

Program EVALPLOT
(Version 2021-1)

by

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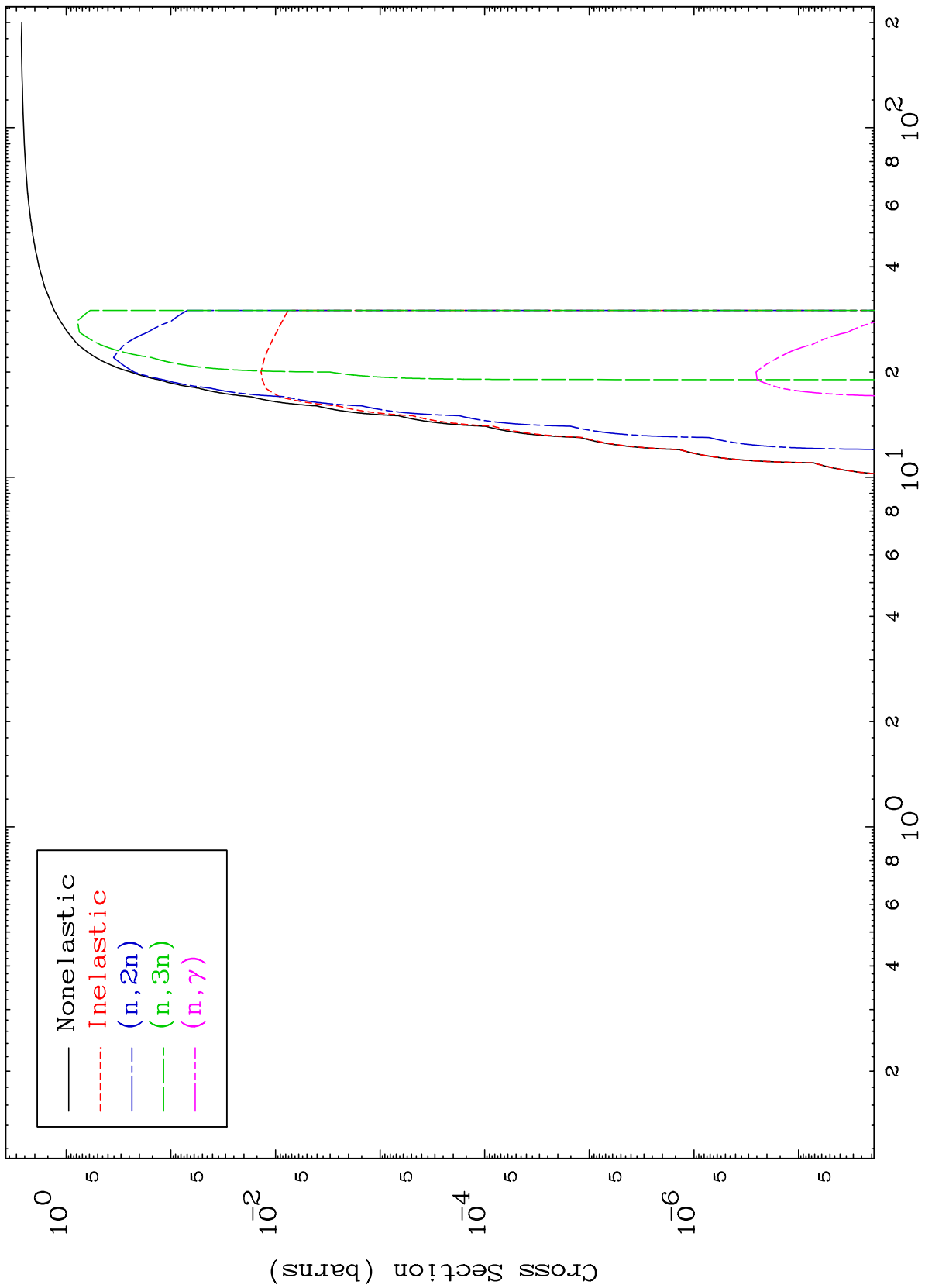
Press Mouse Button to Start

MAT 7256

0 Kelvin

α Major Cross Sections

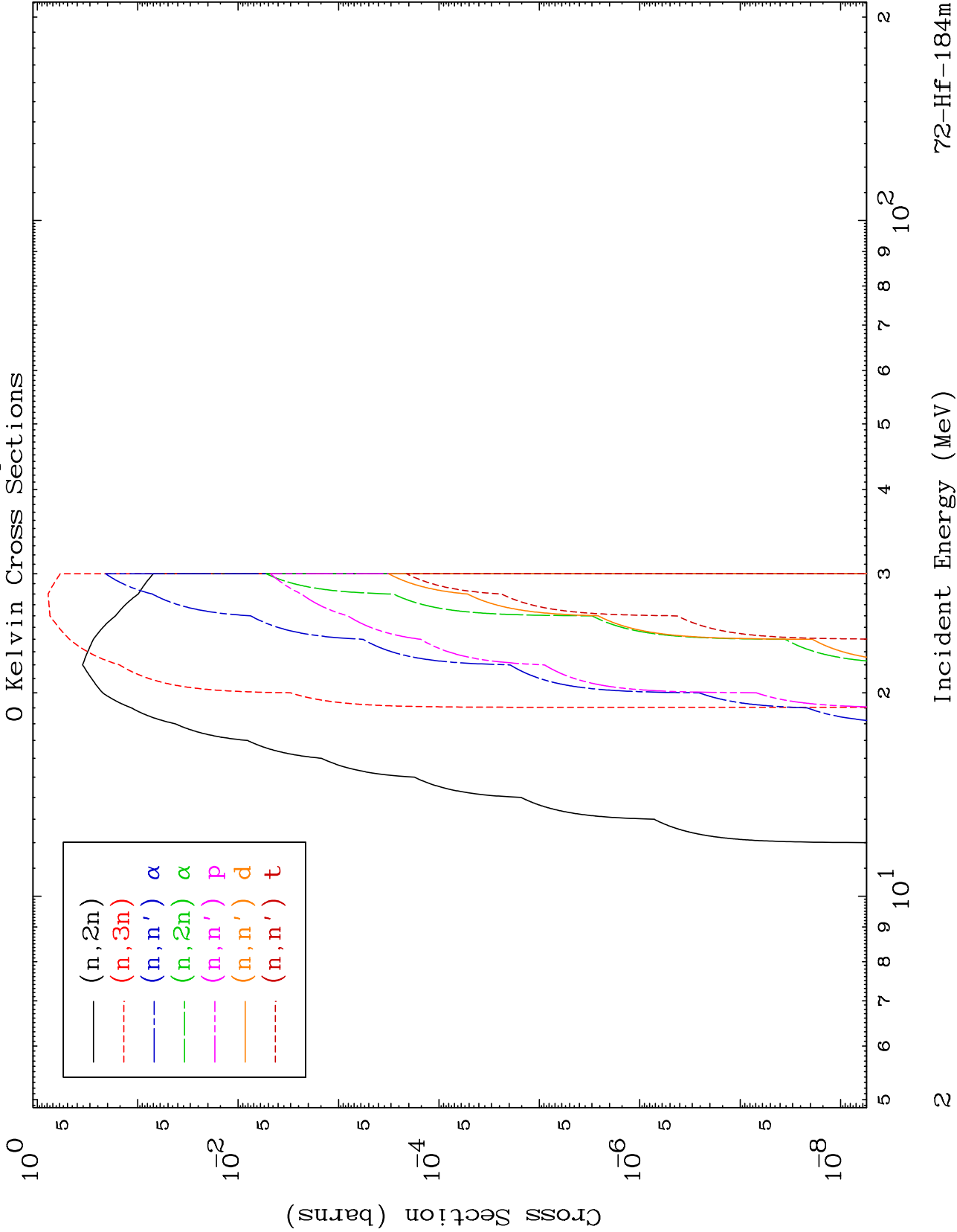
⁷²Hf-184m



MAT 7256

α Neutron Absorption
0 Kelvin Cross Sections

⁷²Hf-184m



⁷²Hf-184m

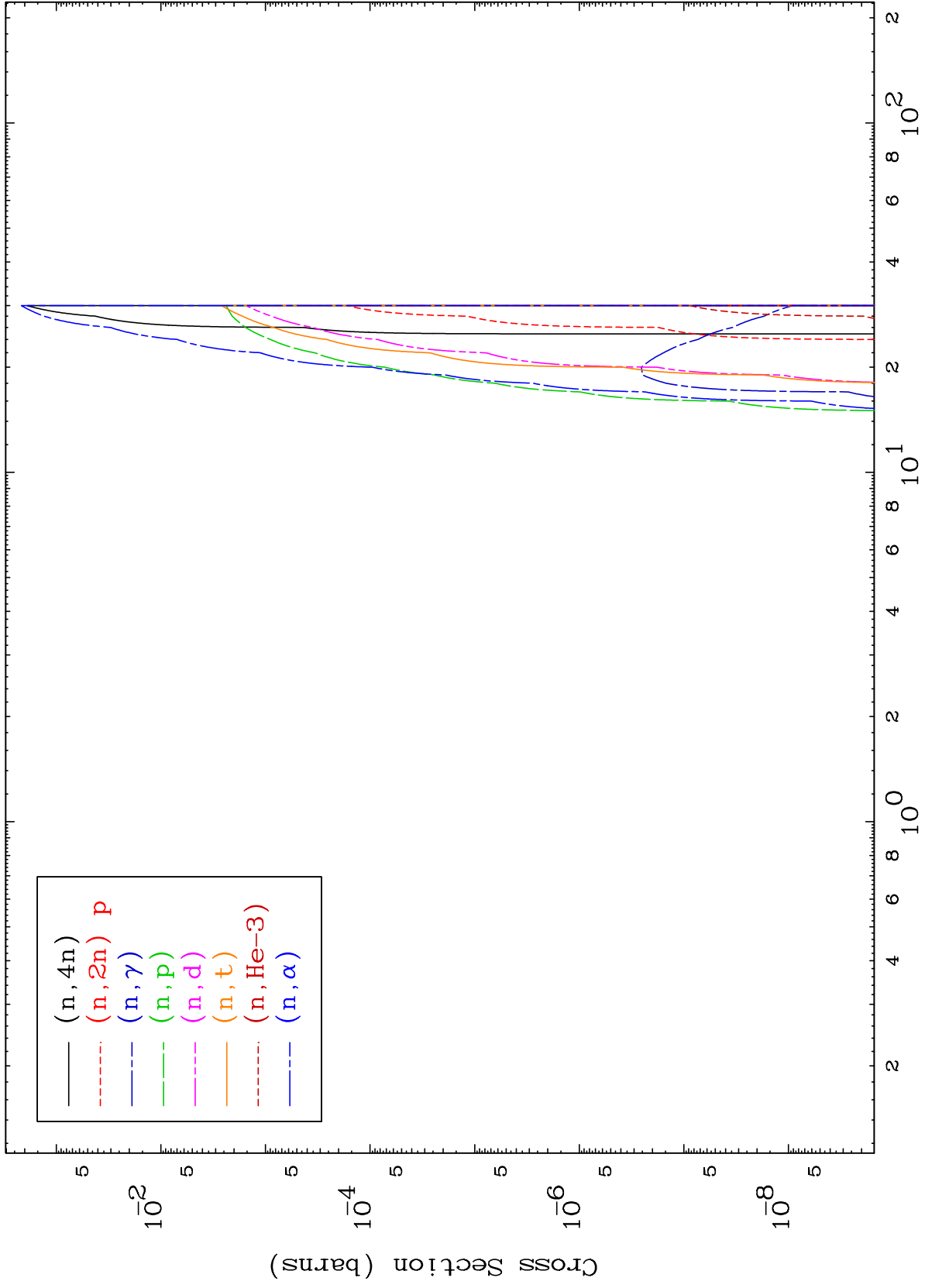
Incident Energy (MeV)

2

MAT 7256

α Neutron Absorption
0 Kelvin Cross Sections

⁷²Hf-184m



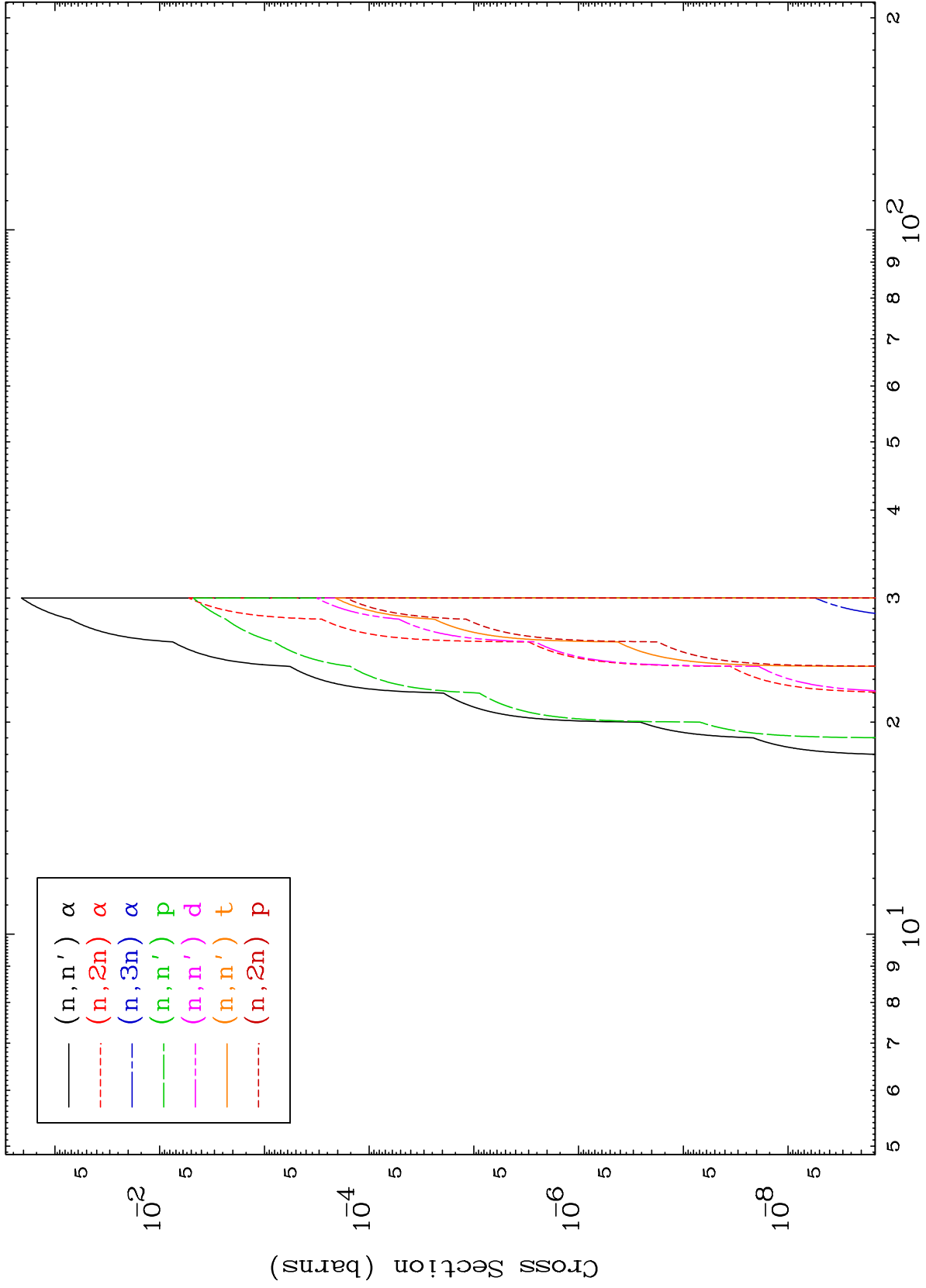
Incident Energy (MeV)

⁷²Hf-184m

MAT 7256

α Charged Particle
0 Kelvin Cross Sections

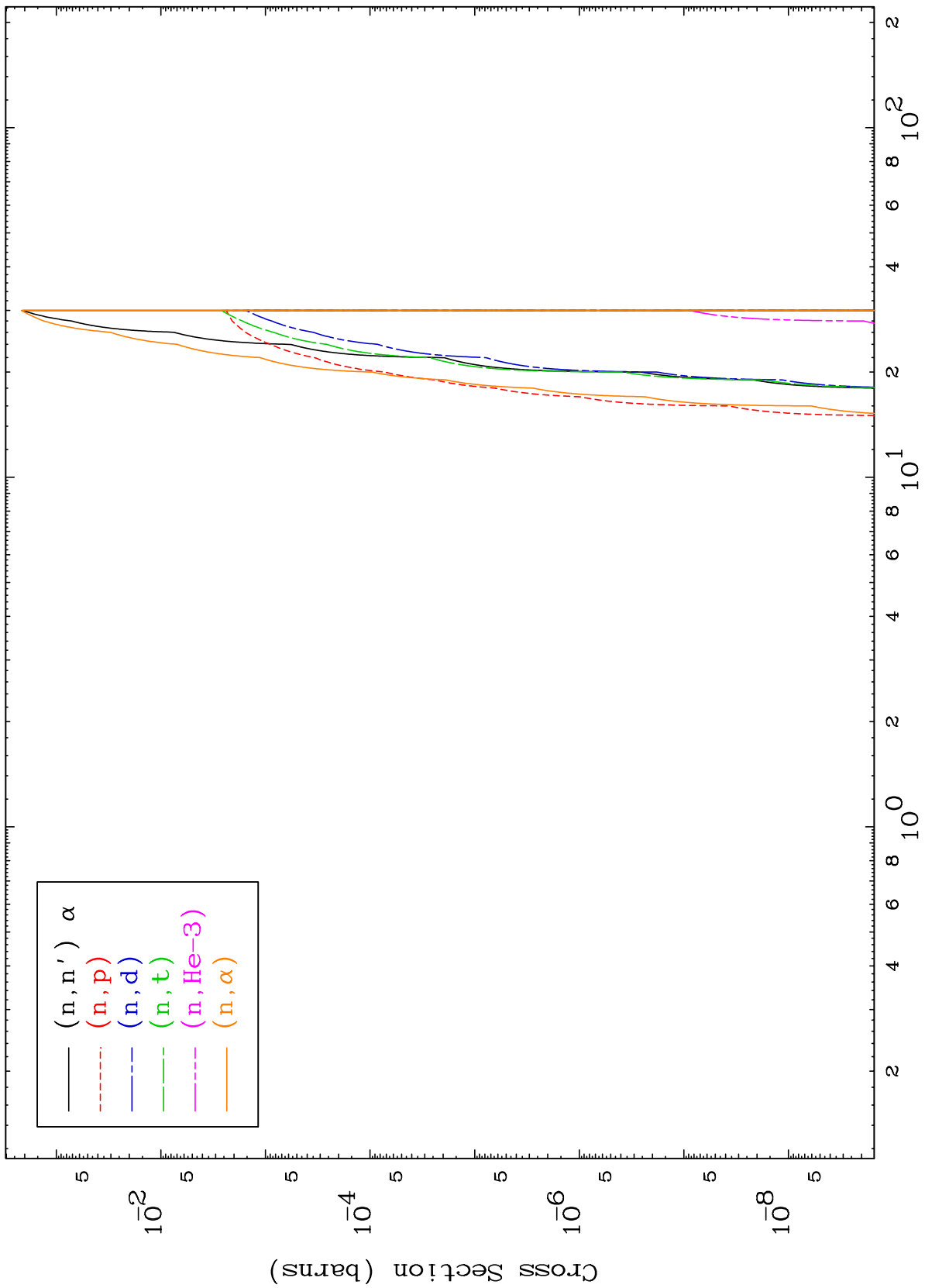
72-Hf-184m



4

Incident Energy (MeV)

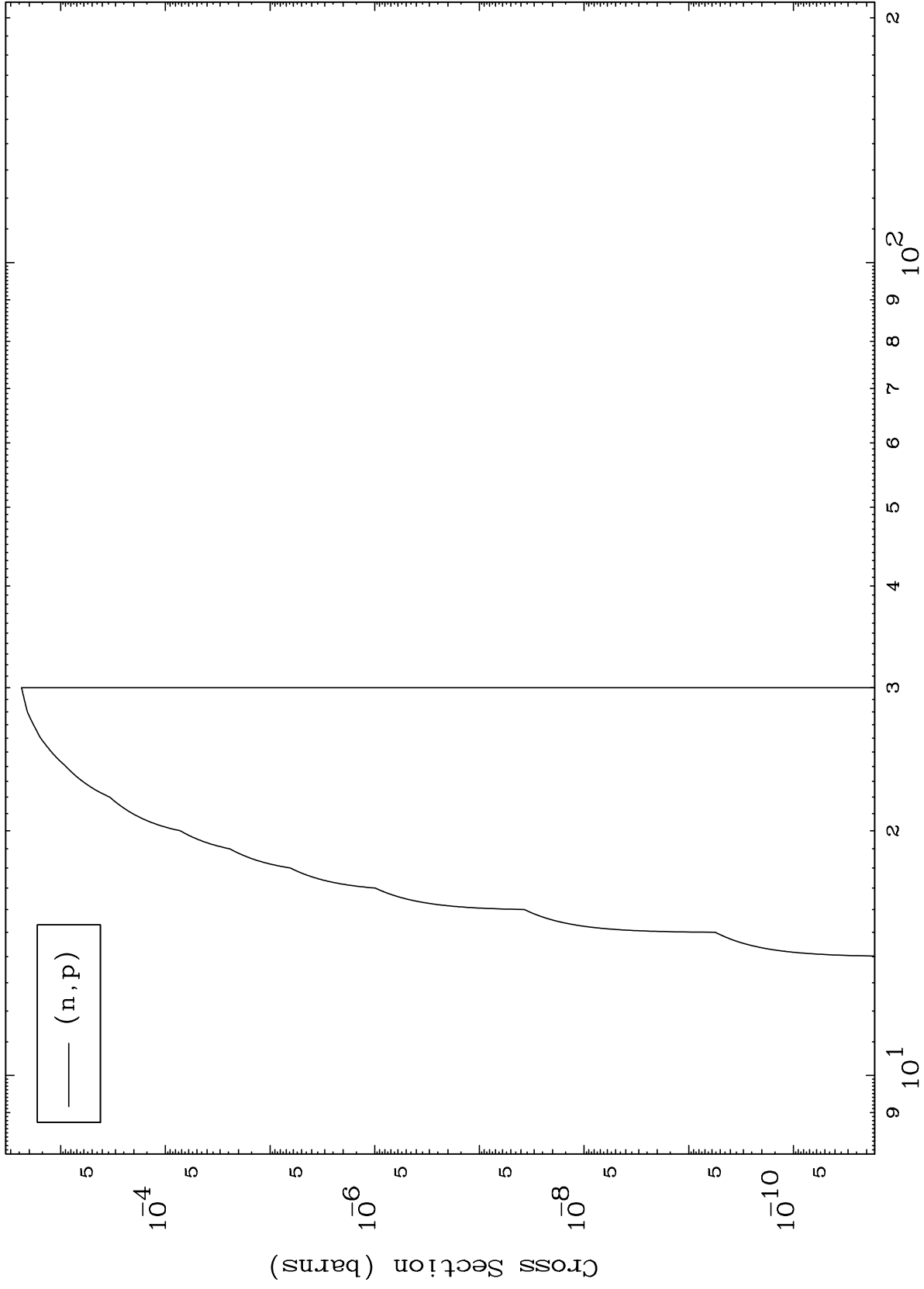
72-Hf-184m



MAT 7256

(α, p) Levels
0 Kelvin Cross Sections

$^{72}\text{Hf}-184\text{m}$



6

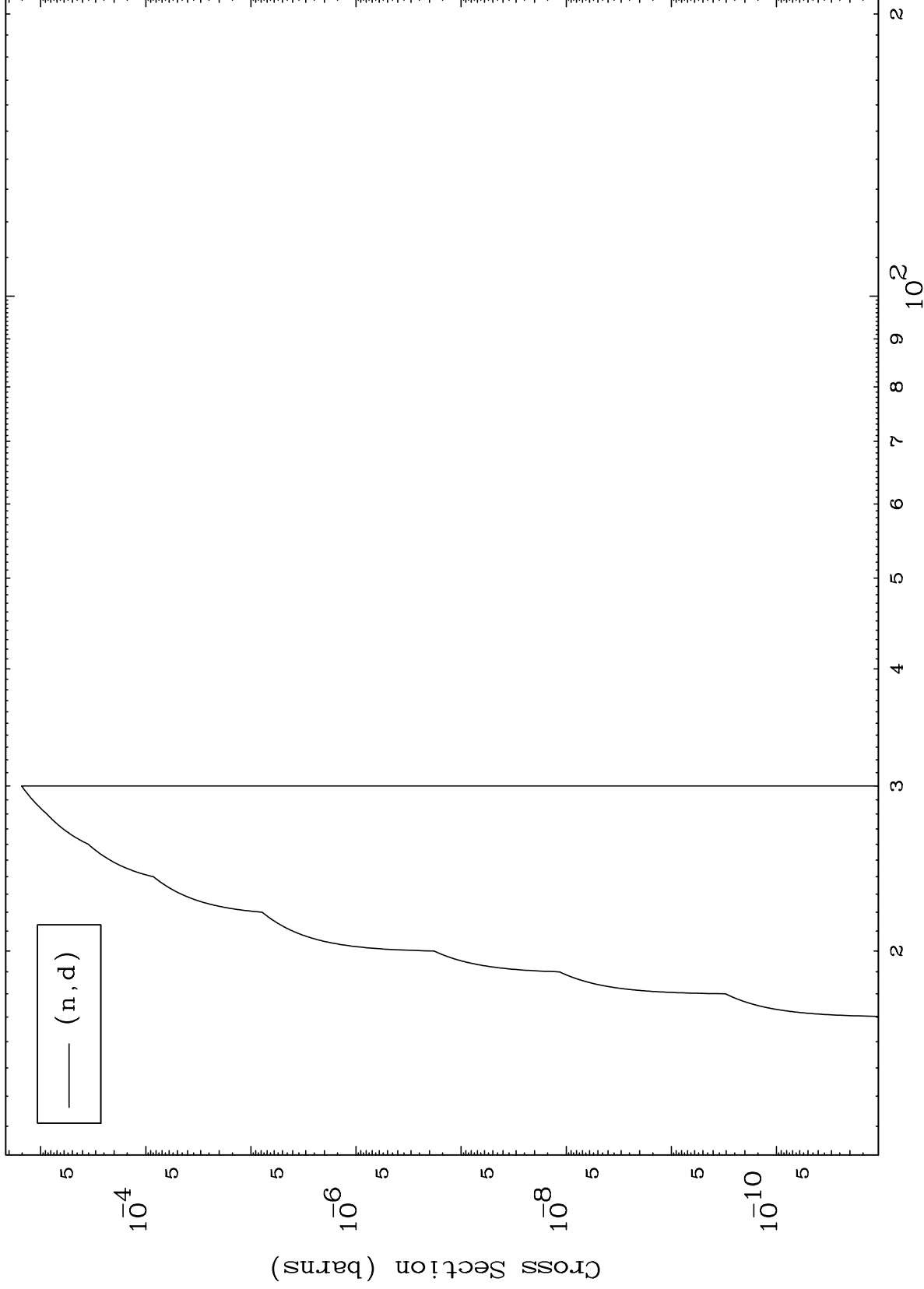
Incident Energy (MeV)

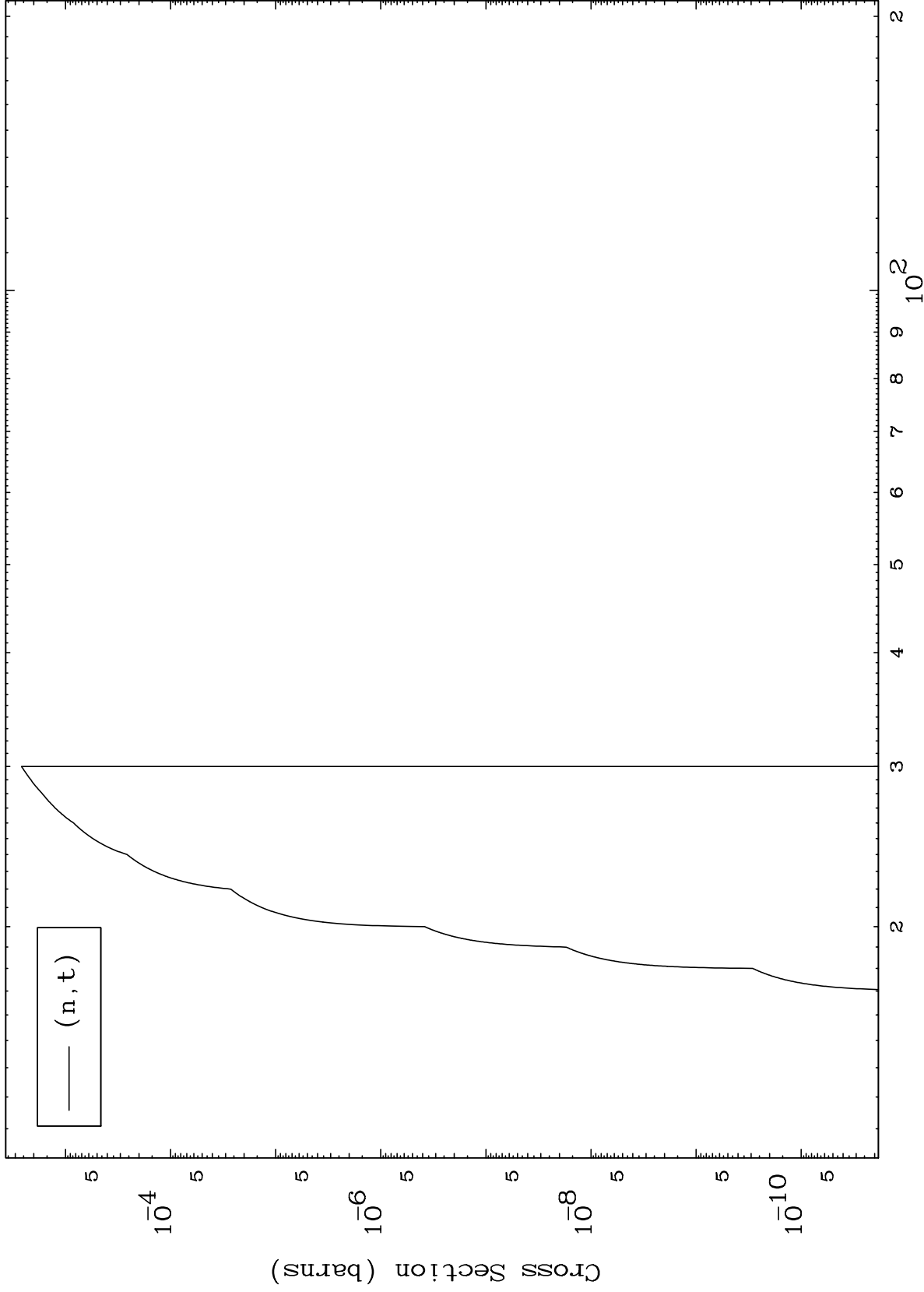
$^{72}\text{Hf}-184\text{m}$

MAT 7256

(α, d) Levels
0 Kelvin Cross Sections

⁷²Hf-184m

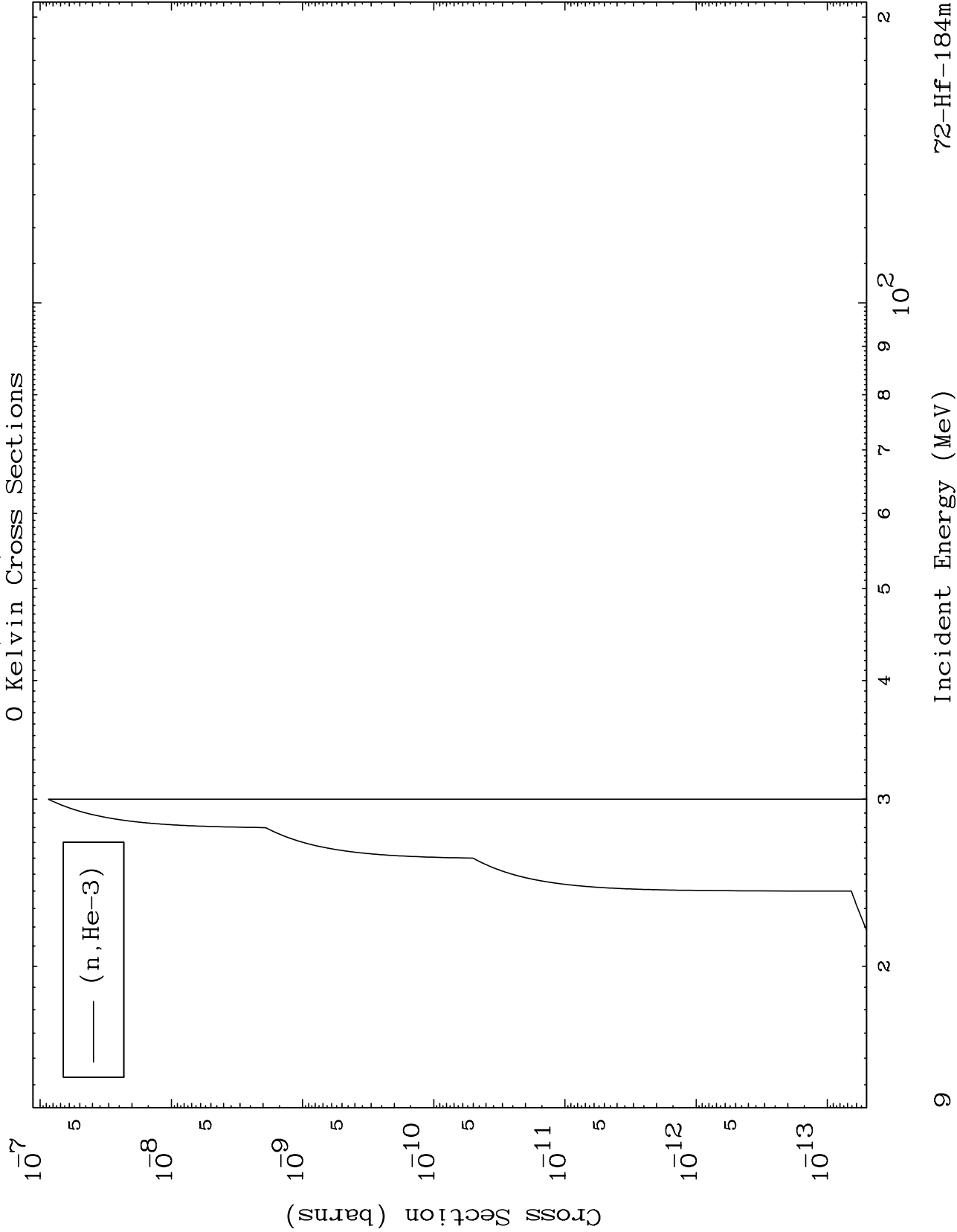




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($\alpha, \text{He3}$) Levels

$^{72}\text{Hf}-184\text{m}$

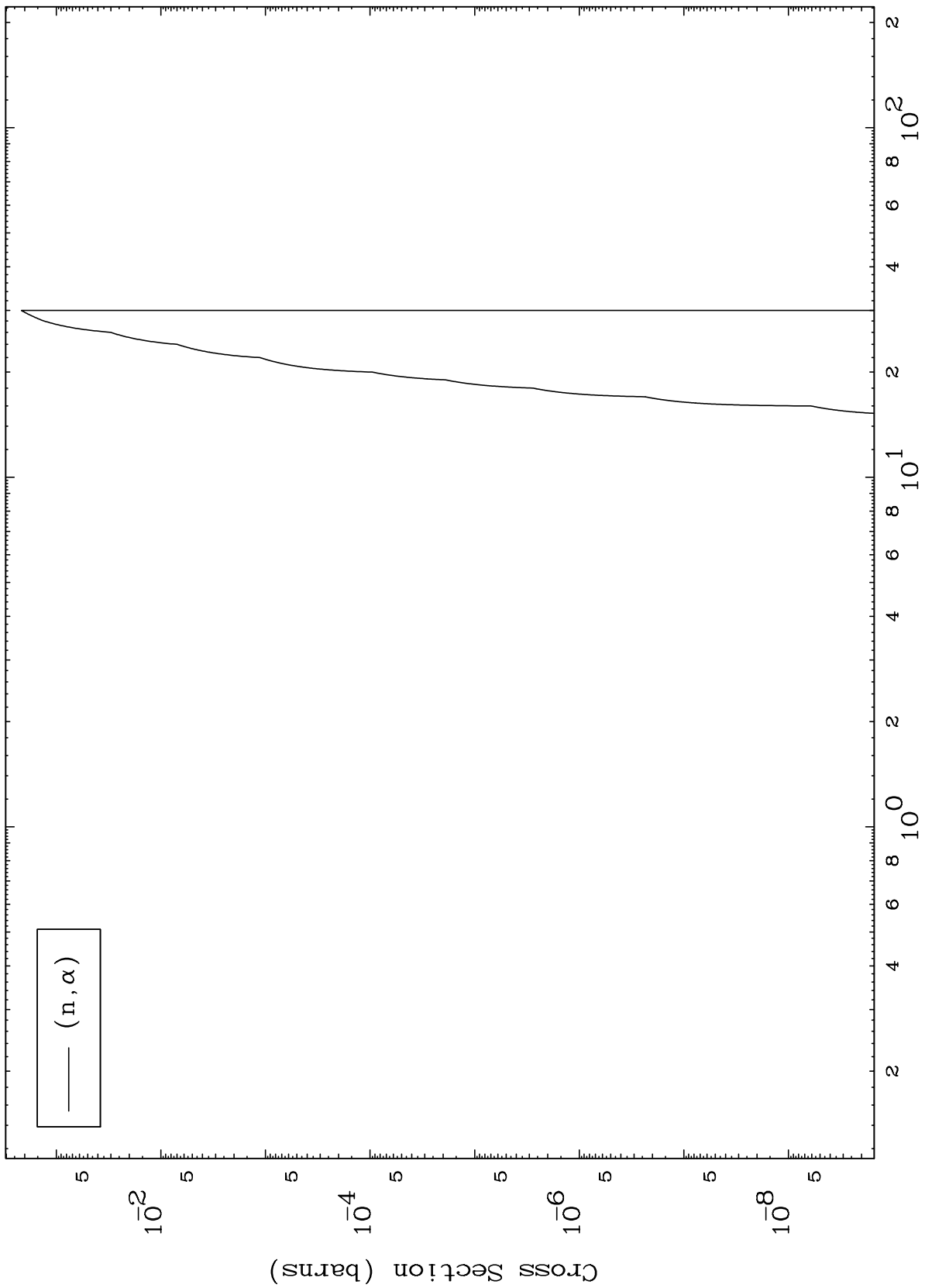


MAT 7256

(α, α) Levels

72-Hf-184m

0 Kelvin Cross Sections



10

Incident Energy (MeV)

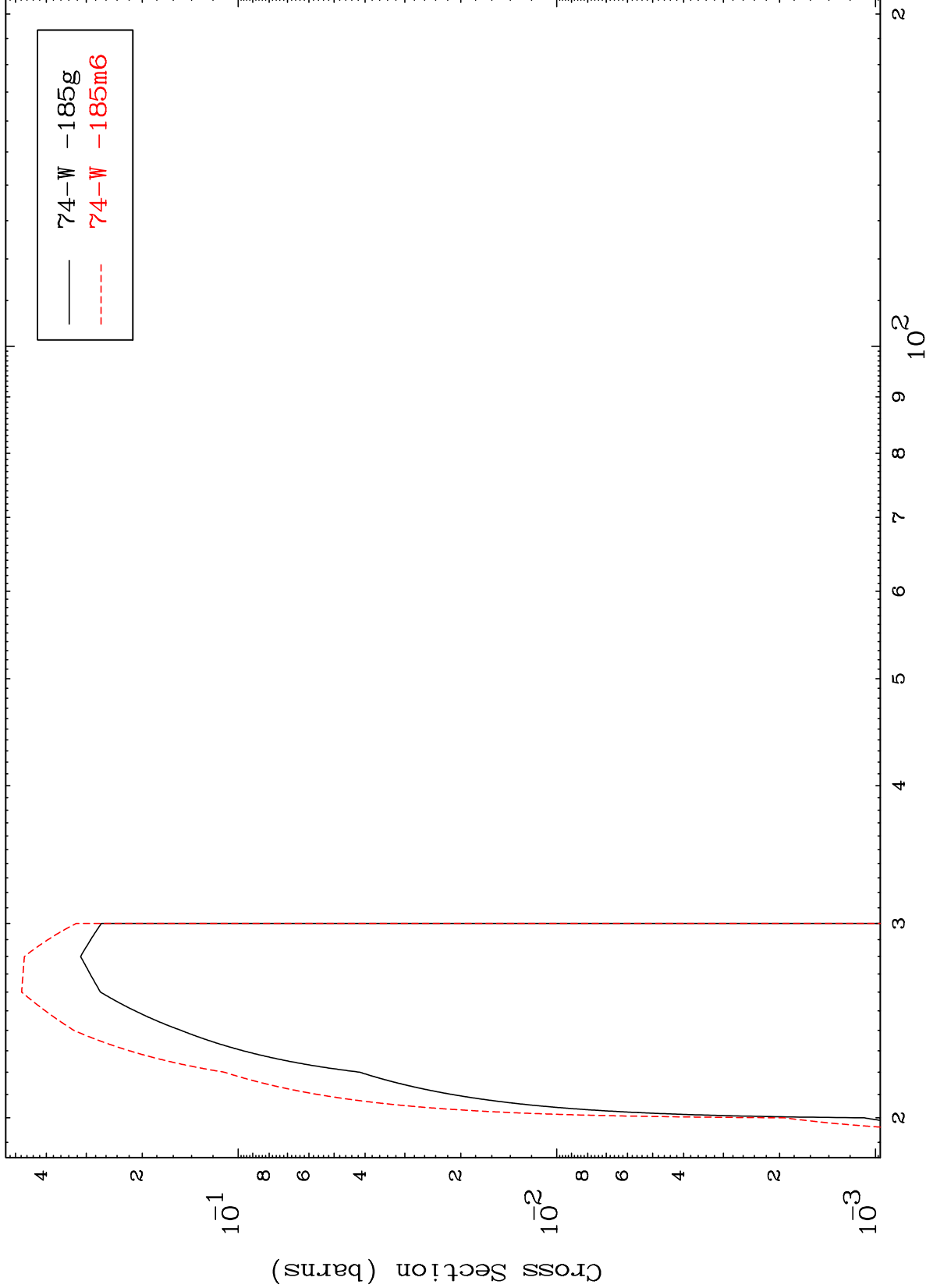
72-Hf-184m

MAT 7256

(n,3n)

⁷²Hf-184m

Radionuclide Production Cross Section



11

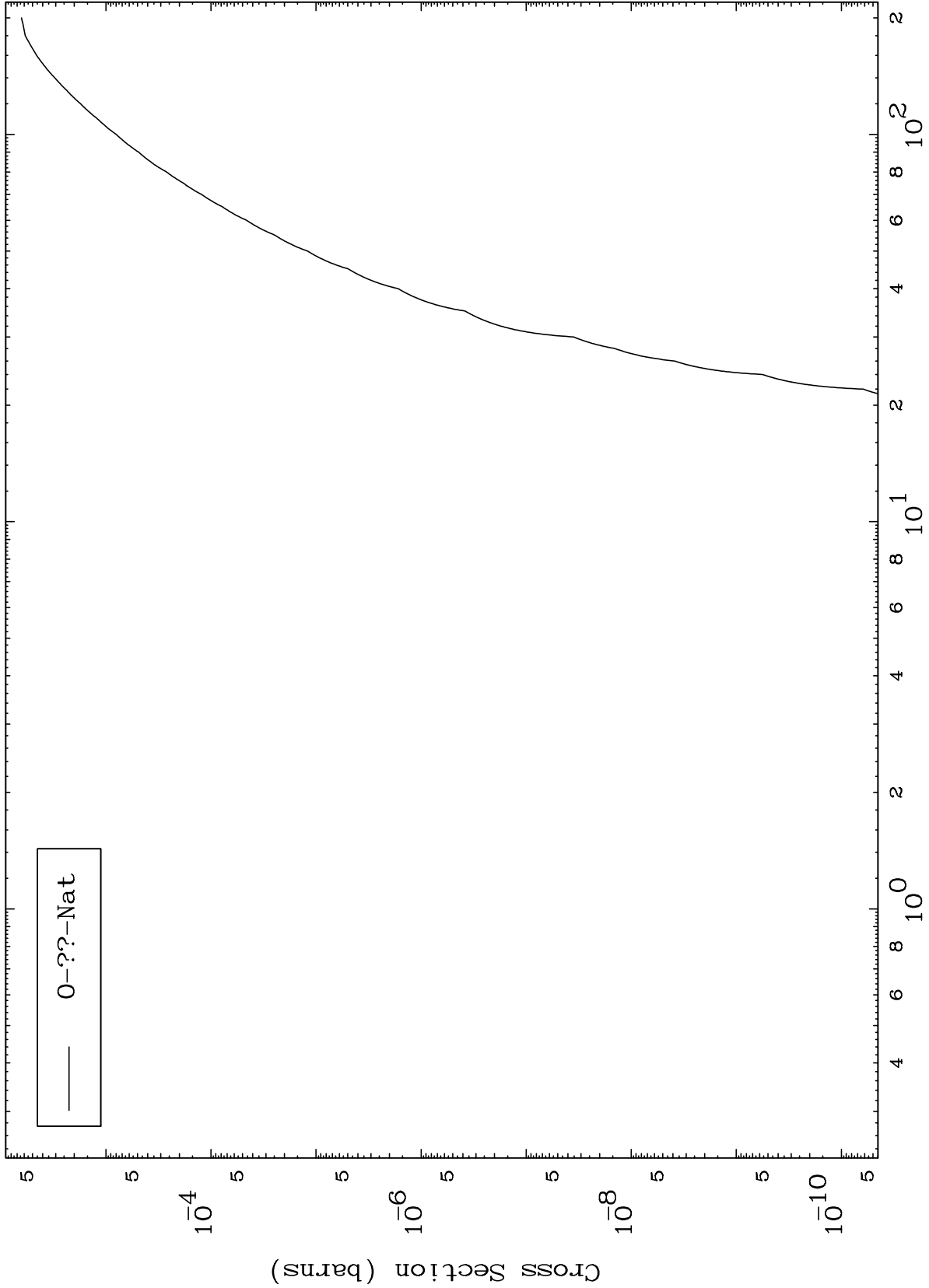
Incident Energy (MeV)

⁷²Hf-184m

MAT 7256

Fission
Radionuclide Production Cross Section

⁷²Hf-184m



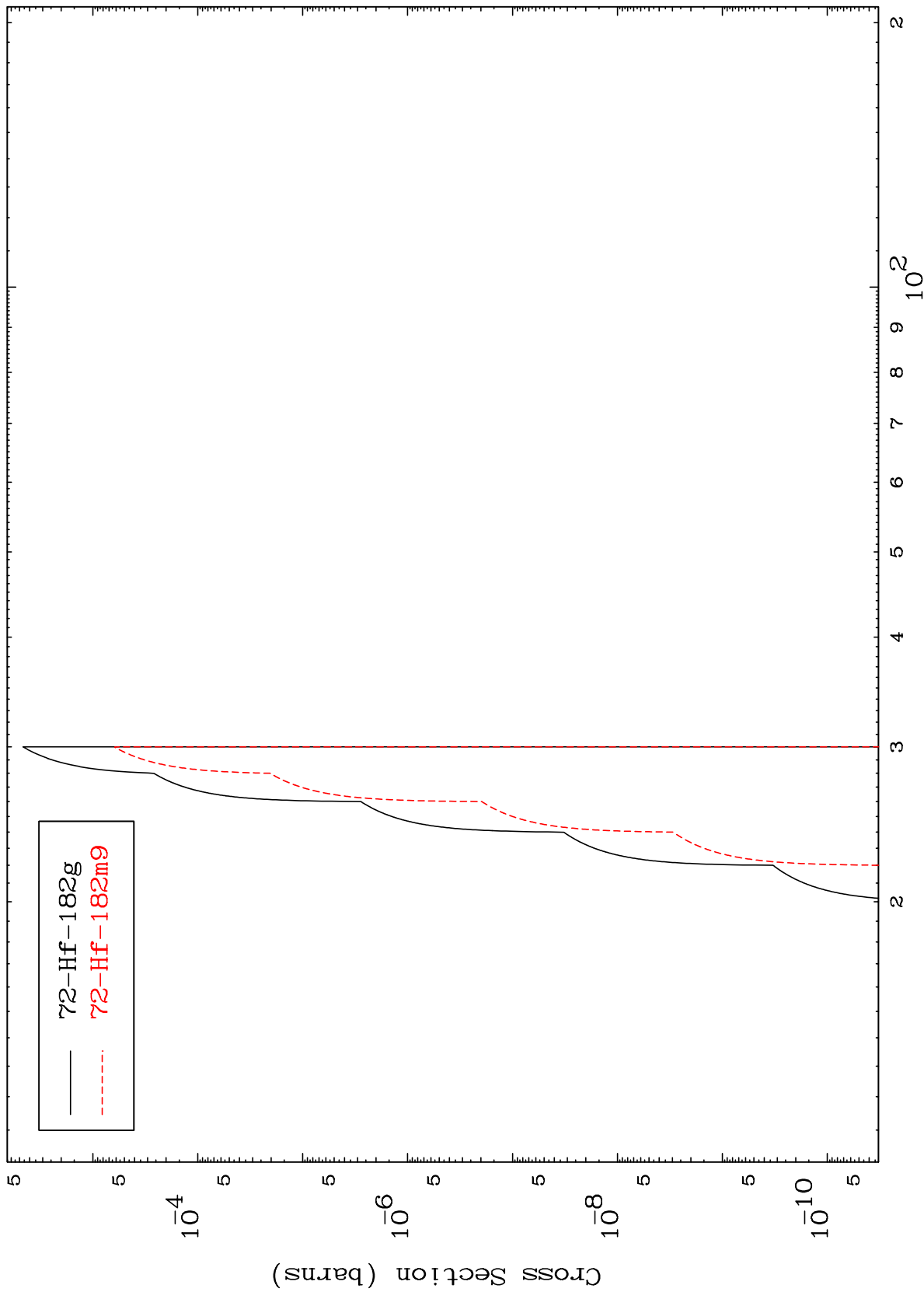
⁷²Hf-184m

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(n,2n) α

⁷²Hf-184m

Radionuclide Production Cross Section



MAT 7256

(n, α)

⁷²Hf-184m

Radionuclide Production Cross Section

