

Program EVALPLOT  
(Version 2021-1)

by

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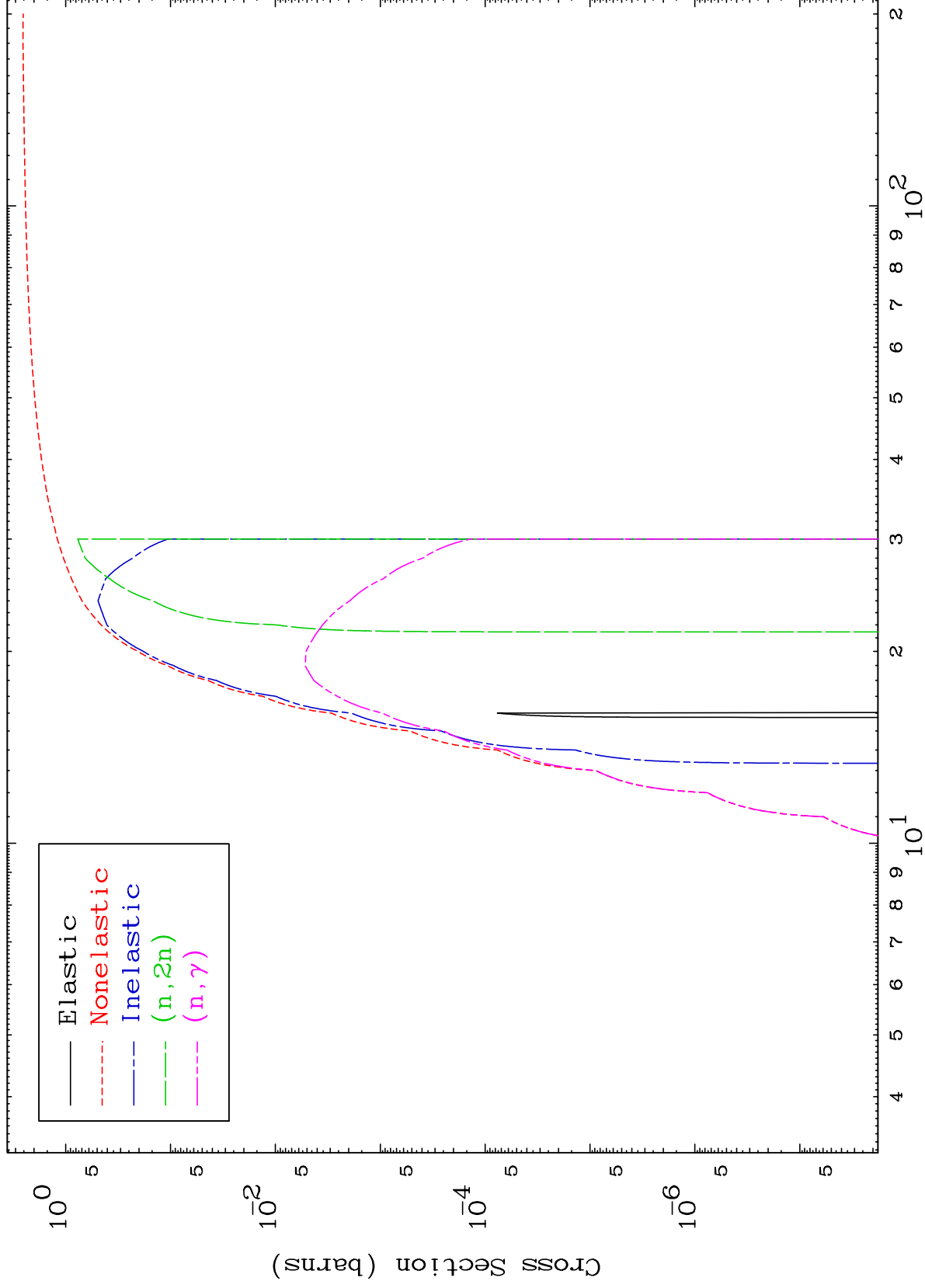
Press Mouse Button to Start

MAT 7101

0 Kelvin

$\alpha$  Major Cross Sections

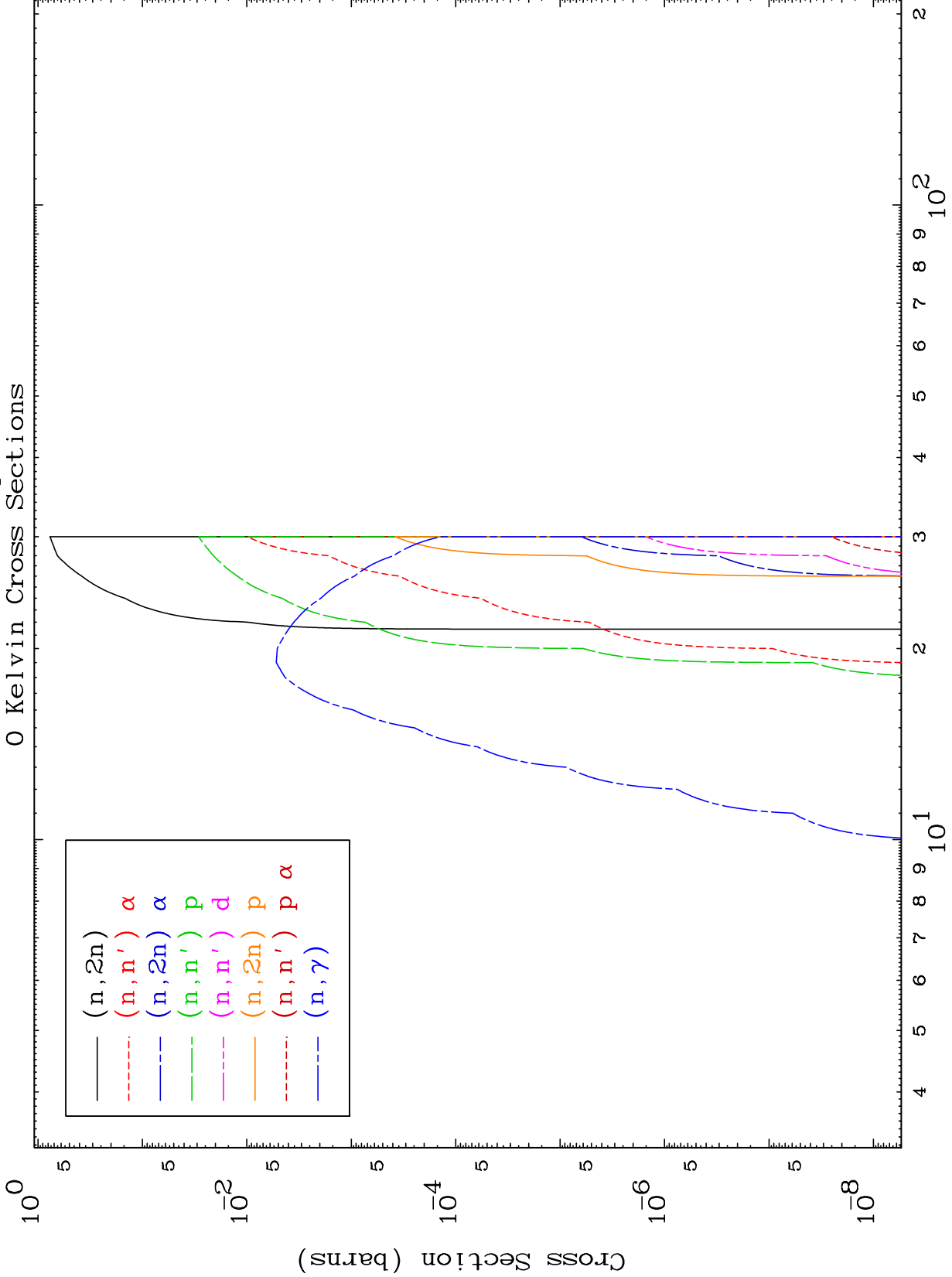
71-Lu-167



1

Incident Energy (MeV)

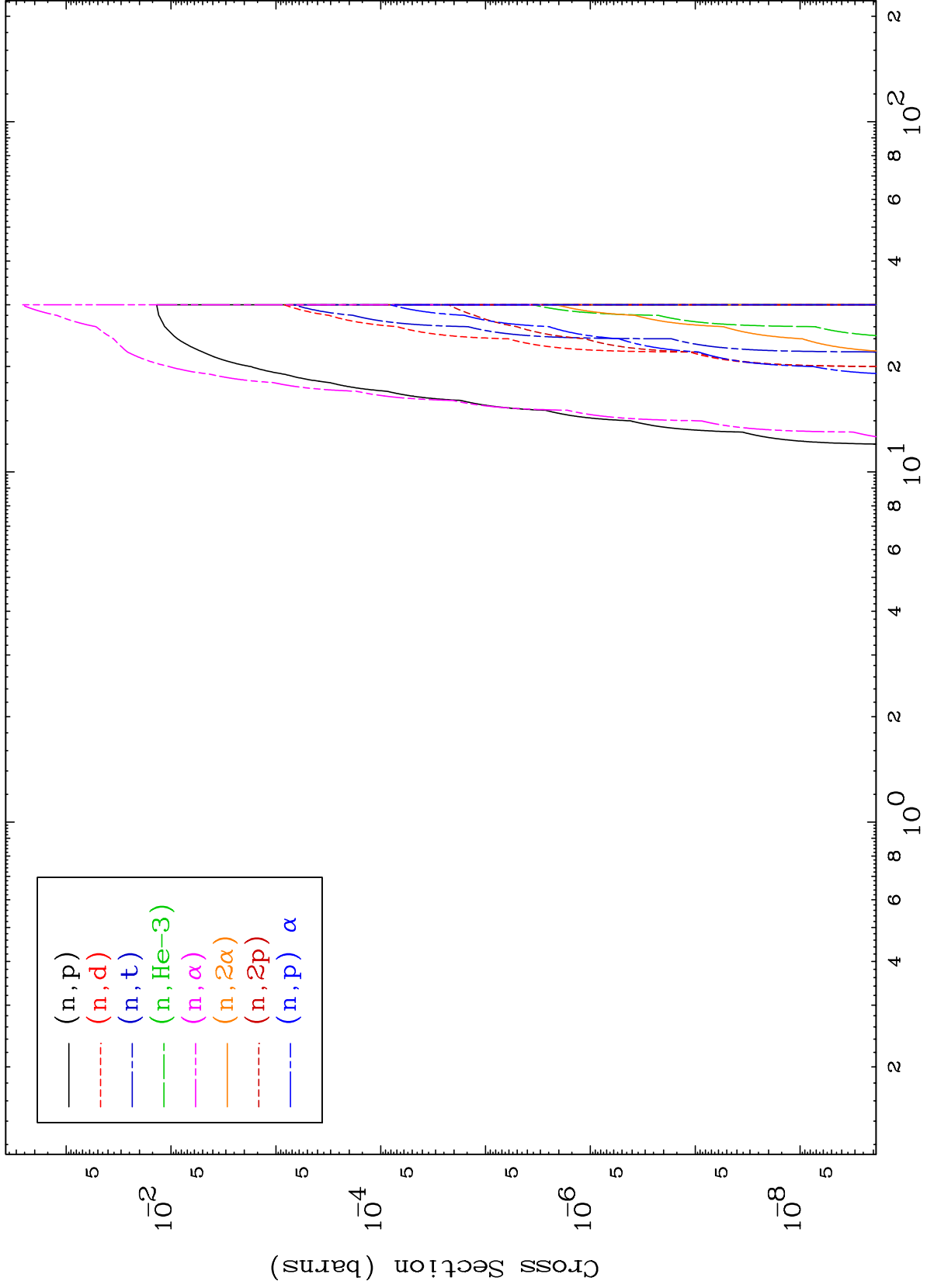
71-Lu-167

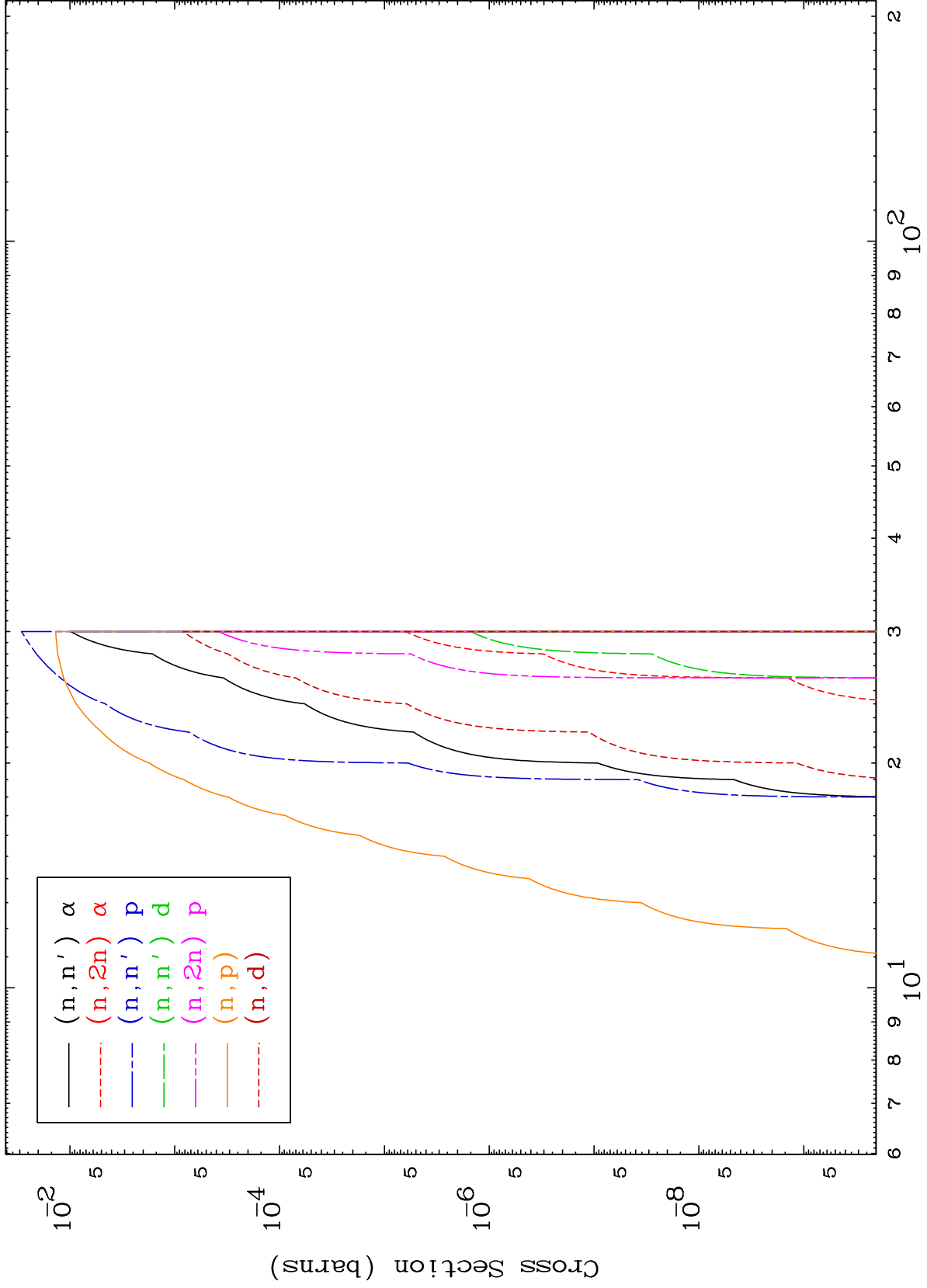


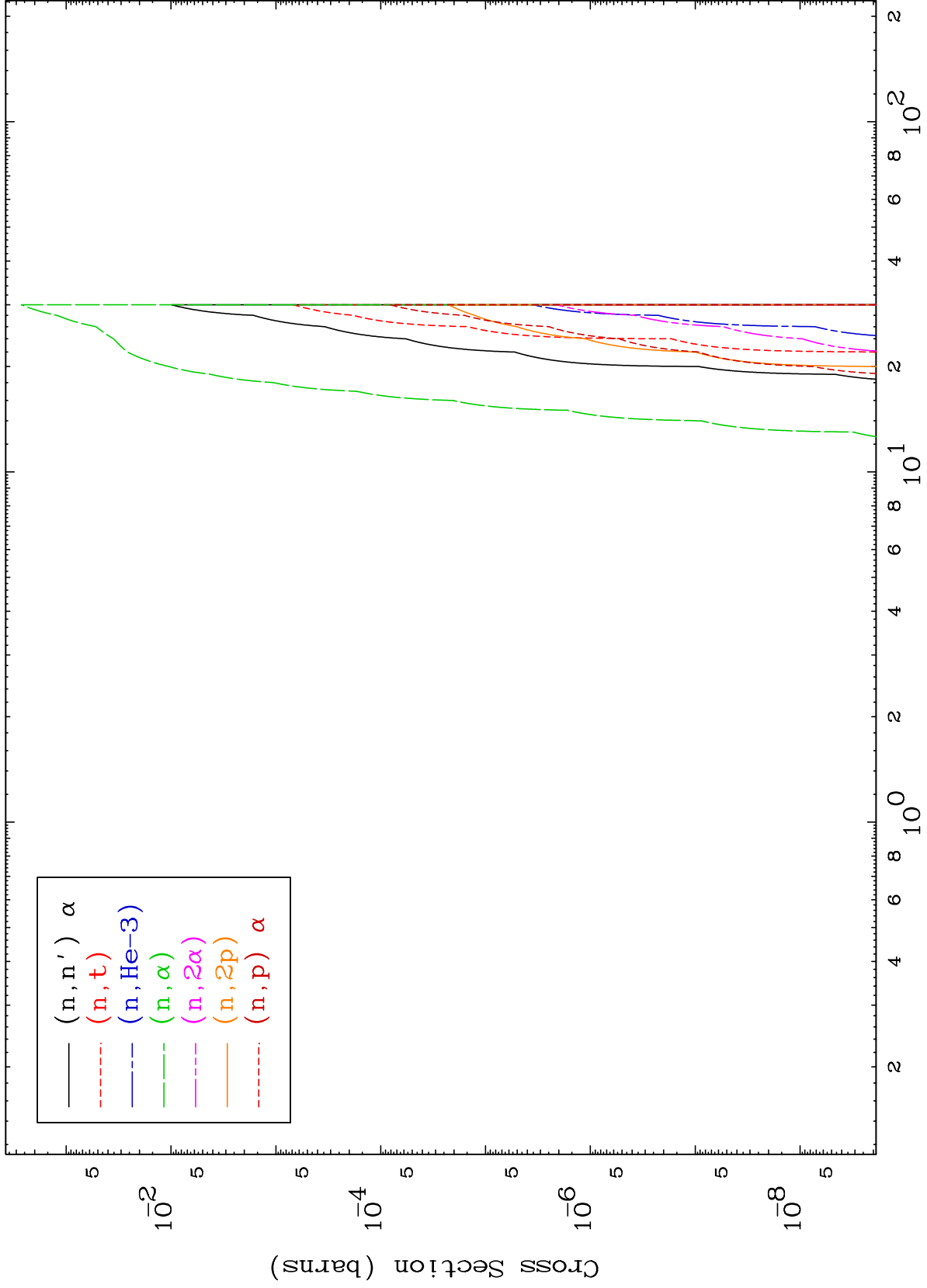
MAT 7101

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

71-Lu-167



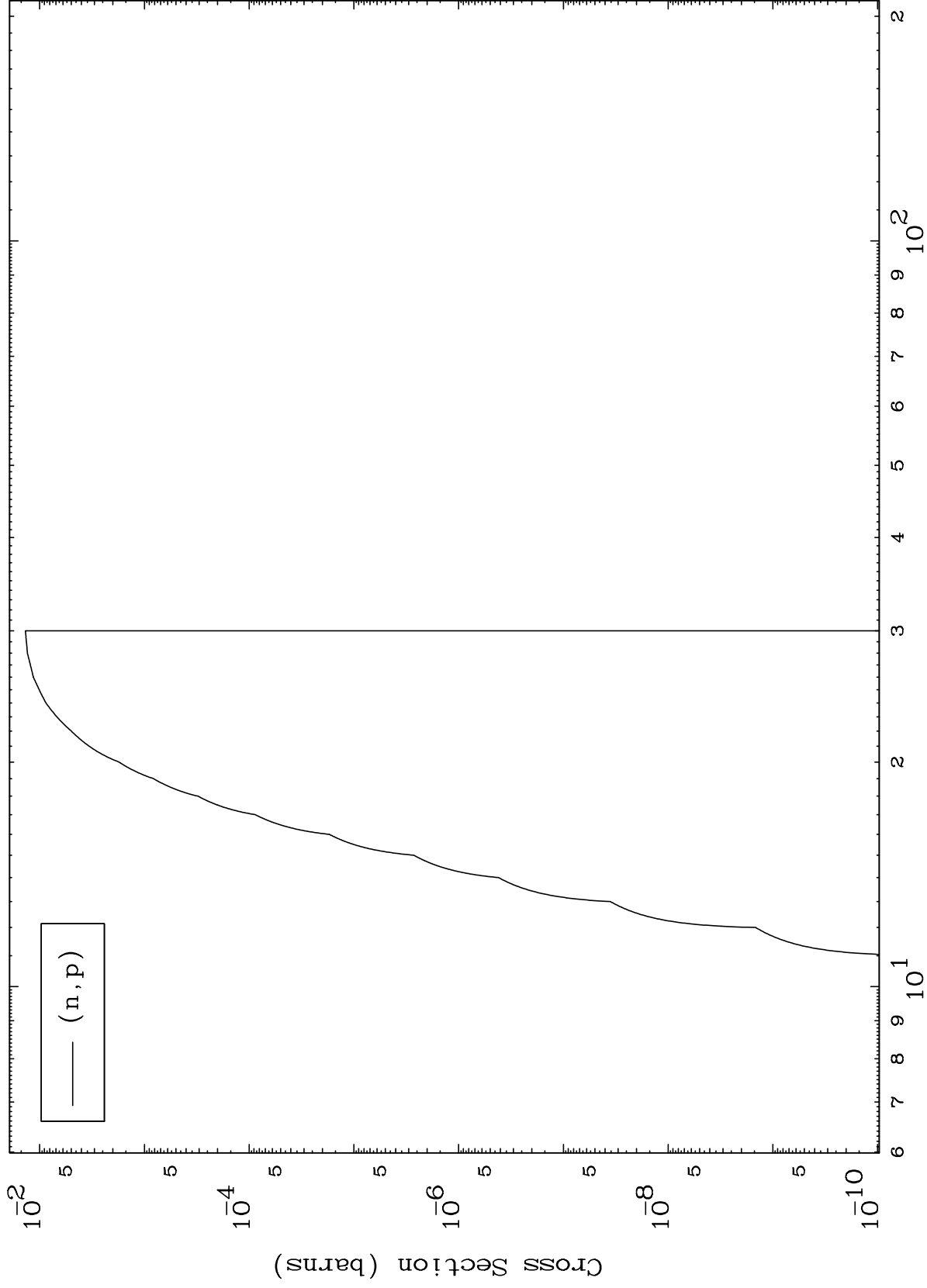




MAT 7101

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

71-Lu-167



6

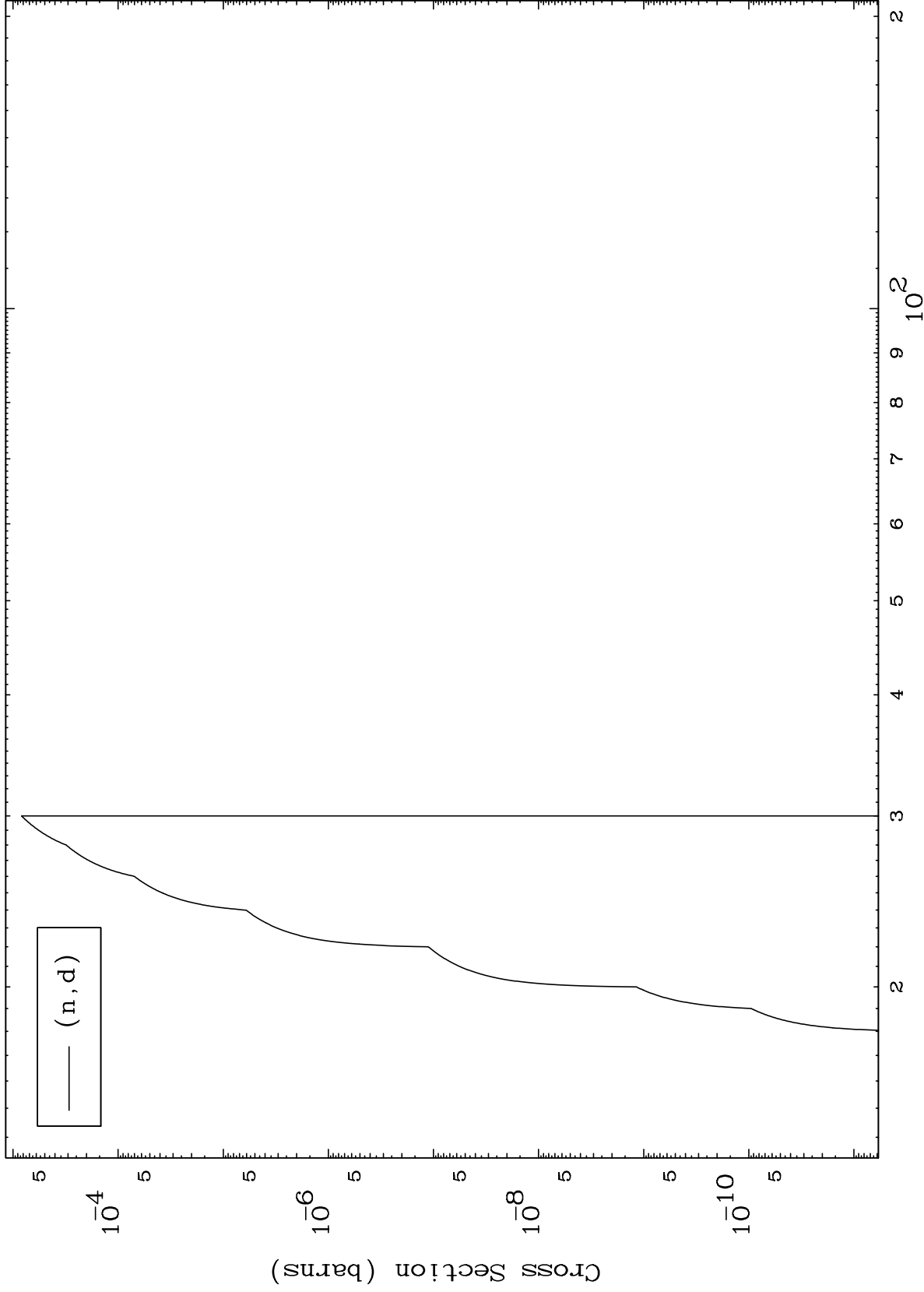
Incident Energy (MeV)

71-Lu-167

MAT 7101

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

71-Lu-167

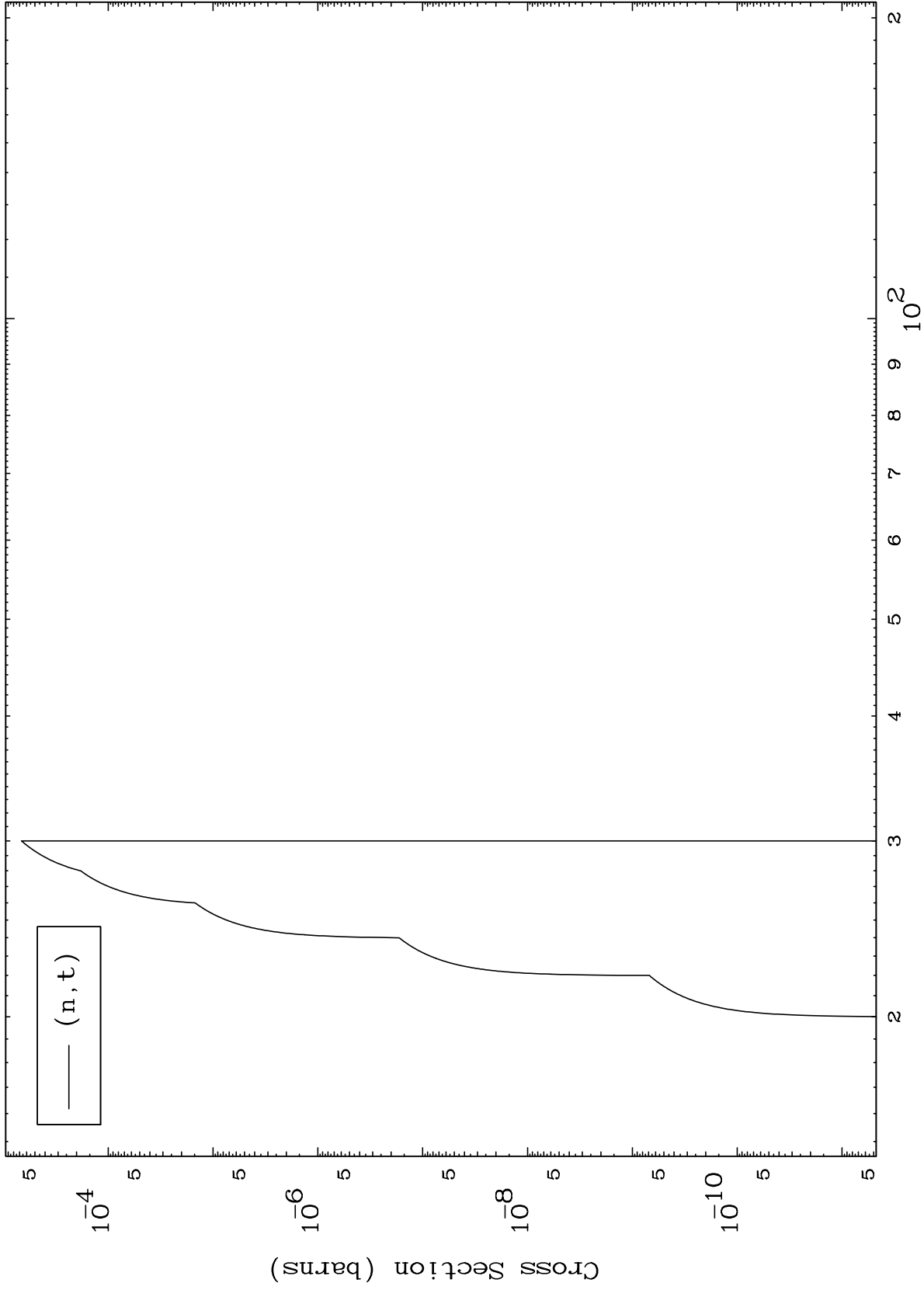


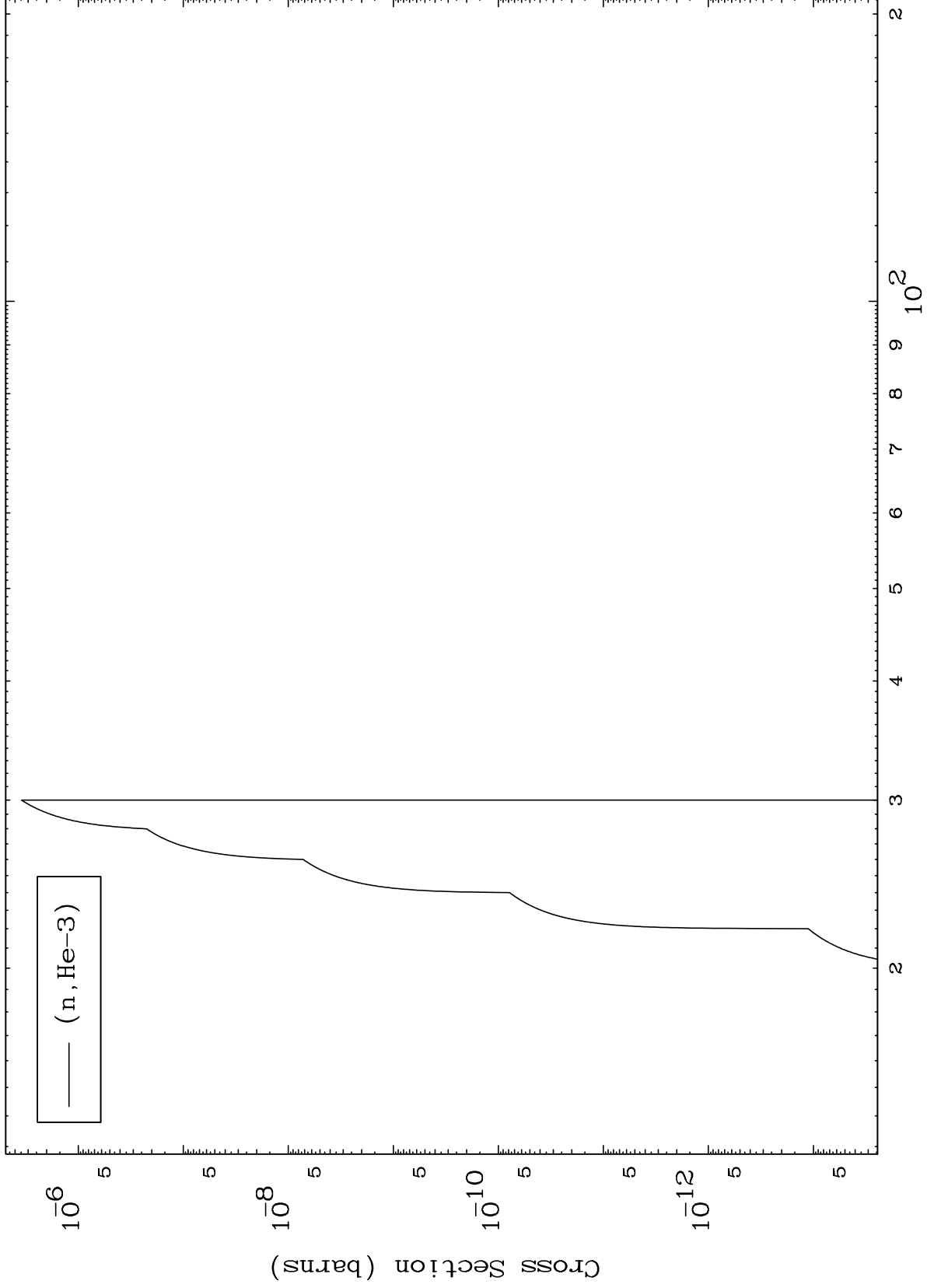
7

Incident Energy (MeV)

71-Lu-167





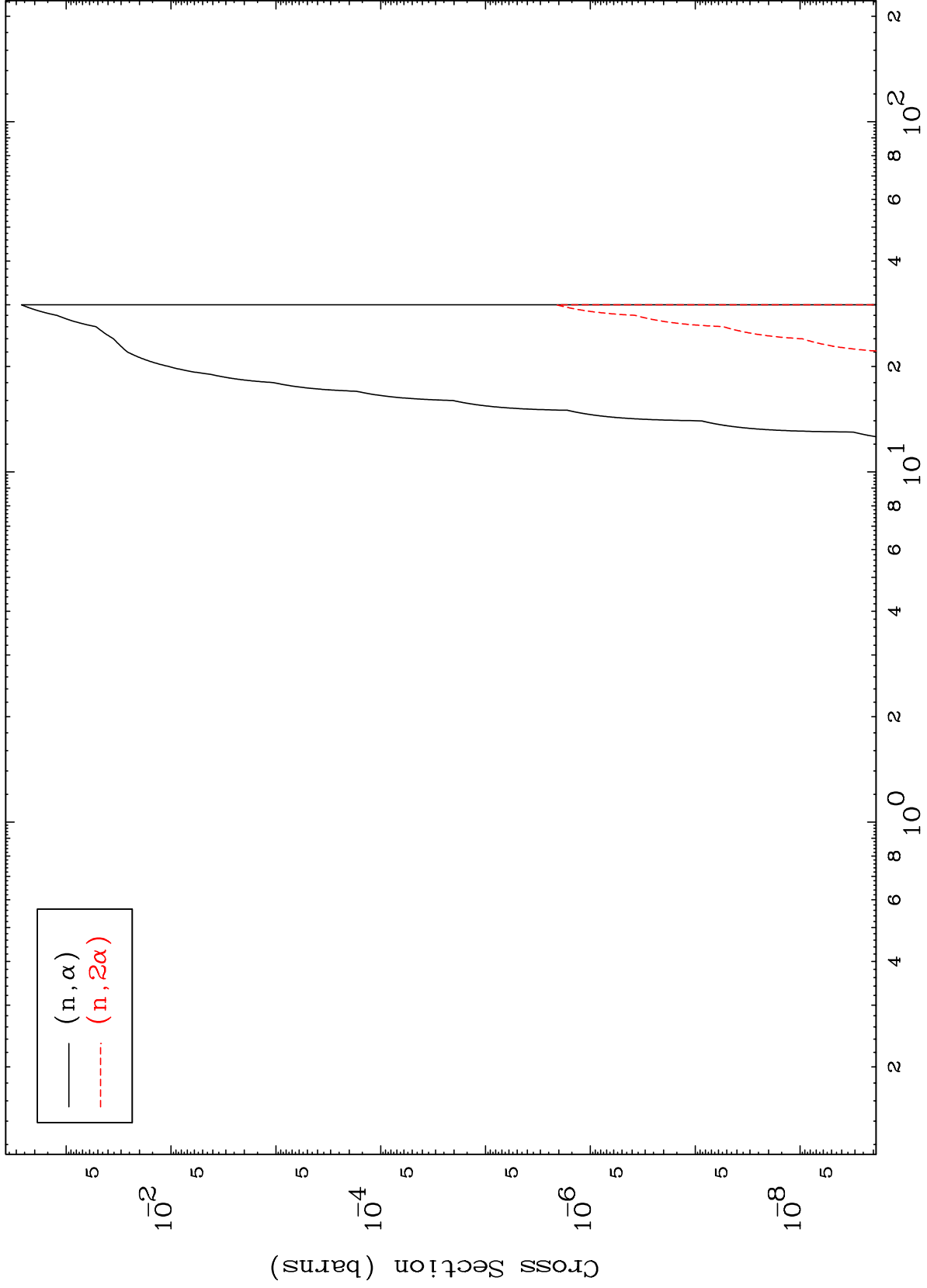


MAT 7101

( $\alpha, \alpha$ ) Levels

71-Lu-167

0 Kelvin Cross Sections



— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

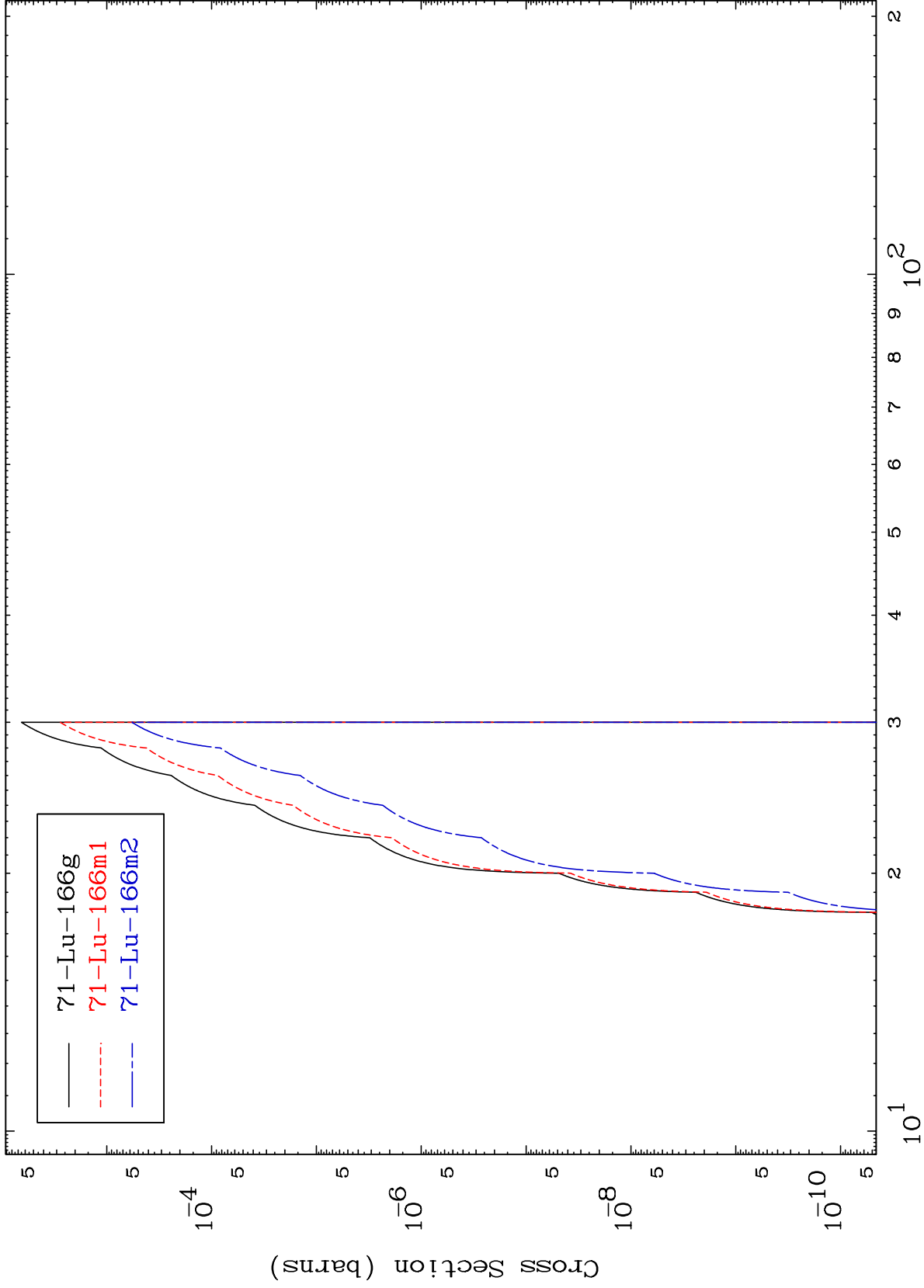
71-Lu-167

MAT 7101

$(n, n') \alpha$

$^{71}\text{Lu-167}$

Radionuclide Production Cross Section



11

Incident Energy (MeV)

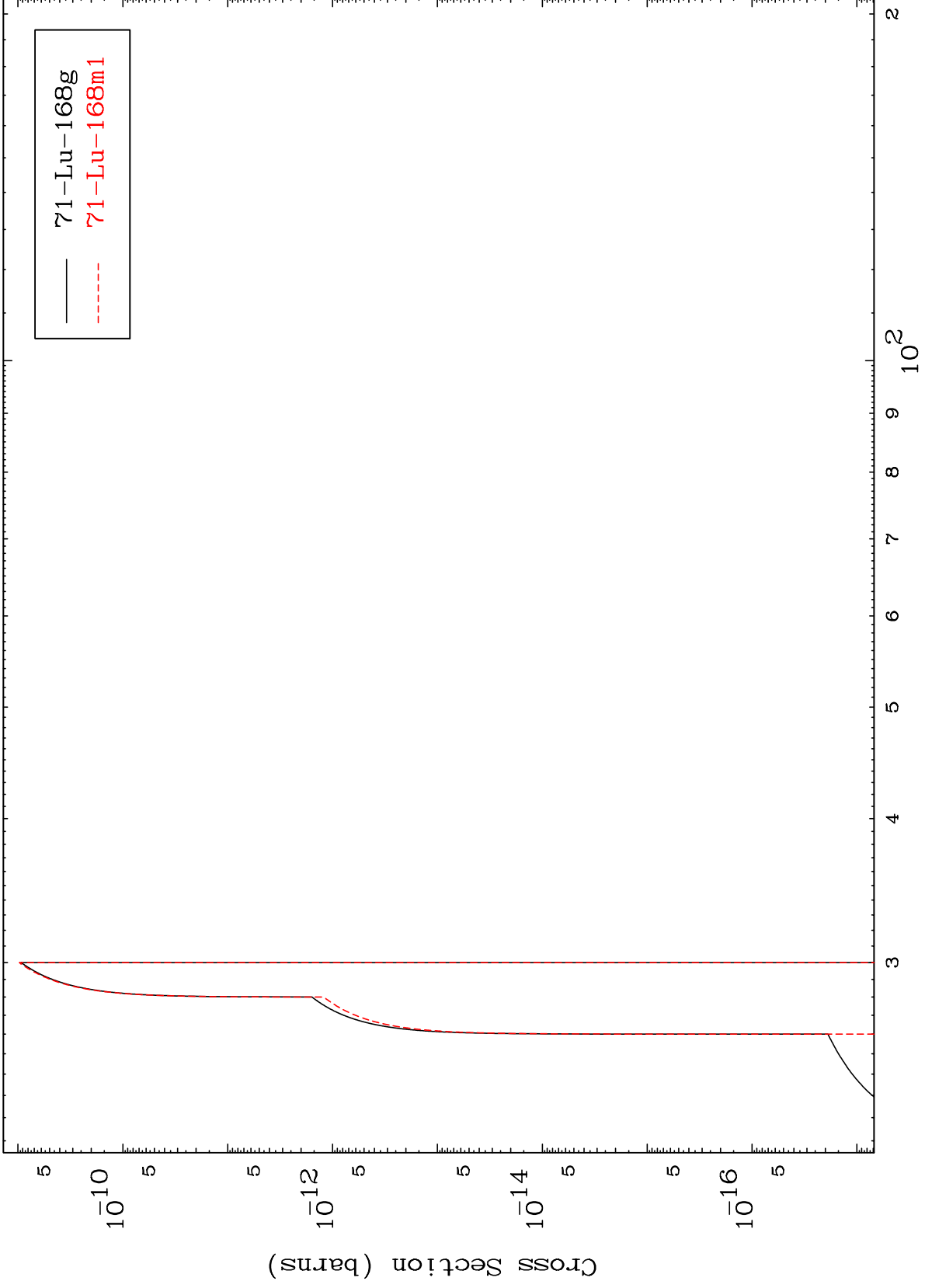
$^{71}\text{Lu-167}$

MAT 7101

(n,2n) p

71-Lu-167

Radionuclide Production Cross Section



12

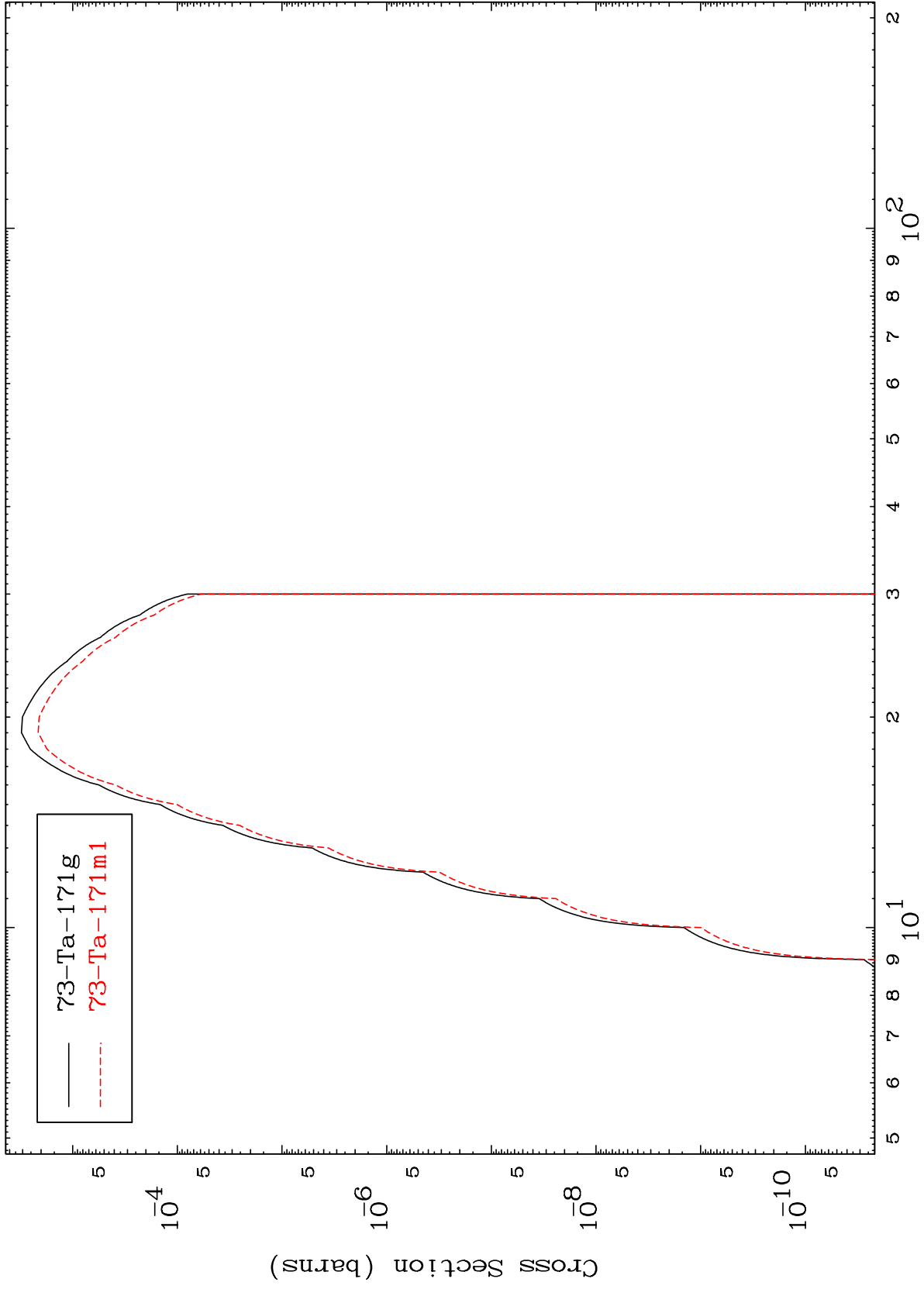
Incident Energy (MeV)

71-Lu-167

MAT 7101

71-Lu-167

(n,γ)  
Radionuclide Production Cross Section



— 73-Ta-171g  
- - - 73-Ta-171m1

13

Incident Energy (MeV)

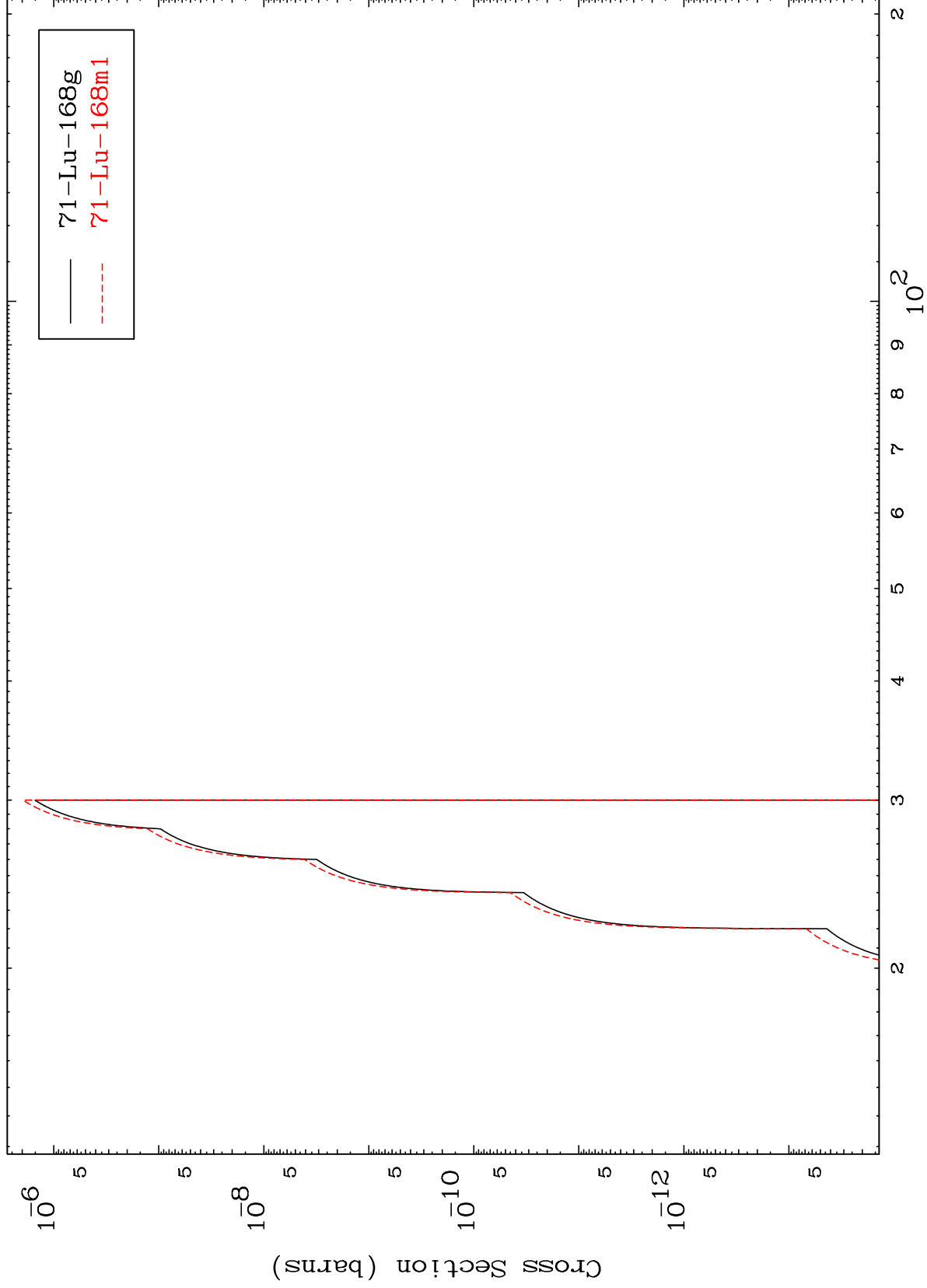
71-Lu-167

MAT 7101

(n,He-3)

71-Lu-167

Radionuclide Production Cross Section



14

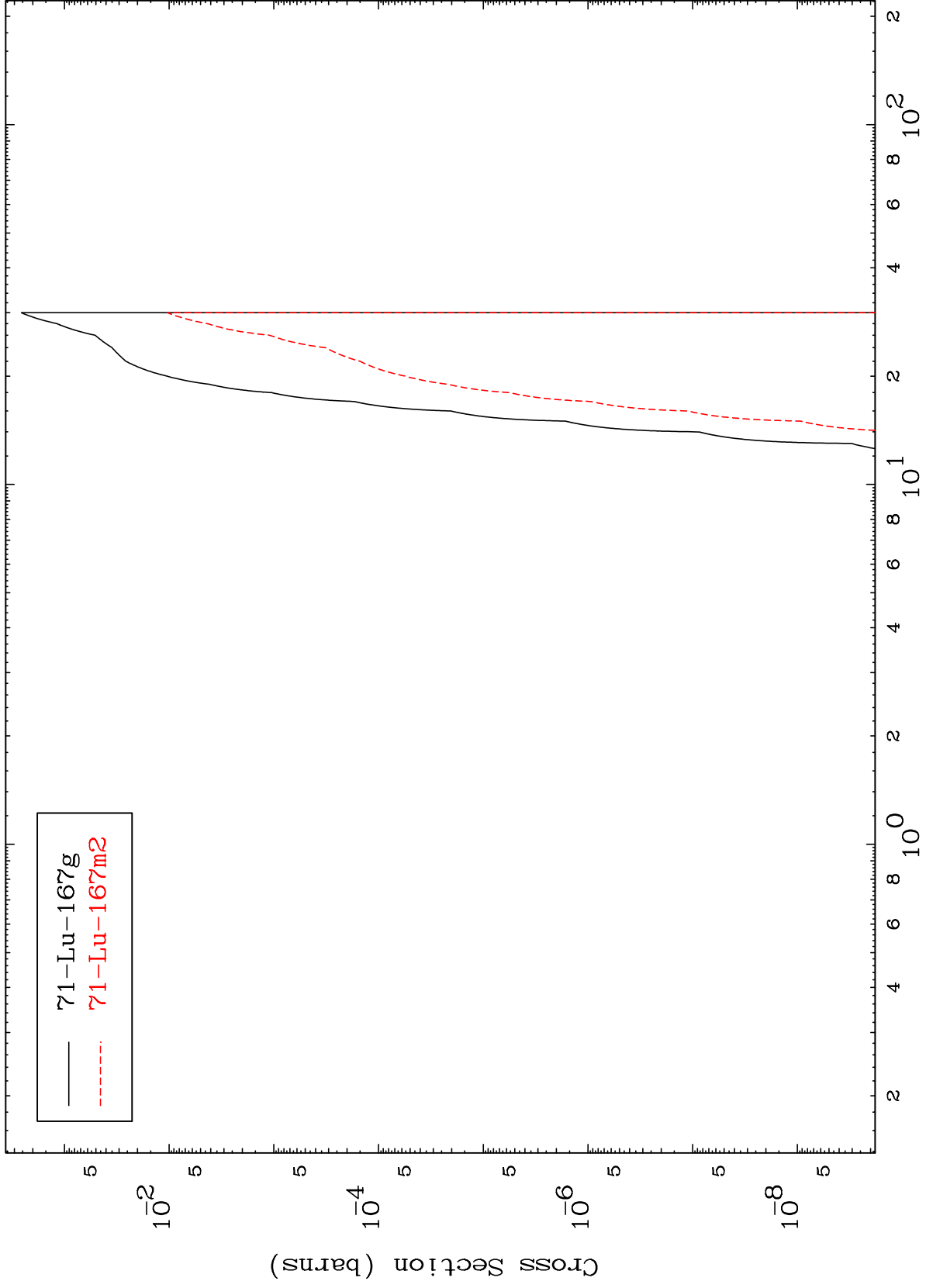
Incident Energy (MeV)

71-Lu-167

MAT 7101

71-Lu-167

Radionuclide Production Cross Section  
(n,  $\alpha$ )



15

Incident Energy (MeV)

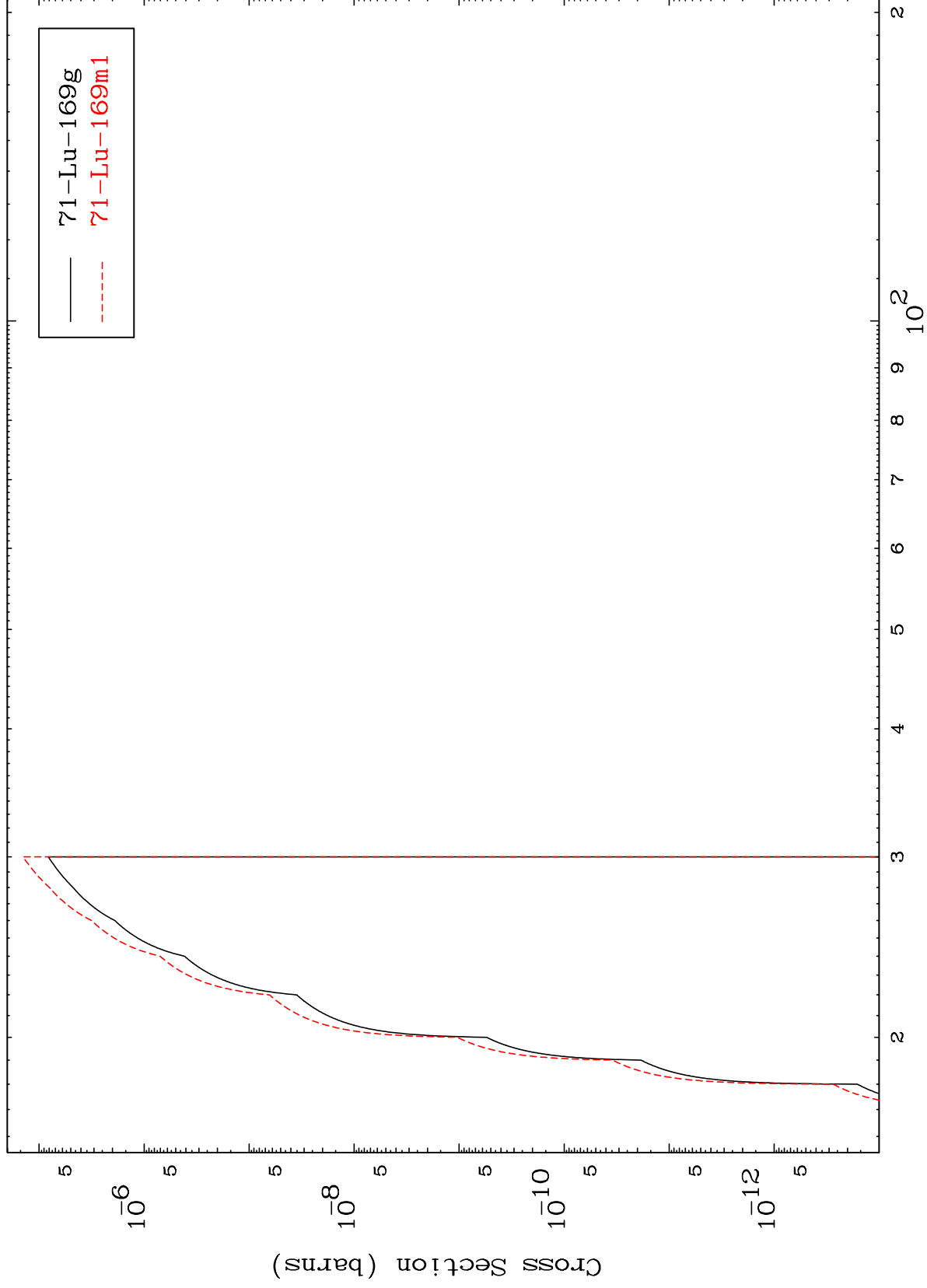
71-Lu-167



MAT 7101

71-Lu-167

(n,2p)  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

71-Lu-167