

Program EVALPLOT  
(Version 2021-1)

by

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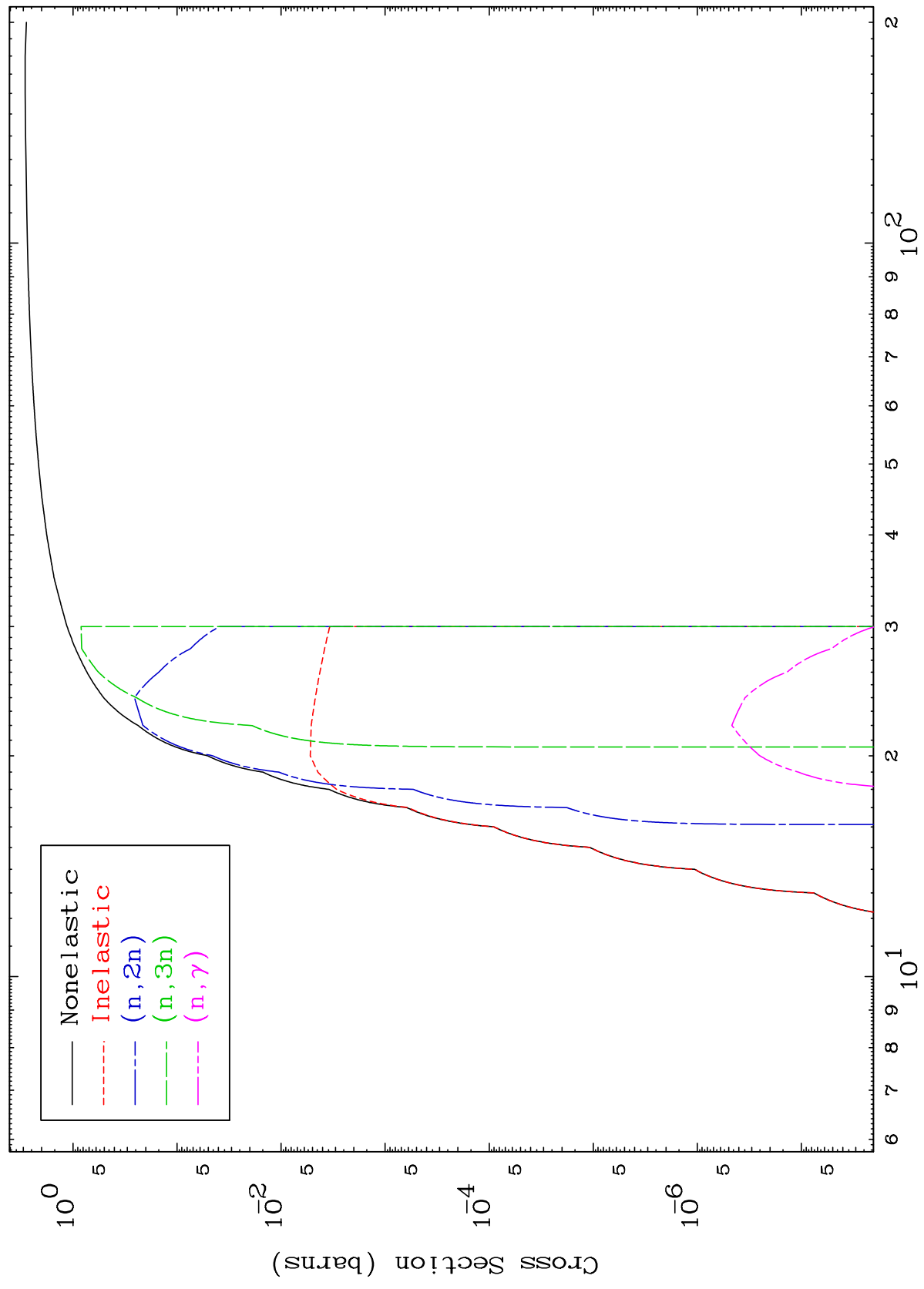
Press Mouse Button to Start

MAT 8347

0 Kelvin

$\alpha$  Major

83-Bi-216m



1

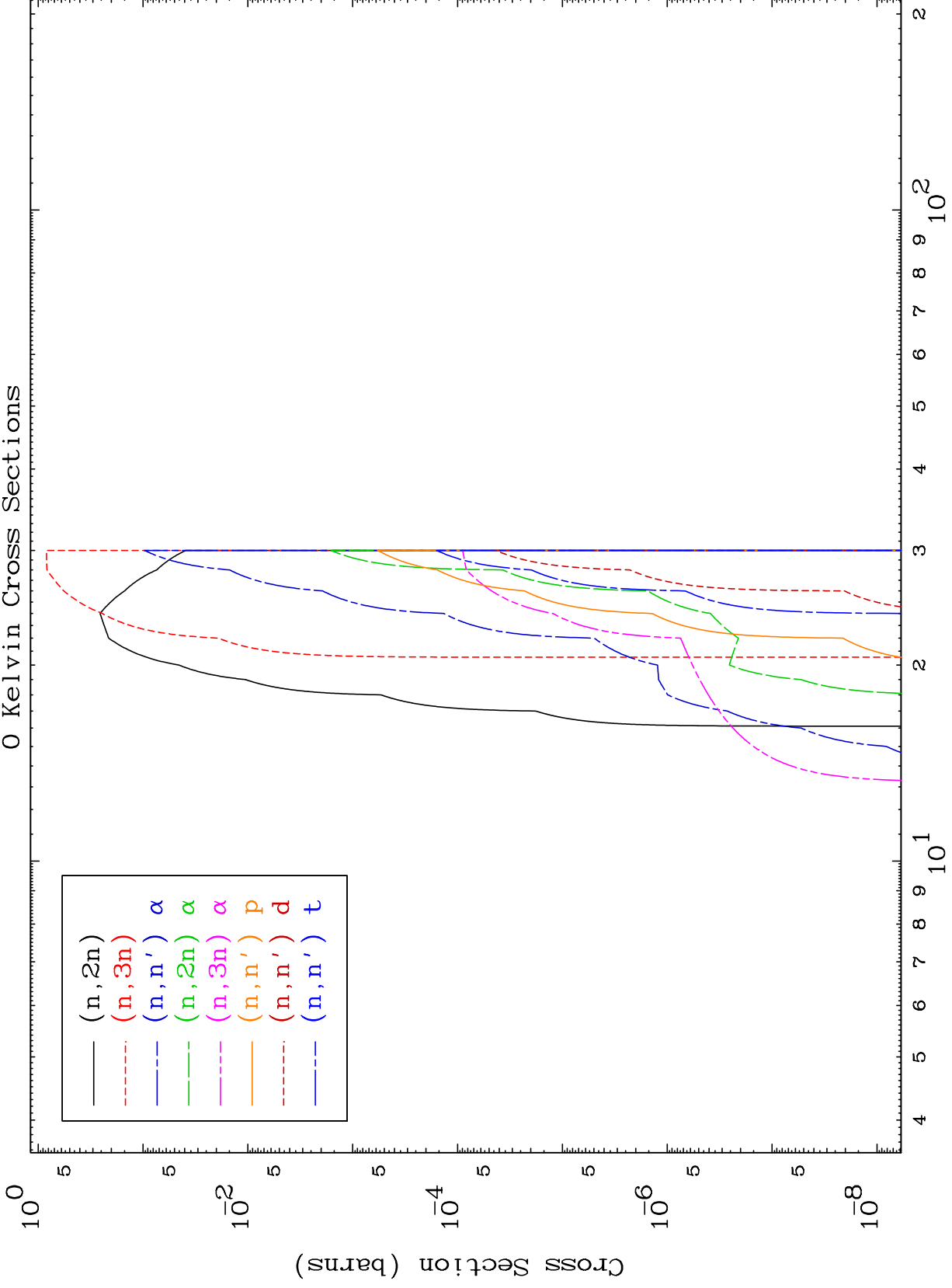
Incident Energy (MeV)

83-Bi-216m

MAT 8347

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

83-Bi-216m



2

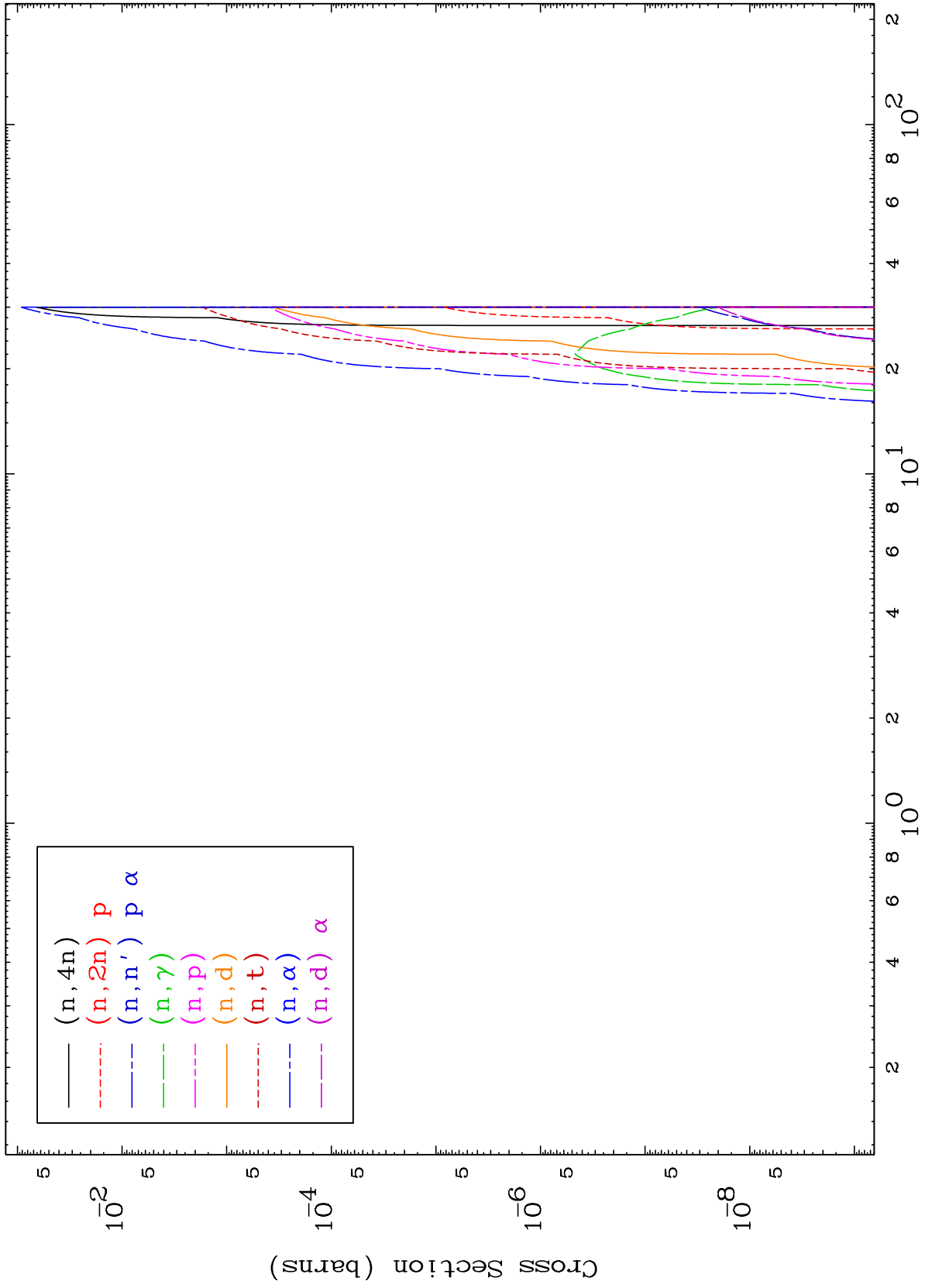
Incident Energy (MeV)

83-Bi-216m

MAT 8347

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

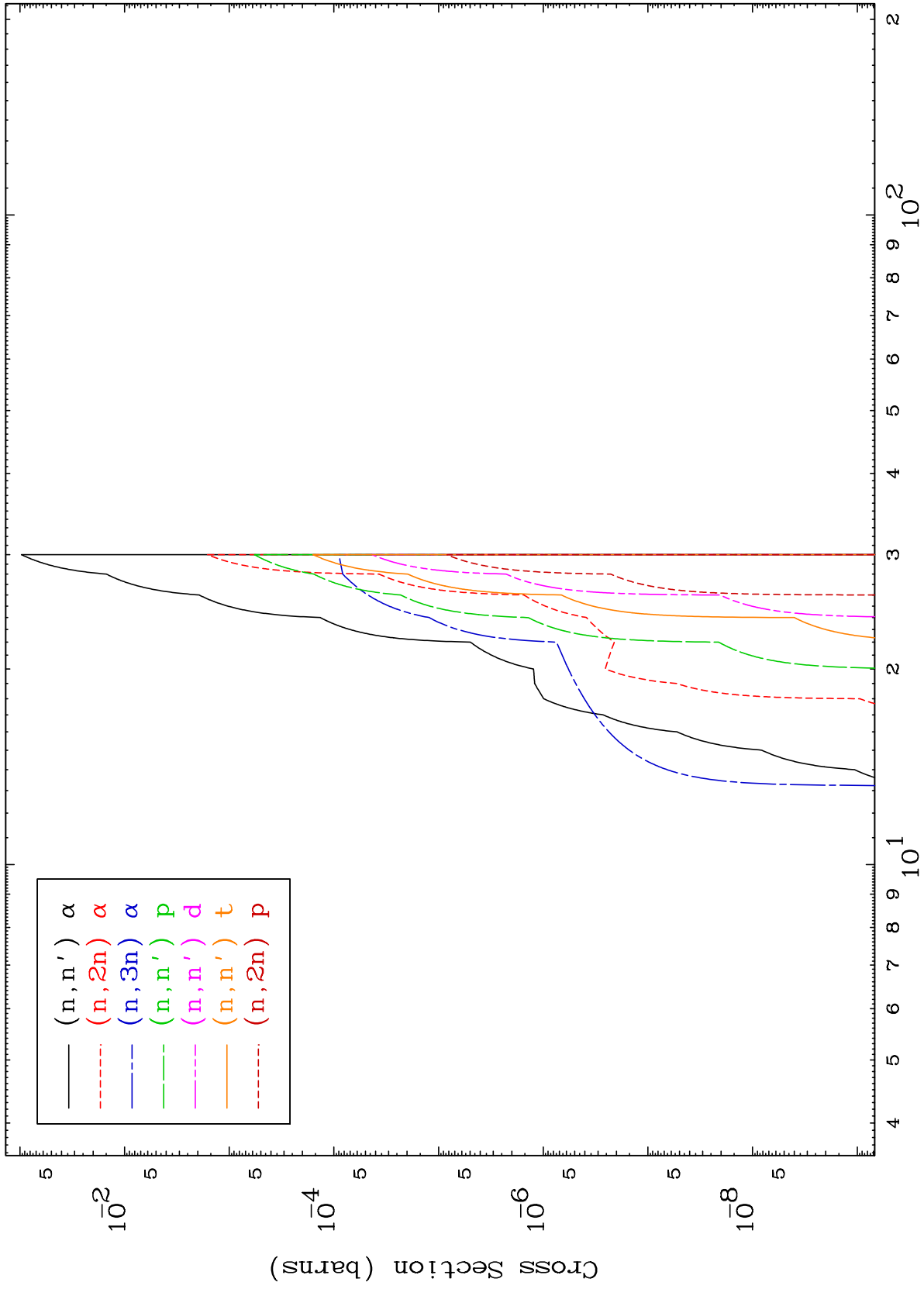
83-Bi-216m



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$\alpha$  Charged Particle  
0 Kelvin Cross Sections

83-Bi-216m



4

Incident Energy (MeV)

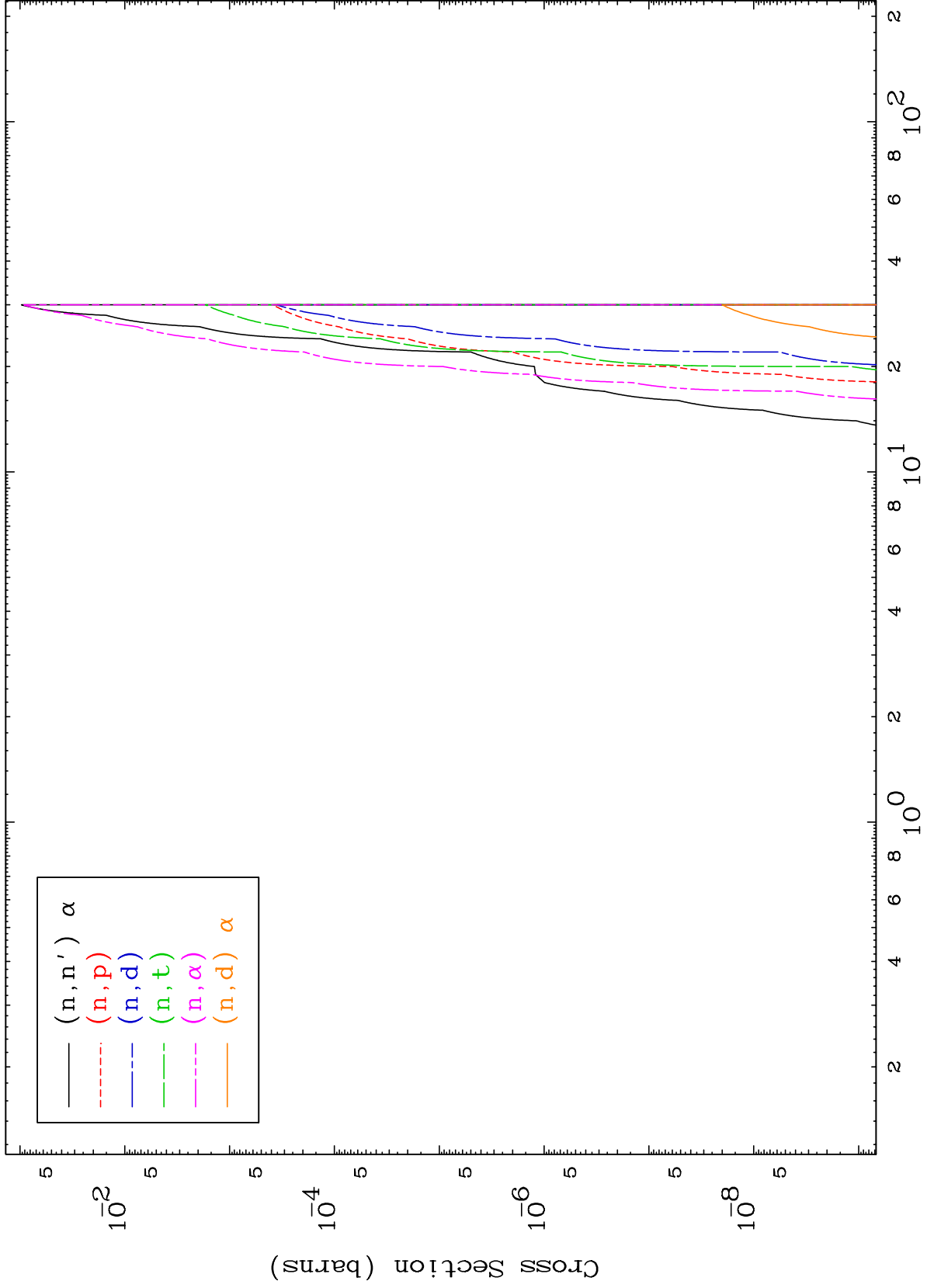
83-Bi-216m

MAT 8347

$\alpha$  Charged Particle

83-Bi-216m

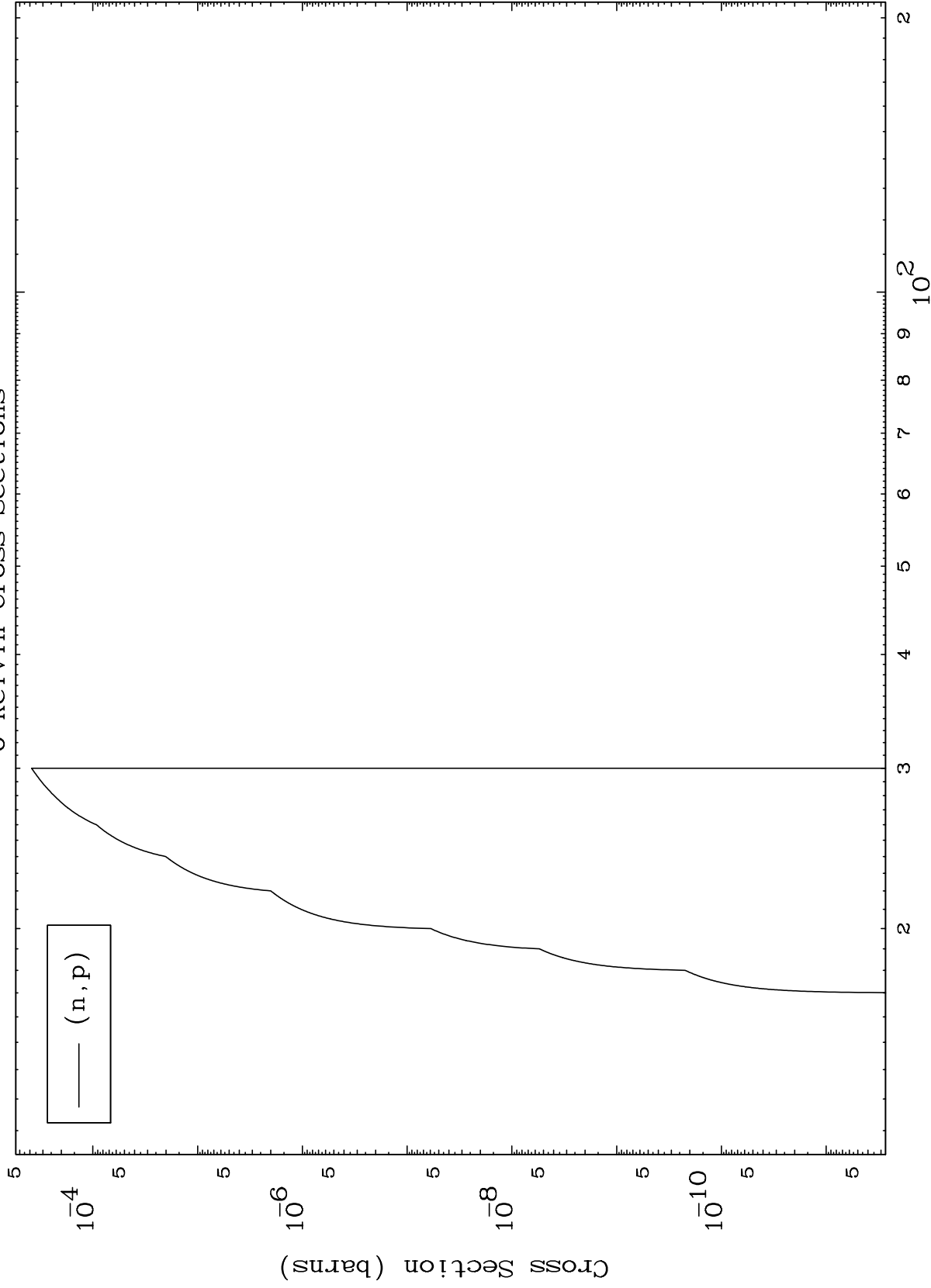
0 Kelvin Cross Sections



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( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

83-Bi-216m



6

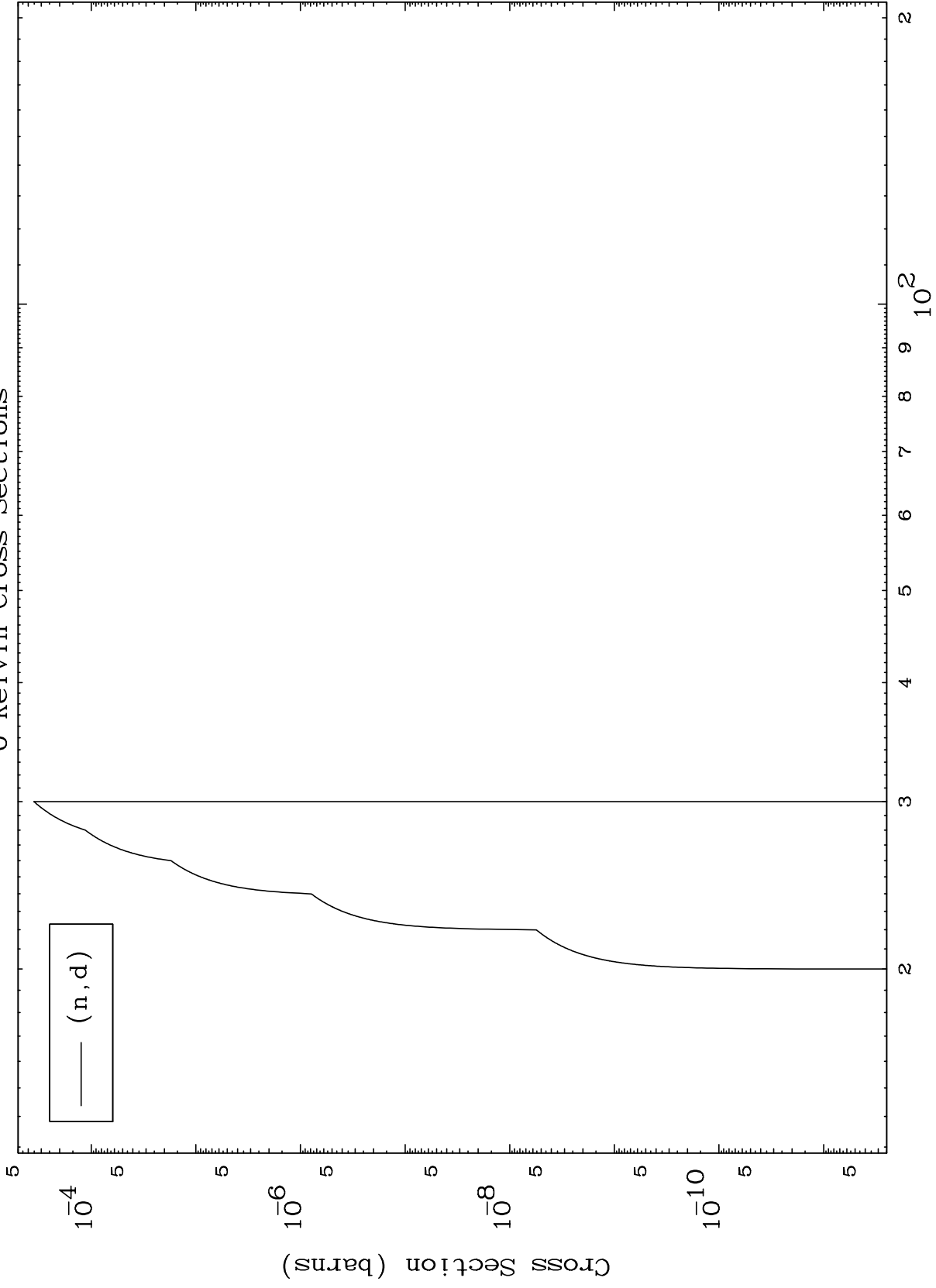
Incident Energy (MeV)

83-Bi-216m

MAT 8347

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

$^{83}\text{Bi}-216\text{m}$



7

Incident Energy (MeV)

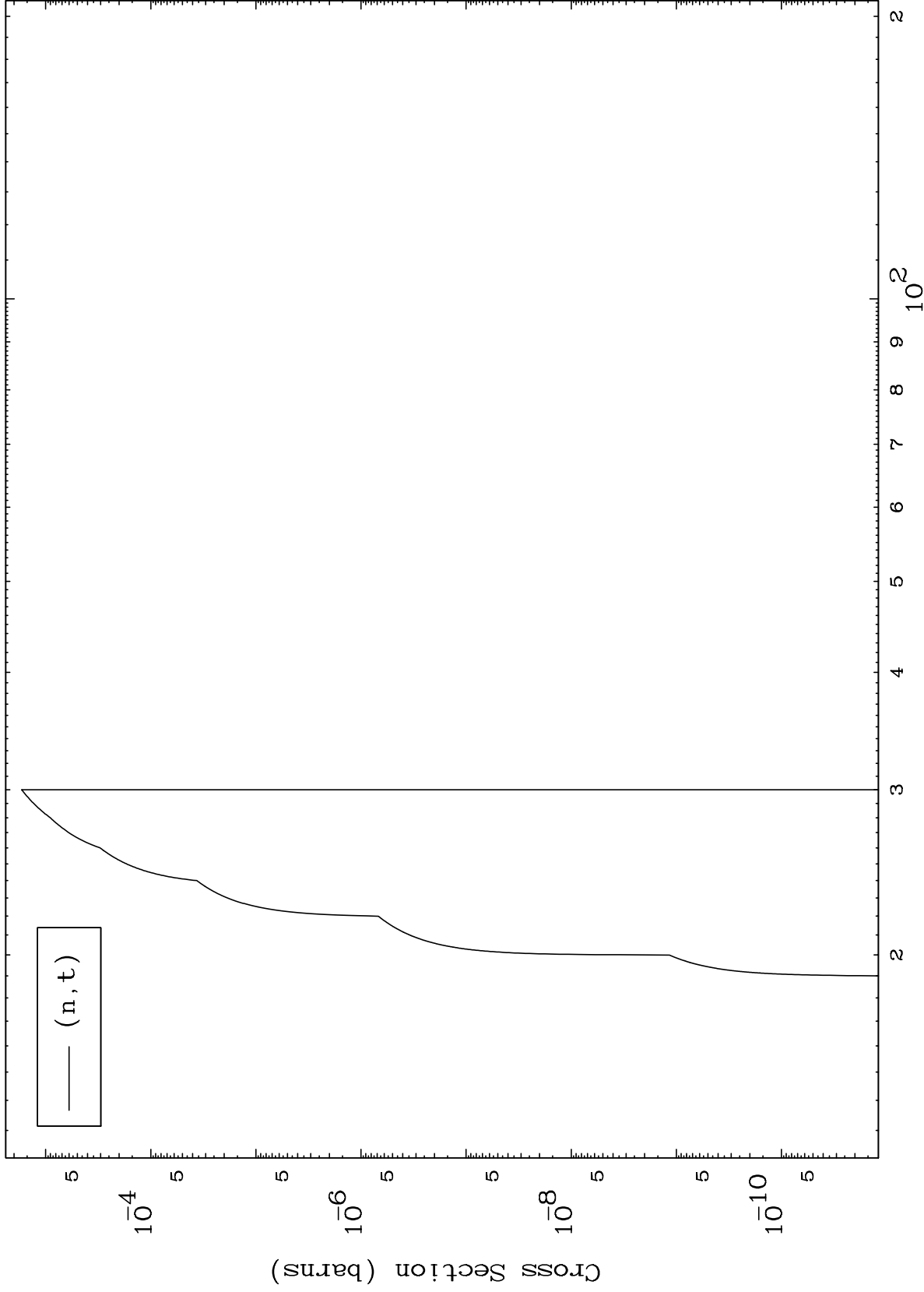
$^{83}\text{Bi}-216\text{m}$



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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

83-Bi-216m

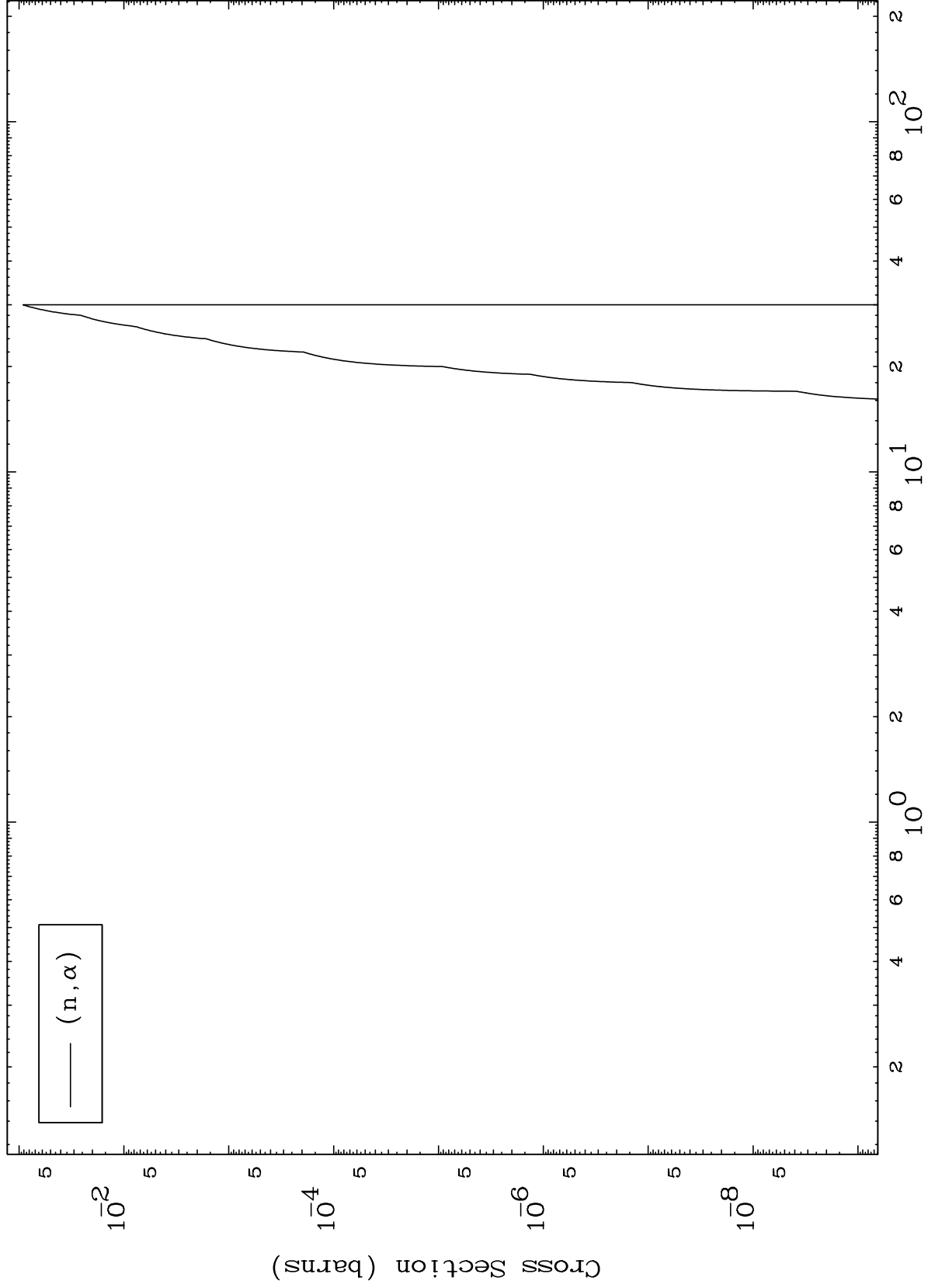


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( $\alpha, \alpha$ ) Levels

83-Bi-216m

0 Kelvin Cross Sections

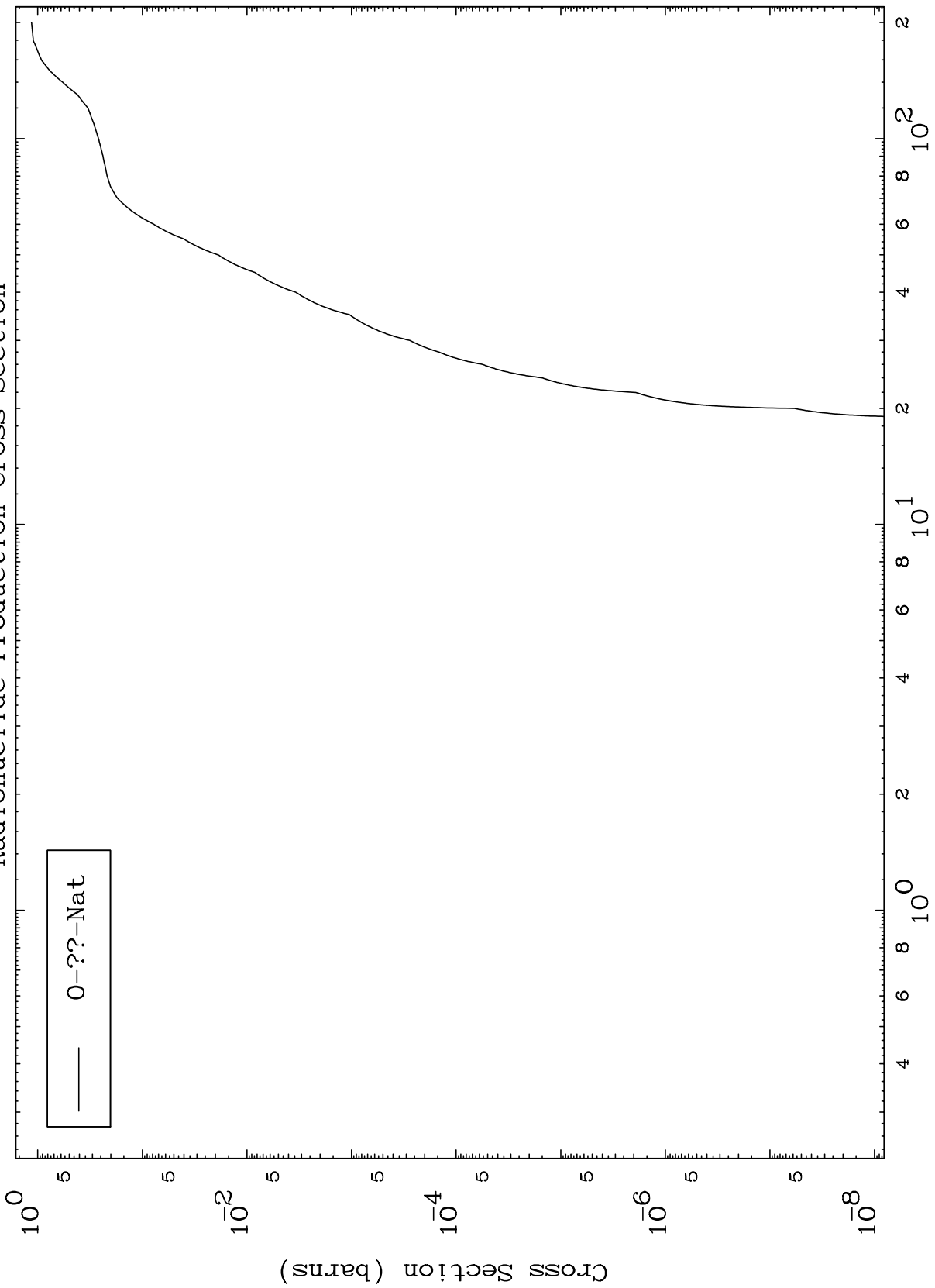


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Fission

<sup>83</sup>Bi-216m

Radionuclide Production Cross Section



10

Incident Energy (MeV)

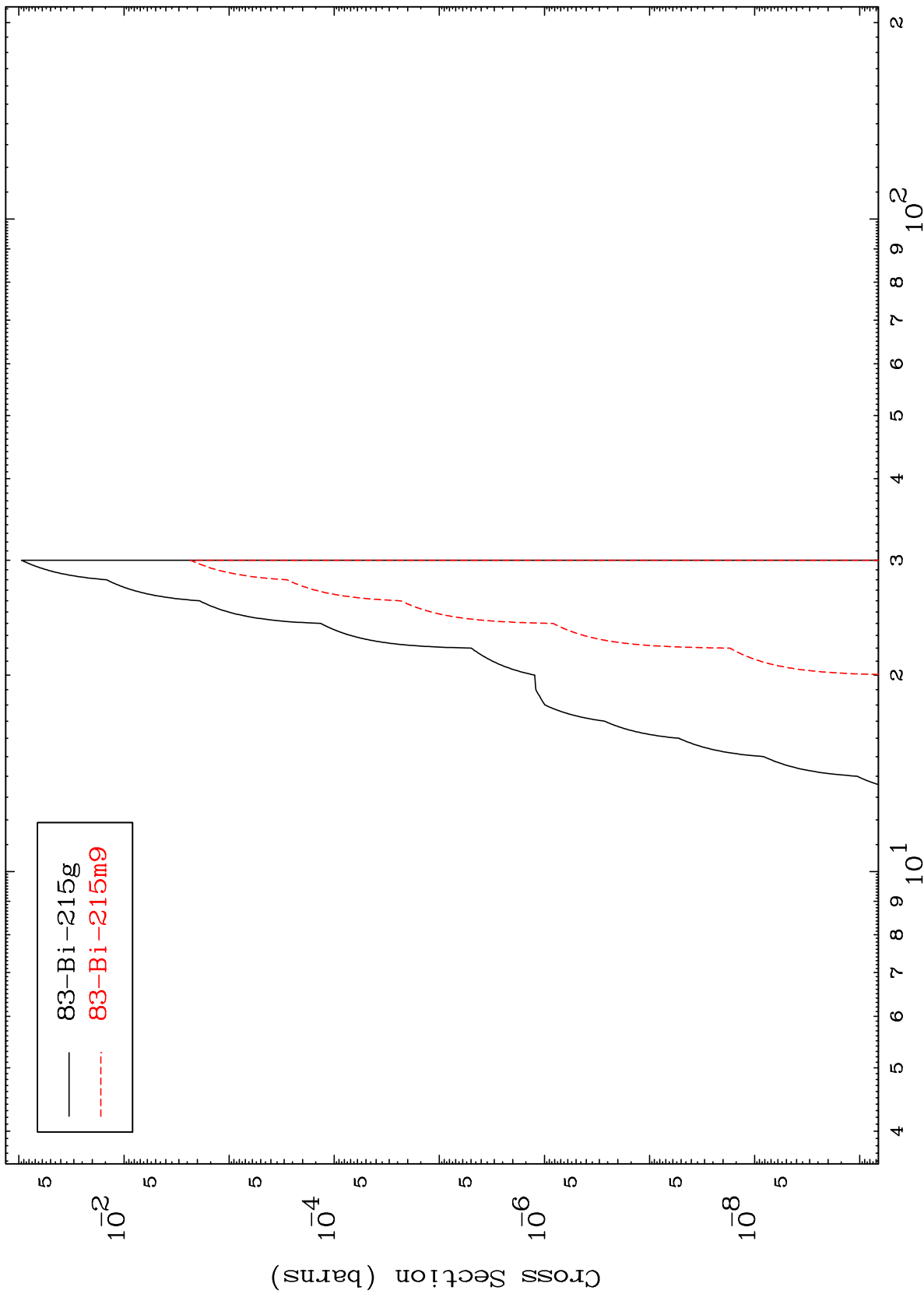
<sup>83</sup>Bi-216m

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$(n, n') \alpha$

83-Bi-216m

Radionuclide Production Cross Section



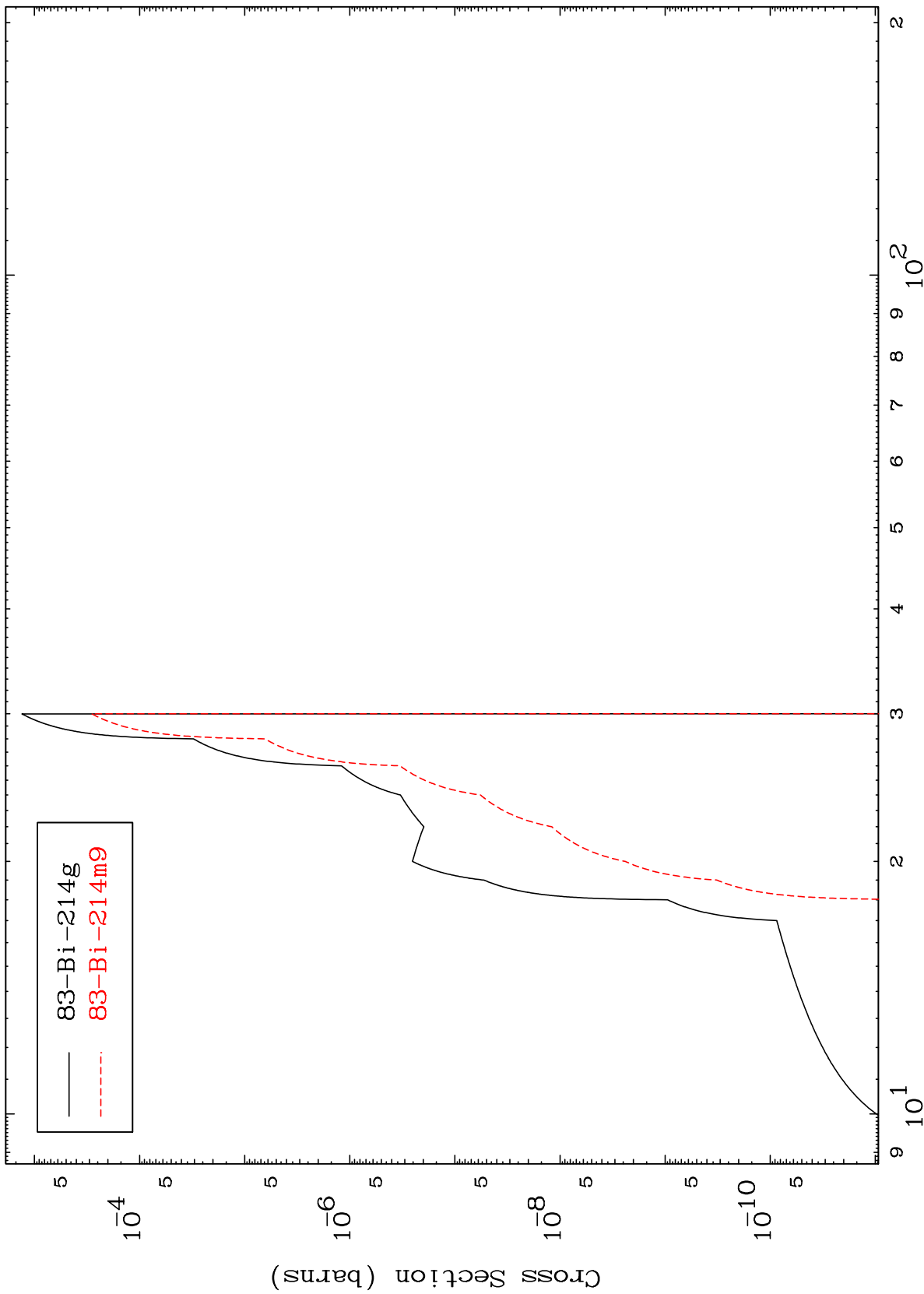
83-Bi-215g  
83-Bi-215m9

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(n,2n)  $\alpha$

$^{83}\text{Bi}-216\text{m}$

Radionuclide Production Cross Section



83-Bi-214g  
83-Bi-214m9

12

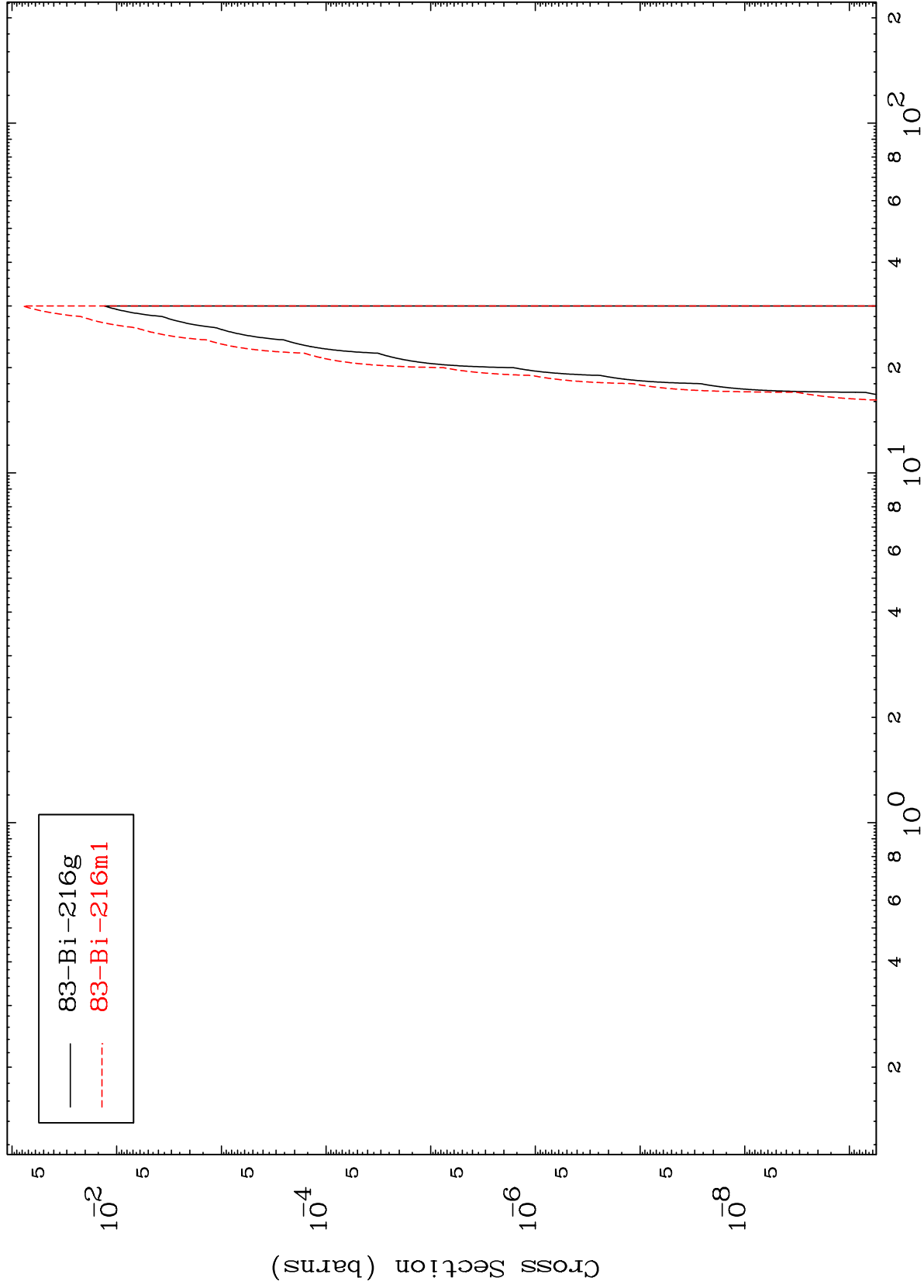
Incident Energy (MeV)

$^{83}\text{Bi}-216\text{m}$

MAT 8347

<sup>83</sup>Bi-216m

Radionuclide Production Cross Section



<sup>83</sup>Bi-216m

Incident Energy (MeV)

13