

Program EVALPLOT
(Version 2021-1)

by

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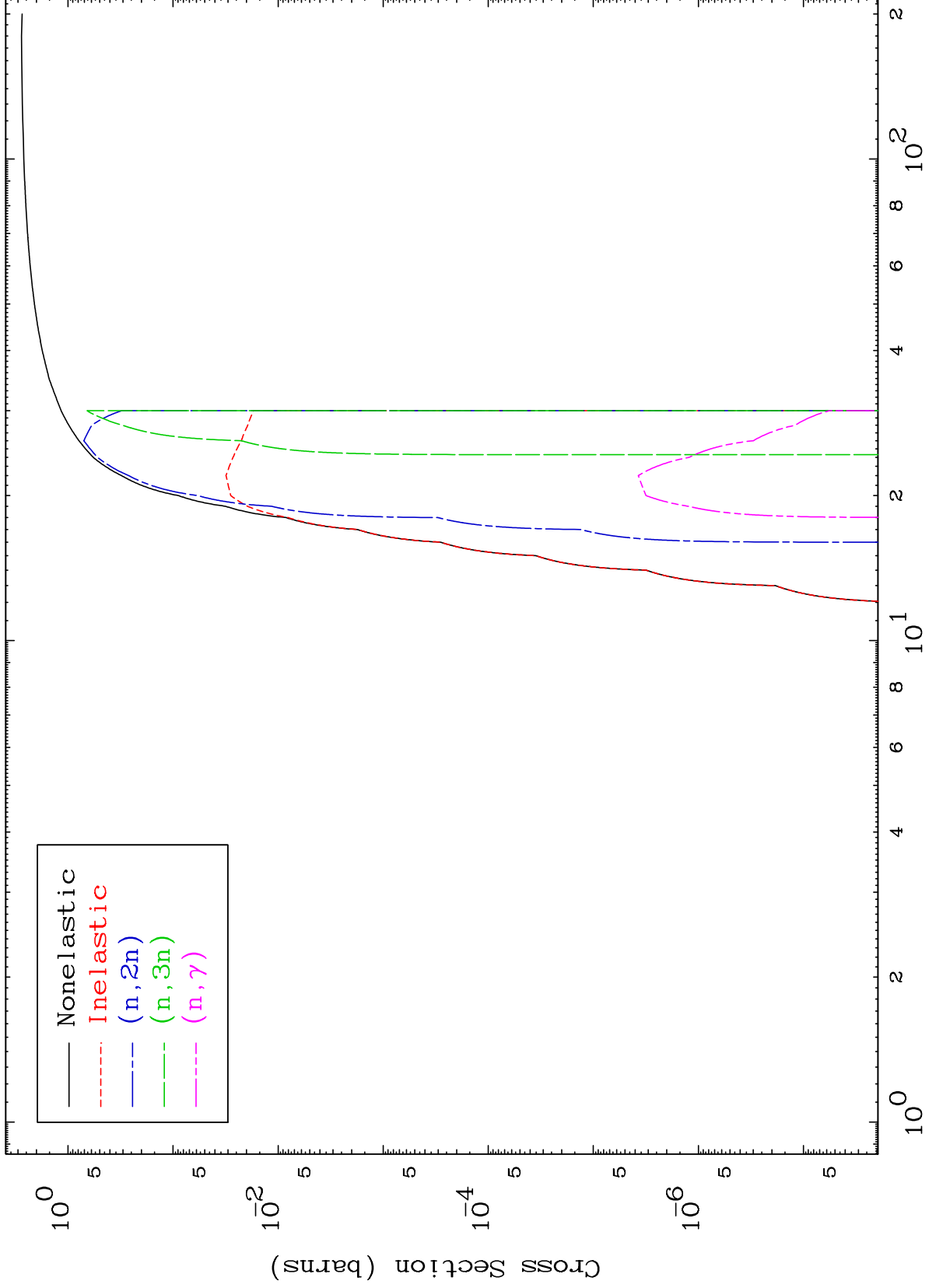
Press Mouse Button to Start

MAT 7931

α Major

79-Au-199

0 Kelvin Cross Sections



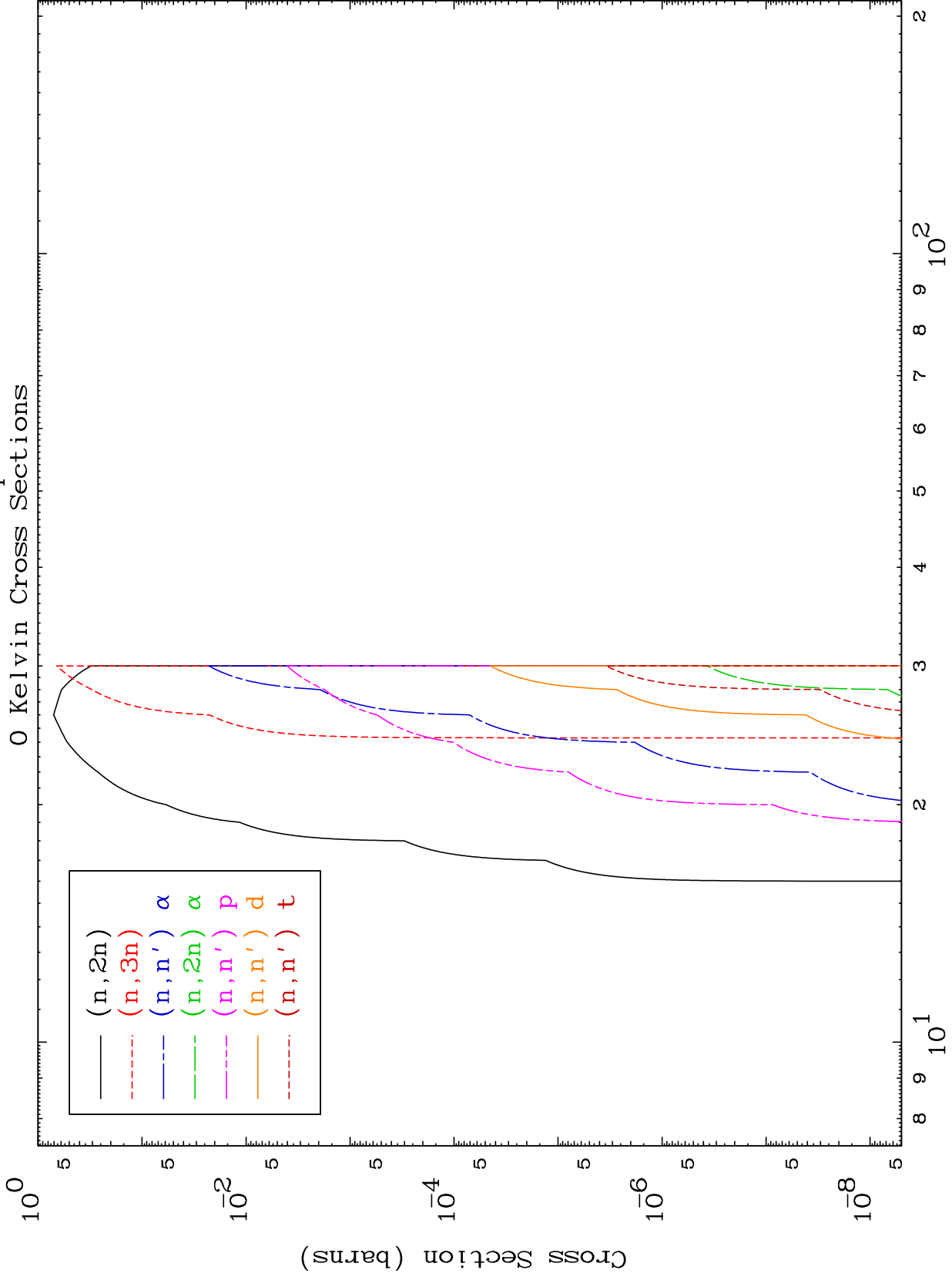
Incident Energy (MeV)

79-Au-199

MAT 7931

α Neutron Absorption
0 Kelvin Cross Sections

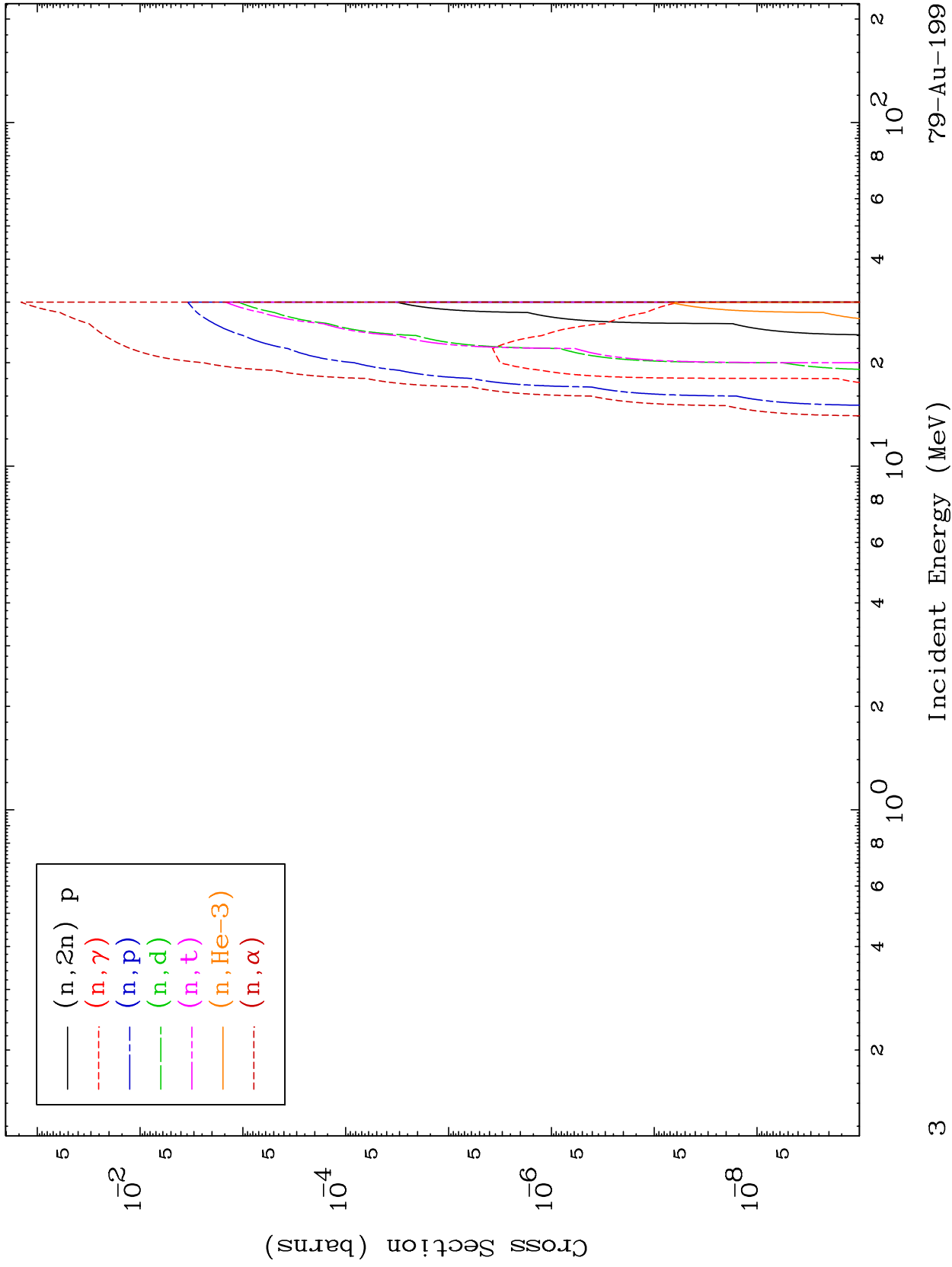
79-Au-199

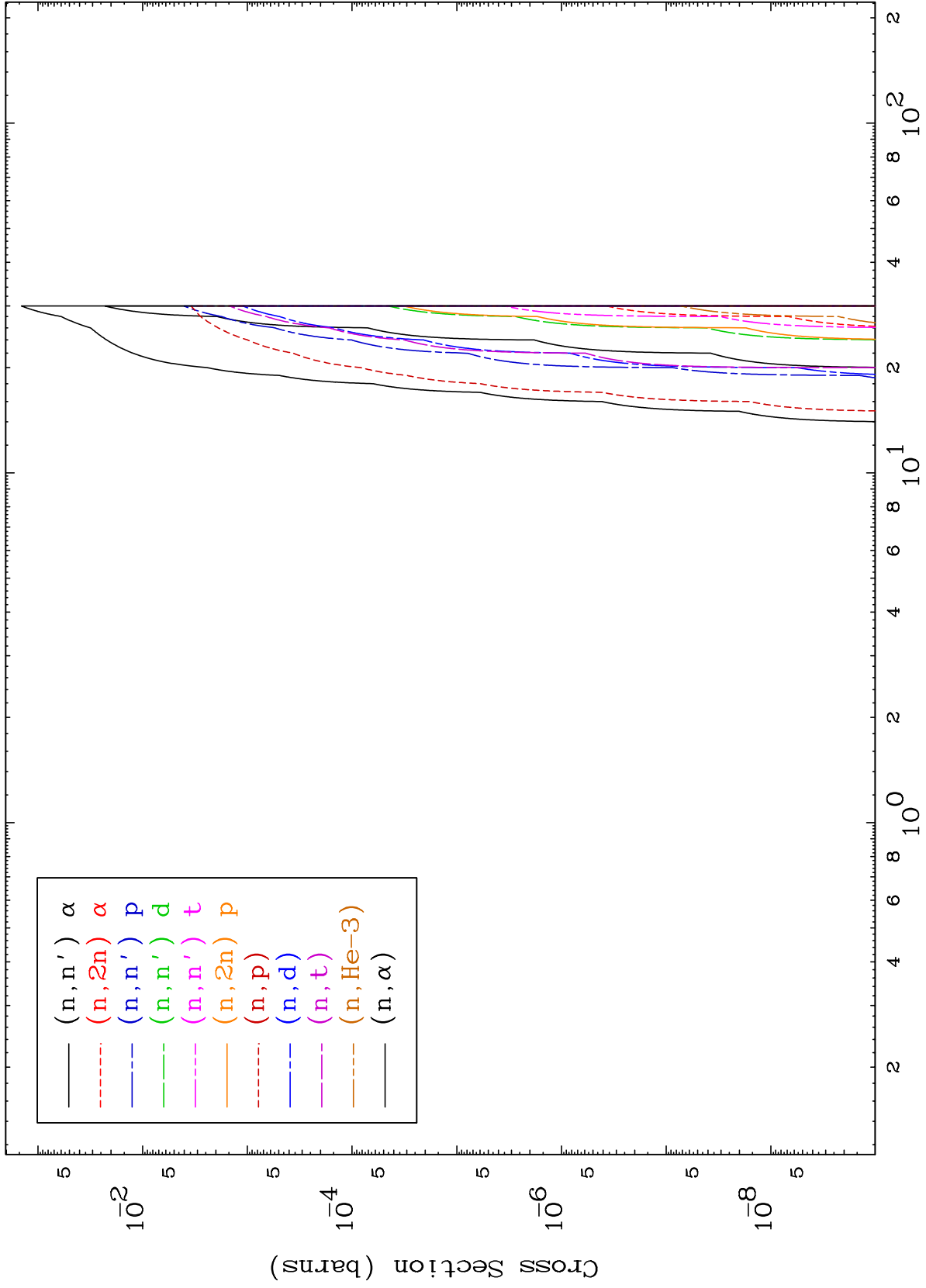


2

Incident Energy (MeV)

79-Au-199

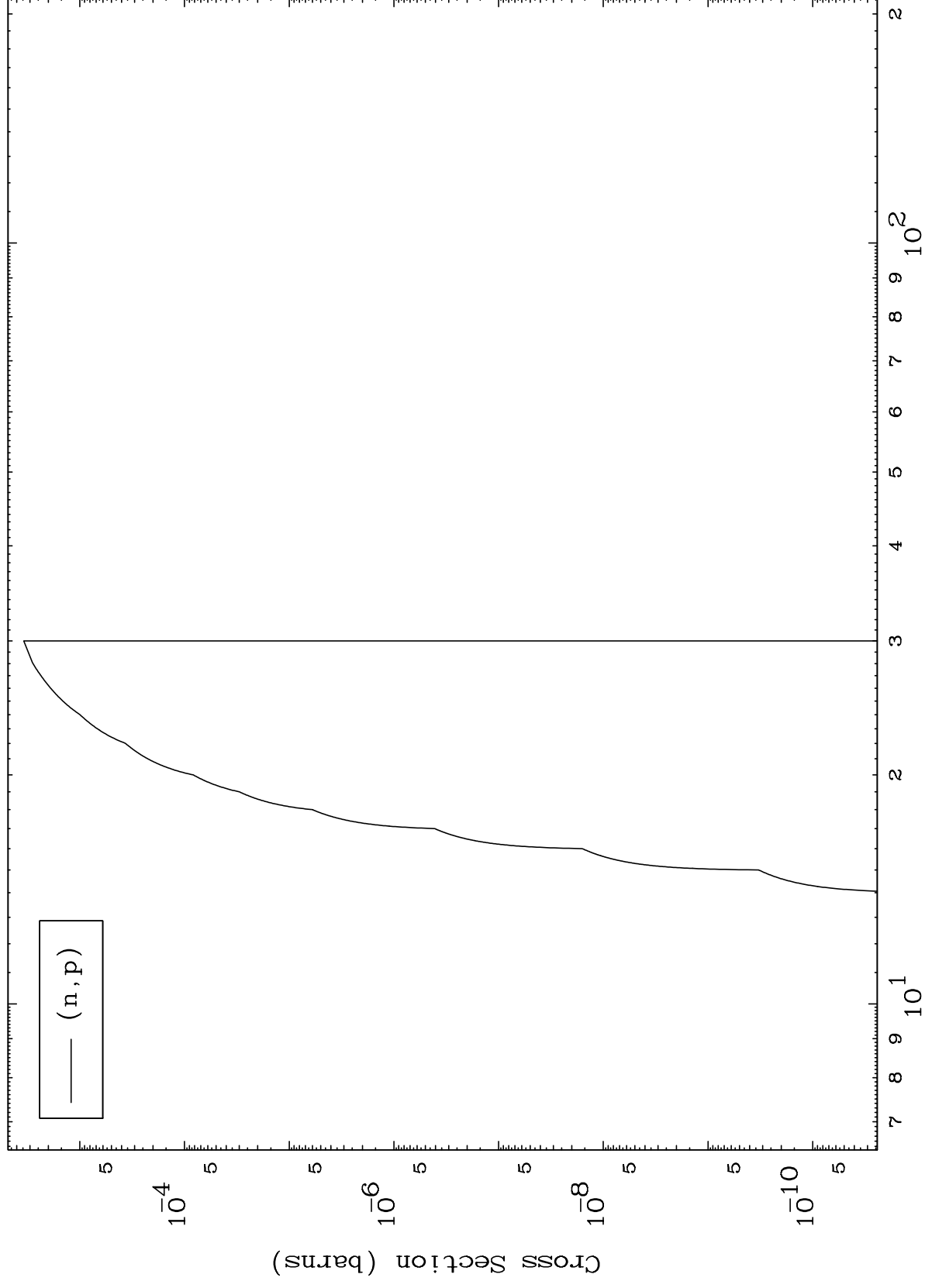




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79-Au-199

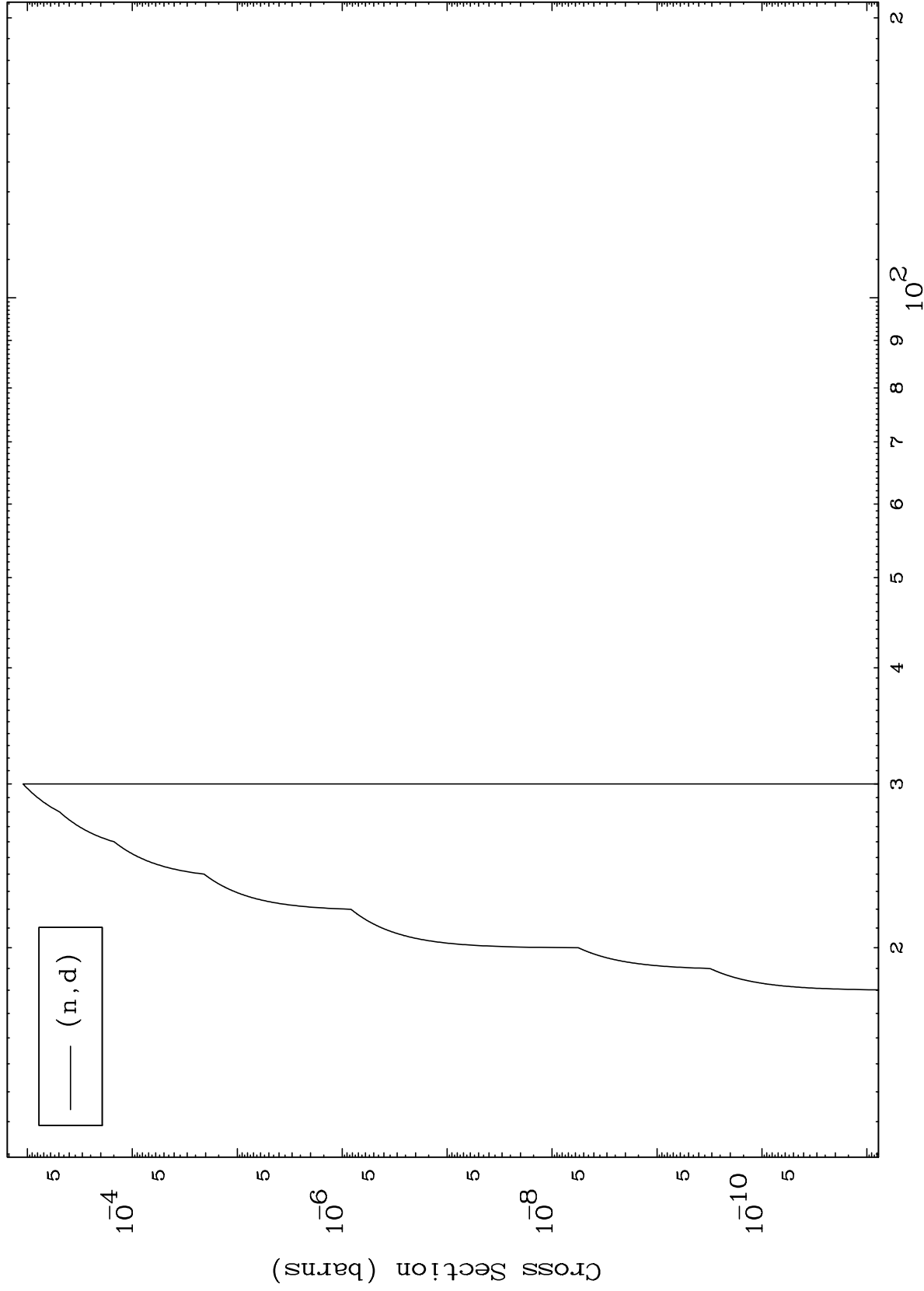
(α, p) Levels
0 Kelvin Cross Sections

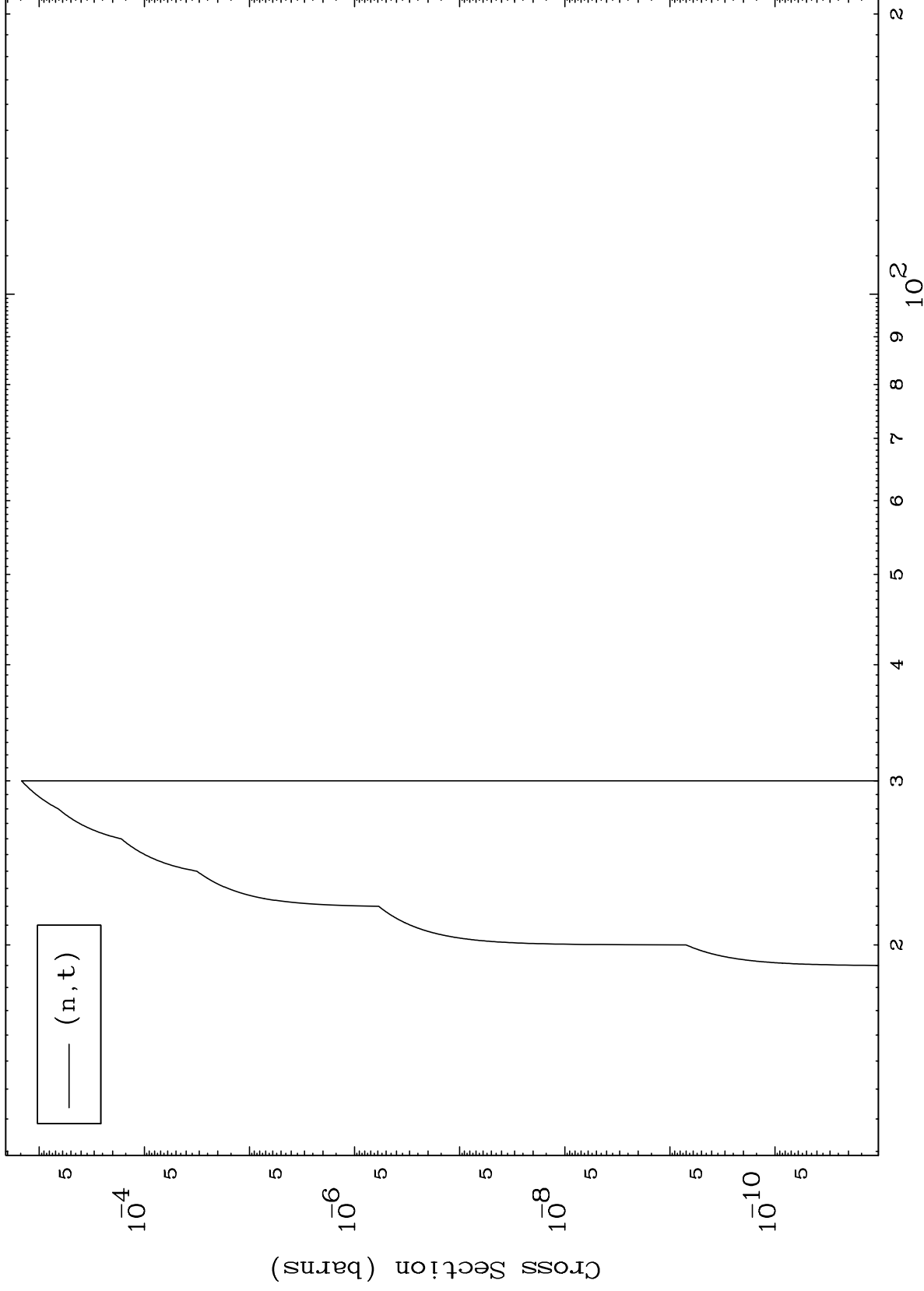


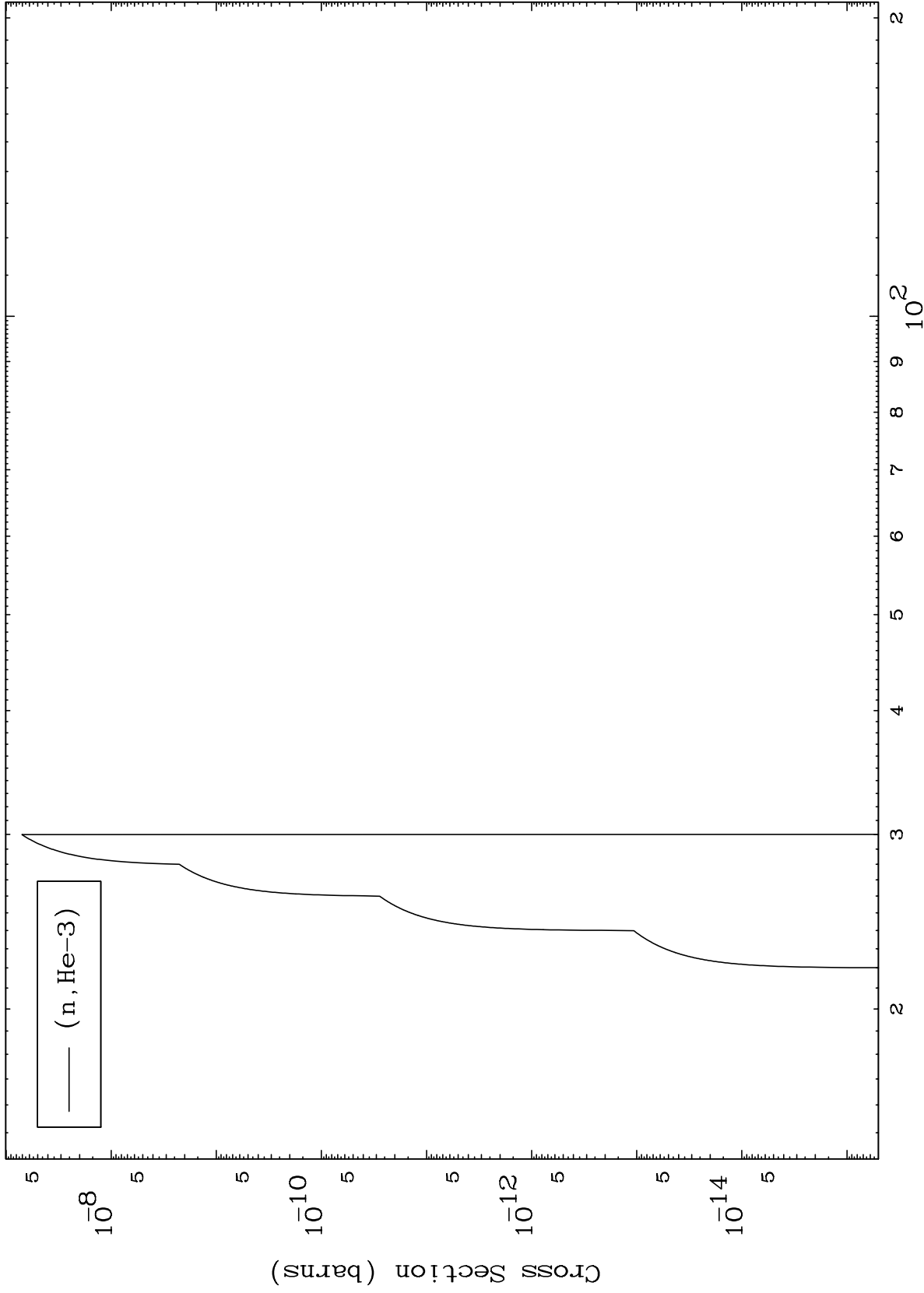
79-Au-199

Incident Energy (MeV)

5





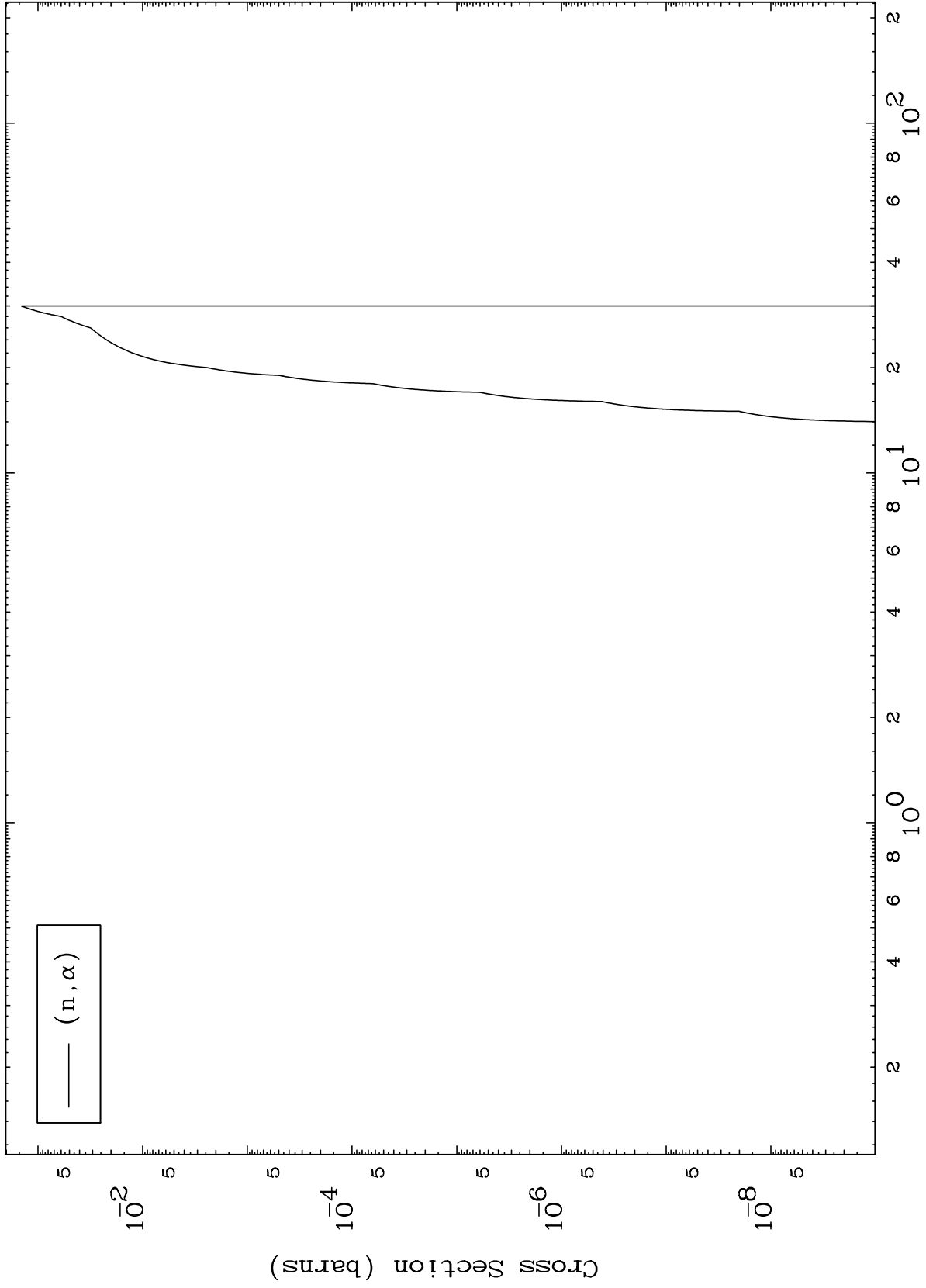


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(α, α) Levels

79-Au-199

0 Kelvin Cross Sections

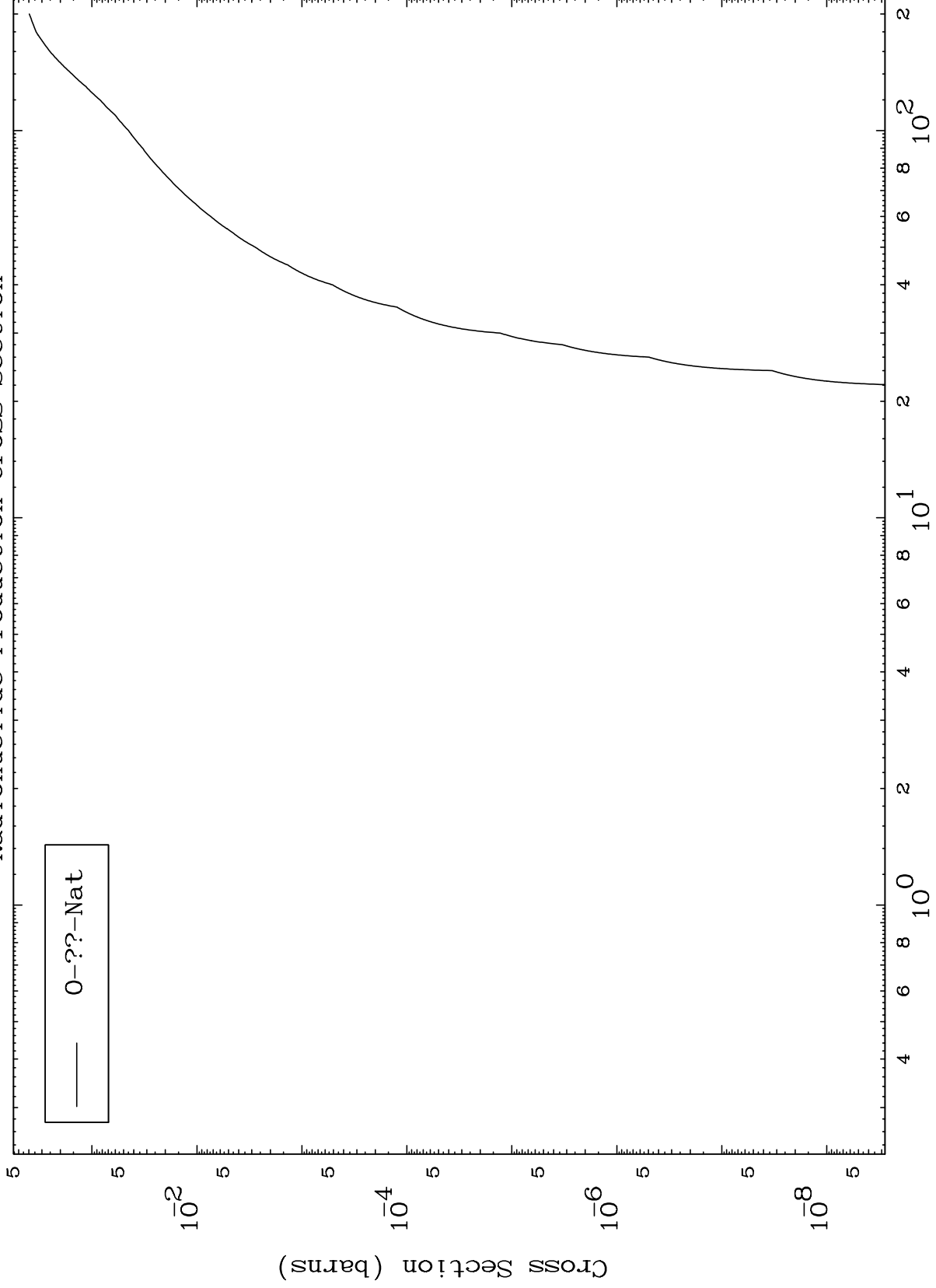


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Fission

79-Au-199

Radionuclide Production Cross Section



10

Incident Energy (MeV)

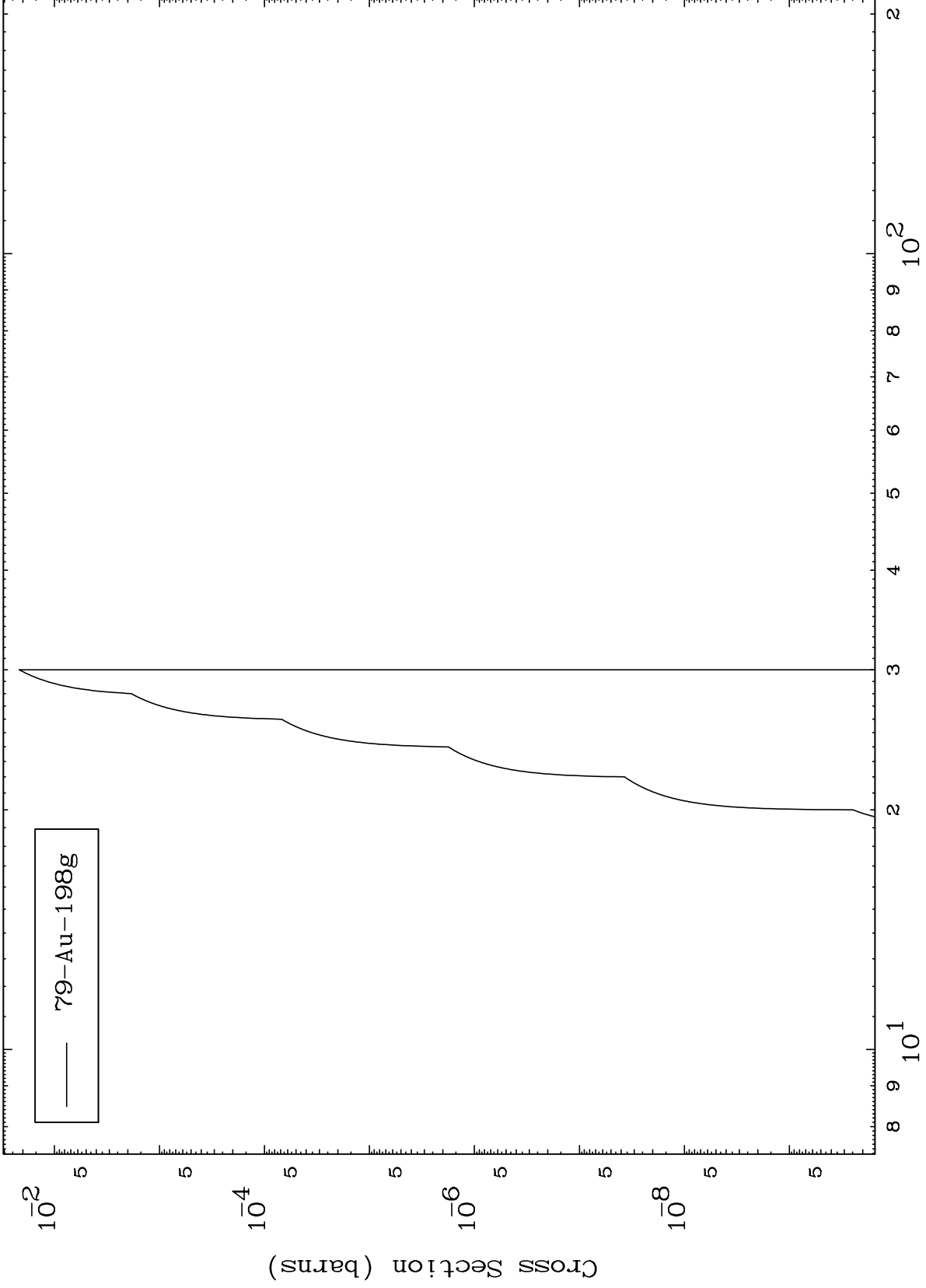
79-Au-199

MAT 7931

$(n, n') \alpha$

79-Au-199

Radionuclide Production Cross Section

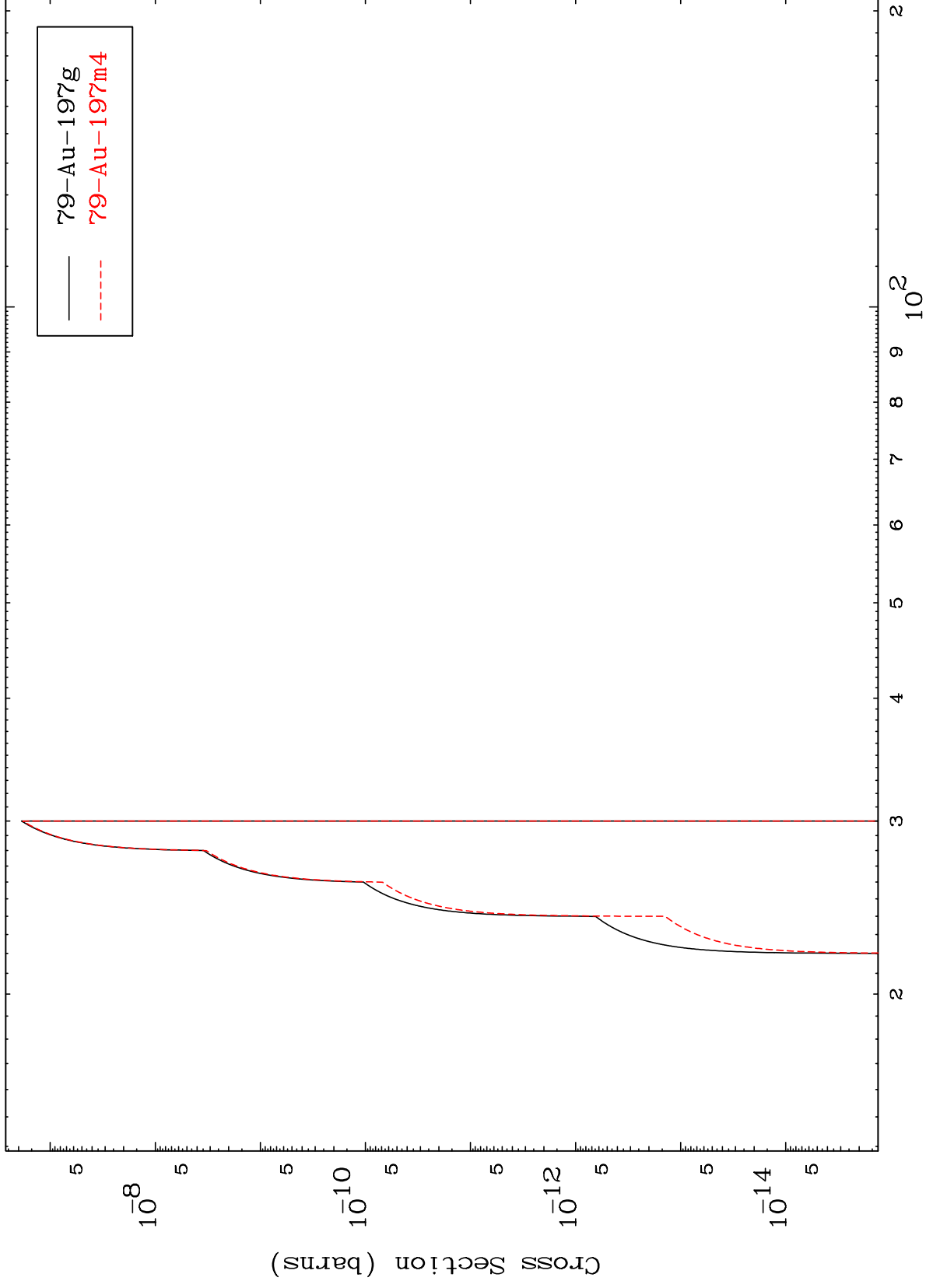


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$(n,2n) \alpha$

$^{79}\text{Au-199}$

Radionuclide Production Cross Section



12

Incident Energy (MeV)

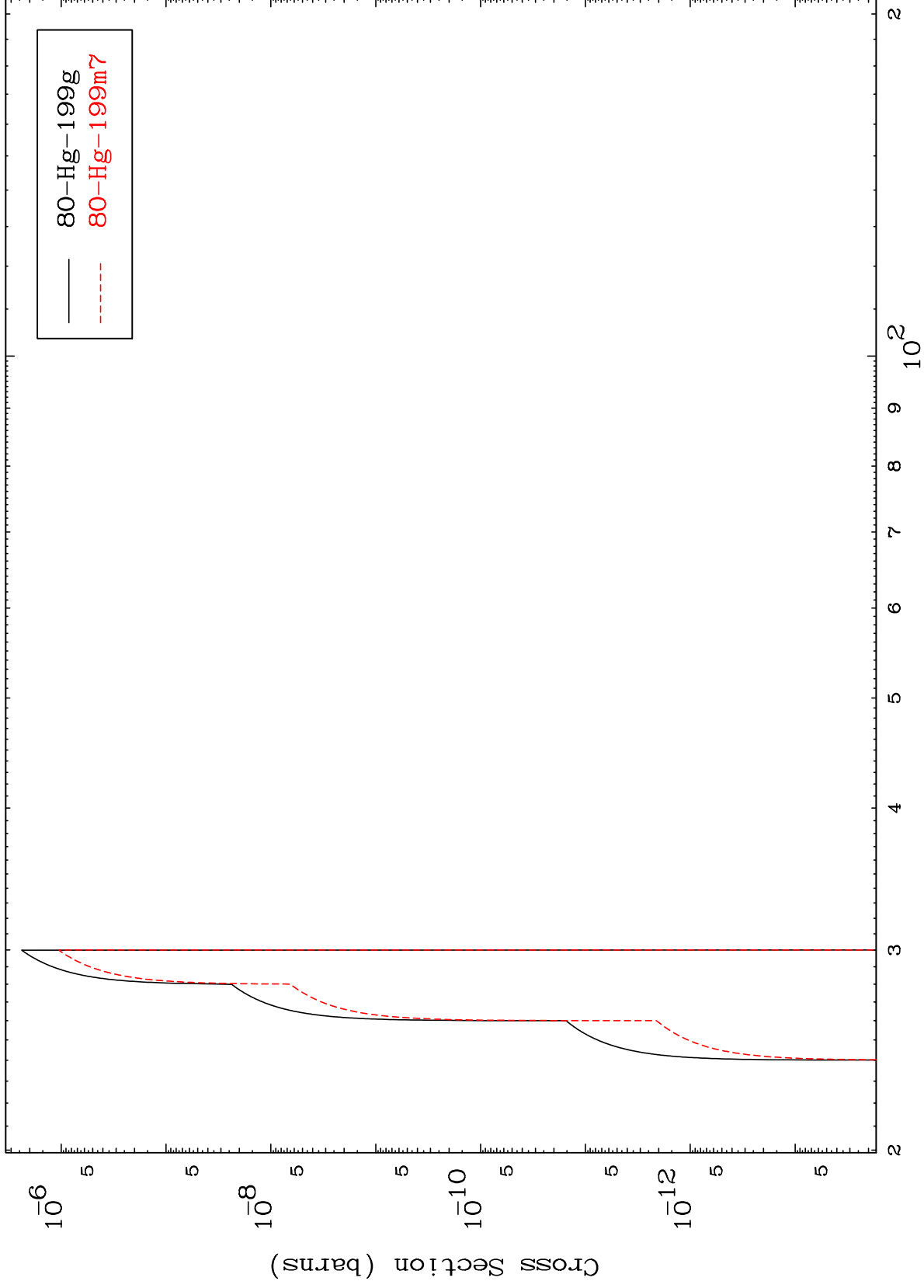
$^{79}\text{Au-199}$

MAT 7931

(n,n') t

79-Au-199

Radionuclide Production Cross Section



13

Incident Energy (MeV)

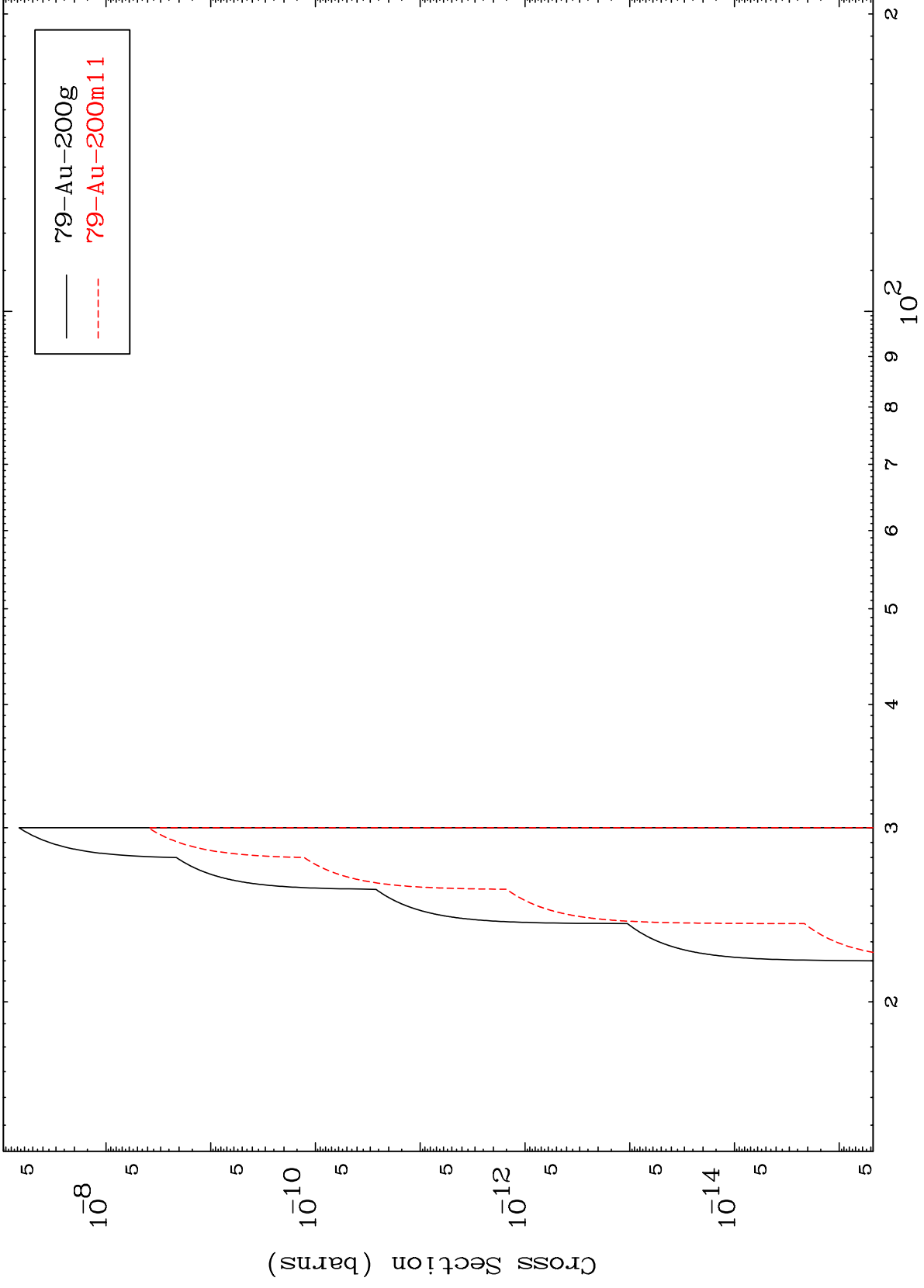
79-Au-199

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(n,He-3)

79-Au-199

Radionuclide Production Cross Section



14

Incident Energy (MeV)

79-Au-199