

Program EVALPLOT  
(Version 2021-1)

by

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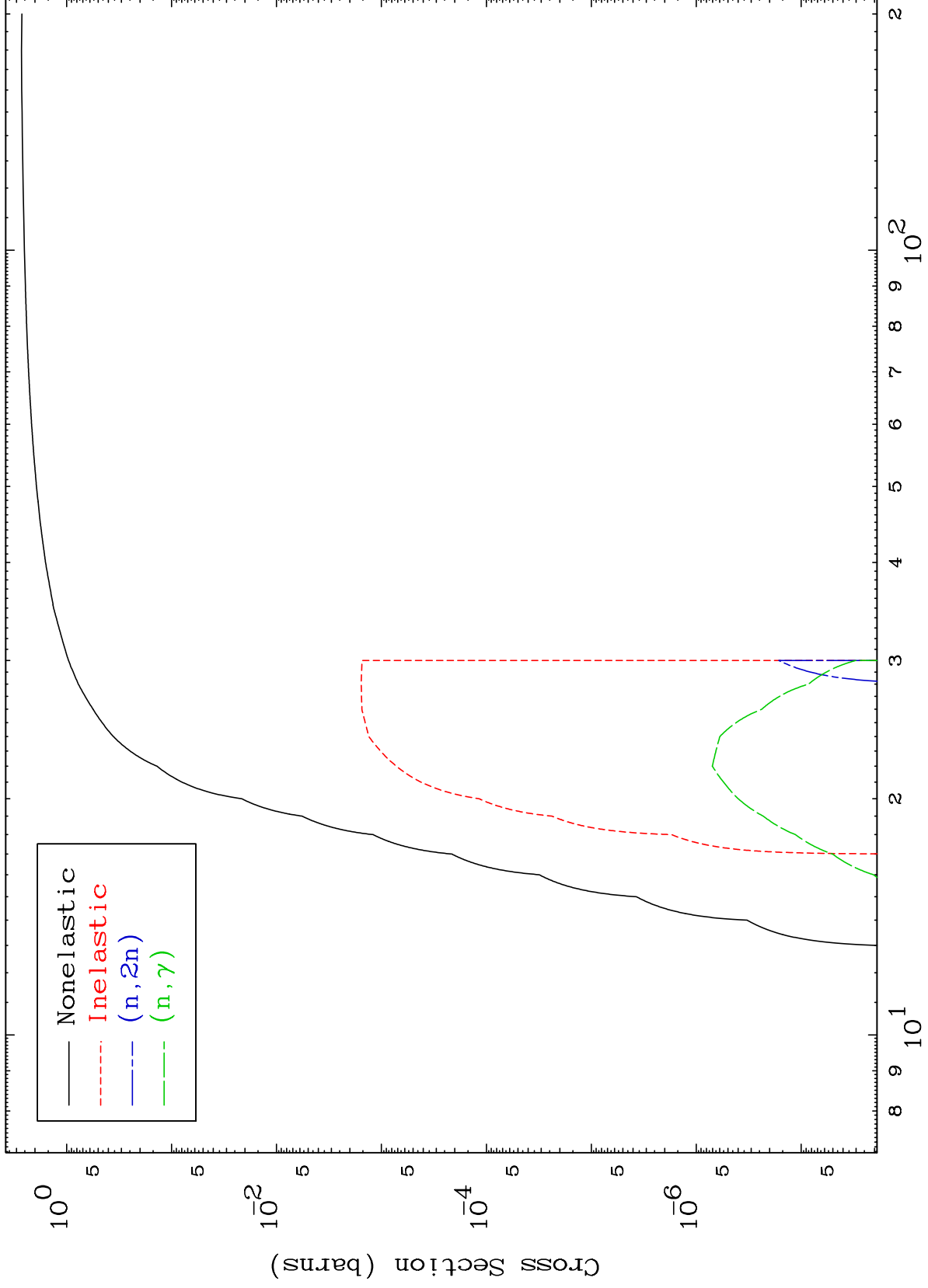
Press Mouse Button to Start

MAT 8269

0 Kelvin Cross Sections

$\alpha$  Major

$^{83}\text{Bi}$ -190m



1

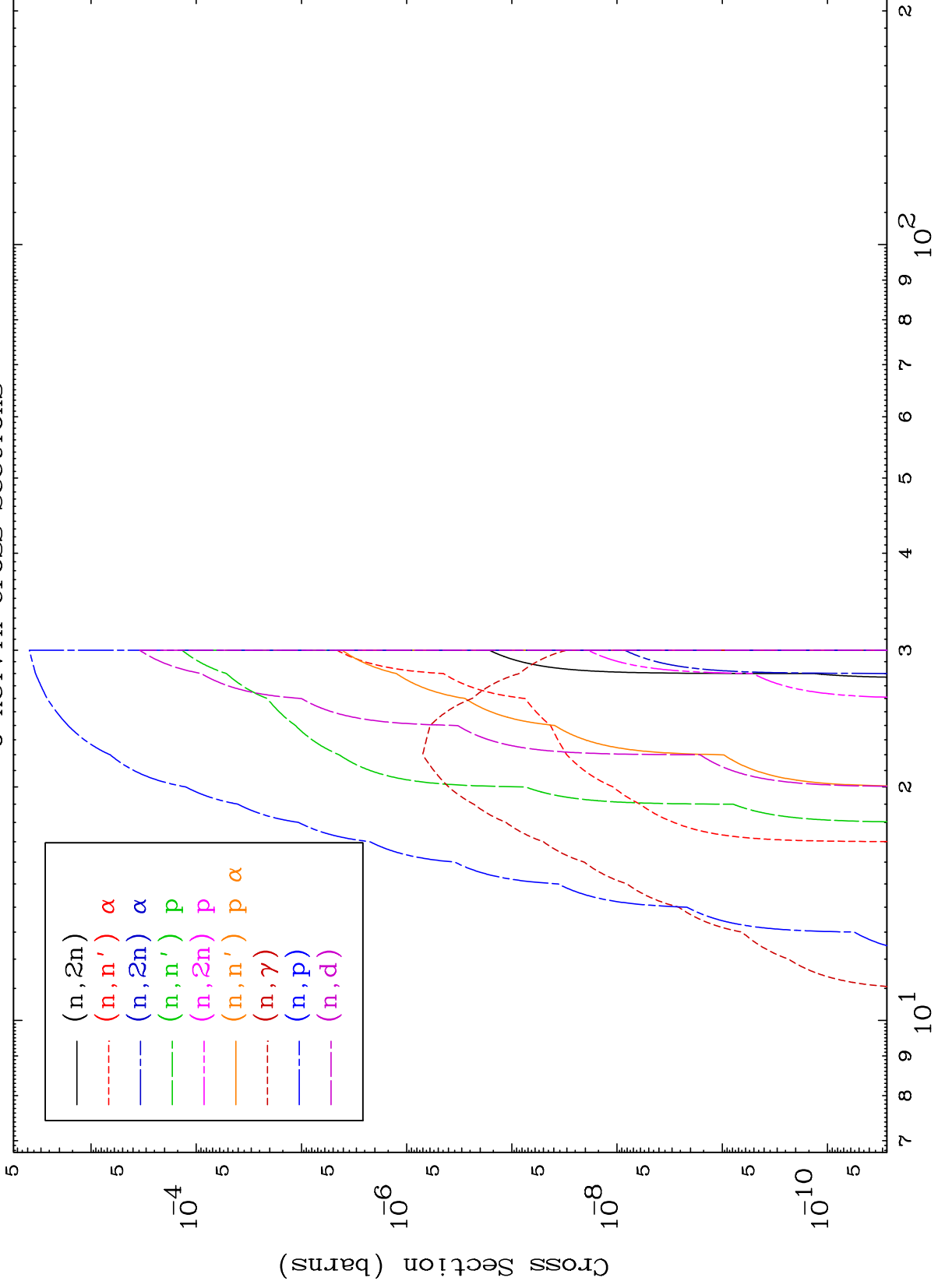
Incident Energy (MeV)

$^{83}\text{Bi}$ -190m

MAT 8269

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-190m



2

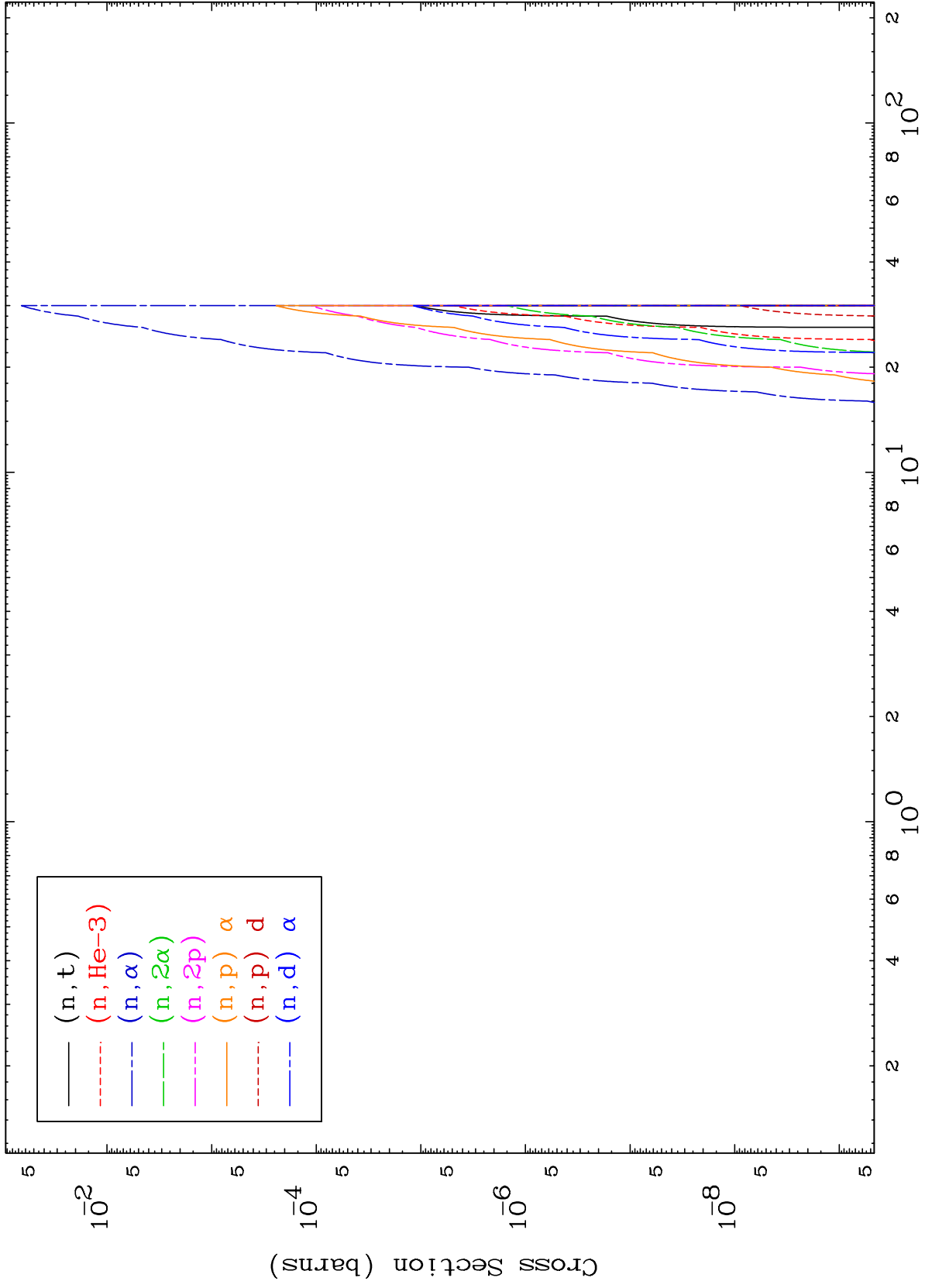
Incident Energy (MeV)

83-Bi-190m

MAT 8269

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-190m



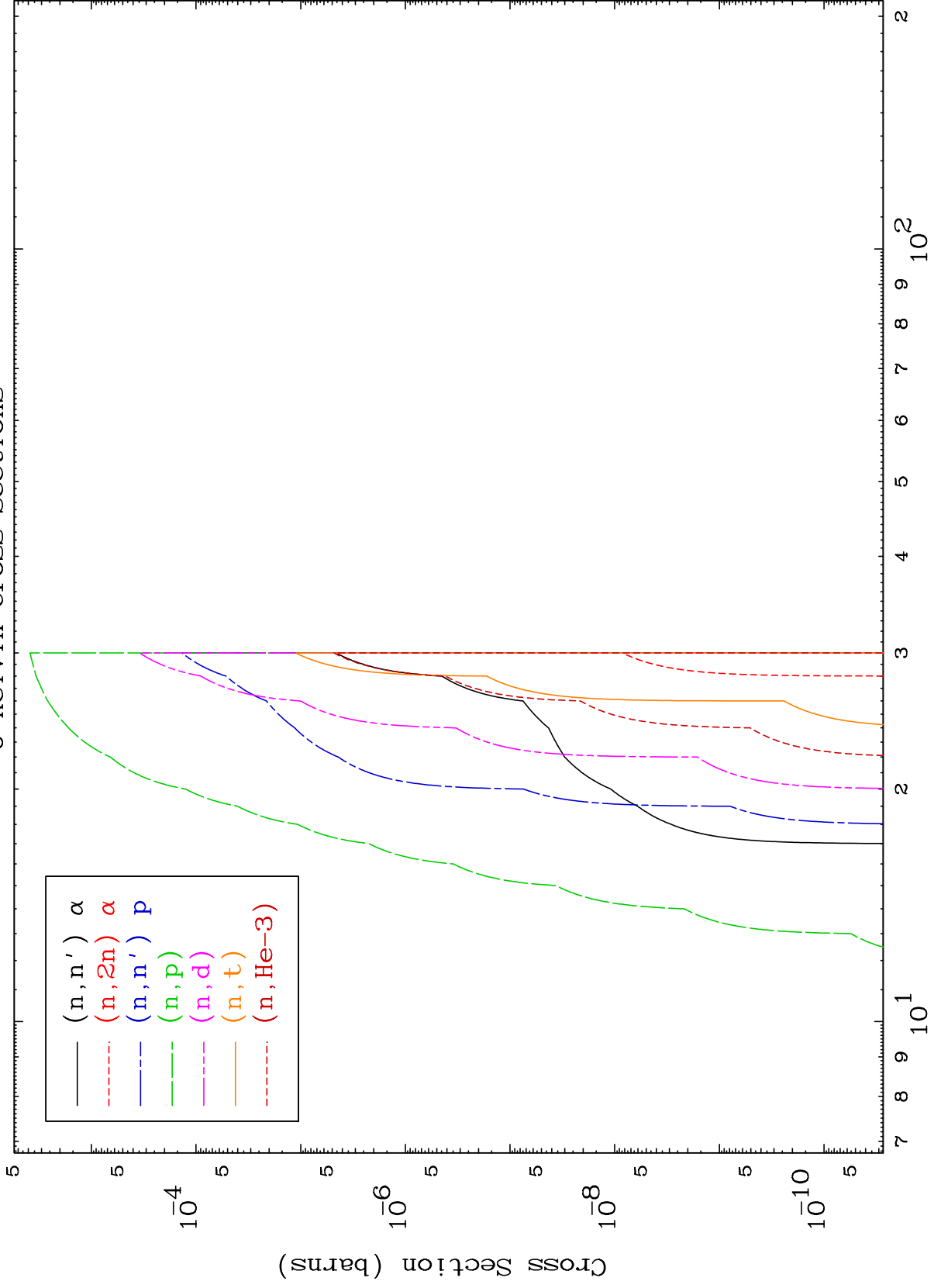
Incident Energy (MeV)

83-Bi-190m

MAT 8269

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-190m



4

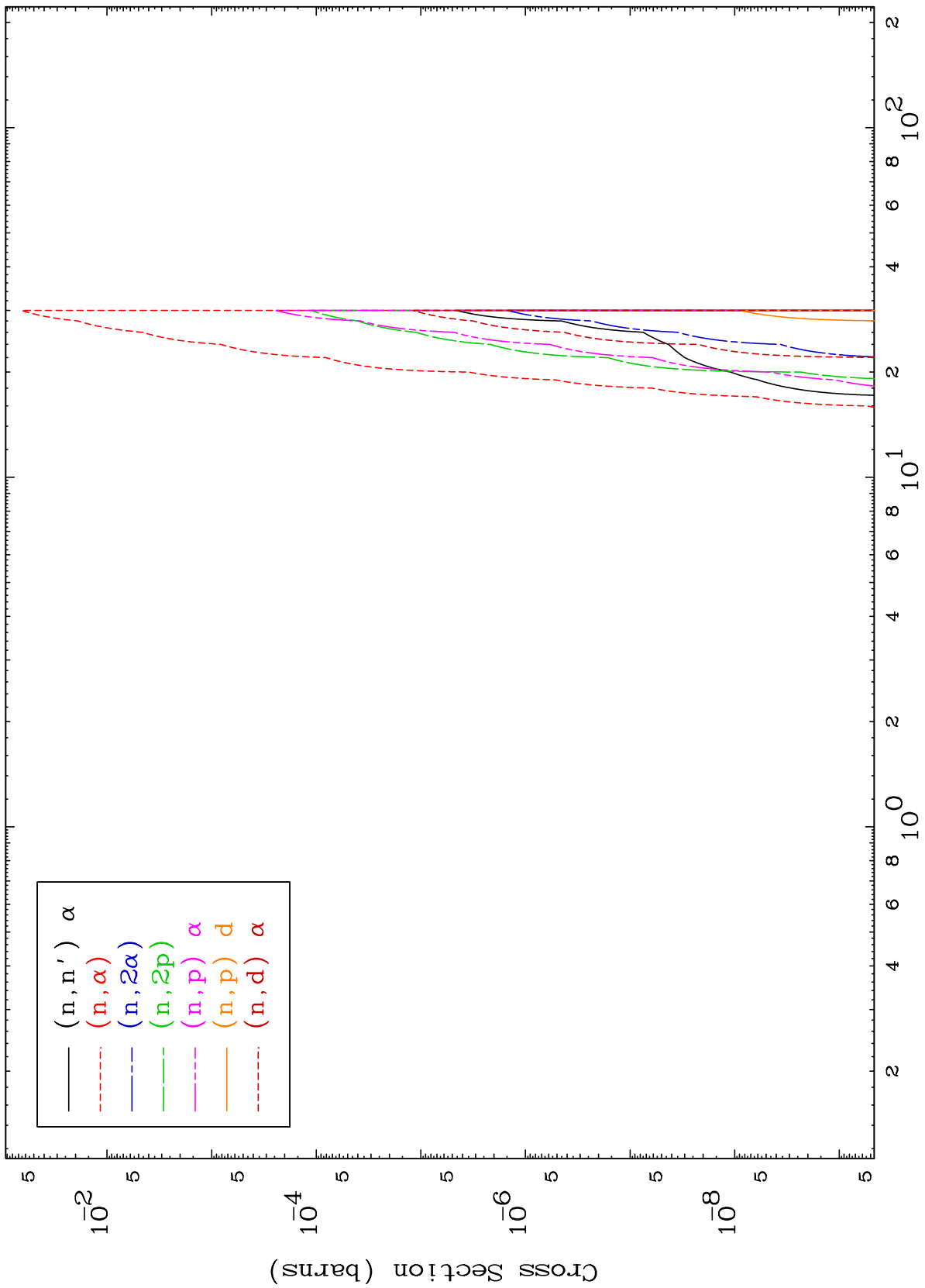
Incident Energy (MeV)

83-Bi-190m

MAT 8269

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

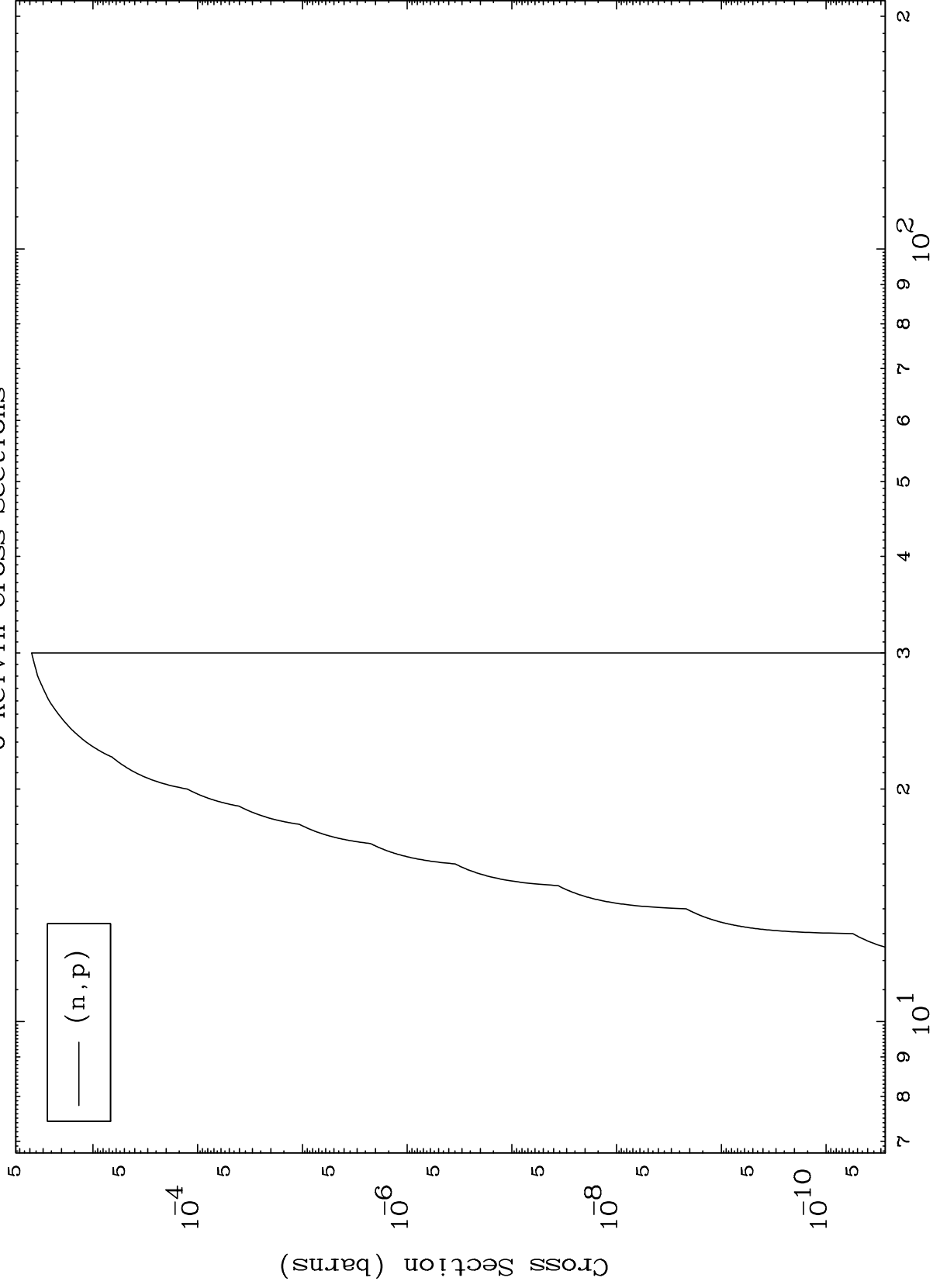
83-Bi-190m



MAT 8269

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

$^{83}\text{Bi}$ -190m



6

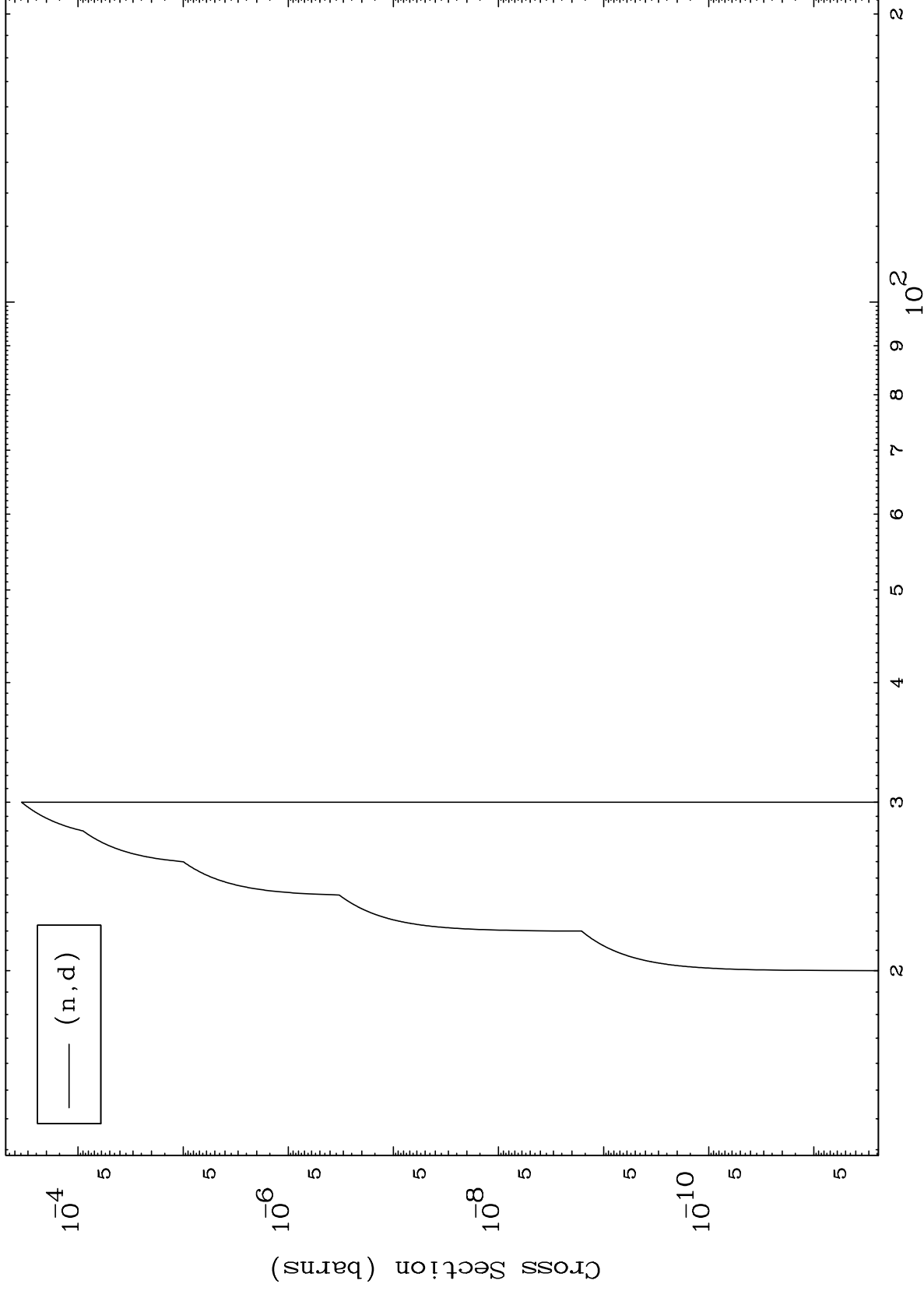
Incident Energy (MeV)

$^{83}\text{Bi}$ -190m

MAT 8269

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

$^{83}\text{Bi}-190\text{m}$

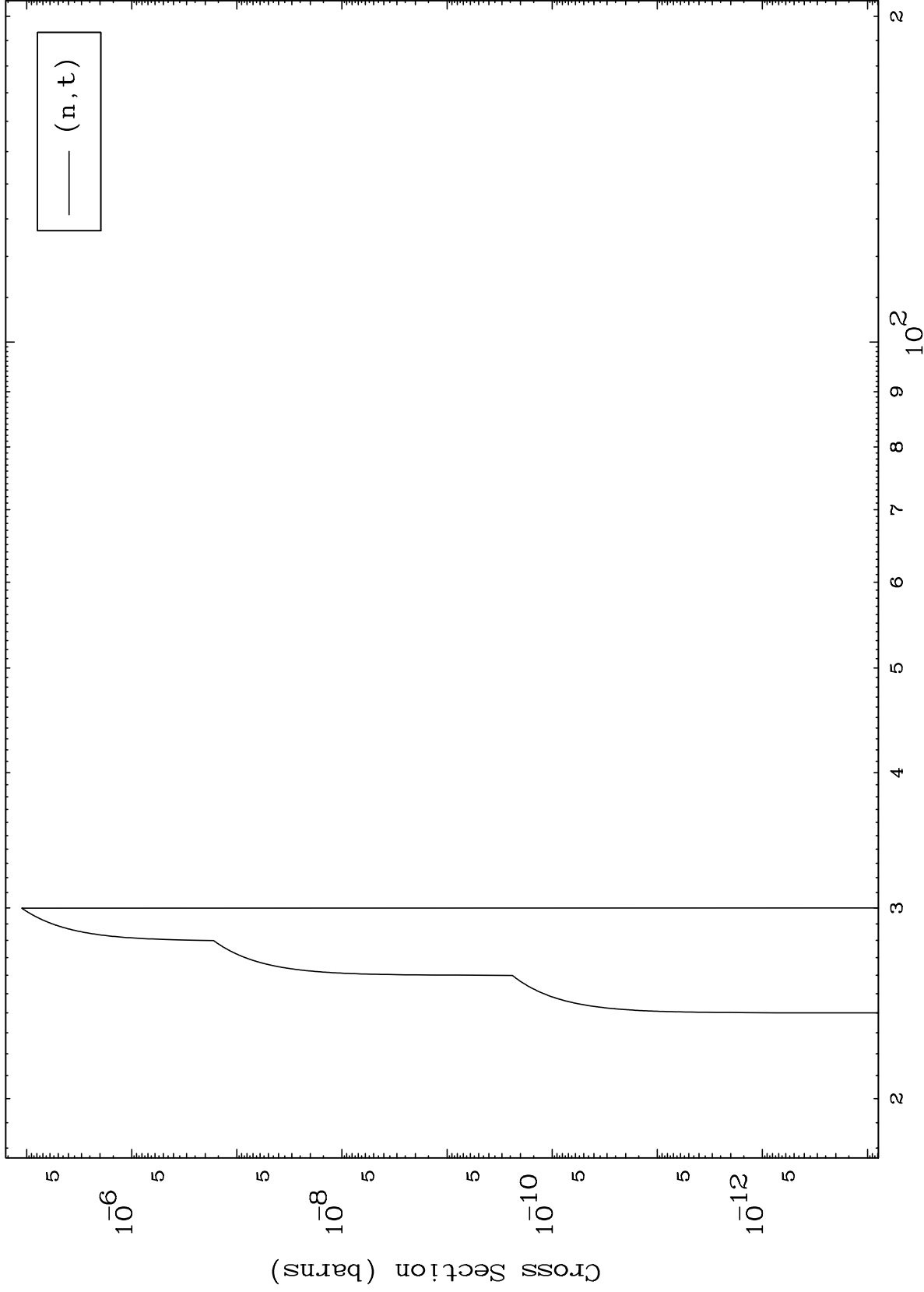


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Incident Energy (MeV)

$^{83}\text{Bi}-190\text{m}$

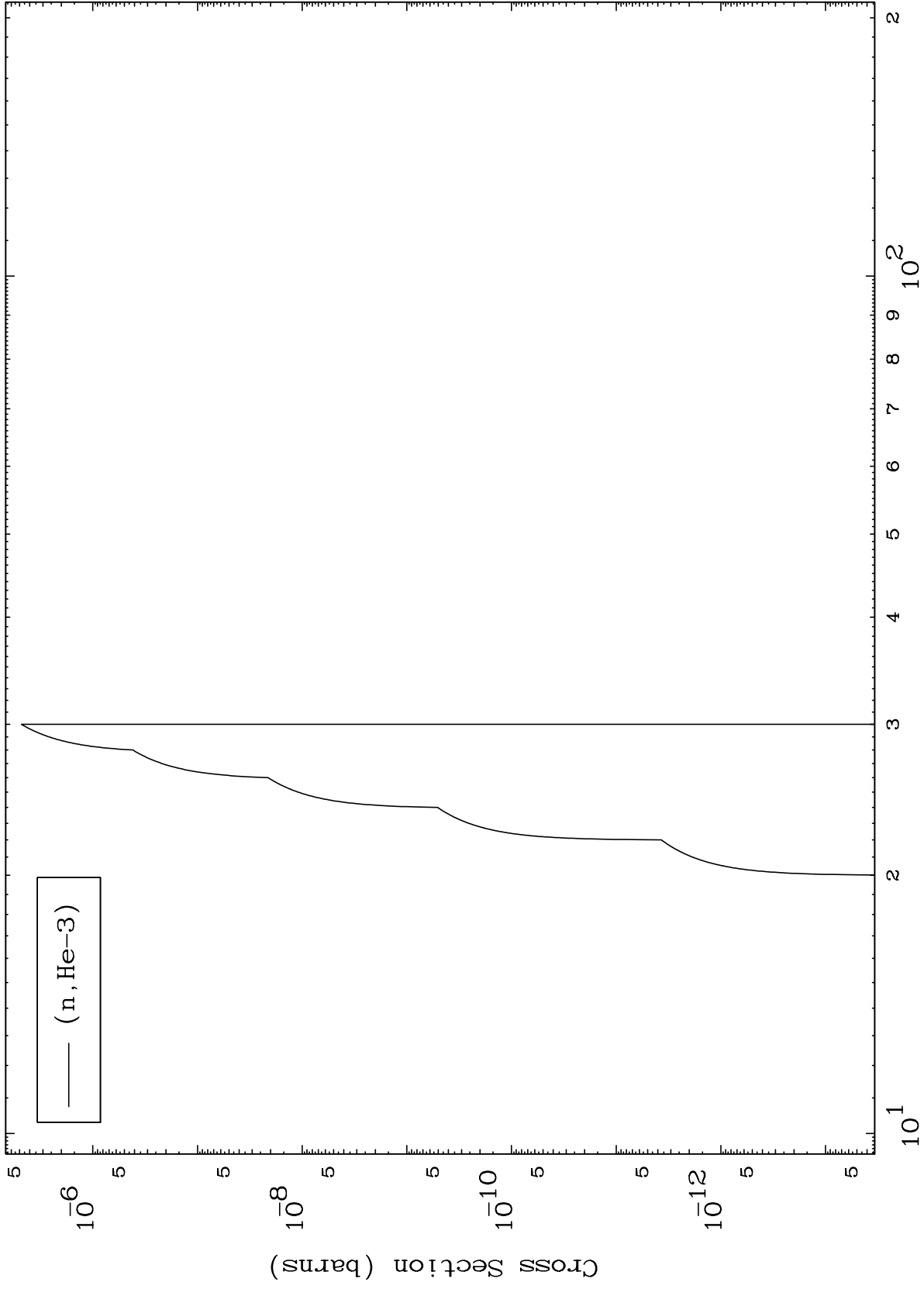




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( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

$^{83}\text{Bi}-190\text{m}$



Incident Energy (MeV)

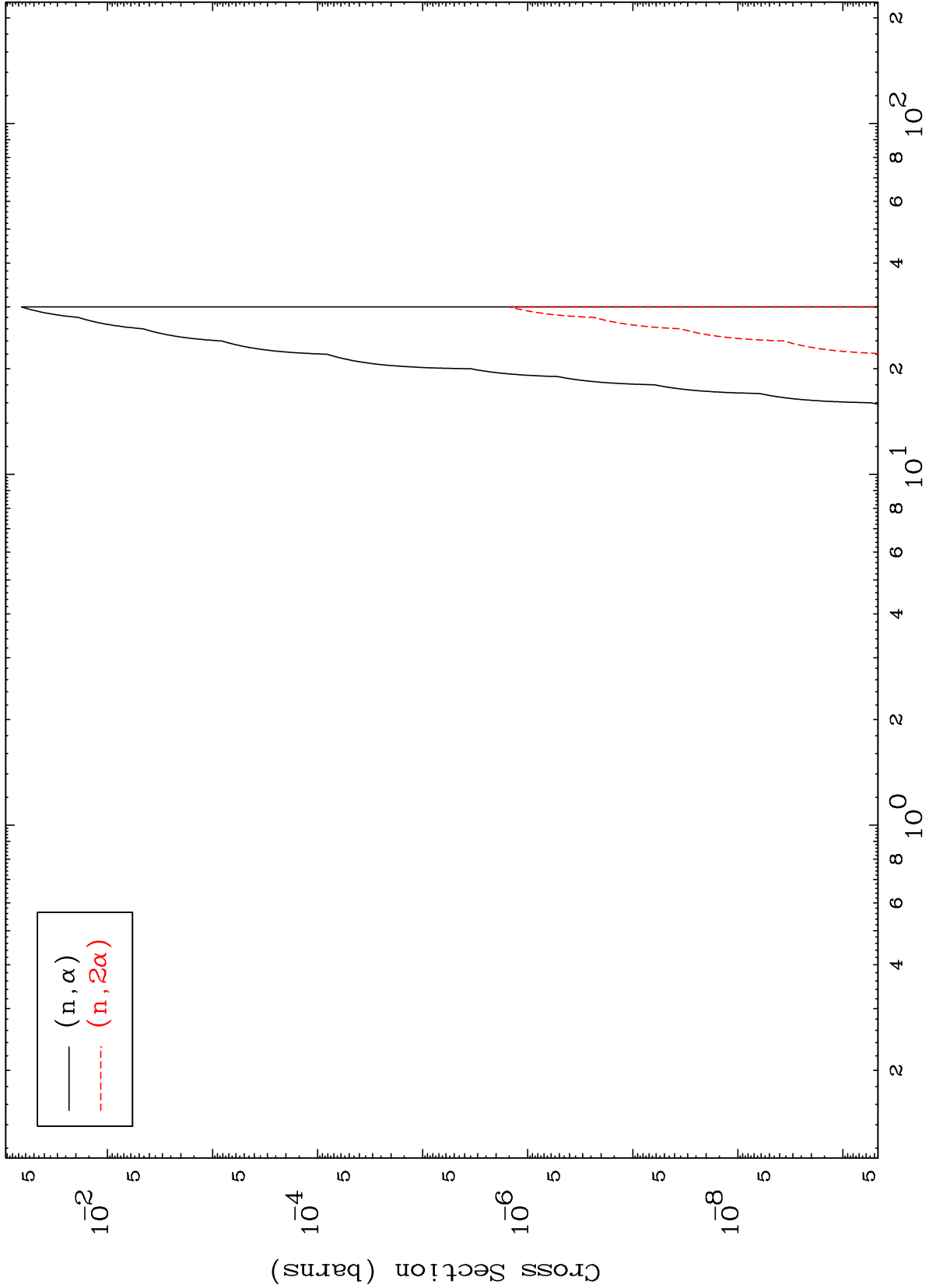
$^{83}\text{Bi}-190\text{m}$

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( $\alpha, \alpha$ ) Levels

$^{83}\text{Bi}-190\text{m}$

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

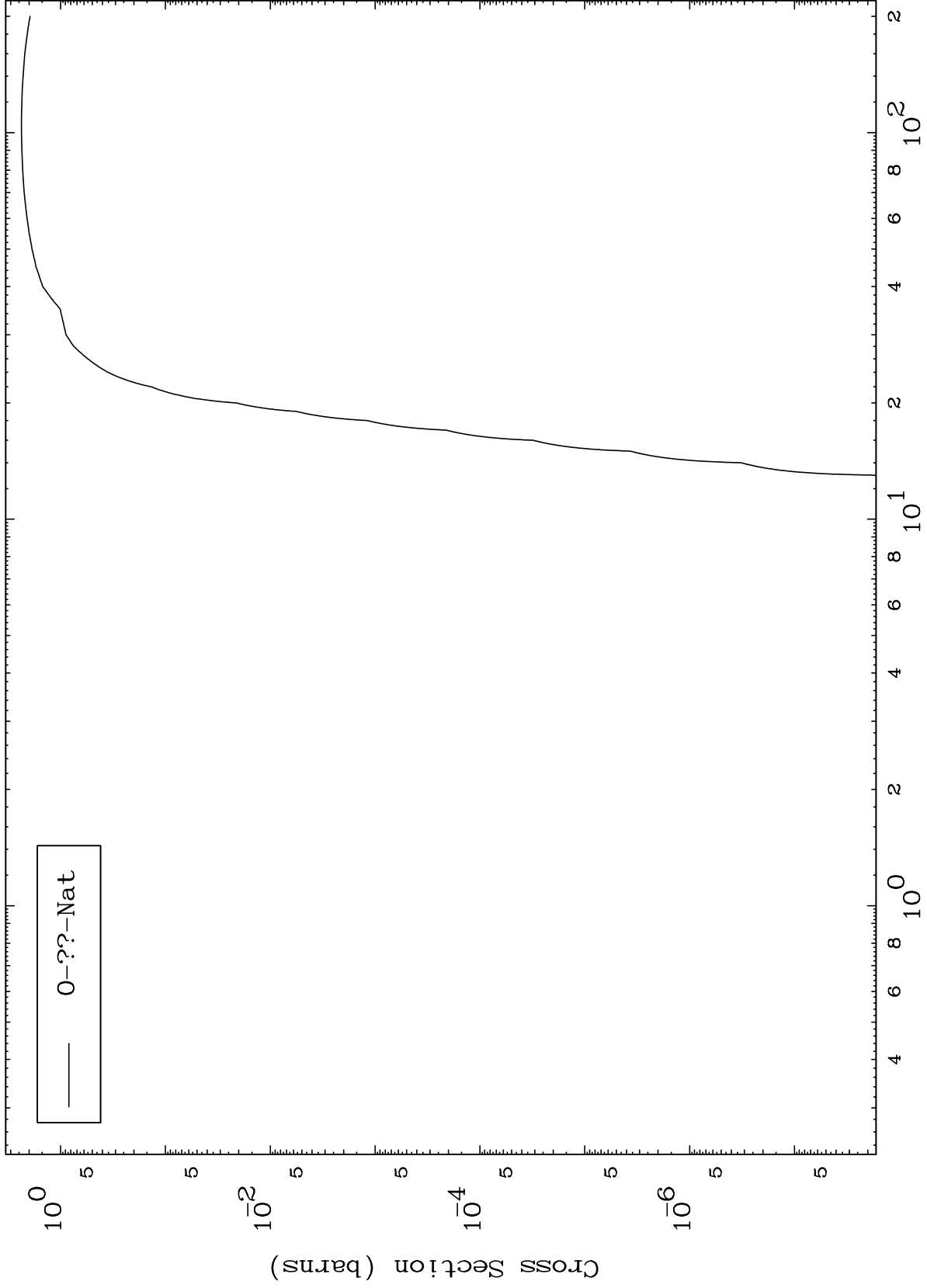
$^{83}\text{Bi}-190\text{m}$

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Fission

<sup>83</sup>Bi-190m

Radionuclide Production Cross Section

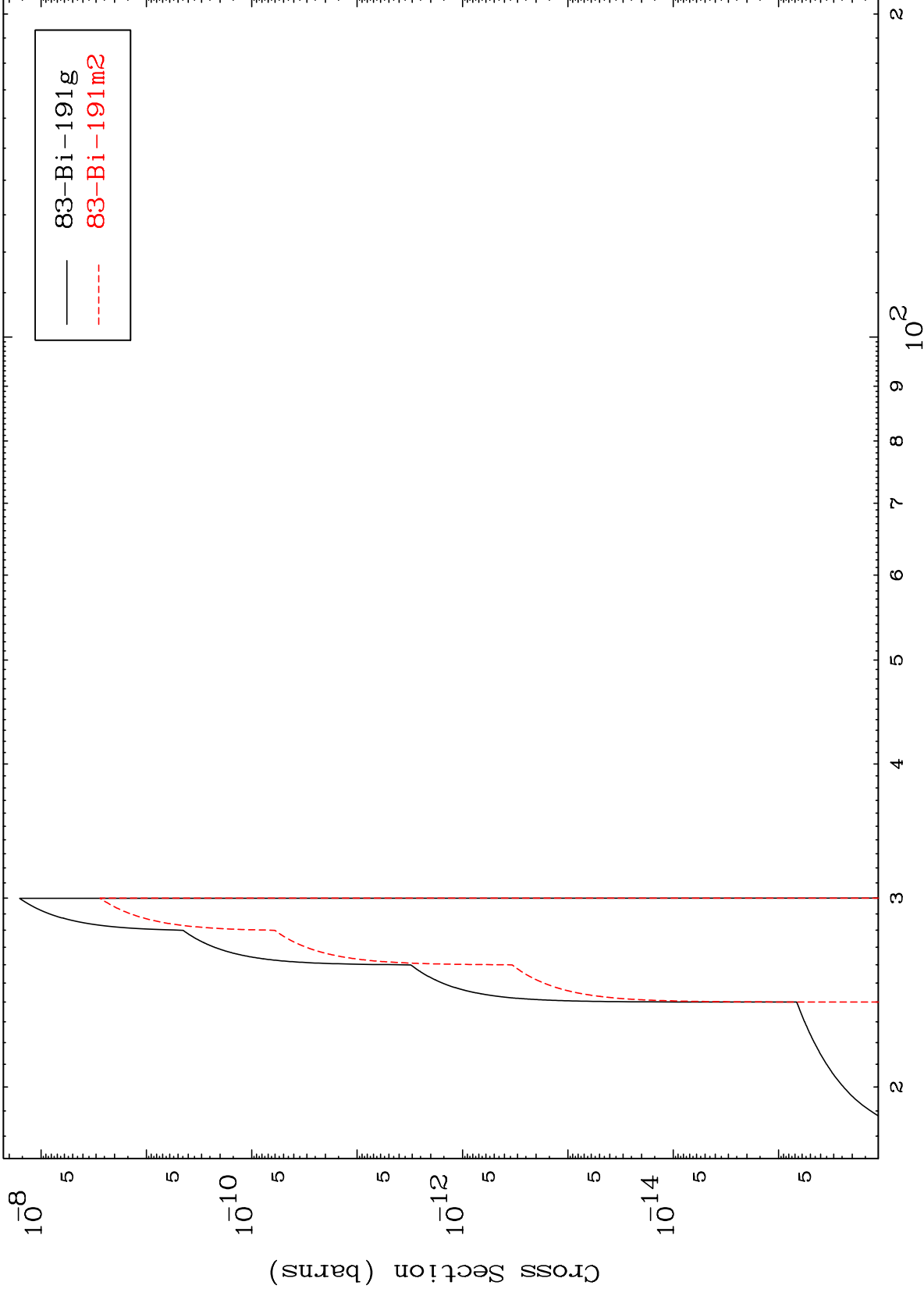


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(n,2n) p

83-Bi-190m

Radionuclide Production Cross Section



12

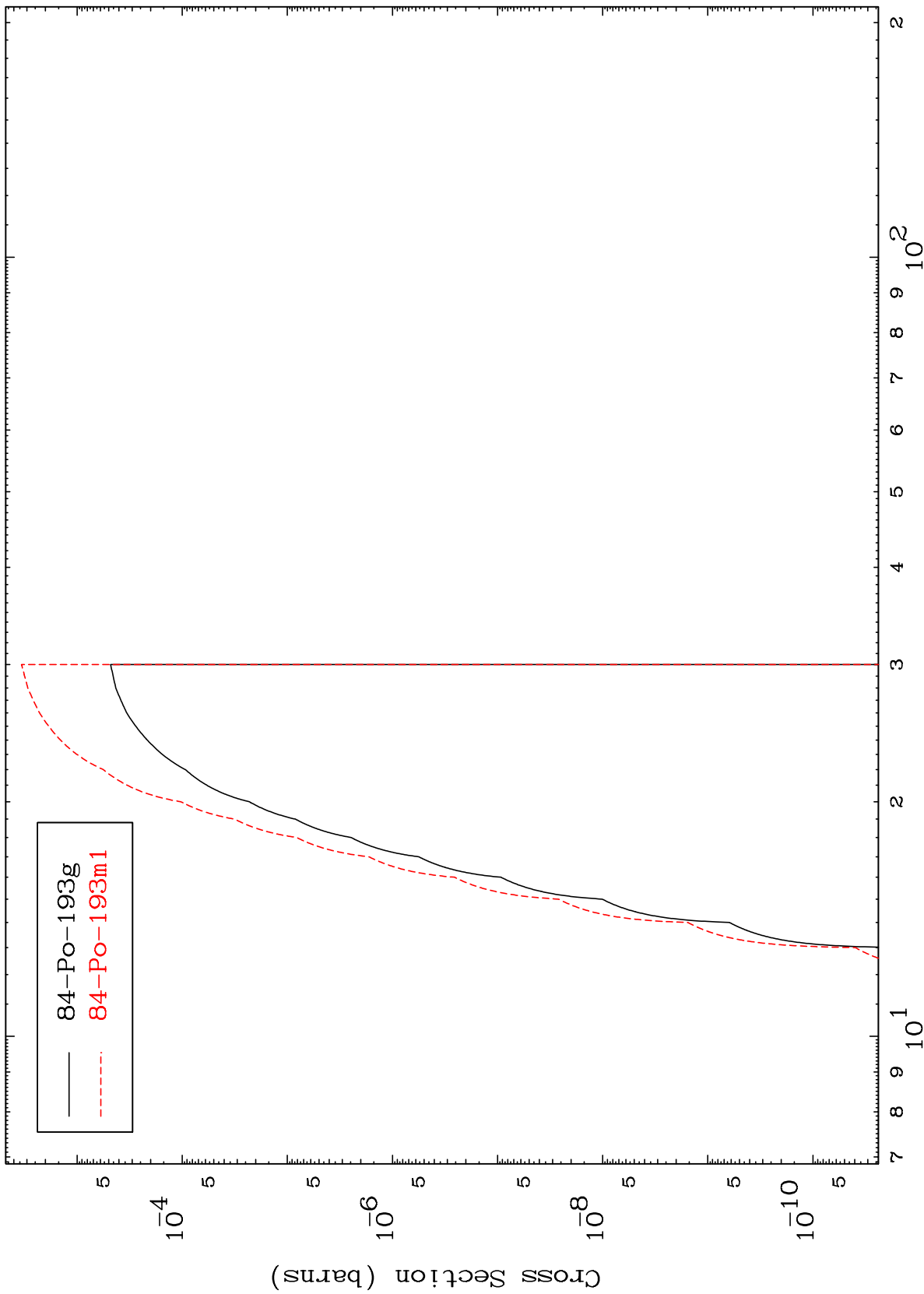
Incident Energy (MeV)

83-Bi-190m

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83-Bi-190m

(n,p)  
Radionuclide Production Cross Section



83-Bi-190m

Incident Energy (MeV)

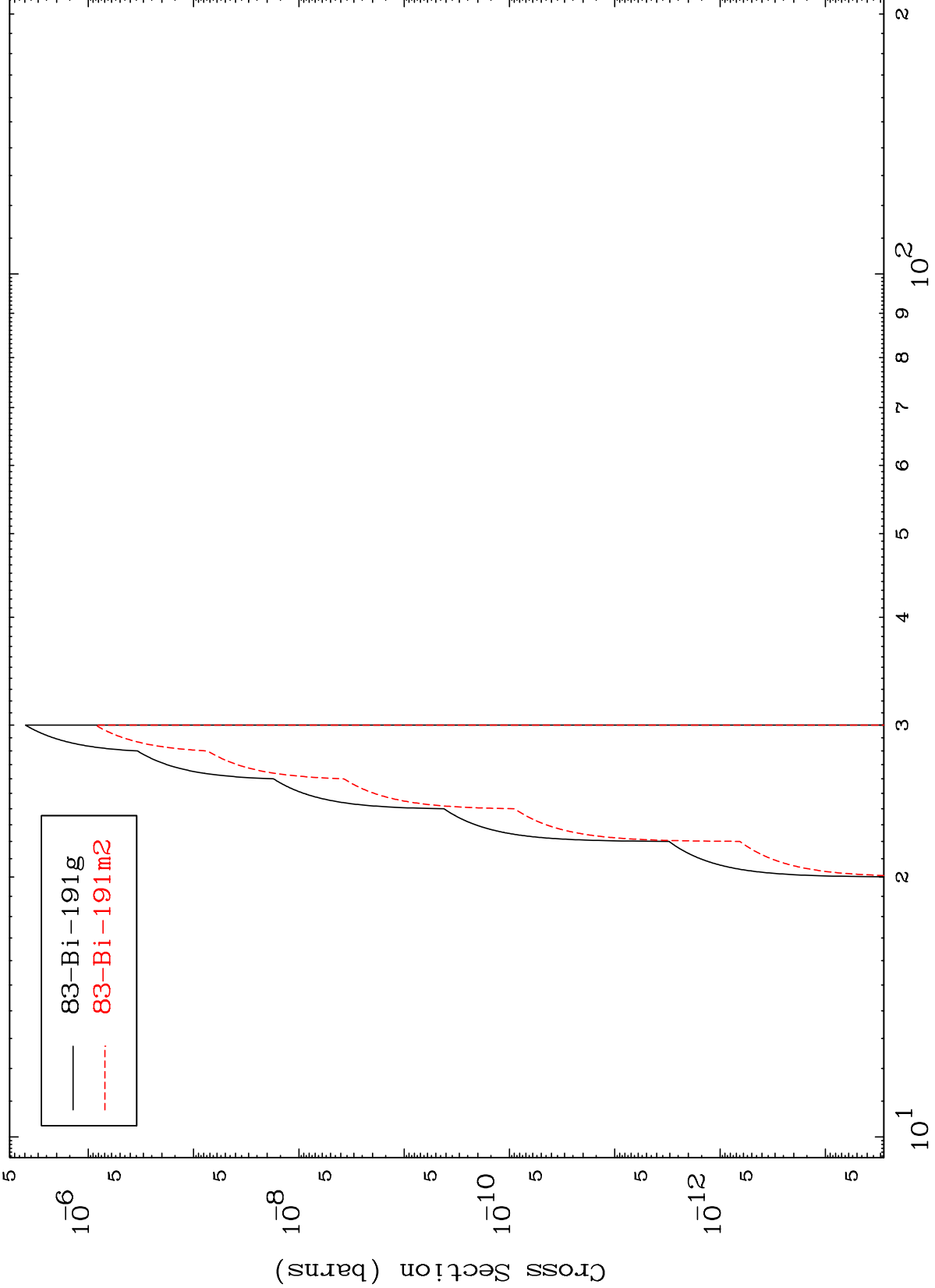
13

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(n,He-3)

83-Bi-190m

Radionuclide Production Cross Section



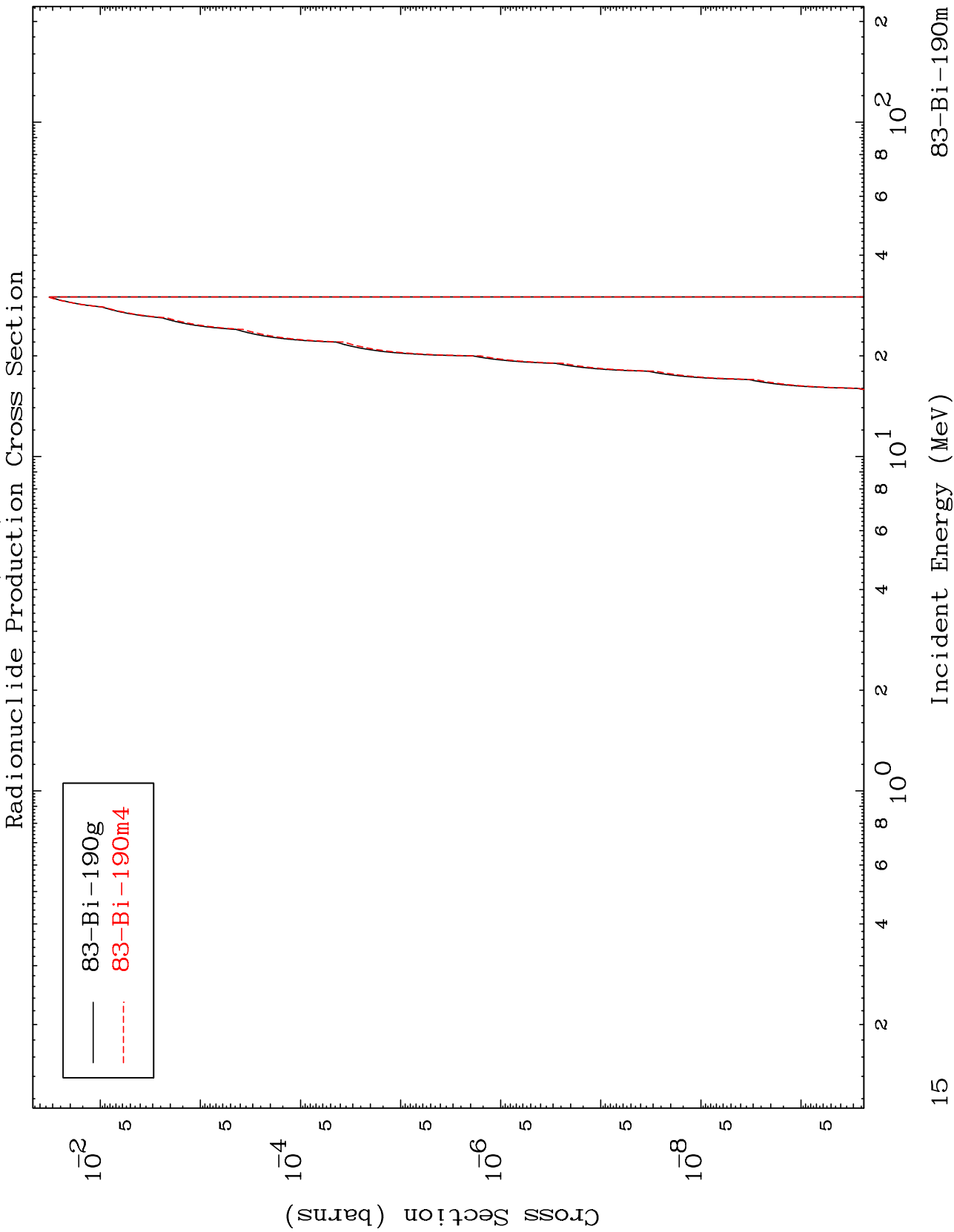
83-Bi-191g  
83-Bi-191m2

Incident Energy (MeV)

83-Bi-190m

MAT 8269

<sup>83</sup>Bi-190m

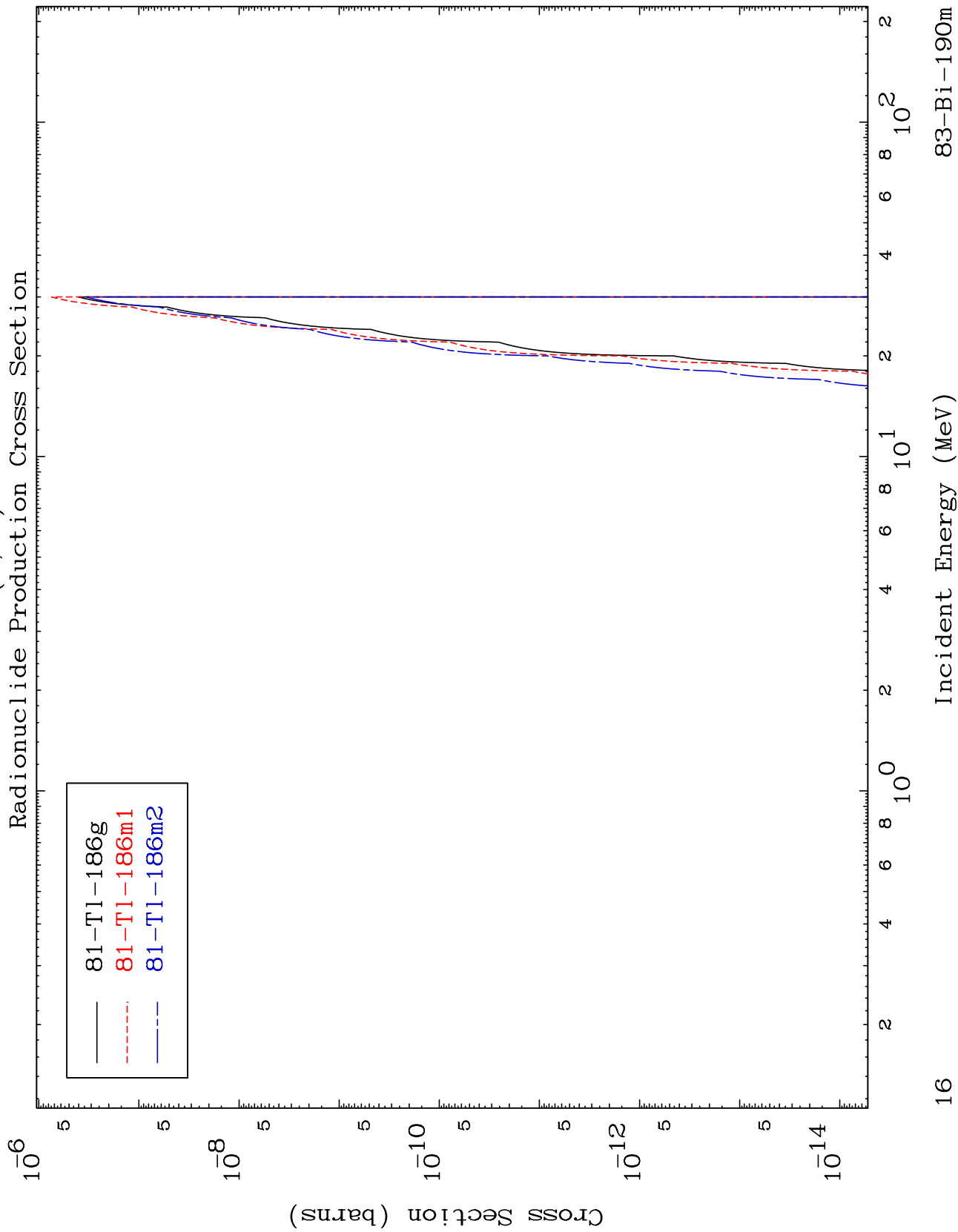


— 83-Bi-190g  
- - - 83-Bi-190m4



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$^{83}\text{Bi}-190\text{m}$

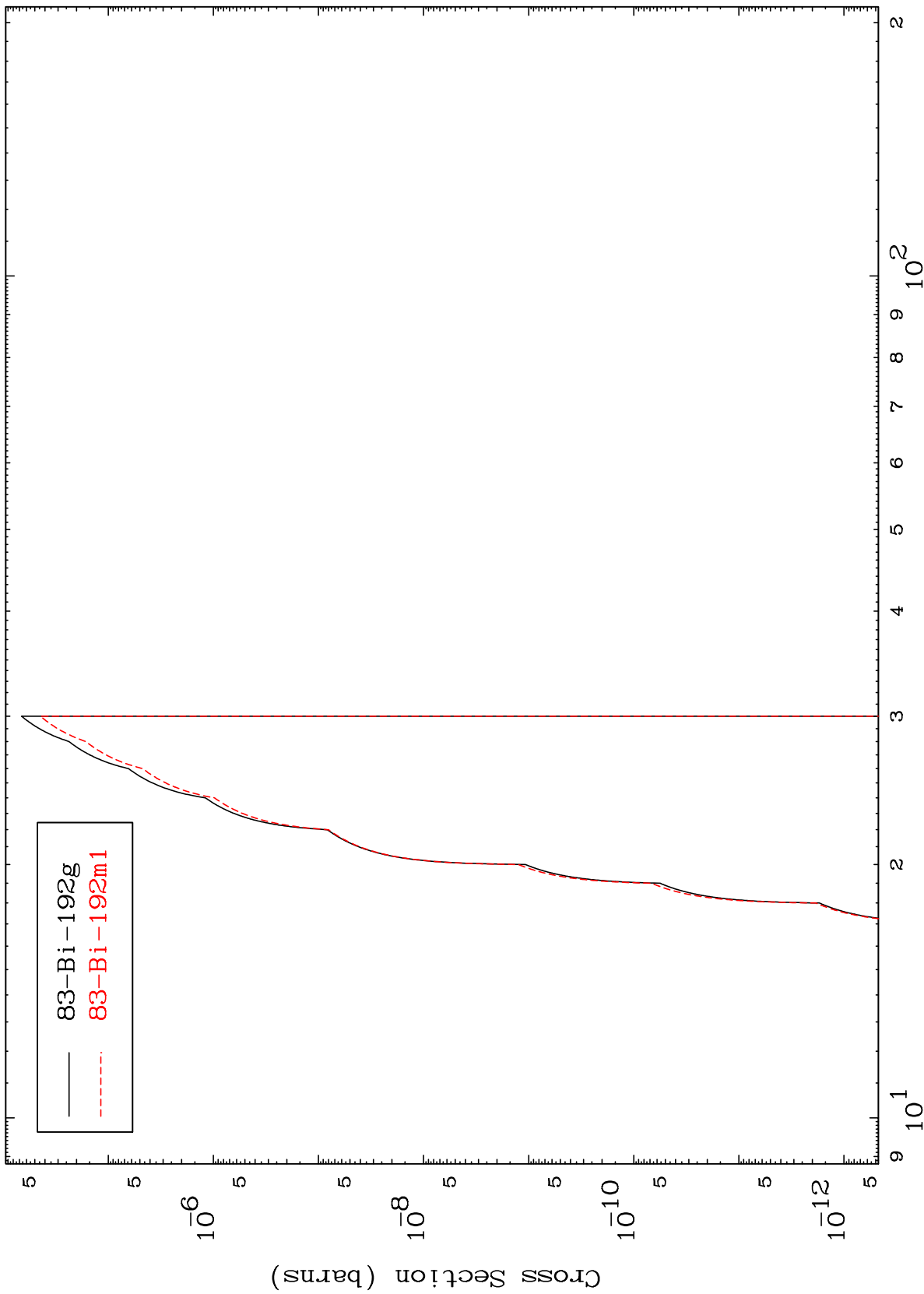


81-Tl-186g  
81-Tl-186m1  
81-Tl-186m2

MAT 8269

<sup>83</sup>Bi-190m

(n,2p)  
Radionuclide Production Cross Section



<sup>83</sup>Bi-190m

Incident Energy (MeV)

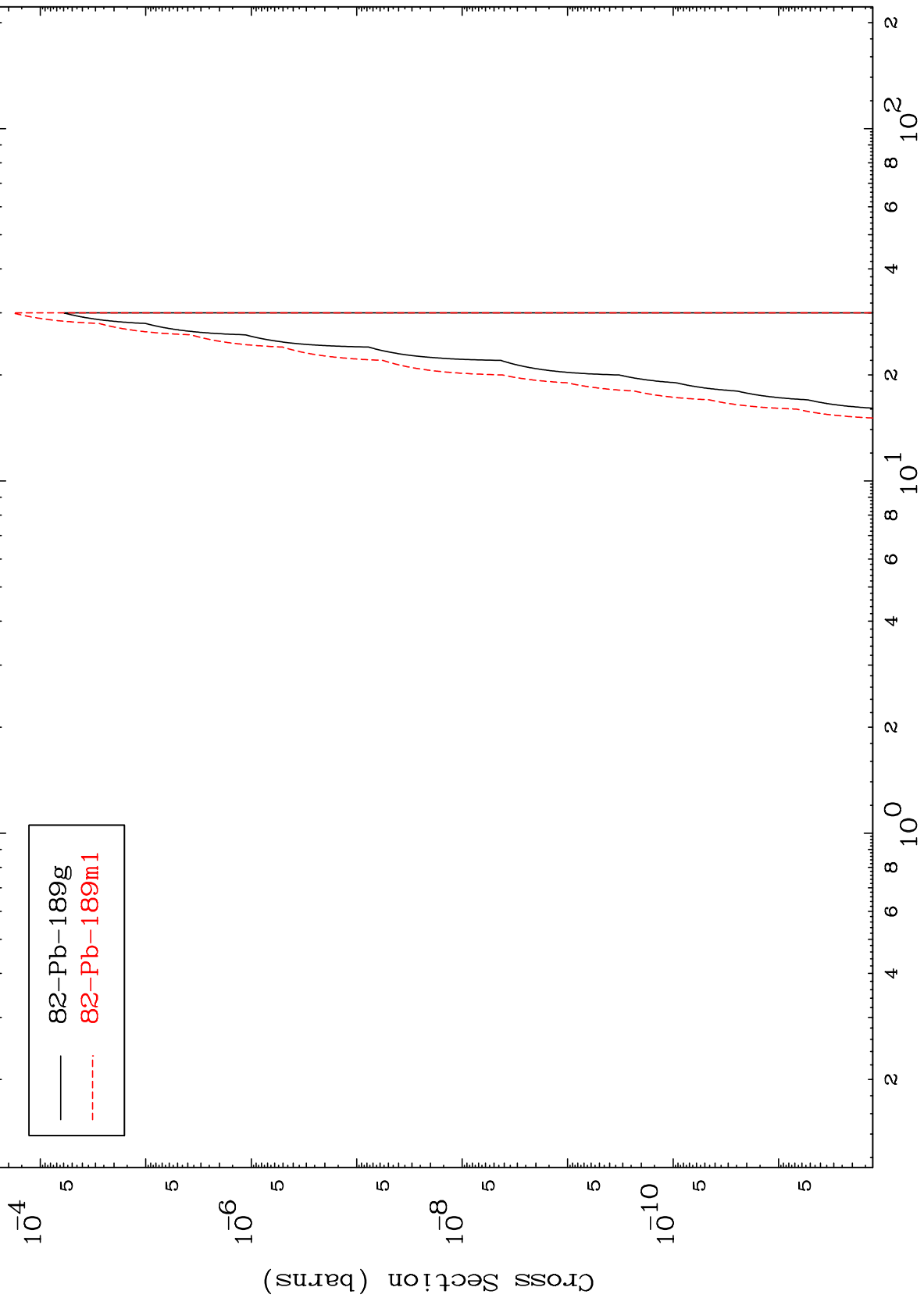
17

MAT 8269

(n,p)  $\alpha$

83-Bi-190m

Radionuclide Production Cross Section

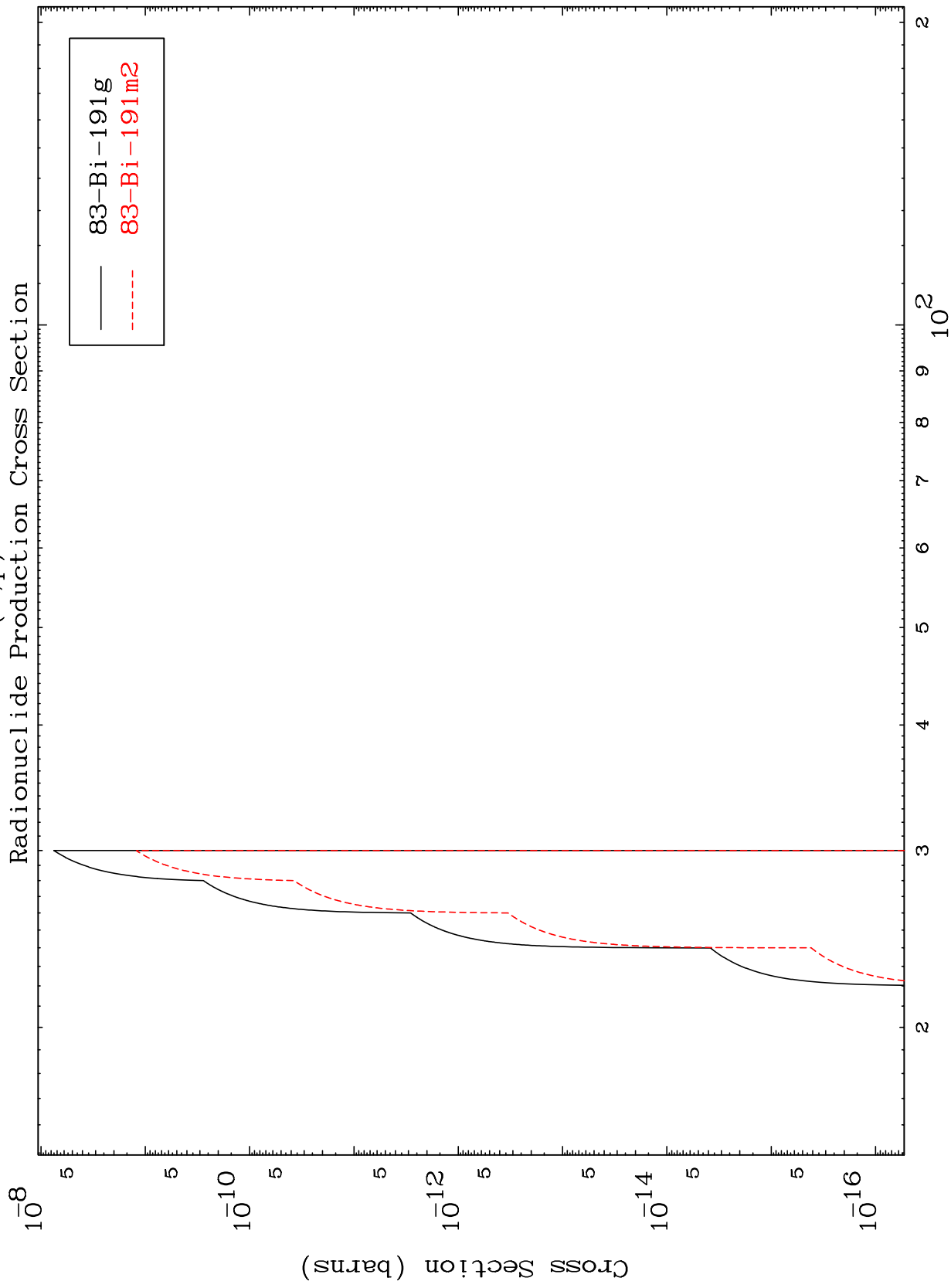


82-Pb-189g  
82-Pb-189m1

MAT 8269

(n,p) d

83-Bi-190m



19

Incident Energy (MeV)

83-Bi-190m