

Program EVALPLOT  
(Version 2021-1)

by

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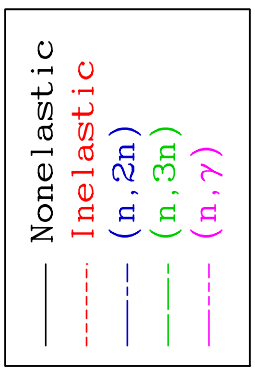
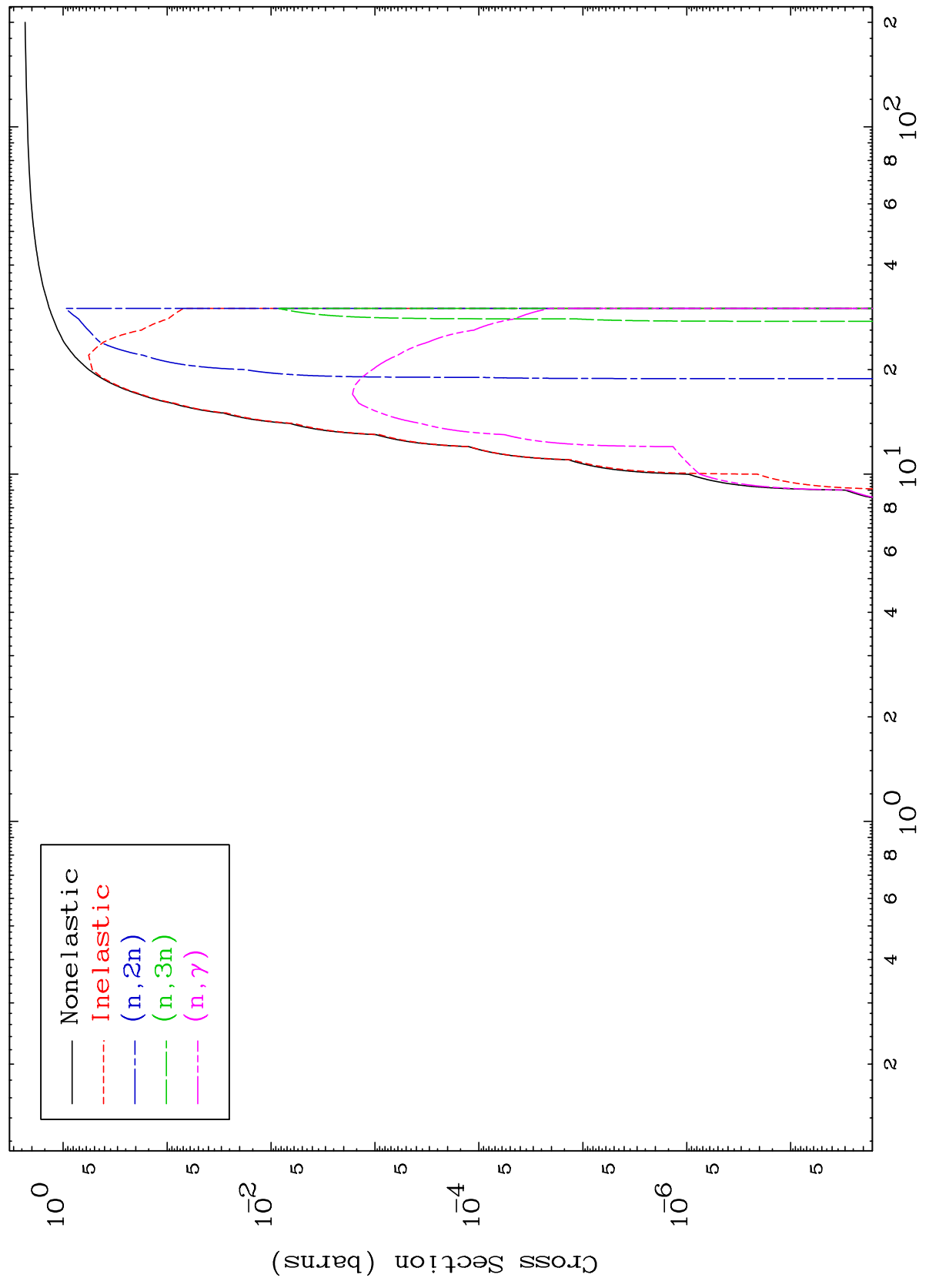
Press Mouse Button to Start

MAT 5823

0 Kelvin

$\alpha$  Major Cross Sections

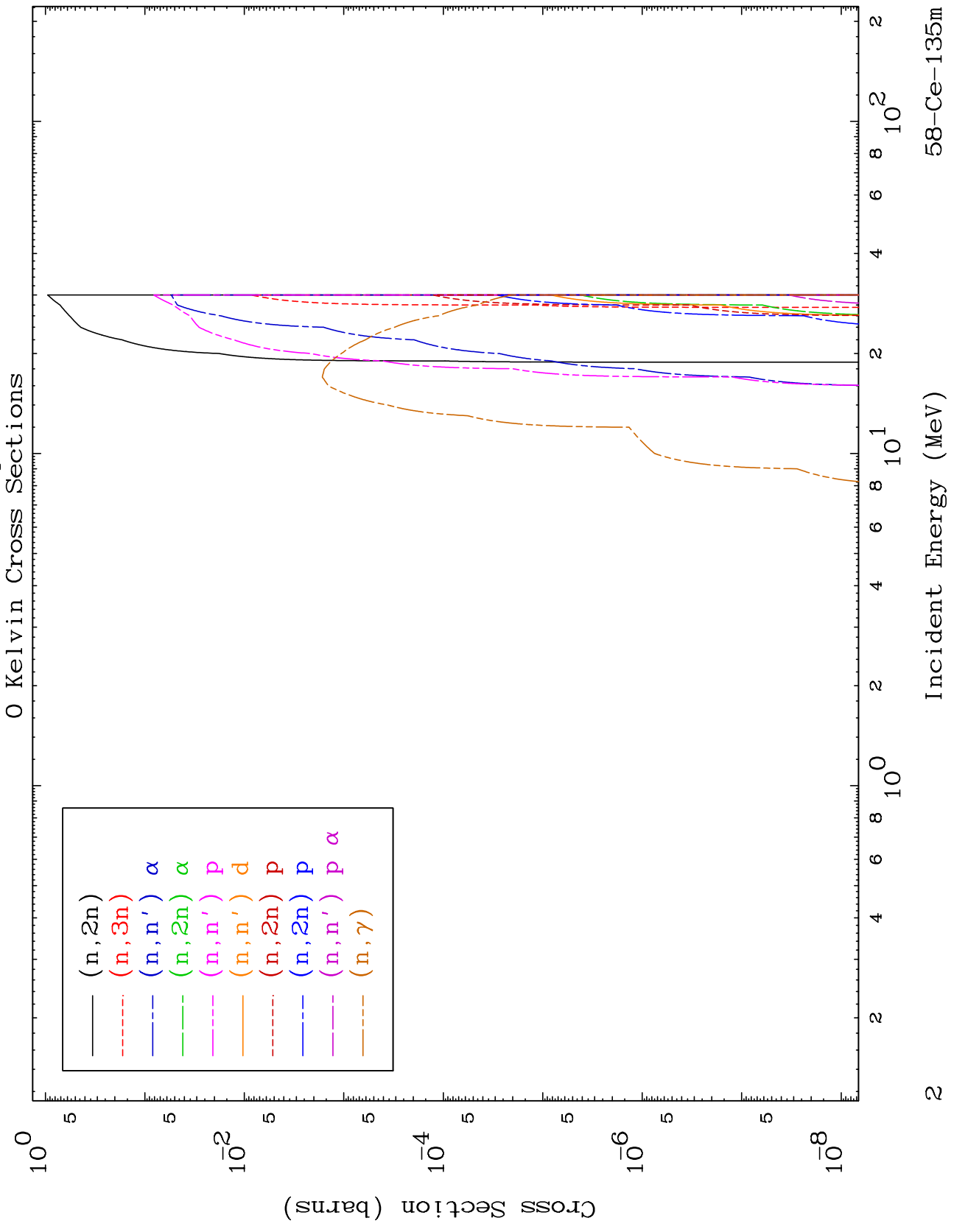
58-Ce-135m



MAT 5823

$\alpha$  Neutron Absorption

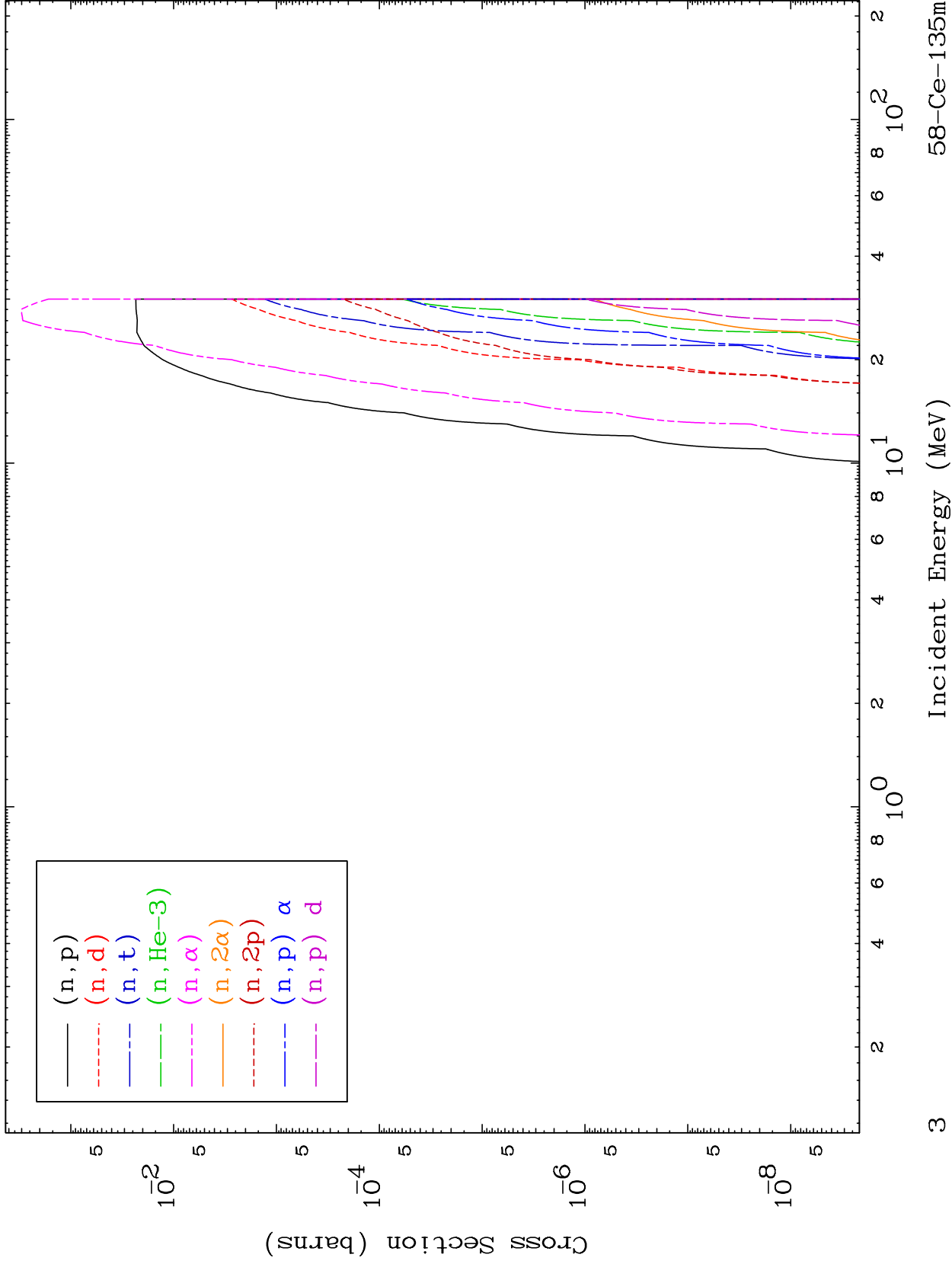
58-Ce-135m

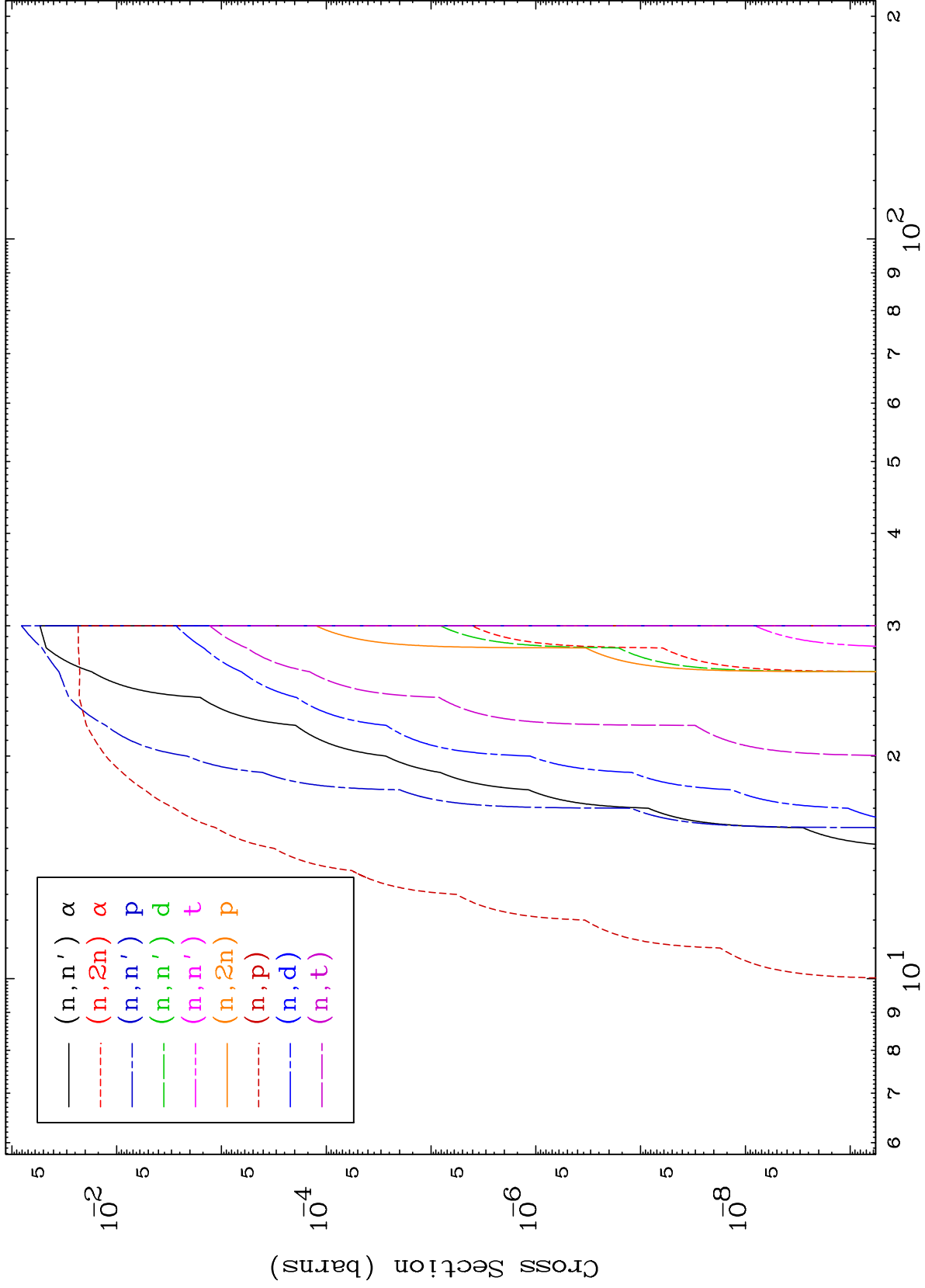


MAT 5823

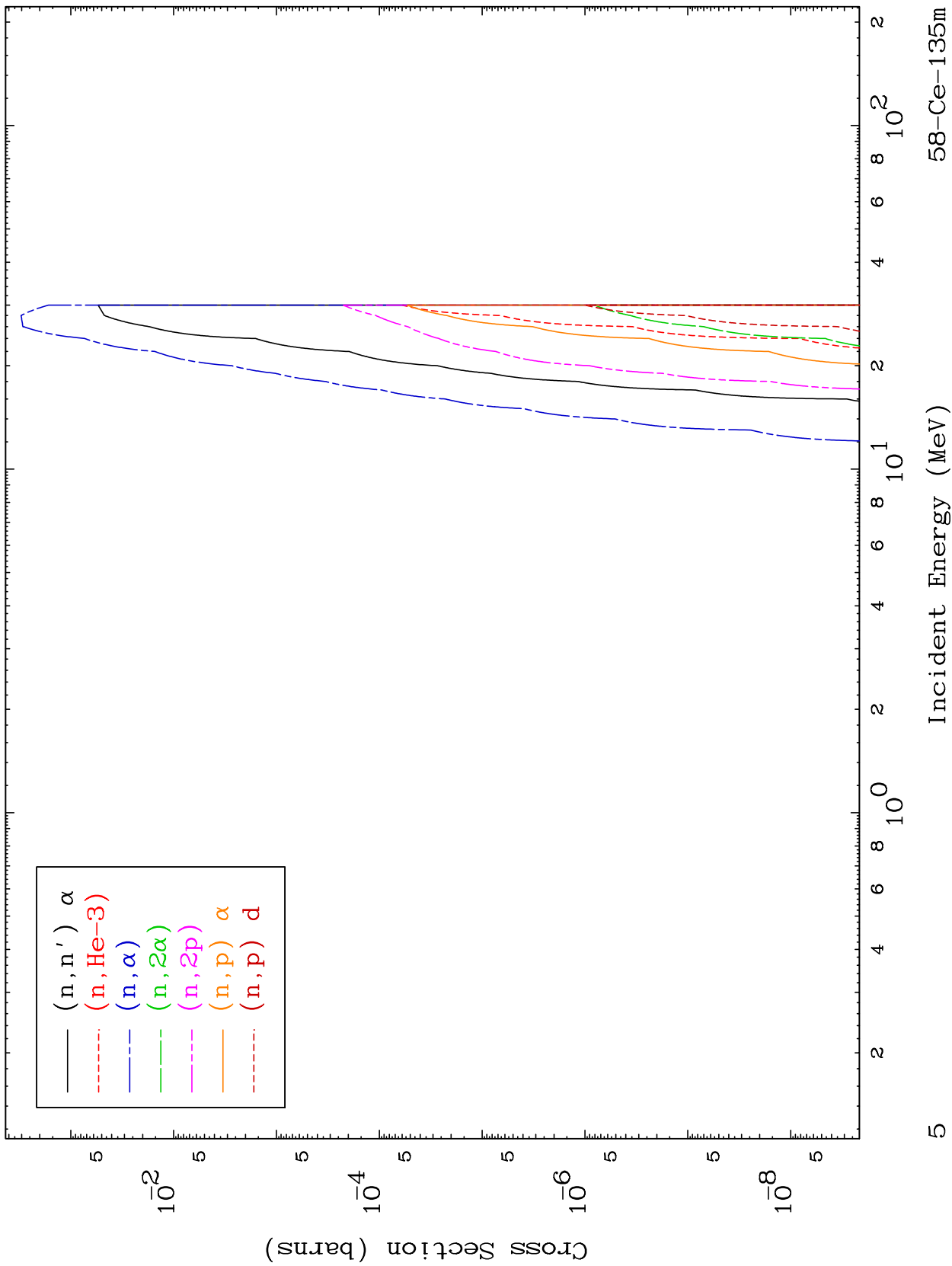
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

58-Ce-135m





0 Kelvin Cross Sections

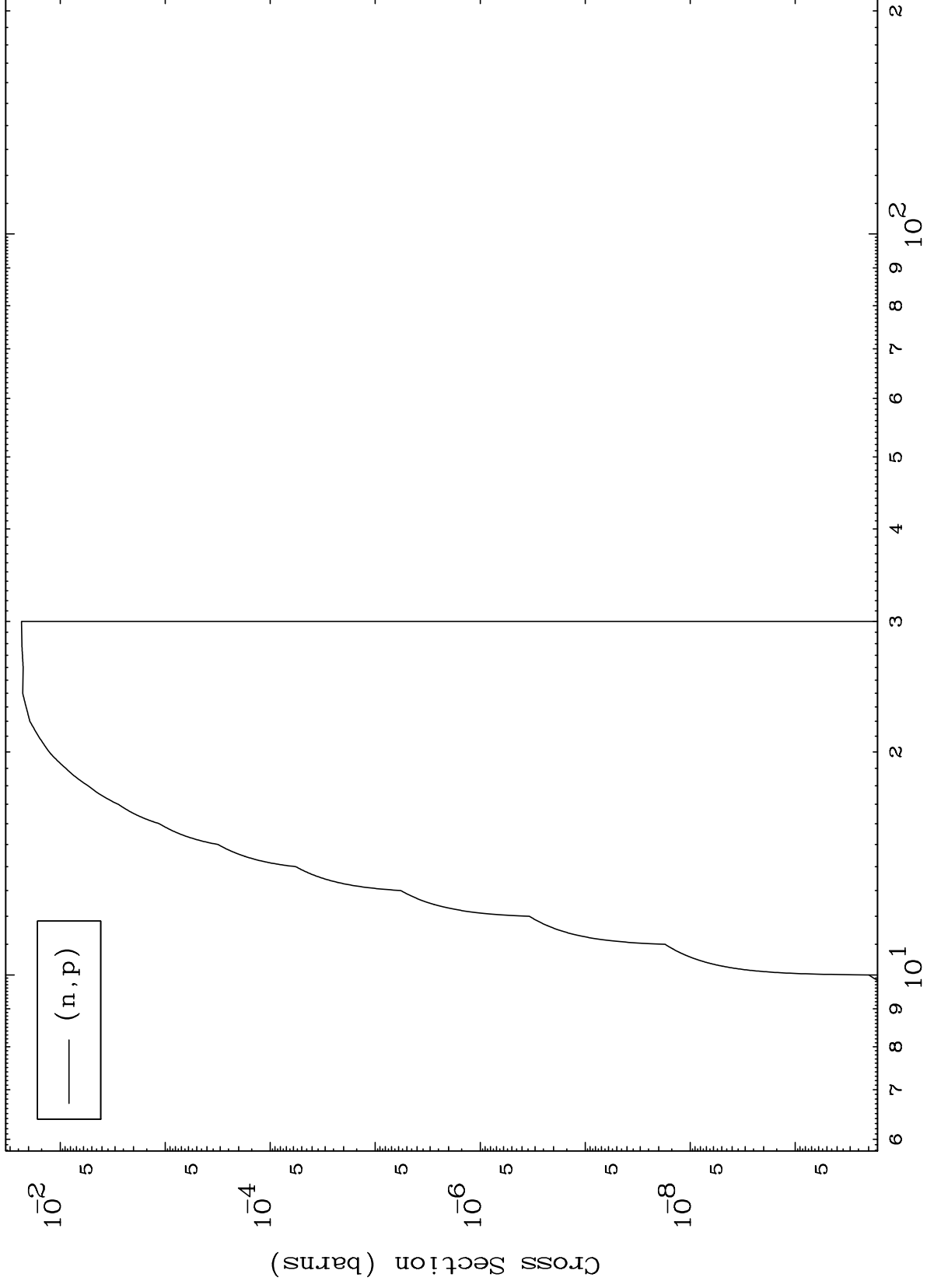


MAT 5823

( $\alpha, p$ ) Levels

58-Ce-135m

0 Kelvin Cross Sections



6

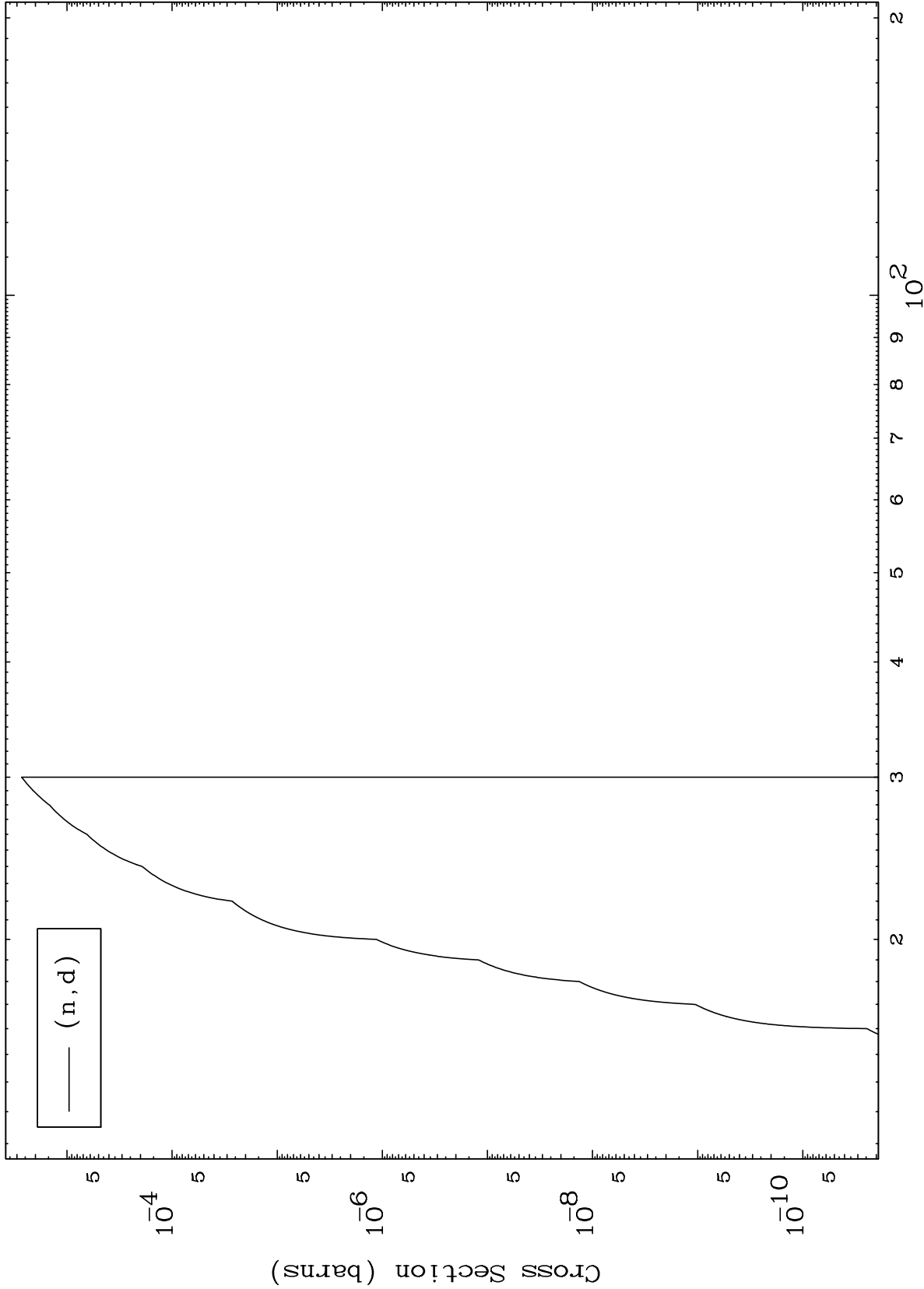
Incident Energy (MeV)

58-Ce-135m

MAT 5823

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

58-Ce-135m

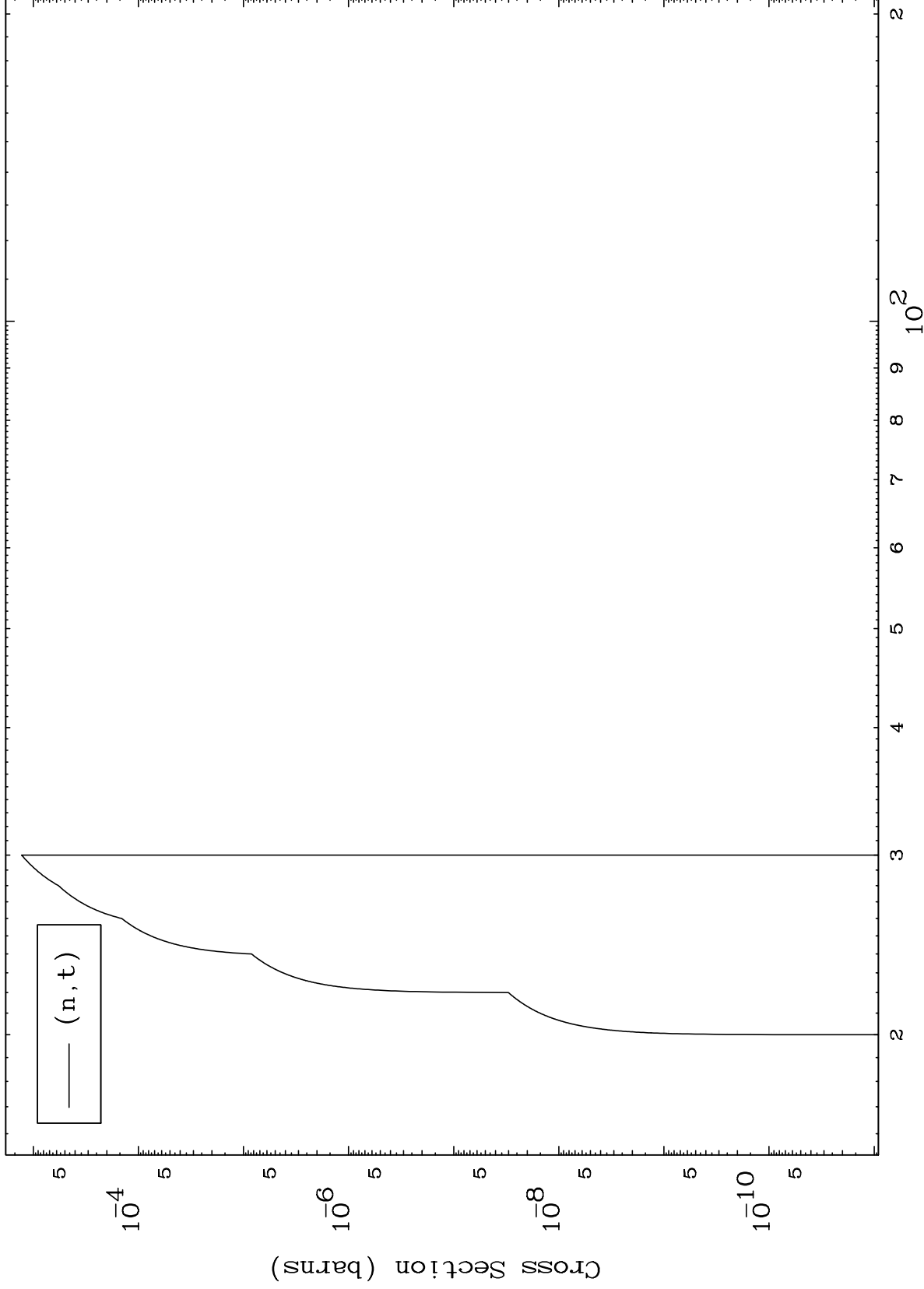




MAT 5823

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

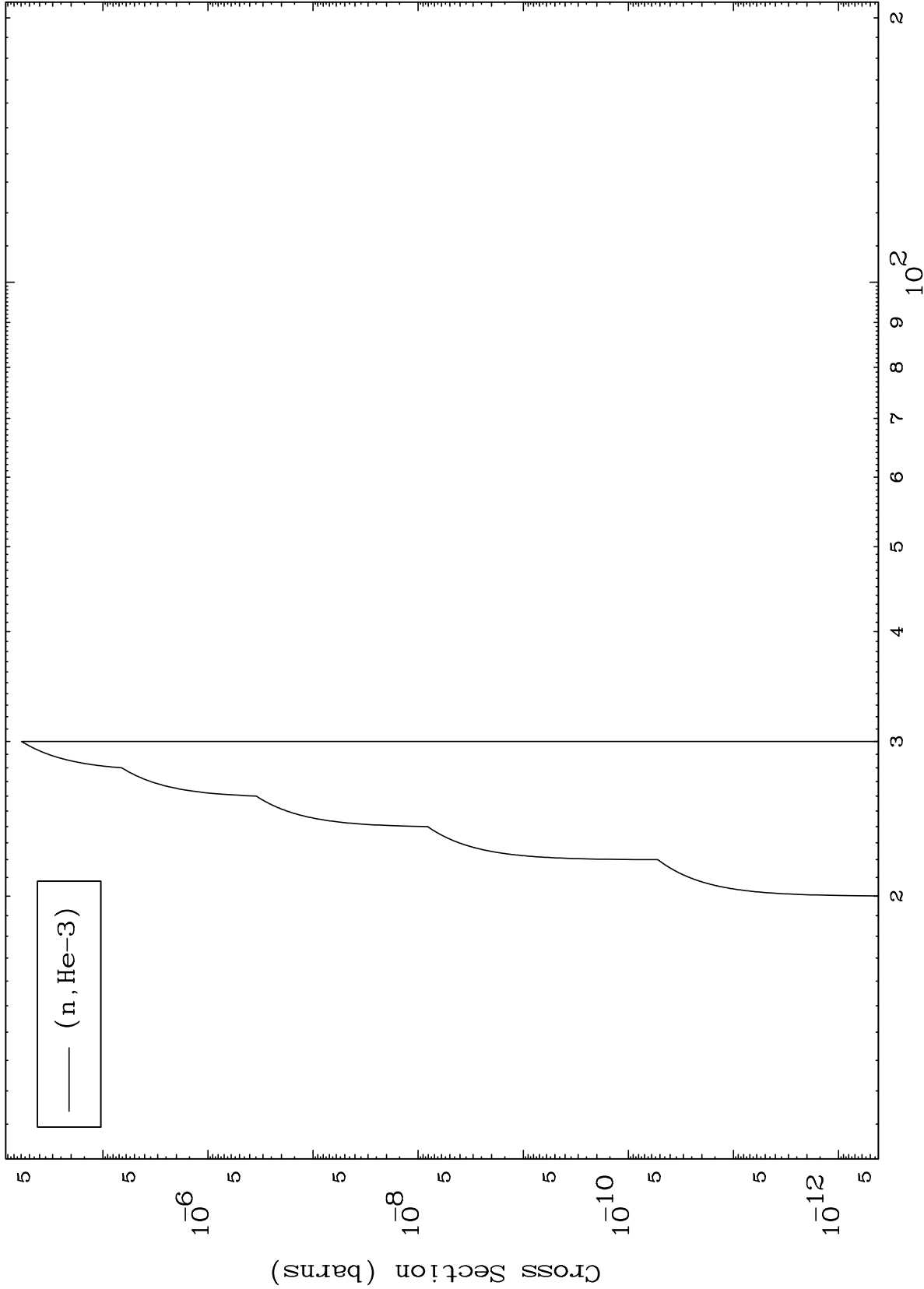
58-Ce-135m



8

Incident Energy (MeV)

58-Ce-135m

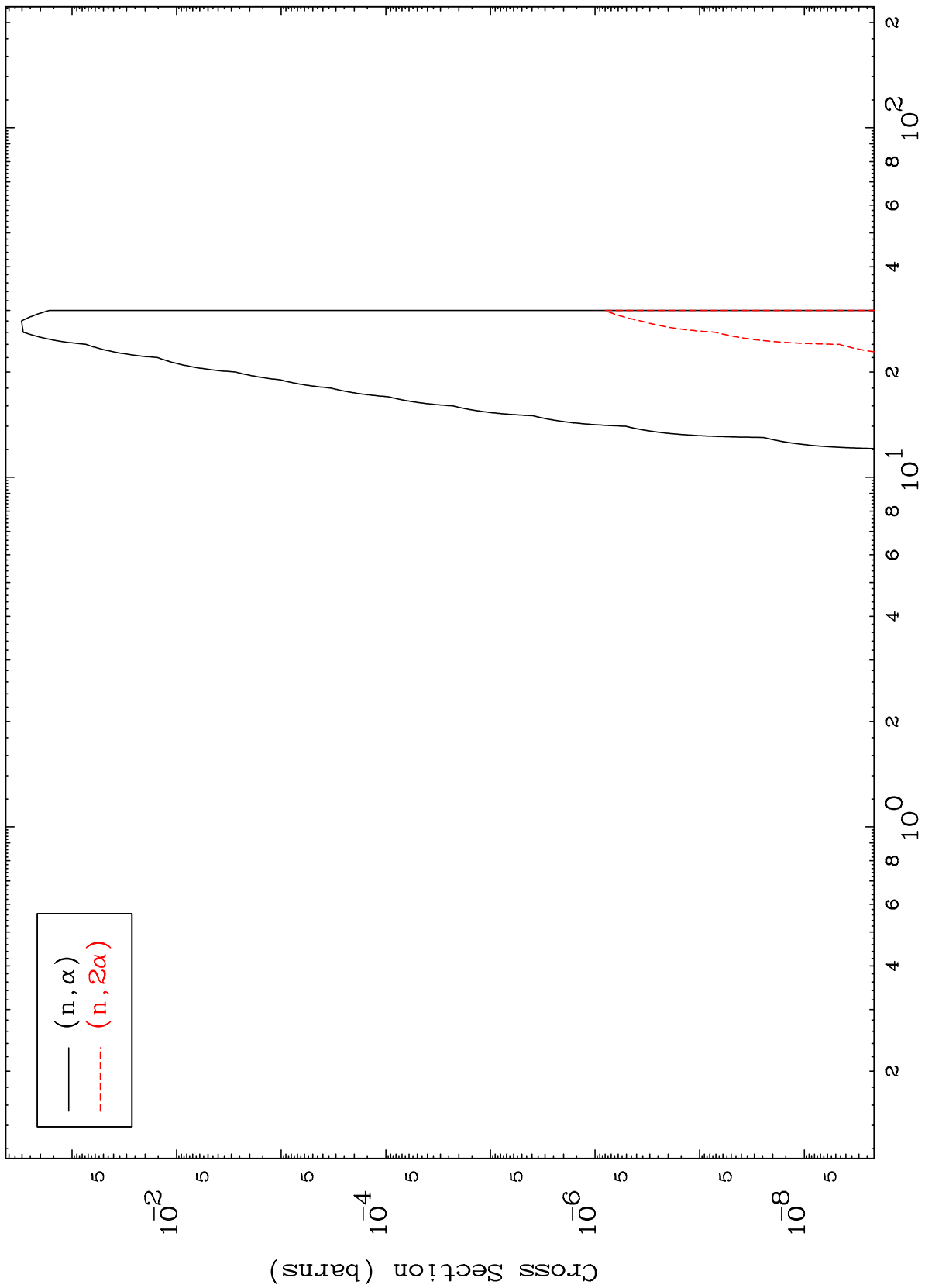


MAT 5823

( $\alpha, \alpha$ ) Levels

58-Ce-135m

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

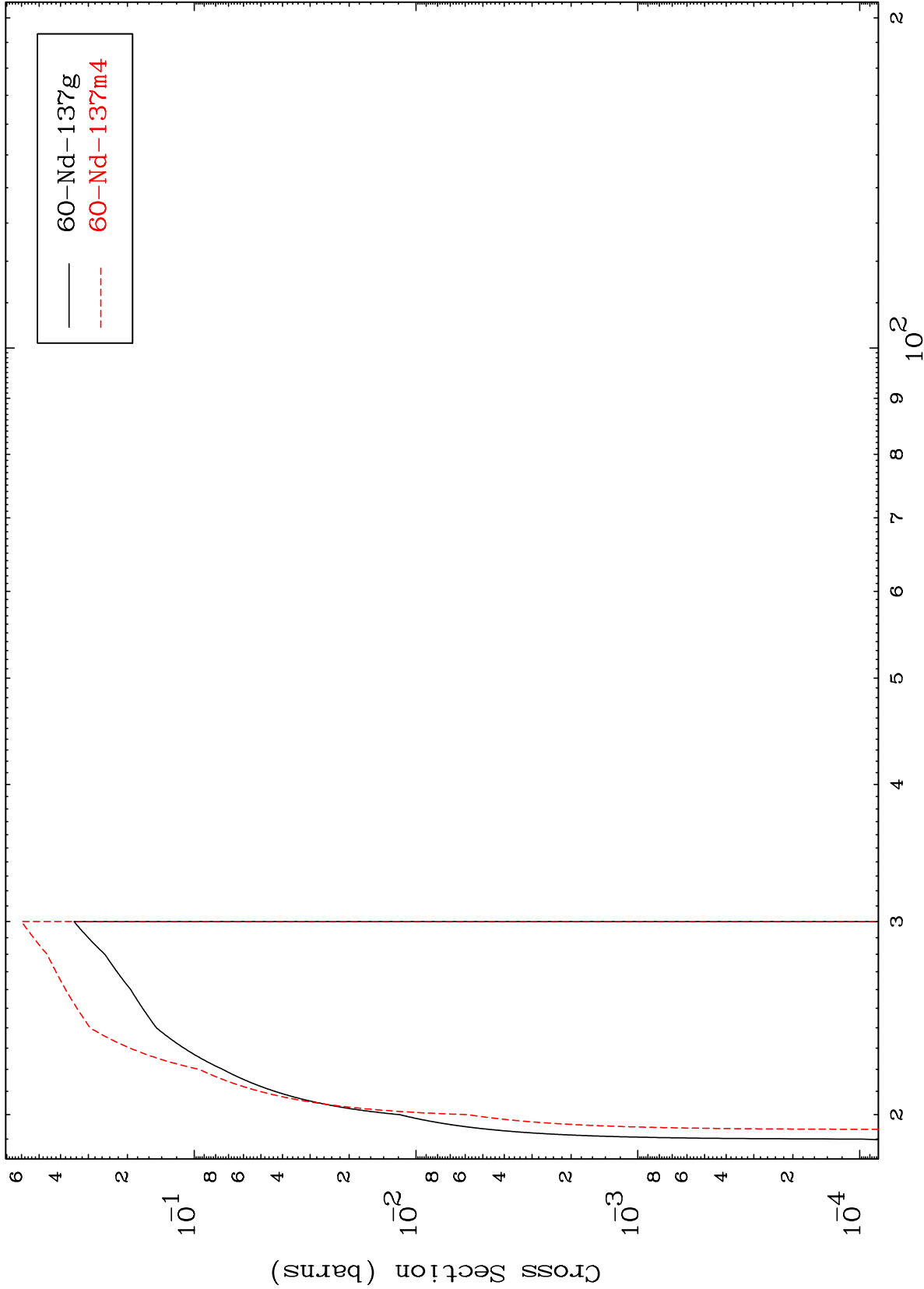
58-Ce-135m

MAT 5823

(n,2n)

58-Ce-135m

Radionuclide Production Cross Section



11

Incident Energy (MeV)

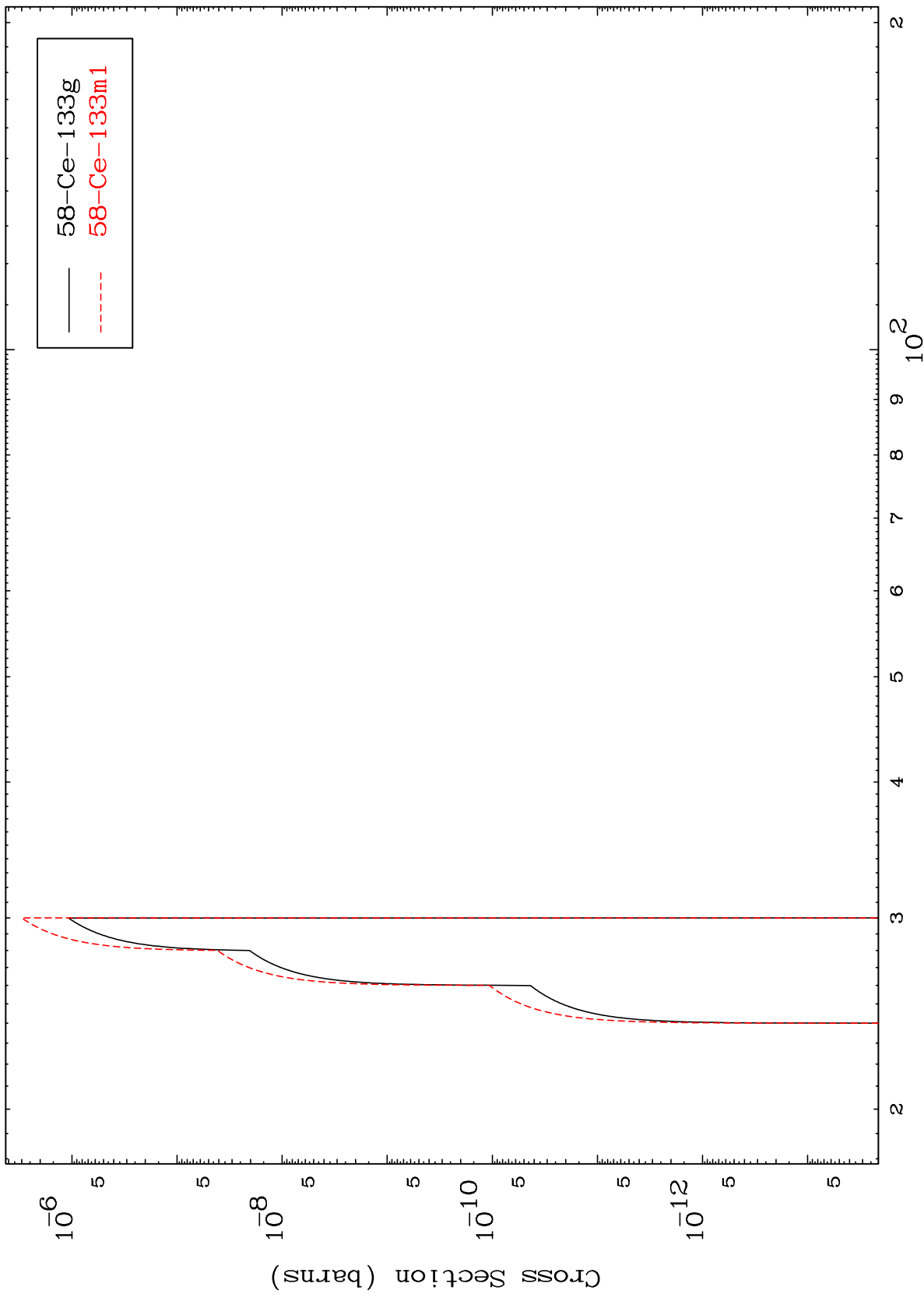
58-Ce-135m

MAT 5823

(n,2n)  $\alpha$

58-Ce-135m

Radionuclide Production Cross Section



12

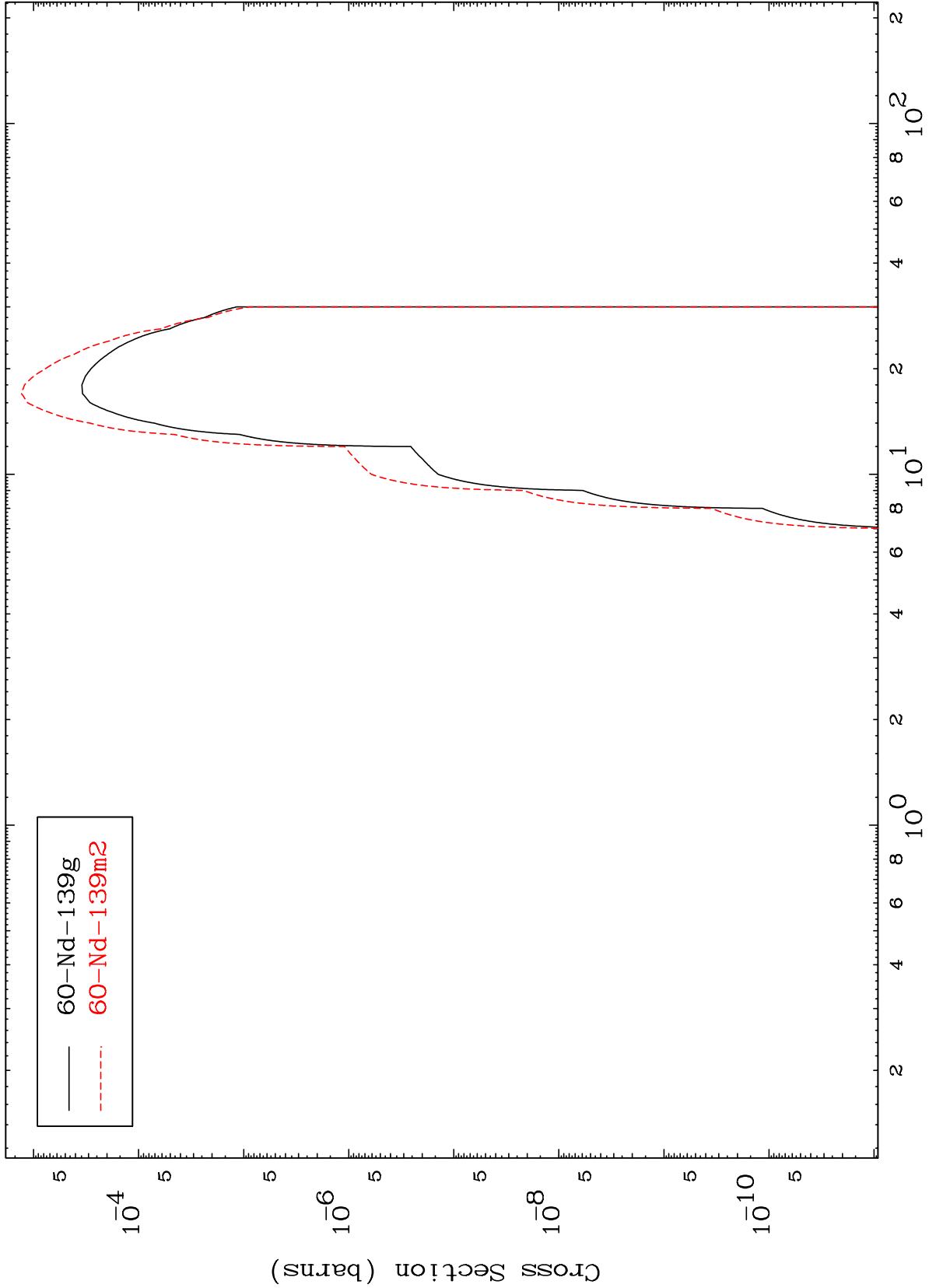
Incident Energy (MeV)

58-Ce-135m

MAT 5823

58-Ce-135m

(n,  $\gamma$ )  
Radionuclide Production Cross Section



— 60-Nd-139g  
- - - 60-Nd-139m2

58-Ce-135m

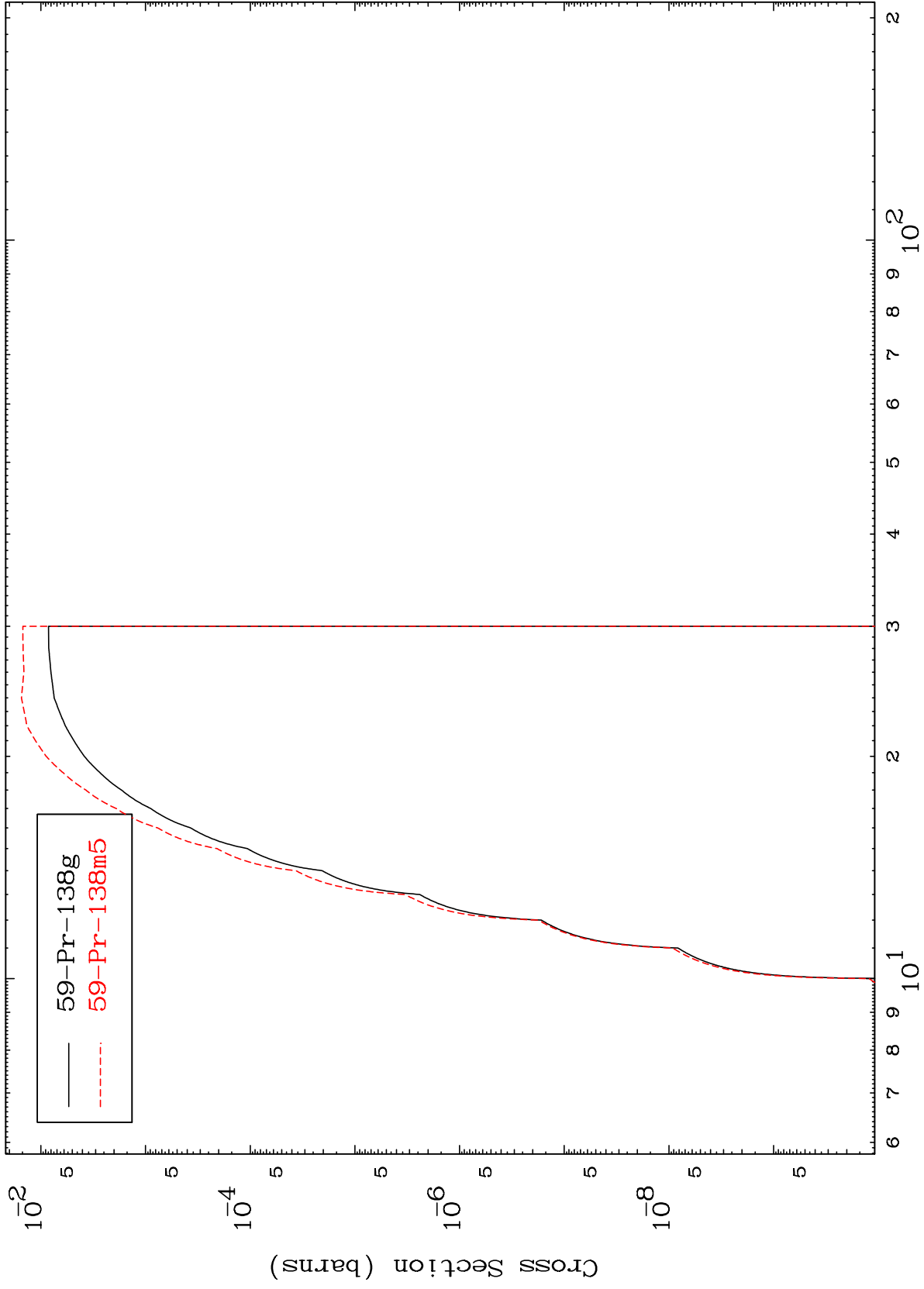
Incident Energy (MeV)

13

MAT 5823

58-Ce-135m

(n,p)  
Radionuclide Production Cross Section



14

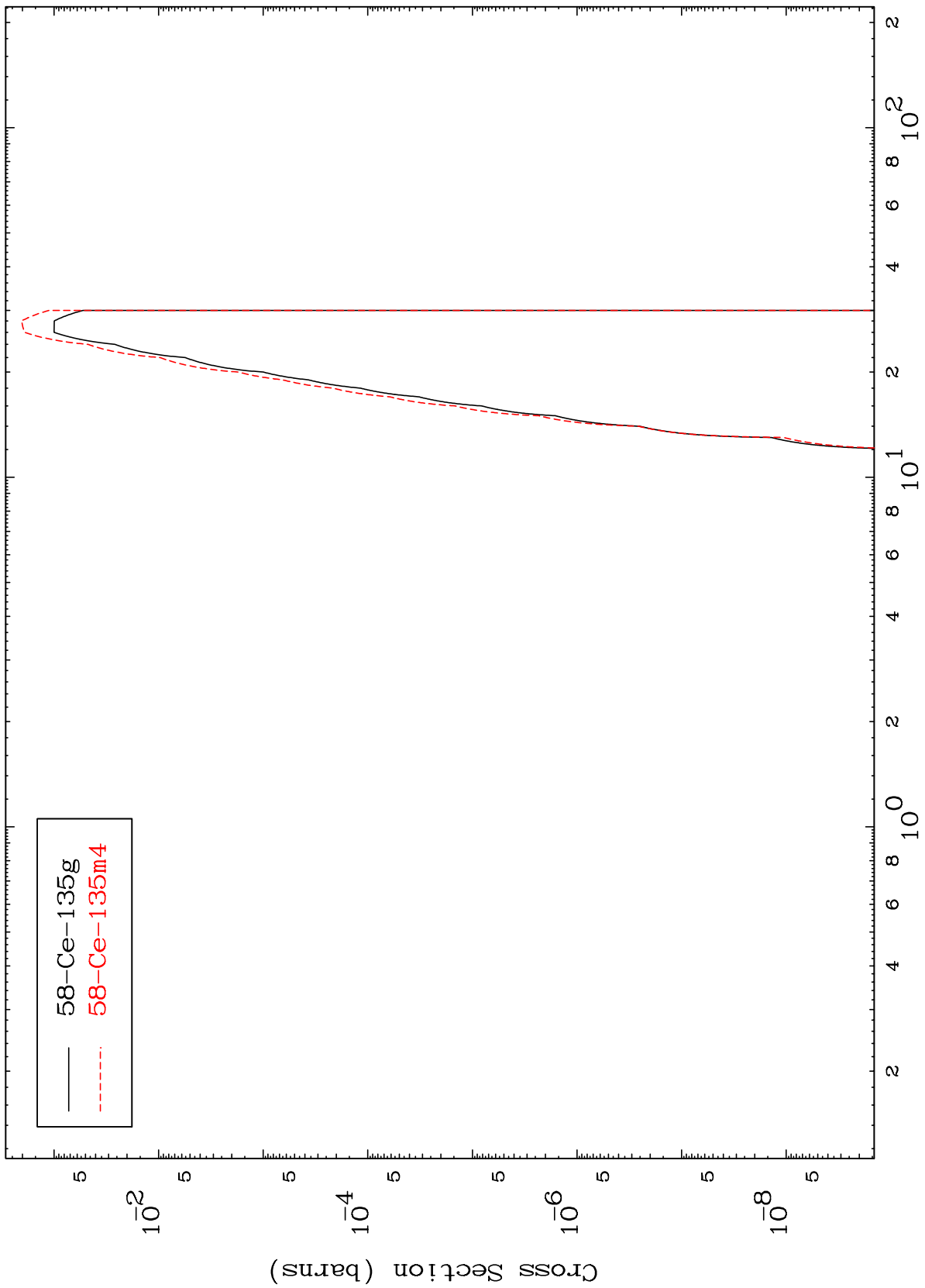
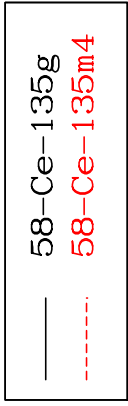
Incident Energy (MeV)

58-Ce-135m

MAT 5823

58-Ce-135m

Radionuclide Production Cross Section



58-Ce-135m

Incident Energy (MeV)

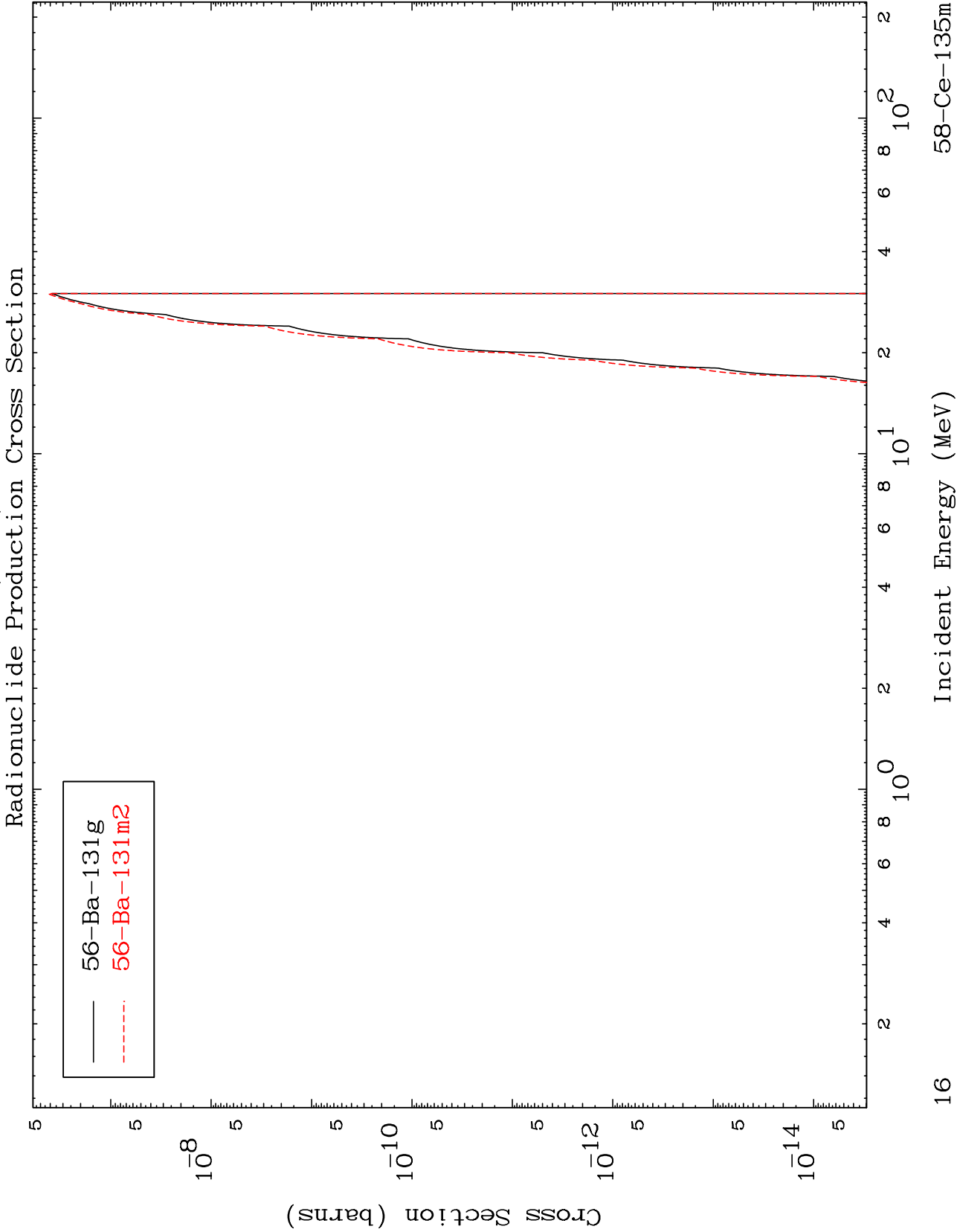
15



MAT 5823

(n,2α)

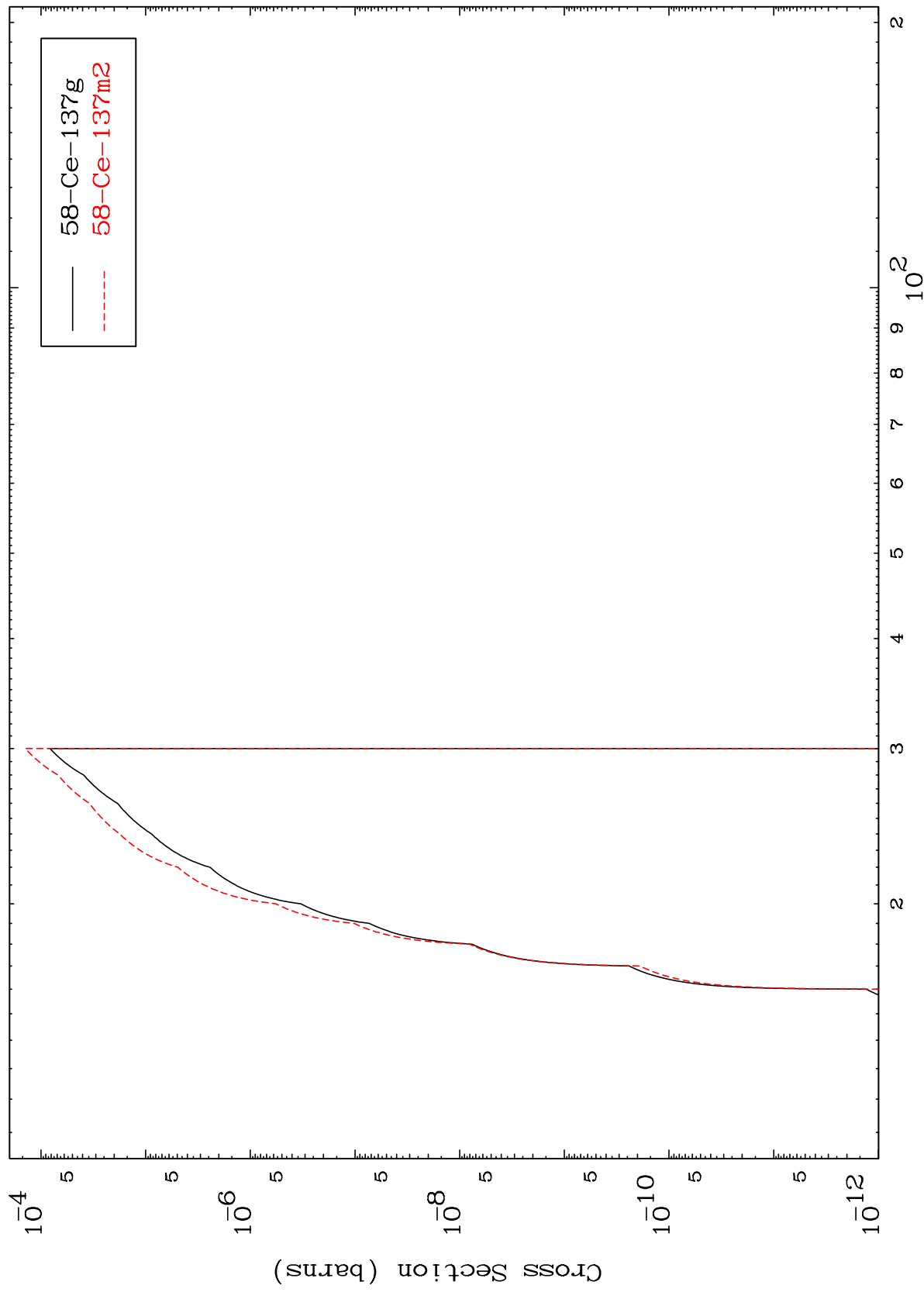
58-Ce-135m



MAT 5823

58-Ce-135m

(n,2p)  
Radionuclide Production Cross Section



17

Incident Energy (MeV)

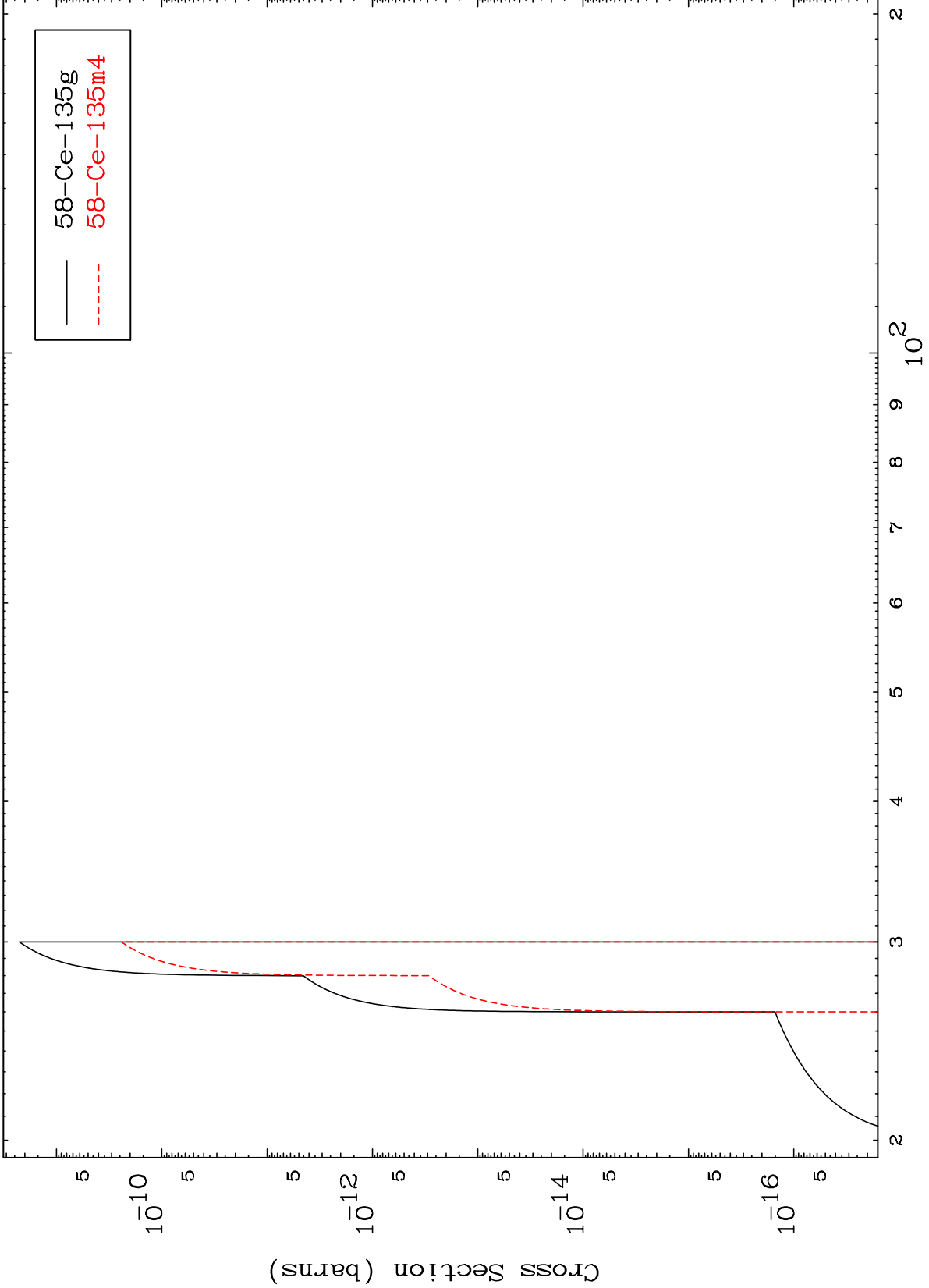
58-Ce-135m

MAT 5823

(n,p) t

58-Ce-135m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

58-Ce-135m