

Program EVALPLOT  
(Version 2021-1)

by

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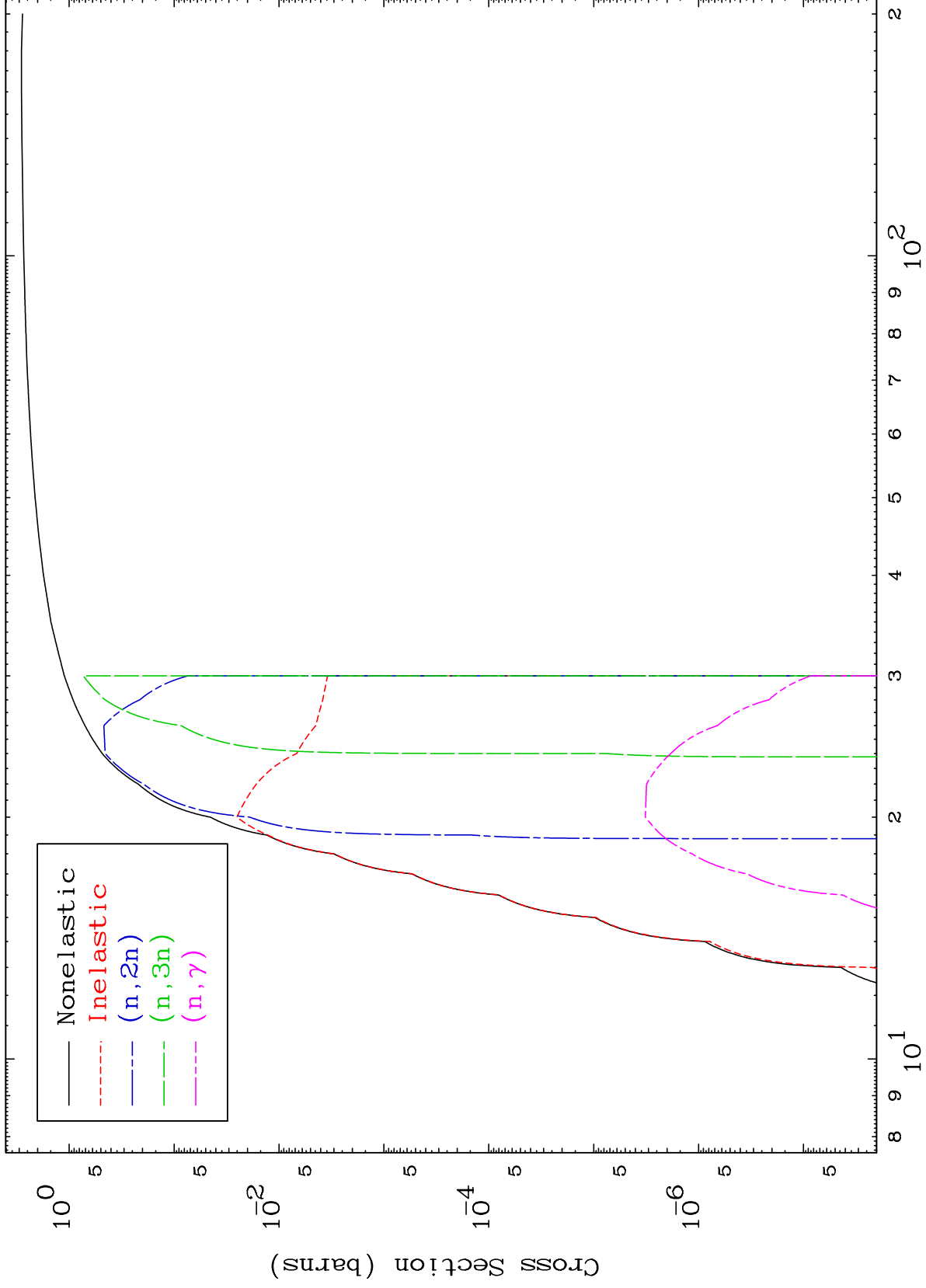
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8334

0 Kelvin  
 $\alpha$  Major Cross Sections

83-Bi-212



1

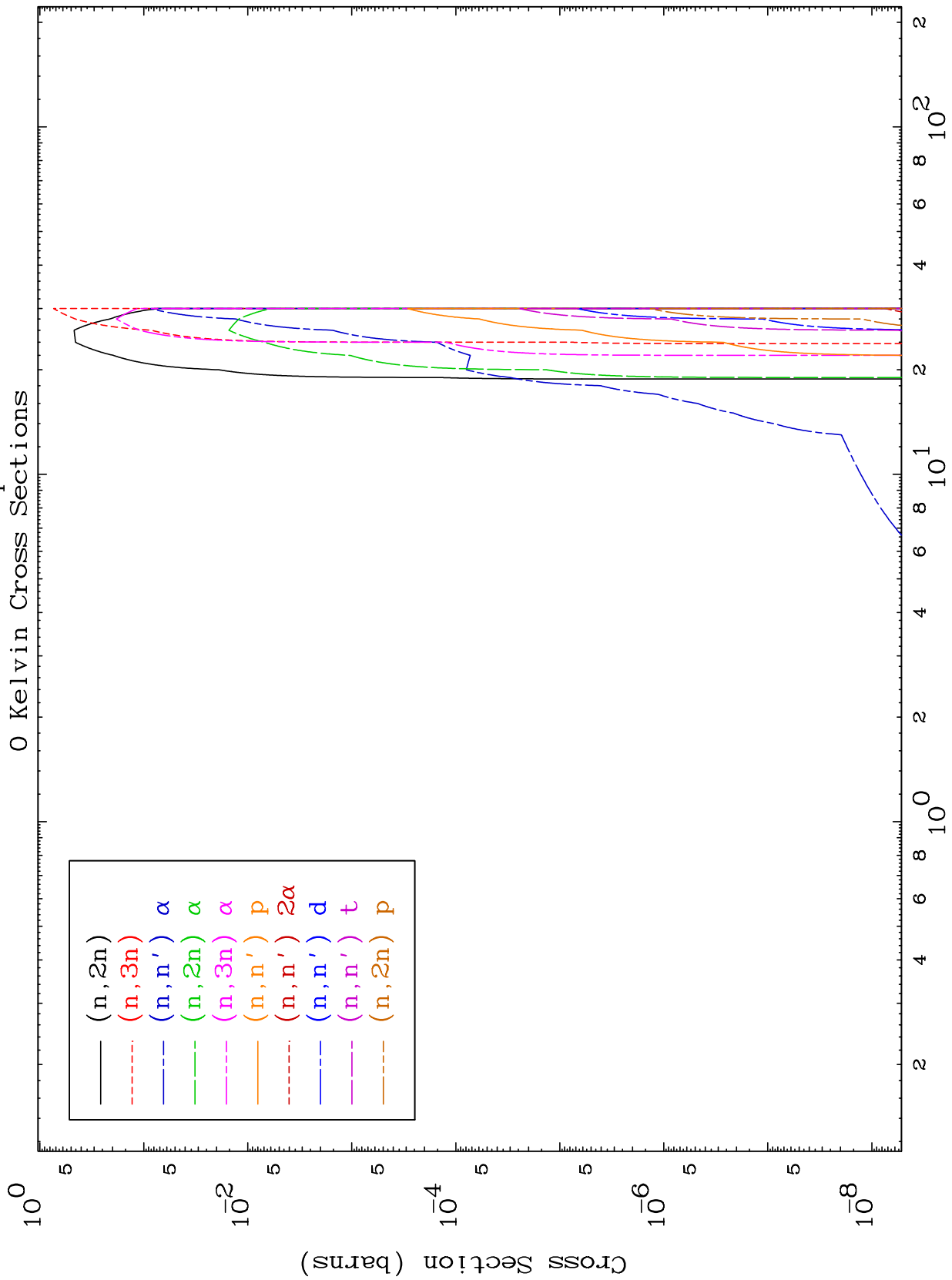
Incident Energy (MeV)

83-Bi-212

MAT 8334

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-212

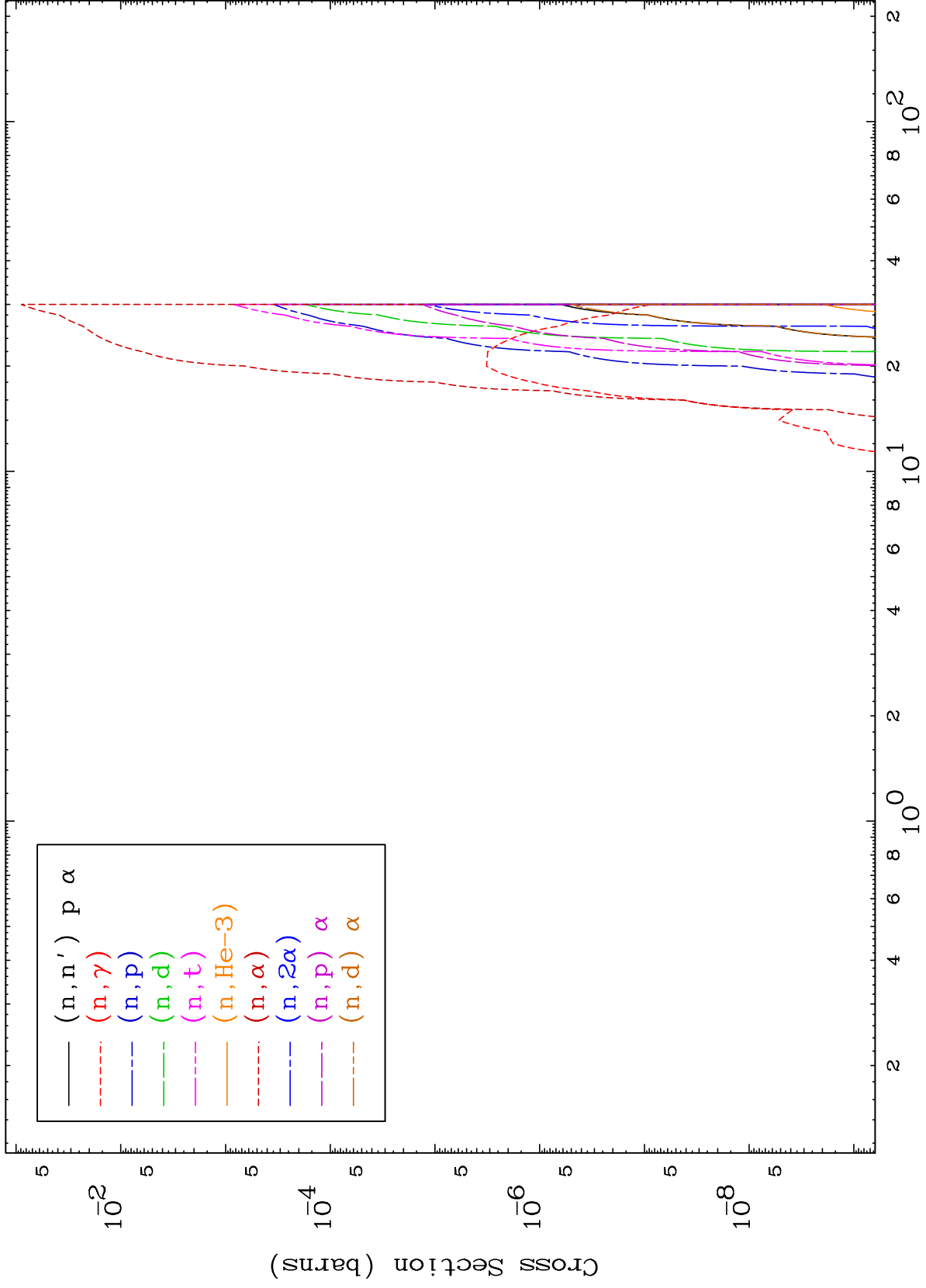


—	(n, 2n)
- - -	(n, 3n)
- · - ·	(n, n')
- · - ·	(n, 2n)
- · - ·	(n, 3n)
- · - ·	(n, n')
- · - ·	(n, n')
- · - ·	(n, n')
- · - ·	(n, n')
- · - ·	(n, n')
- · - ·	(n, 2n)

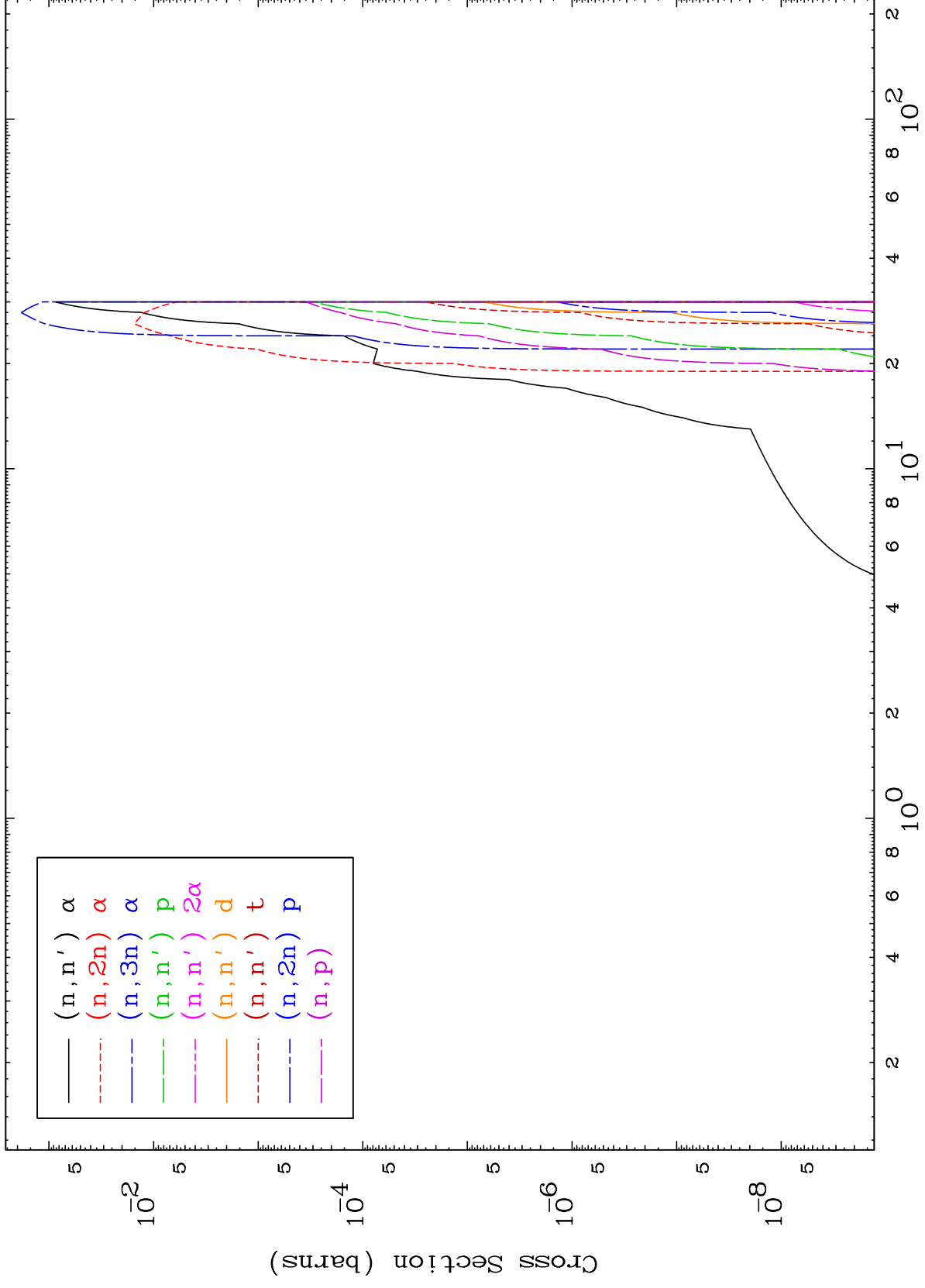
83-Bi-212

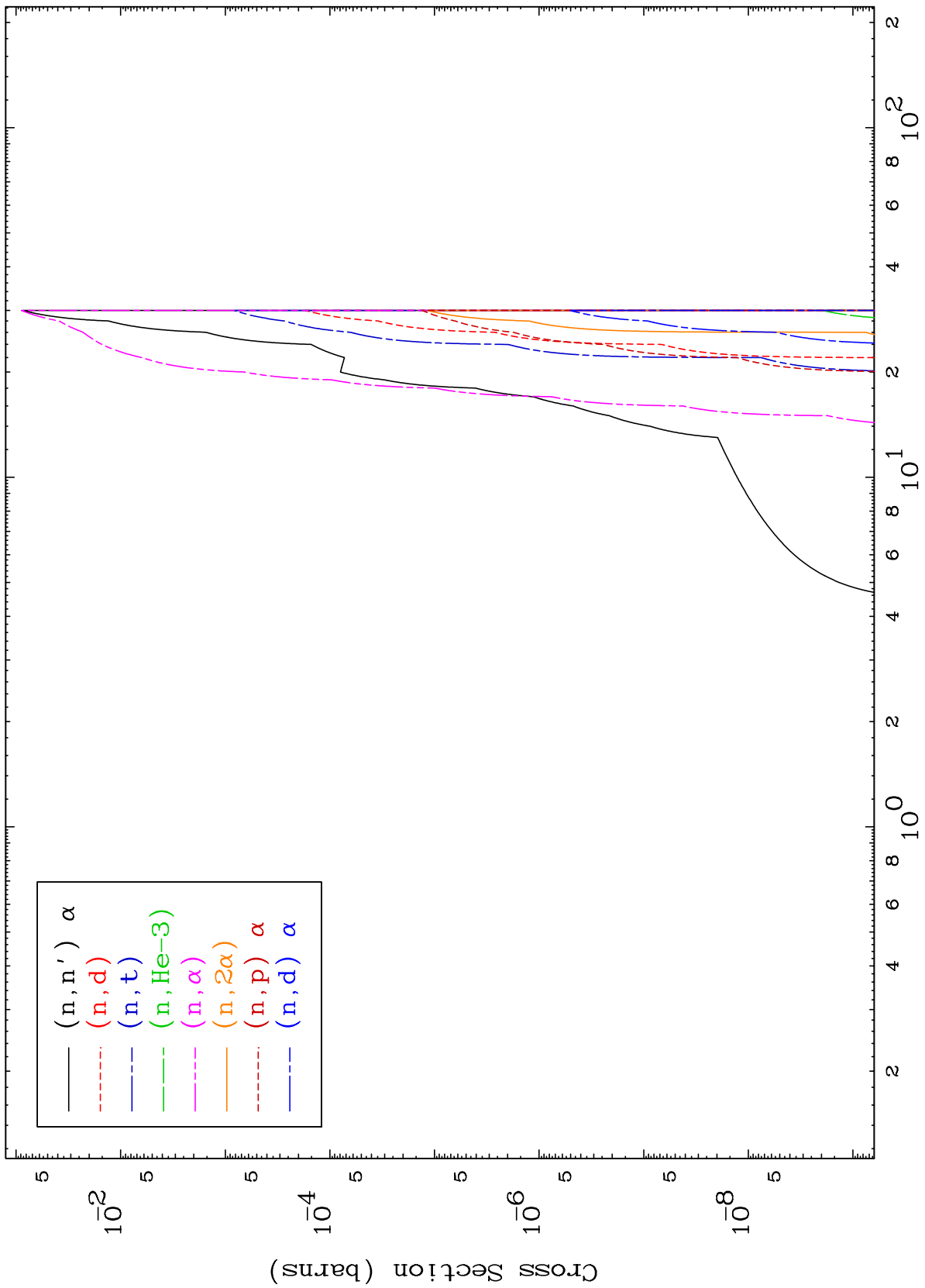
Incident Energy (MeV)

2



0 Kelvin Cross Sections

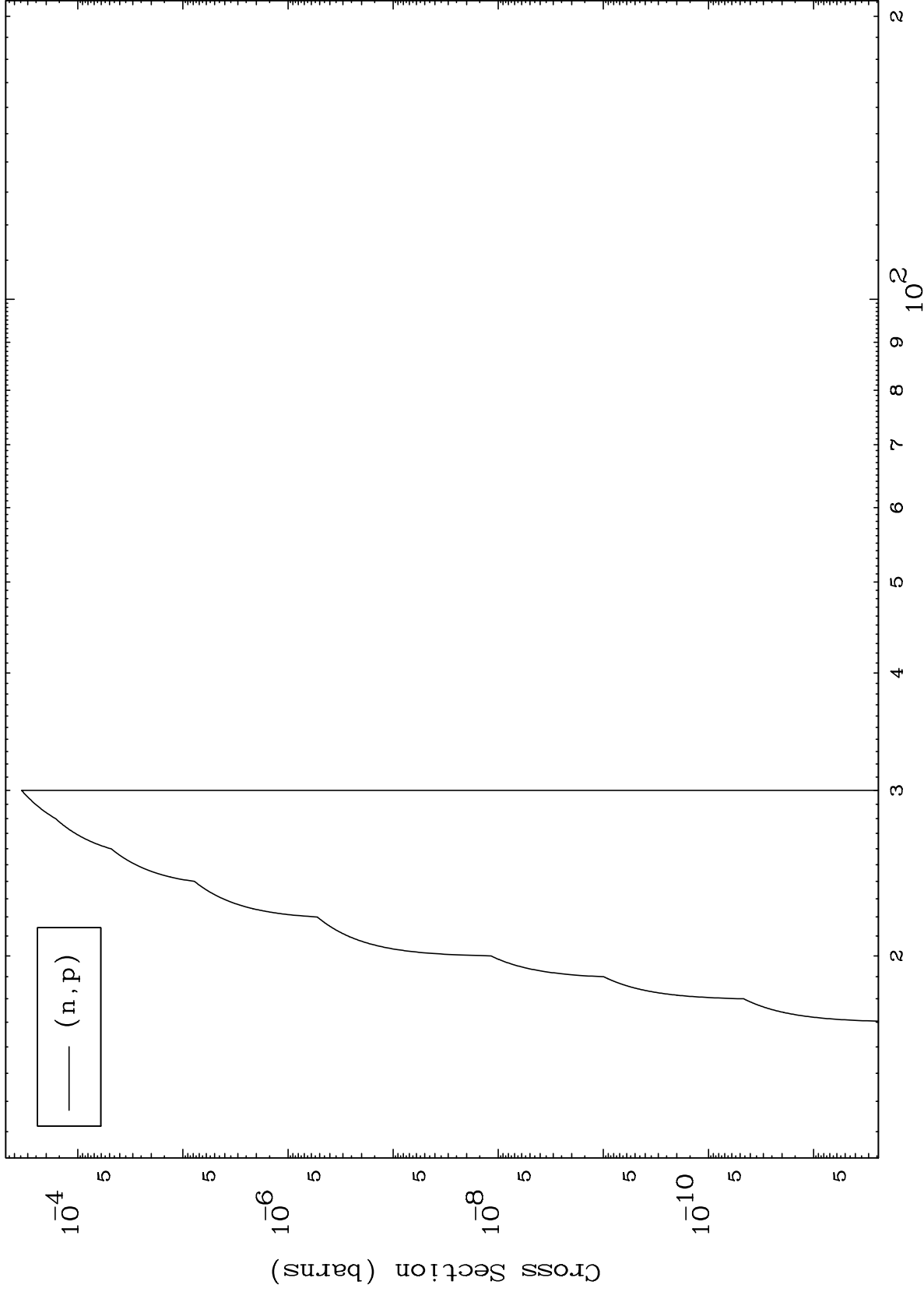




MAT 8334

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

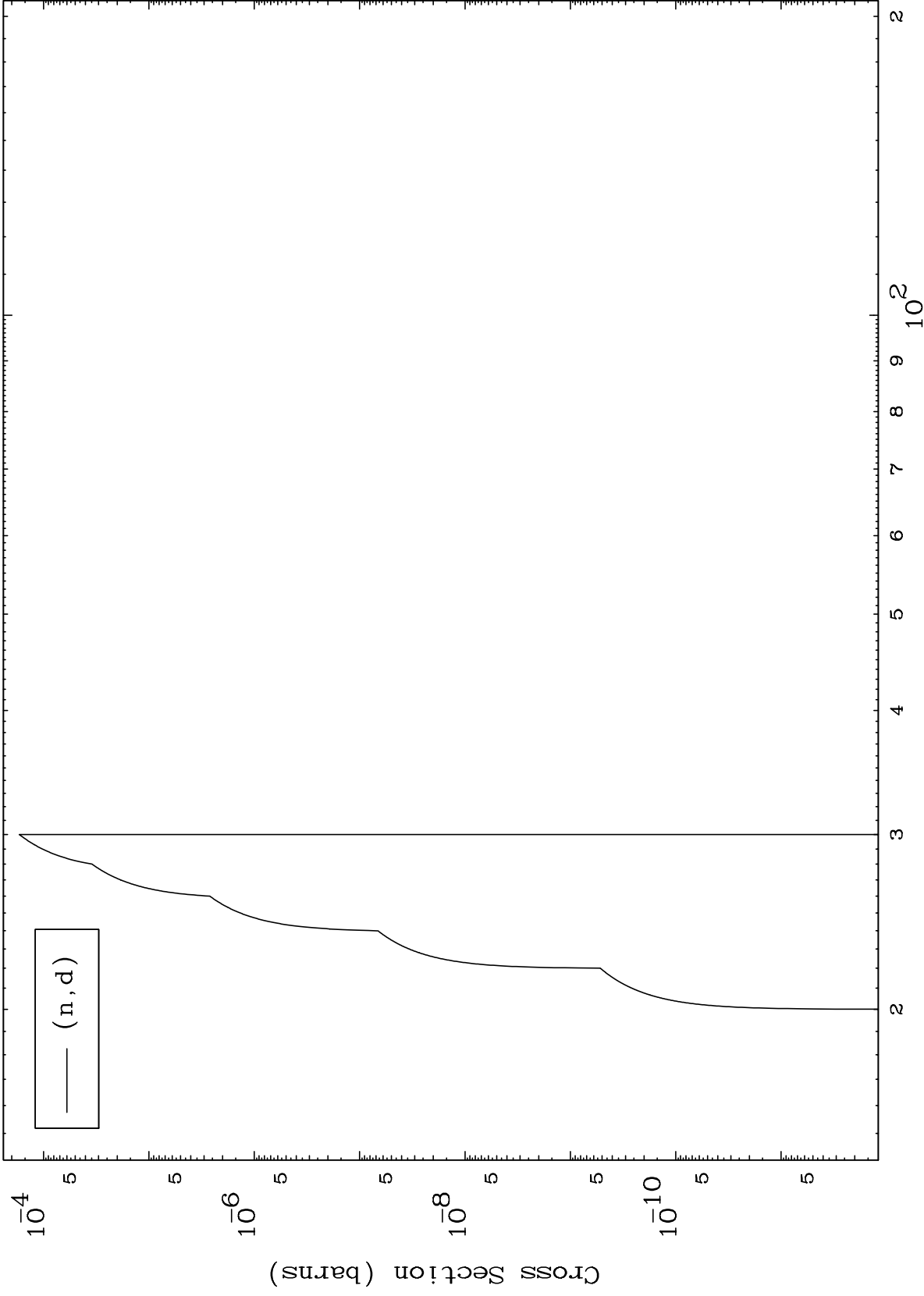
83-Bi-212



MAT 8334

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

83-Bi-212

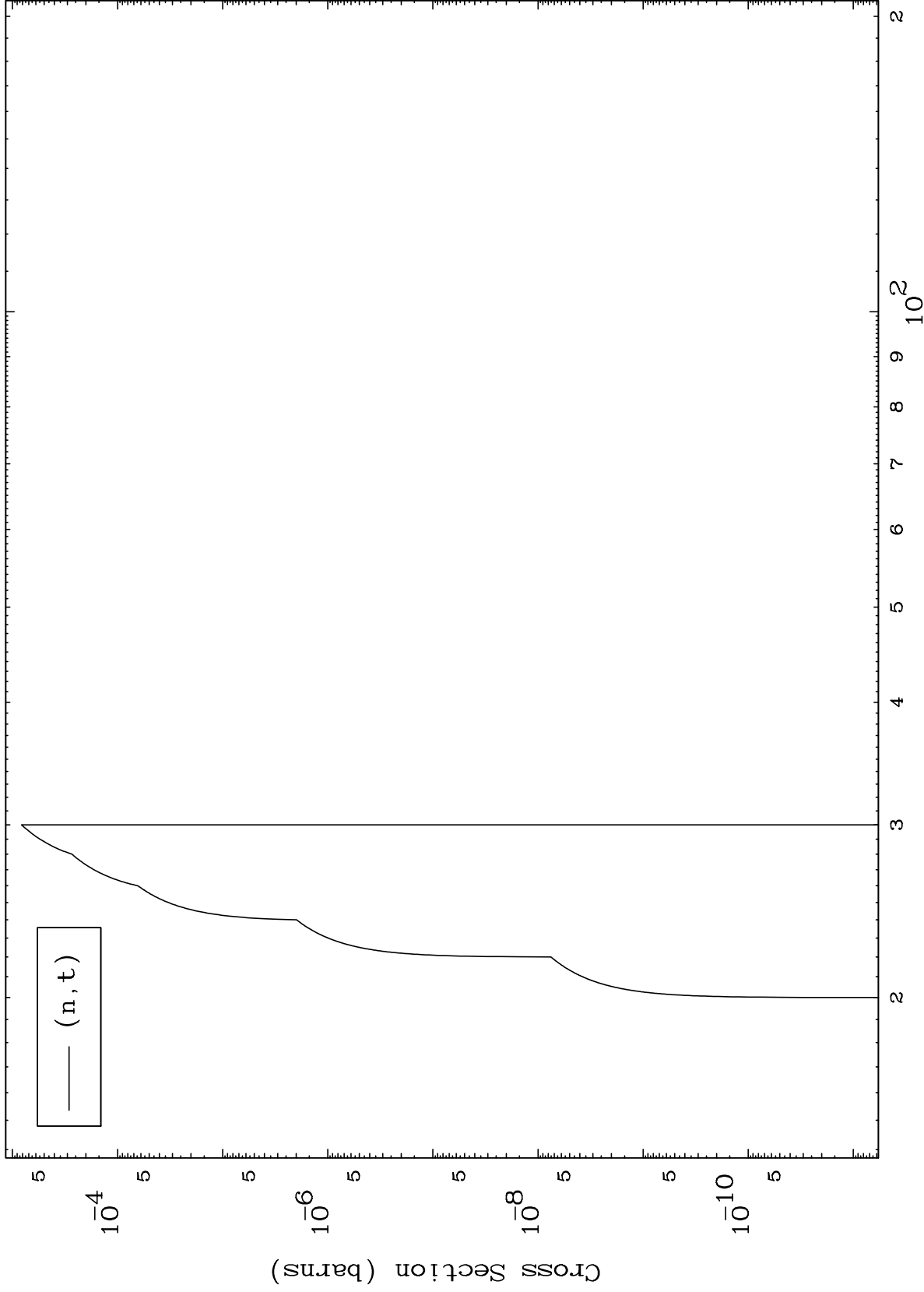


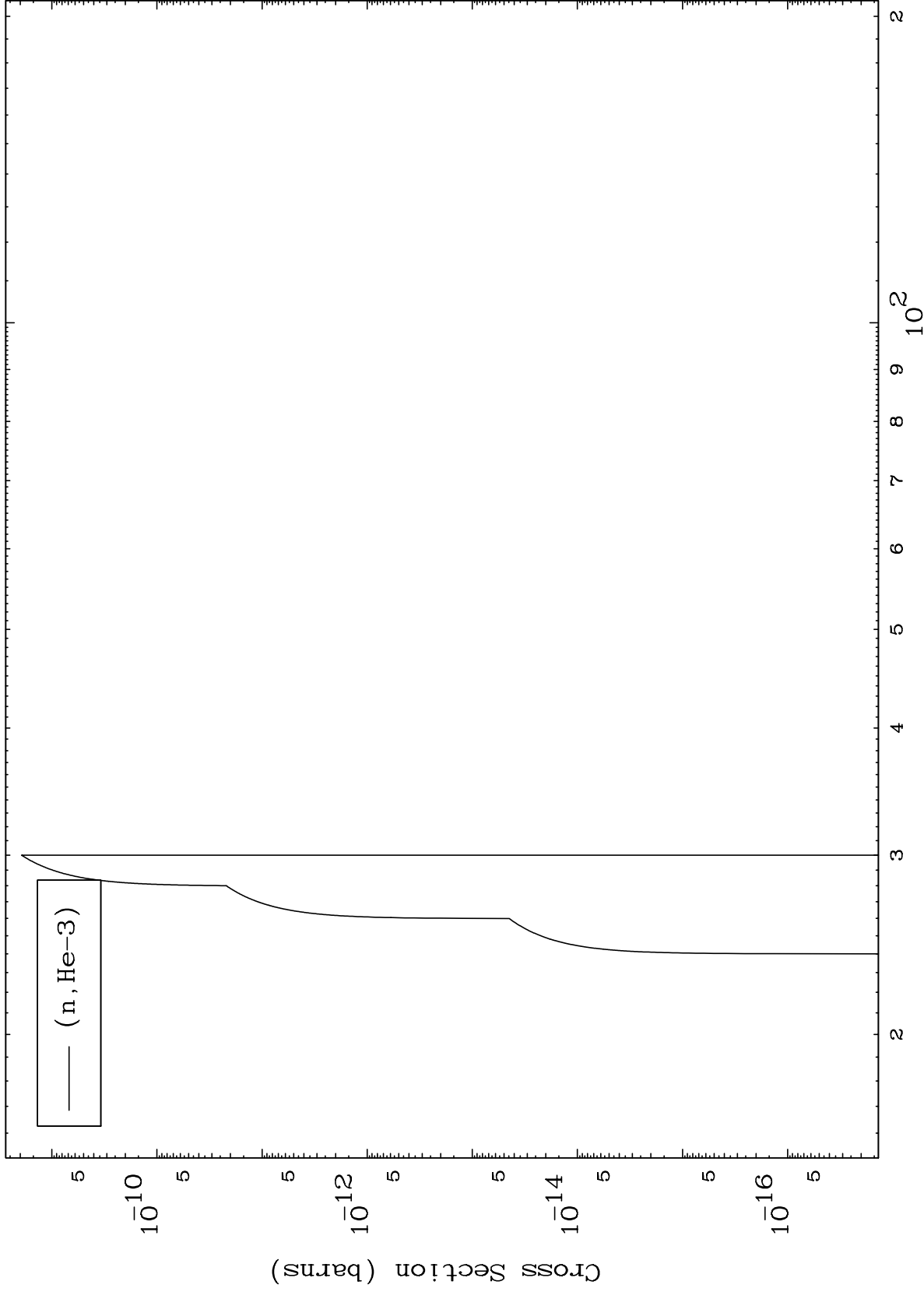
7

Incident Energy (MeV)

83-Bi-212





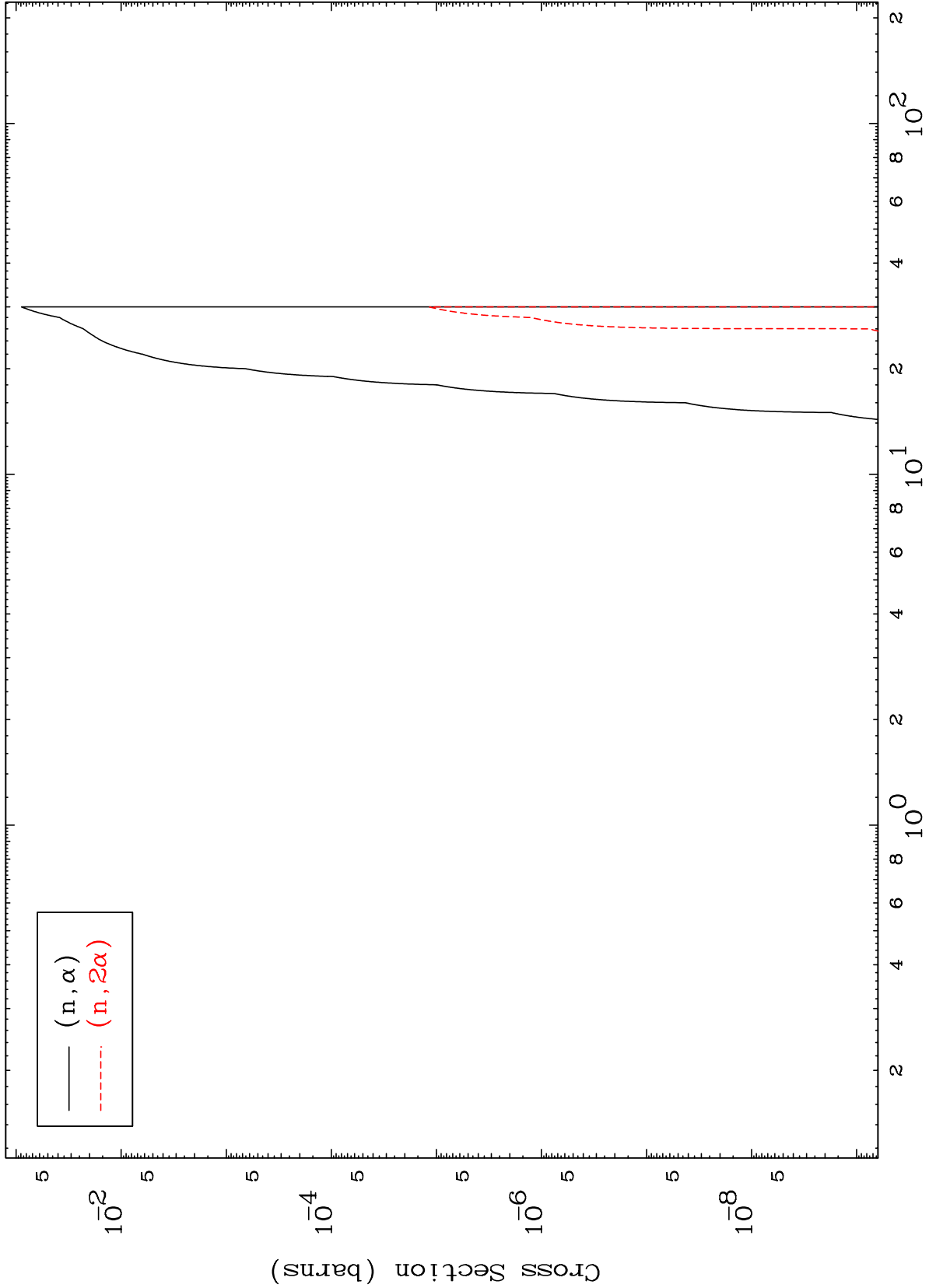


MAT 8334

( $\alpha, \alpha$ ) Levels

83-Bi-212

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

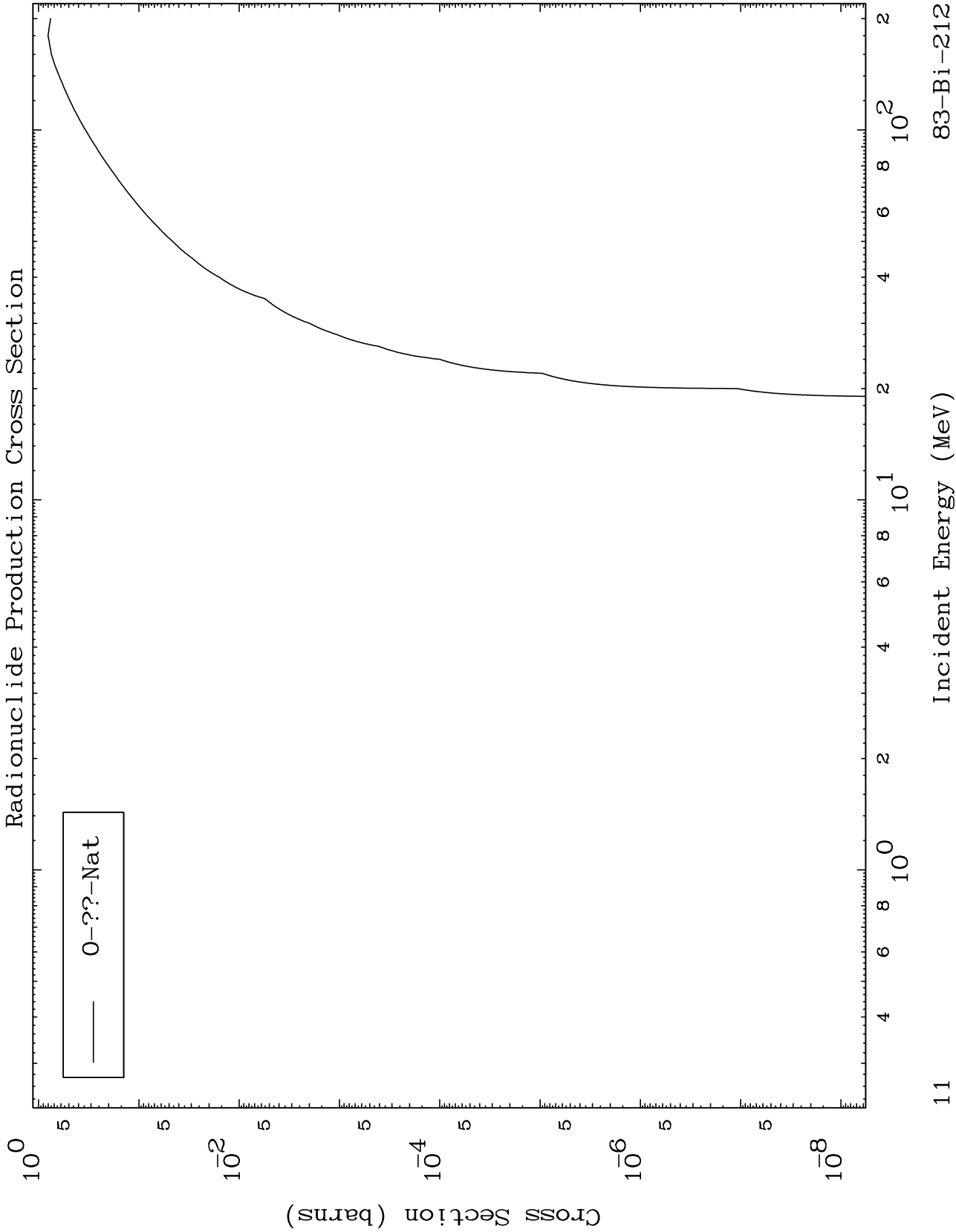
Incident Energy (MeV)

83-Bi-212

MAT 8334

Fission

83-Bi-212

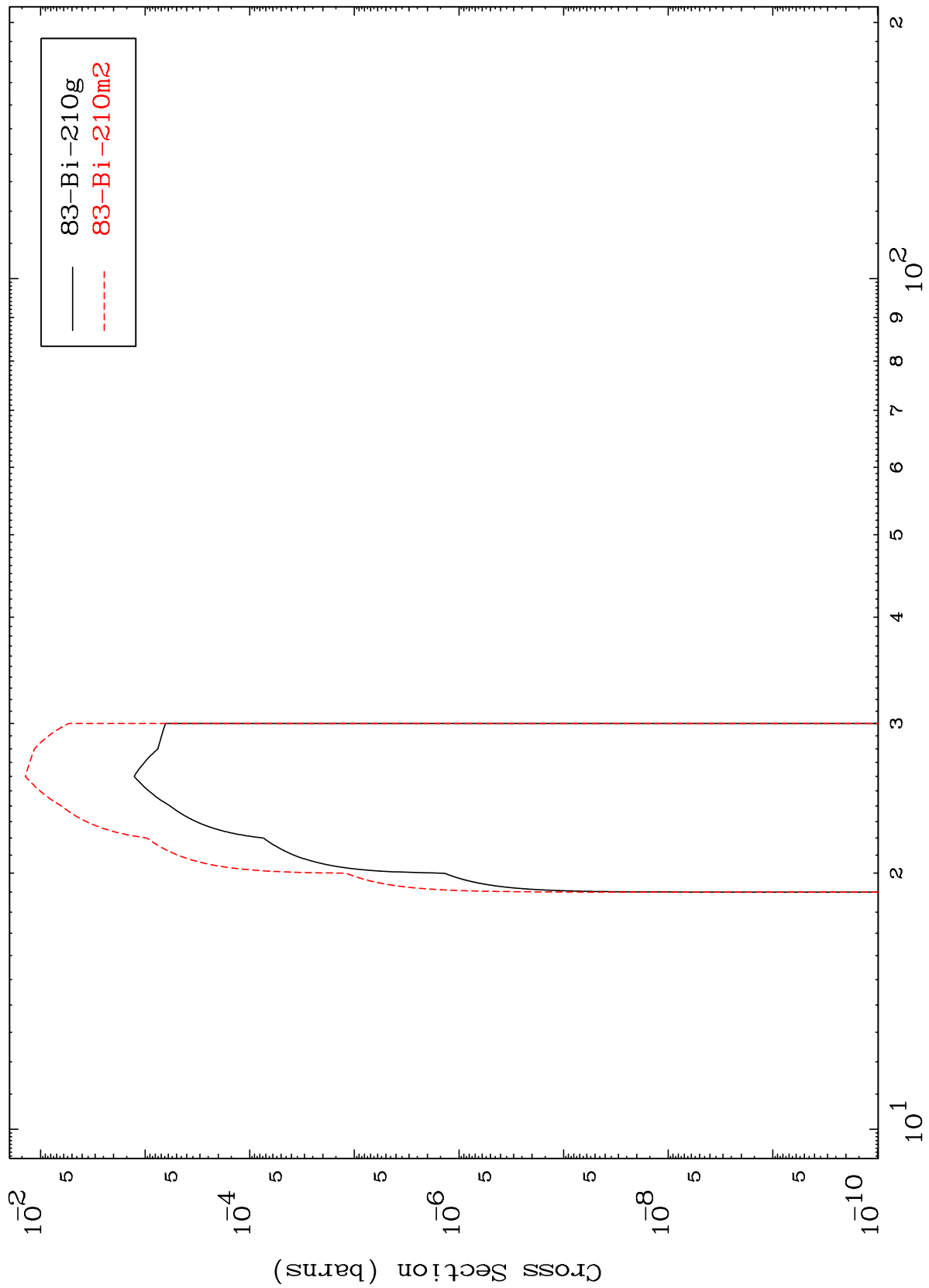


MAT 8334

83-Bi-212

(n,2n)  $\alpha$

Radionuclide Production Cross Section



83-Bi-212

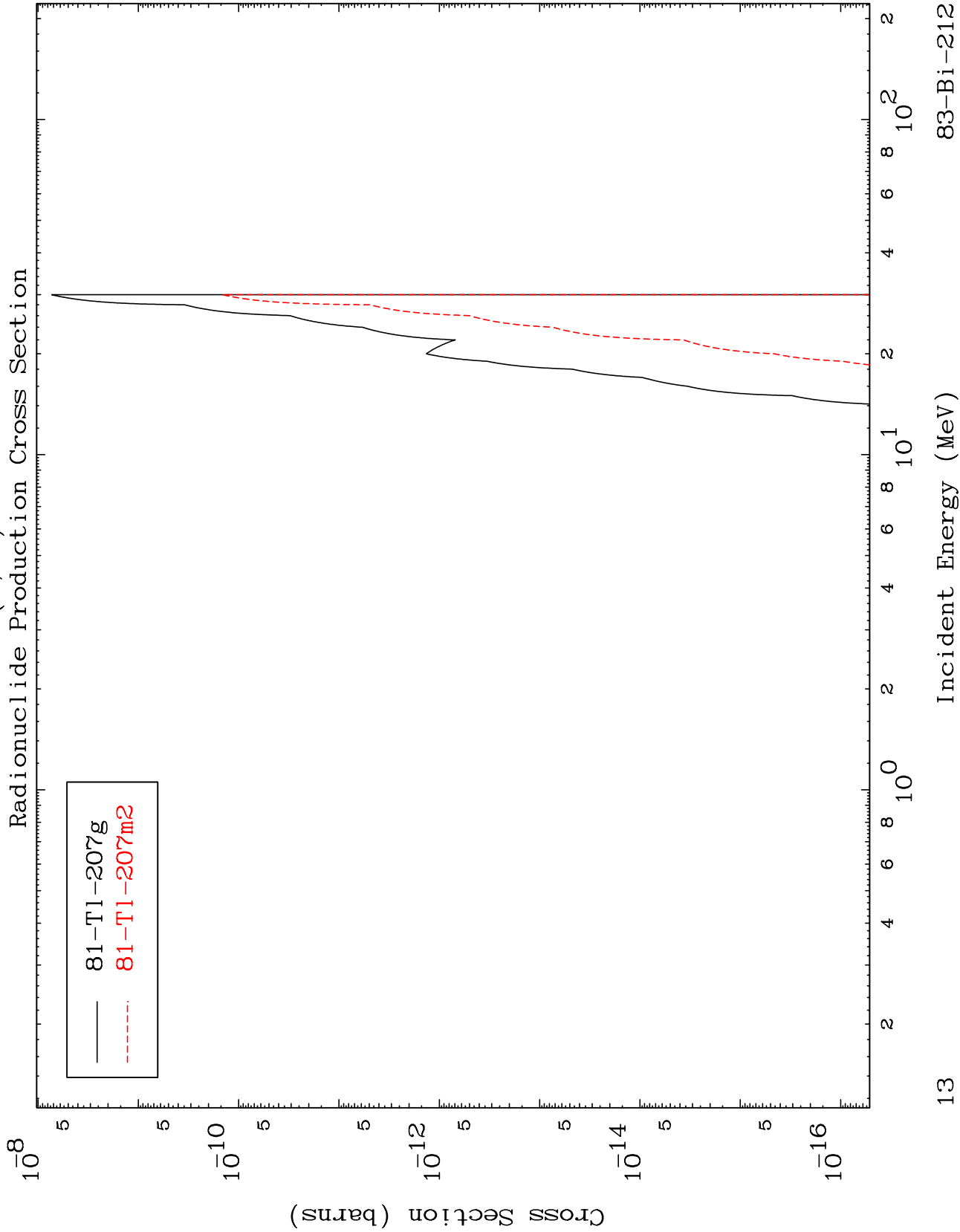
Incident Energy (MeV)

12

MAT 8334

(n,n') 2 $\alpha$

83-Bi-212

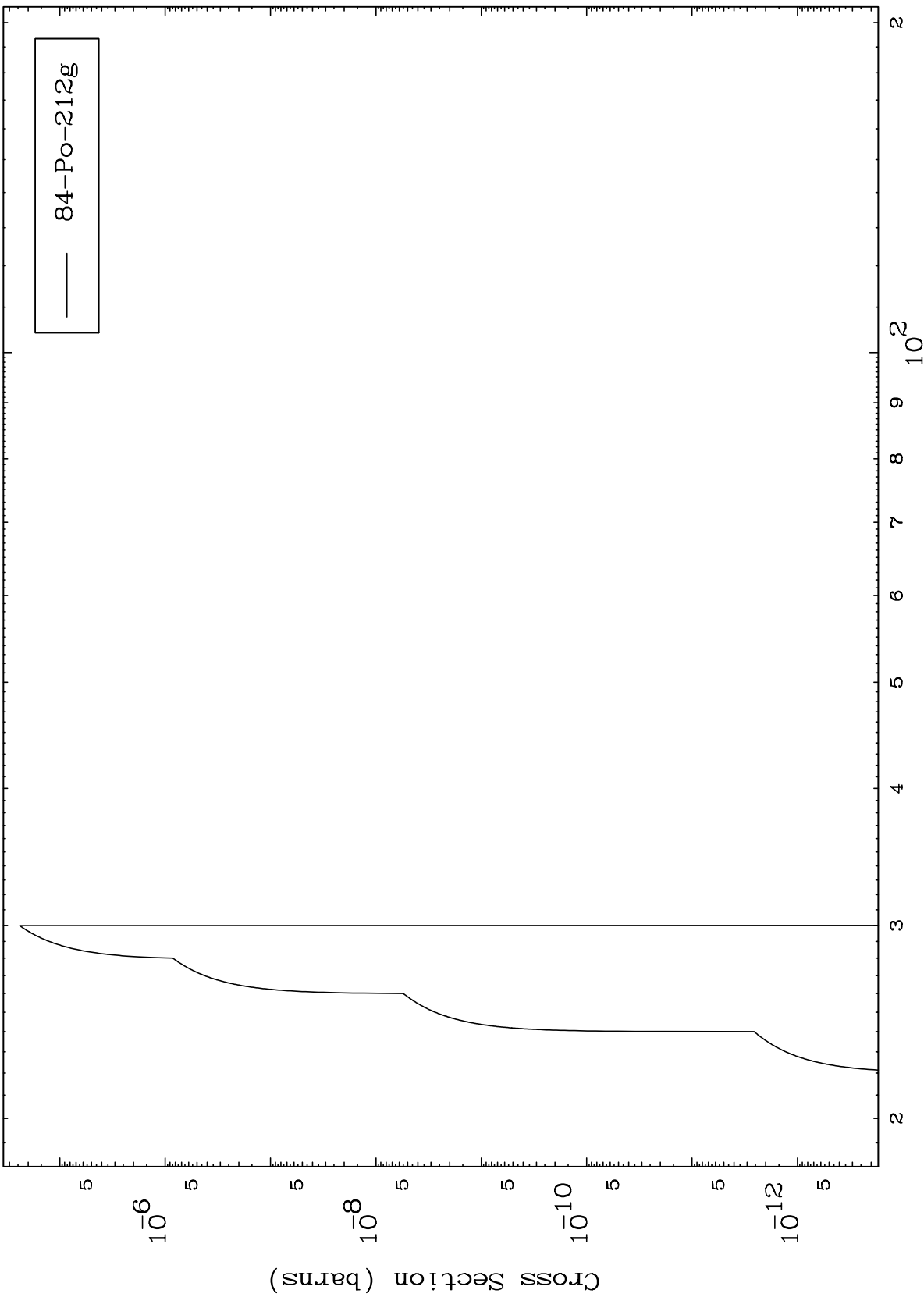


MAT 8334

(n,n') t

83-Bi-212

Radionuclide Production Cross Section



14

Incident Energy (MeV)

83-Bi-212

MAT 8334

83-Bi-212

(n,α)  
Radionuclide Production Cross Section

