

Program EVALPLOT  
(Version 2021-1)

by

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Press Mouse Button to Start

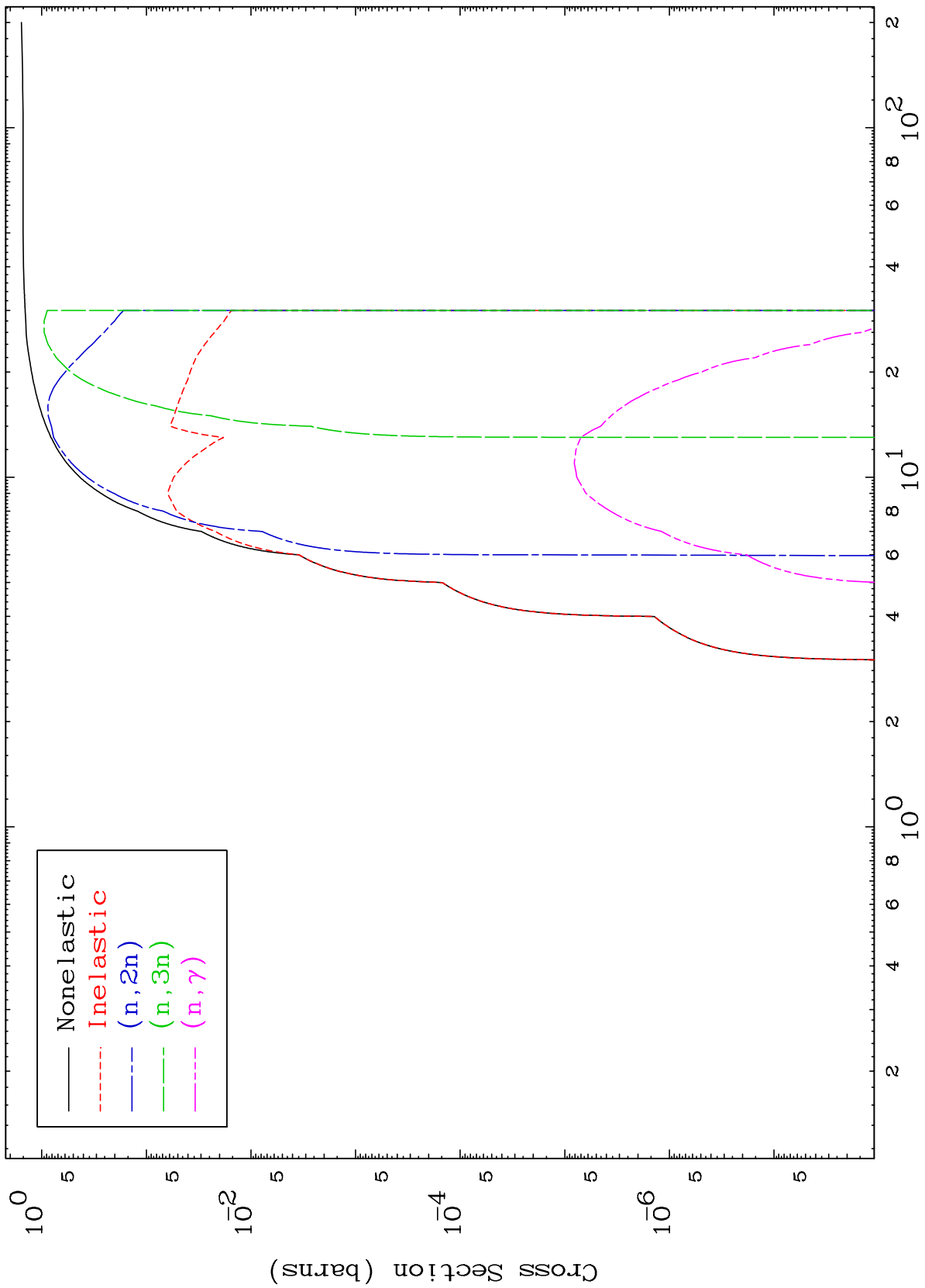
MAT 2446

0 Kelvin

$\alpha$  Major

Cross Sections

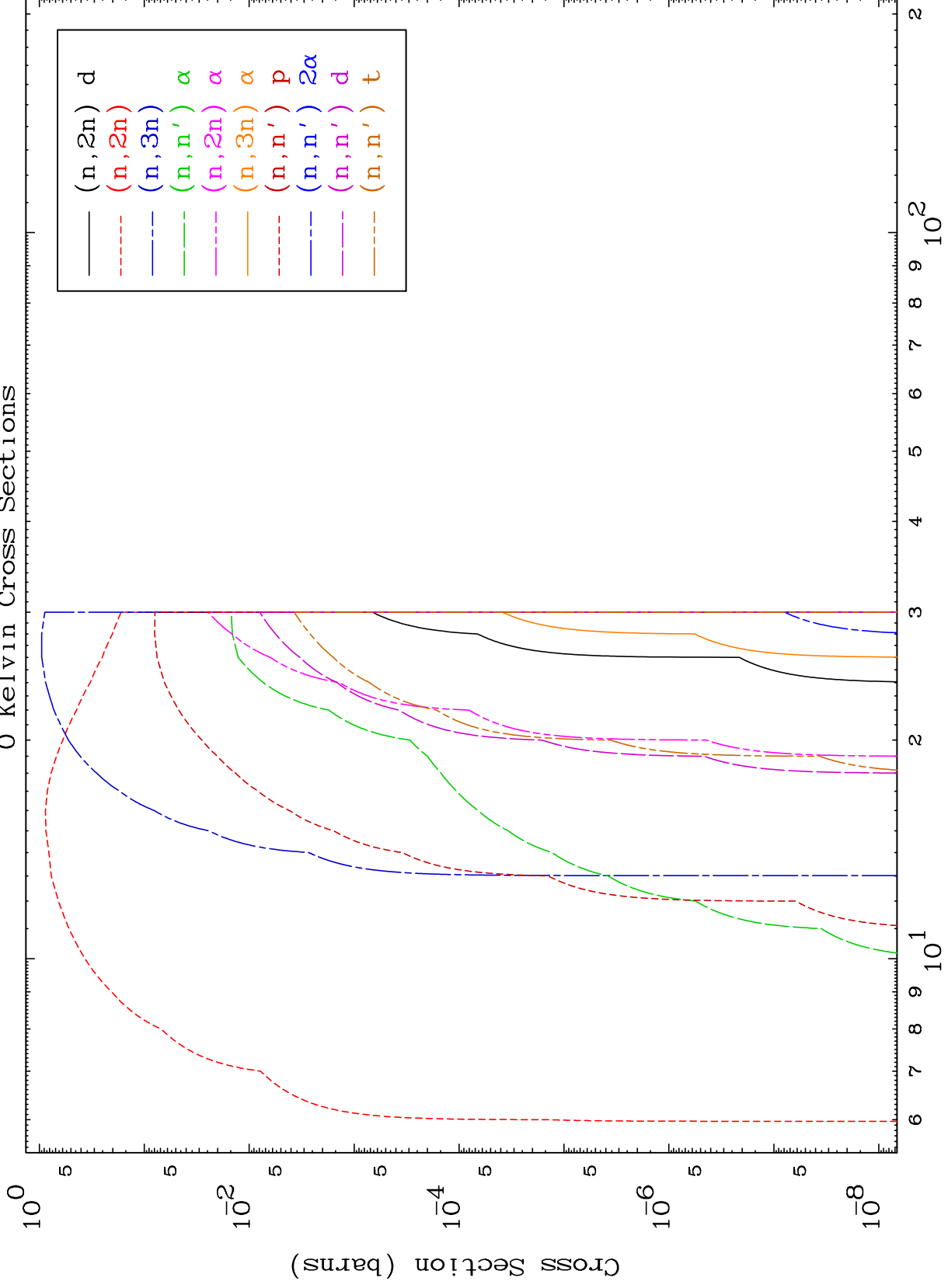
24-Cr-57



MAT 2446

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

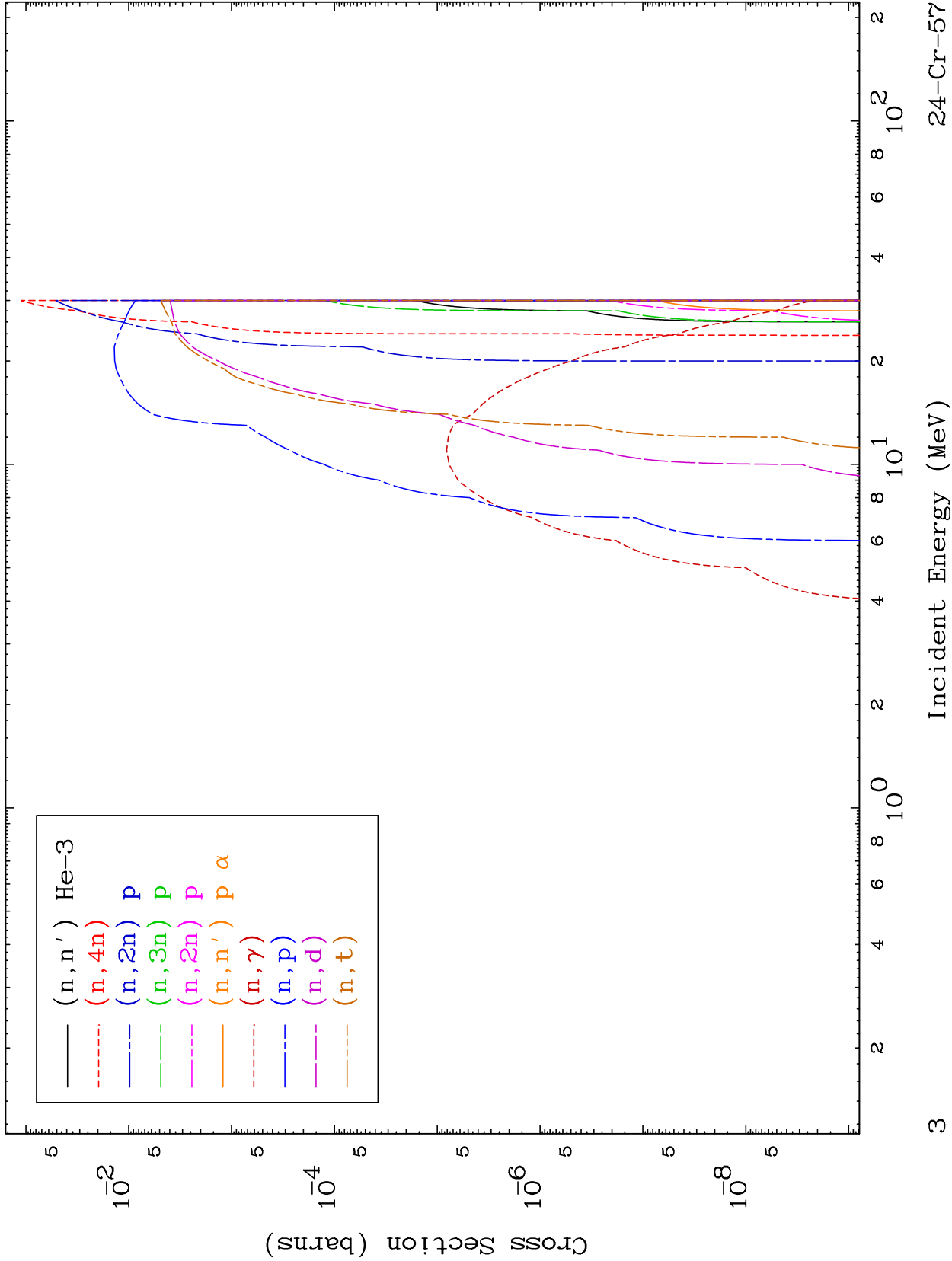
24-Cr-57



2

Incident Energy (MeV)

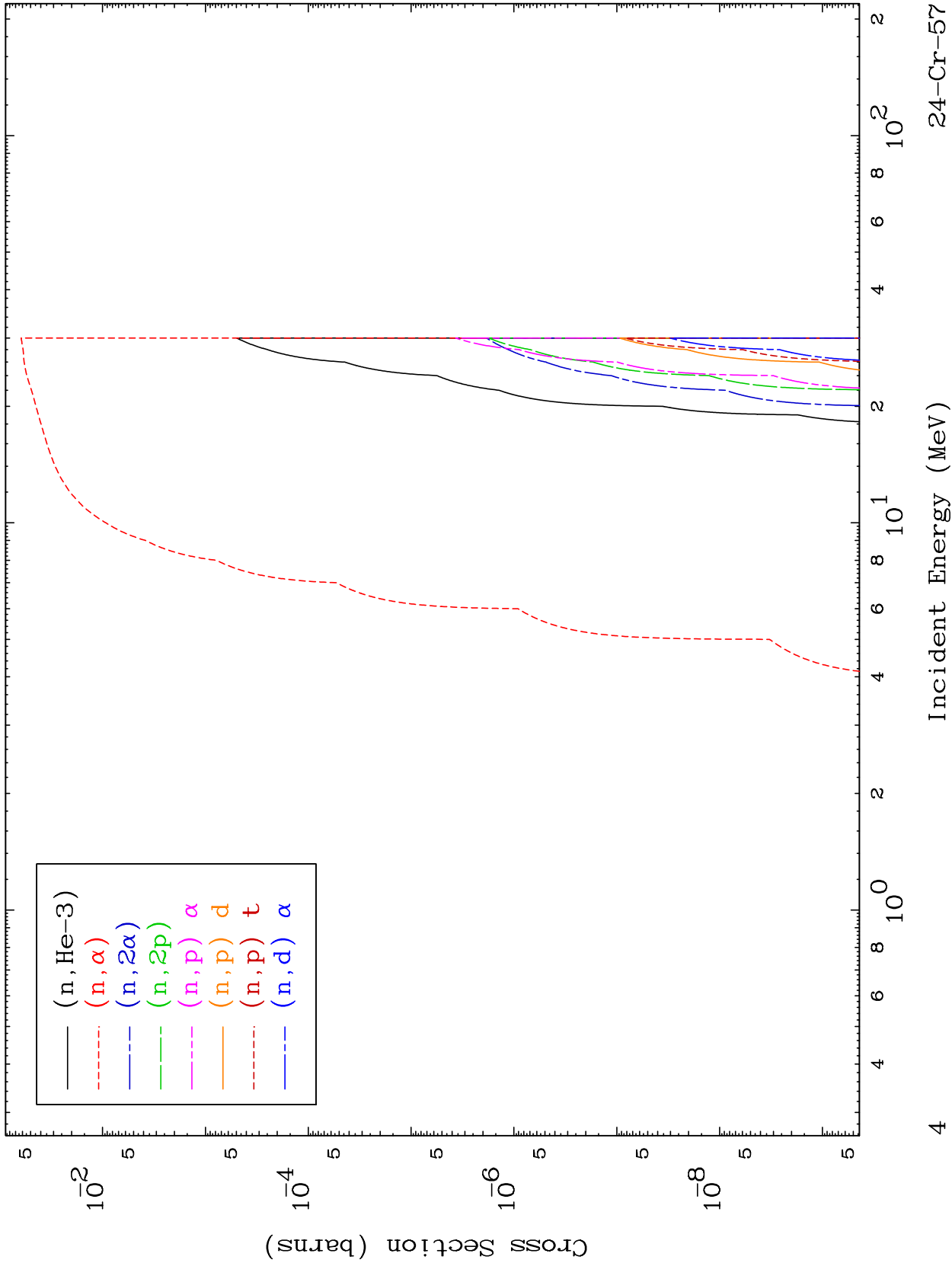
24-Cr-57

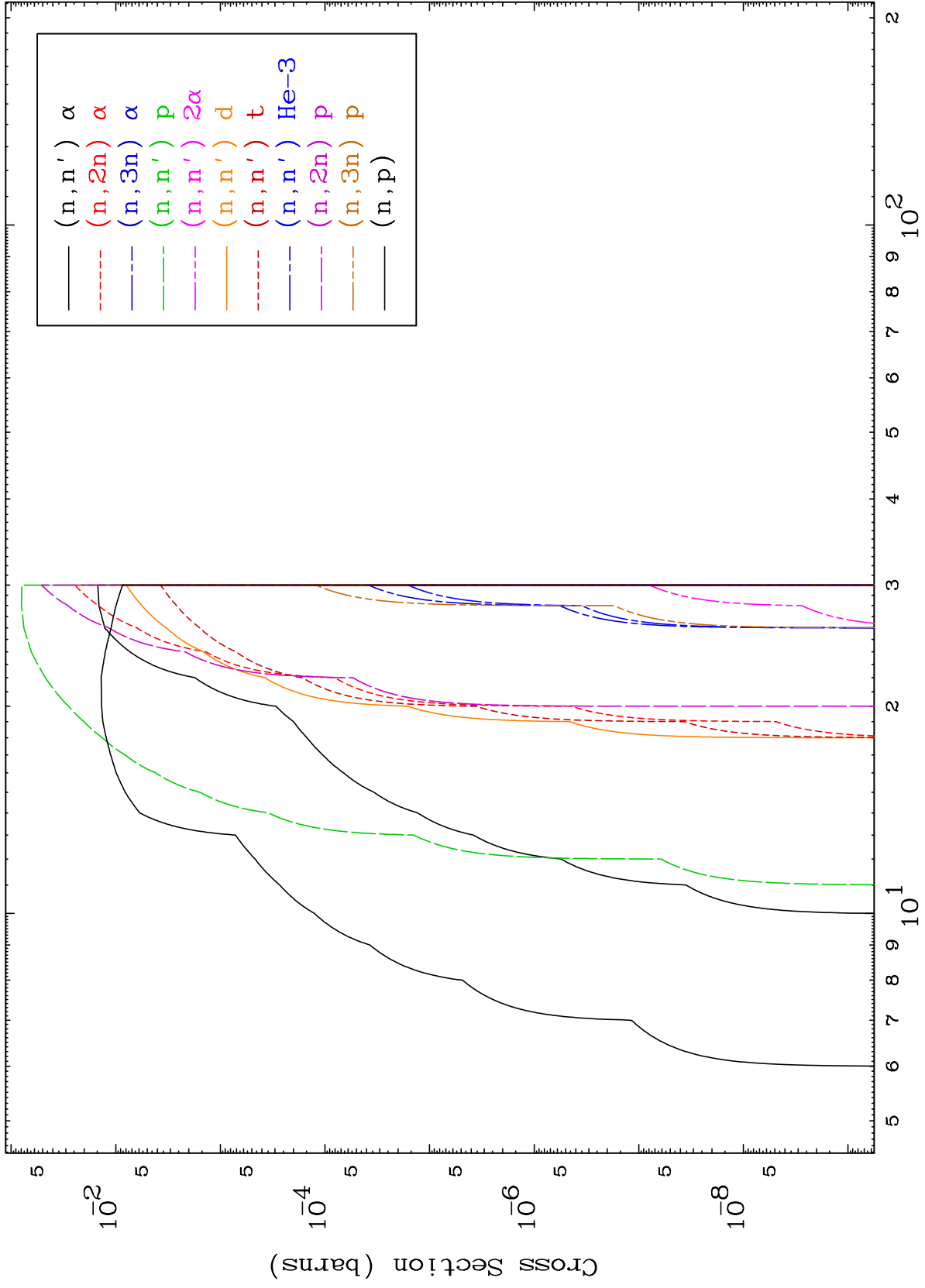


MAT 2446

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

24-Cr-57

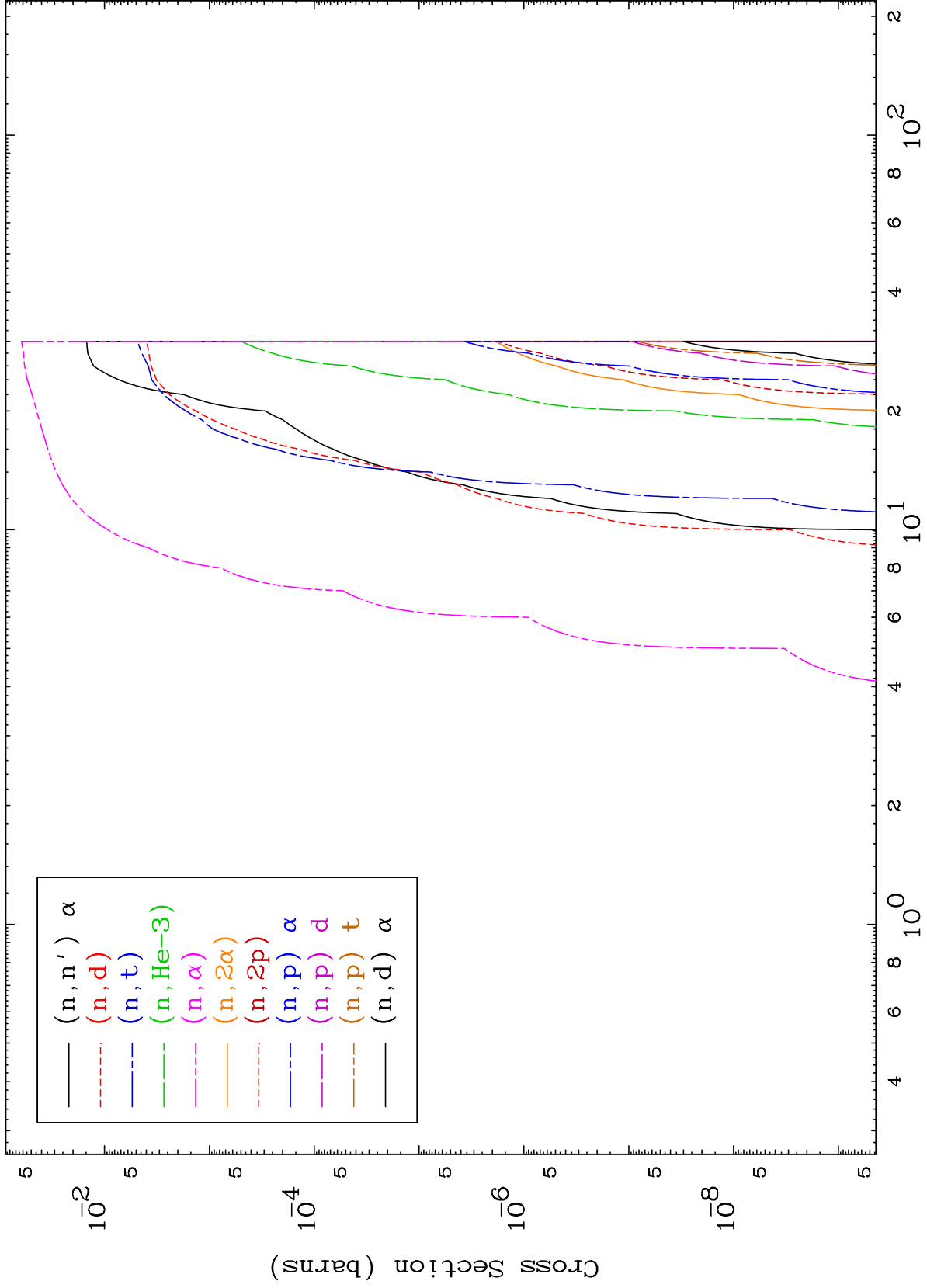




MAT 2446

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

24-Cr-57



6

Incident Energy (MeV)

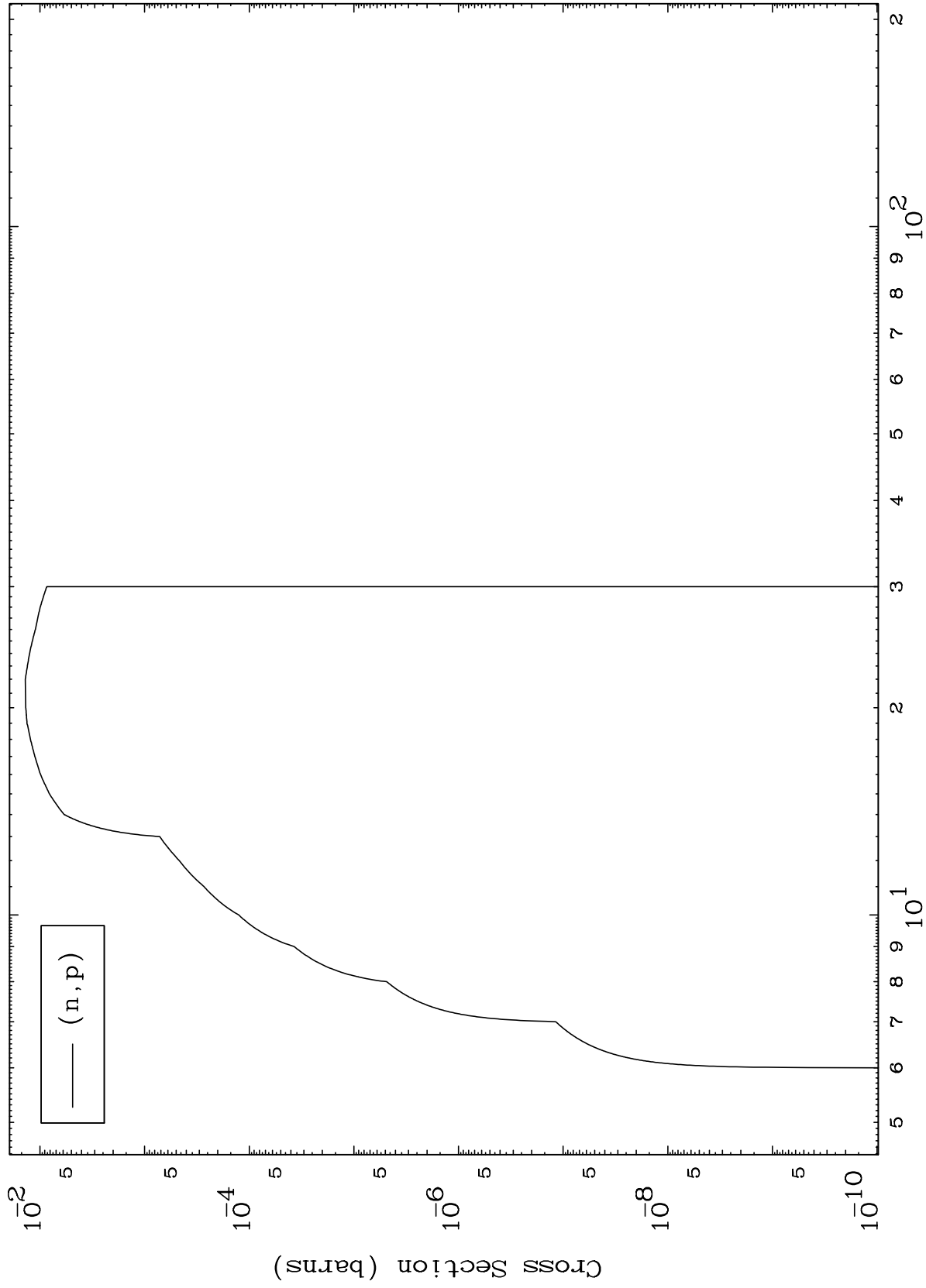
24-Cr-57

MAT 2446

( $\alpha, p$ ) Levels

24-Cr-57

0 Kelvin Cross Sections



7

Incident Energy (MeV)

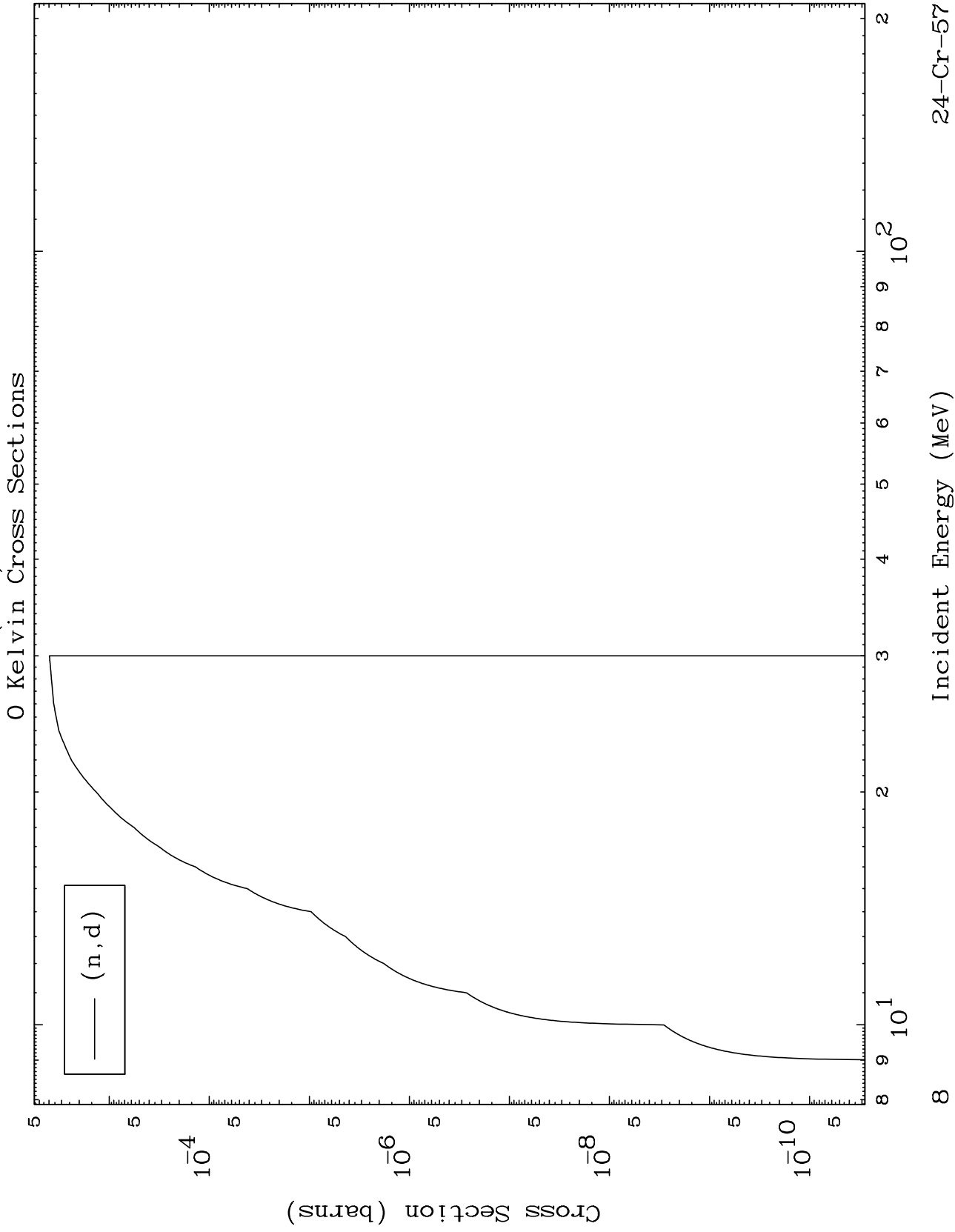
24-Cr-57



MAT 2446

( $\alpha, d$ ) Levels

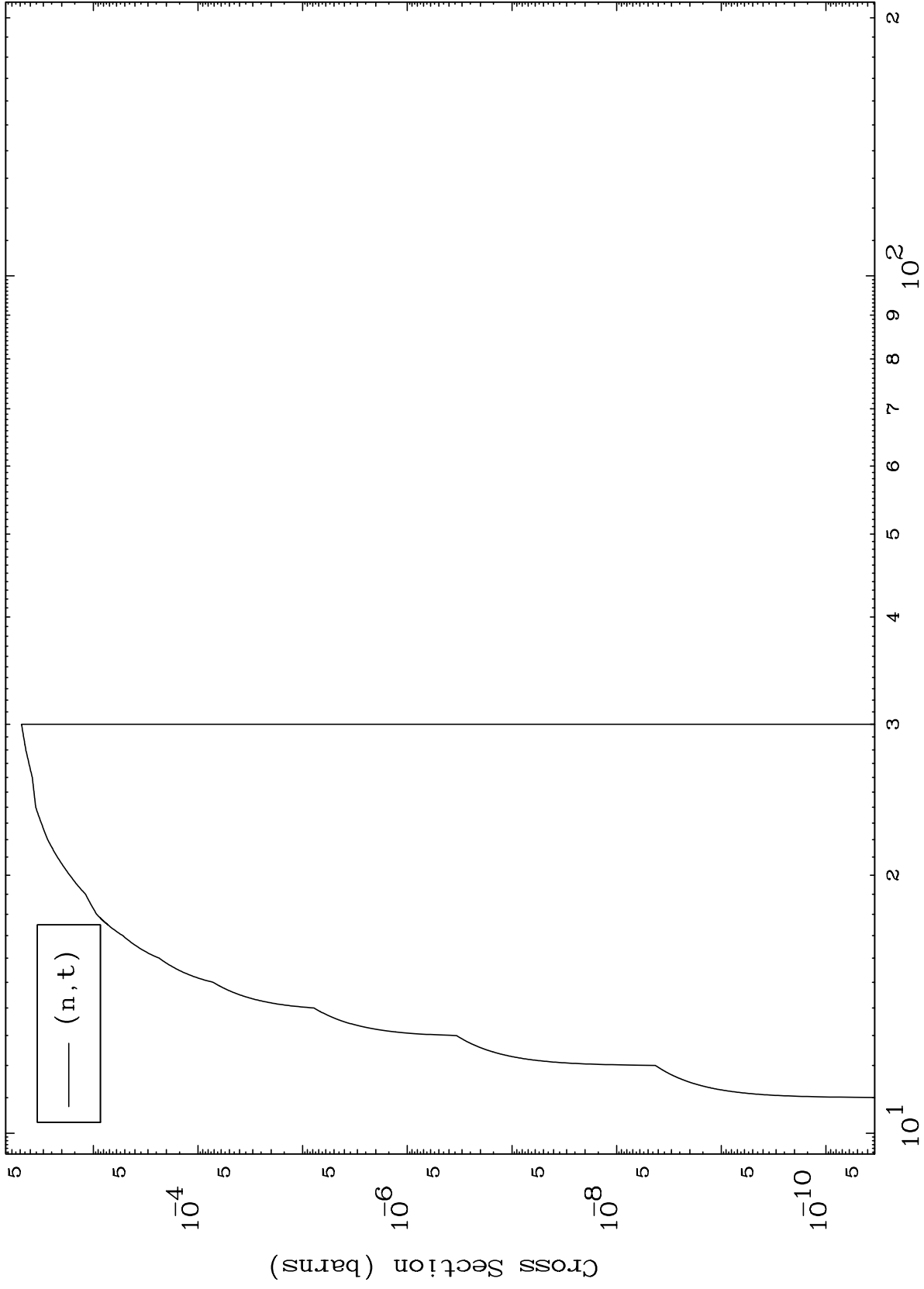
24-Cr-57



MAT 2446

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

24-Cr-57



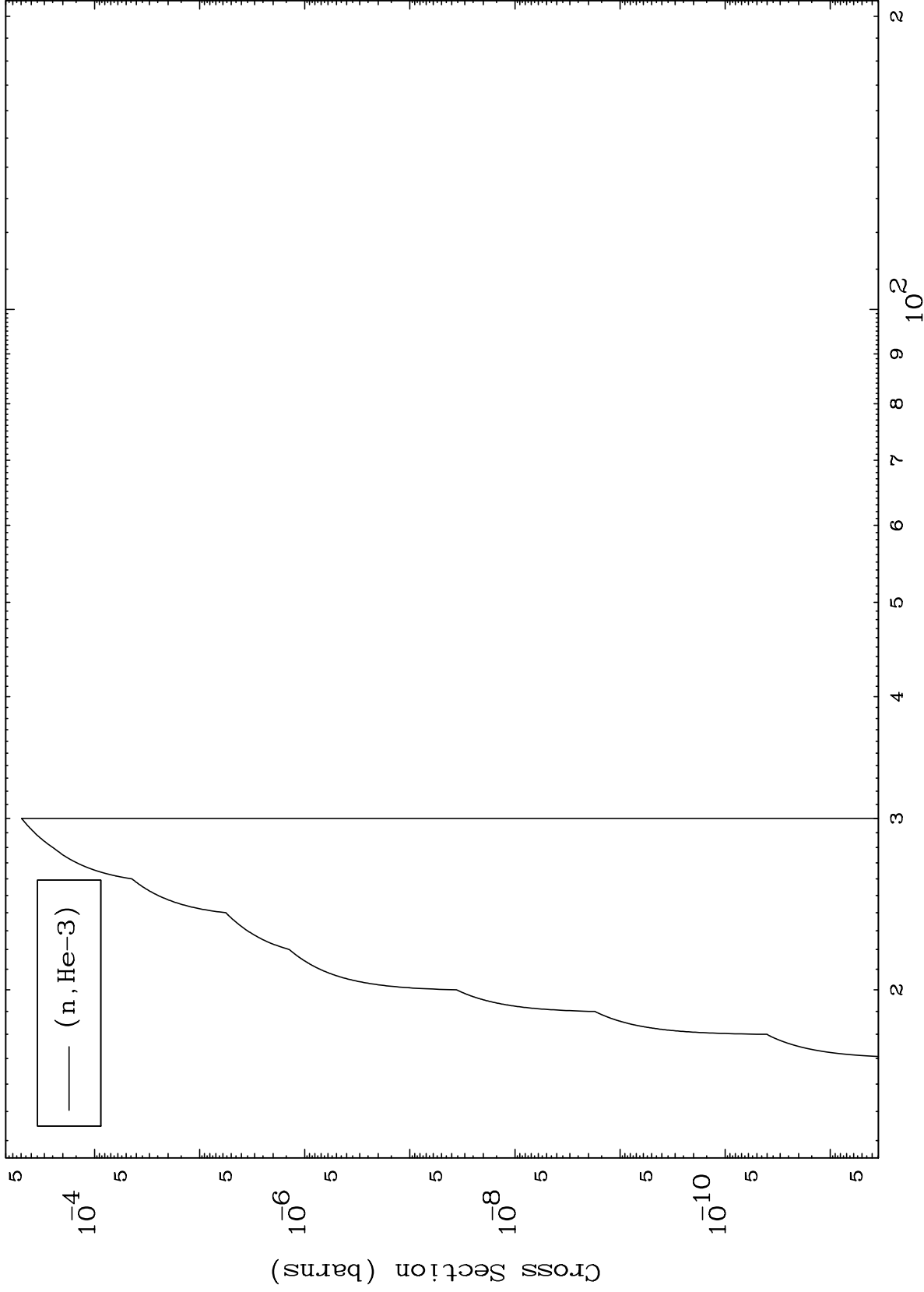
Incident Energy (MeV)

24-Cr-57

MAT 2446

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

24-Cr-57



10

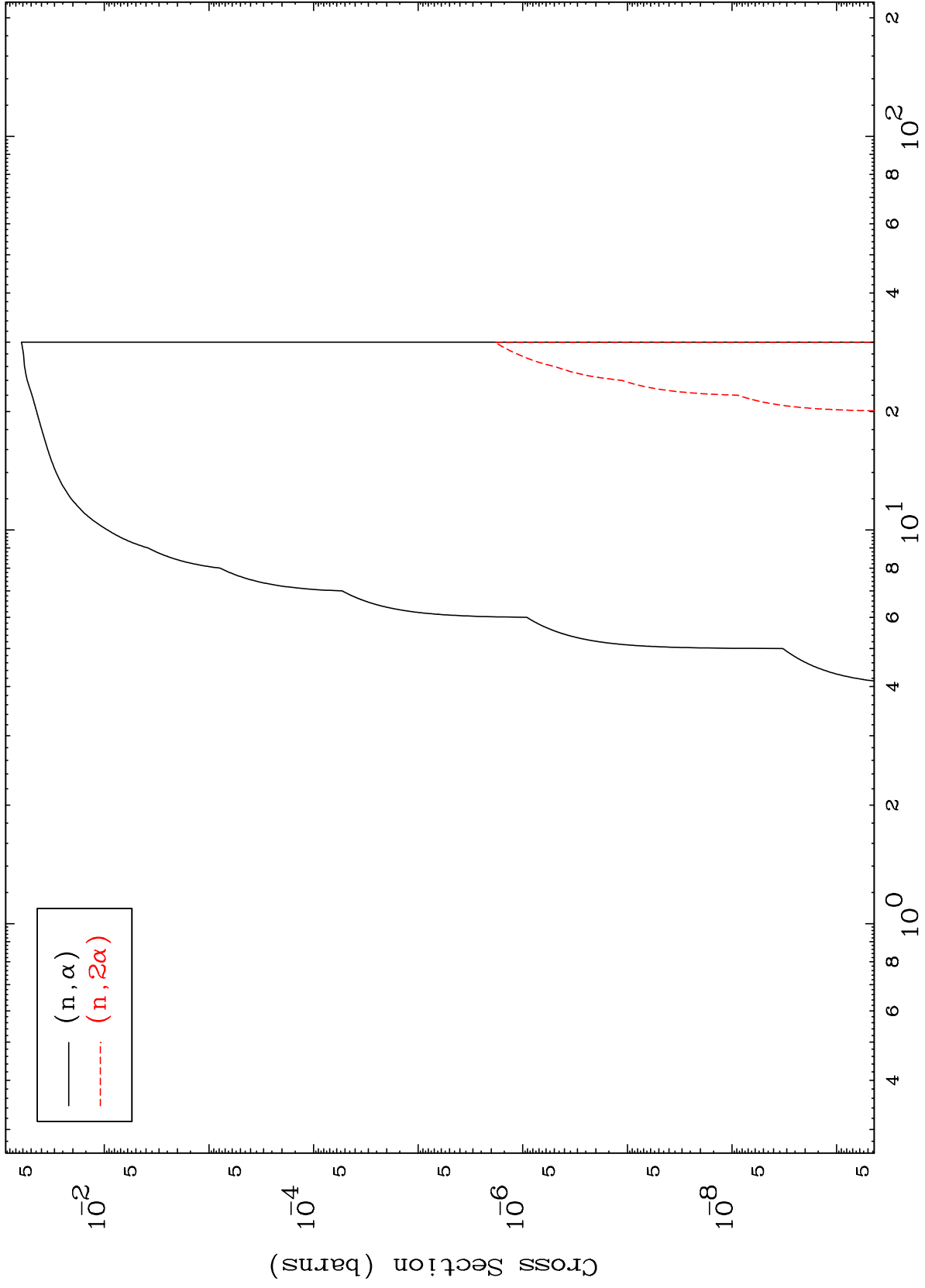
Incident Energy (MeV)

24-Cr-57

MAT 2446

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

24-Cr-57

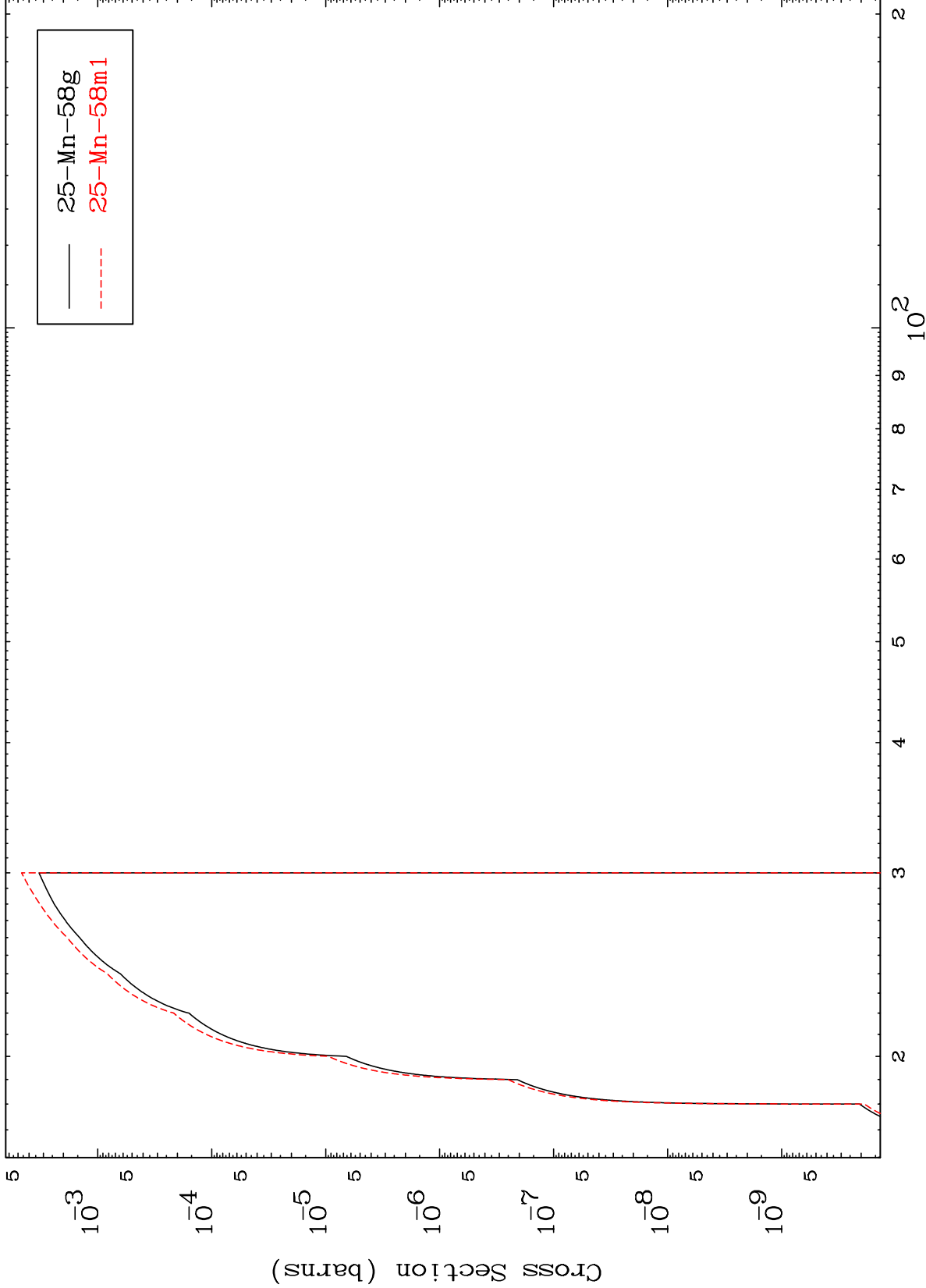


MAT 2446

(n,n') d

24-Cr-57

Radionuclide Production Cross Section



12

Incident Energy (MeV)

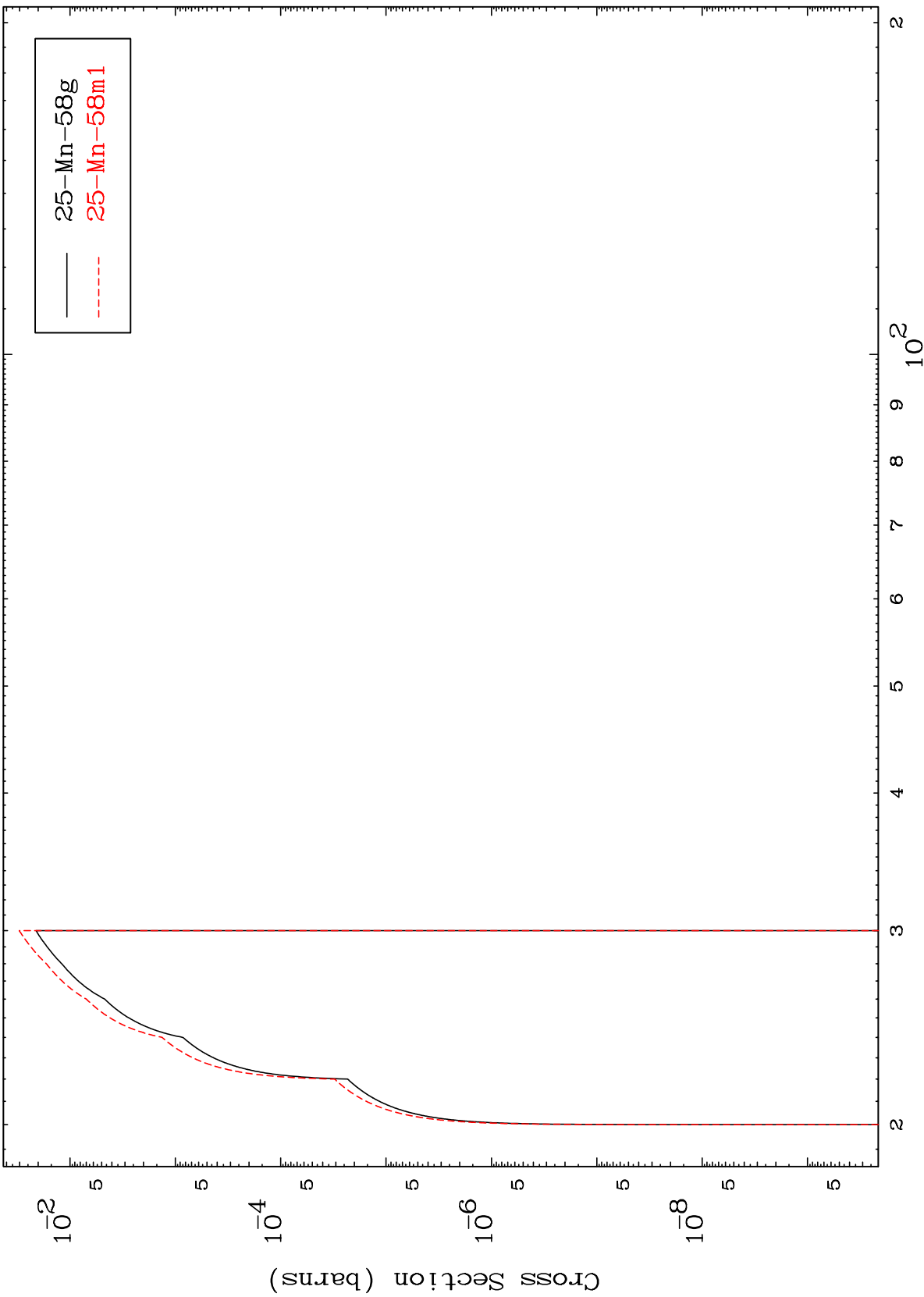
24-Cr-57

MAT 2446

(n,2n) p

24-Cr-57

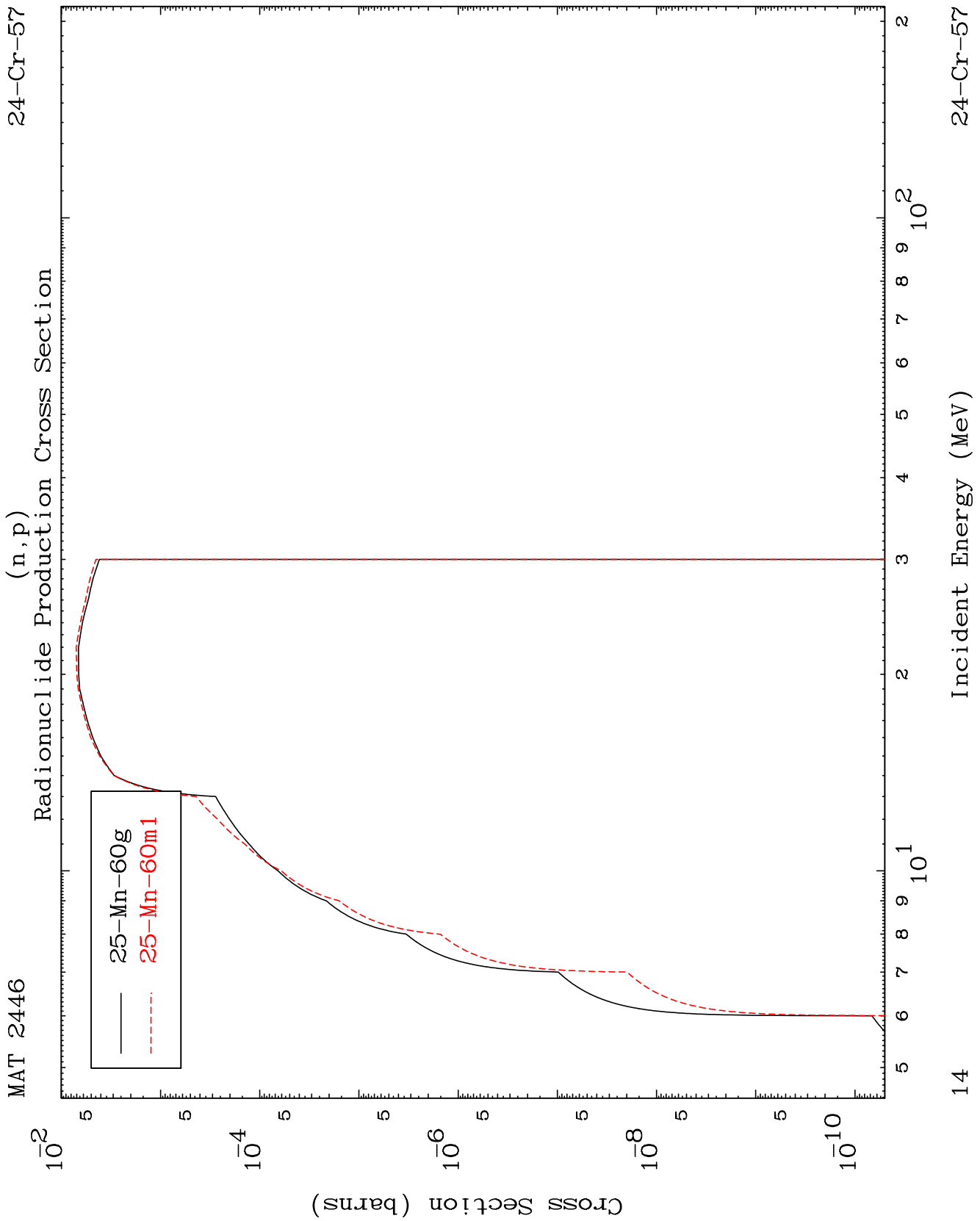
Radionuclide Production Cross Section



13

Incident Energy (MeV)

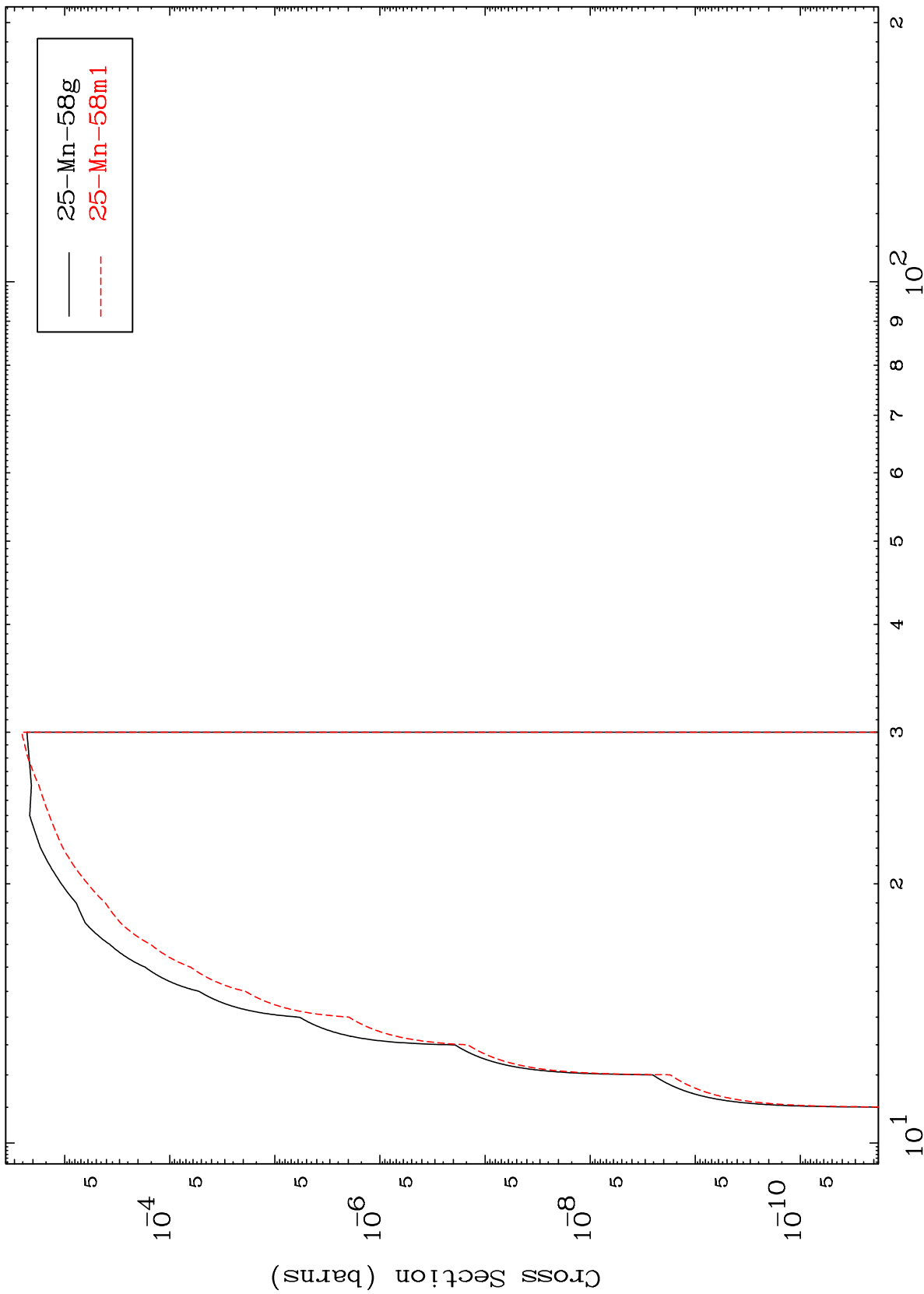
24-Cr-57



MAT 2446

24-Cr-57

(n,t)  
Radionuclide Production Cross Section



24-Cr-57

Incident Energy (MeV)