

Program EVALPLOT  
(Version 2021-1)

by

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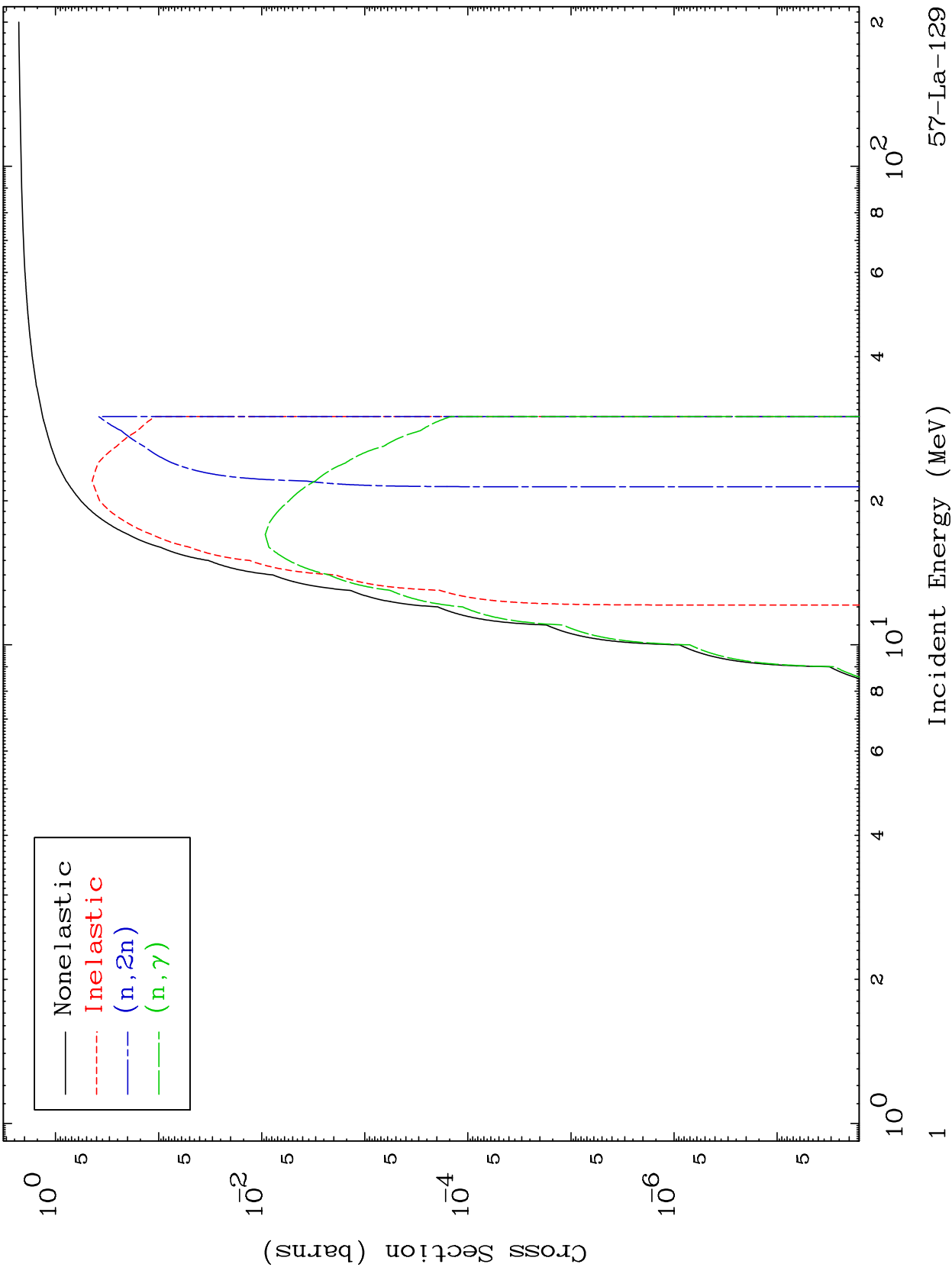
Press Mouse Button to Start

MAT 5698

$\alpha$  Major

57-La-129

0 Kelvin Cross Sections



Legend:  
— Nonelastic  
- - - Inelastic  
- - - (n, 2n)  
- - - (n,  $\gamma$ )

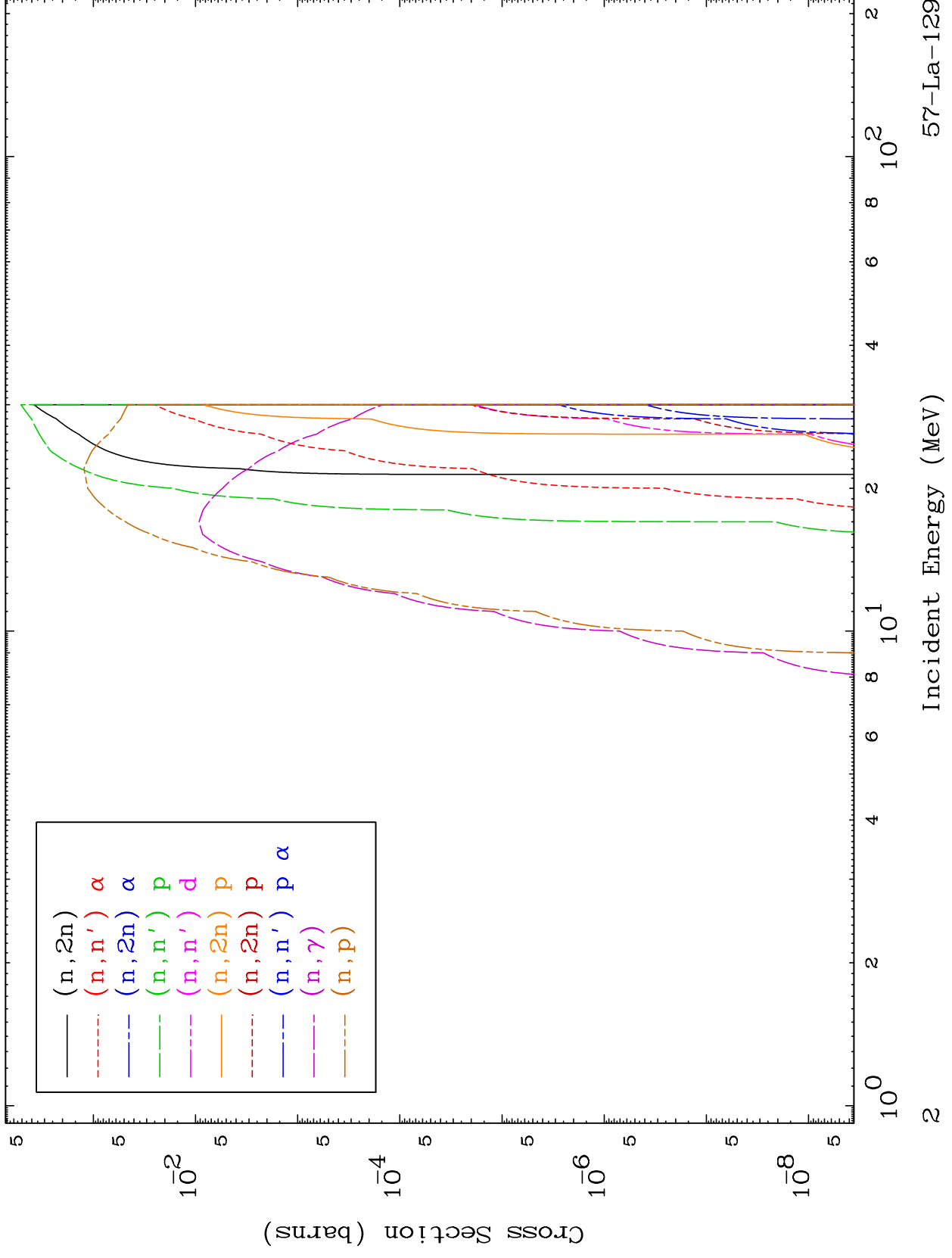
57-La-129

Incident Energy (MeV)

MAT 5698

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

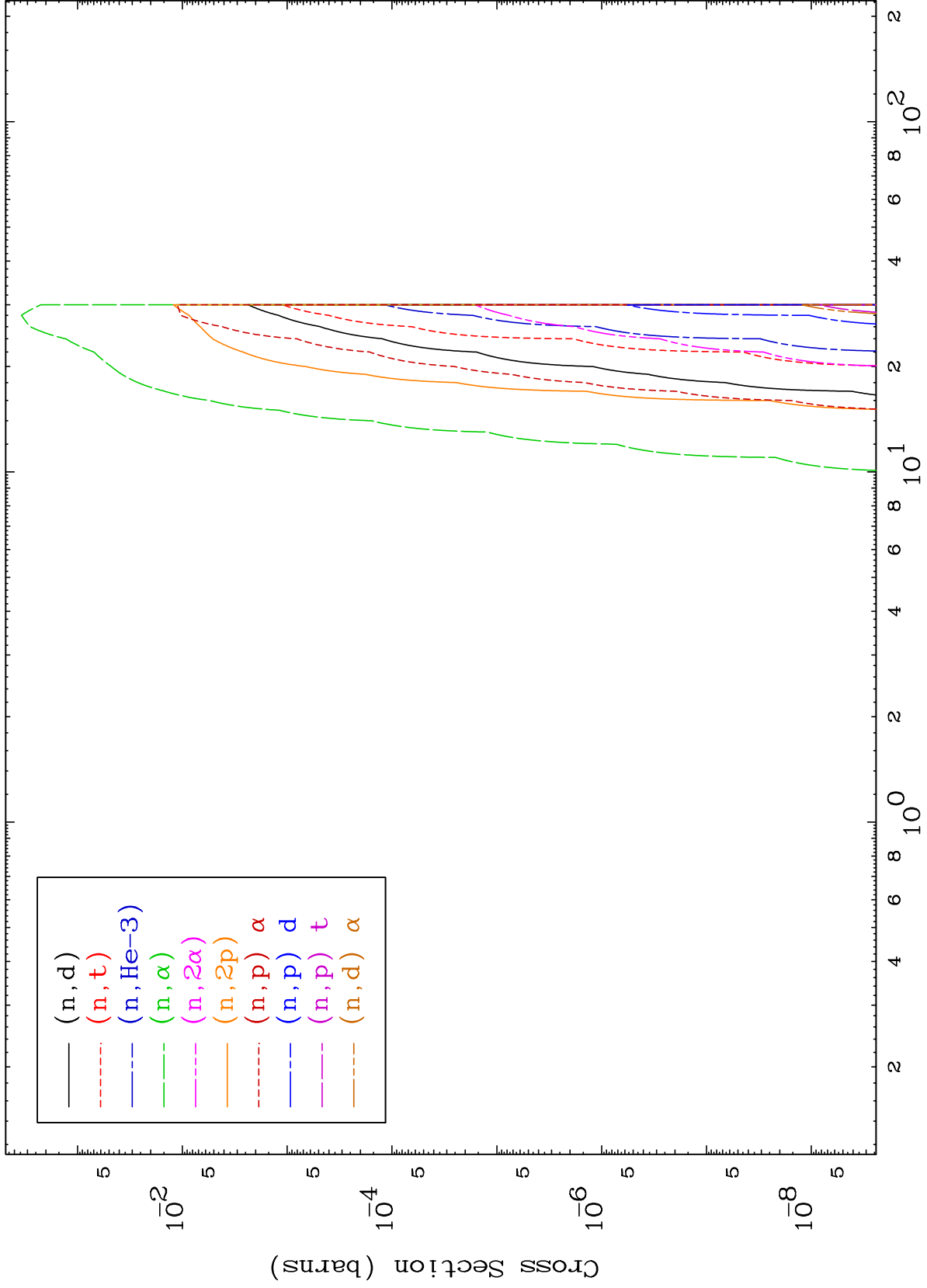
57-La-129

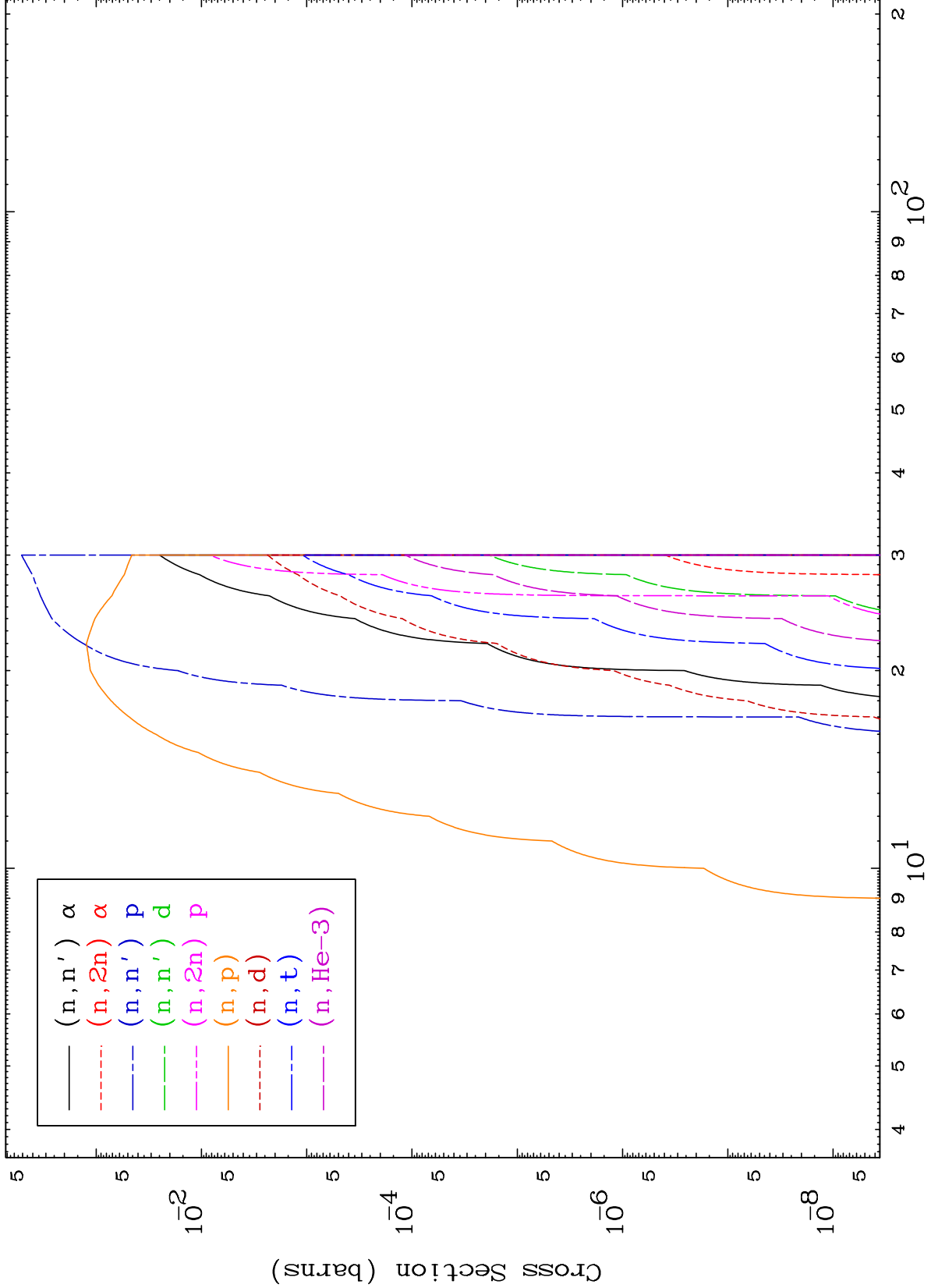


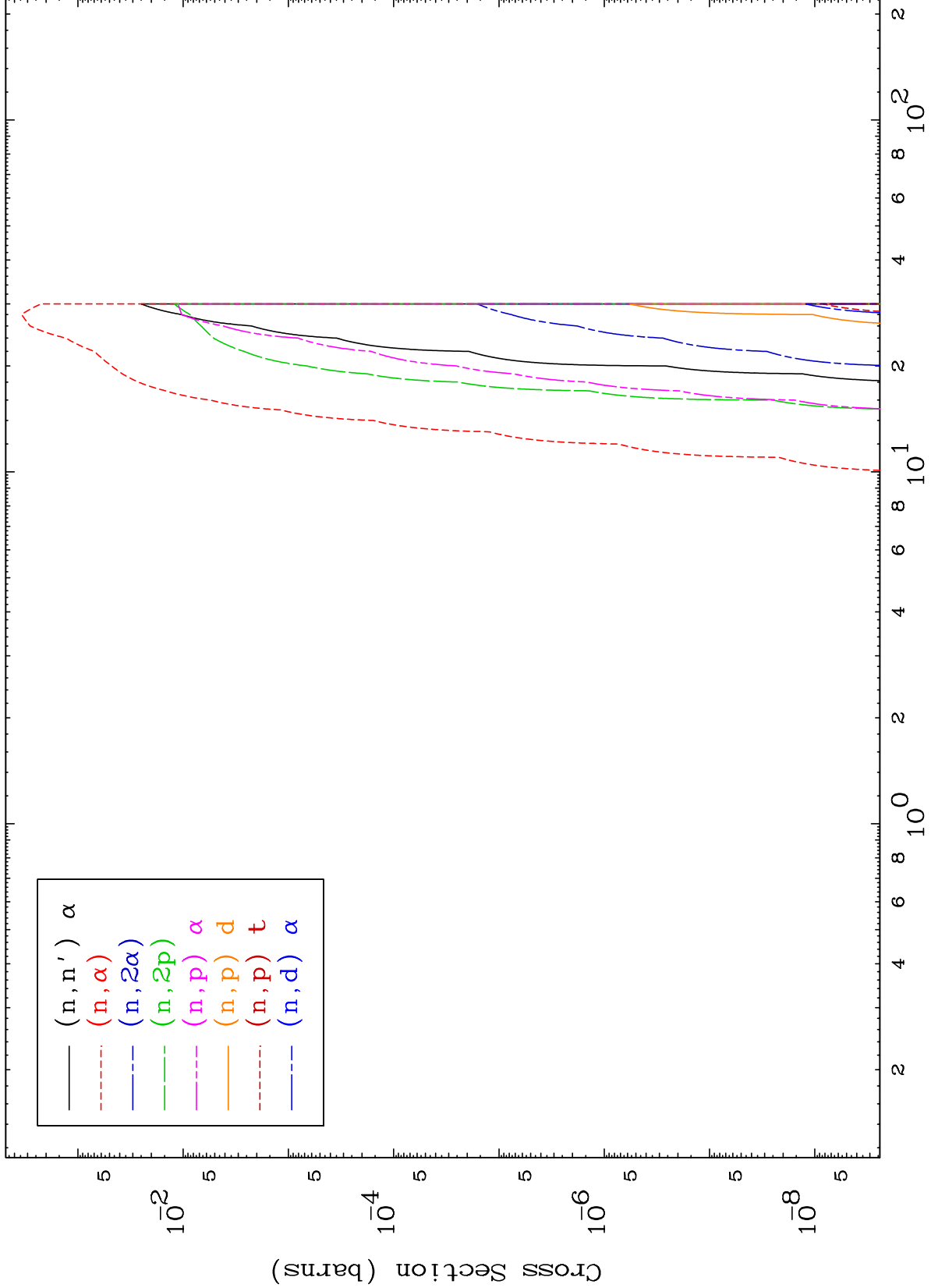
MAT 5698

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

57-La-129





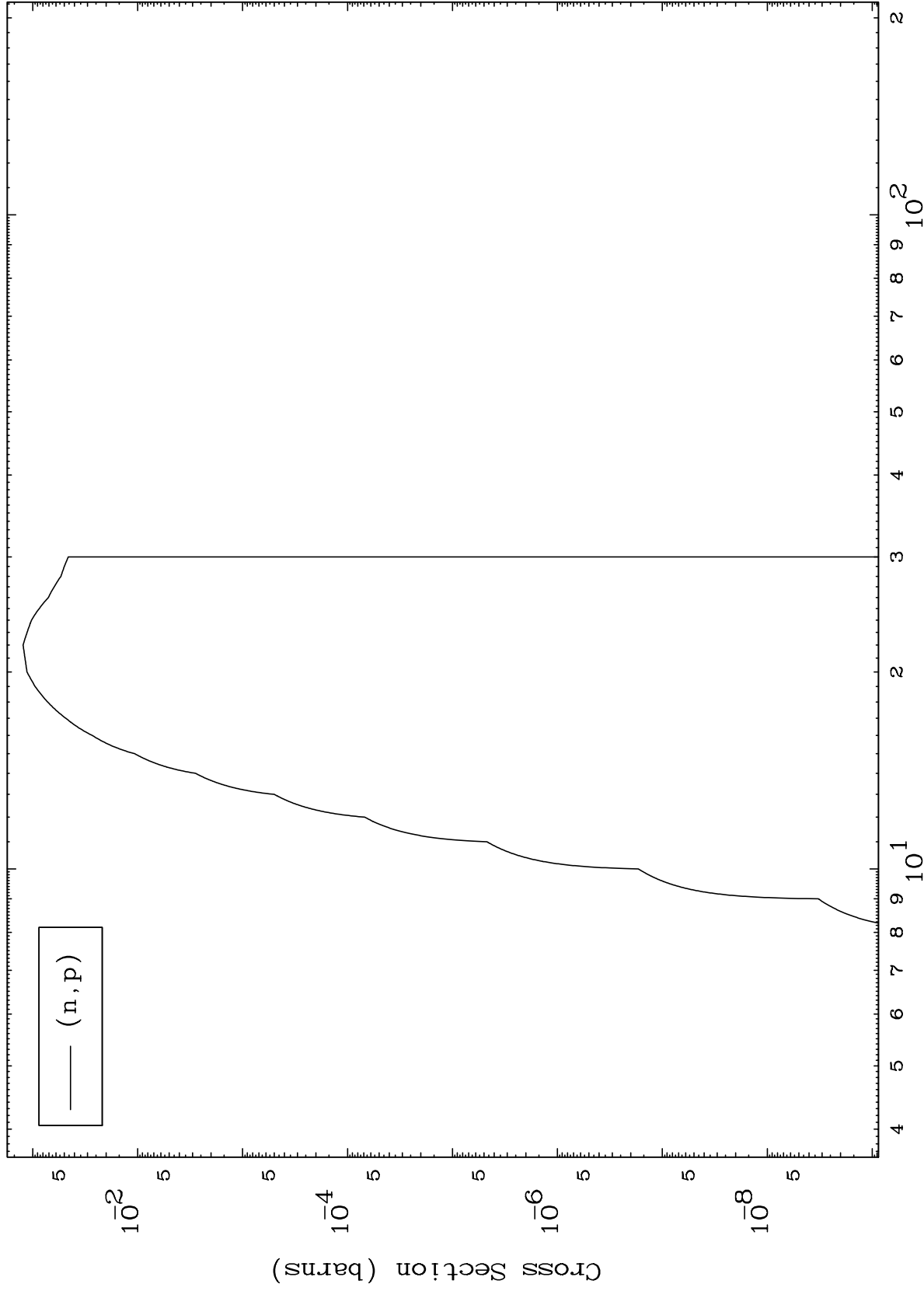


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( $\alpha, p$ ) Levels

57-La-129

0 Kelvin Cross Sections



6

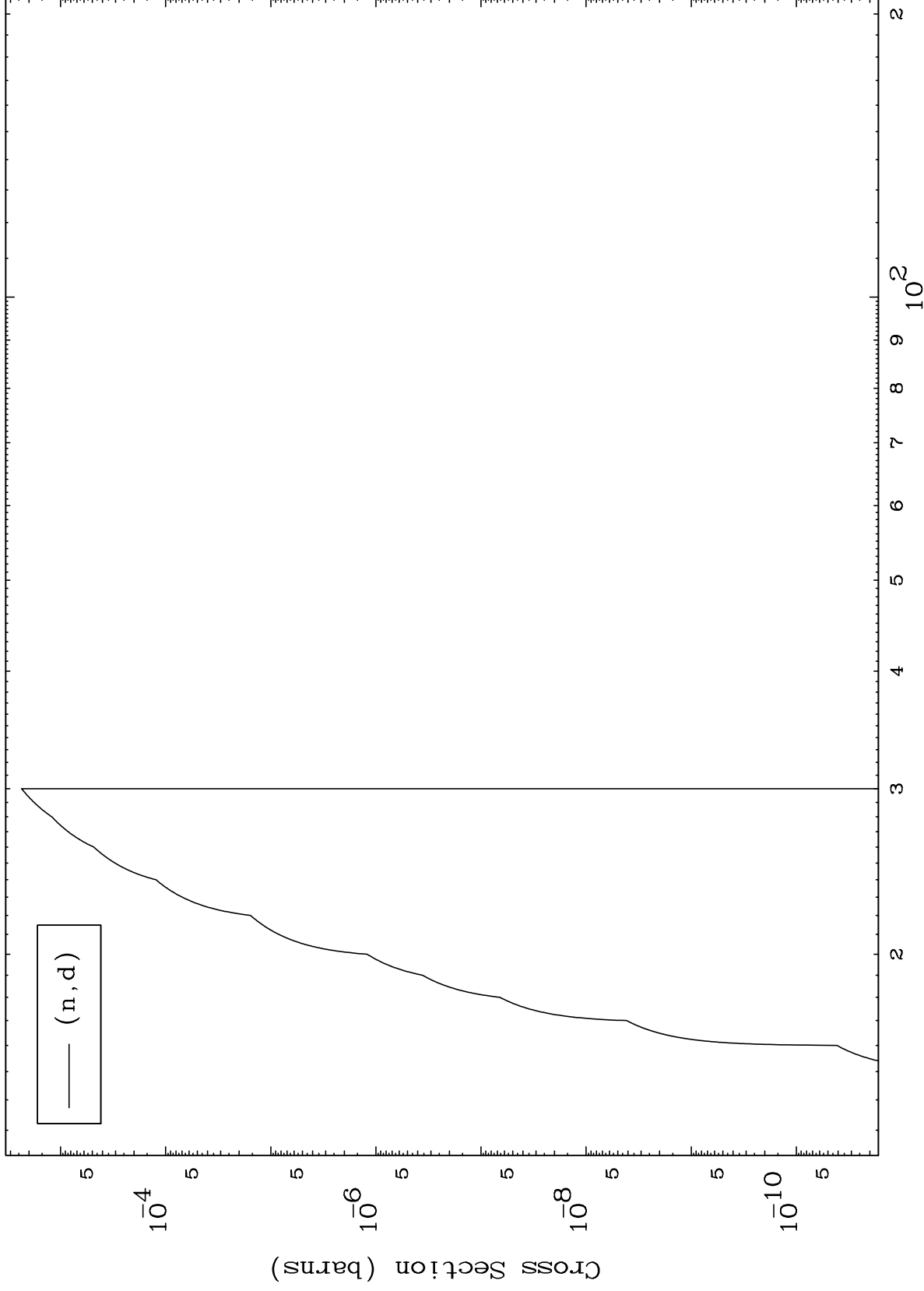
Incident Energy (MeV)

57-La-129

MAT 5698

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

57-La-129

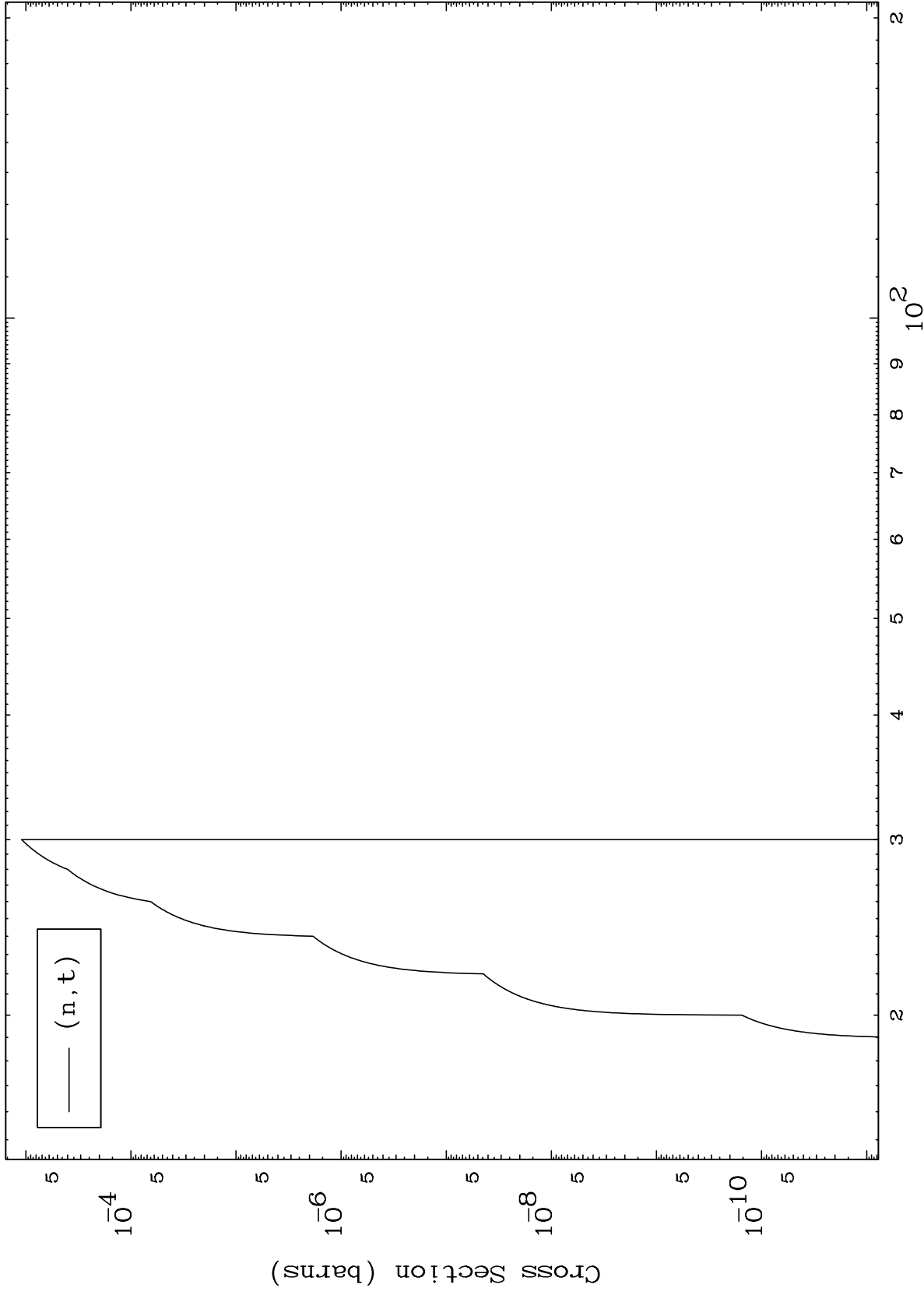


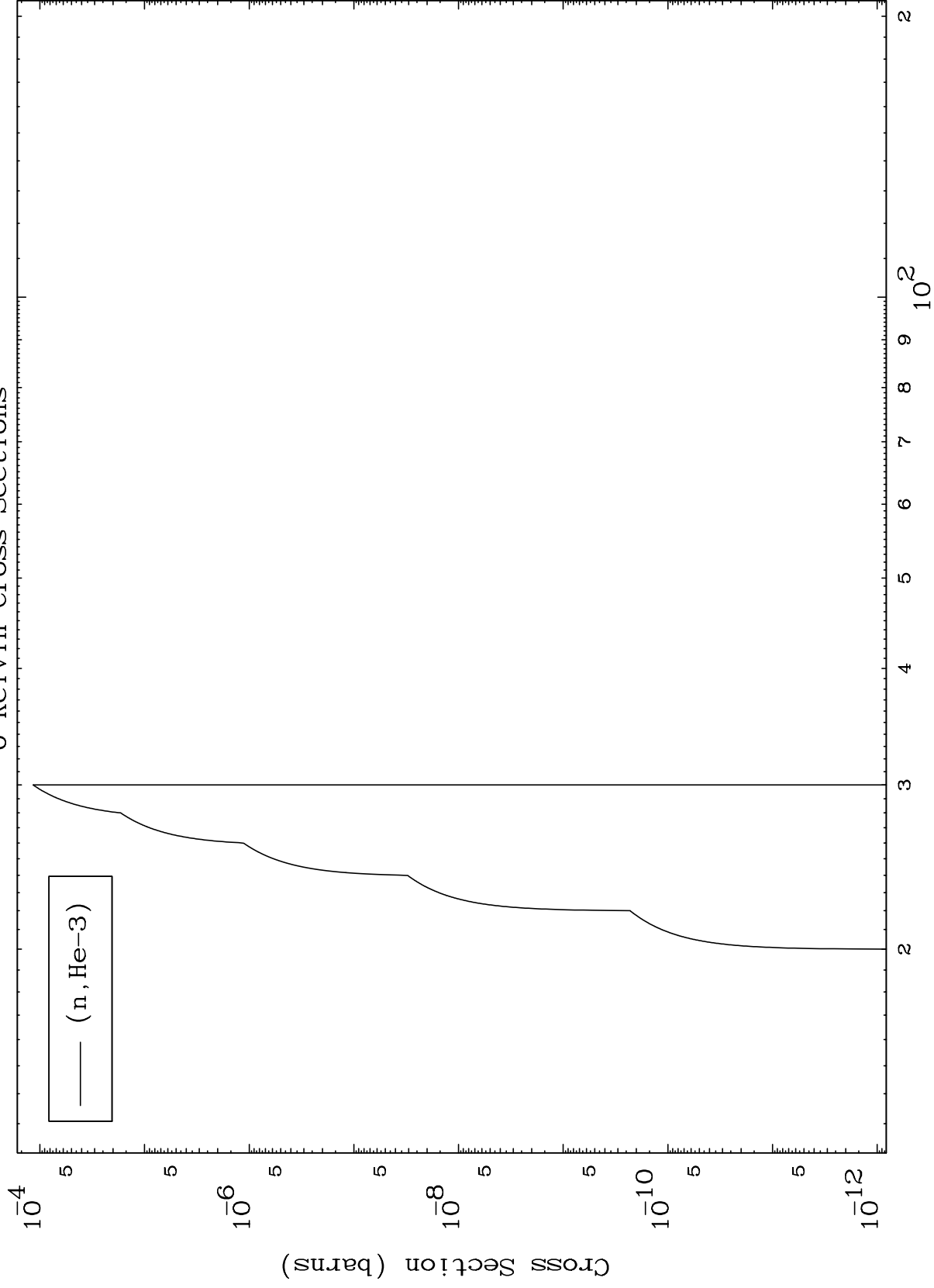
7

Incident Energy (MeV)

57-La-129





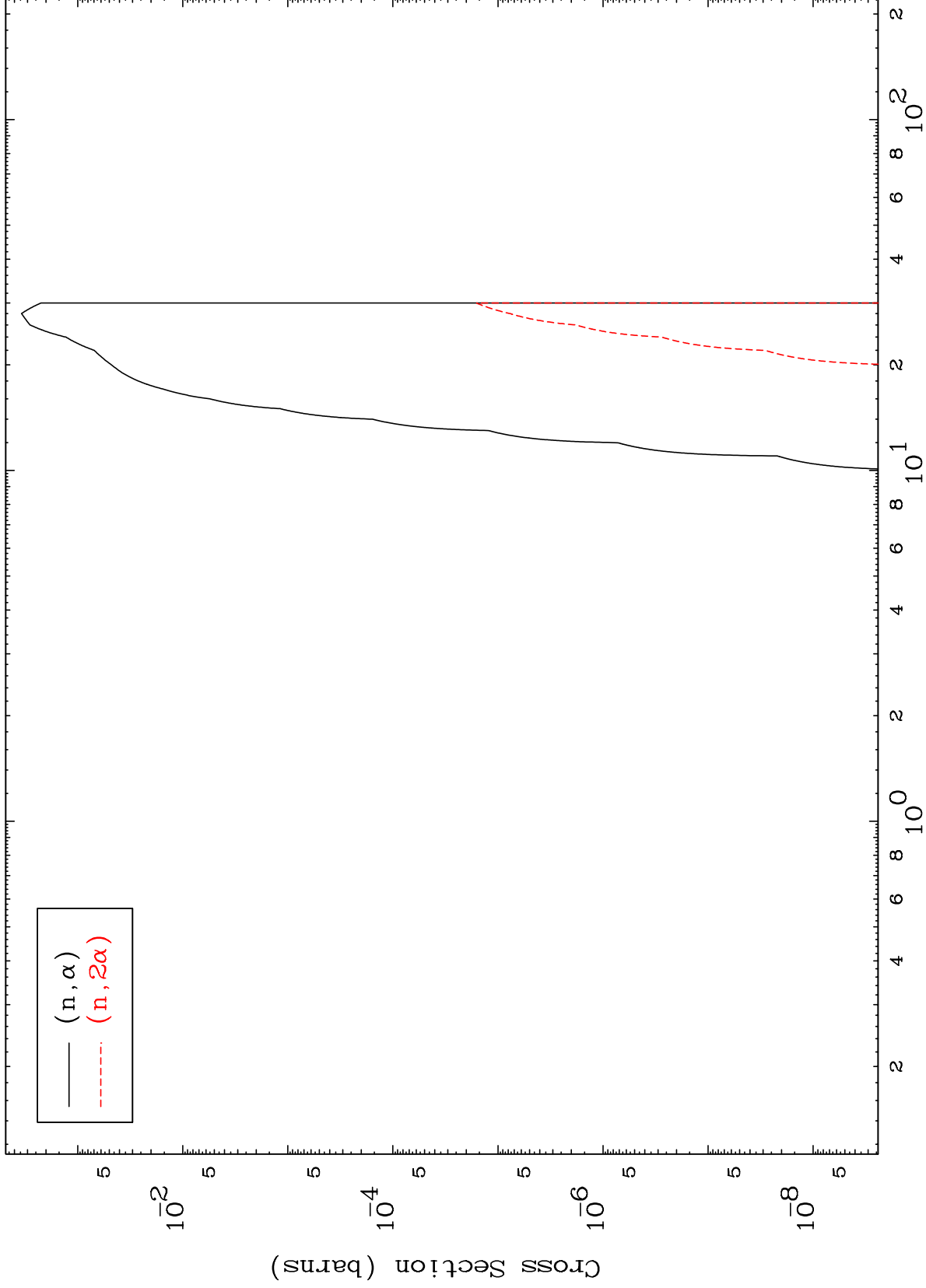


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( $\alpha, \alpha$ ) Levels

57-La-129

0 Kelvin Cross Sections



10

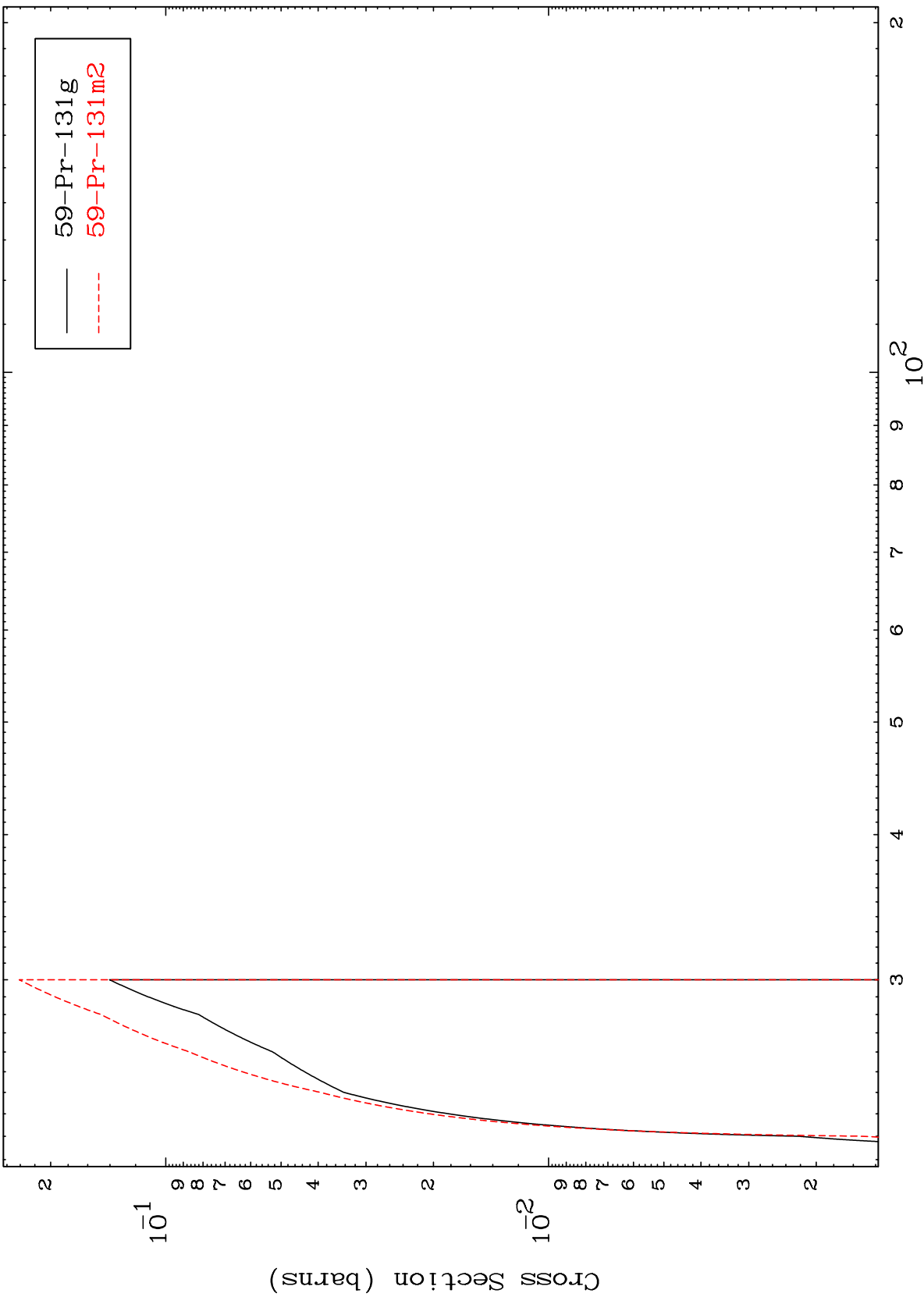
Incident Energy (MeV)

57-La-129

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57-La-129

(n,2n)  
Radionuclide Production Cross Section



11

Incident Energy (MeV)

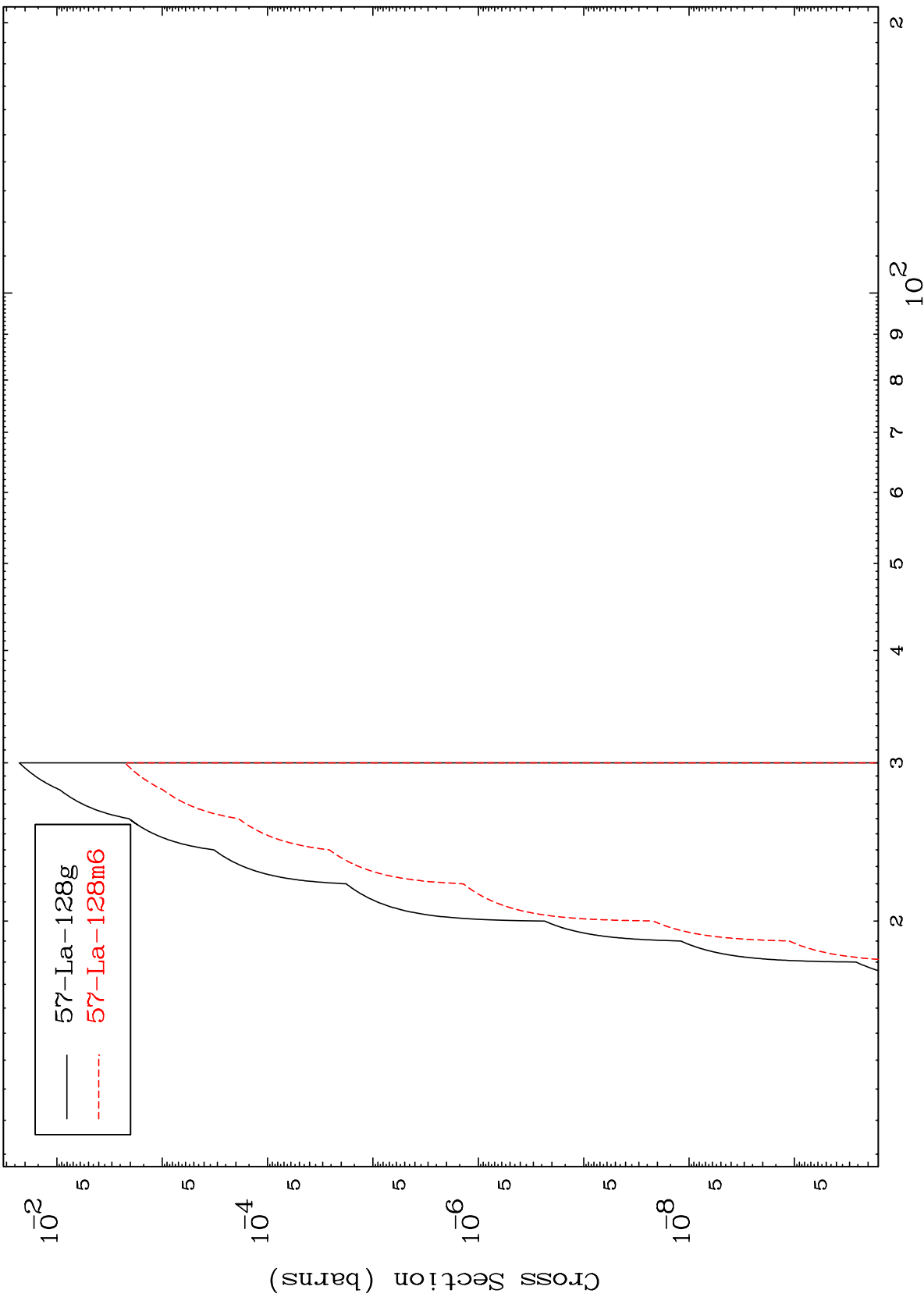
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(n,n')  $\alpha$

57-La-129

Radionuclide Production Cross Section

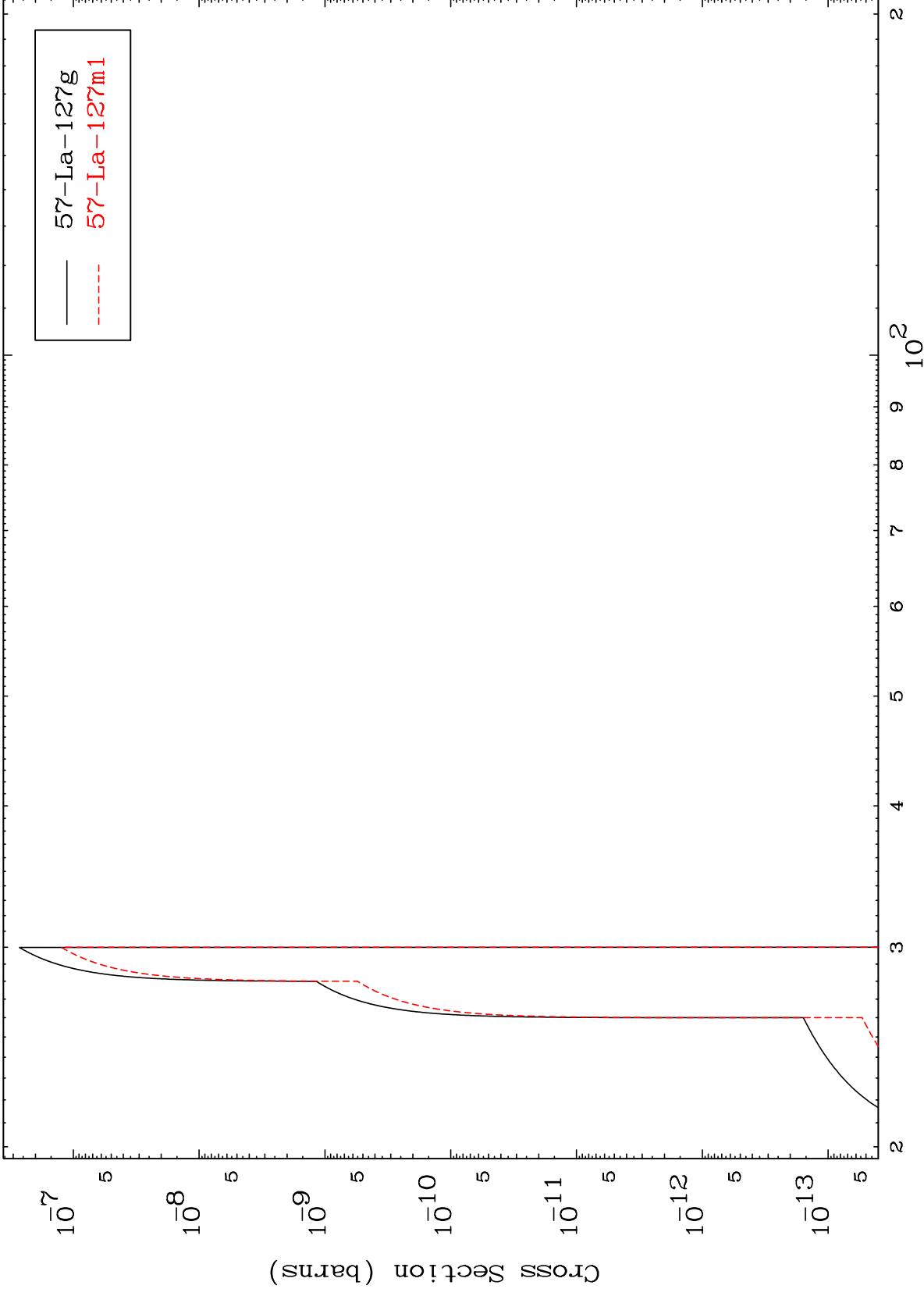


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$(n,2n) \alpha$

57-La-129

Radionuclide Production Cross Section



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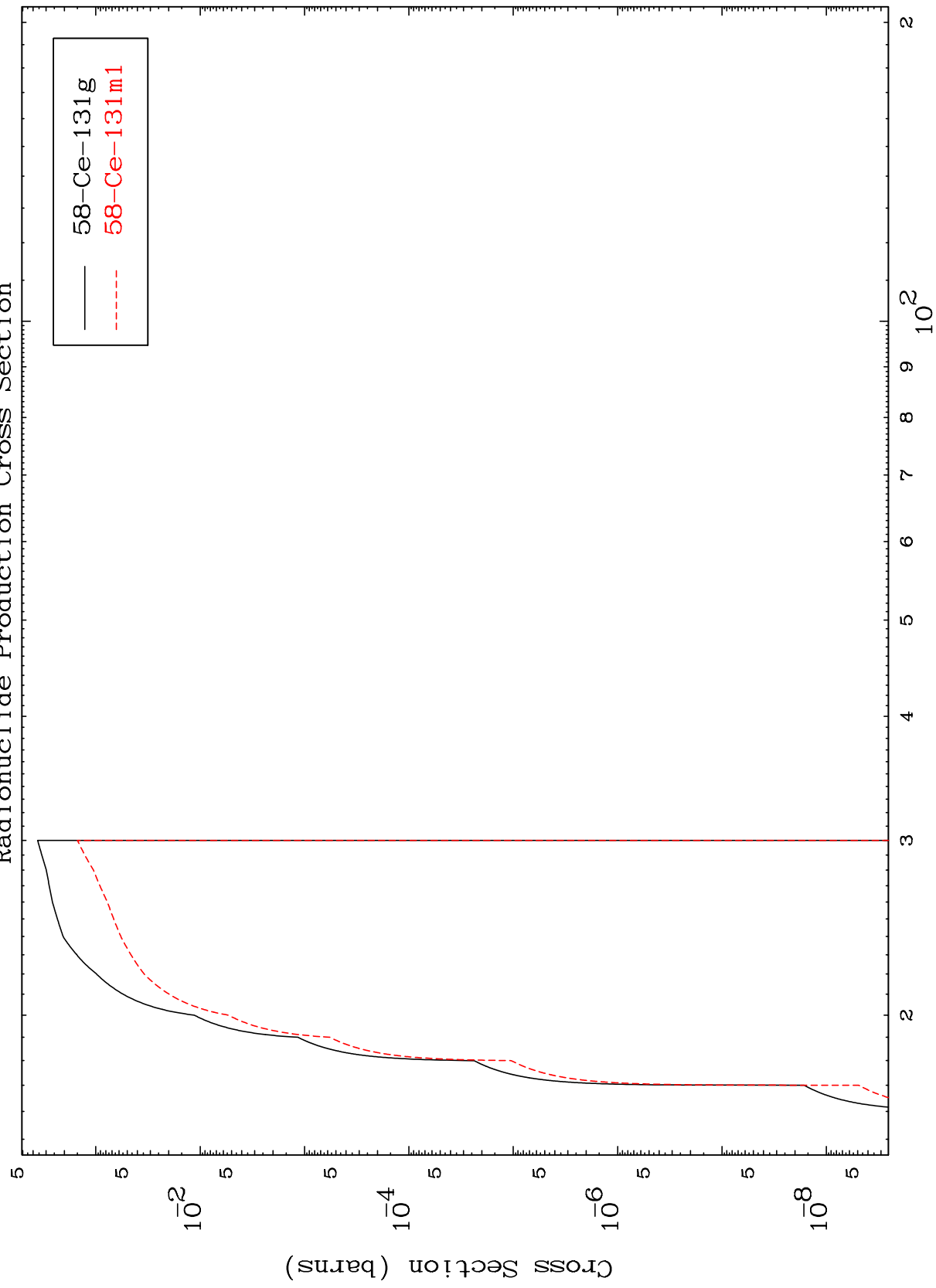
Incident Energy (MeV)

57-La-129

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57-La-129

$(n, n')$  p  
Radionuclide Production Cross Section



14

57-La-129

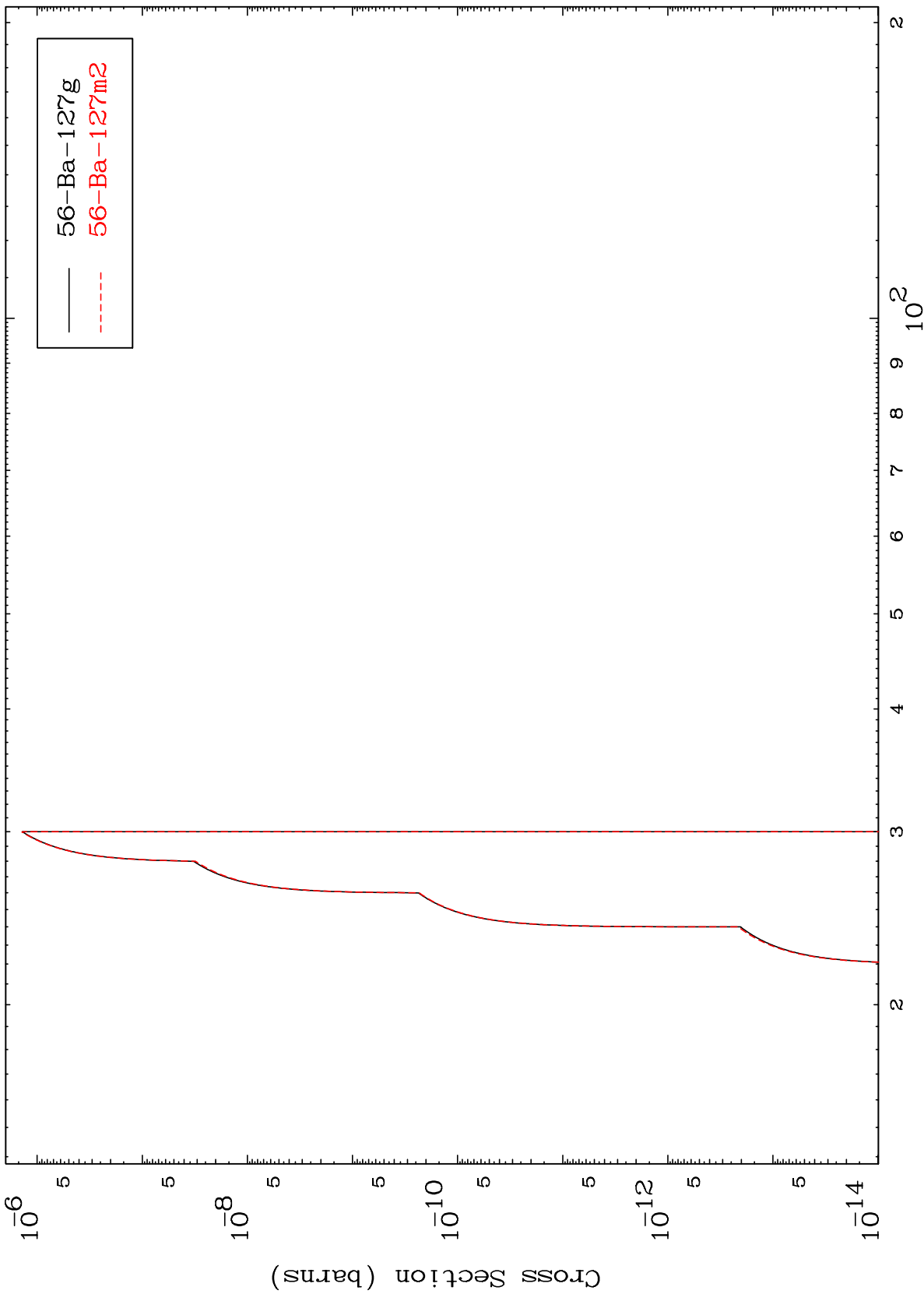
Incident Energy (MeV)

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(n,n') p  $\alpha$

57-La-129

Radionuclide Production Cross Section



15

Incident Energy (MeV)

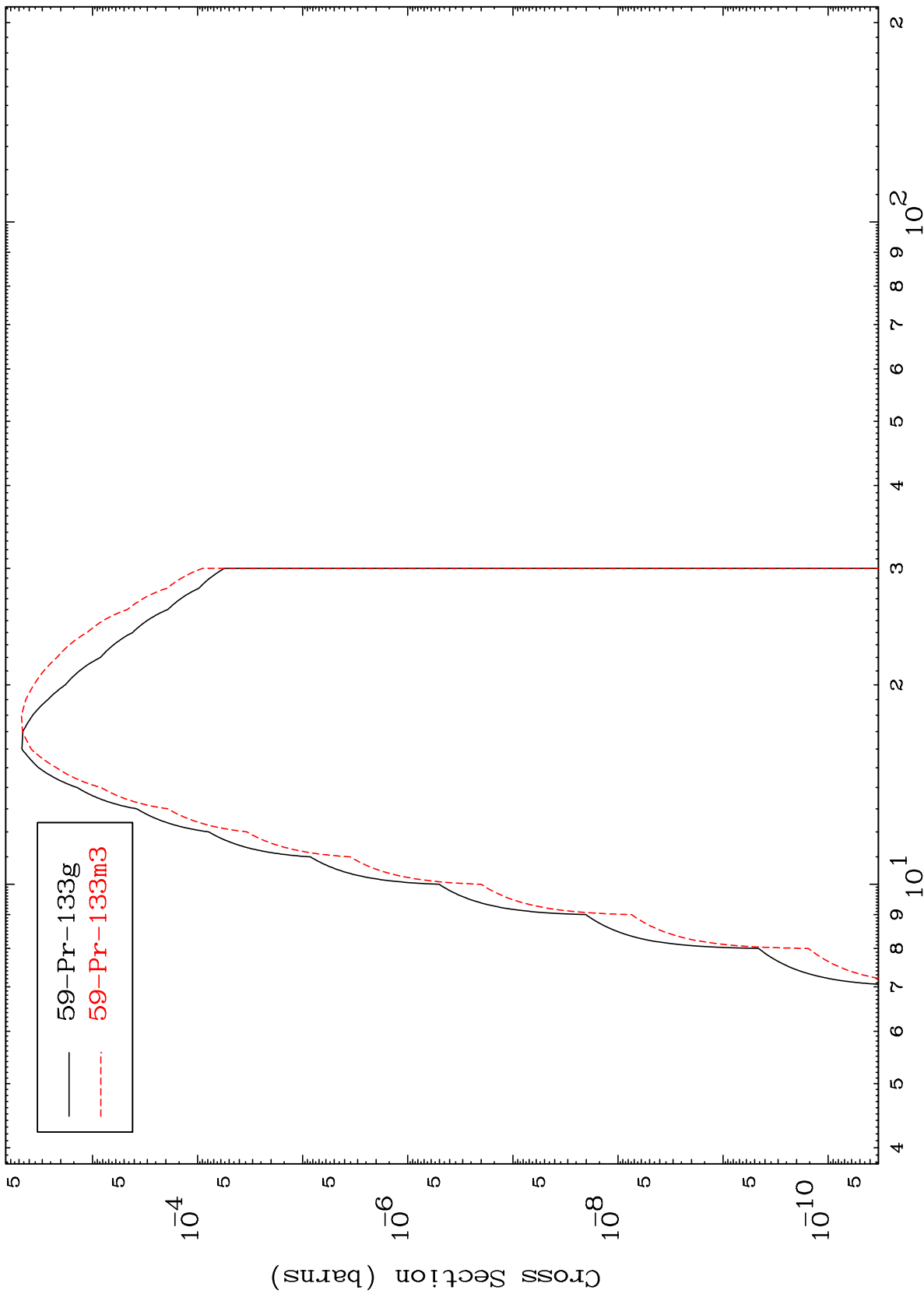
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(n,  $\gamma$ )  
Radionuclide Production Cross Section



57-La-129

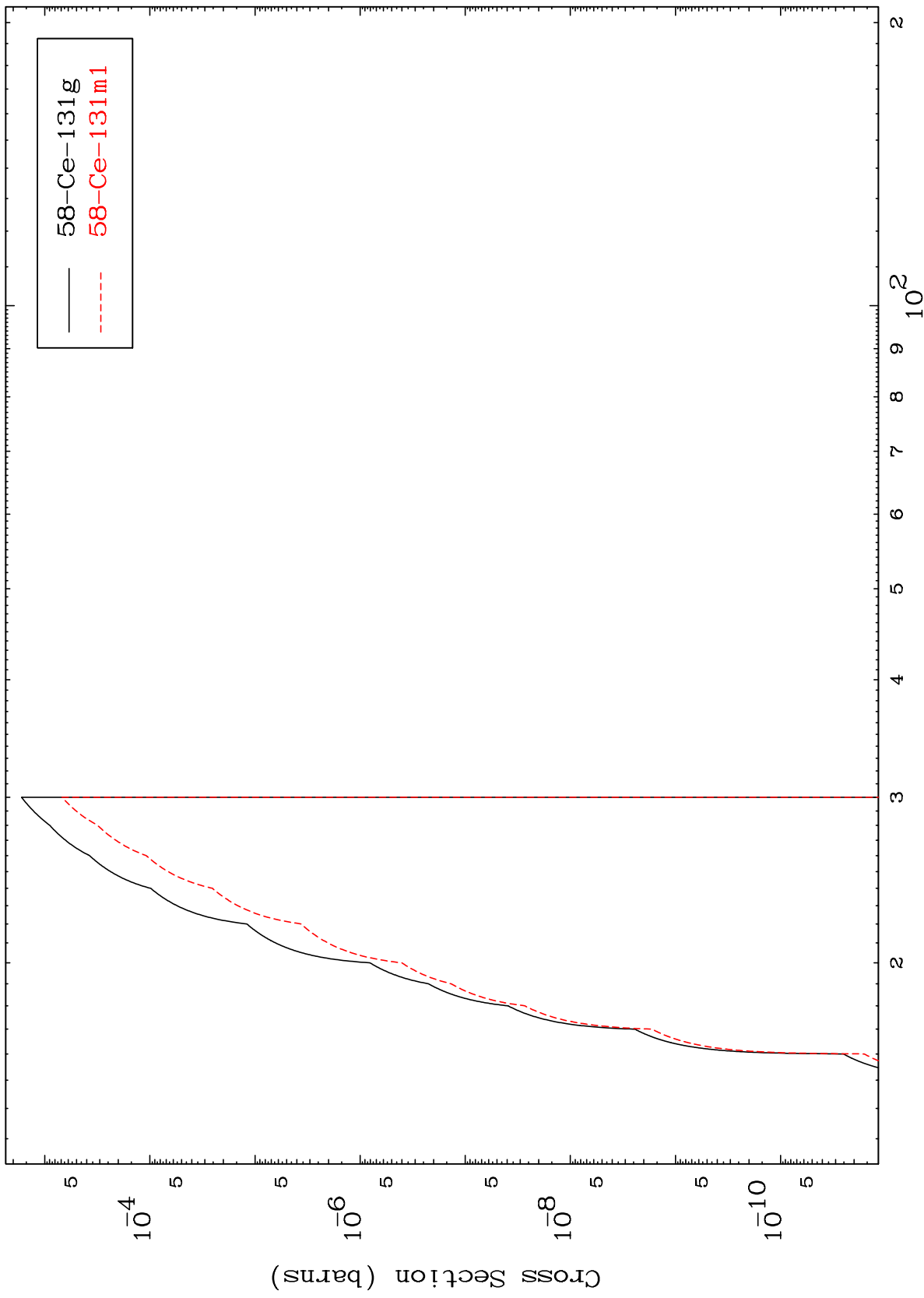
Incident Energy (MeV)

16

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(n,d)  
Radionuclide Production Cross Section



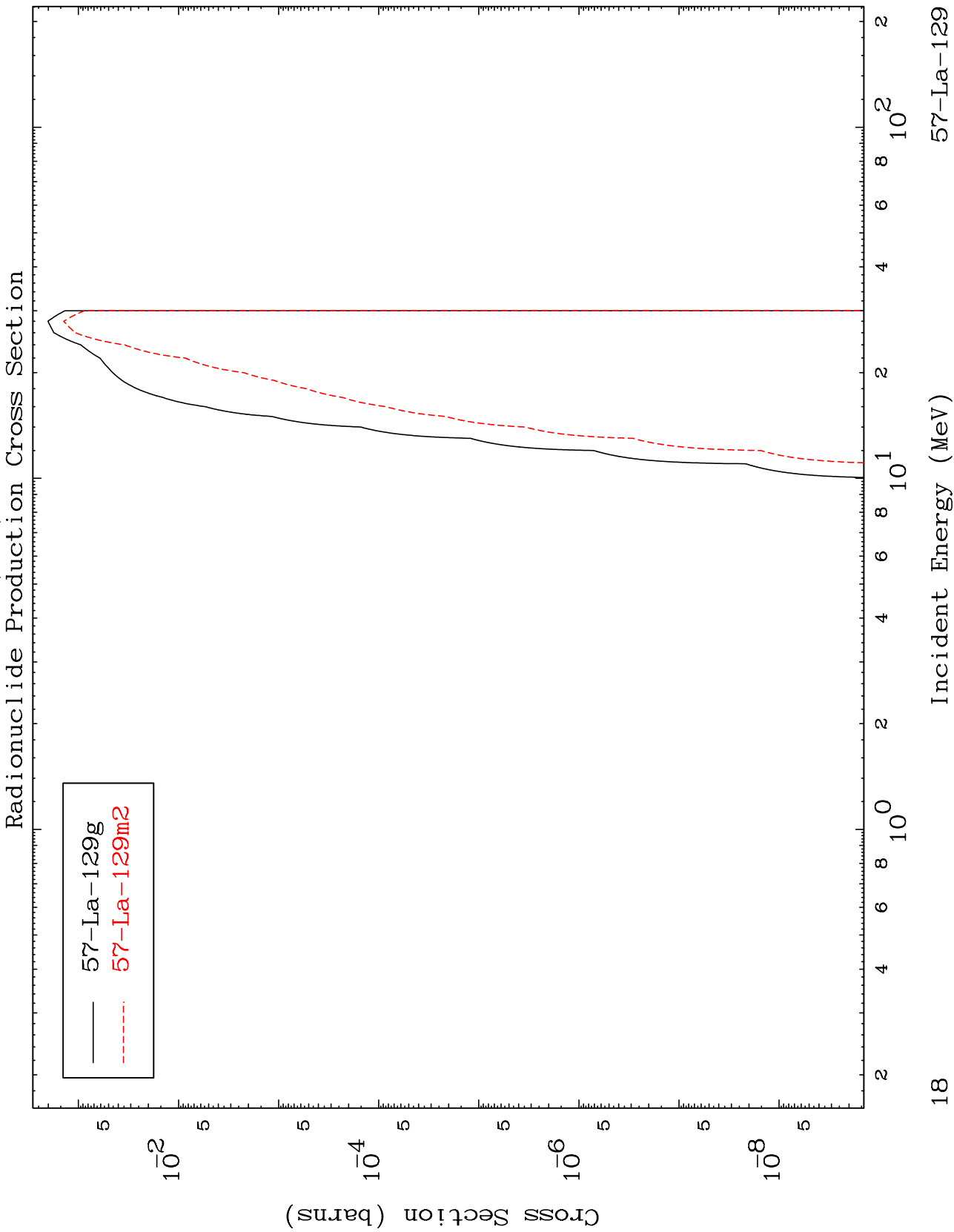
17

57-La-129

Incident Energy (MeV)

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57-La-129



18

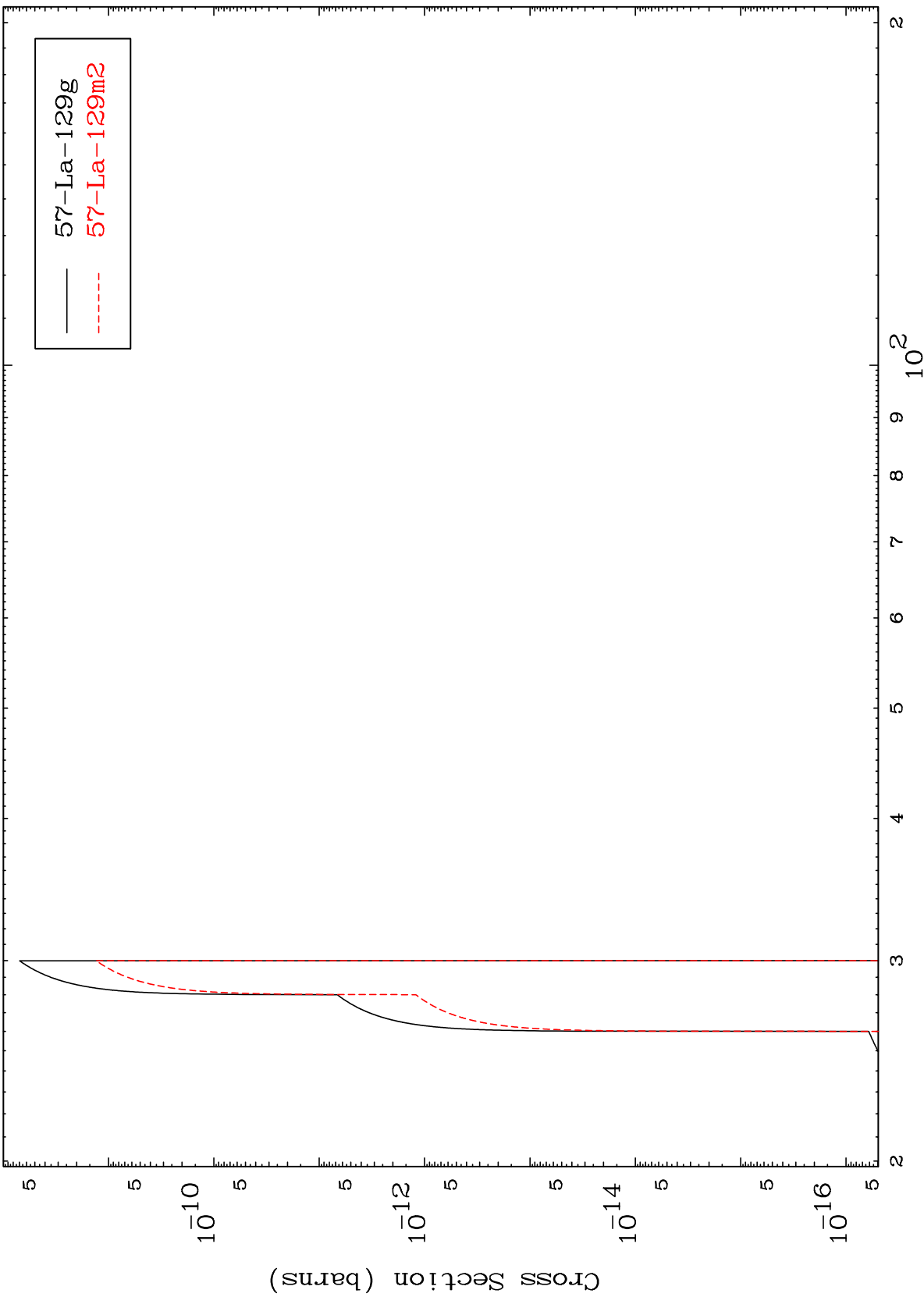
57-La-129

MAT 5698

(n,p) t

57-La-129

Radionuclide Production Cross Section



19

Incident Energy (MeV)

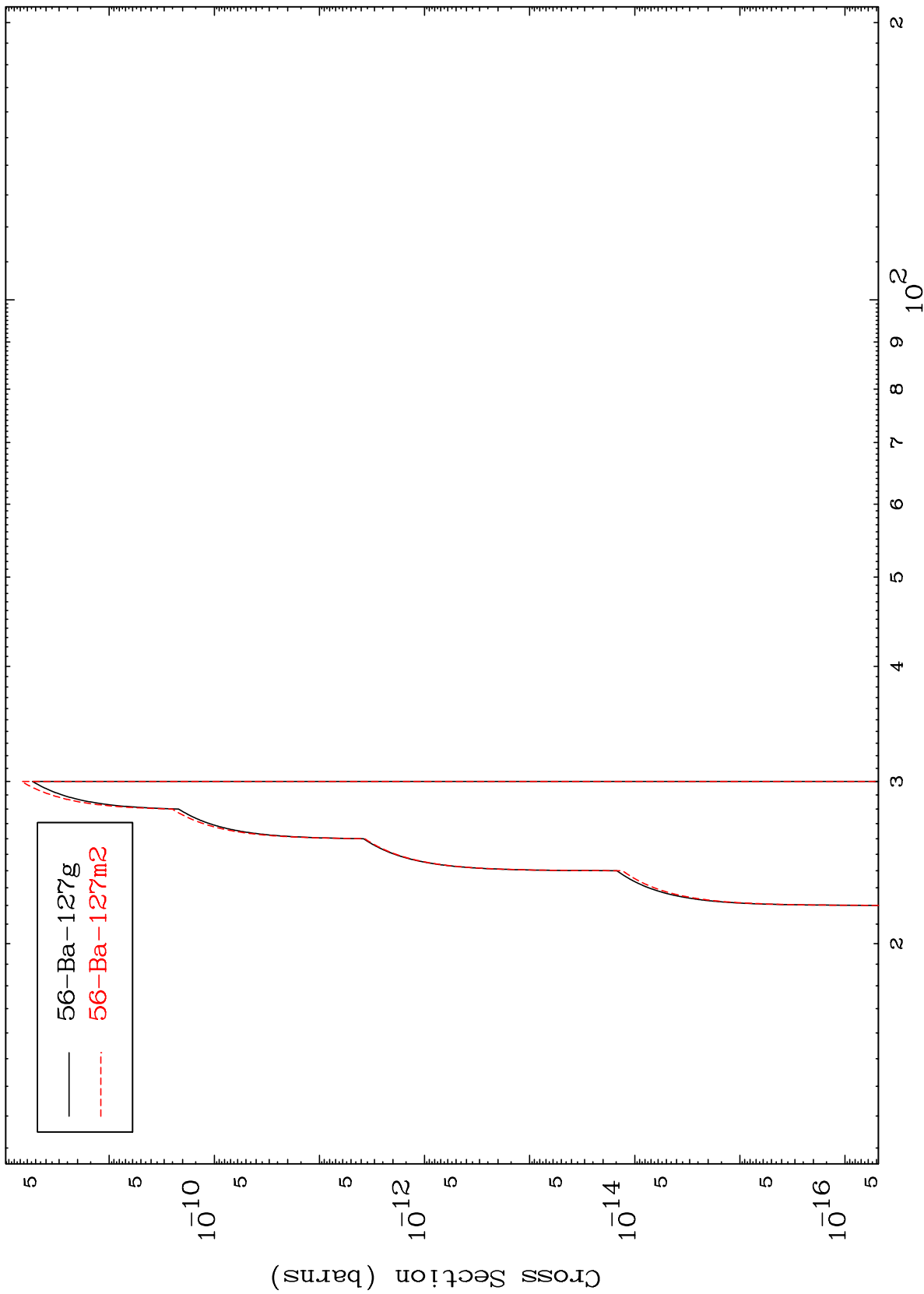
57-La-129

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(n,d)  $\alpha$

57-La-129

Radionuclide Production Cross Section



20

Incident Energy (MeV)

57-La-129