

Program EVALPLOT
(Version 2021-1)

by

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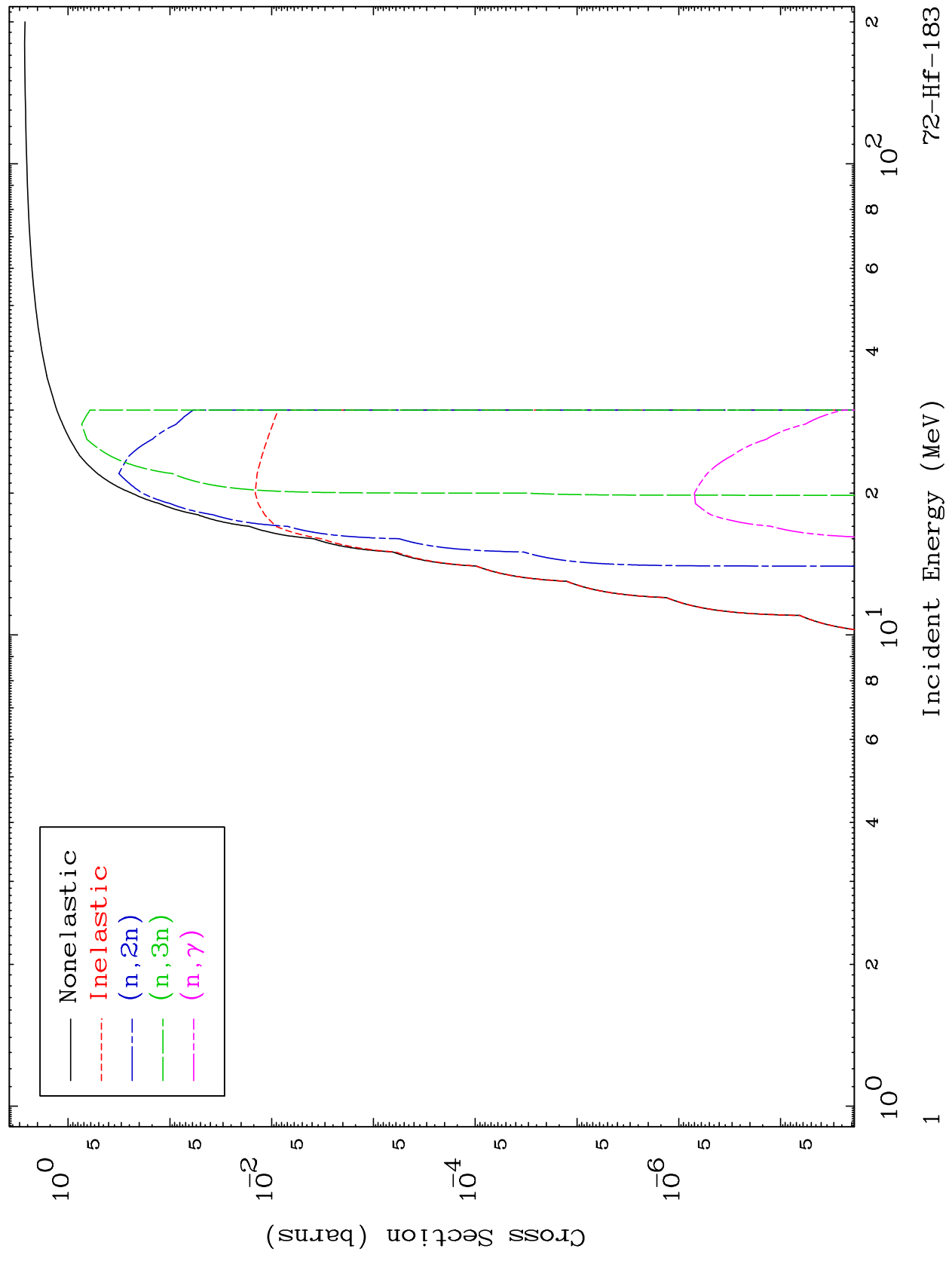
Press Mouse Button to Start

MAT 7252

0 Kelvin

Major Cross Sections

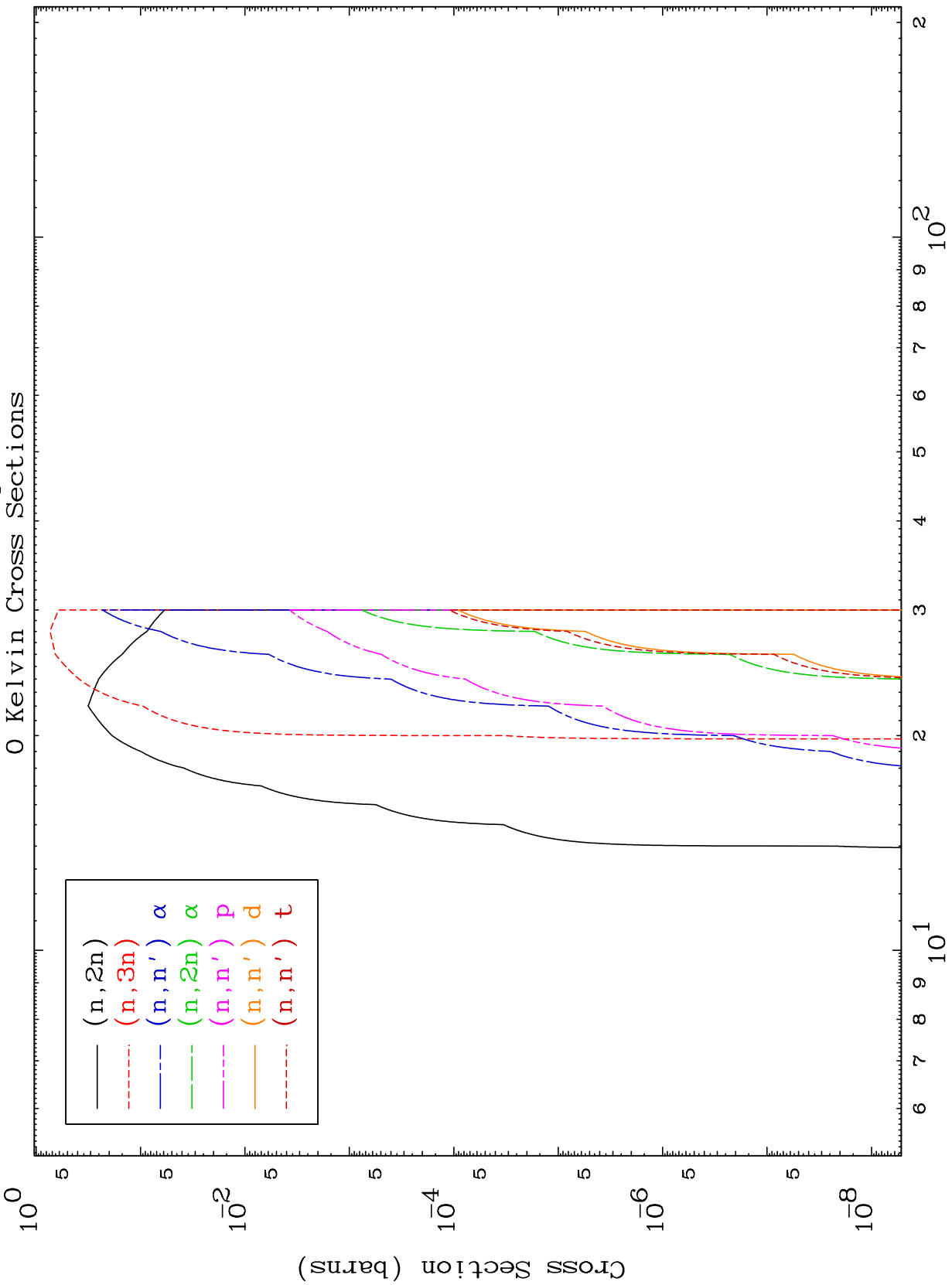
72-Hf-183



72-Hf-183

Incident Energy (MeV)

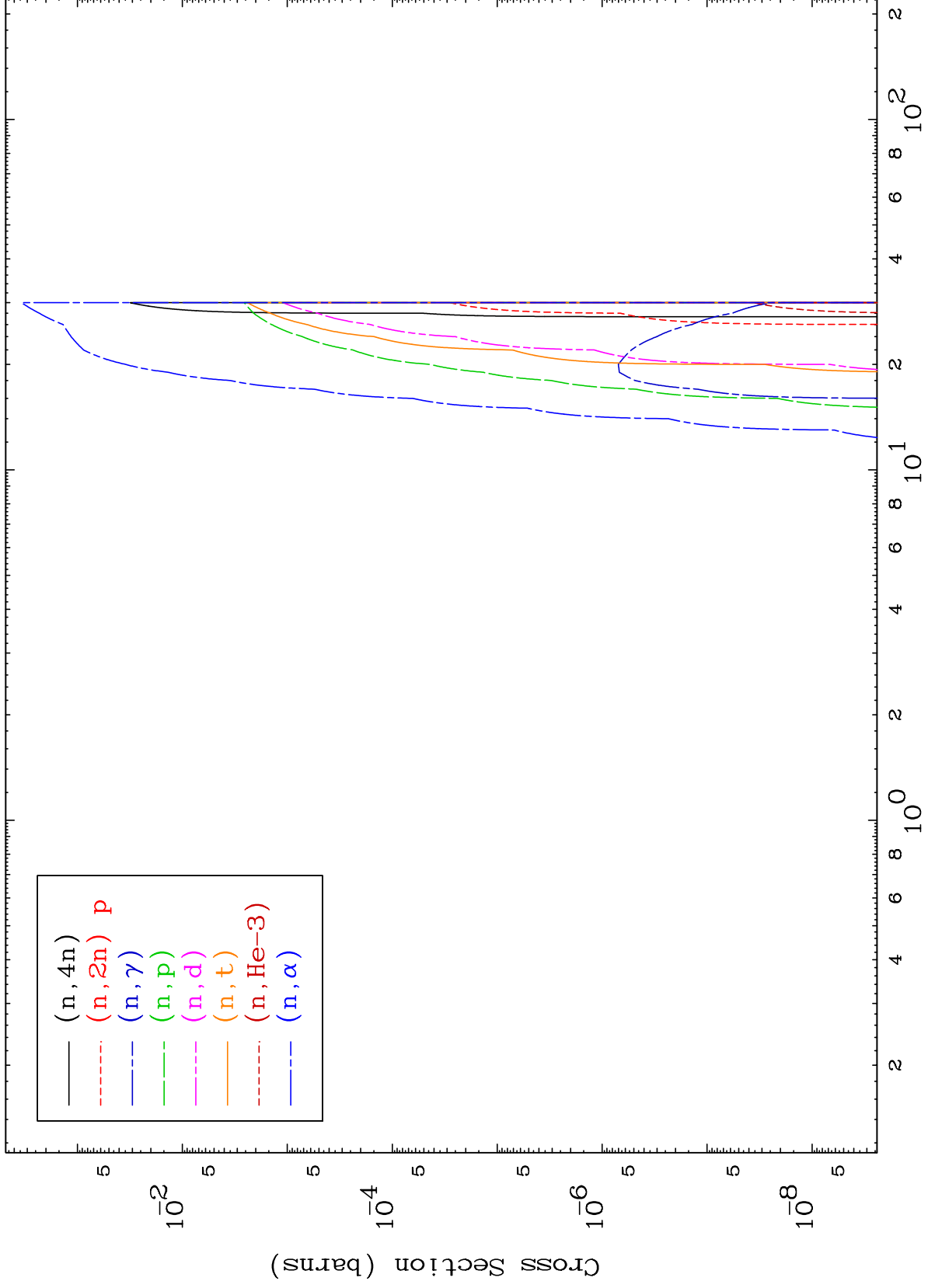
α Neutron Absorption
0 Kelvin Cross Sections

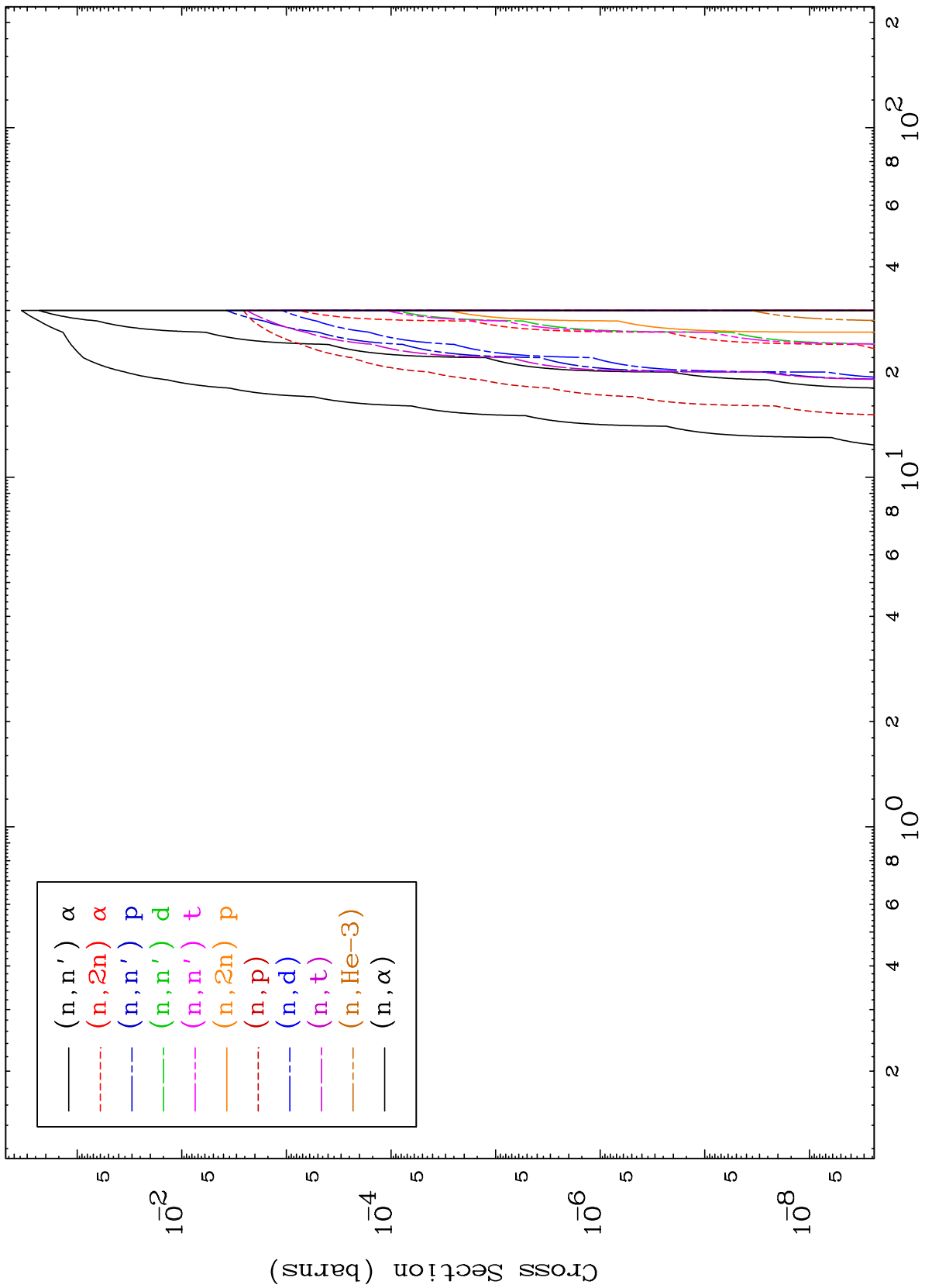


MAT 7252

α Neutron Absorption
0 Kelvin Cross Sections

72-Hf-183



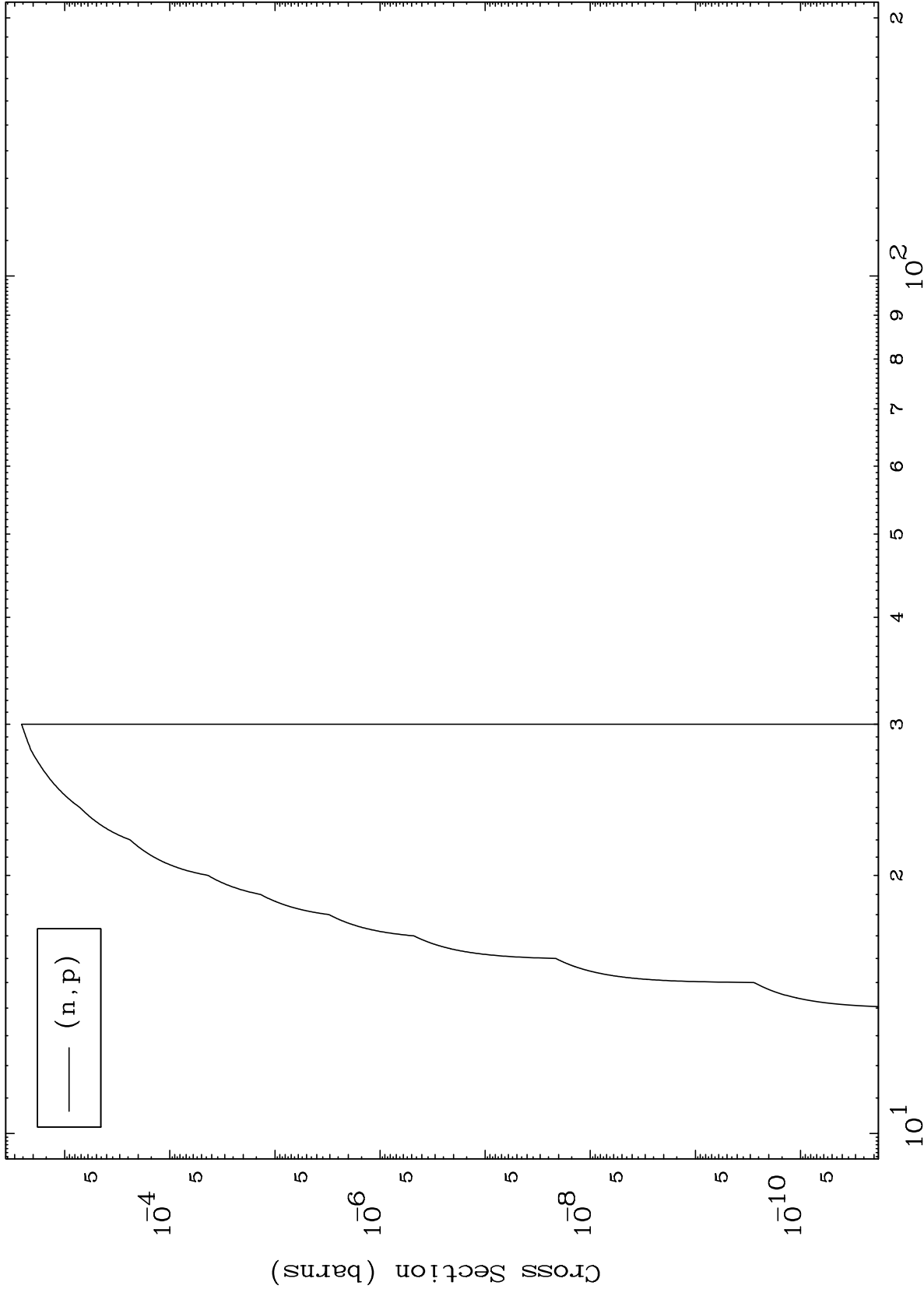


MAT 7252

(α, p) Levels

72-Hf-183

0 Kelvin Cross Sections



(n, p)

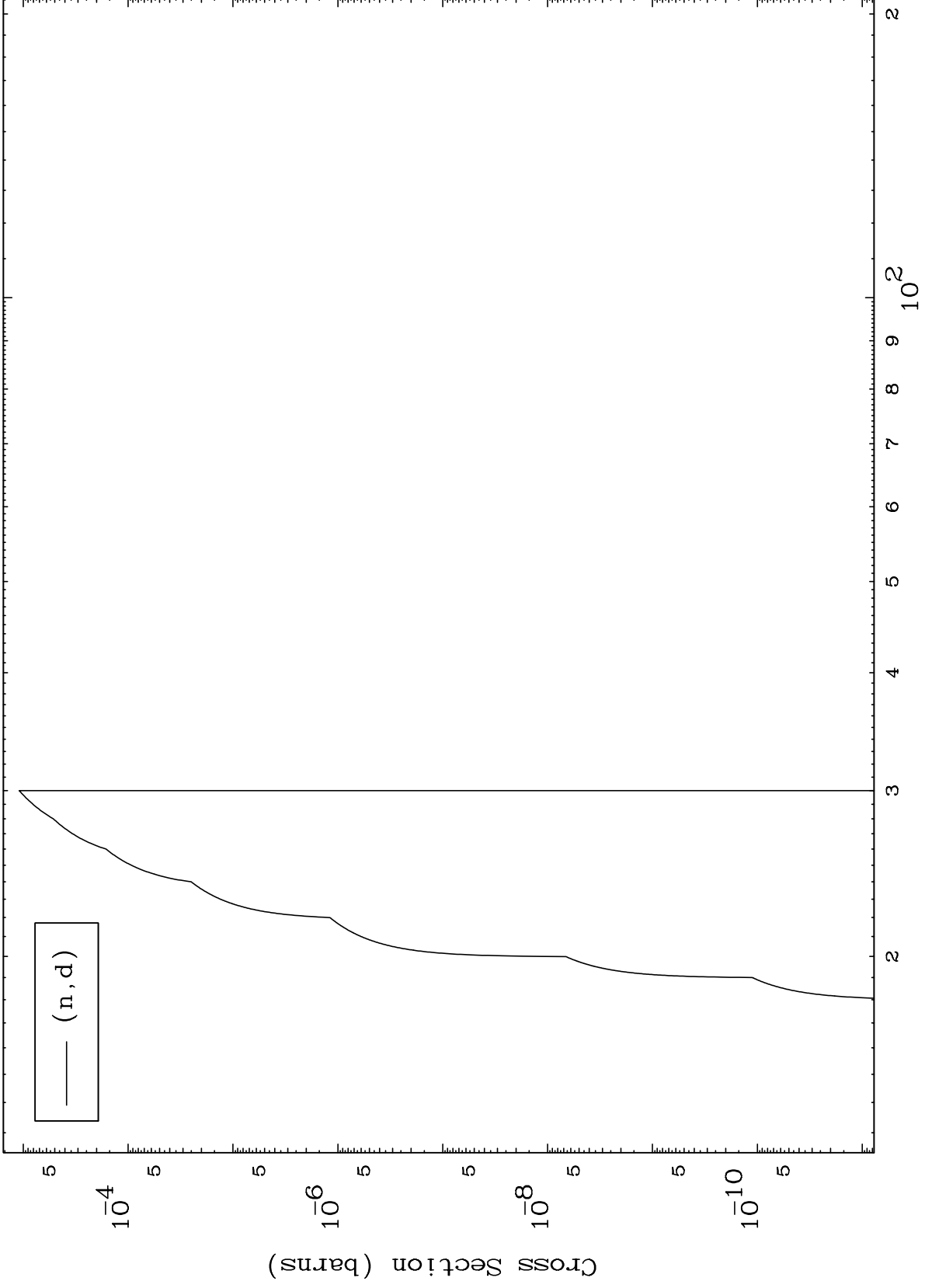
Incident Energy (MeV)

72-Hf-183

MAT 7252

(α, d) Levels
0 Kelvin Cross Sections

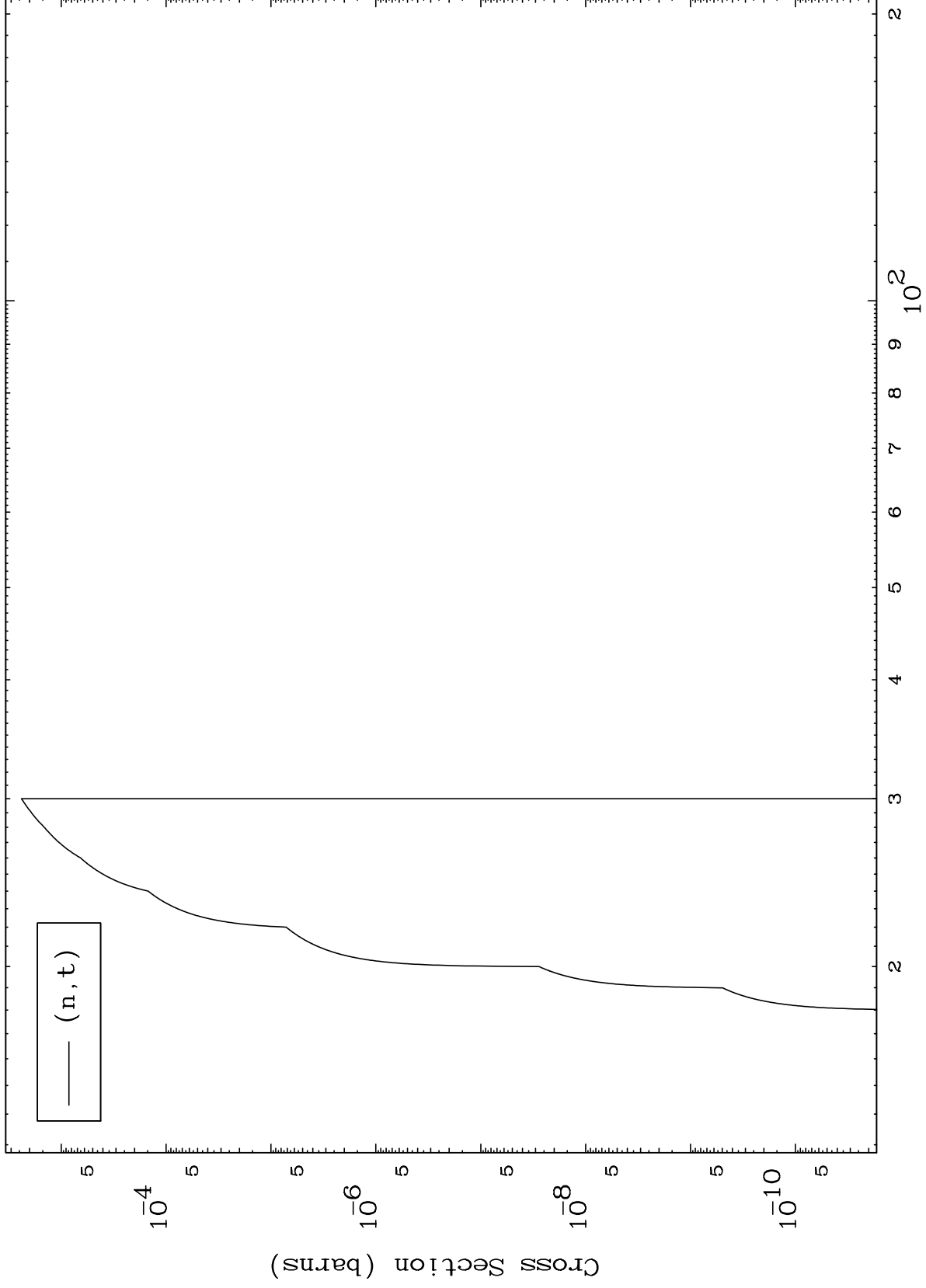
72-Hf-183



MAT 7252

(α, t) Levels
0 Kelvin Cross Sections

72-Hf-183

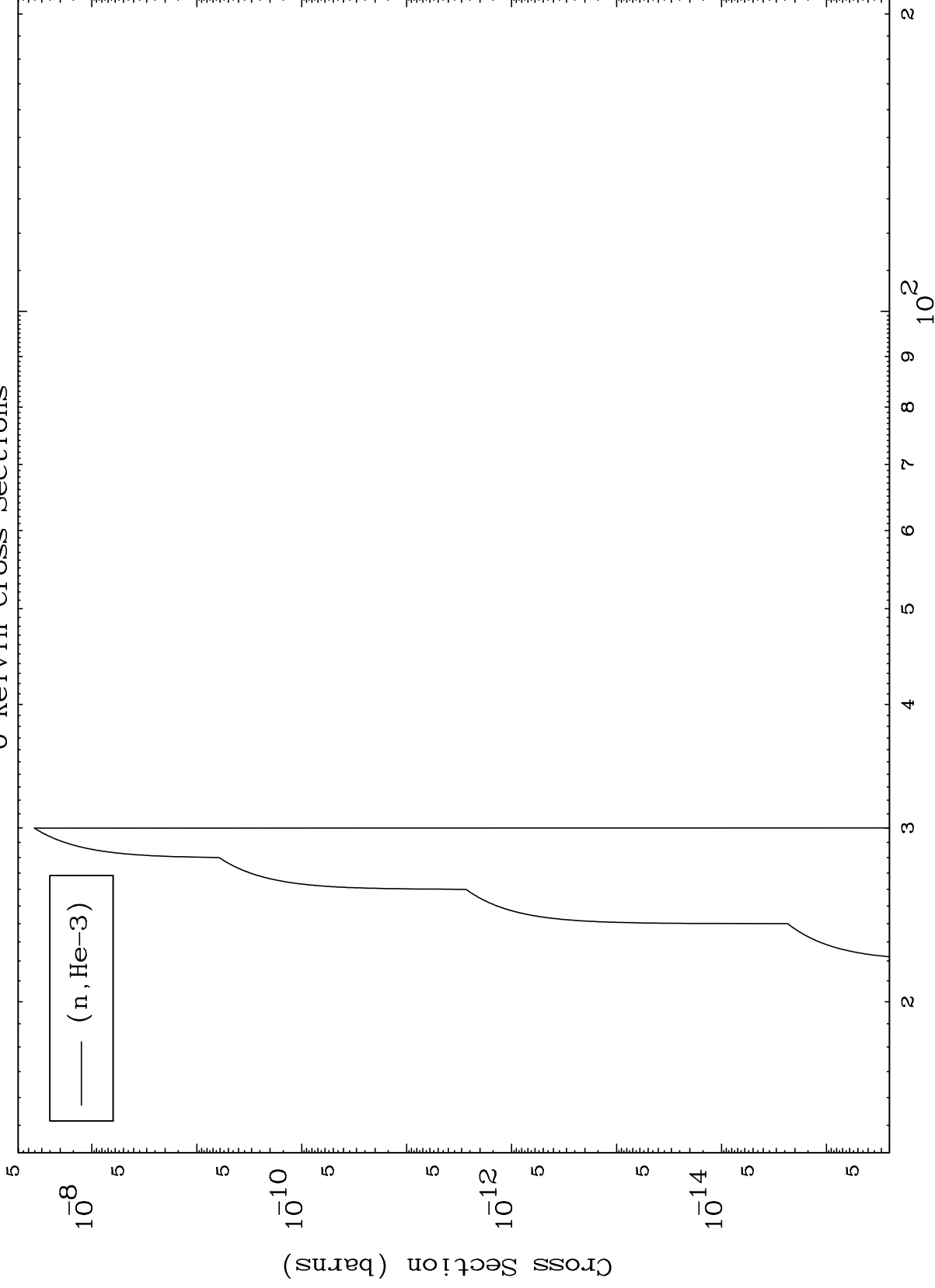


7

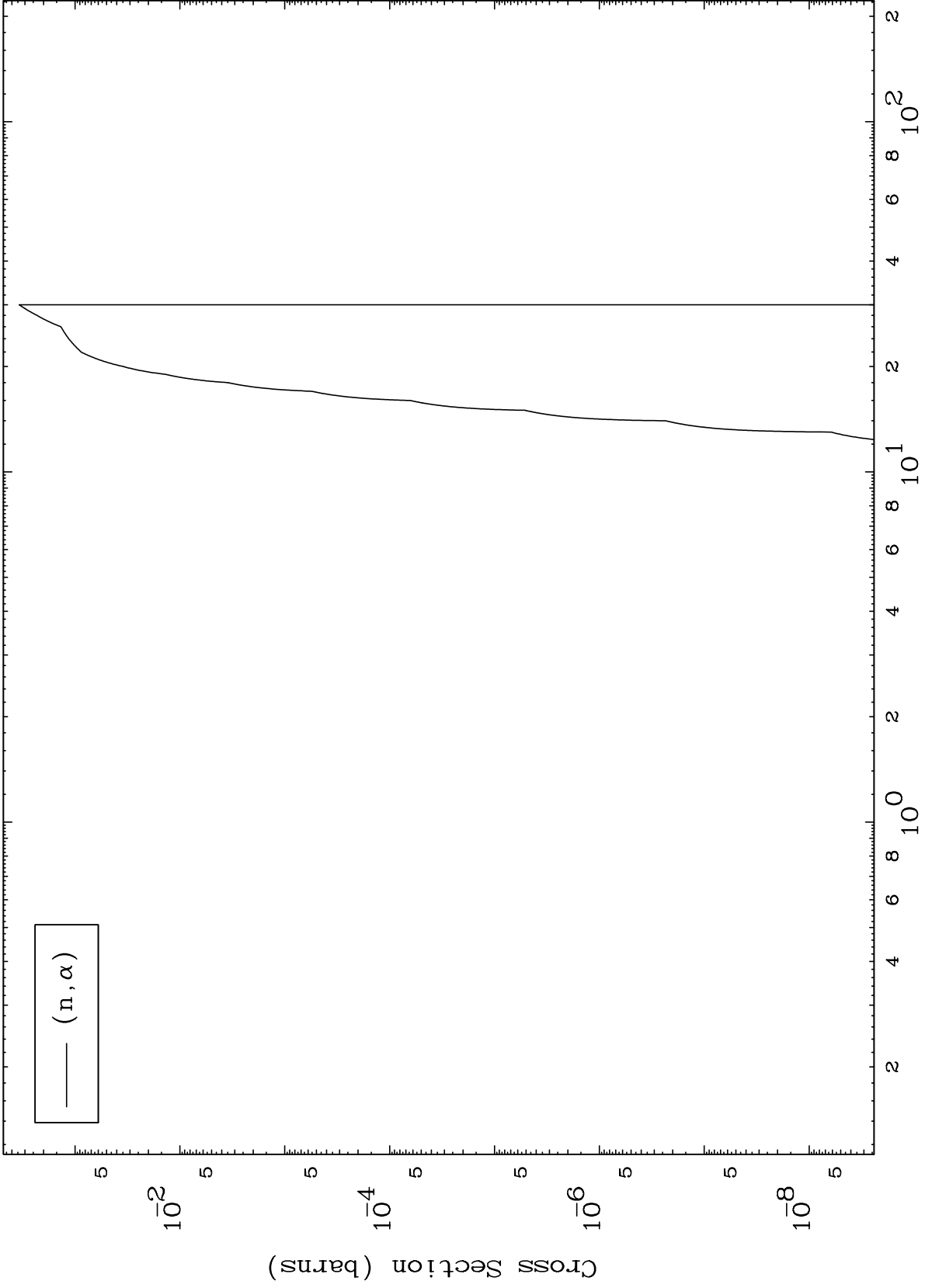
Incident Energy (MeV)

72-Hf-183

0 Kelvin Cross Sections



0 Kelvin Cross Sections

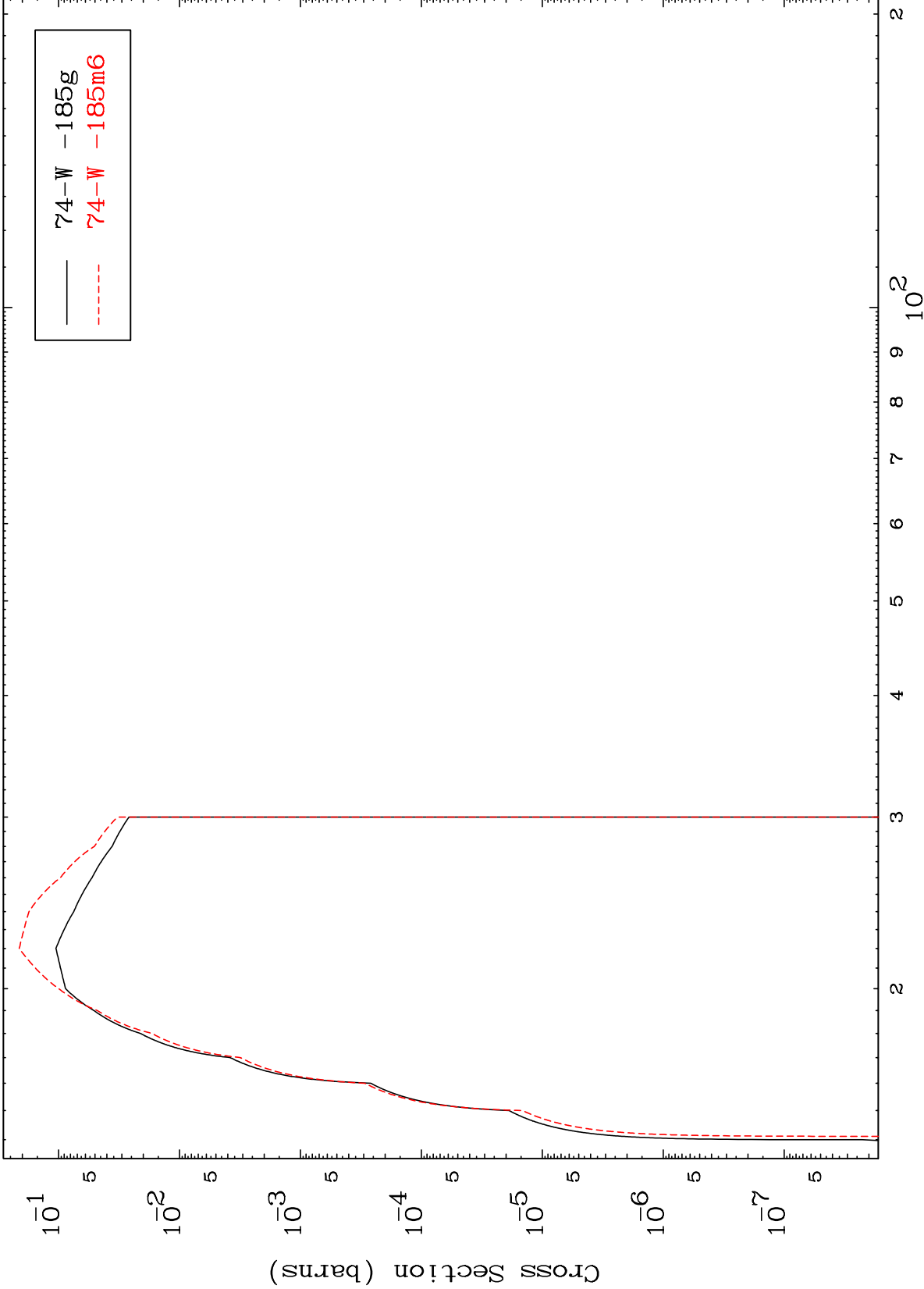


MAT 7252

(n,2n)

72-Hf-183

Radionuclide Production Cross Section



74-W - 185g
74-W - 185m6

10

Incident Energy (MeV)

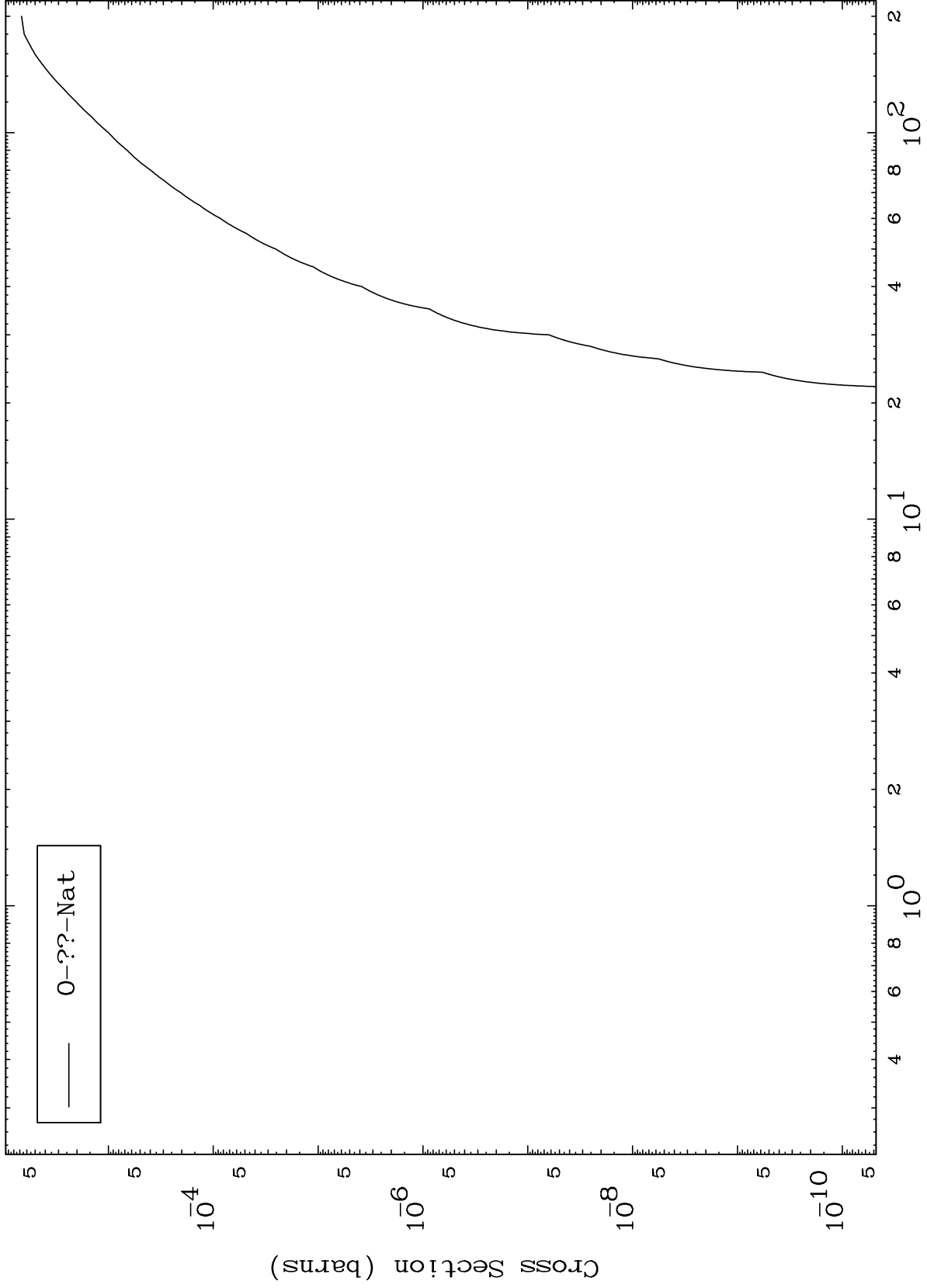
72-Hf-183

MAT 7252

Fission

72-Hf-183

Radionuclide Production Cross Section

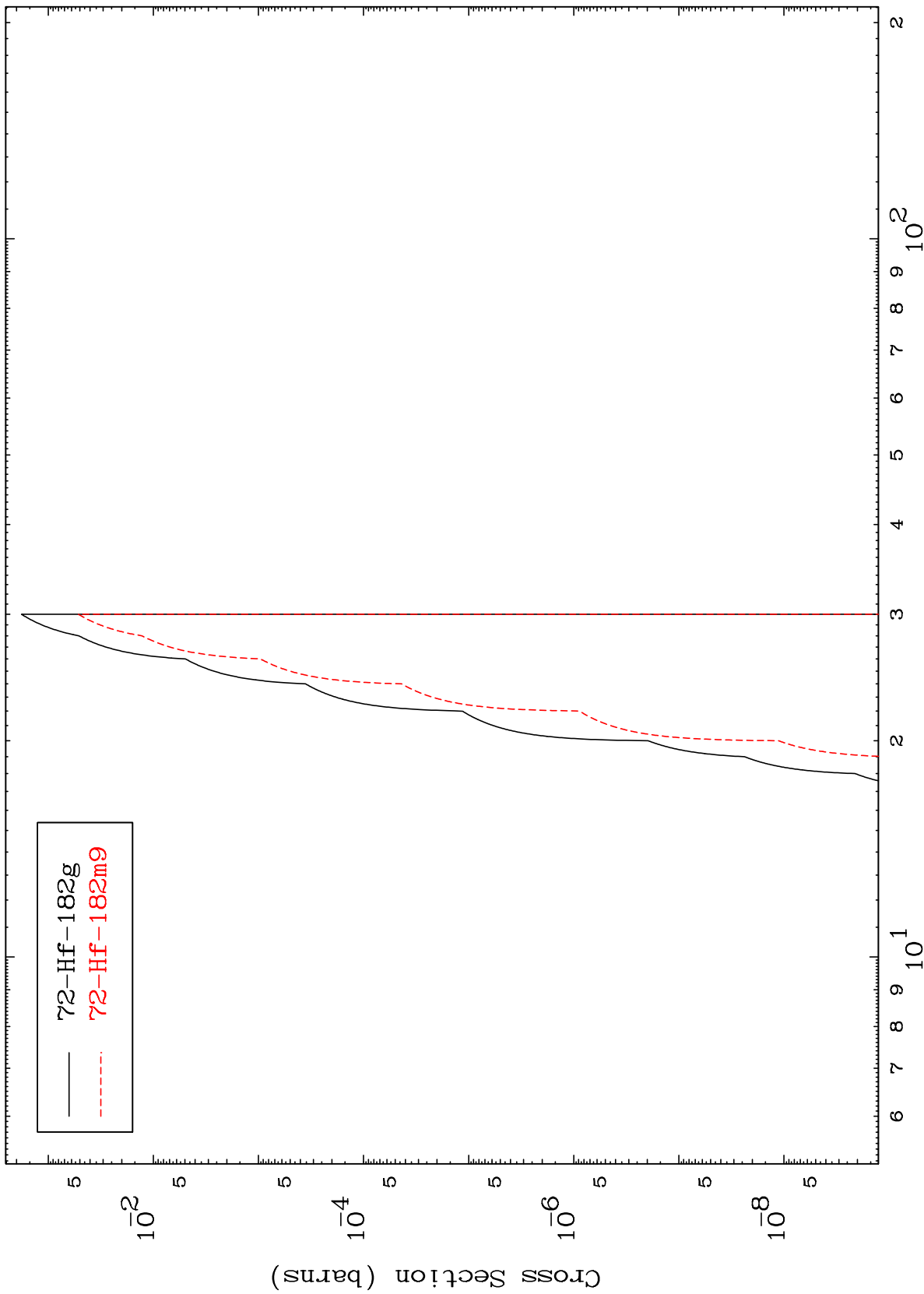


MAT 7252

$(n, n') \alpha$

72-Hf-183

Radionuclide Production Cross Section



72-Hf-182g
72-Hf-182m9

12

Incident Energy (MeV)

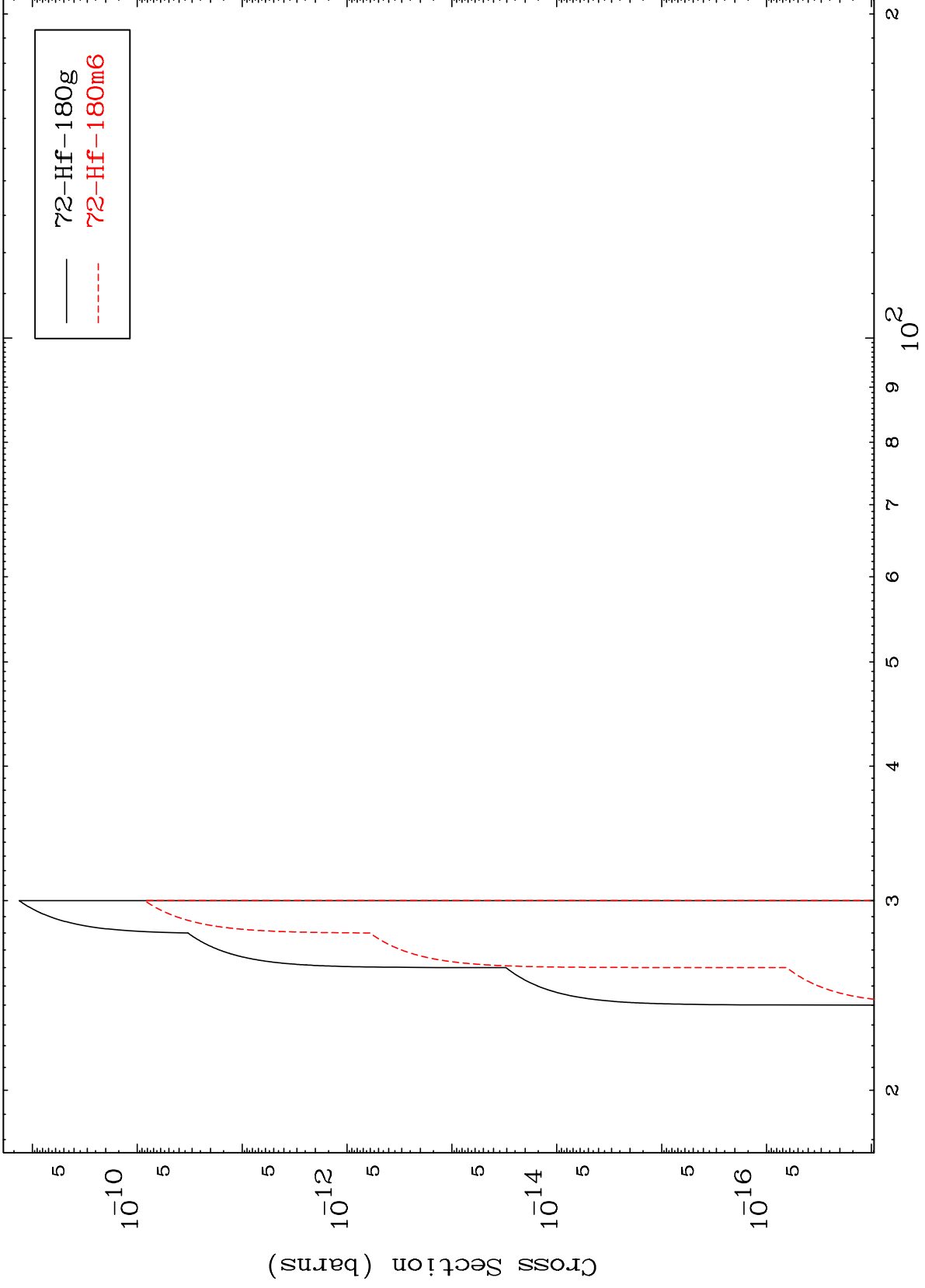
72-Hf-183

MAT 7252

(n,3n) α

72-Hf-183

Radionuclide Production Cross Section



13

Incident Energy (MeV)

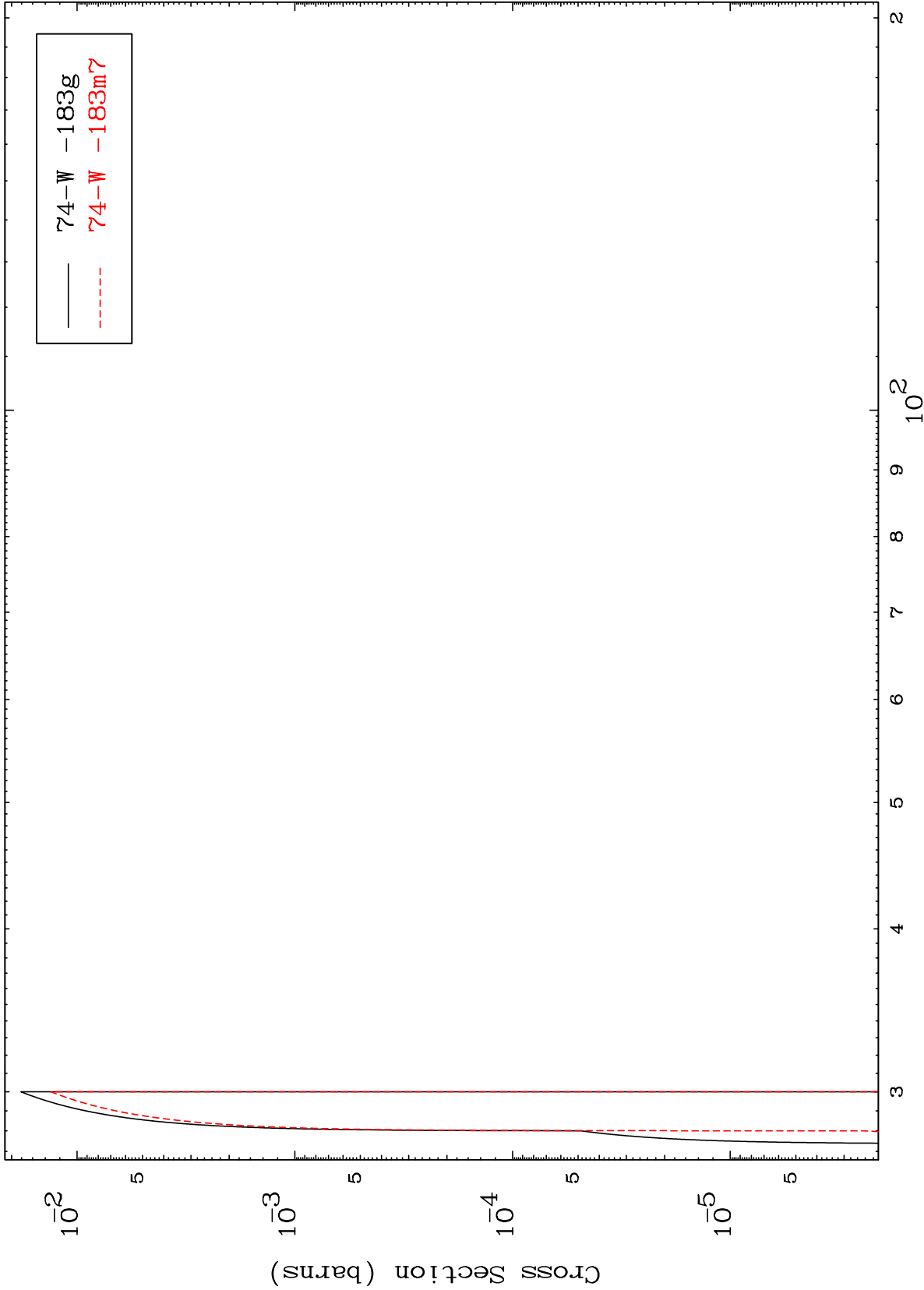
72-Hf-183

MAT 7252

(n,4n)

72-Hf-183

Radionuclide Production Cross Section



14

Incident Energy (MeV)

72-Hf-183

Radionuclide Production Cross Section

