

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

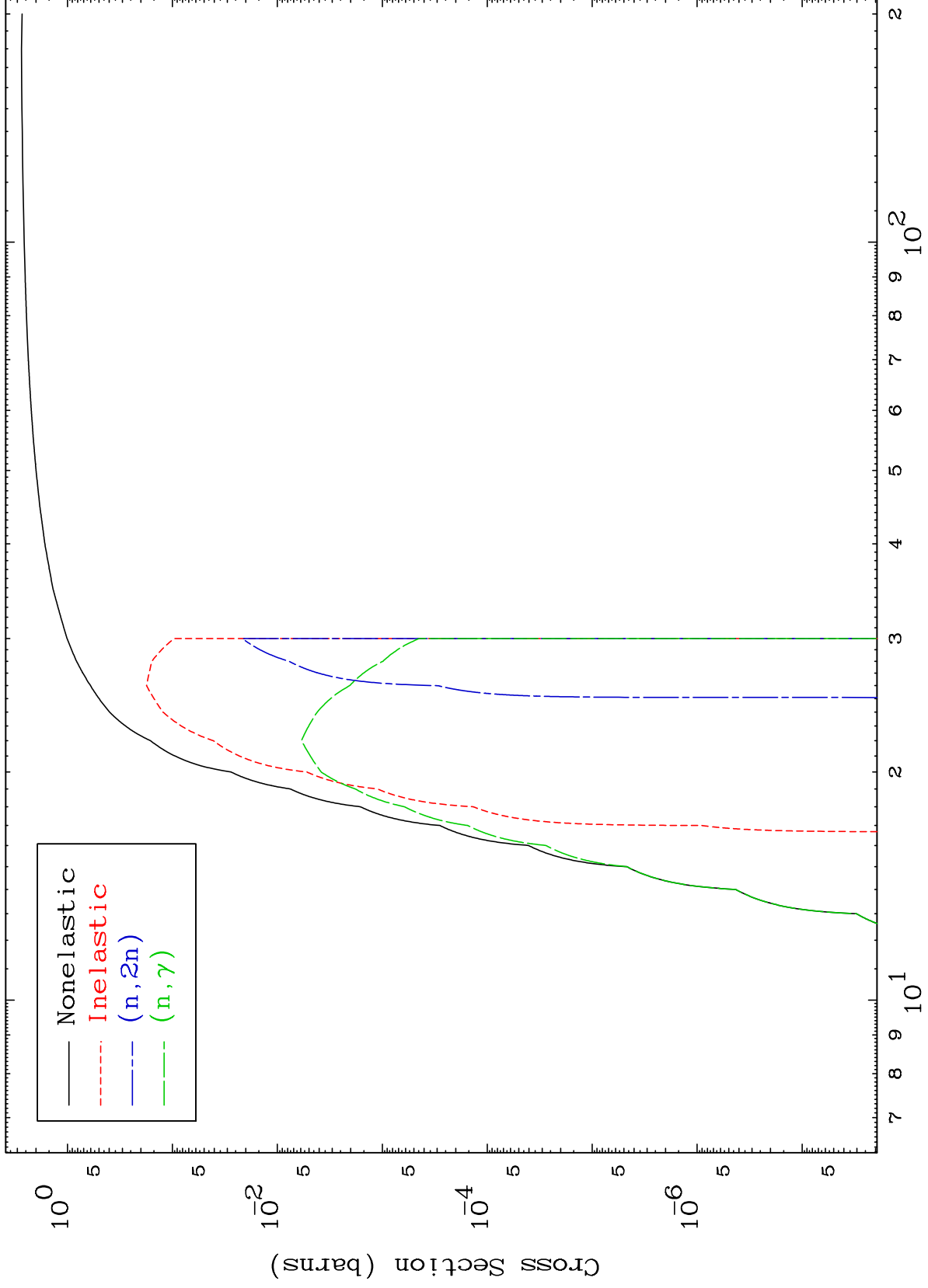
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Press Mouse Button to Start

MAT 8289

0 Kelvin Cross Sections

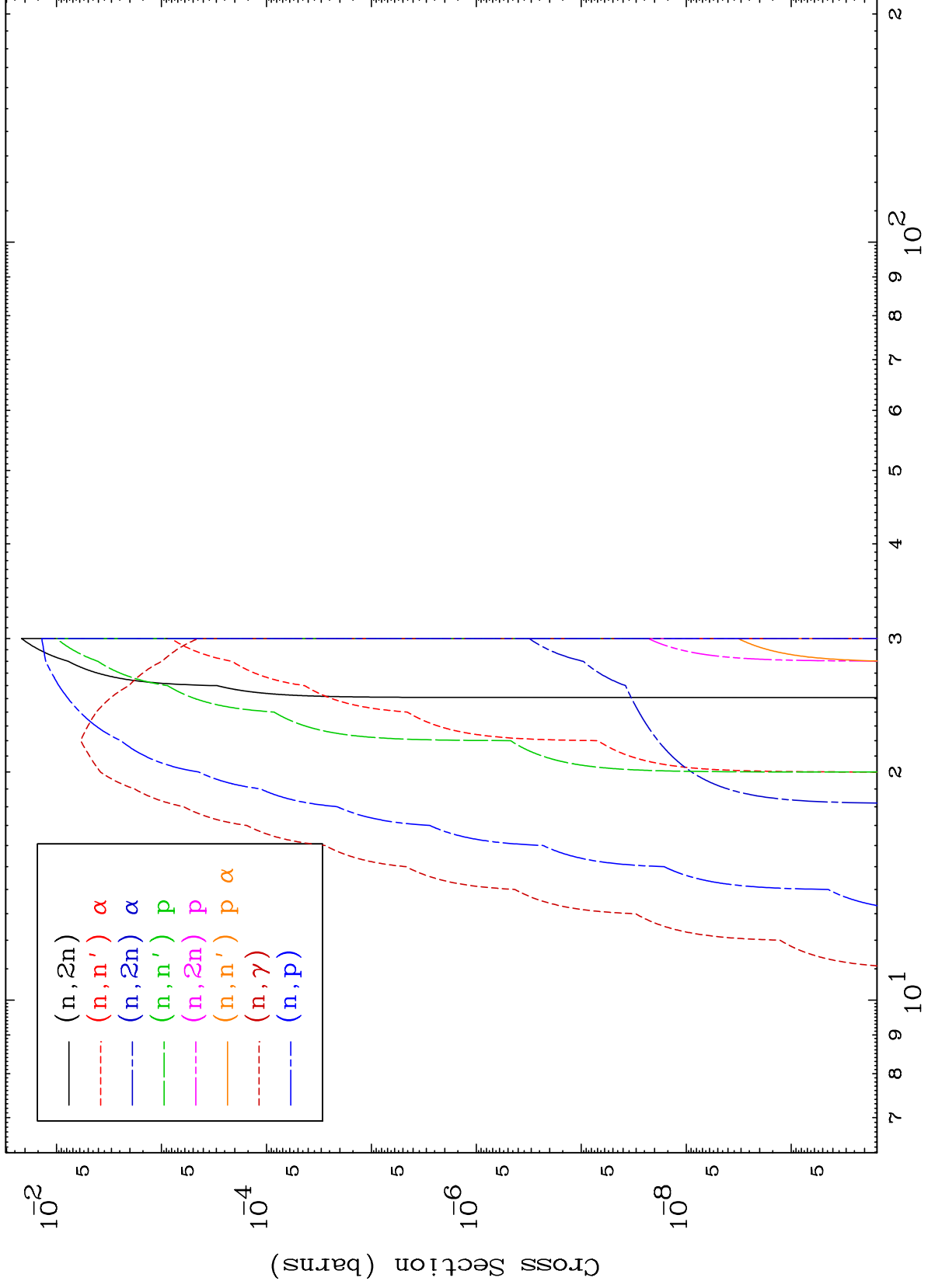
83-Bi-197

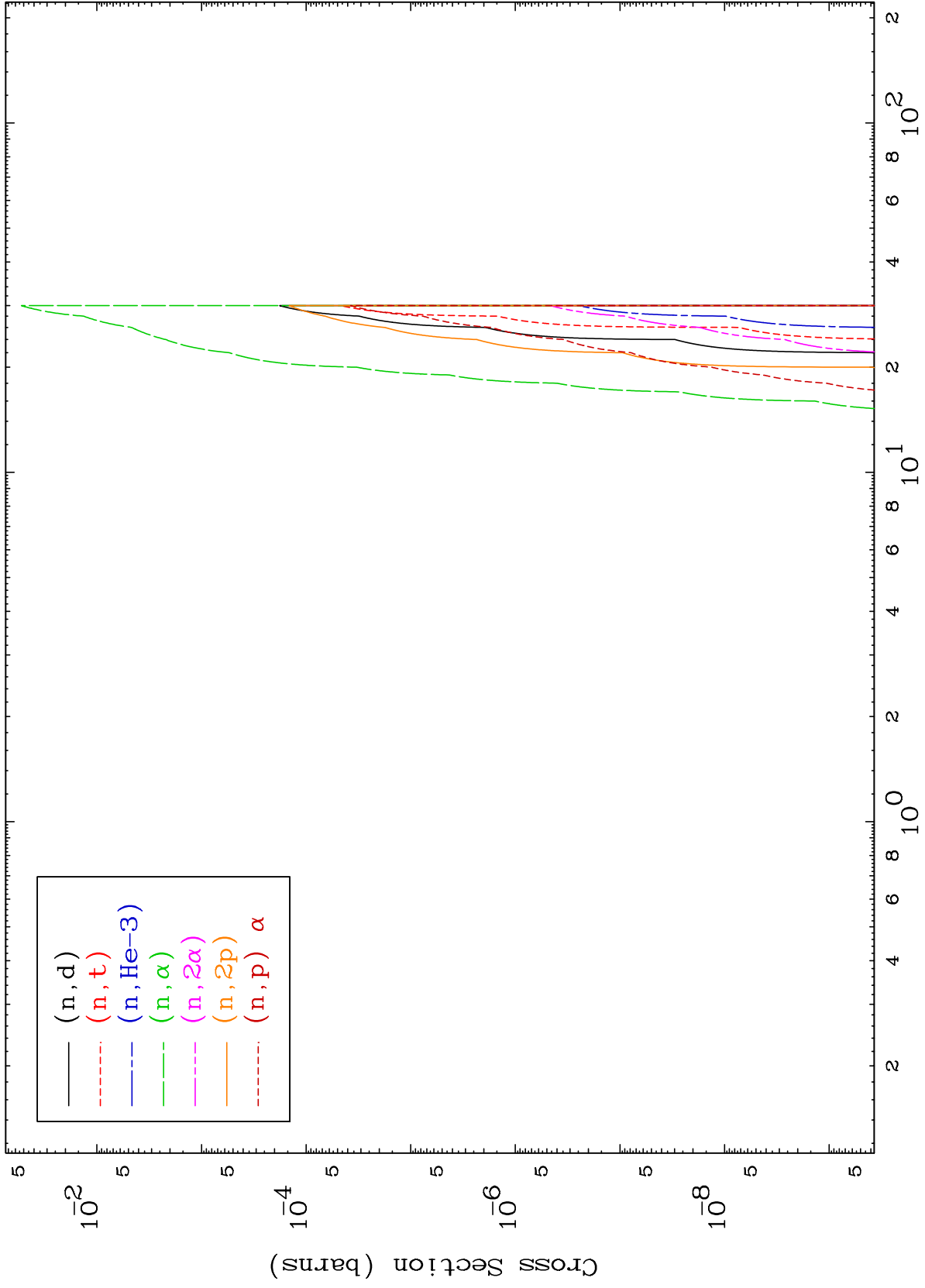


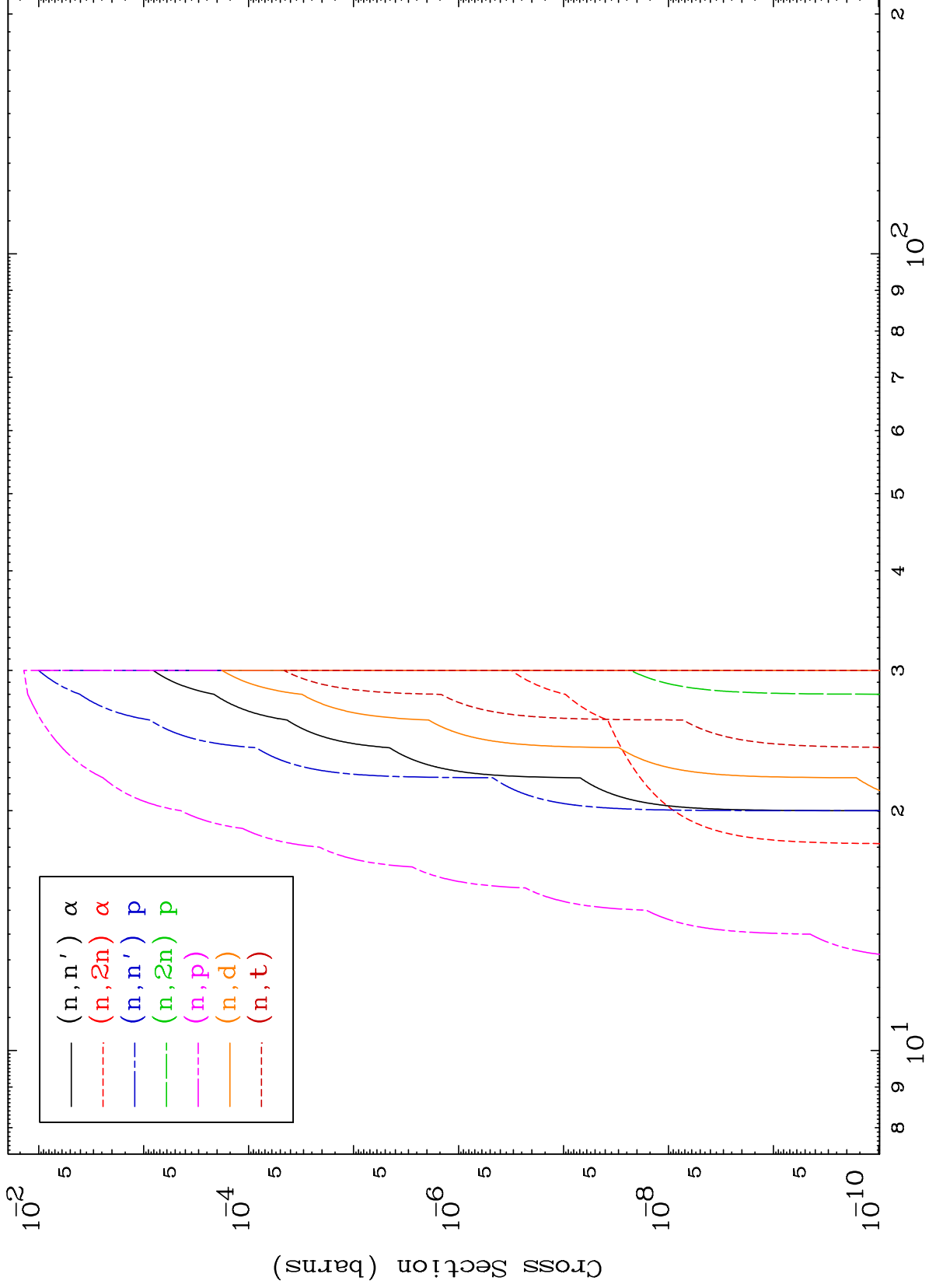
1

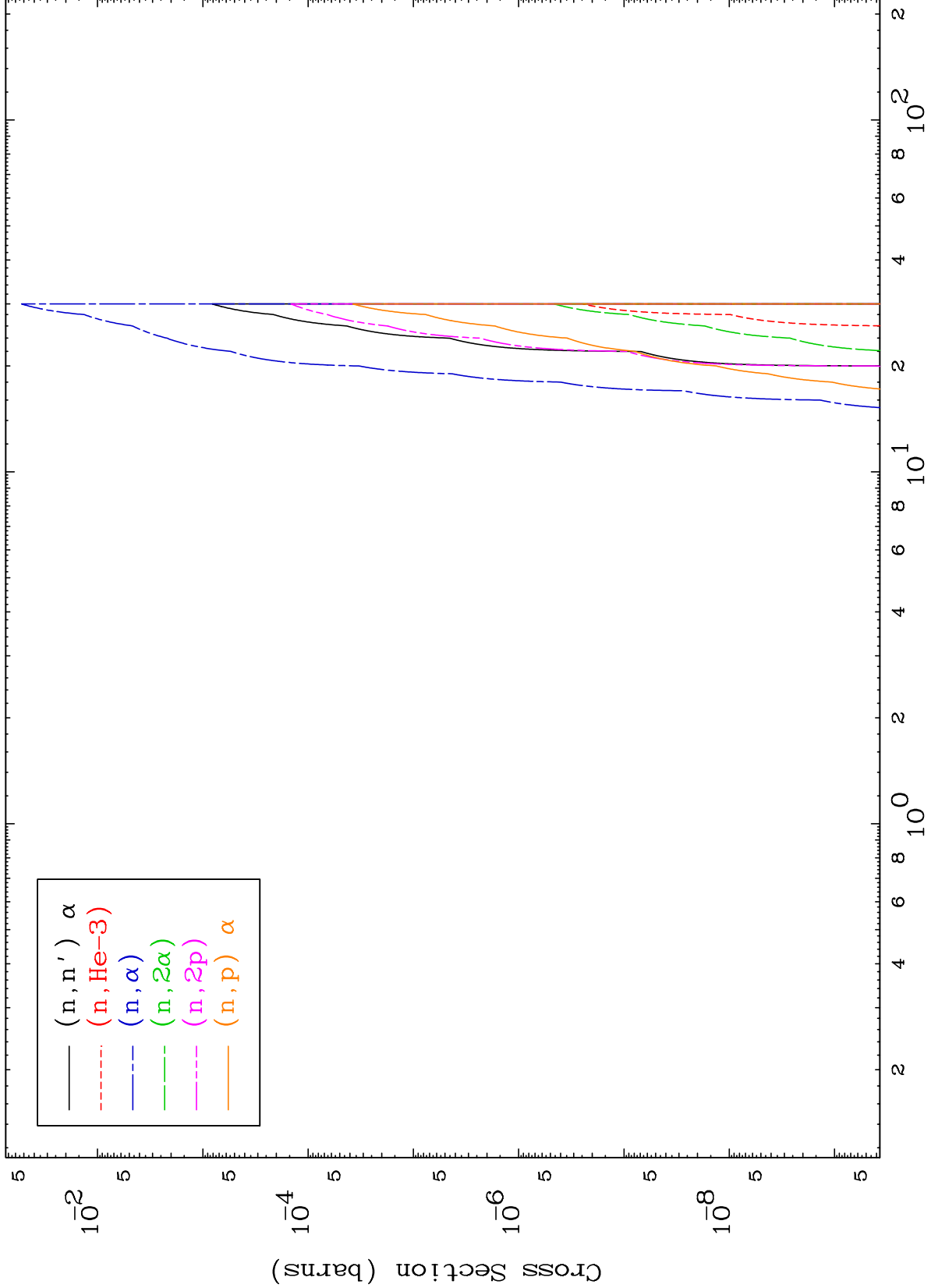
Incident Energy (MeV)

83-Bi-197





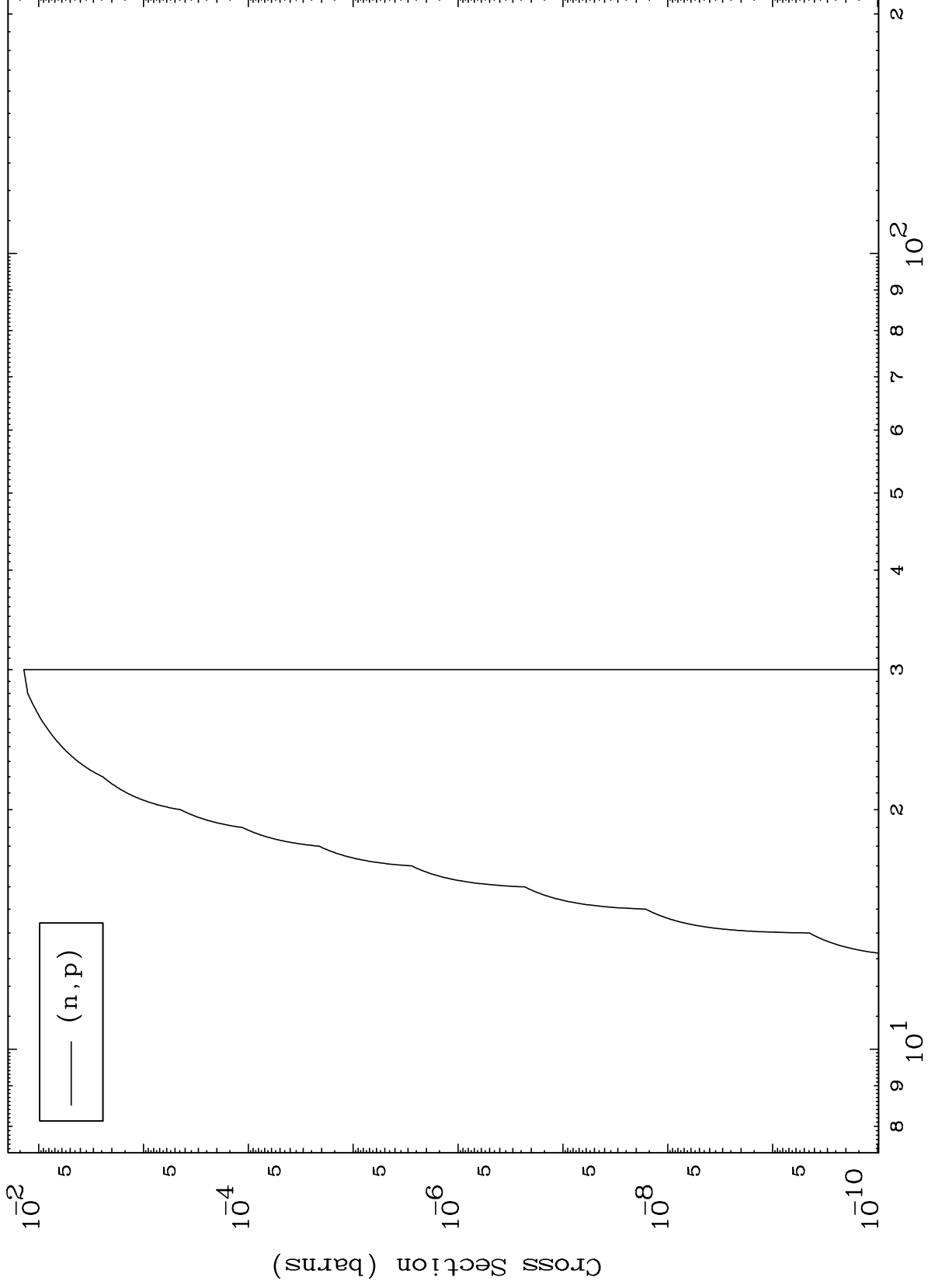




MAT 8289

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

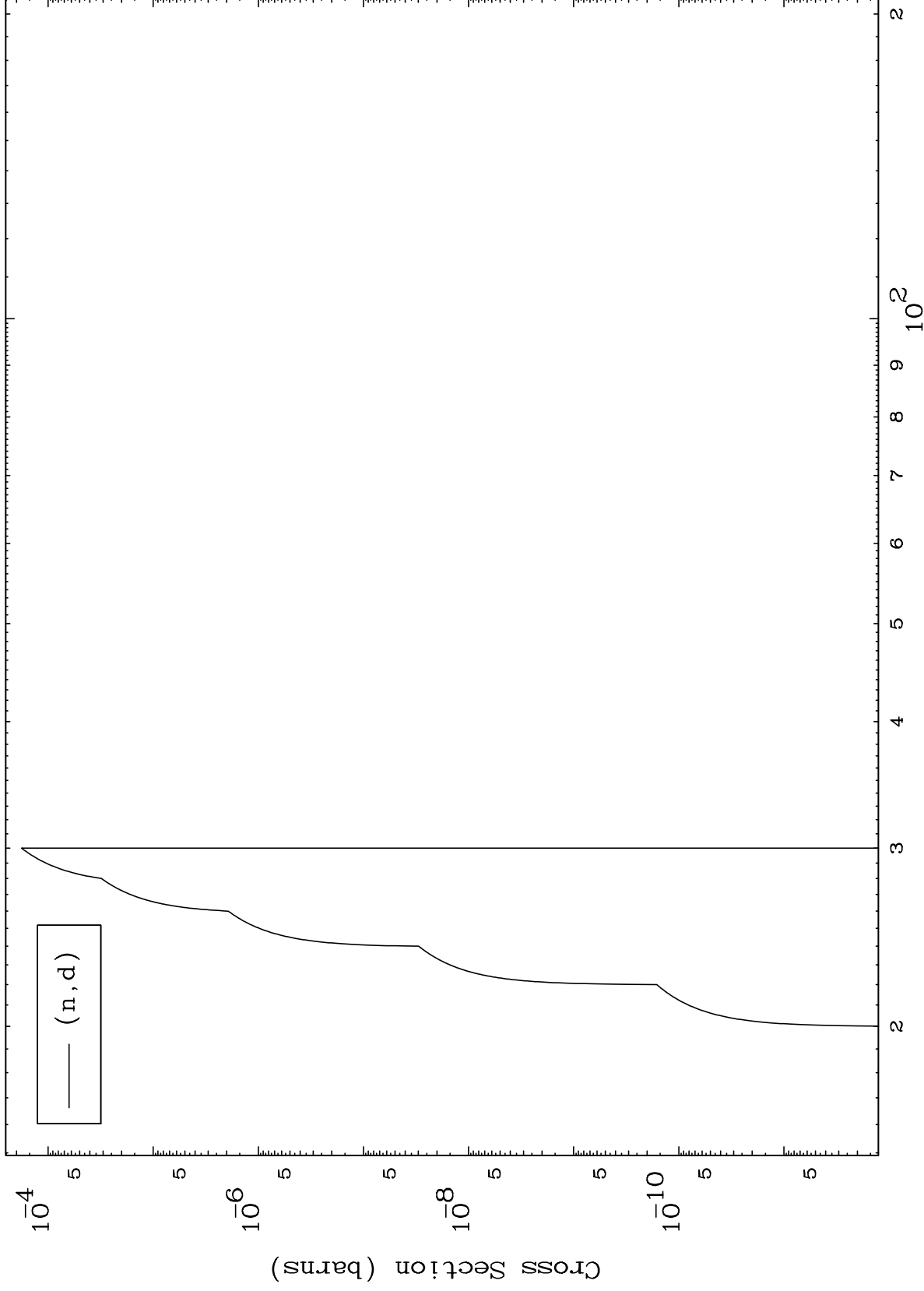
83-Bi-197



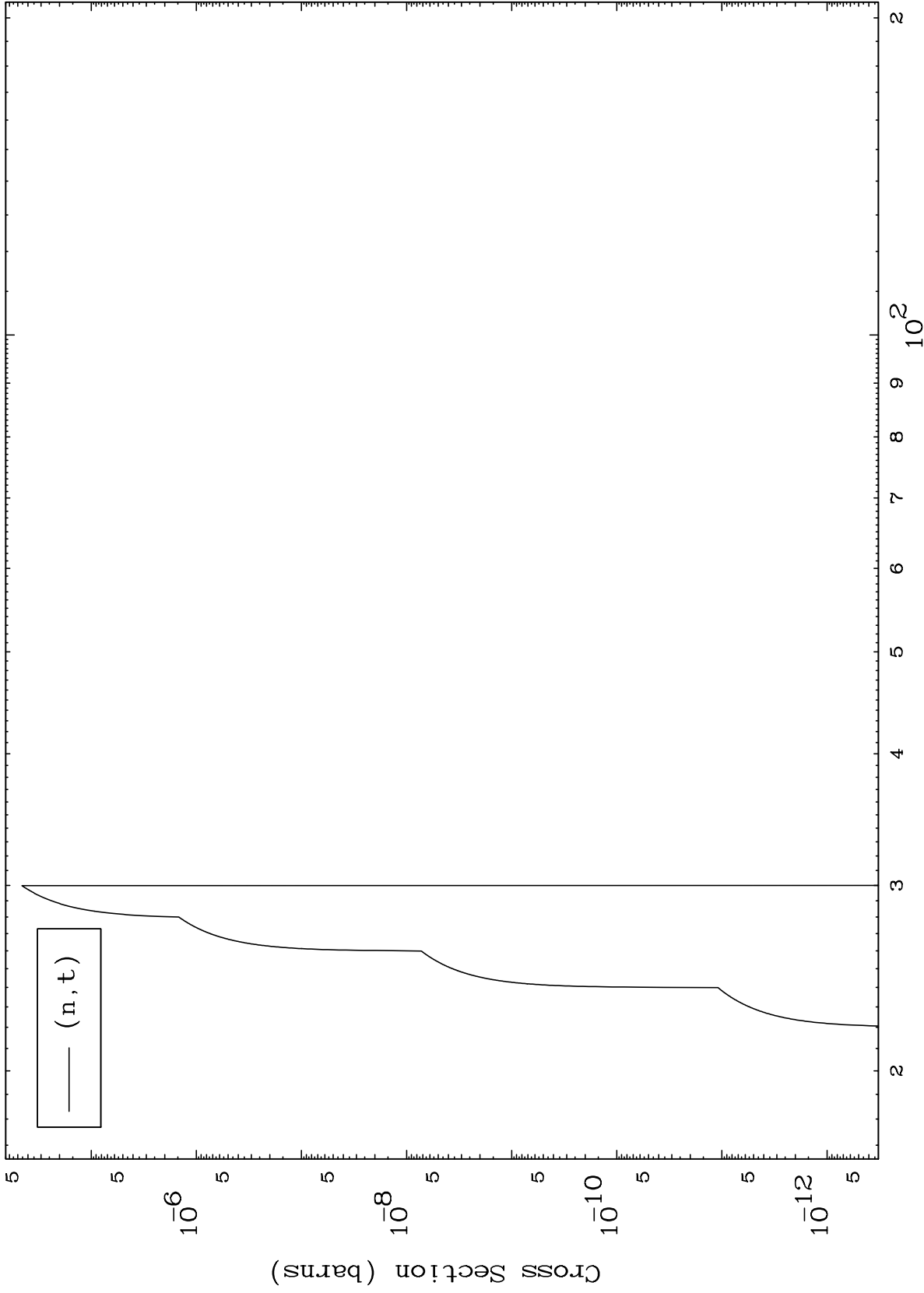
6

Incident Energy (MeV)

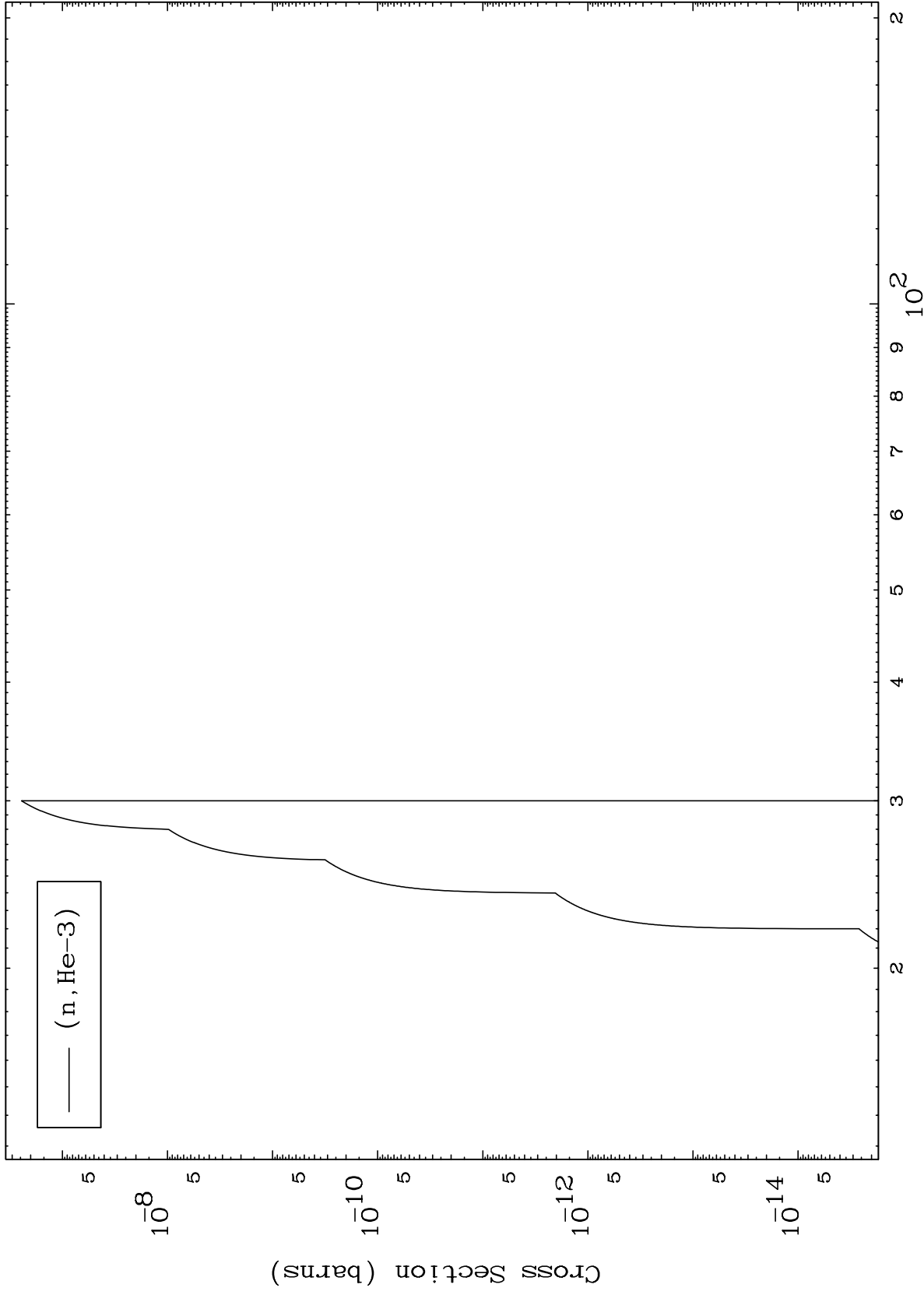
83-Bi-197







( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections



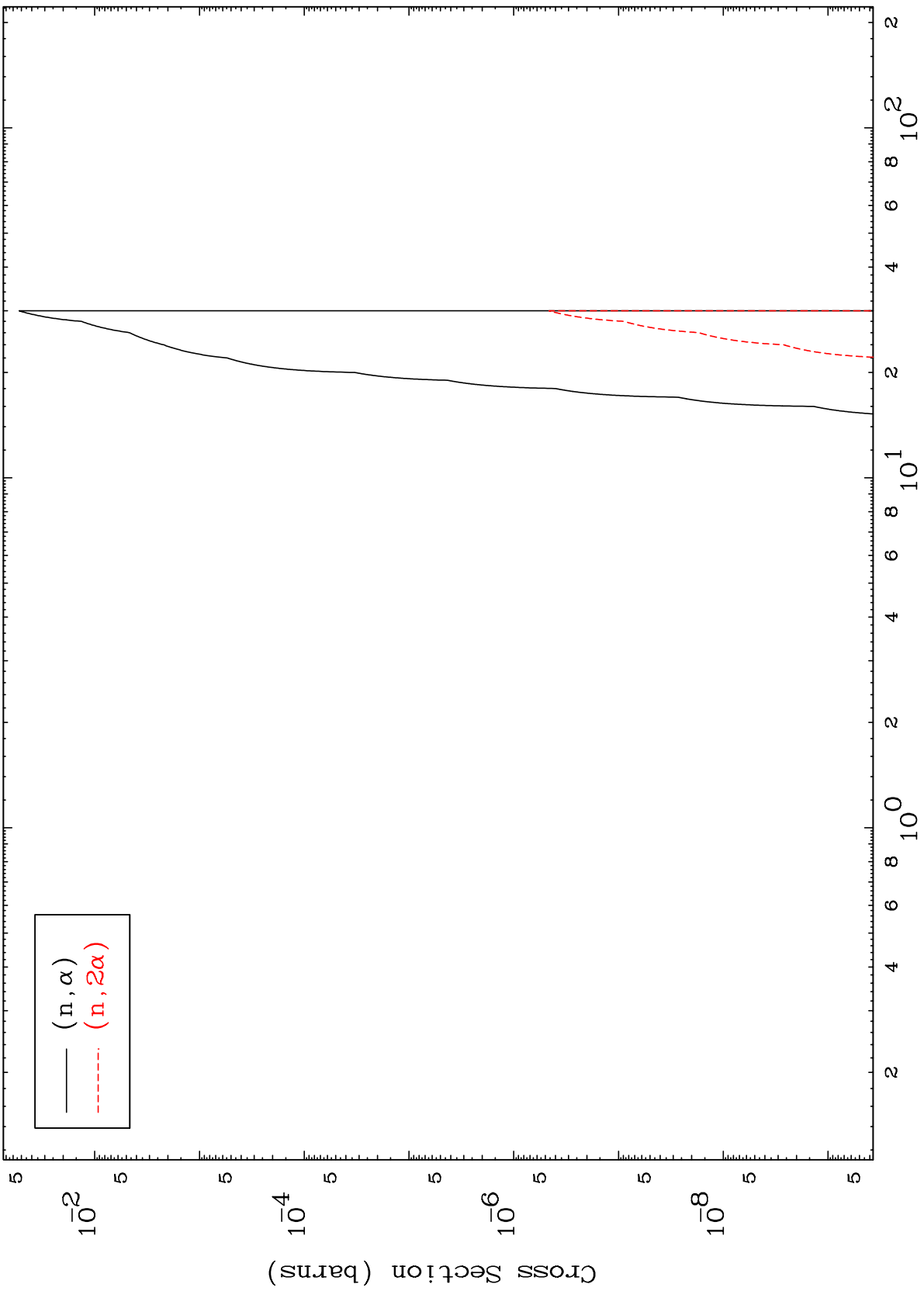
Incident Energy (MeV)

MAT 8289

( $\alpha, \alpha$ ) Levels

83-Bi-197

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

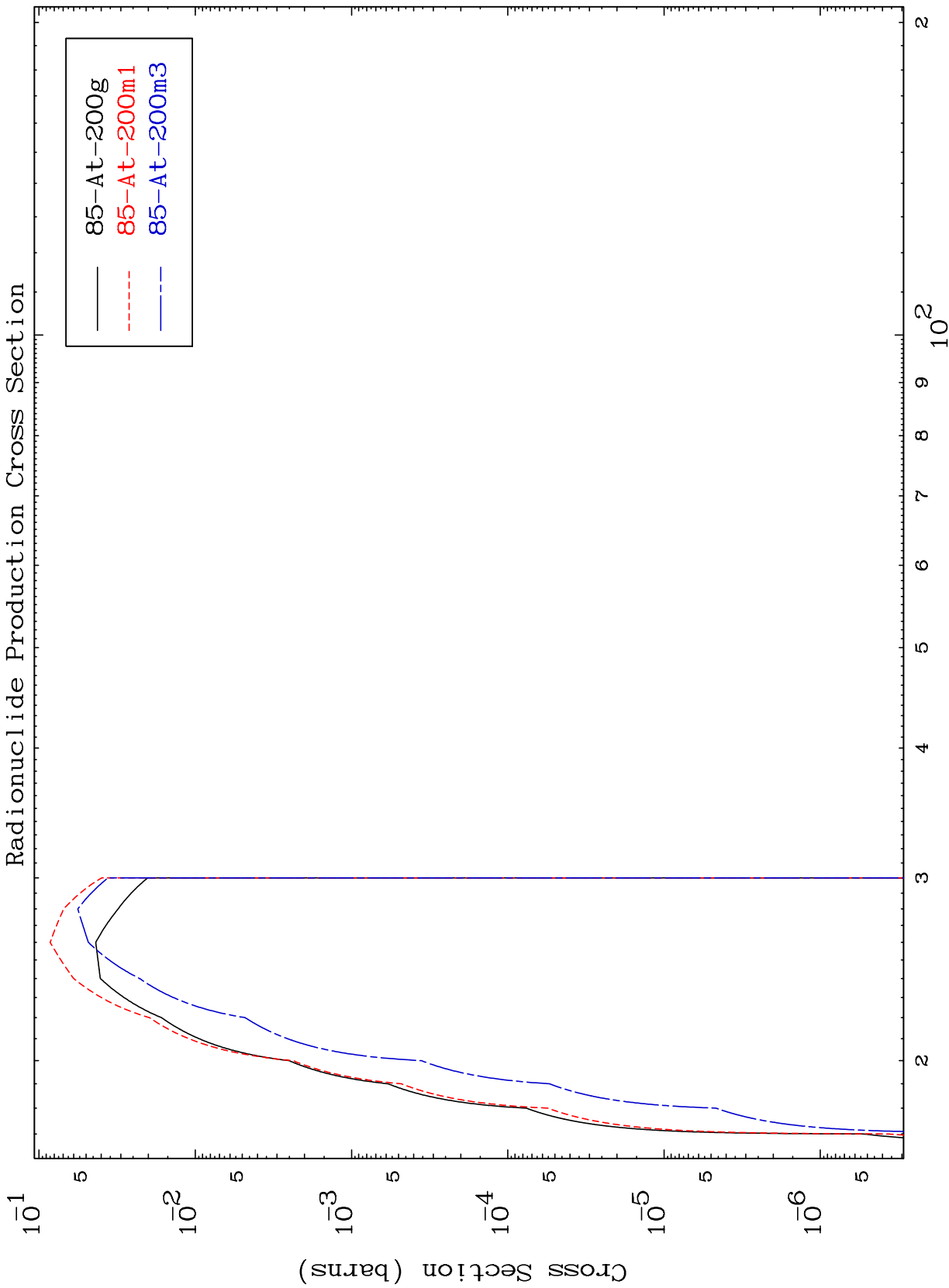
Incident Energy (MeV)

83-Bi-197

MAT 8289

83-Bi-197

Inelastic  
Radionuclide Production Cross Section



11

83-Bi-197

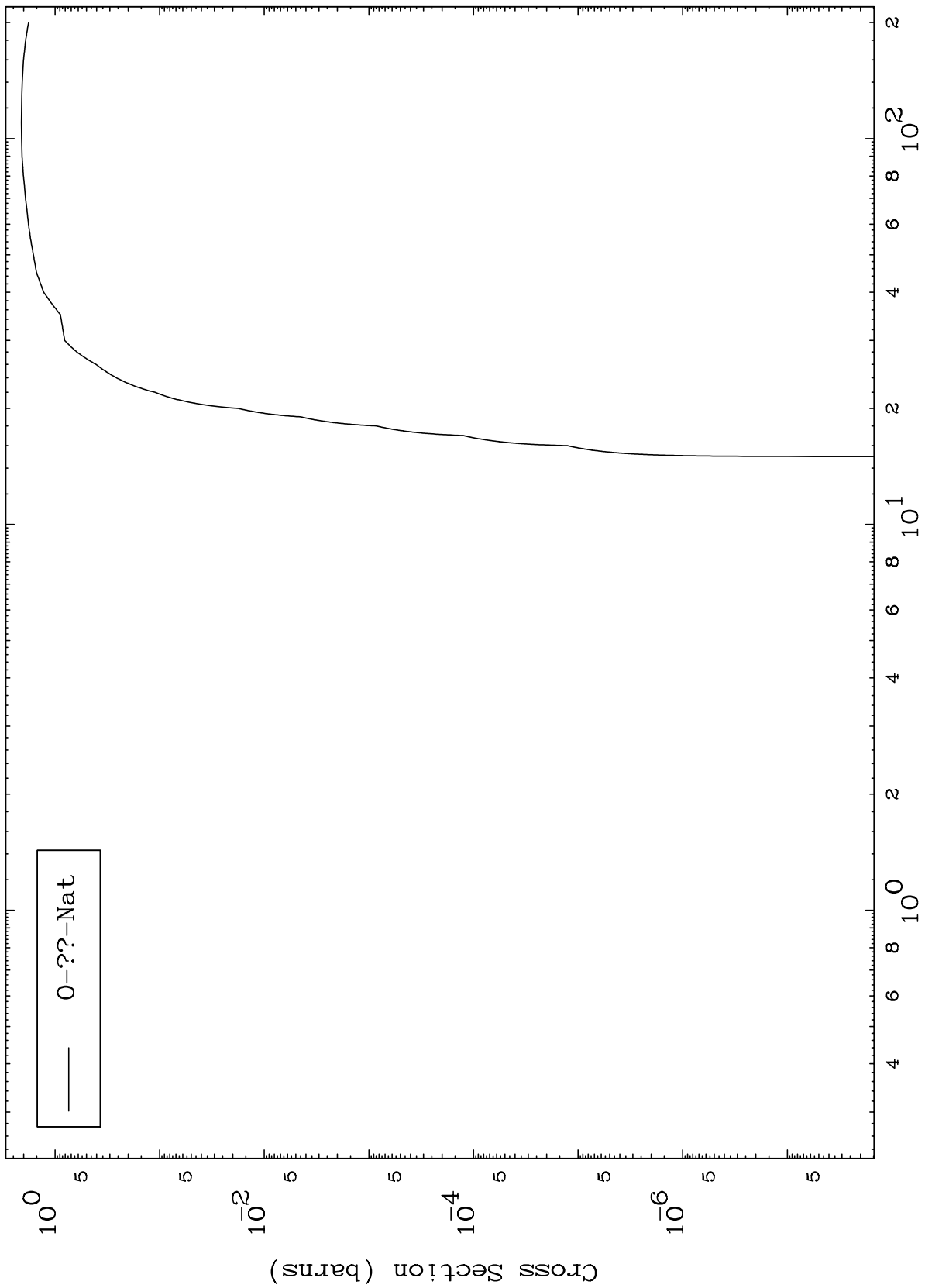
Incident Energy (MeV)

MAT 8289

Fission

83-Bi-197

Radionuclide Production Cross Section

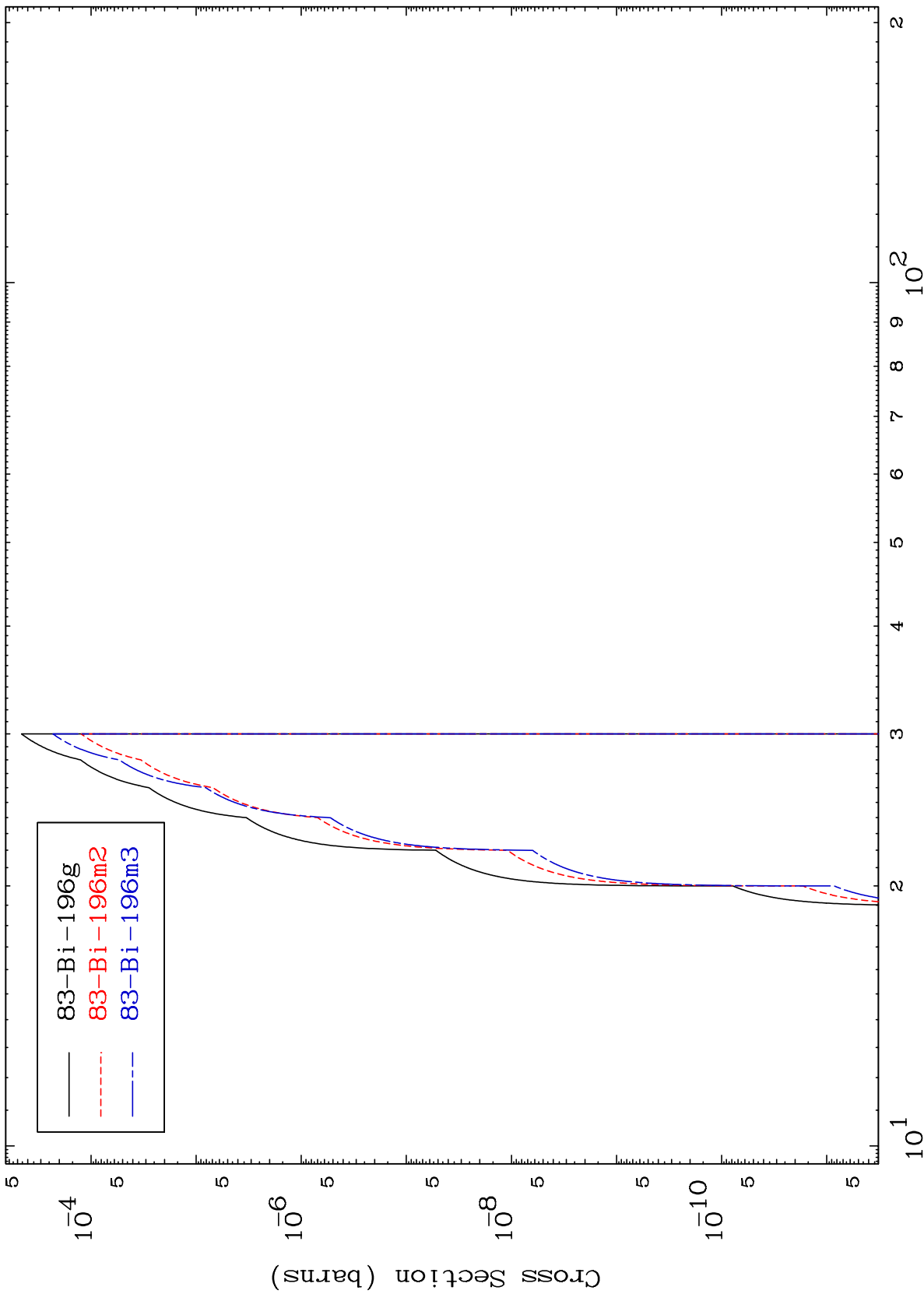


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$(n, n') \alpha$

83-Bi-197

Radionuclide Production Cross Section



Incident Energy (MeV)

83-Bi-197

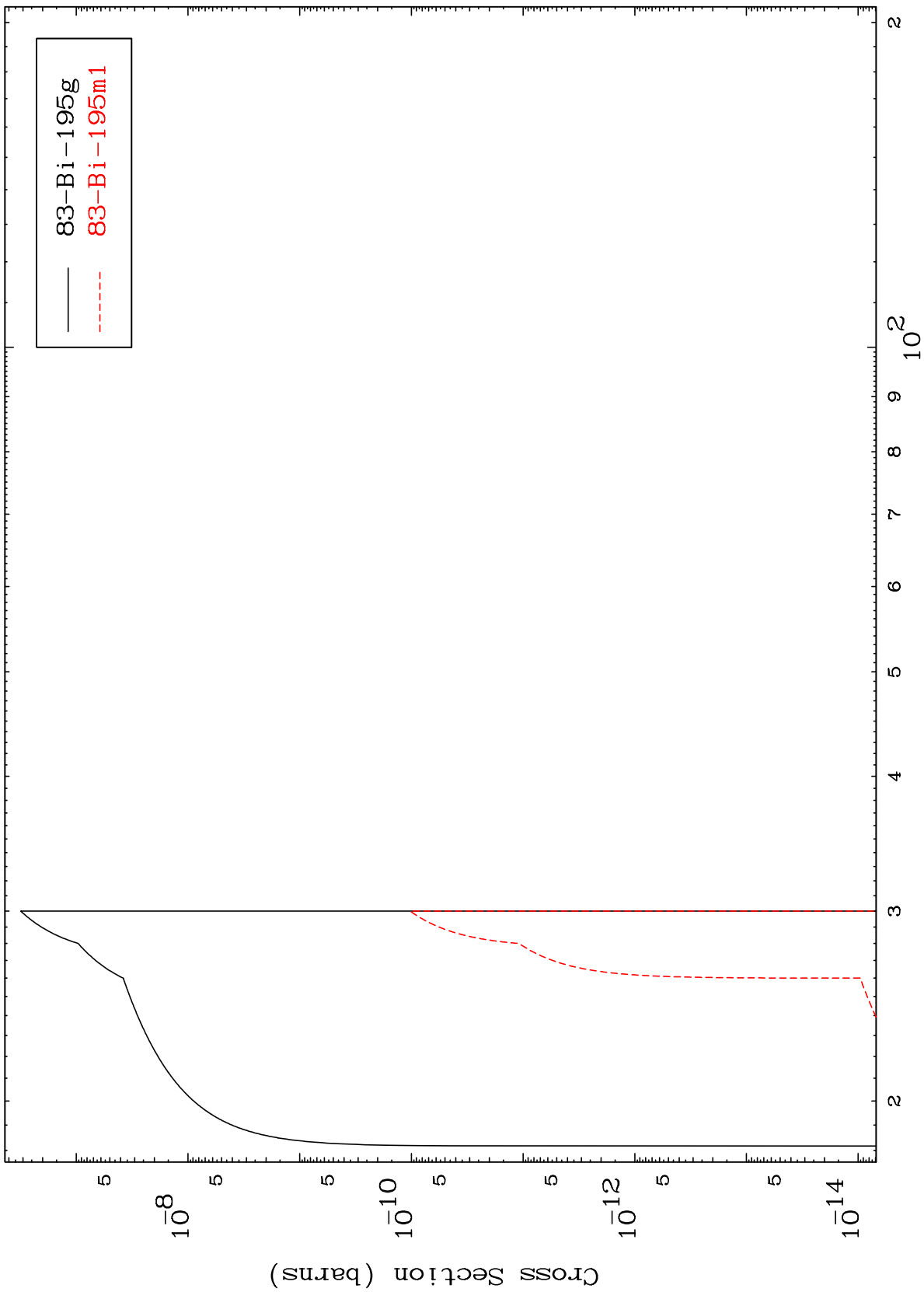
13

MAT 8289

$(n,2n) \alpha$

83-Bi-197

Radionuclide Production Cross Section

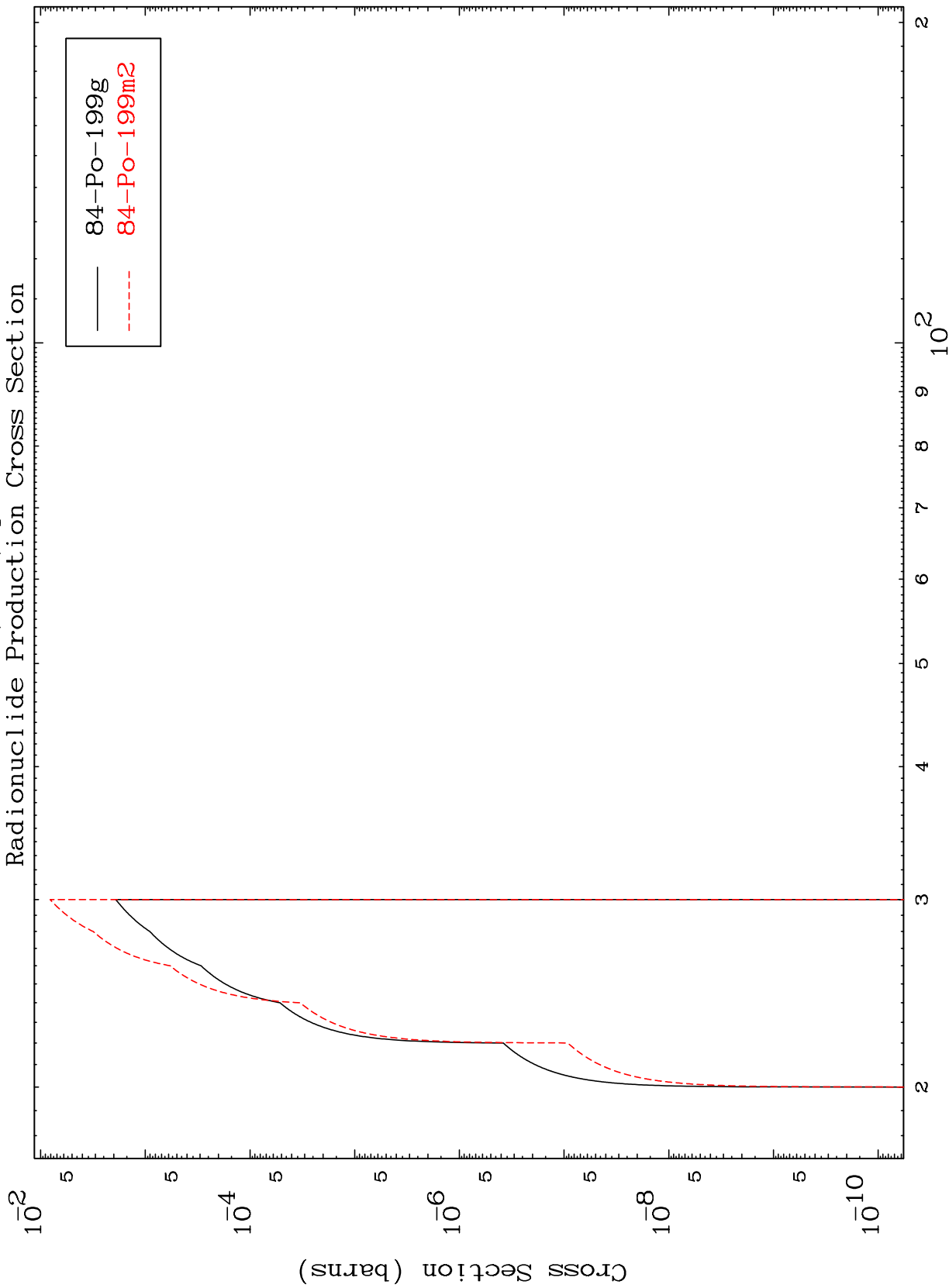


14

Incident Energy (MeV)

83-Bi-197

(n,n') p  
Radionuclide Production Cross Section



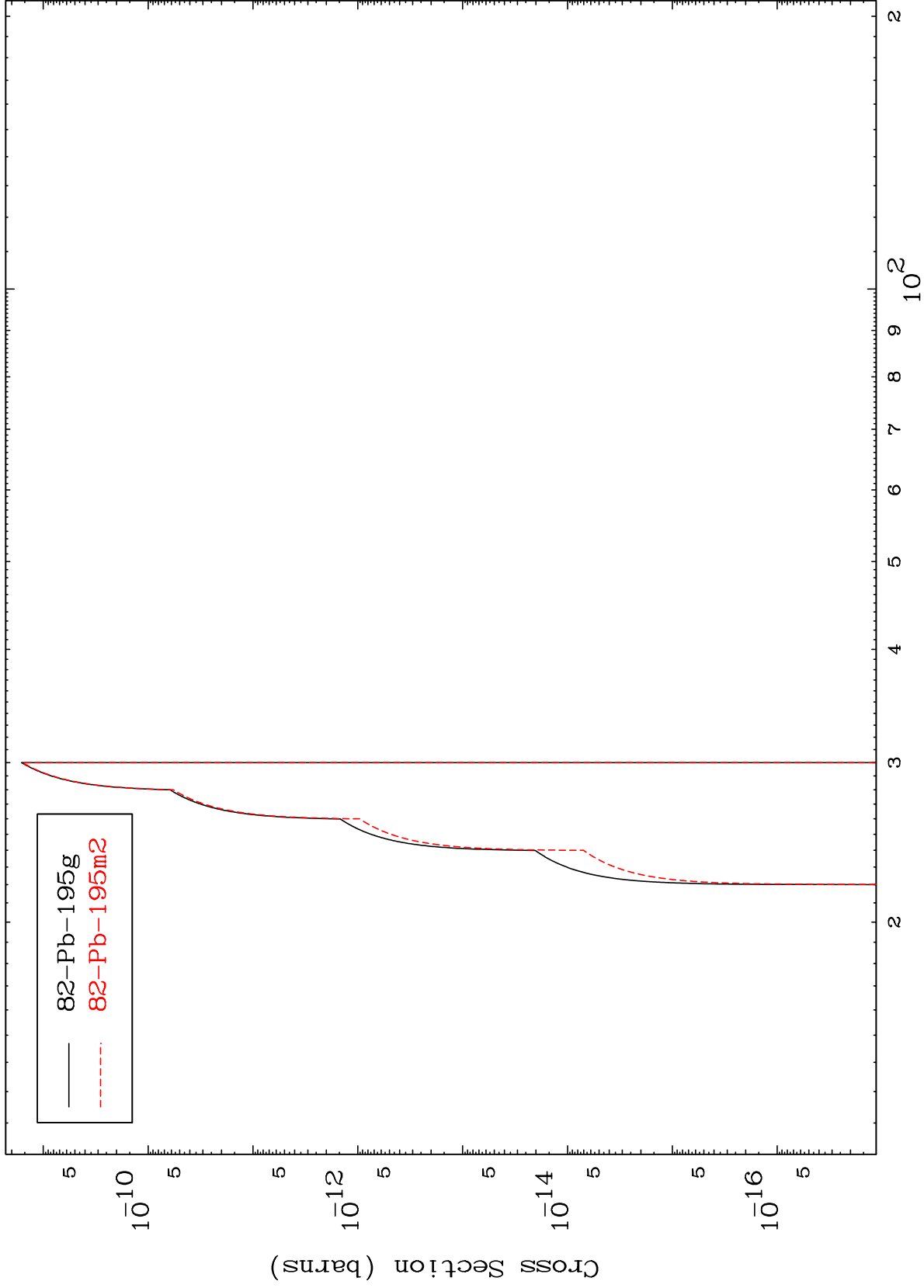


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(n,n') p  $\alpha$

83-Bi-197

Radionuclide Production Cross Section



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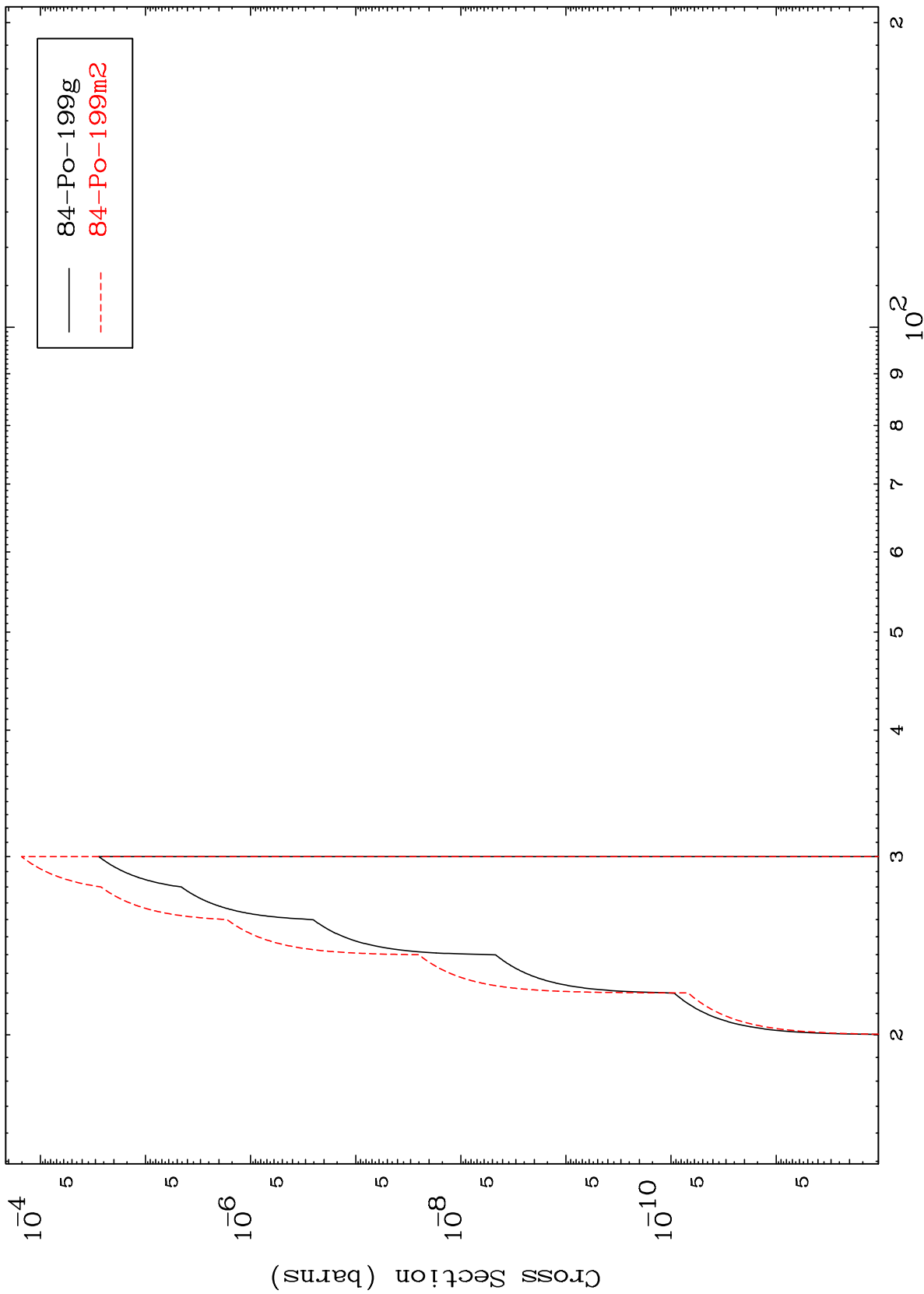
Incident Energy (MeV)

83-Bi-197

MAT 8289

83-Bi-197

(n,d)  
Radionuclide Production Cross Section



17

83-Bi-197

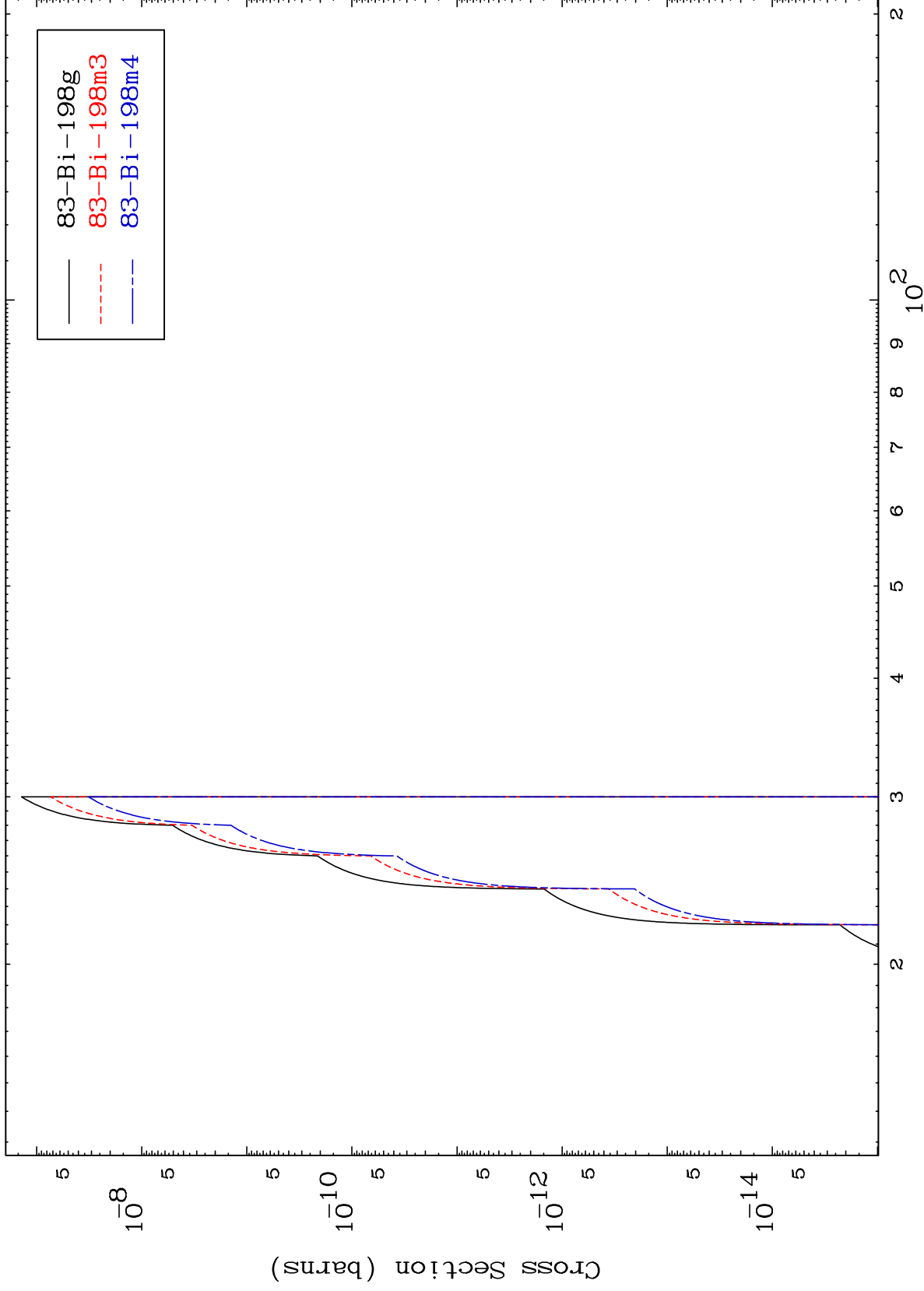
Incident Energy (MeV)

MAT 8289

(n,He-3)

83-Bi-197

Radionuclide Production Cross Section



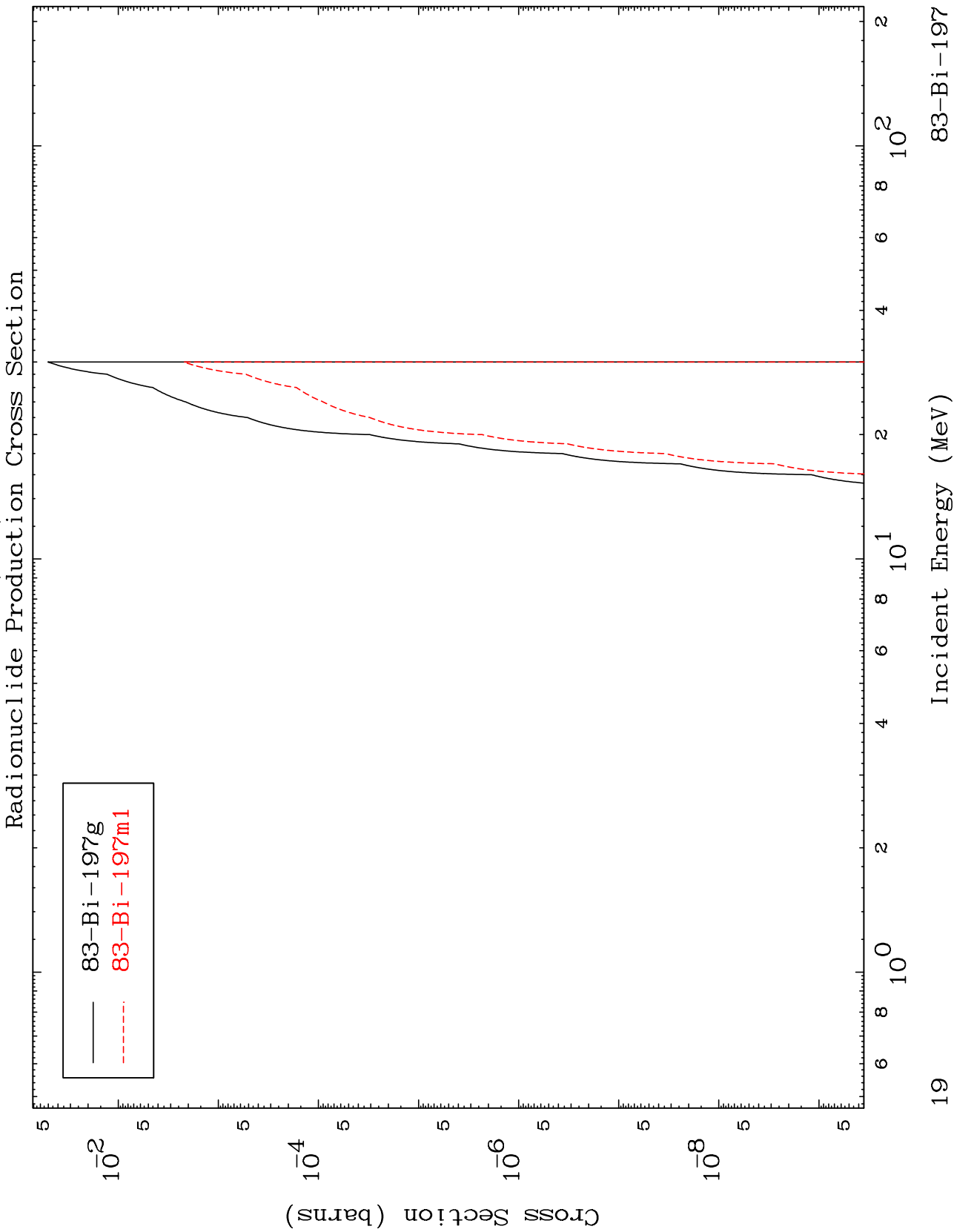
18

Incident Energy (MeV)

83-Bi-197

MAT 8289

83-Bi-197



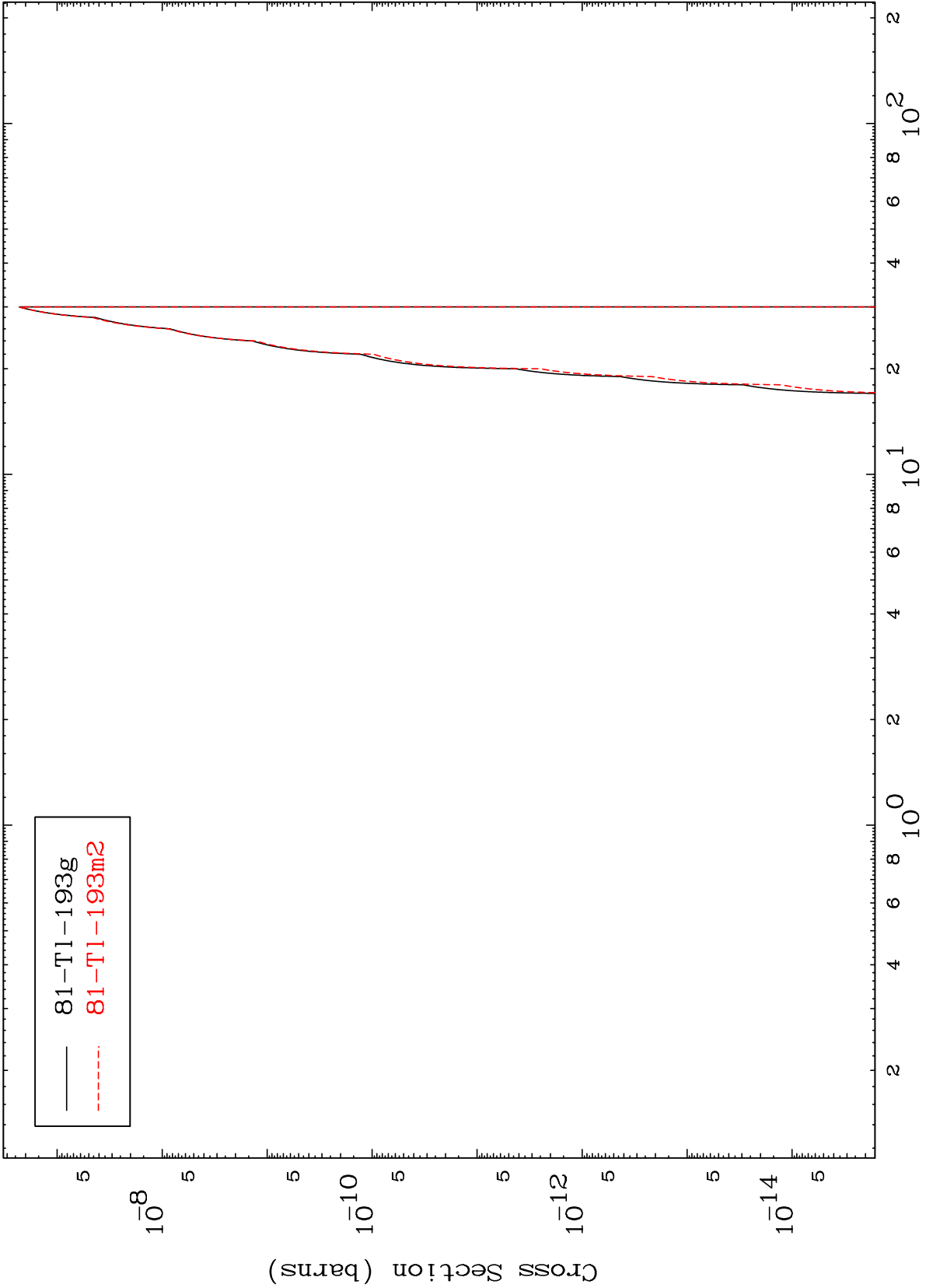
83-Bi-197g  
83-Bi-197m1

MAT 8289

(n,2α)

83-Bi-197

Radionuclide Production Cross Section

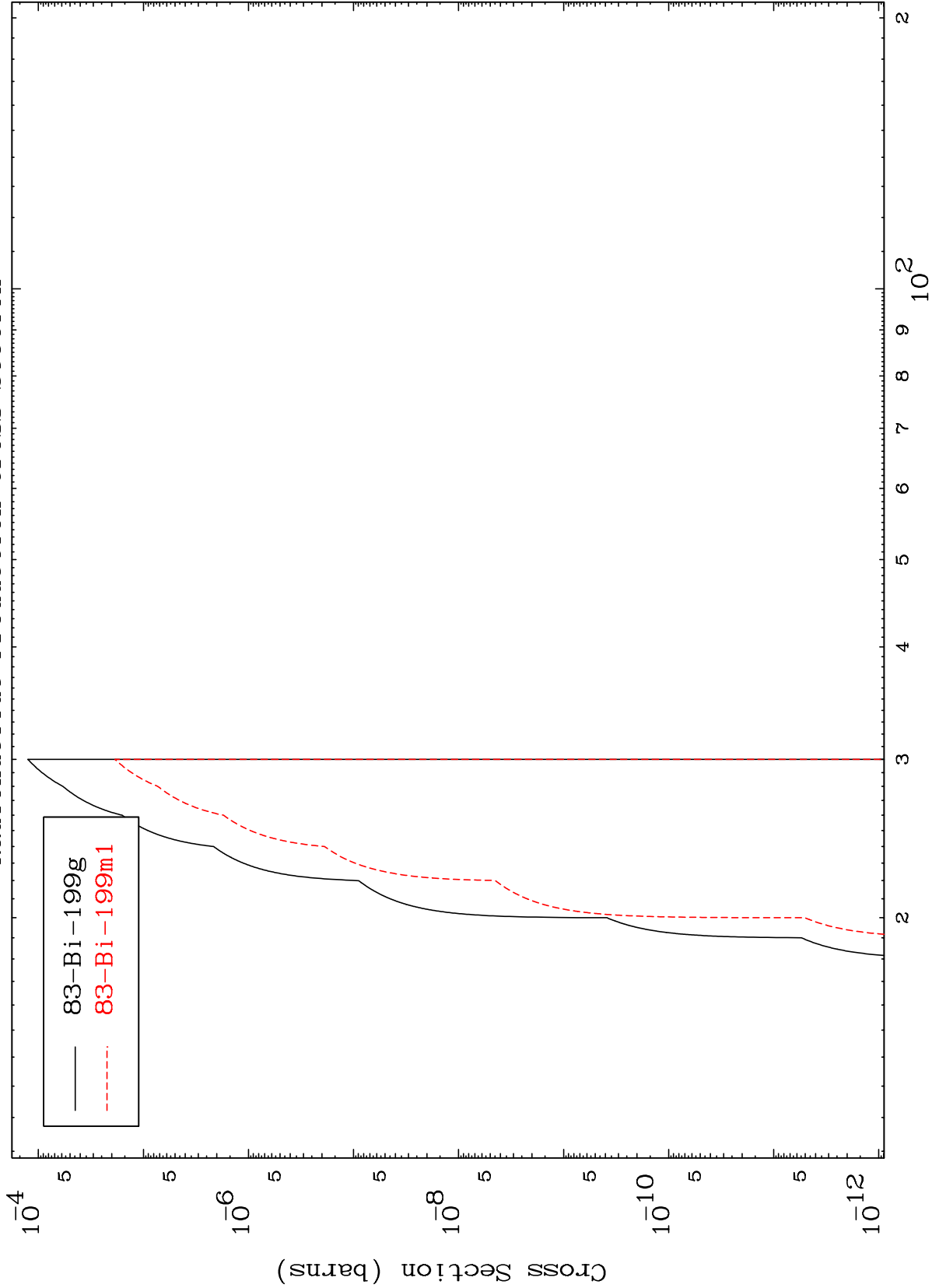


81-Tl-193g  
81-Tl-193m2

MAT 8289

83-Bi-197

(n,2p)  
Radionuclide Production Cross Section



83-Bi-197

Incident Energy (MeV)

21