

Program EVALPLOT
(Version 2021-1)

by

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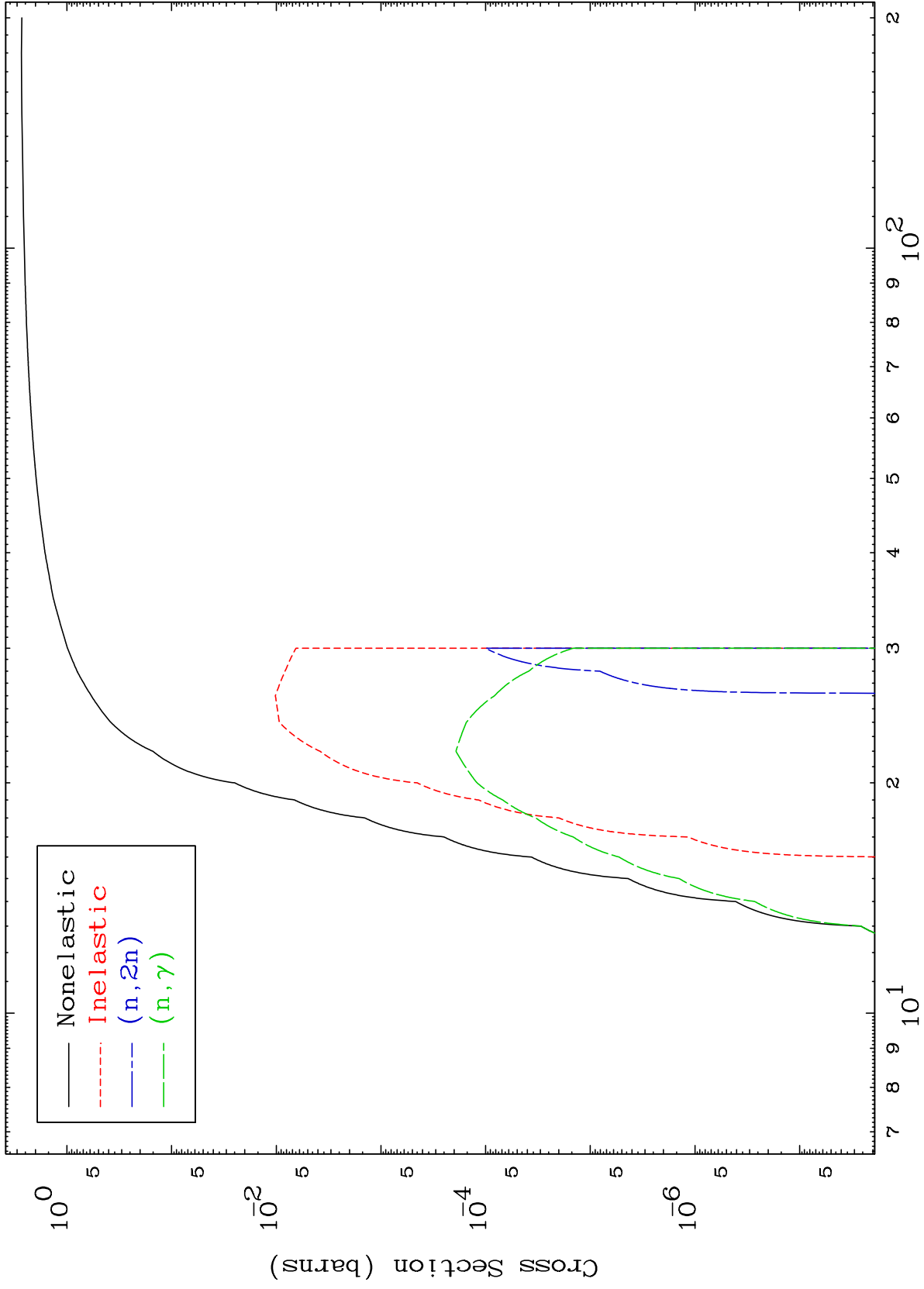
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8282

0 Kelvin Cross Sections

83-Bi-194n



1

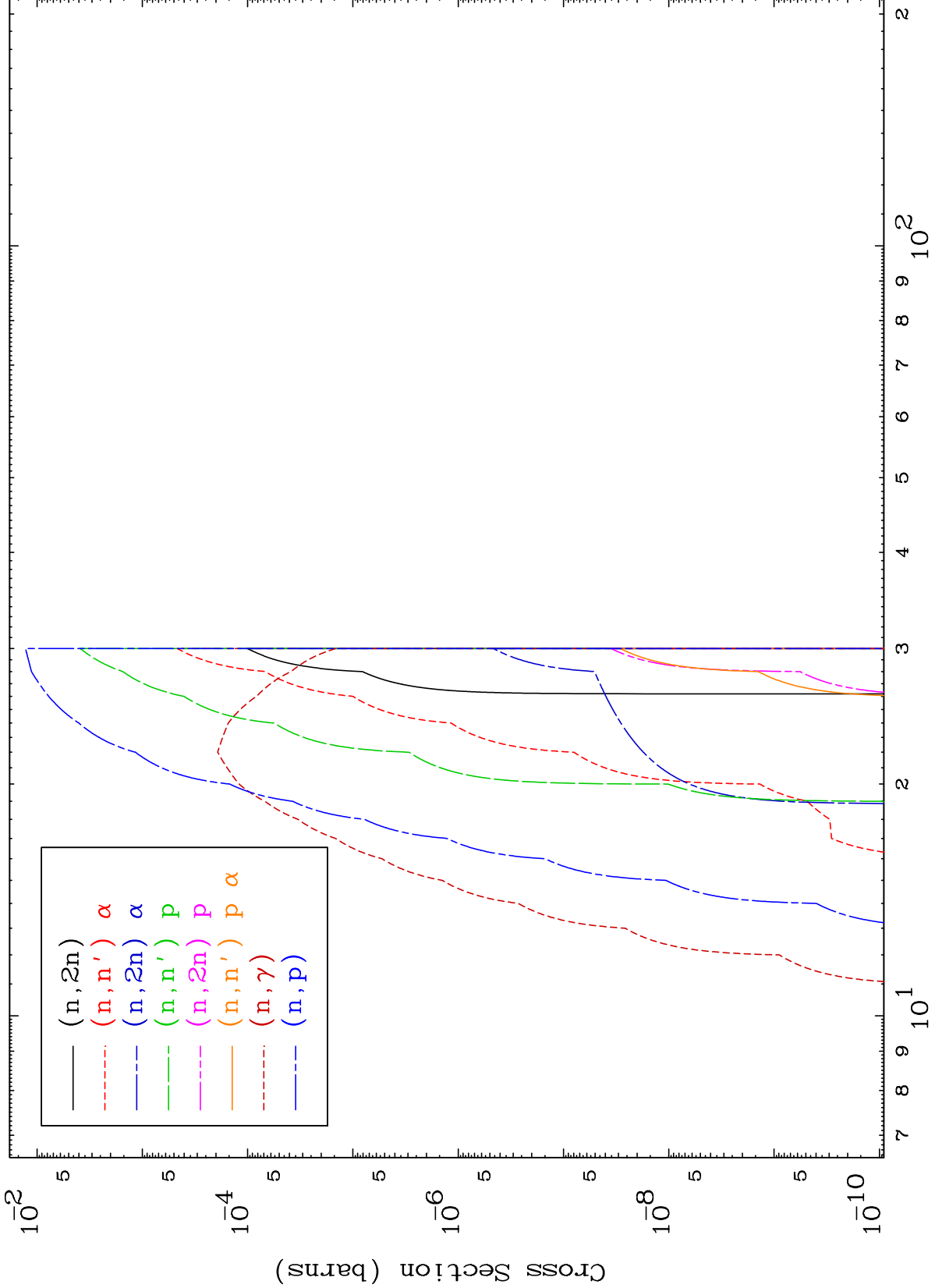
Incident Energy (MeV)

83-Bi-194n

MAT 8282

α Neutron Absorption
0 Kelvin Cross Sections

83-Bi-194n



2

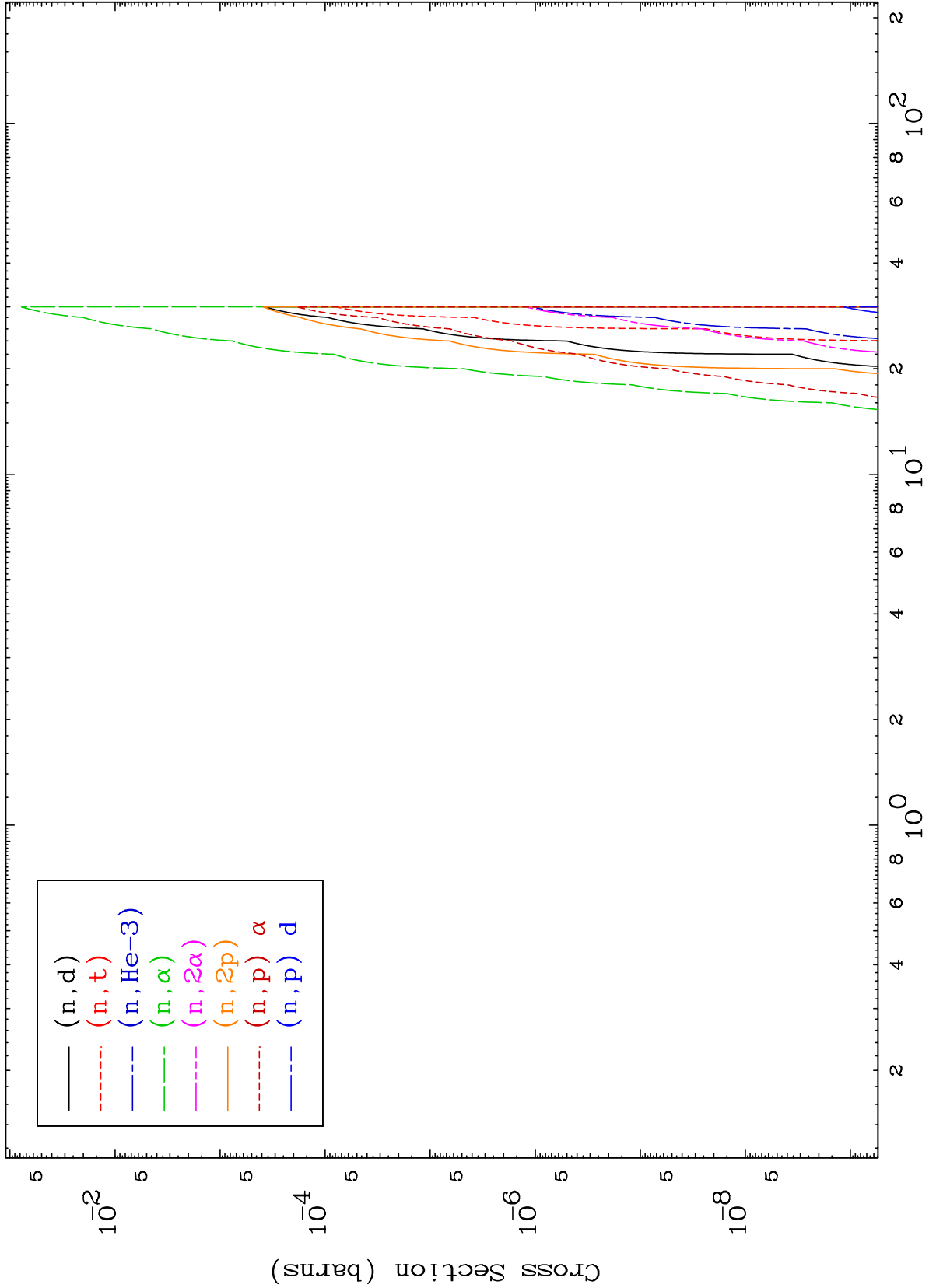
Incident Energy (MeV)

83-Bi-194n

MAT 8282

α Neutron Absorption
0 Kelvin Cross Sections

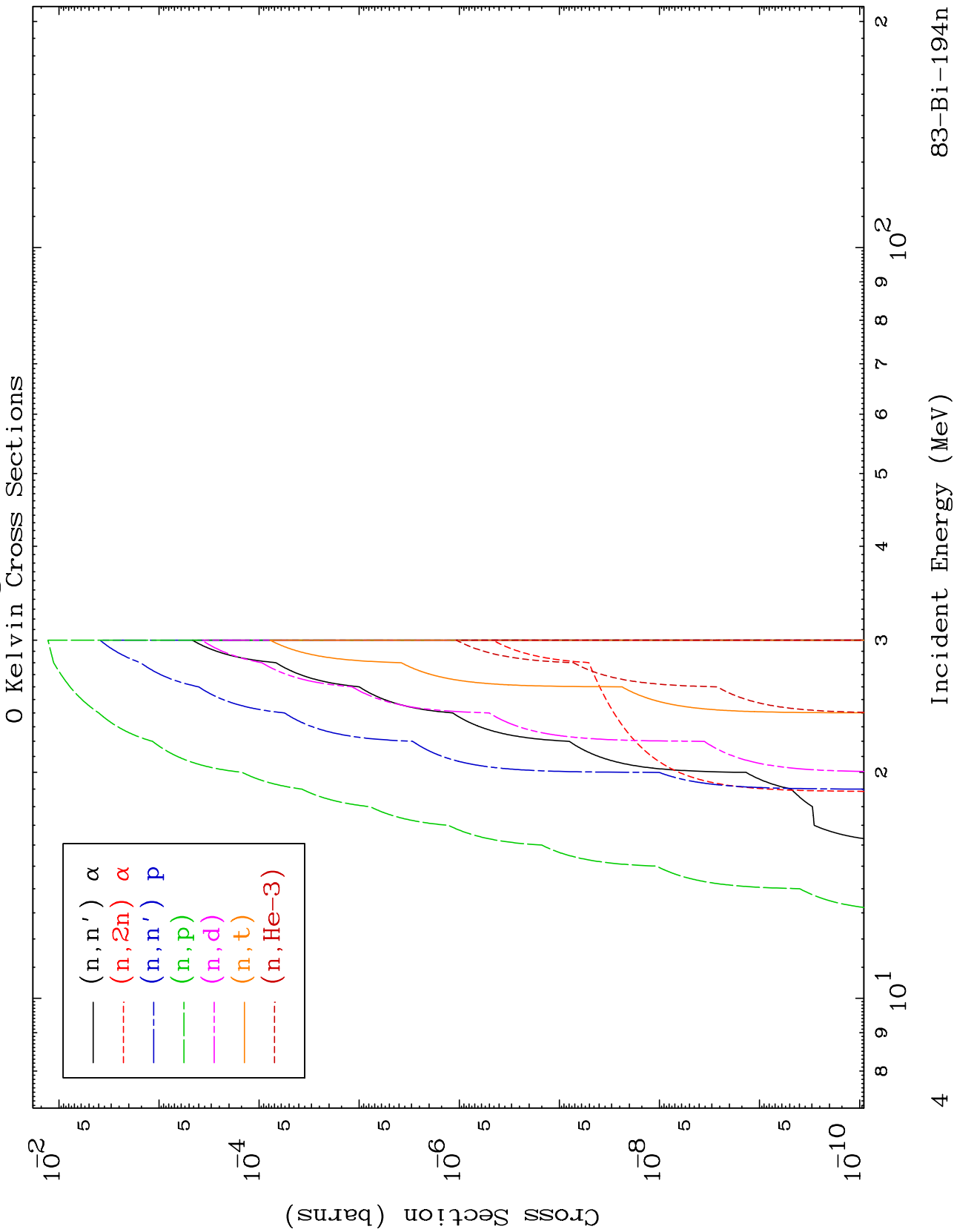
83-Bi-194n

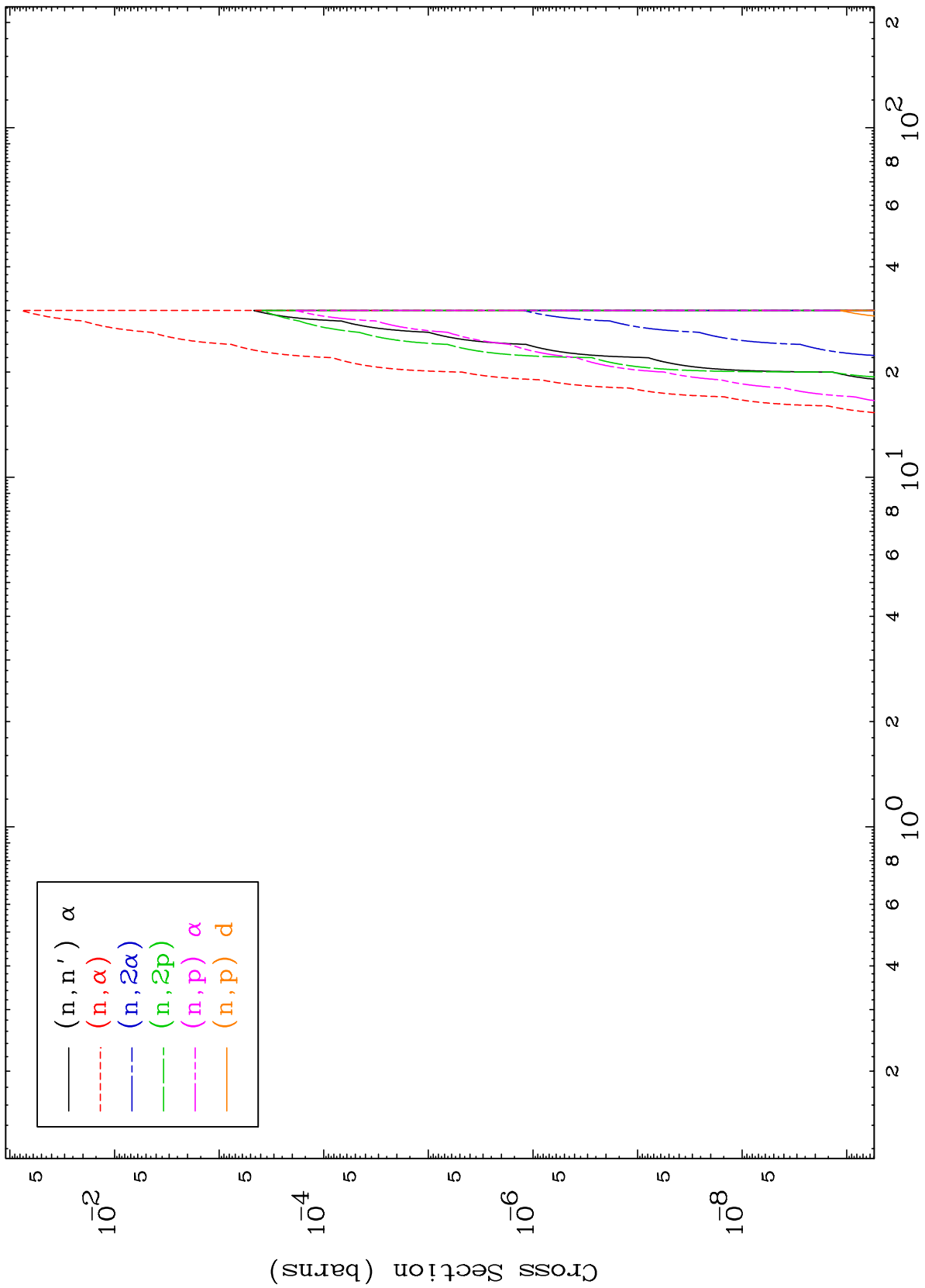


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α Charged Particle

83-Bi-194n



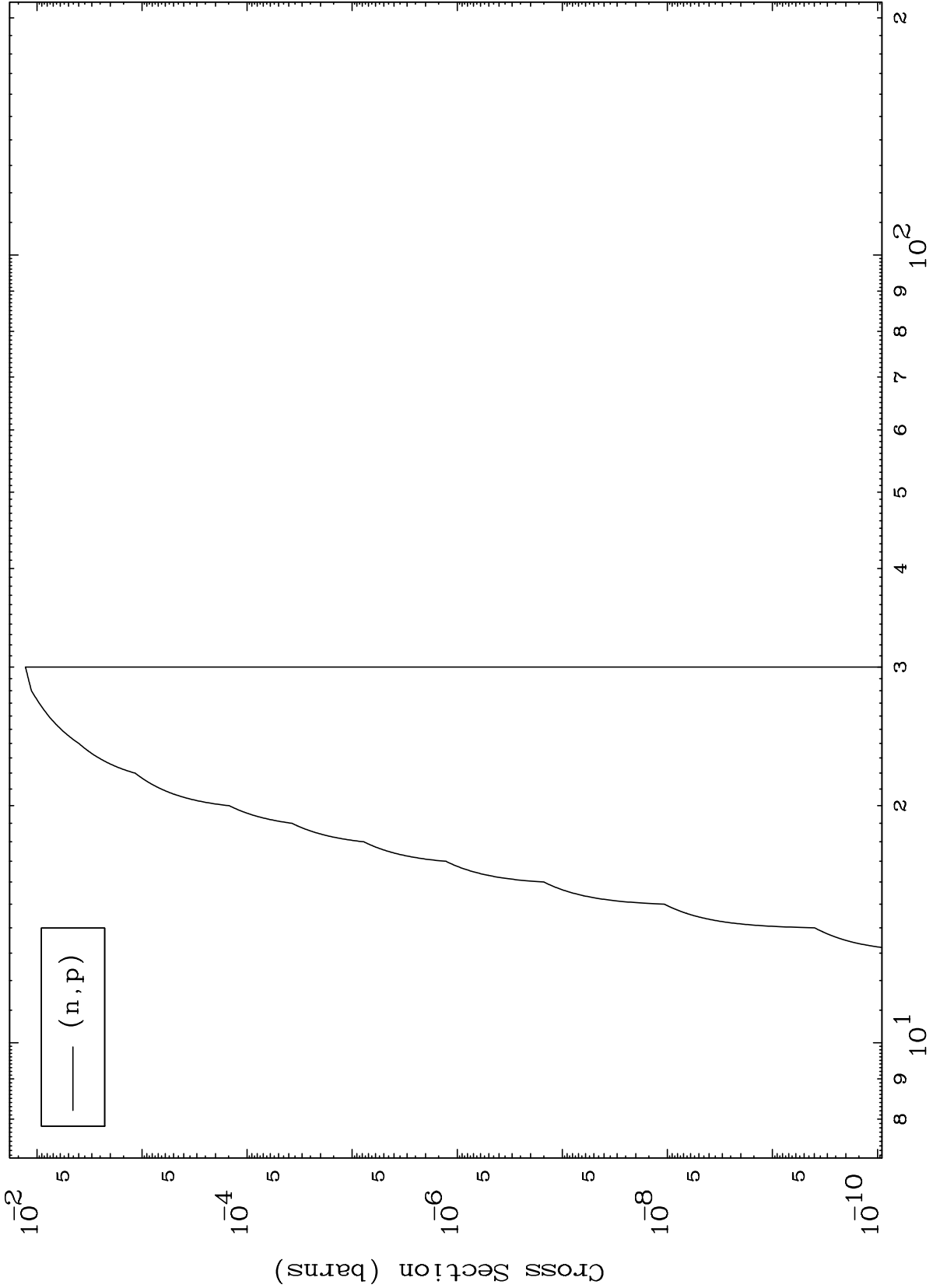


MAT 8282

(α, p) Levels

$^{83}\text{Bi}-194n$

0 Kelvin Cross Sections



6

Incident Energy (MeV)

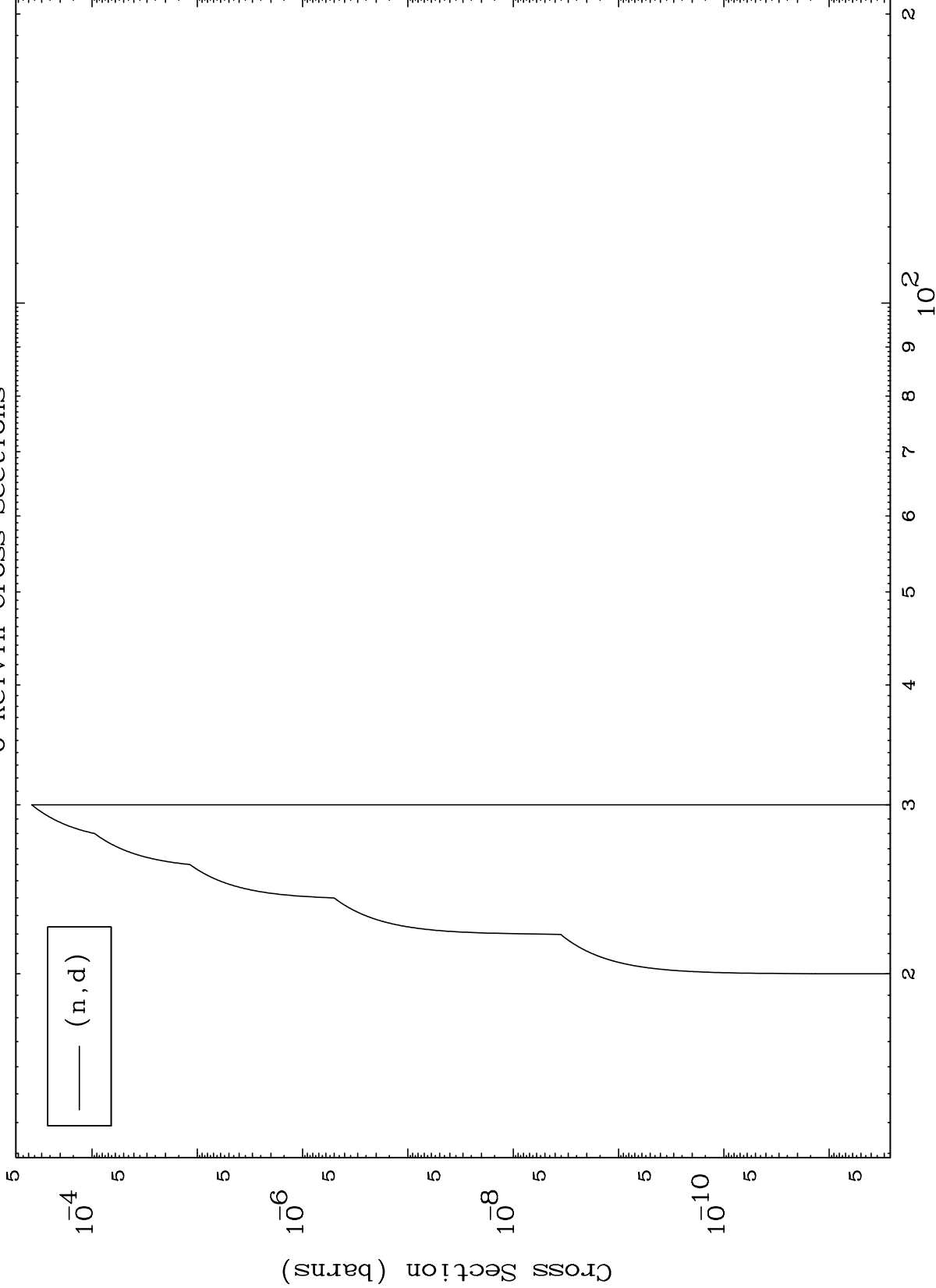
$^{83}\text{Bi}-194n$

MAT 8282

(α, d) Levels

$^{83}\text{Bi}-194\text{n}$

0 Kelvin Cross Sections

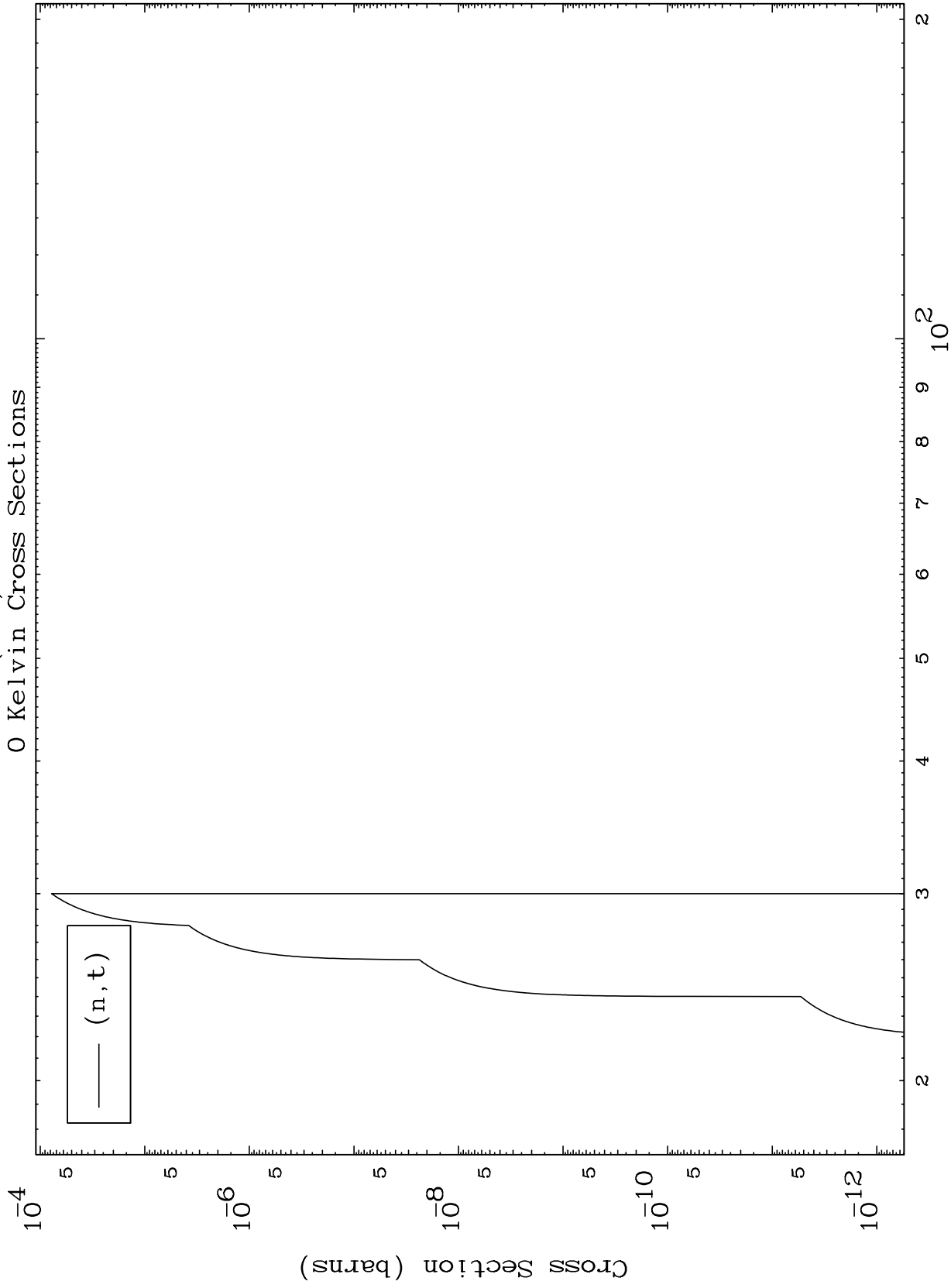


MAT 8282

(α, t) Levels

$^{83}\text{Bi}-194\text{n}$

0 Kelvin Cross Sections



8

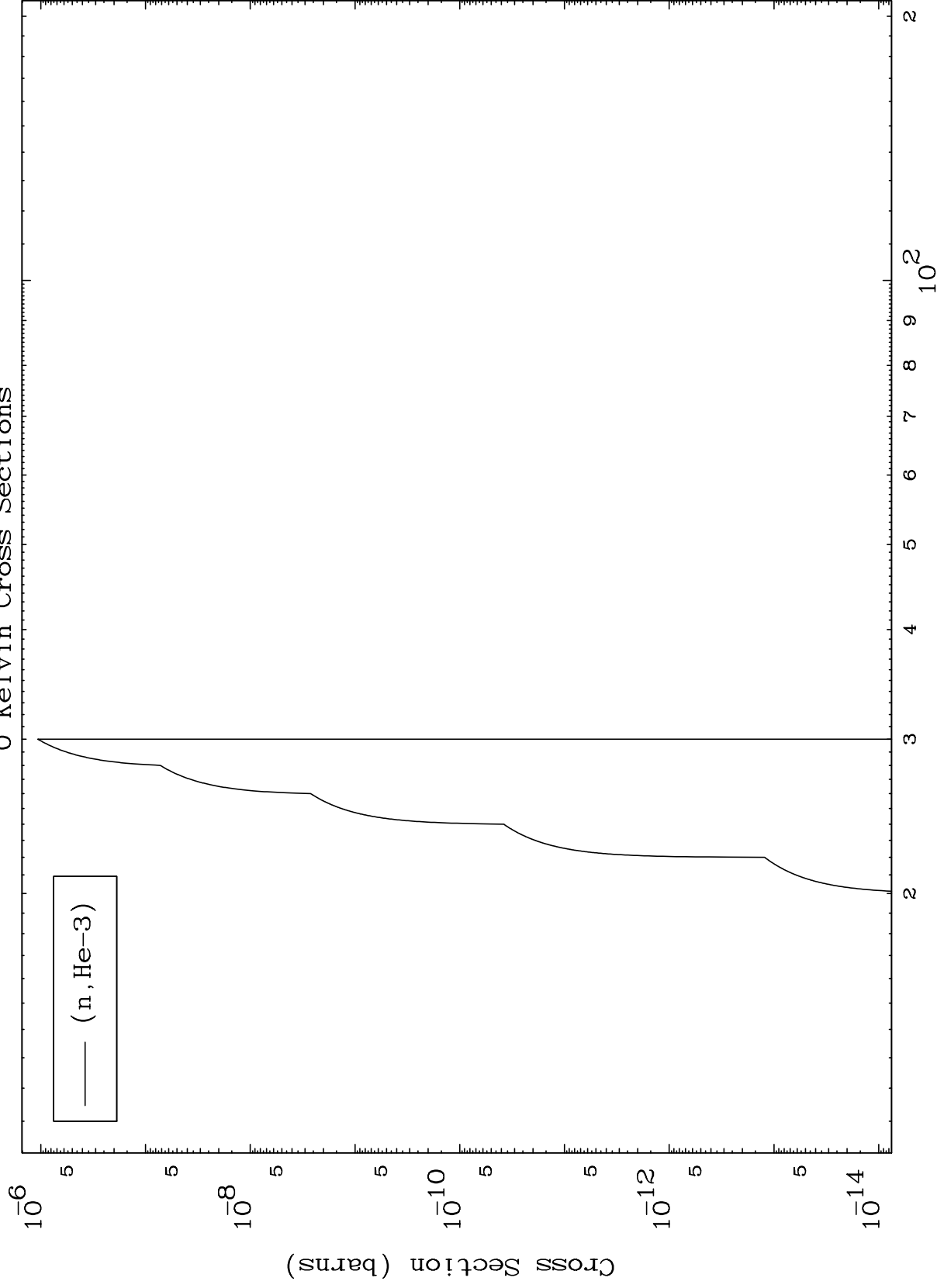
Incident Energy (MeV)

$^{83}\text{Bi}-194\text{n}$

MAT 8282

($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

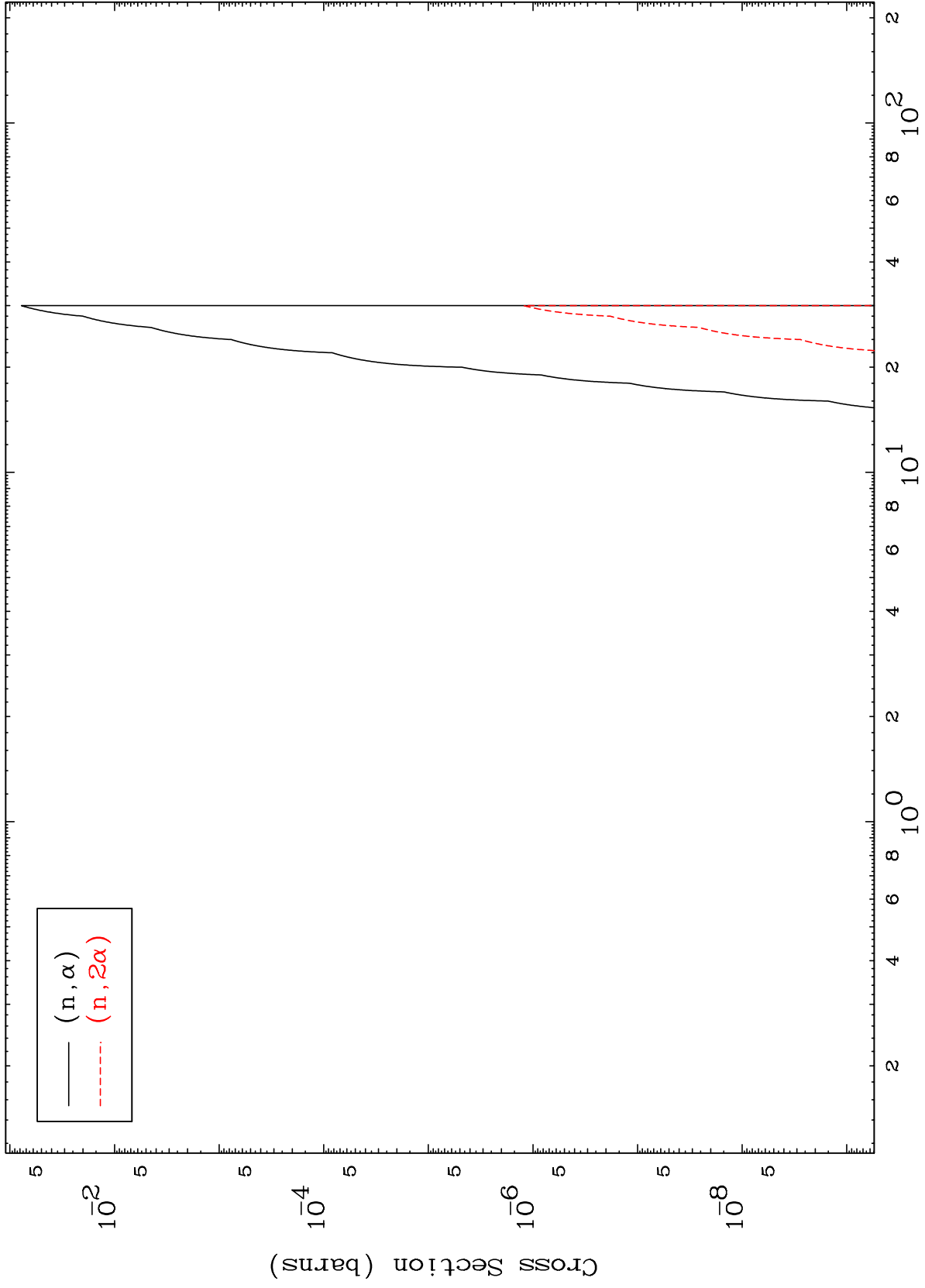
83-Bi-194n



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(α, α) Levels
0 Kelvin Cross Sections

$^{83}\text{Bi}-194\text{n}$



10

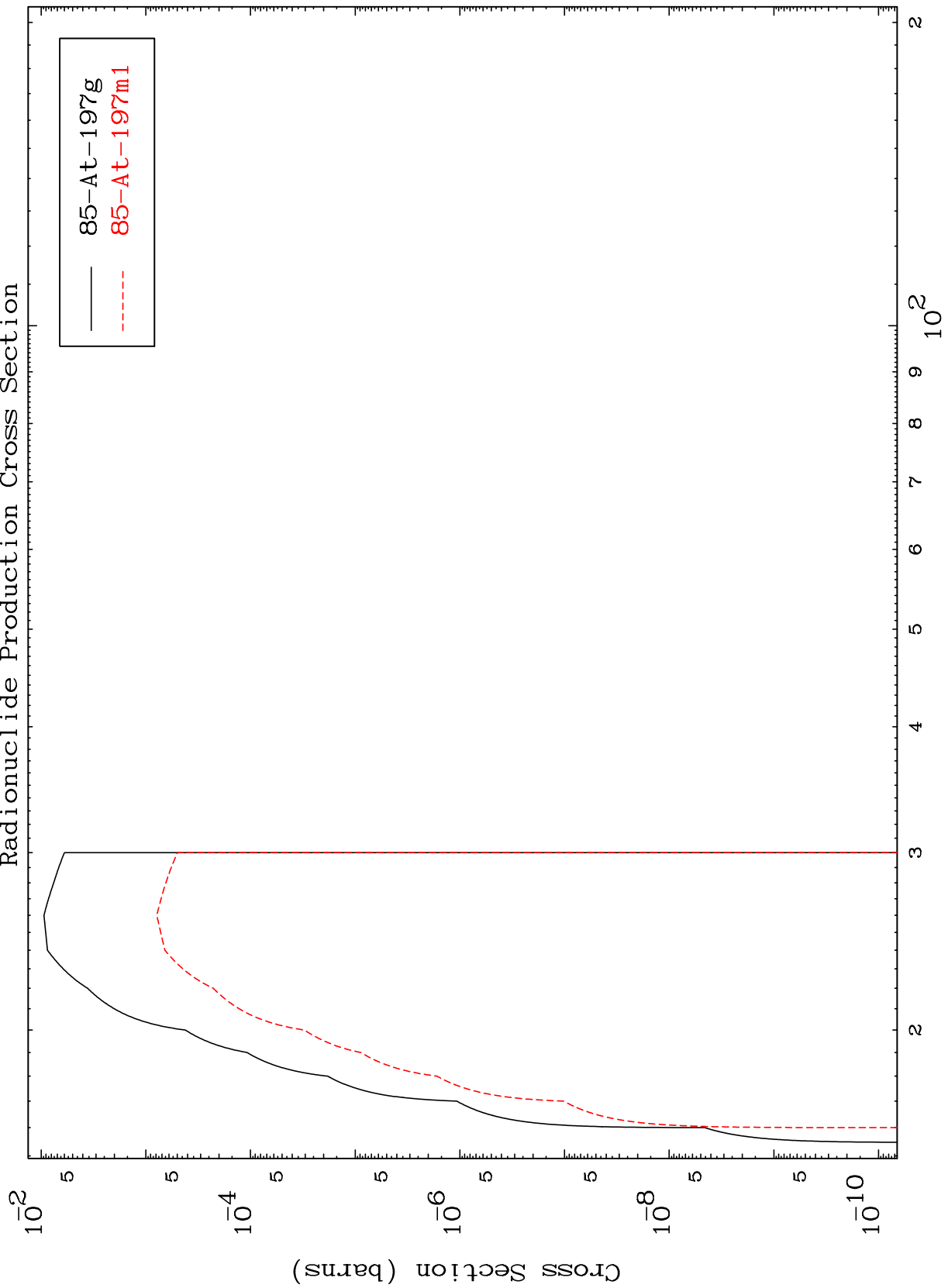
Incident Energy (MeV)

$^{83}\text{Bi}-194\text{n}$

MAT 8282

83-Bi-194n

Inelastic
Radionuclide Production Cross Section

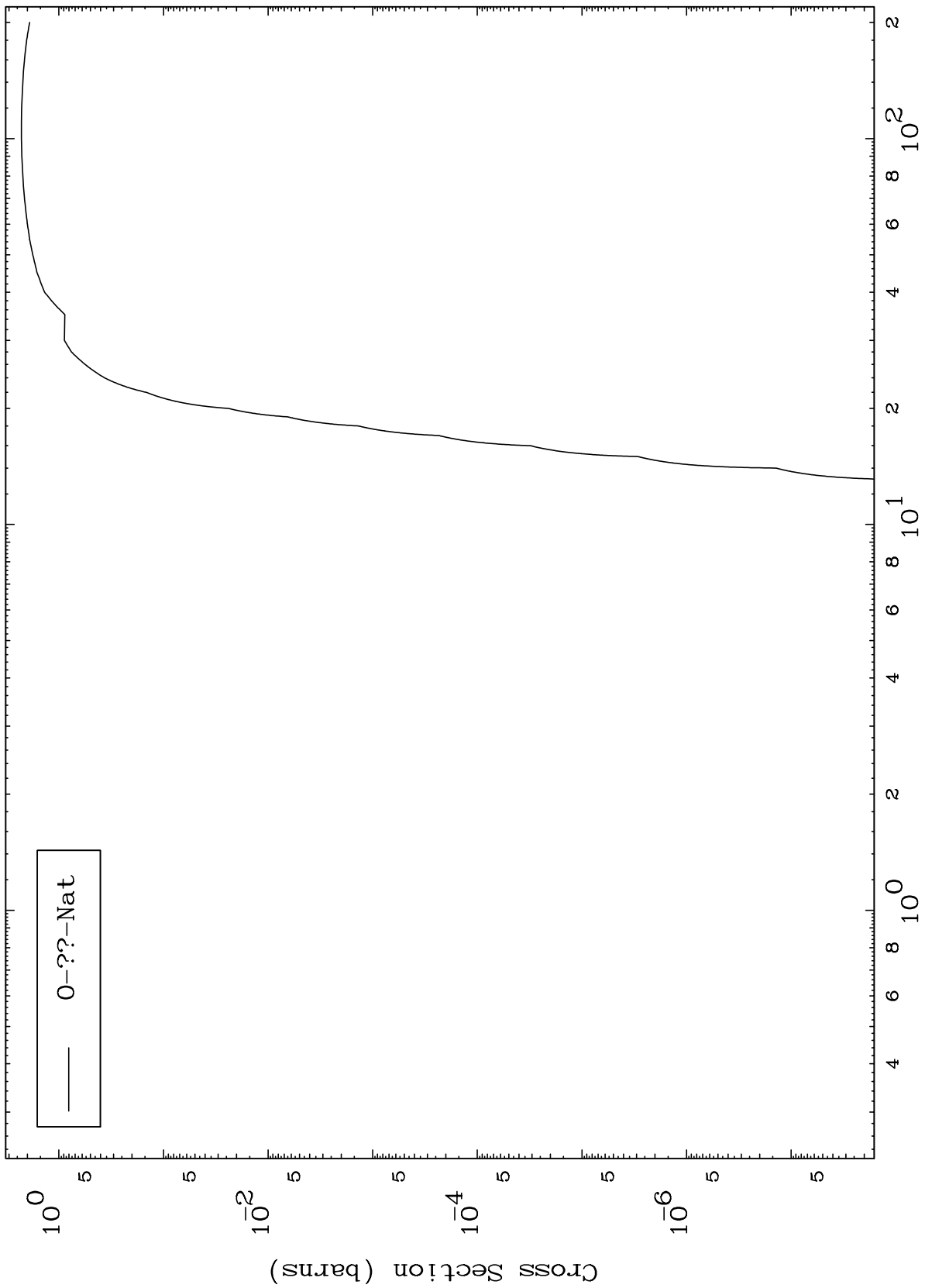


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Fission

⁸³Bi-194n

Radionuclide Production Cross Section

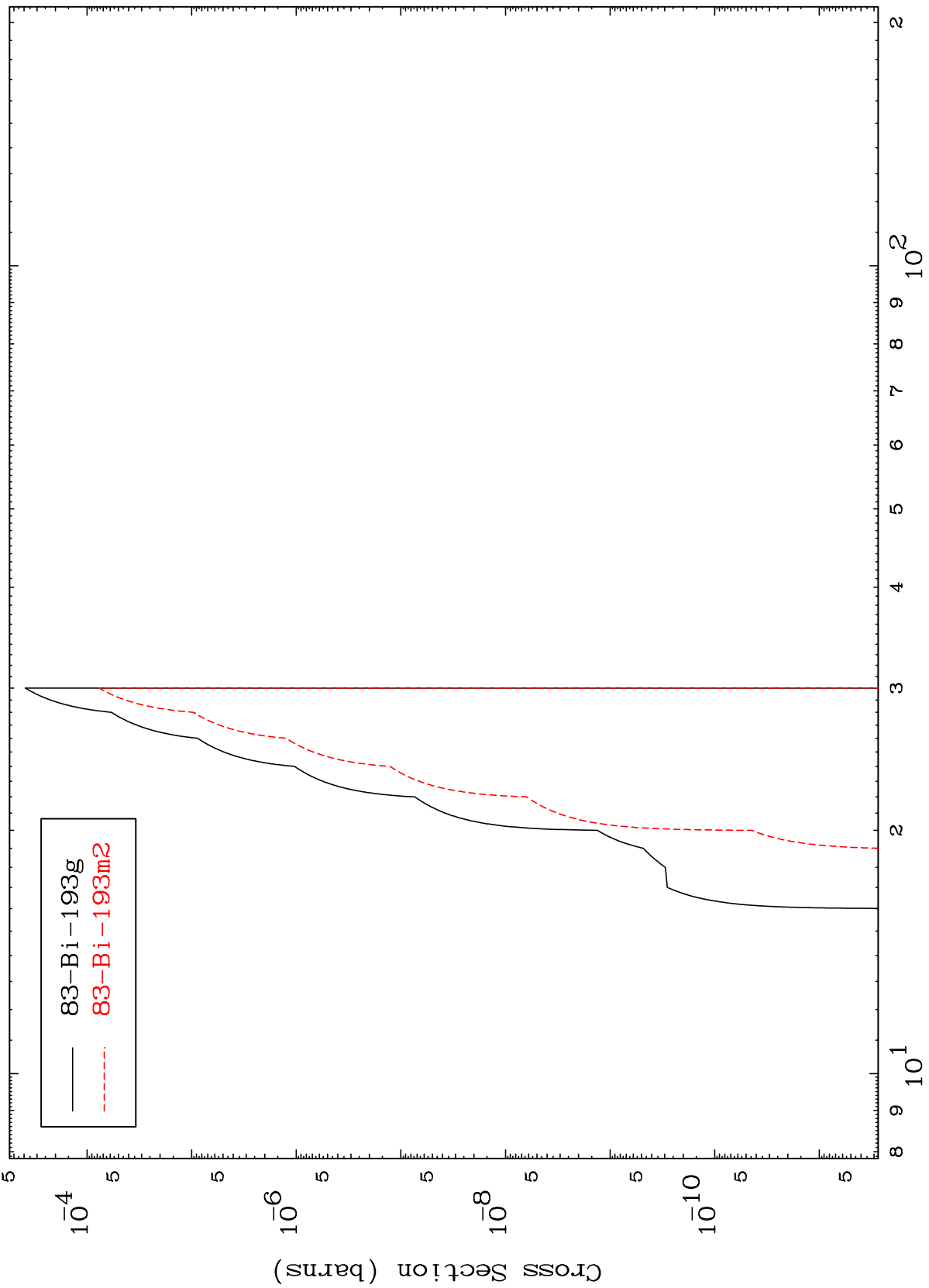


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$(n, n') \alpha$

$^{83}\text{Bi}-194\text{n}$

Radionuclide Production Cross Section



13

Incident Energy (MeV)

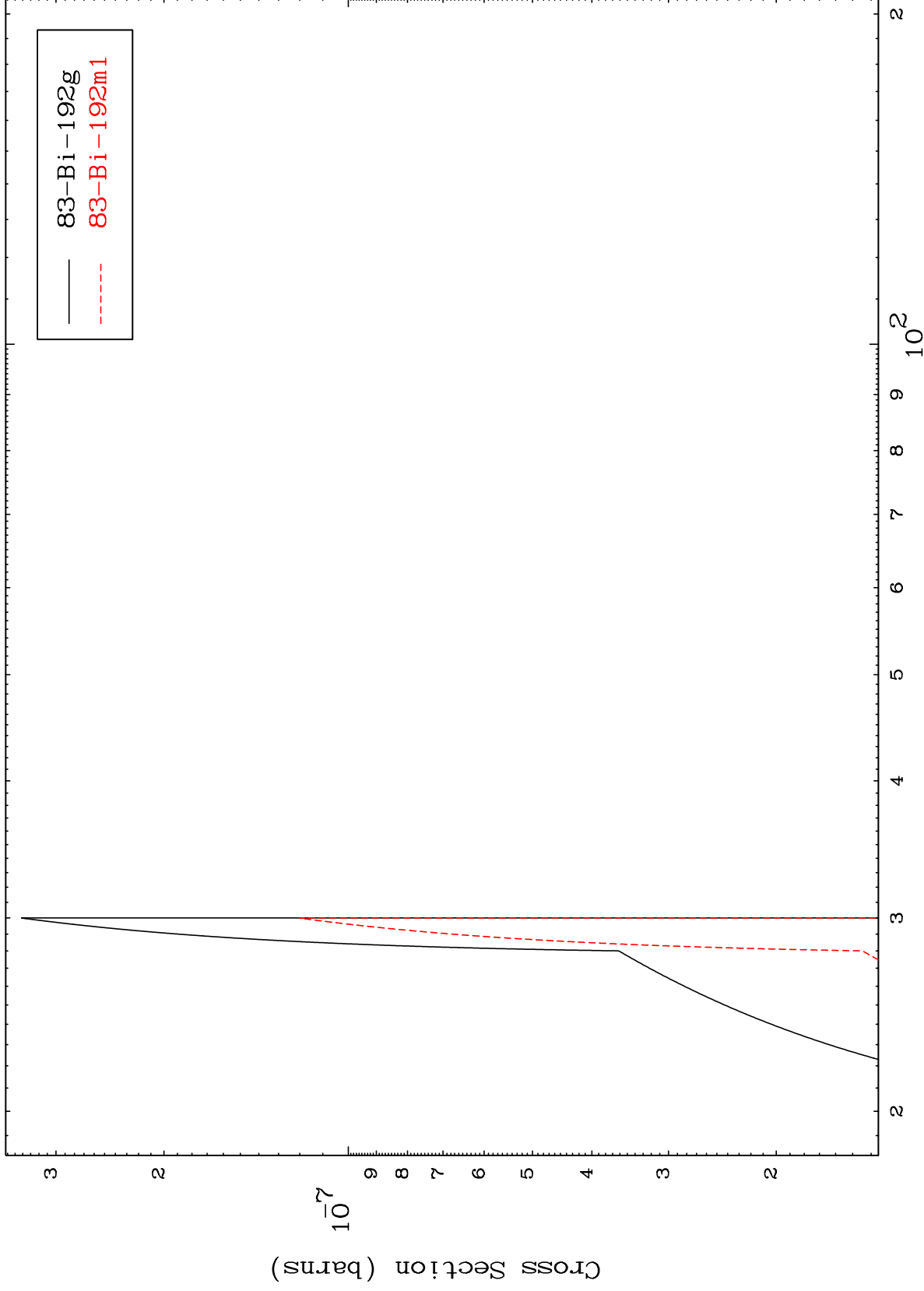
$^{83}\text{Bi}-194\text{n}$

MAT 8282

$(n,2n) \alpha$

$^{83}\text{Bi}-194n$

Radionuclide Production Cross Section



14

Incident Energy (MeV)

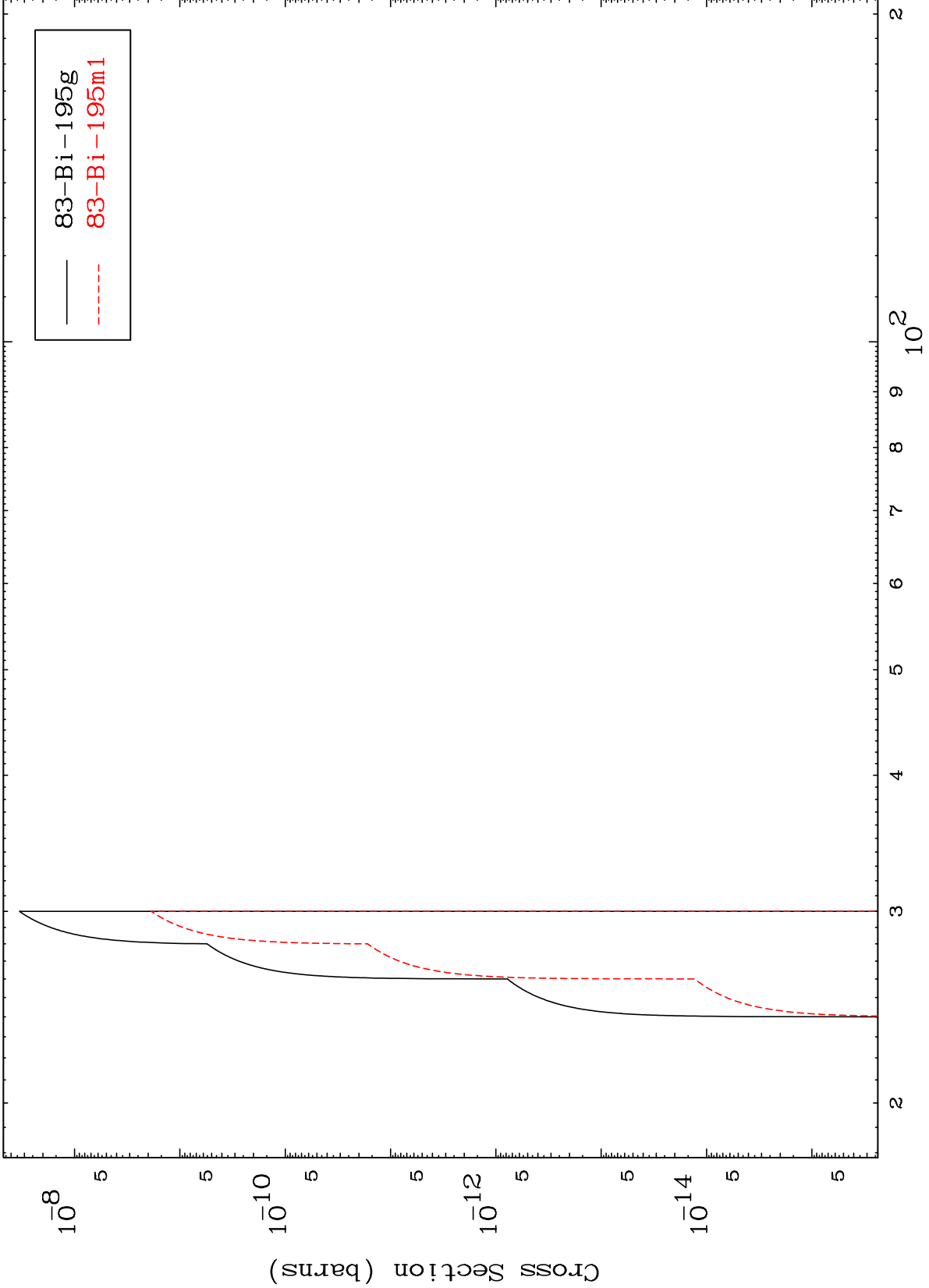
$^{83}\text{Bi}-194n$

MAT 8282

(n,2n) p

83-Bi-194n

Radionuclide Production Cross Section



15

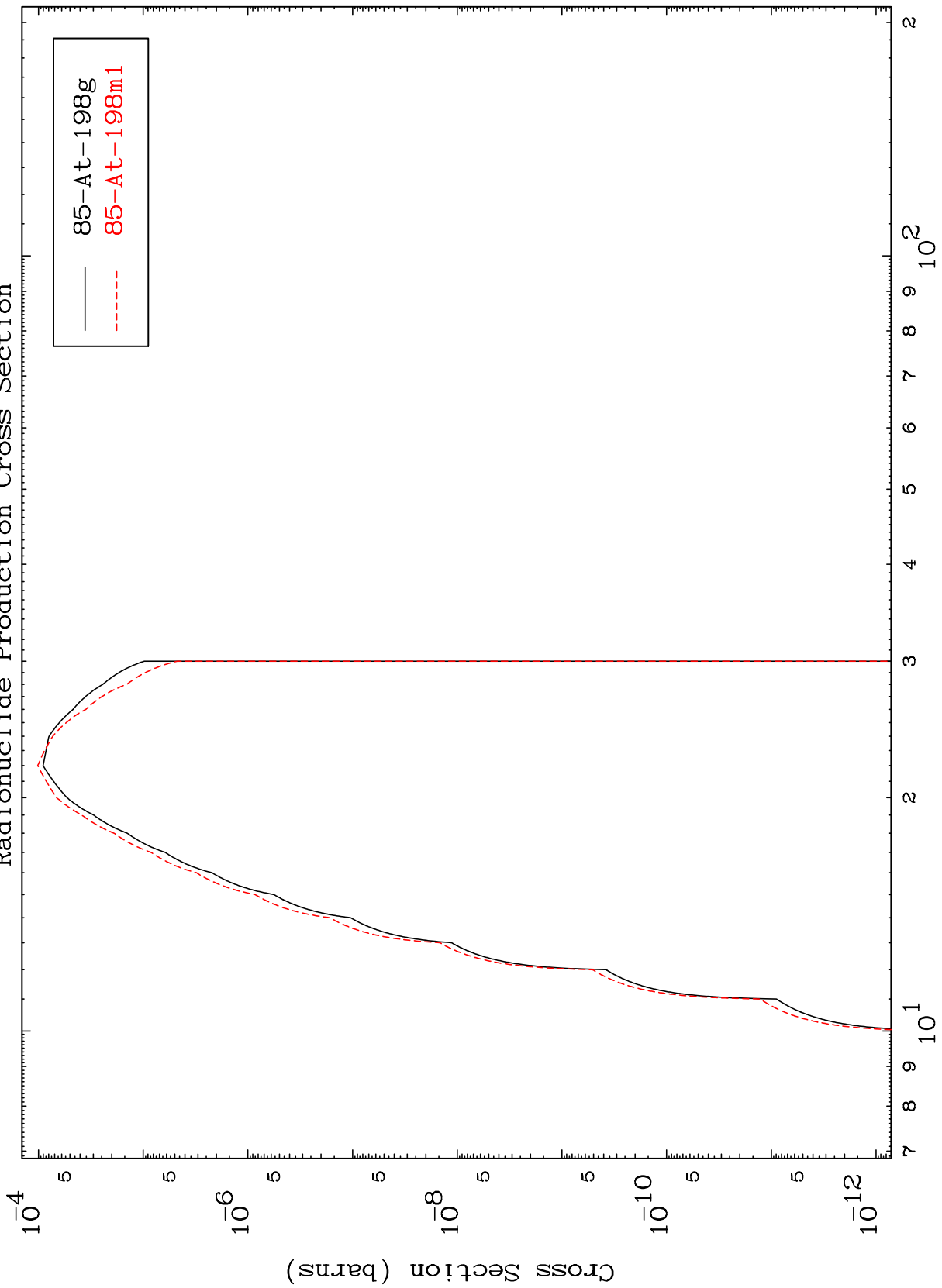
Incident Energy (MeV)

83-Bi-194n

MAT 8282

83-Bi-194n

(n,γ)
Radionuclide Production Cross Section



16

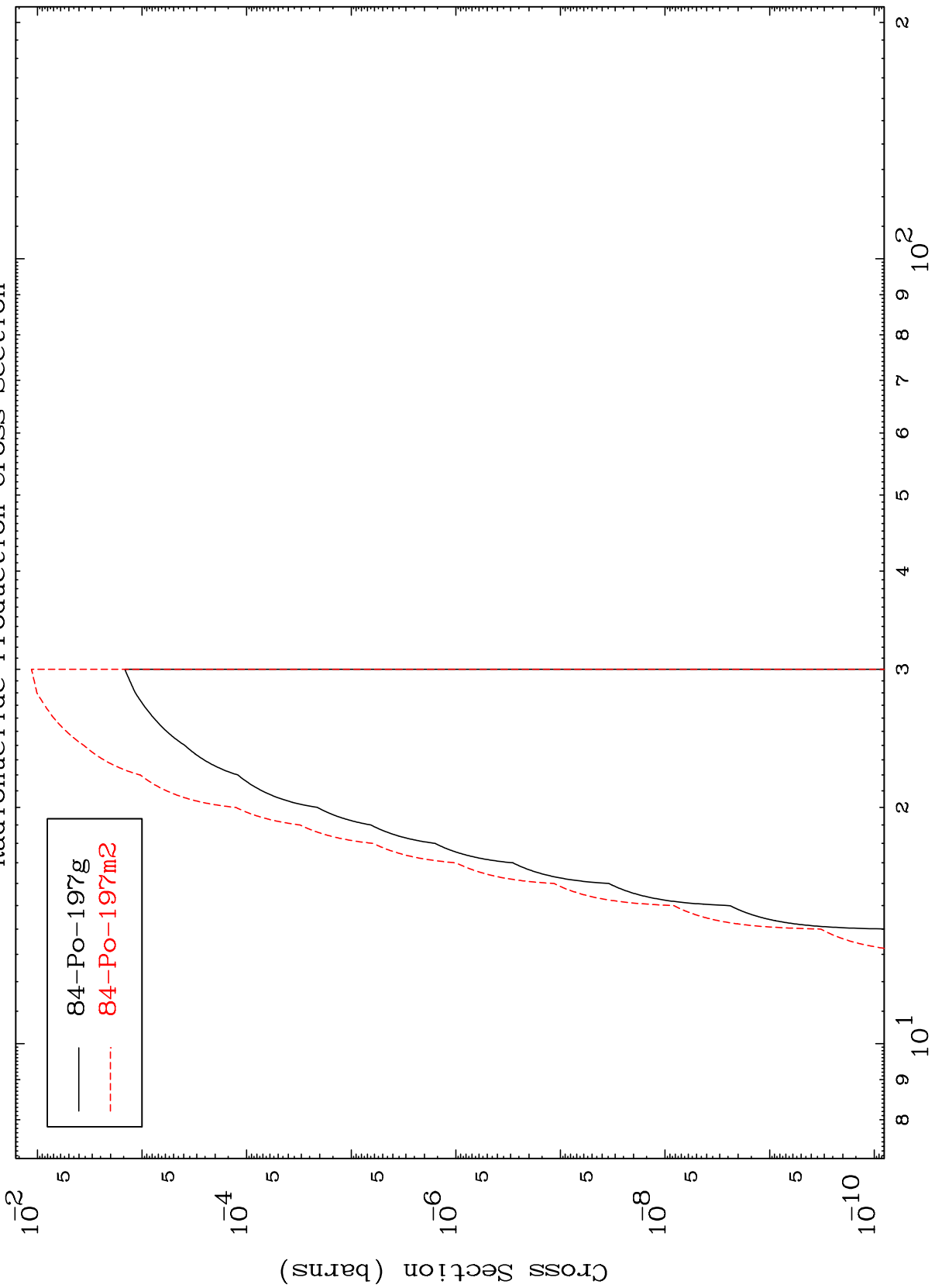
Incident Energy (MeV)

83-Bi-194n

MAT 8282

83-Bi-194n

(n,p)
Radionuclide Production Cross Section



17

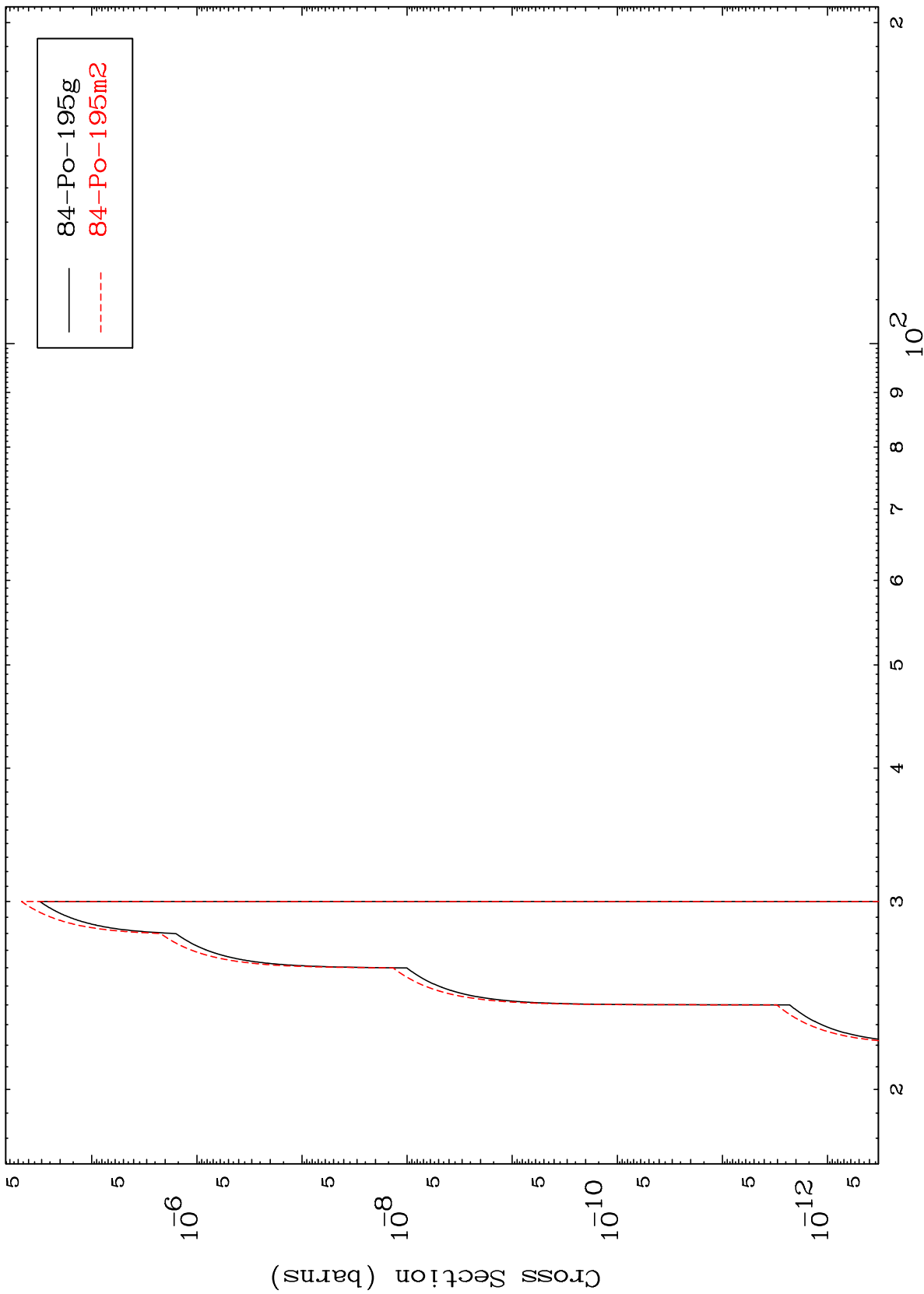
Incident Energy (MeV)

83-Bi-194n

MAT 8282

83-Bi-194n

(n,t)
Radionuclide Production Cross Section



18

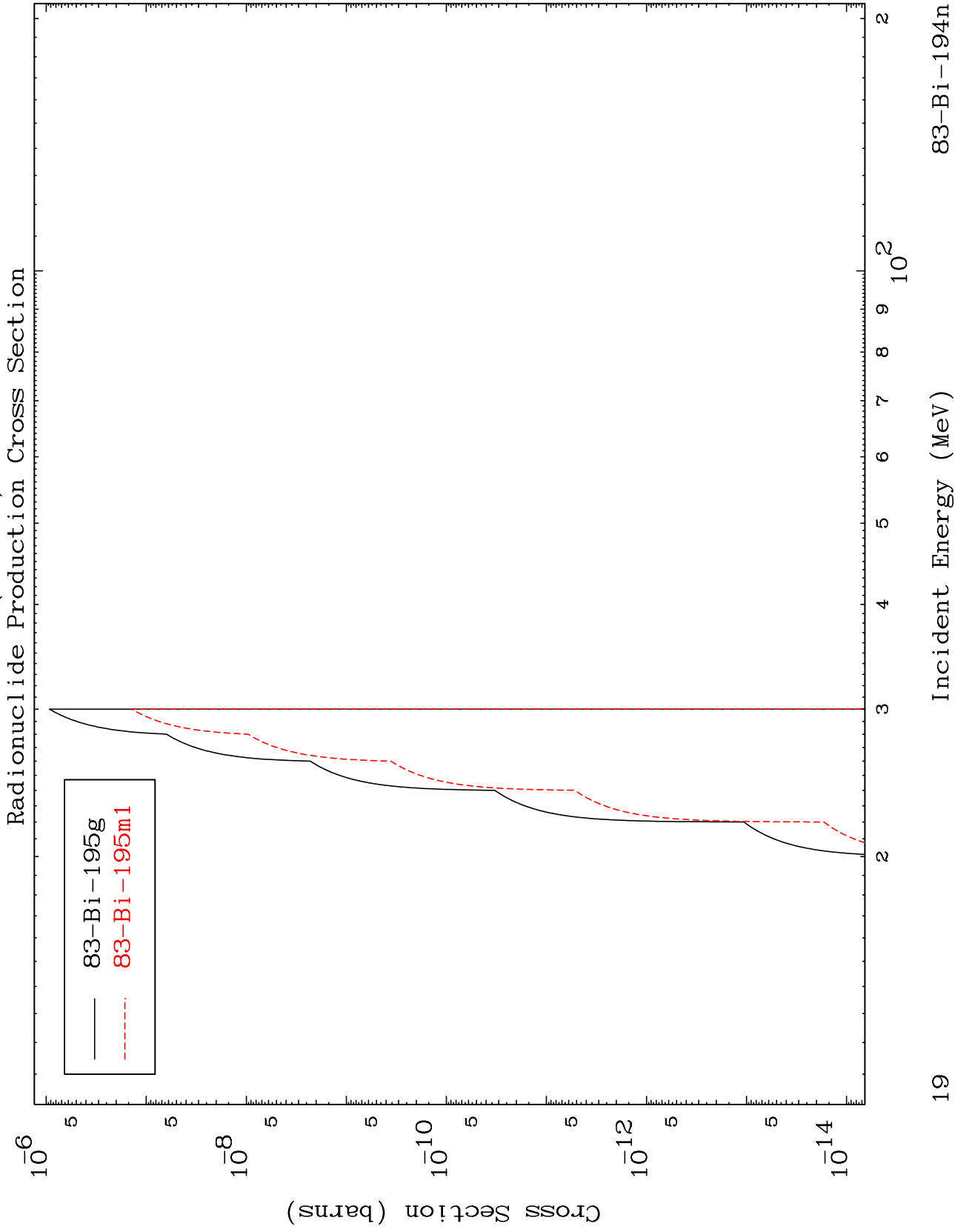
83-Bi-194n

Incident Energy (MeV)

MAT 8282

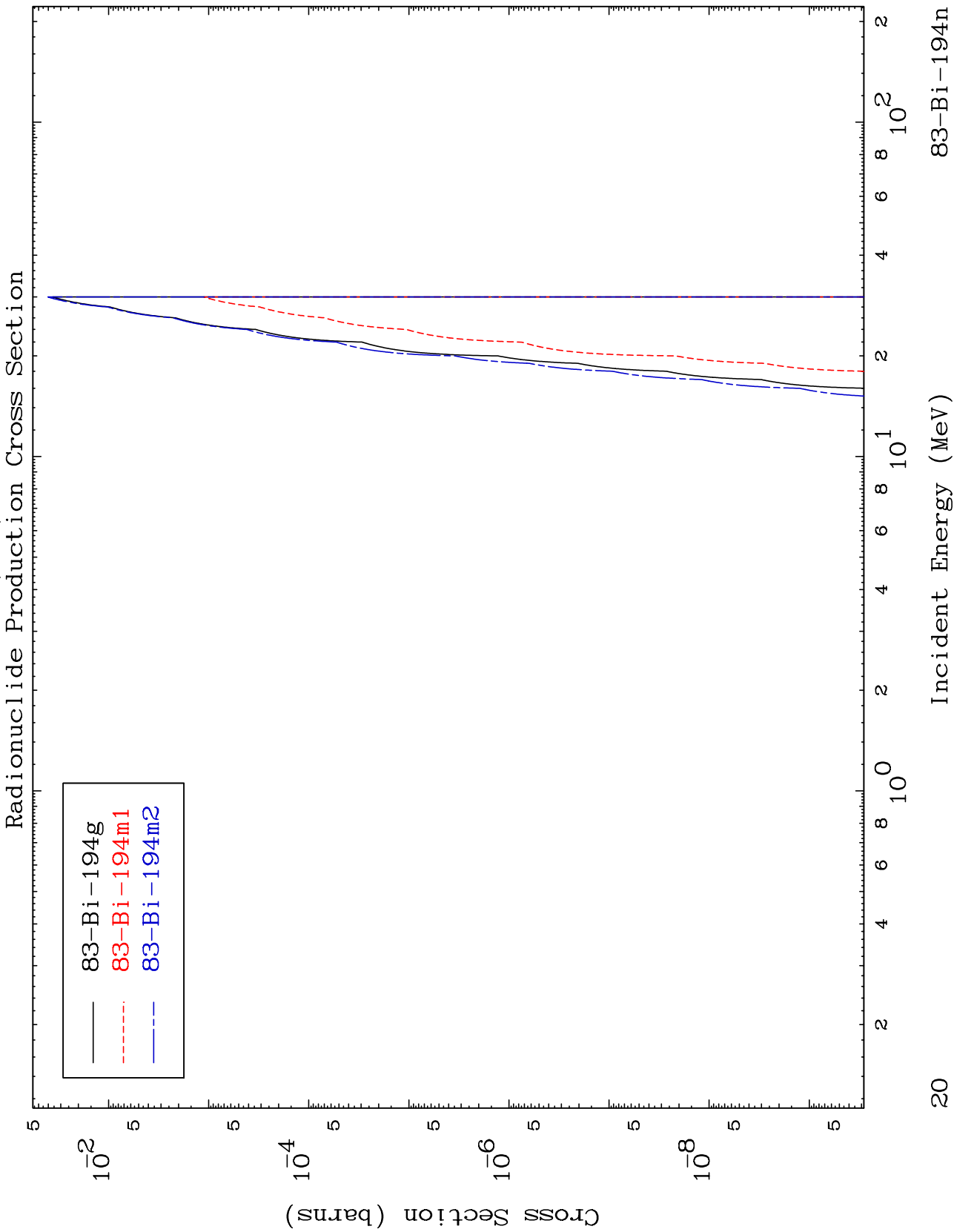
(n,He-3)

83-Bi-194n



MAT 8282

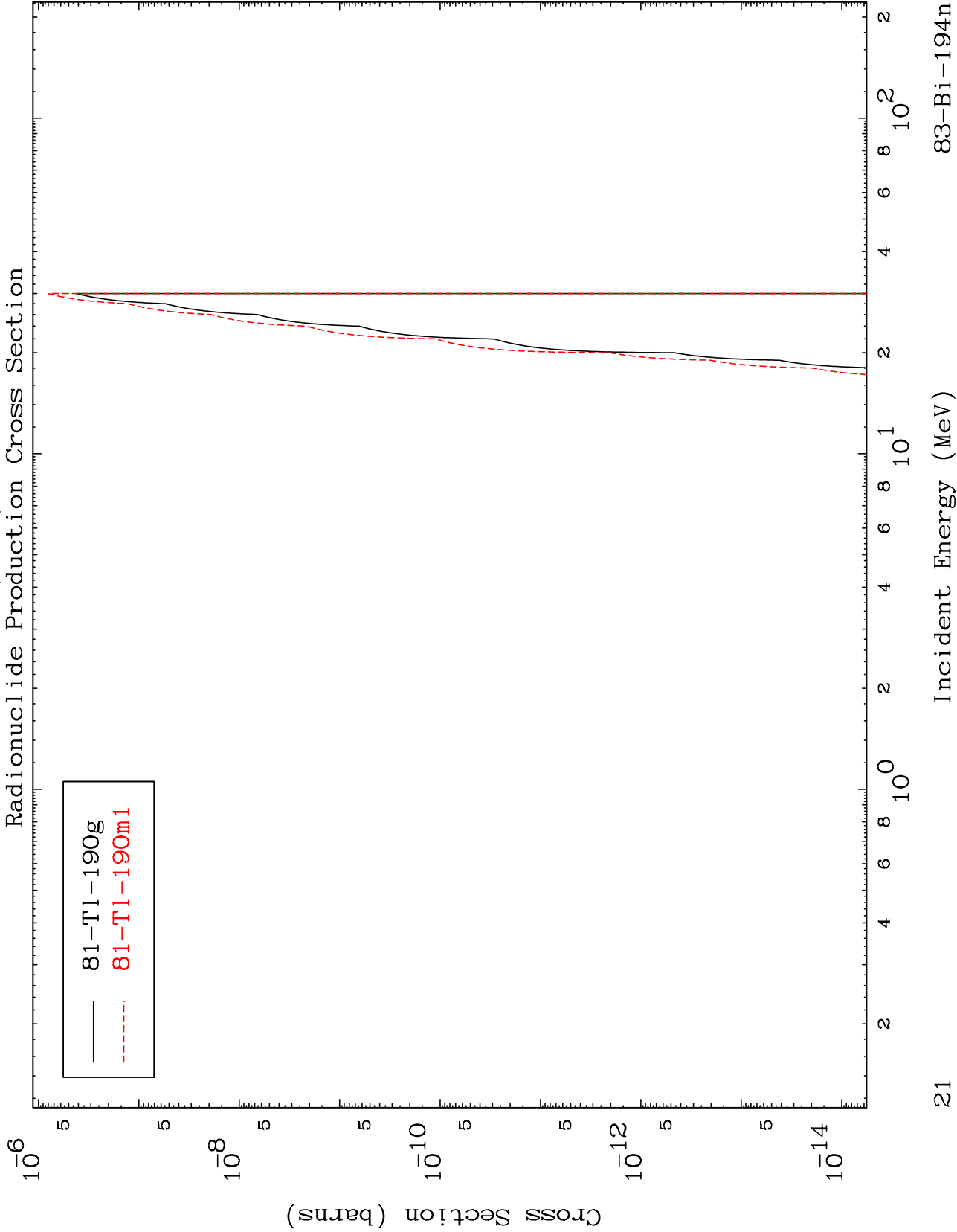
$^{83}\text{Bi}-194\text{n}$



MAT 8282

(n,2α)

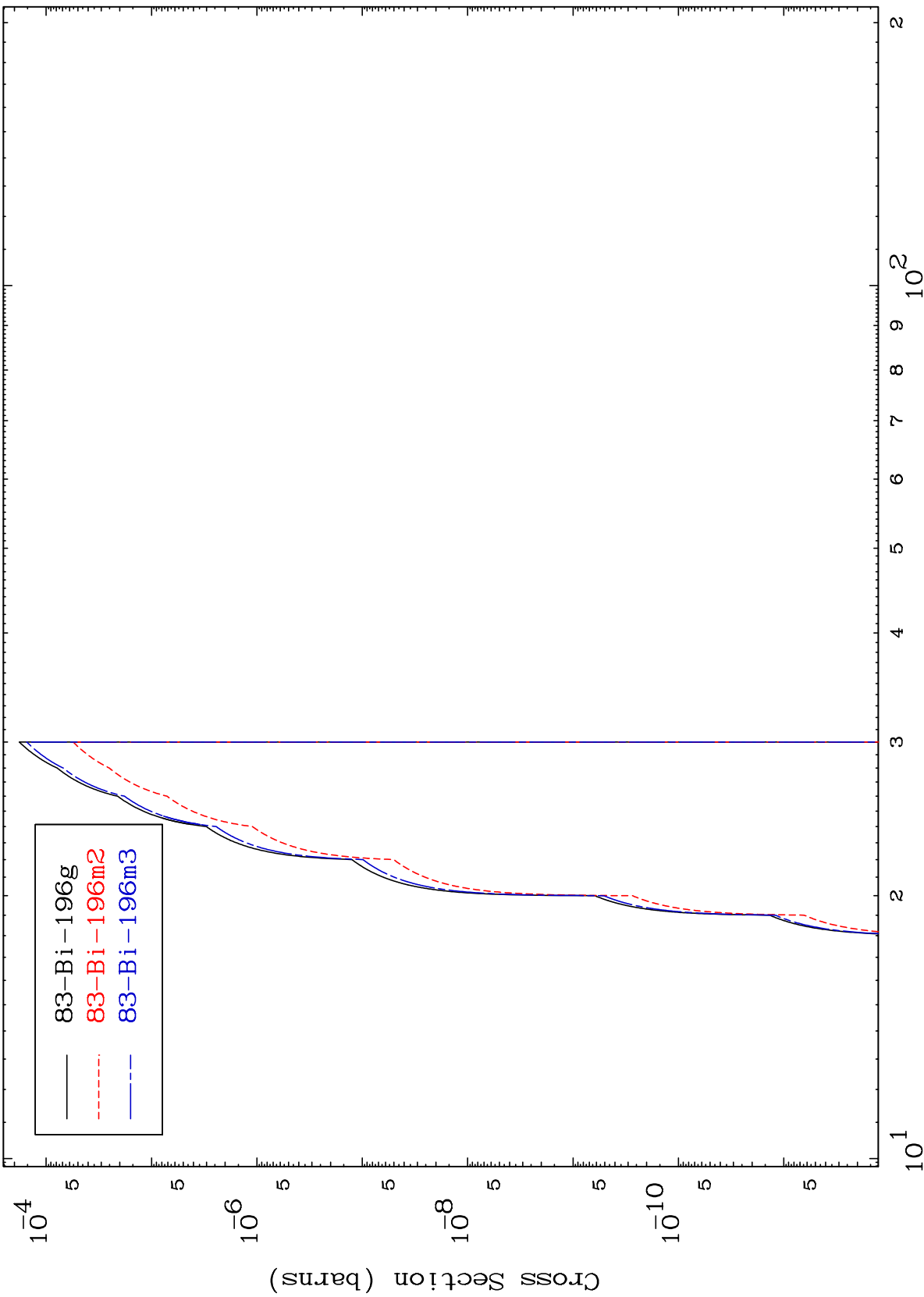
83-Bi-194n



MAT 8282

$^{83}\text{Bi}-194\text{n}$

(n,2p)
Radionuclide Production Cross Section



$^{83}\text{Bi}-194\text{n}$

Incident Energy (MeV)

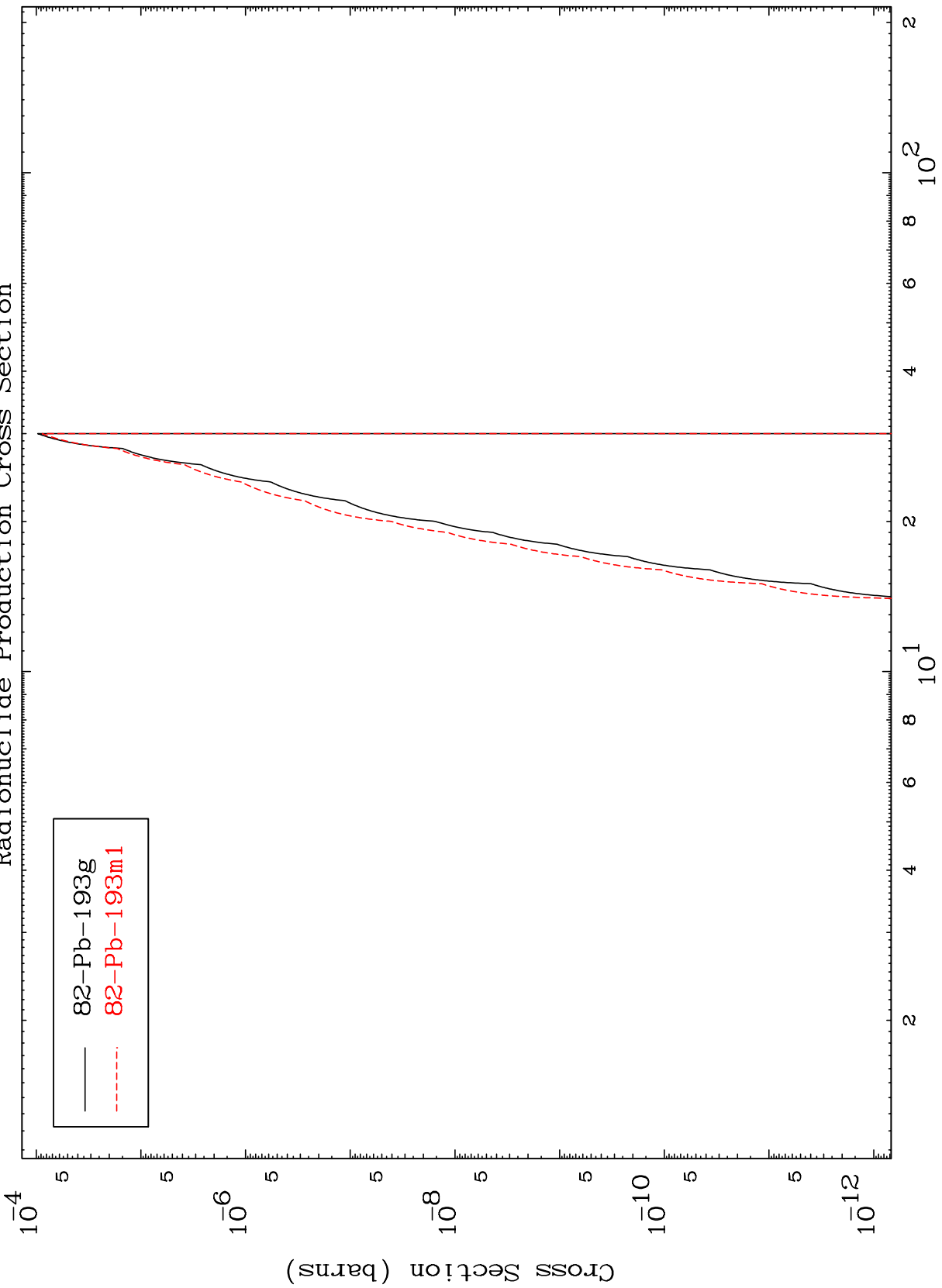
22

MAT 8282

(n,p) α

83-Bi-194n

Radionuclide Production Cross Section



— 82-Pb-193g
- - - 82-Pb-193m1

Incident Energy (MeV)

83-Bi-194n

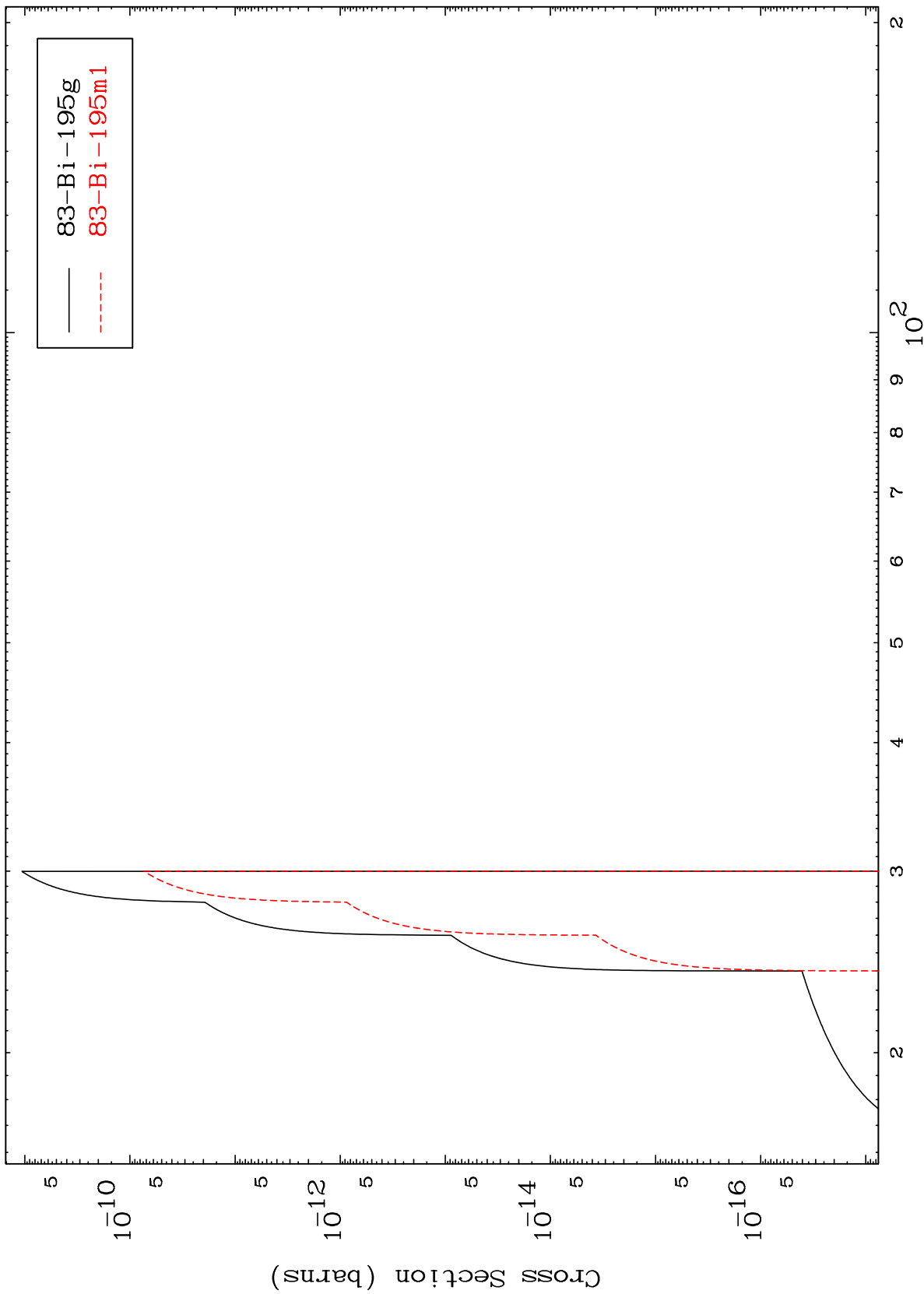
23

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(n,p) d

83-Bi-194n

Radionuclide Production Cross Section



24

Incident Energy (MeV)

83-Bi-194n