

Program EVALPLOT  
(Version 2021-1)

by

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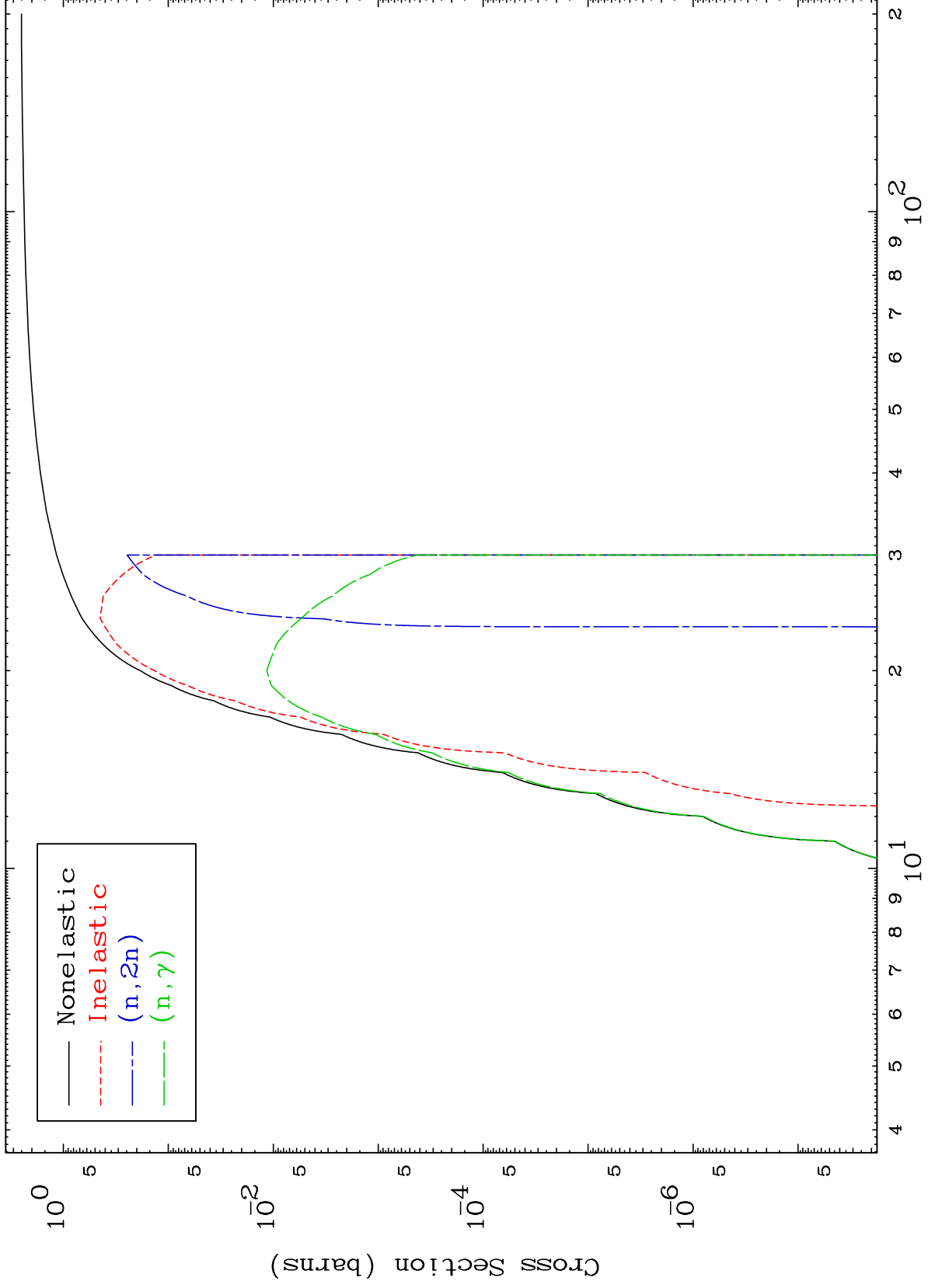
Press Mouse Button to Start

MAT 7088

0 Kelvin Cross Sections

$\alpha$  Major

$^{71}\text{Lu-162n}$



1

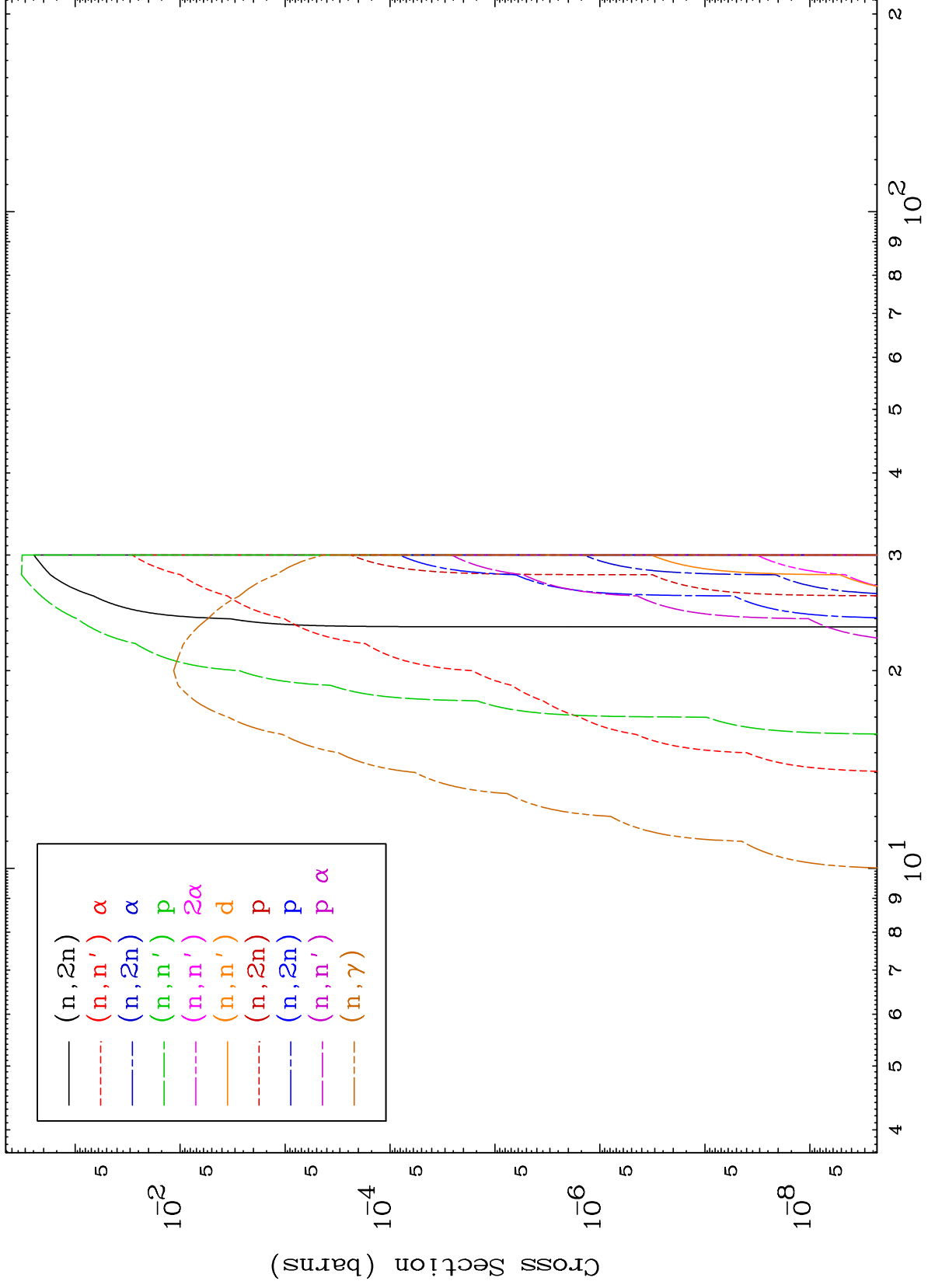
Incident Energy (MeV)

$^{71}\text{Lu-162n}$

MAT 7088

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

$^{71}\text{Lu}-162\text{n}$



2

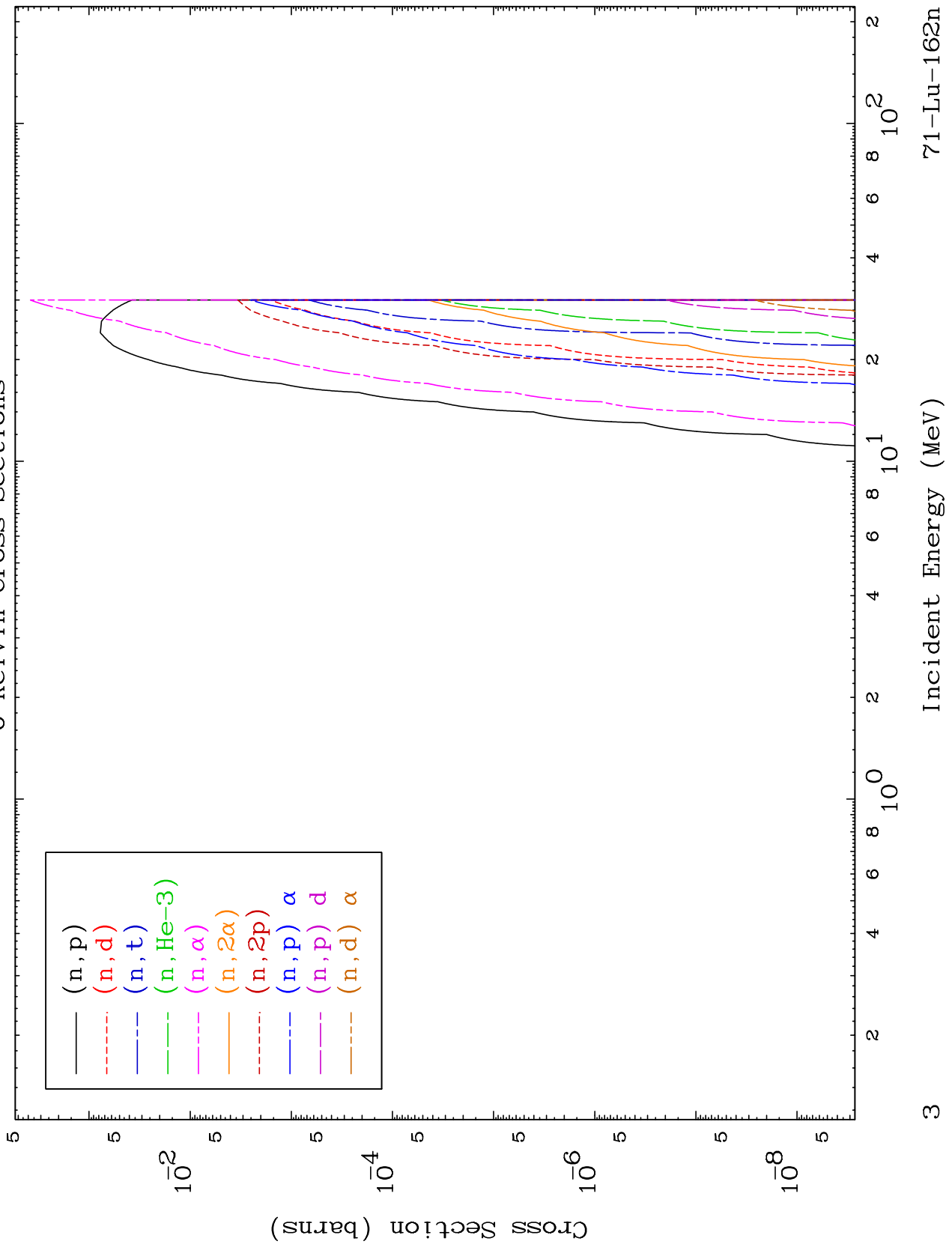
Incident Energy (MeV)

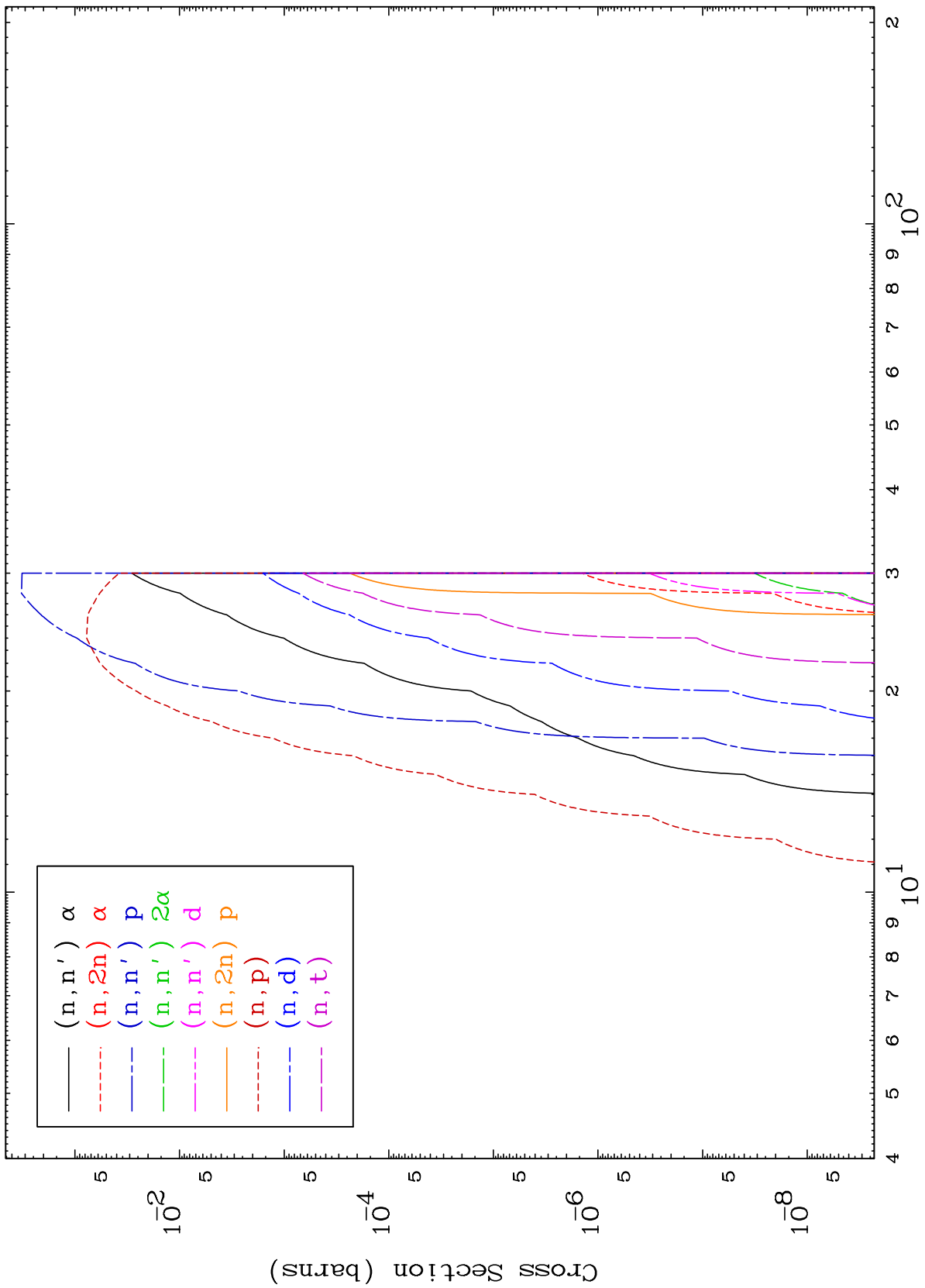
$^{71}\text{Lu}-162\text{n}$

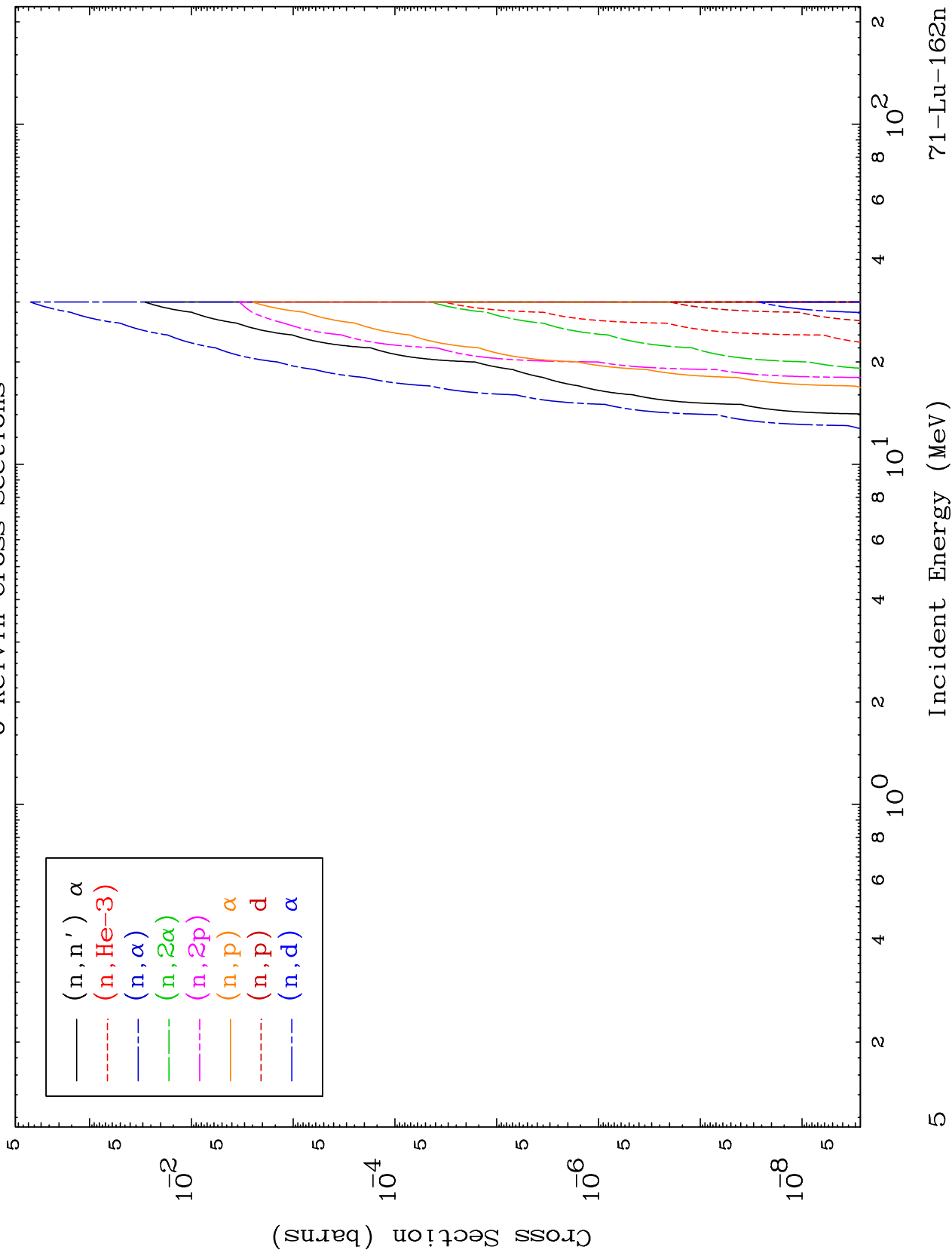
MAT 7088

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

$^{71}\text{Lu-162n}$





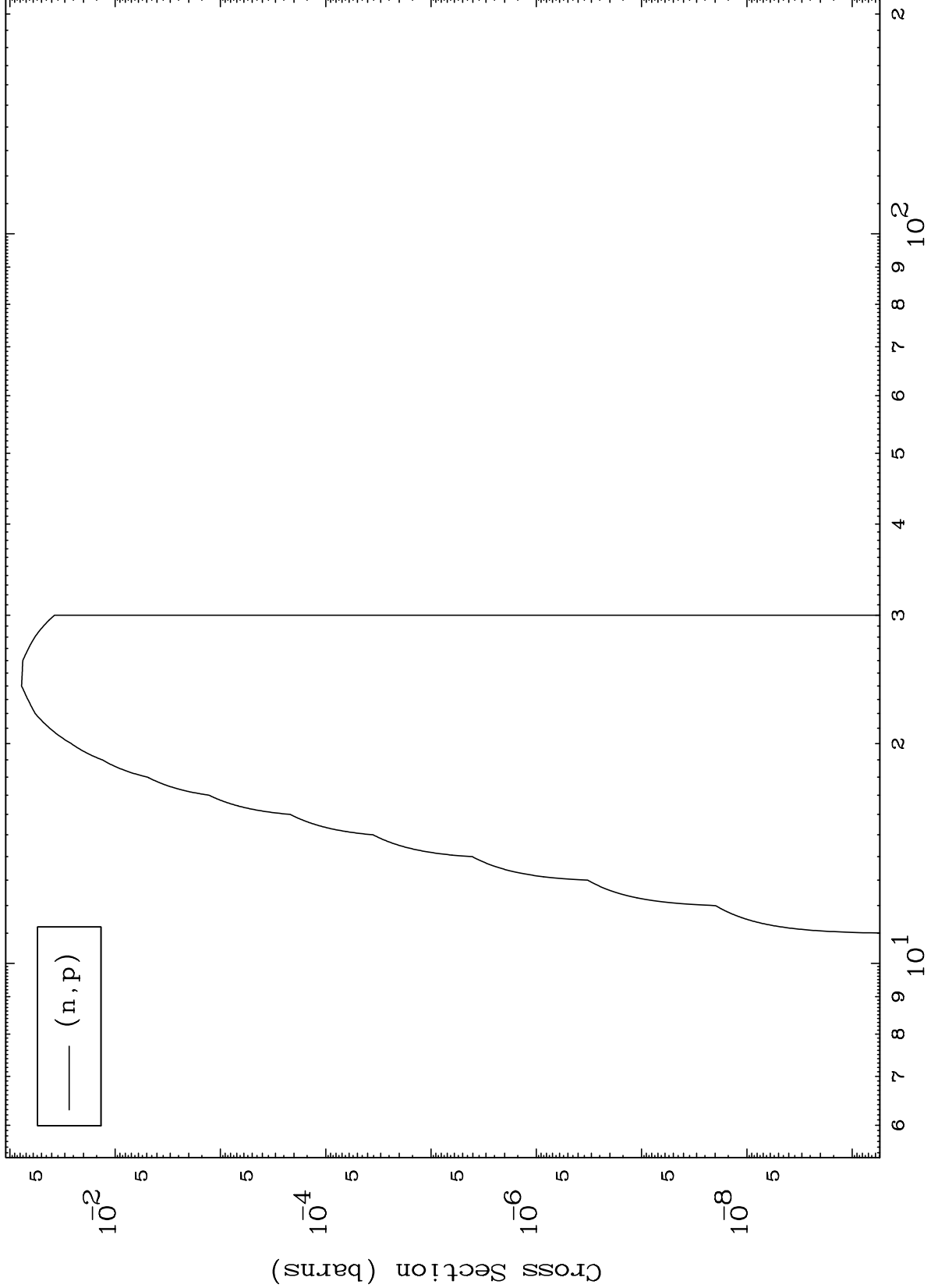


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( $\alpha, p$ ) Levels

$^{71}\text{Lu}-162\text{n}$

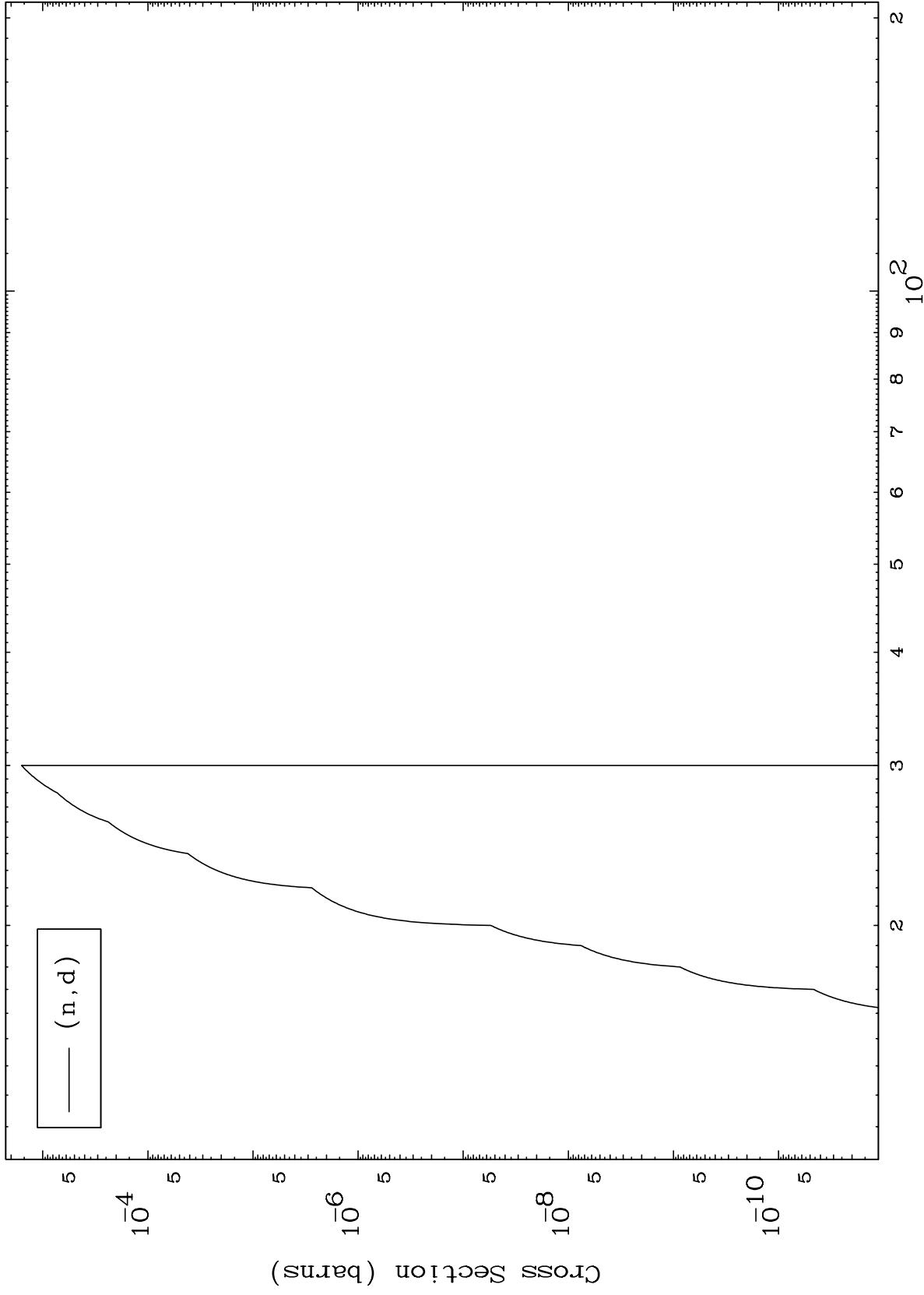
0 Kelvin Cross Sections



6

Incident Energy (MeV)

$^{71}\text{Lu}-162\text{n}$

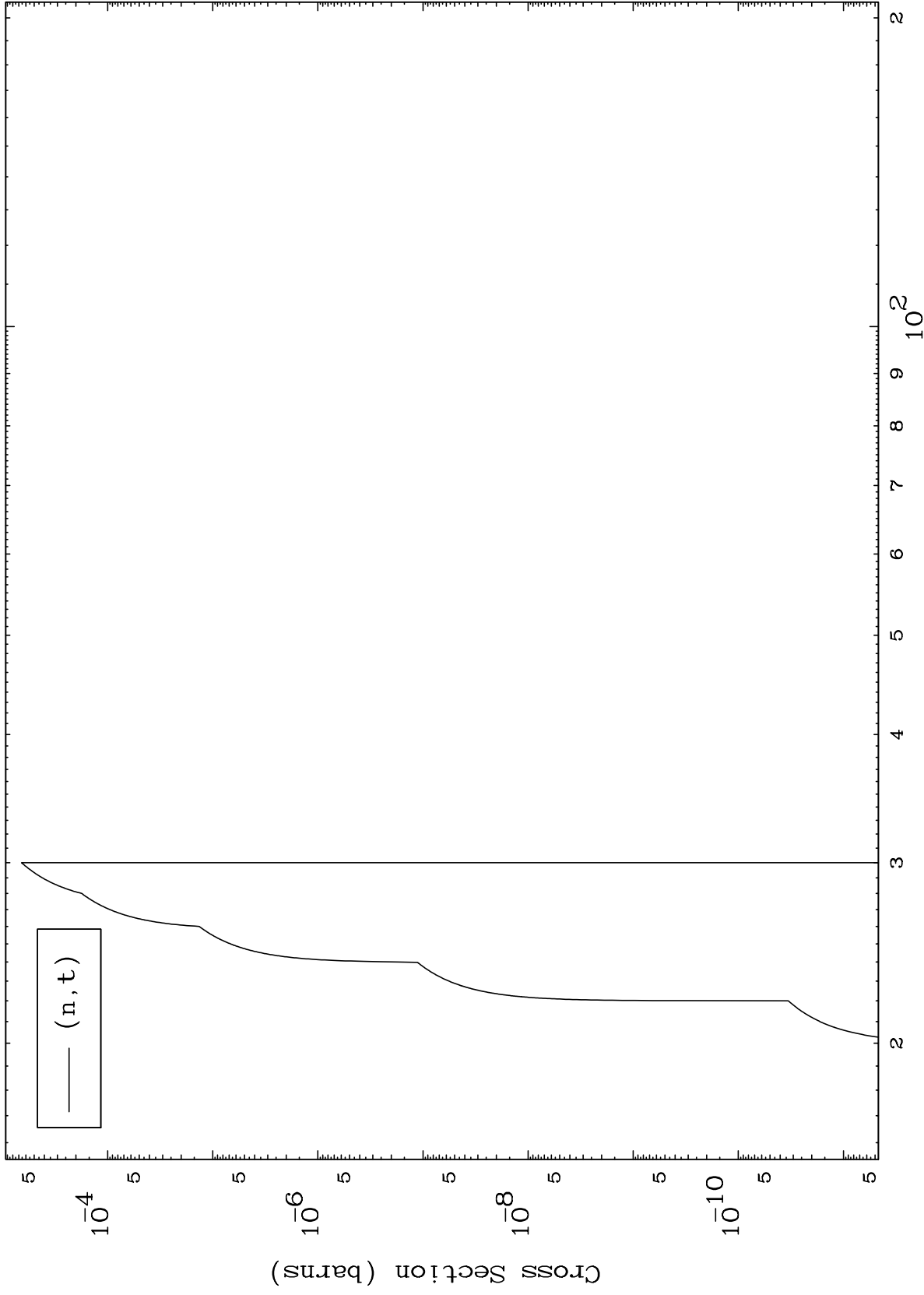




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( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

$^{71}\text{Lu}-162\text{n}$



8

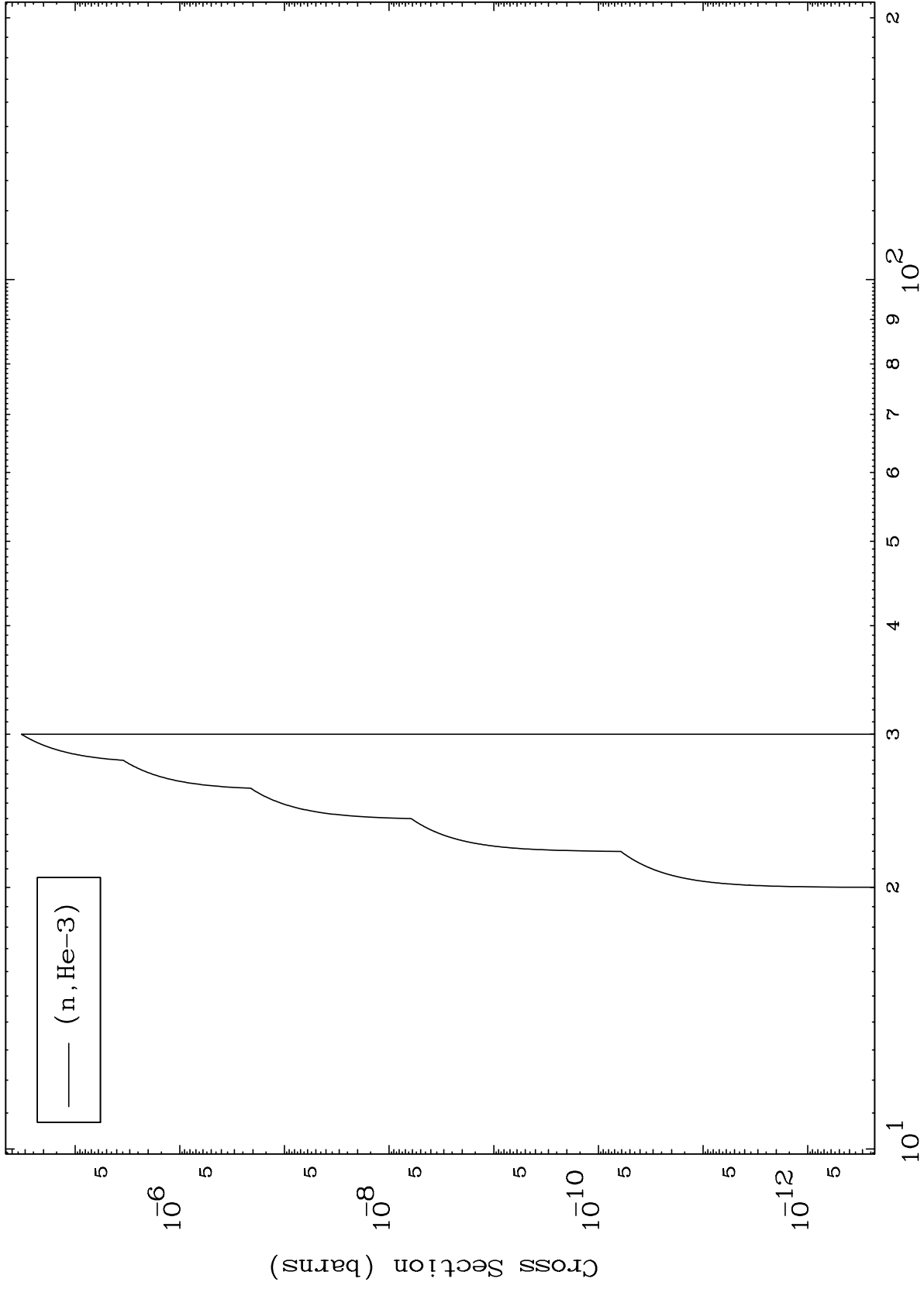
Incident Energy (MeV)

$^{71}\text{Lu}-162\text{n}$

MAT 7088

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

71-Lu-162n



Incident Energy (MeV)

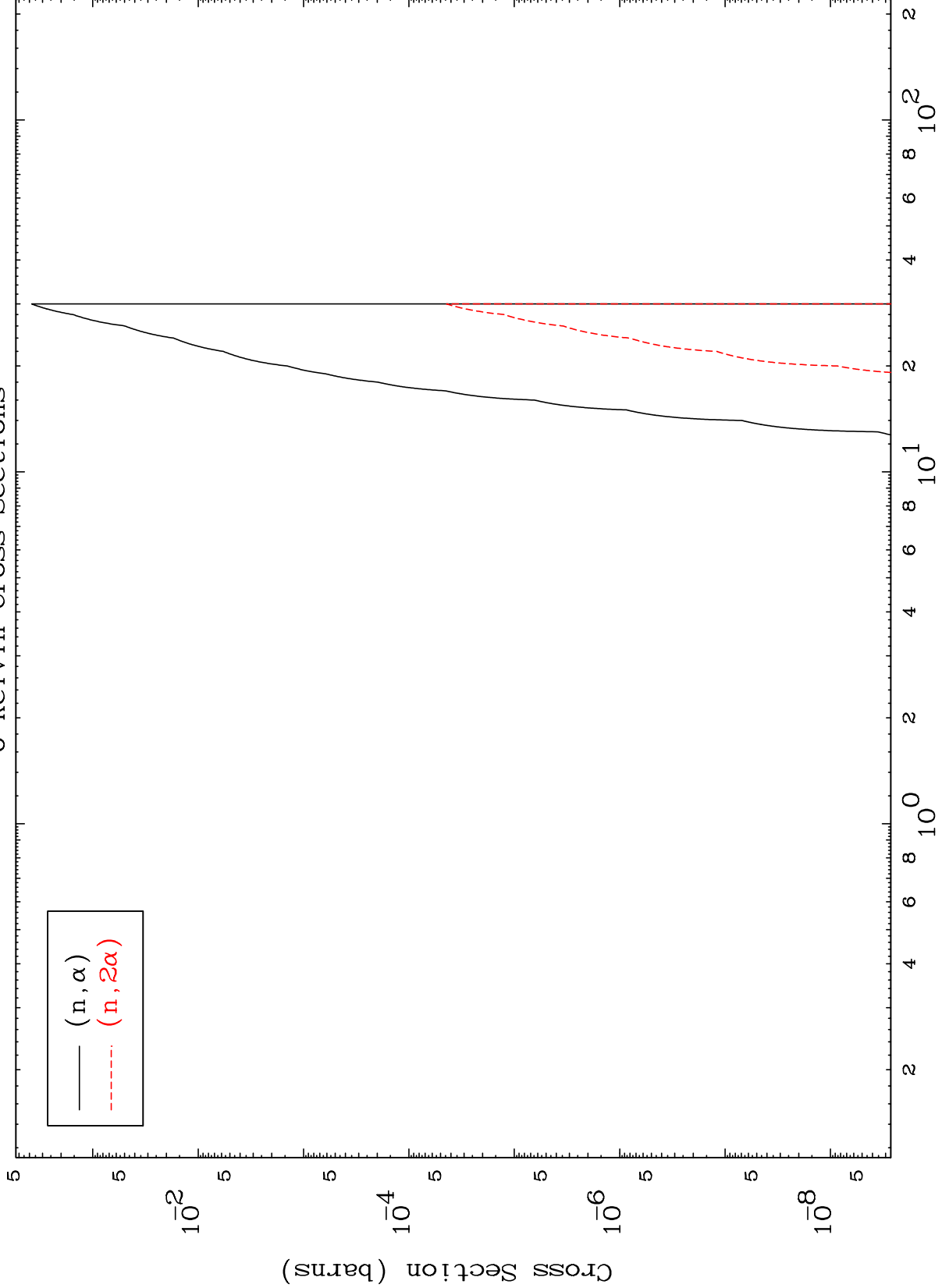
71-Lu-162n

MAT 7088

( $\alpha, \alpha$ ) Levels

$^{71}\text{Lu-162n}$

0 Kelvin Cross Sections



10

Incident Energy (MeV)

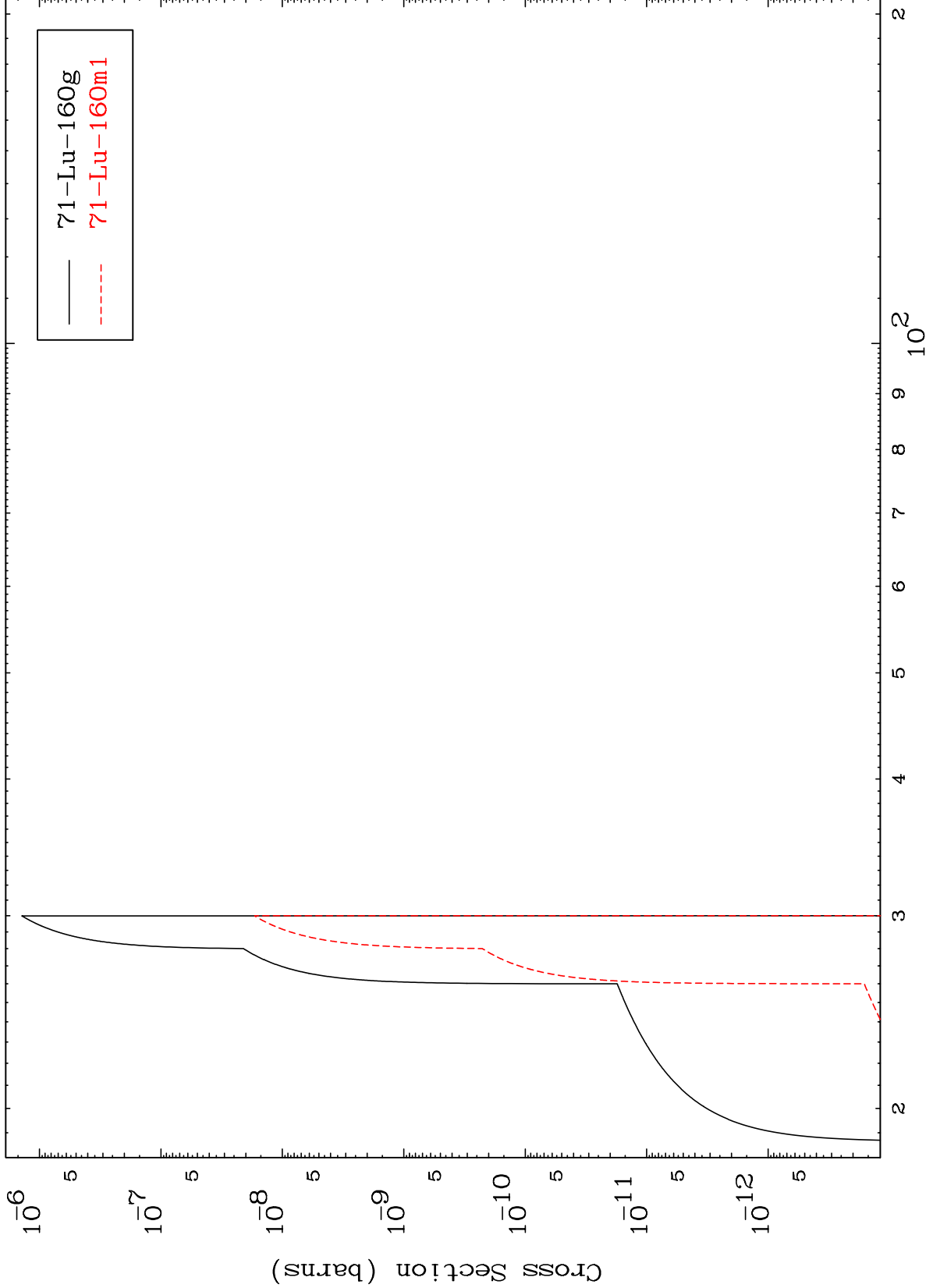
$^{71}\text{Lu-162n}$

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$(n,2n) \alpha$

$^{71}\text{Lu}-162n$

Radionuclide Production Cross Section



11

Incident Energy (MeV)

$^{71}\text{Lu}-162n$

MAT 7088

$^{71}\text{Lu}-162\text{n}$

