

Program EVALPLOT  
(Version 2021-1)

by

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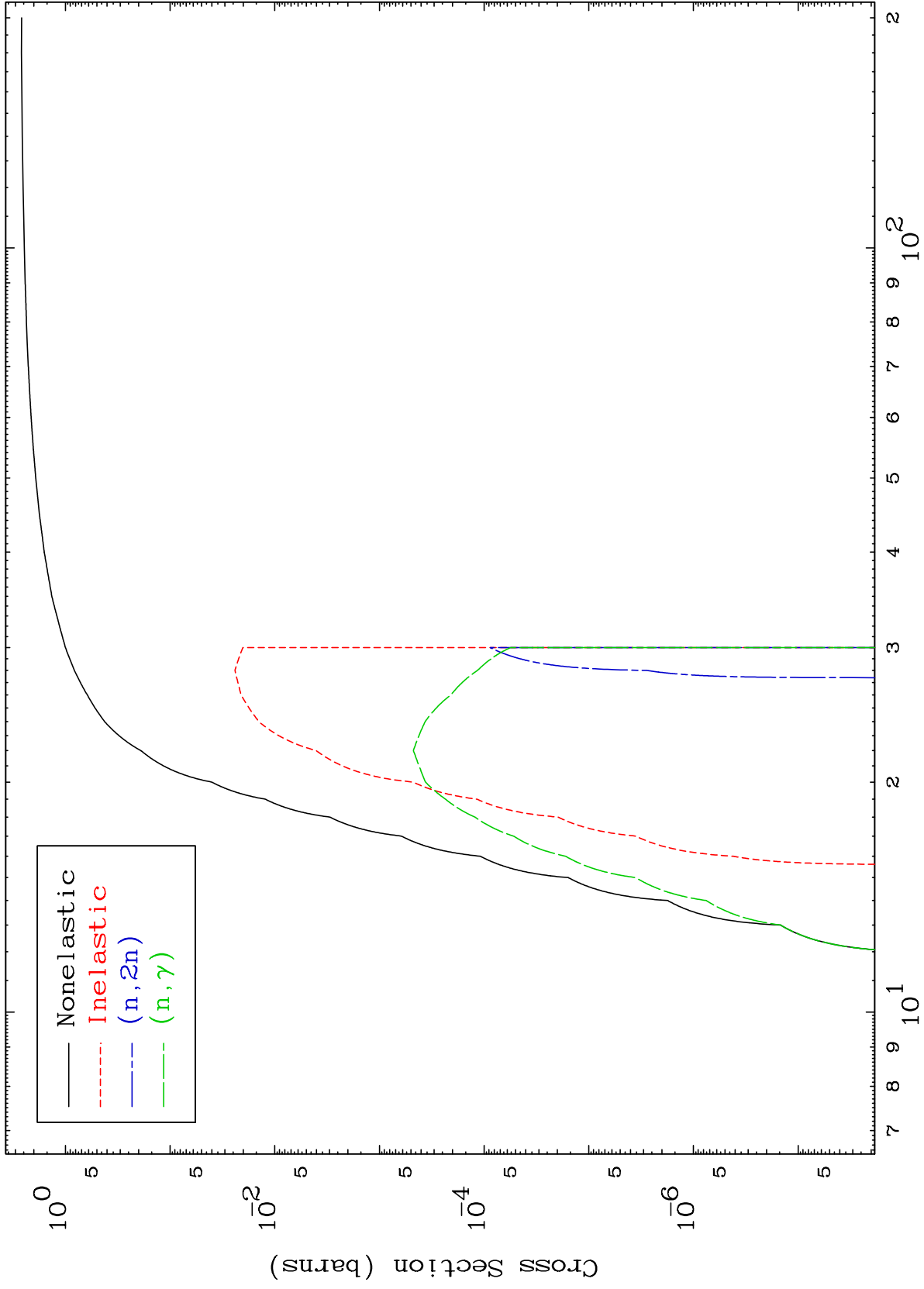
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7980

0 Kelvin Cross Sections

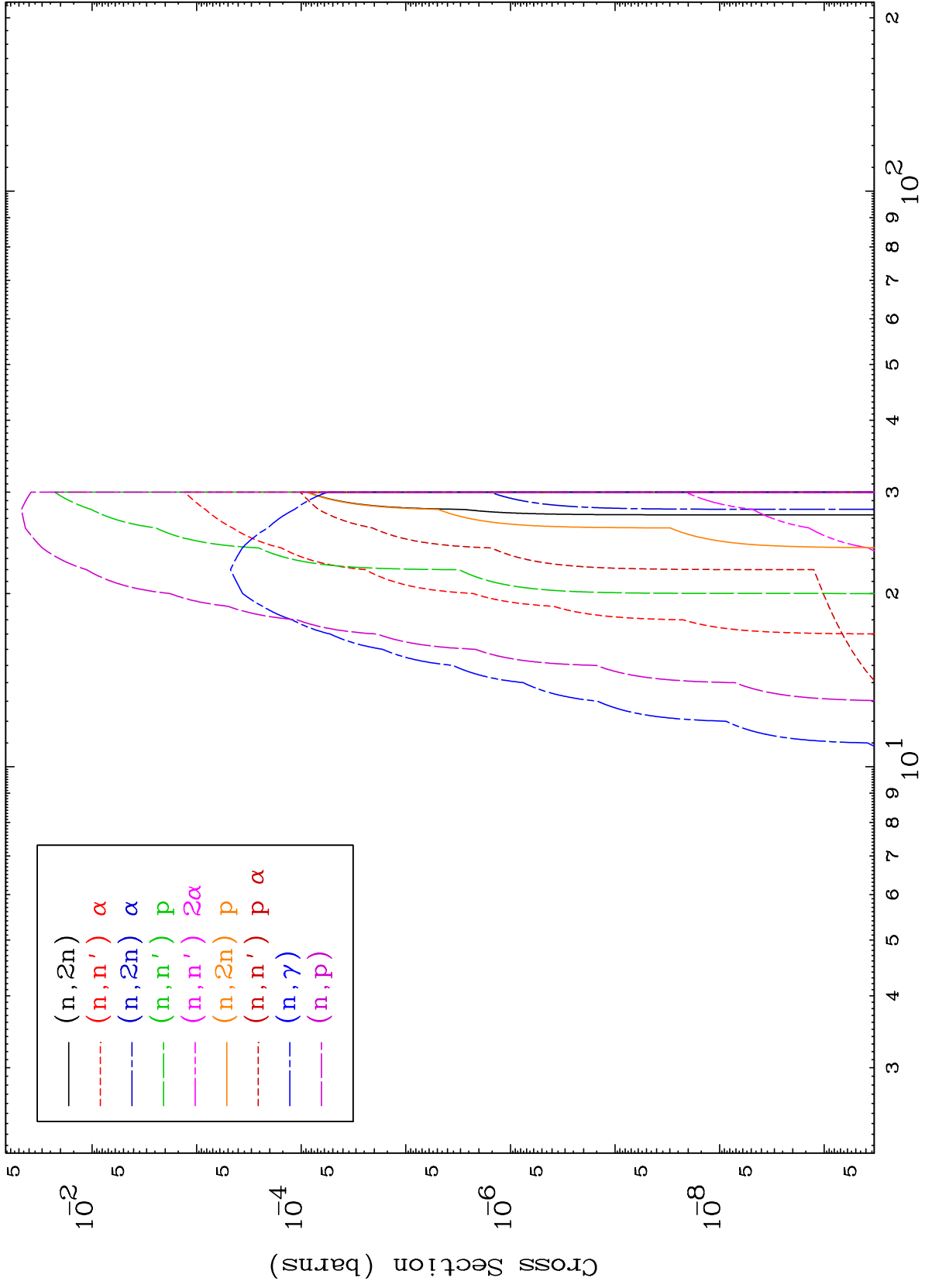
80-Hg-181



1

Incident Energy (MeV)

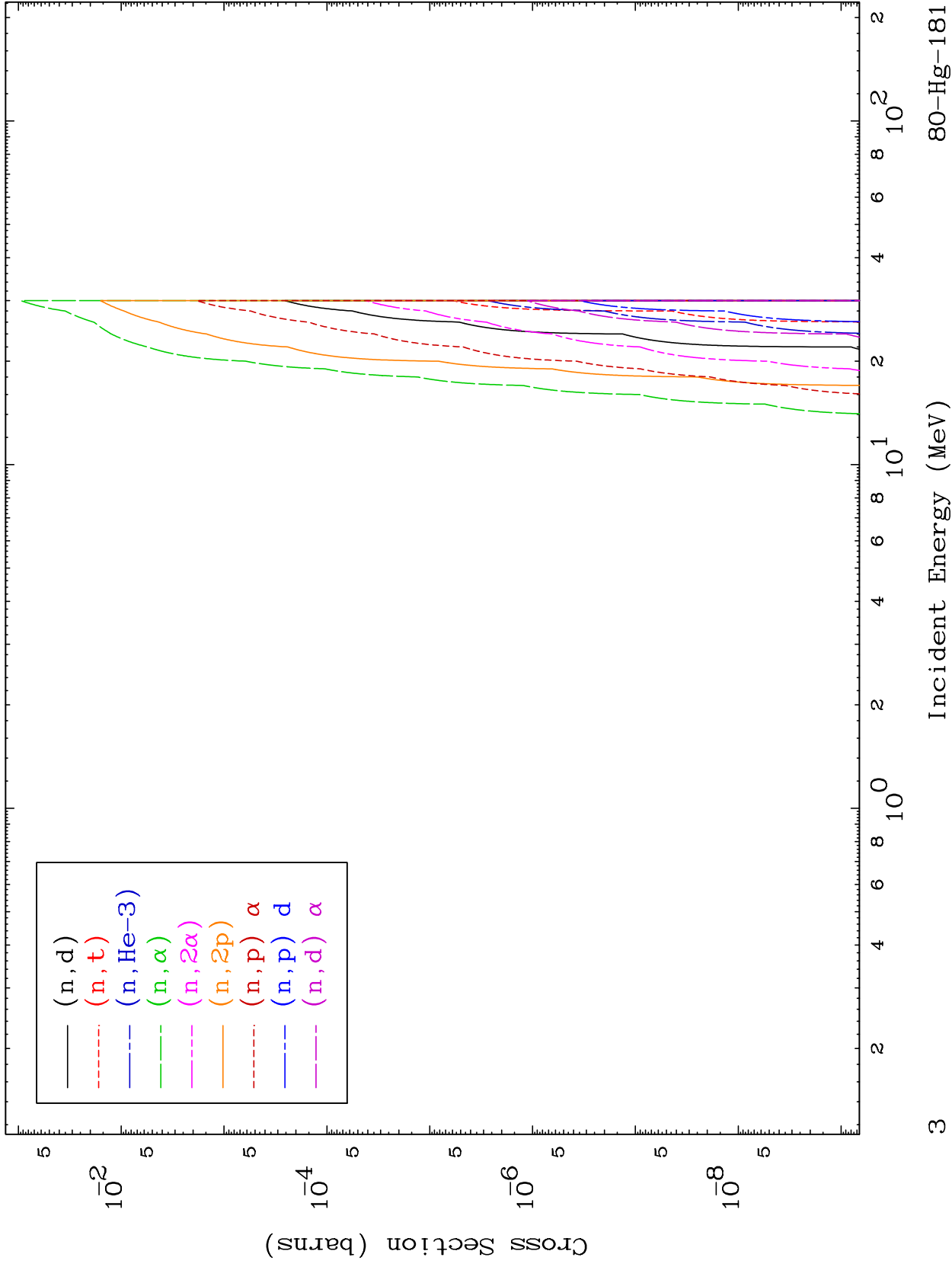
80-Hg-181

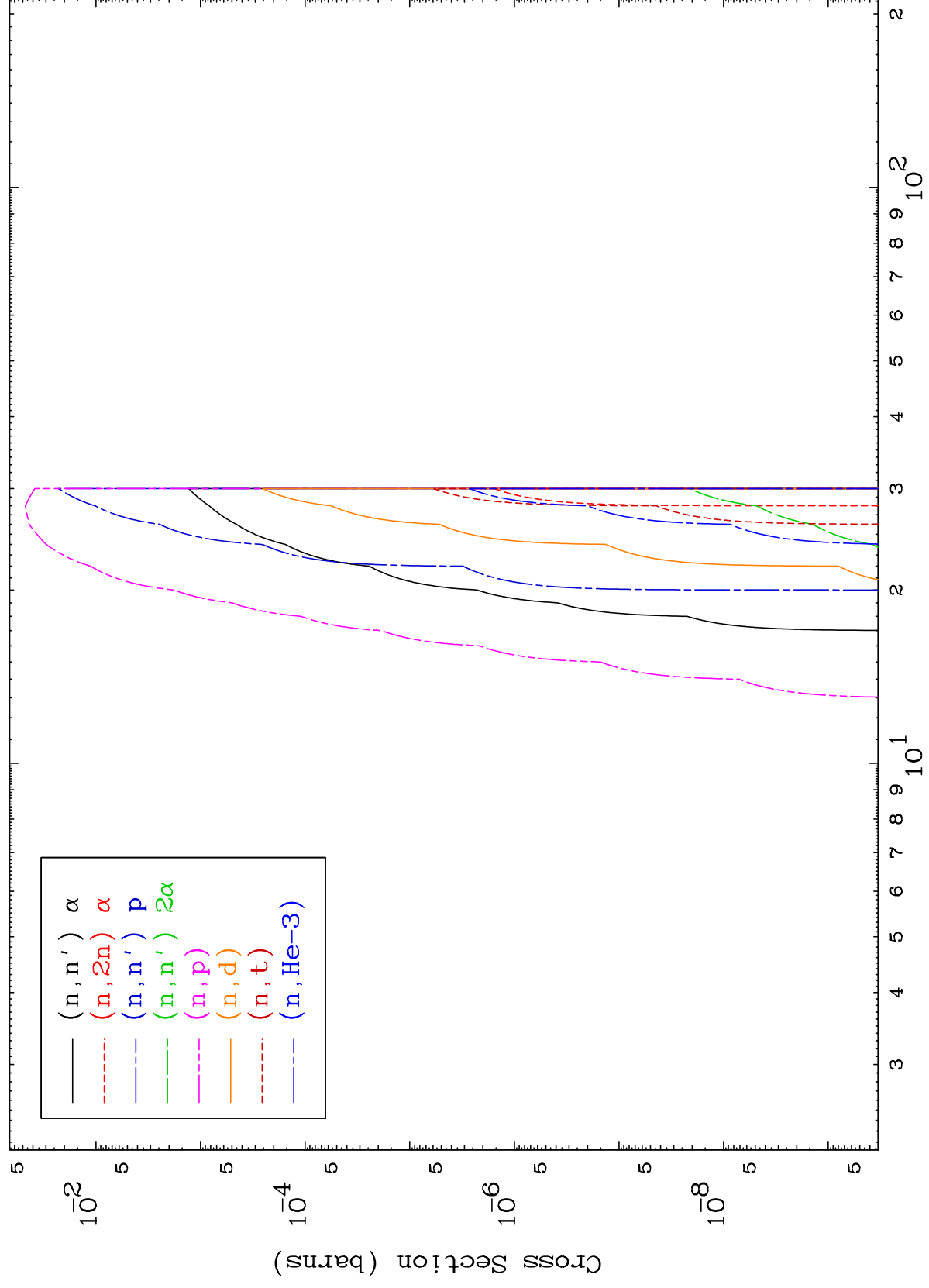


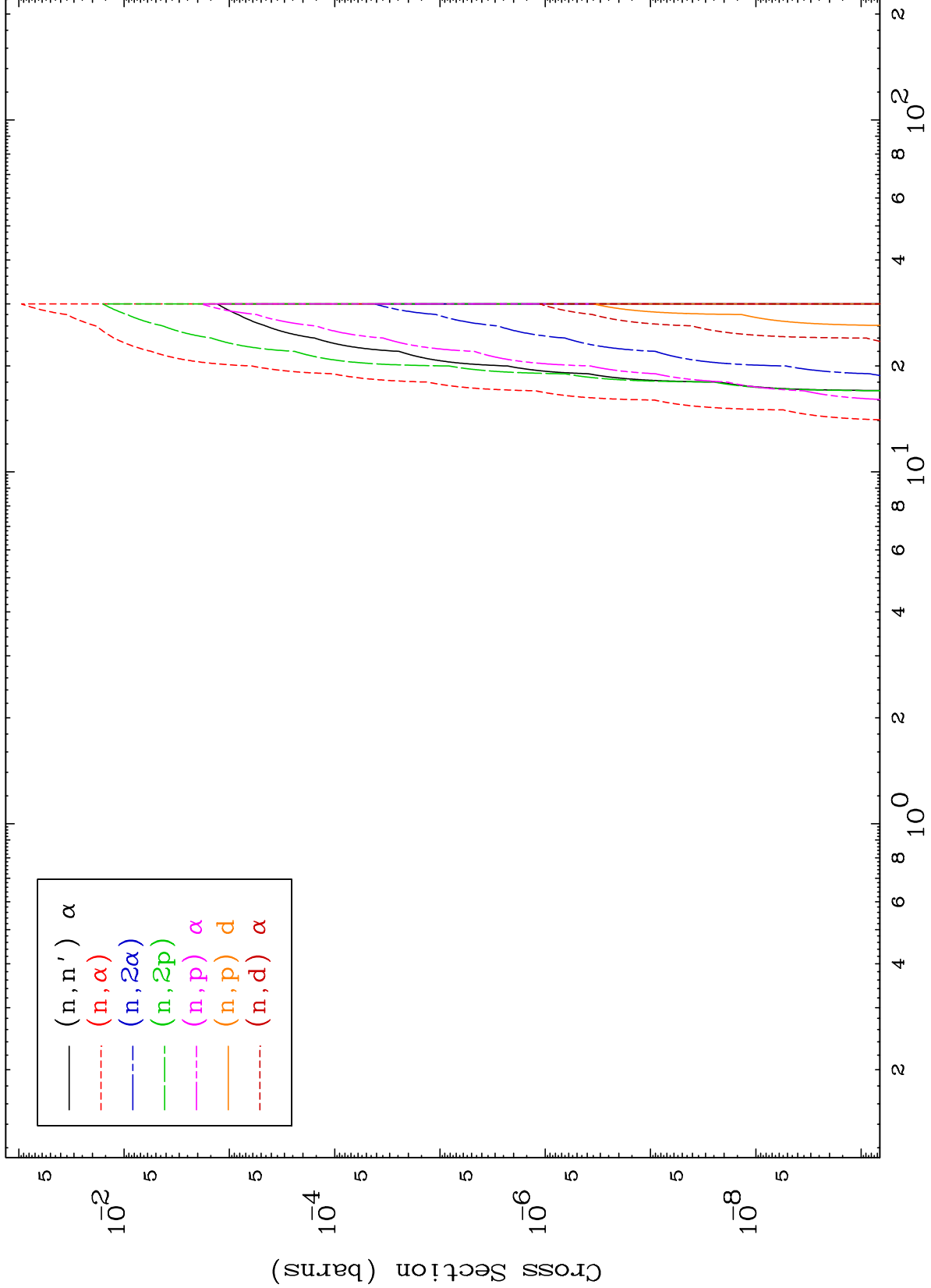
MAT 7980

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

80-Hg-181





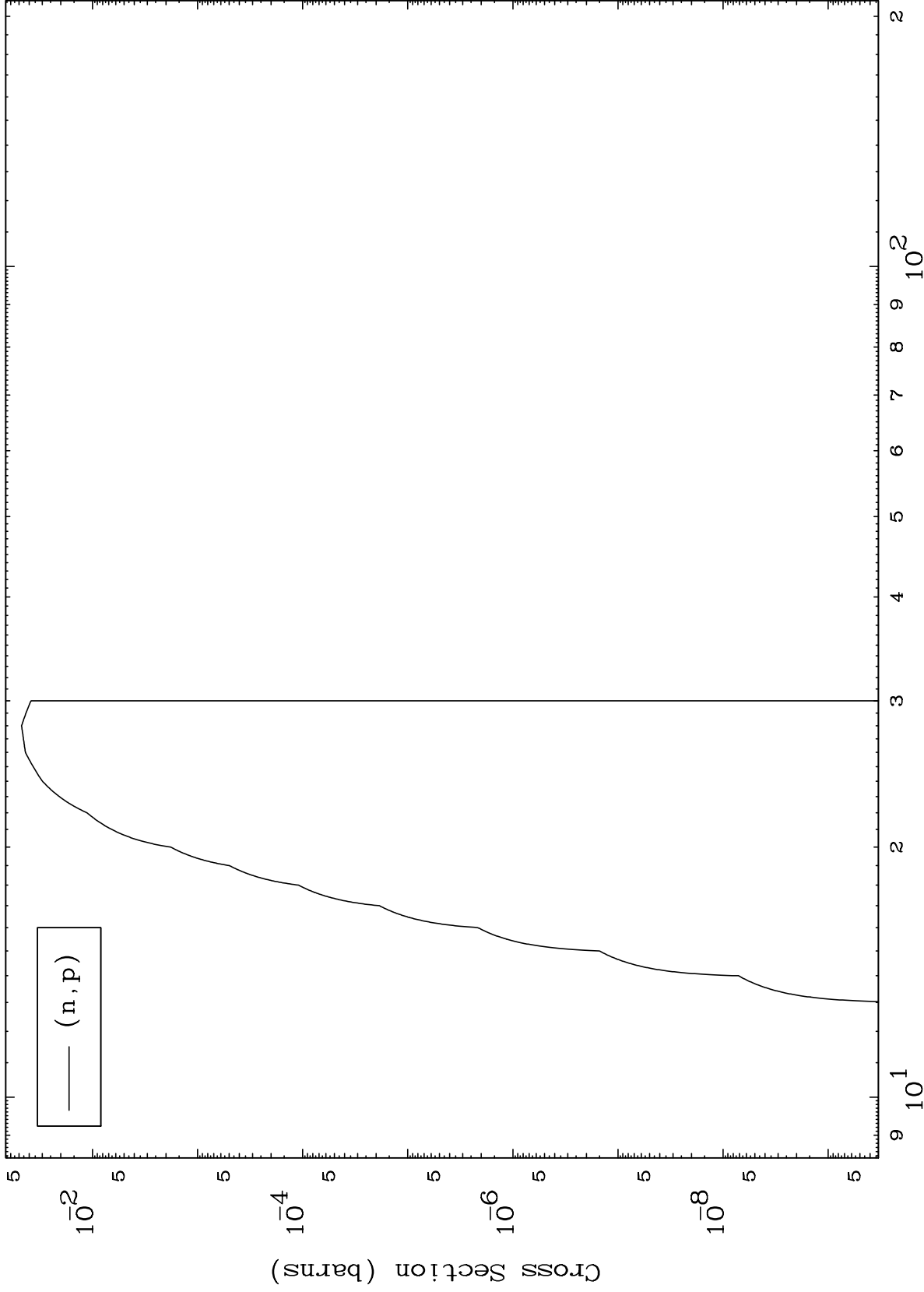


MAT 7980

( $\alpha, p$ ) Levels

80-Hg-181

0 Kelvin Cross Sections



Incident Energy (MeV)

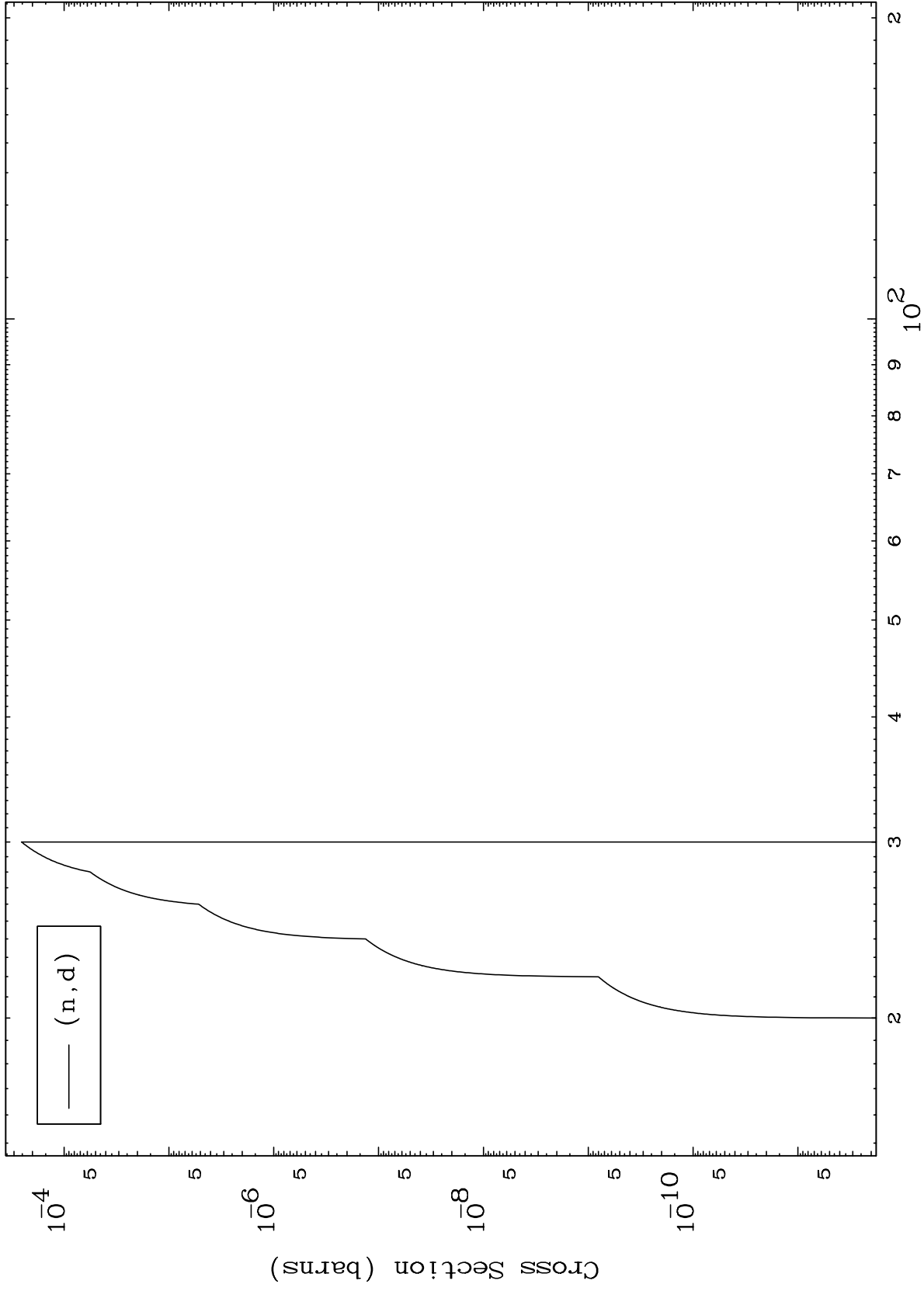
80-Hg-181

6

MAT 7980

80-Hg-181

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



7

80-Hg-181

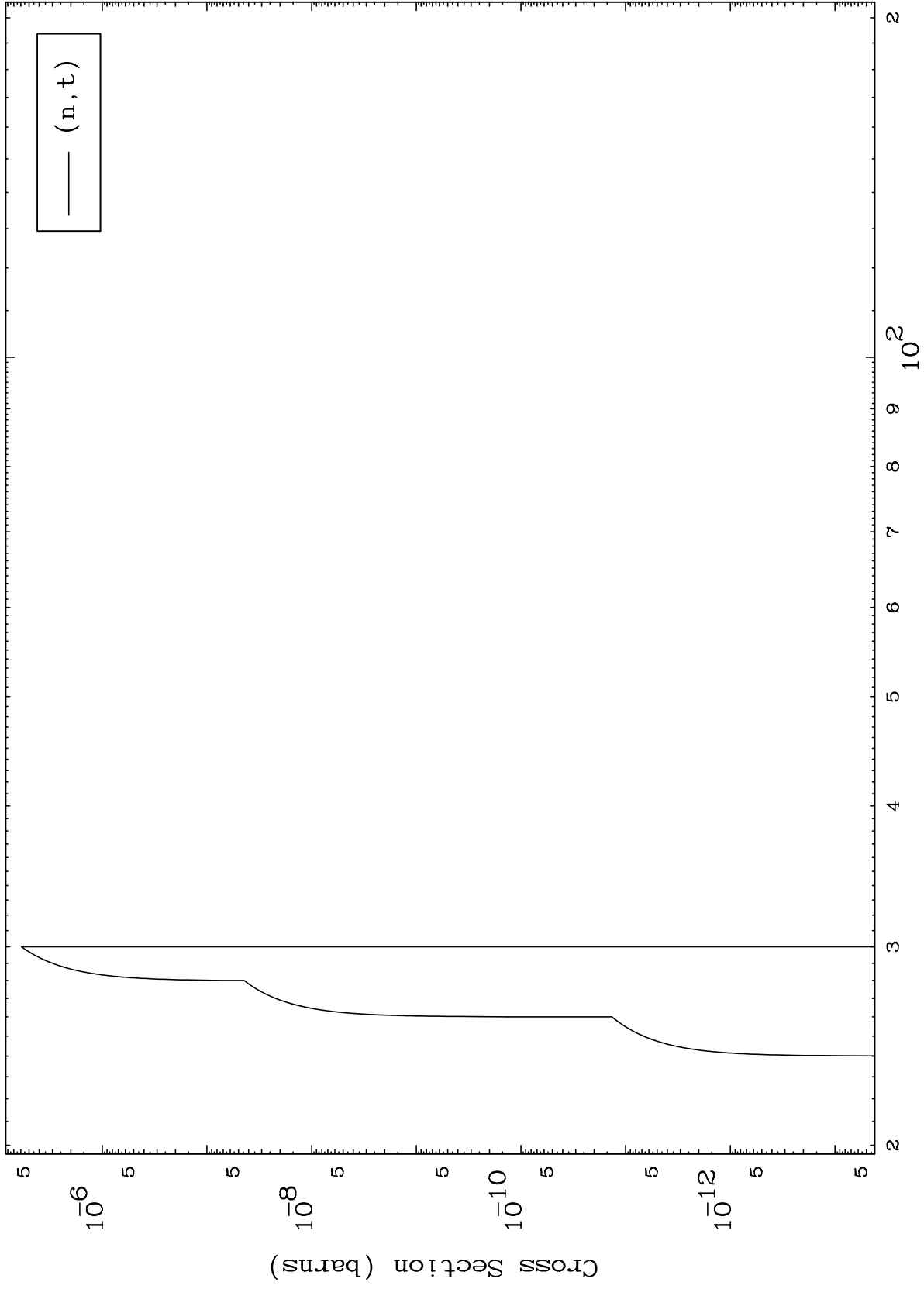
Incident Energy (MeV)



MAT 7980

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

80-Hg-181



8

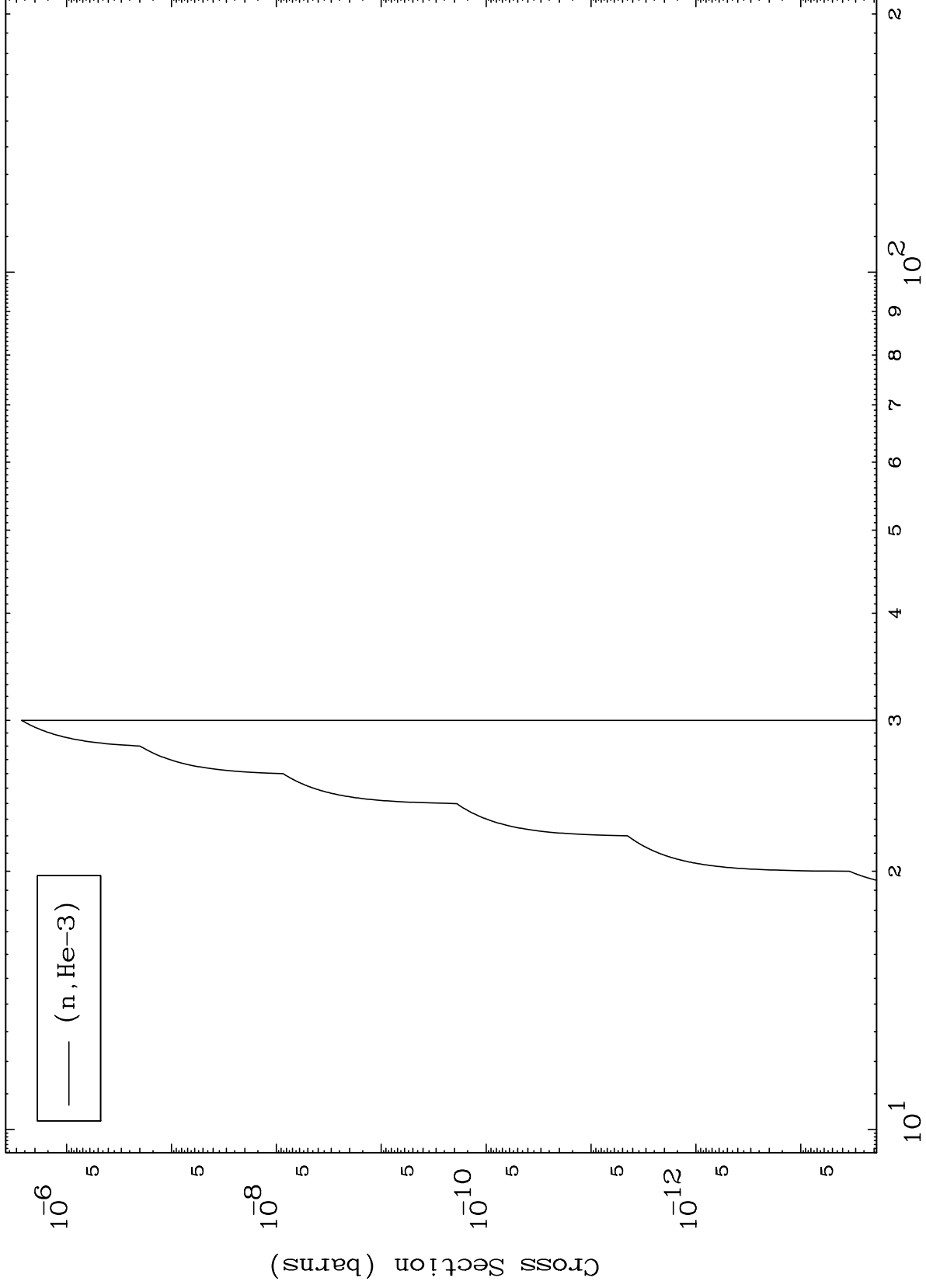
Incident Energy (MeV)

80-Hg-181

MAT 7980

( $\alpha$ , He3) Levels  
0 Kelvin Cross Sections

80-Hg-181



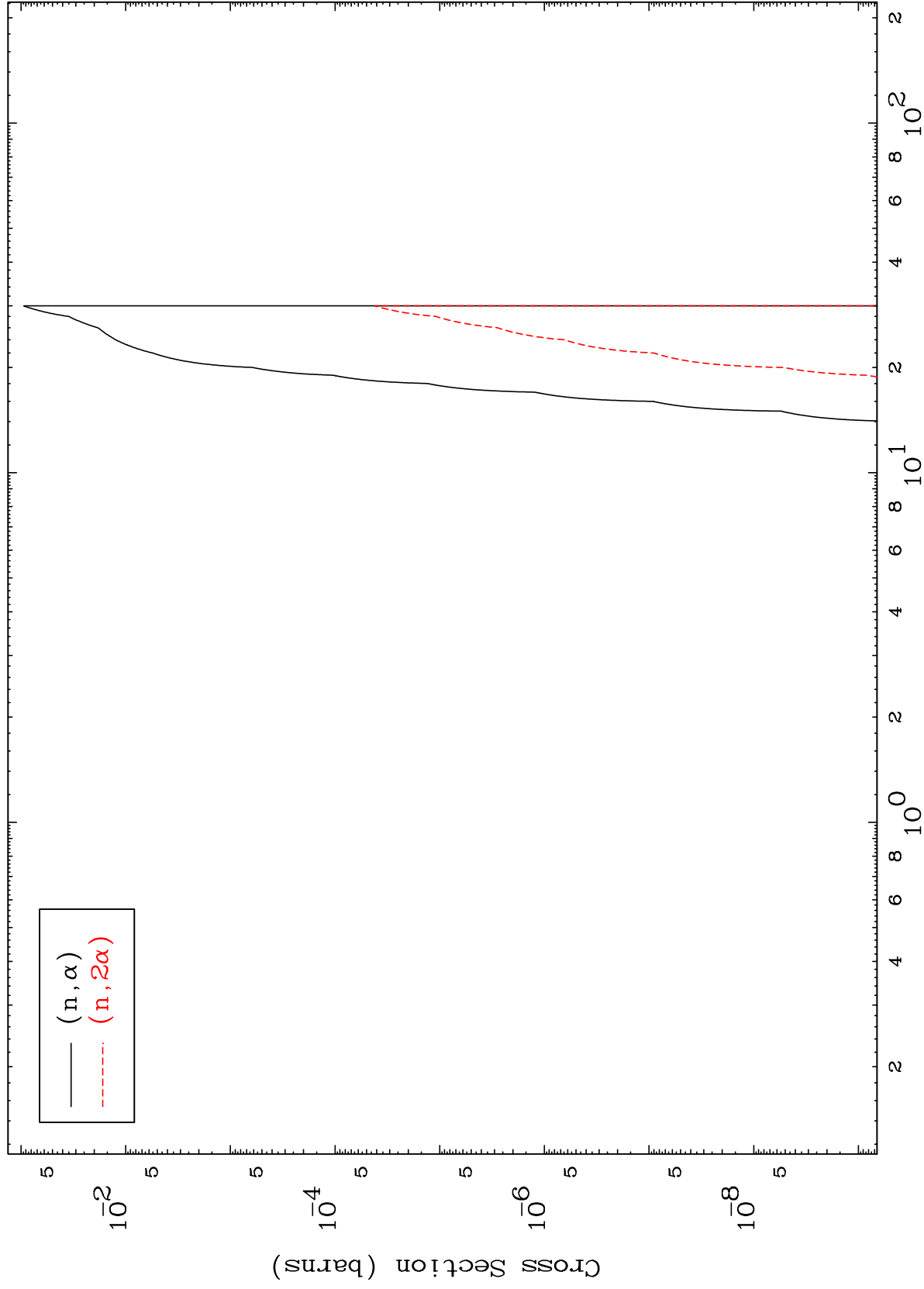
Incident Energy (MeV)

80-Hg-181

MAT 7980

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

80-Hg-181



10

Incident Energy (MeV)

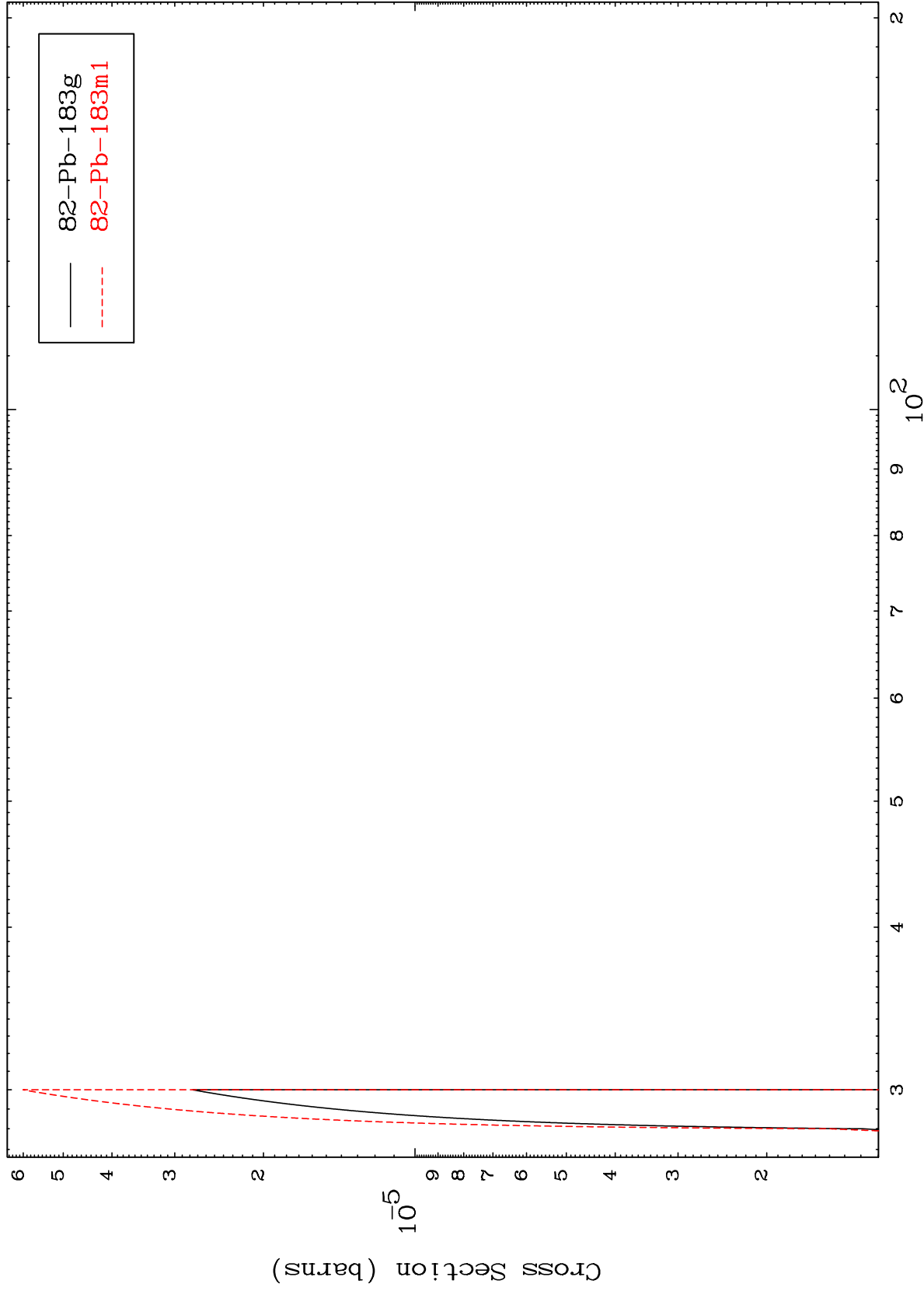
80-Hg-181

MAT 7980

(n,2n)

80-Hg-181

Radionuclide Production Cross Section



11

Incident Energy (MeV)

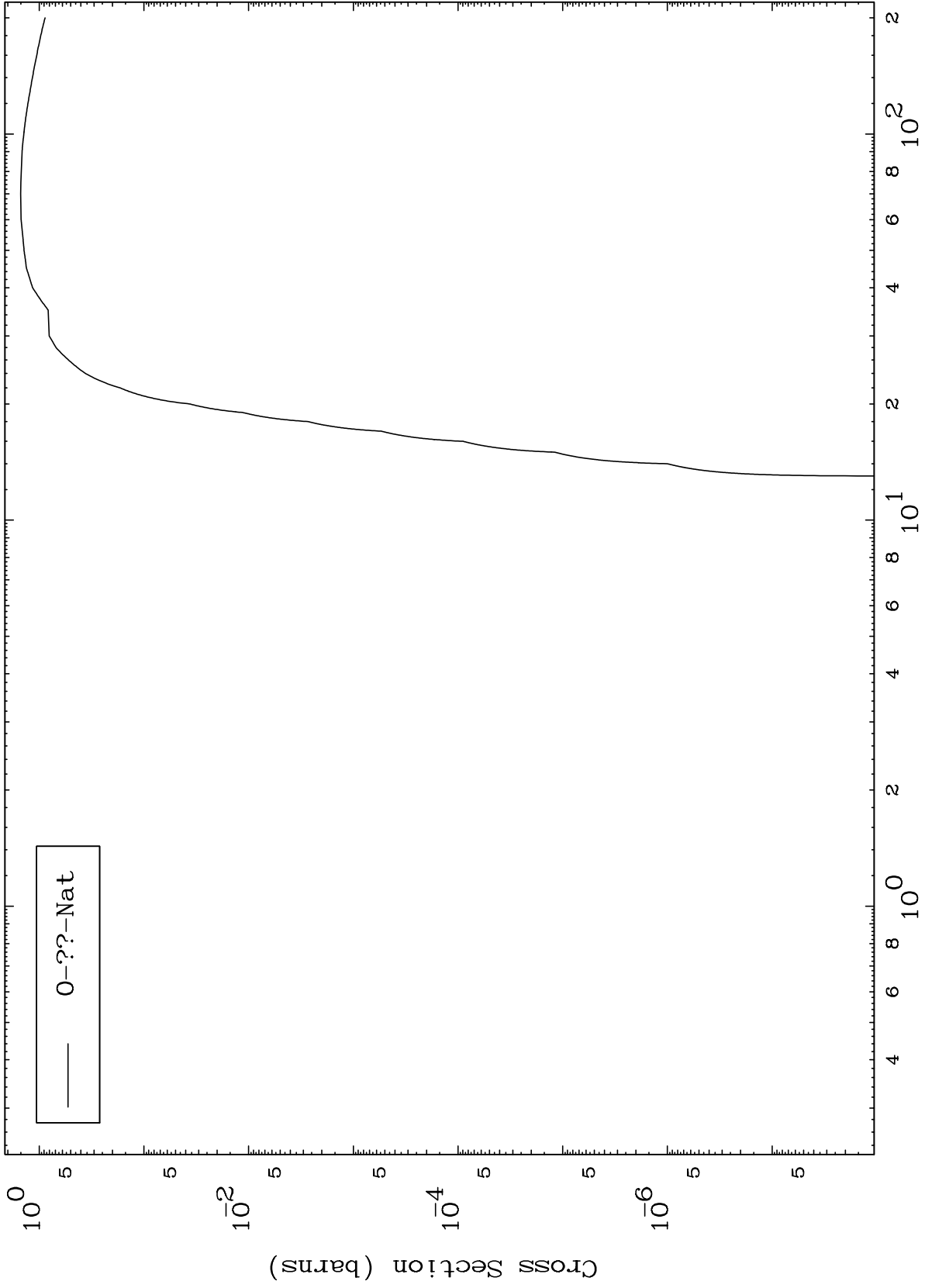
80-Hg-181

MAT 7980

Fission

80-Hg-181

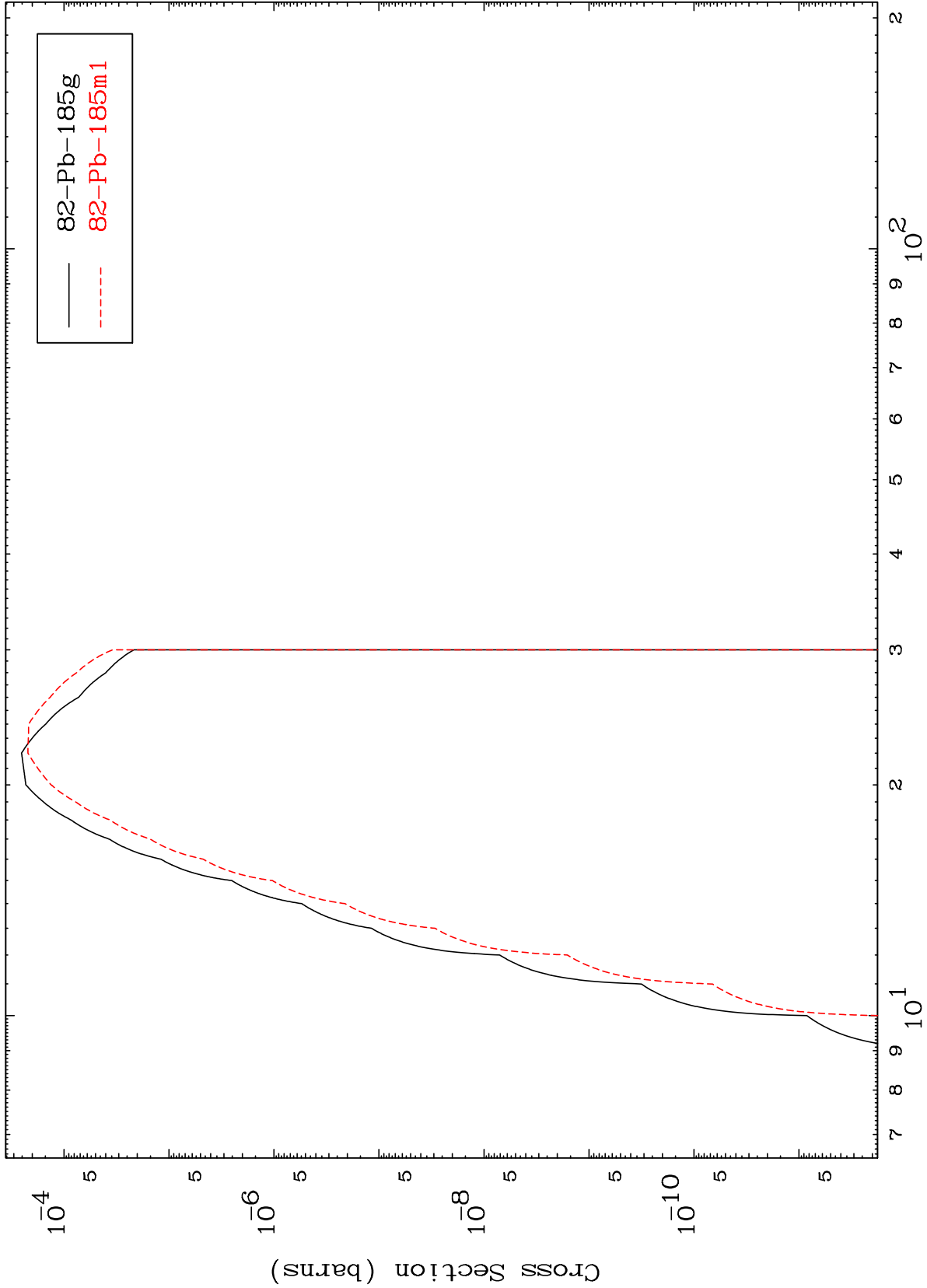
Radionuclide Production Cross Section



MAT 7980

80-Hg-181

(n,γ)  
Radionuclide Production Cross Section



13

Incident Energy (MeV)

80-Hg-181