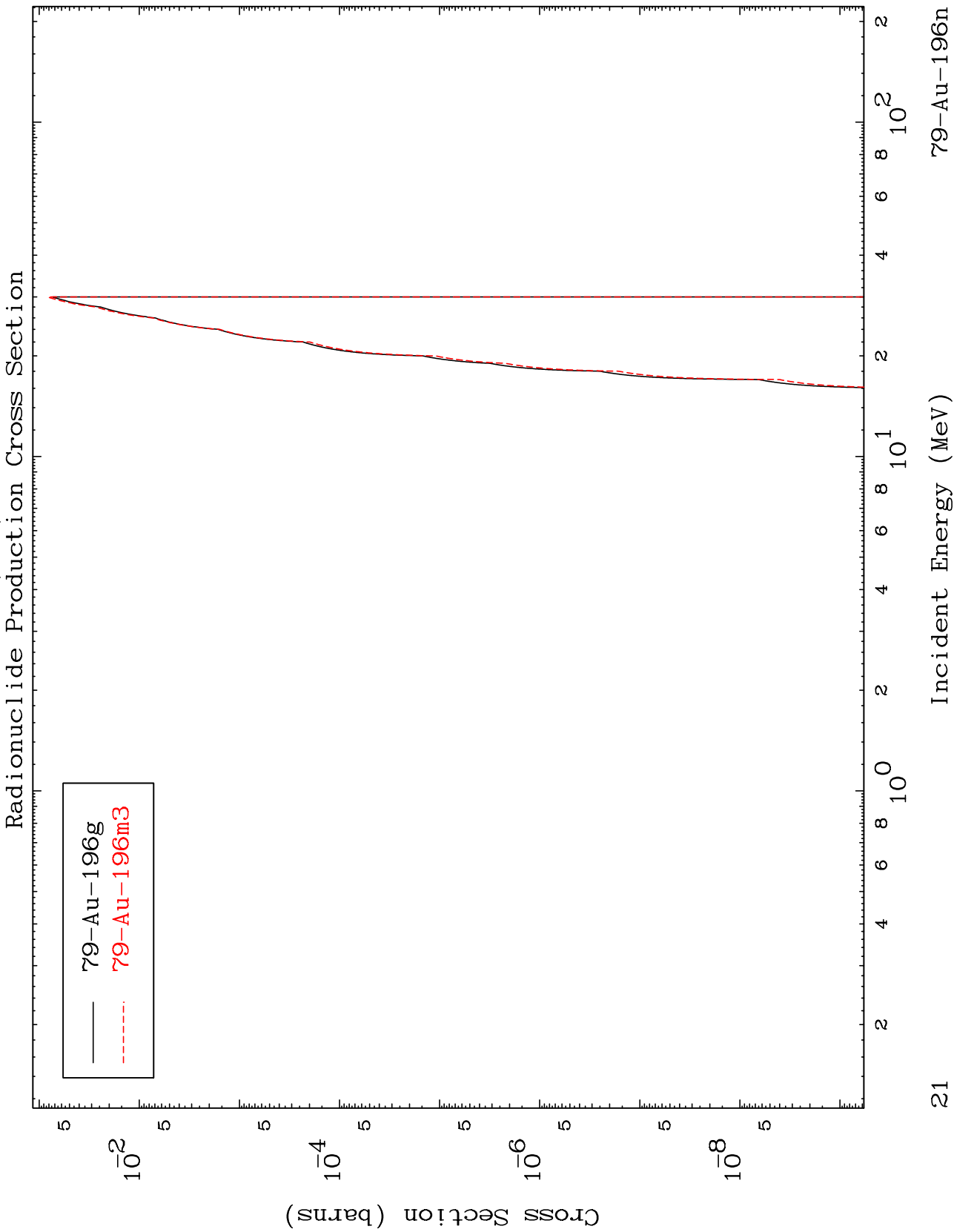
20

Incident Energy (MeV)

79-Au-196n

MAT 7924

⁷⁹Au-196n



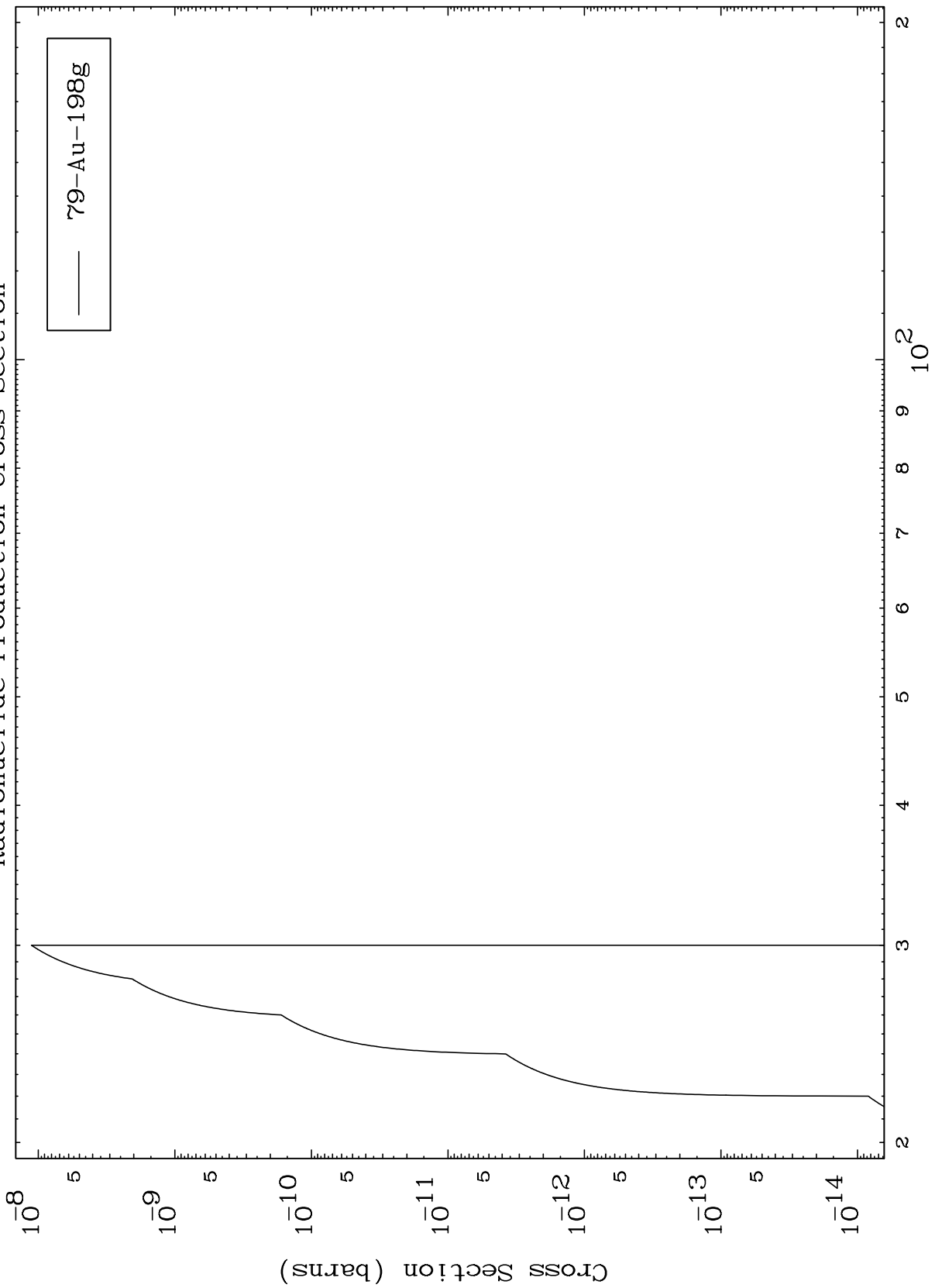
⁷⁹Au-196n

MAT 7924

⁷⁹Au-196n

(n,2p)

Radionuclide Production Cross Section



22

Incident Energy (MeV)

⁷⁹Au-196n

MAT 7924

$^{79}\text{Au-196n}$ $(n,p) \alpha$

$^{79}\text{Au-196n}$

Radionuclide Production Cross Section

