

Program EVALPLOT  
(Version 2021-1)

by

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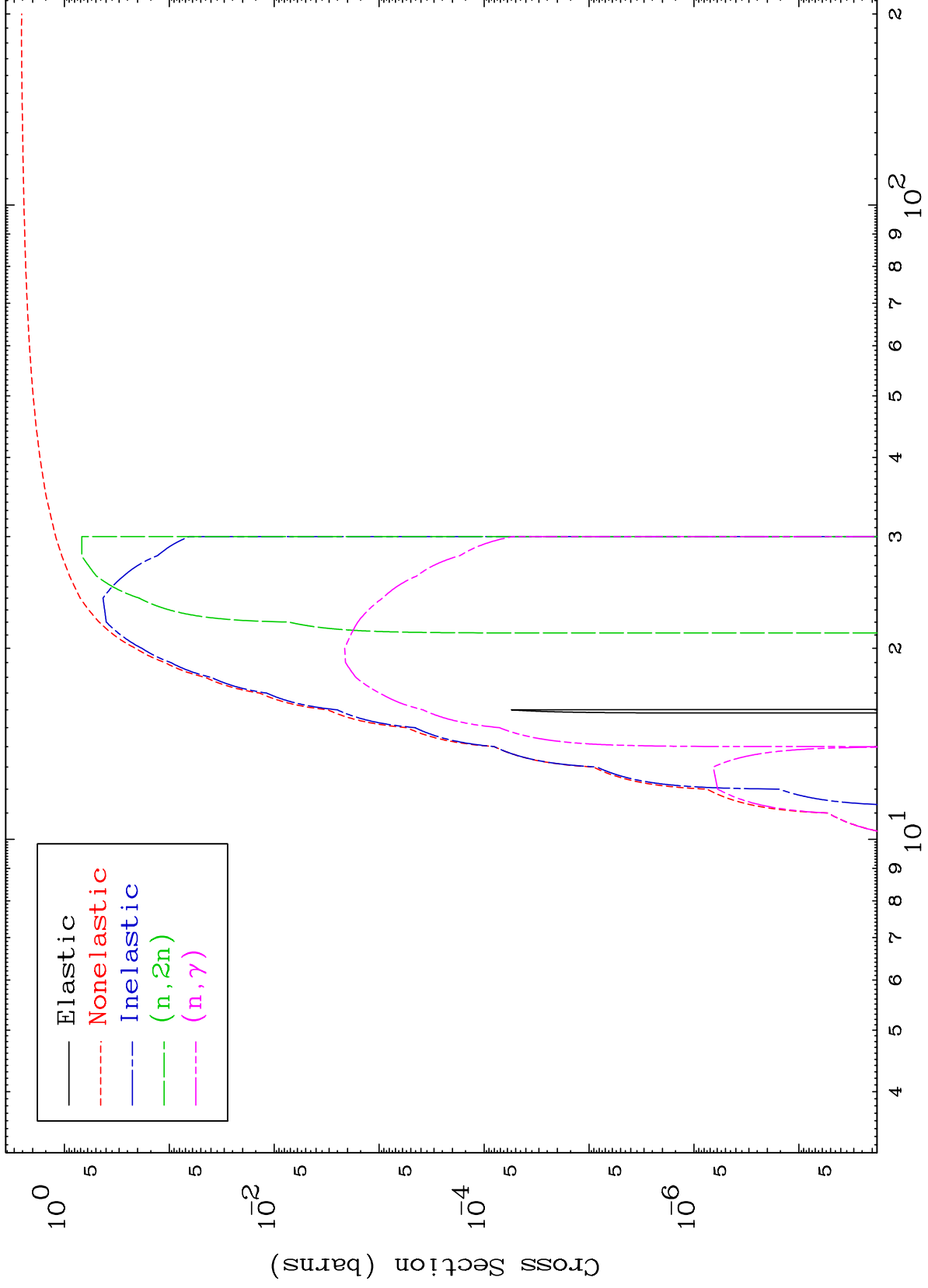
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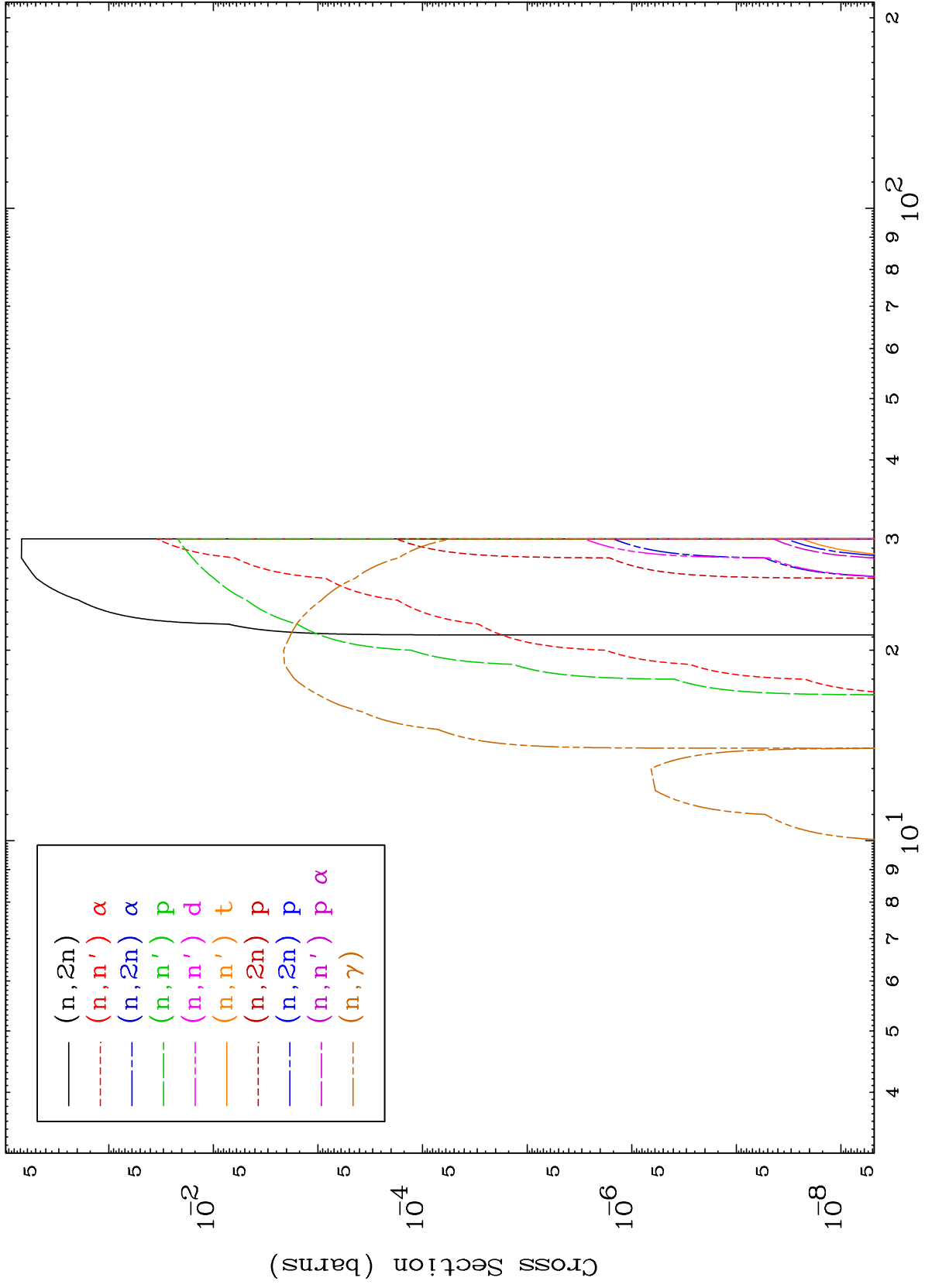
Press Mouse Button to Start

MAT 7104

0 Kelvin  $\alpha$  Major Cross Sections

71-Lu-168

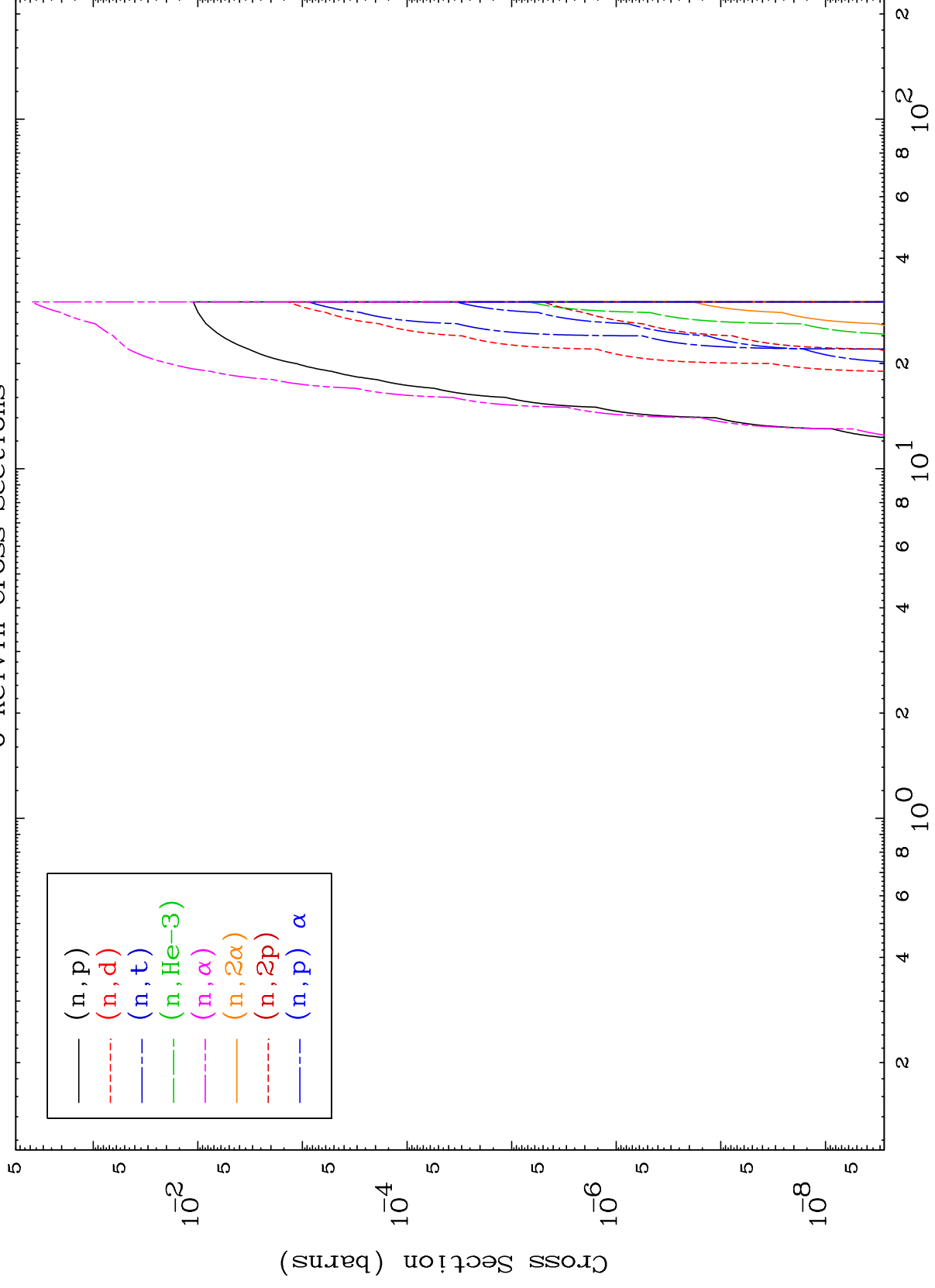


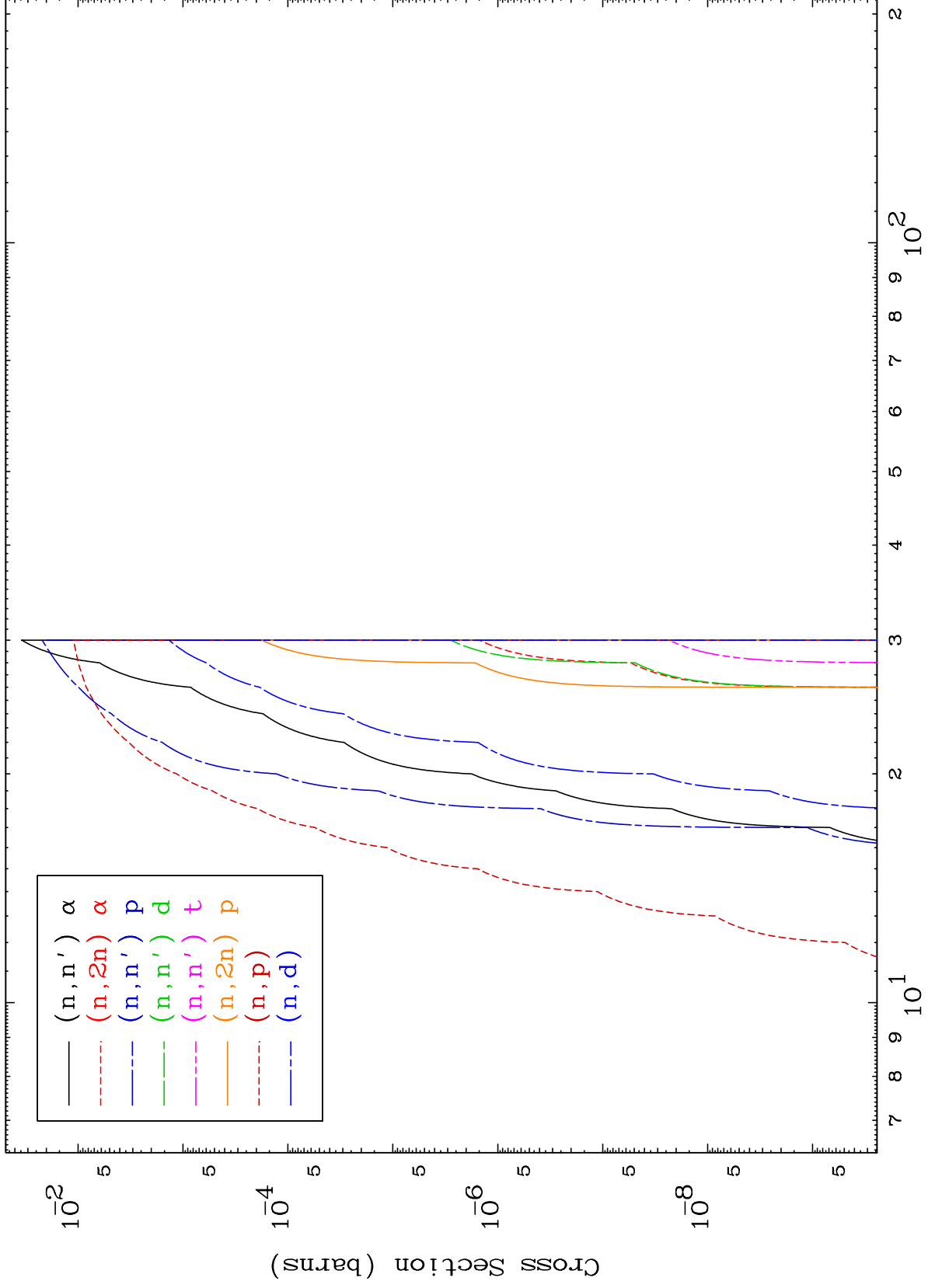


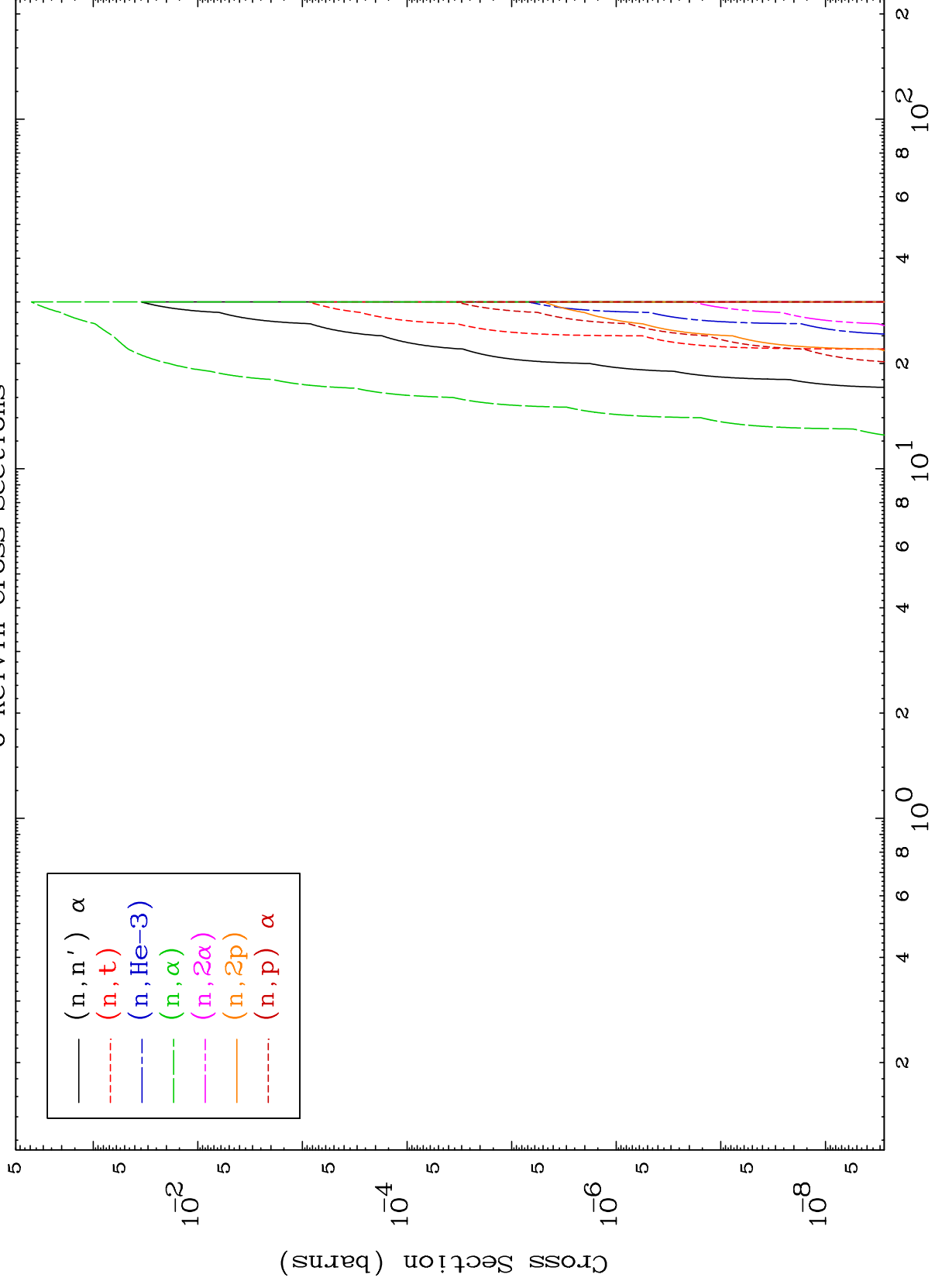
MAT 7104

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

71-Lu-168





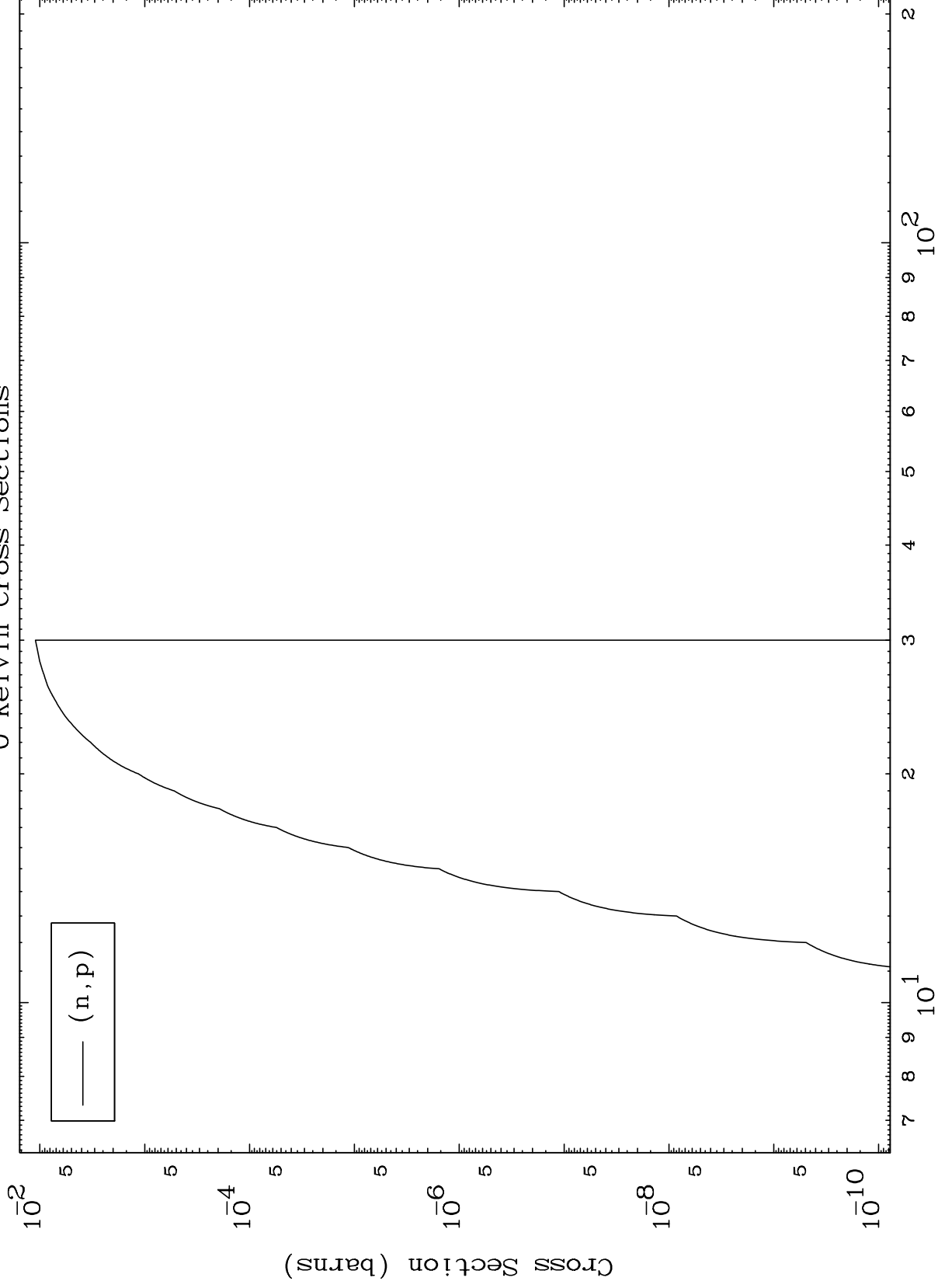


MAT 7104

( $\alpha, p$ ) Levels

71-Lu-168

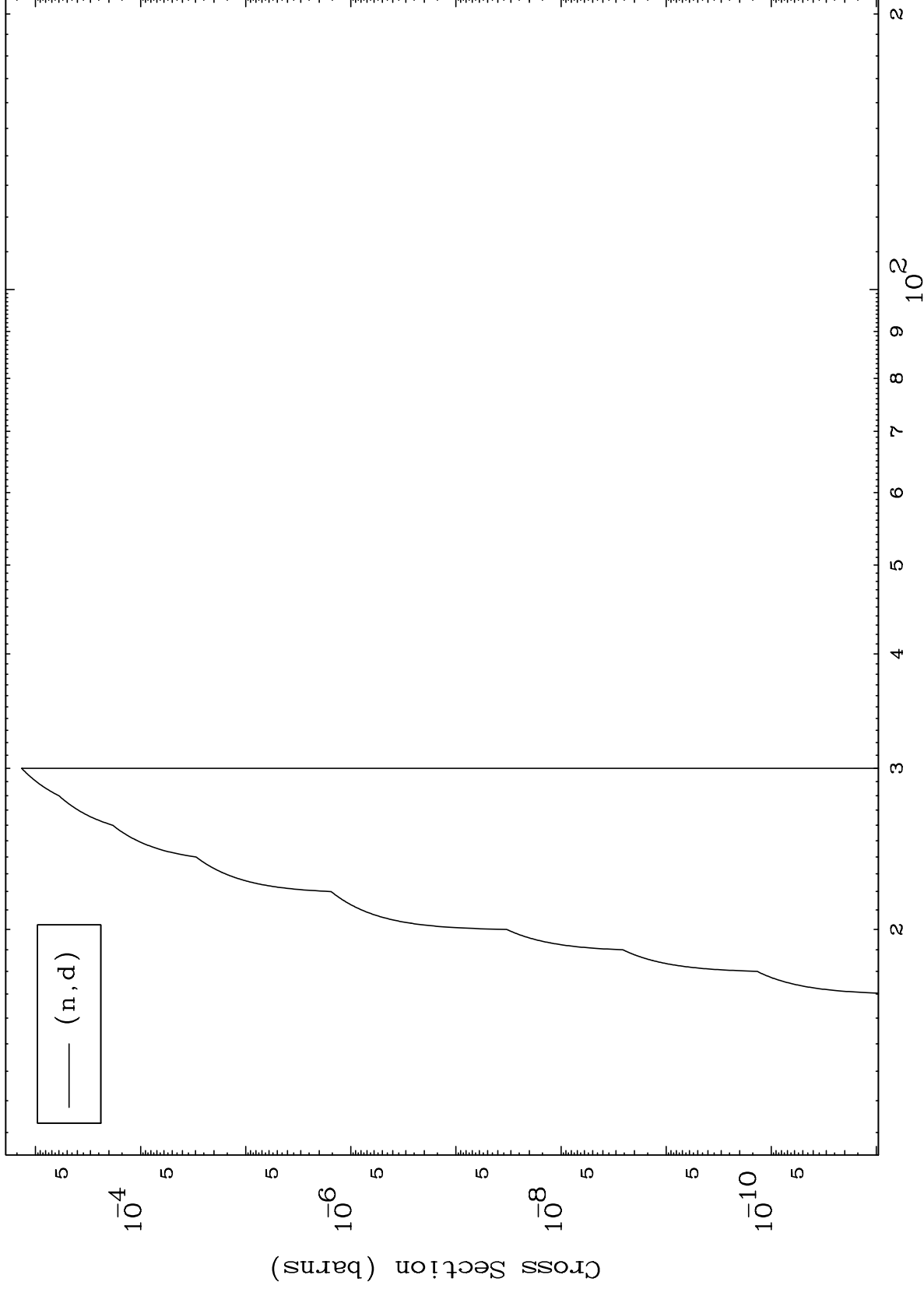
0 Kelvin Cross Sections



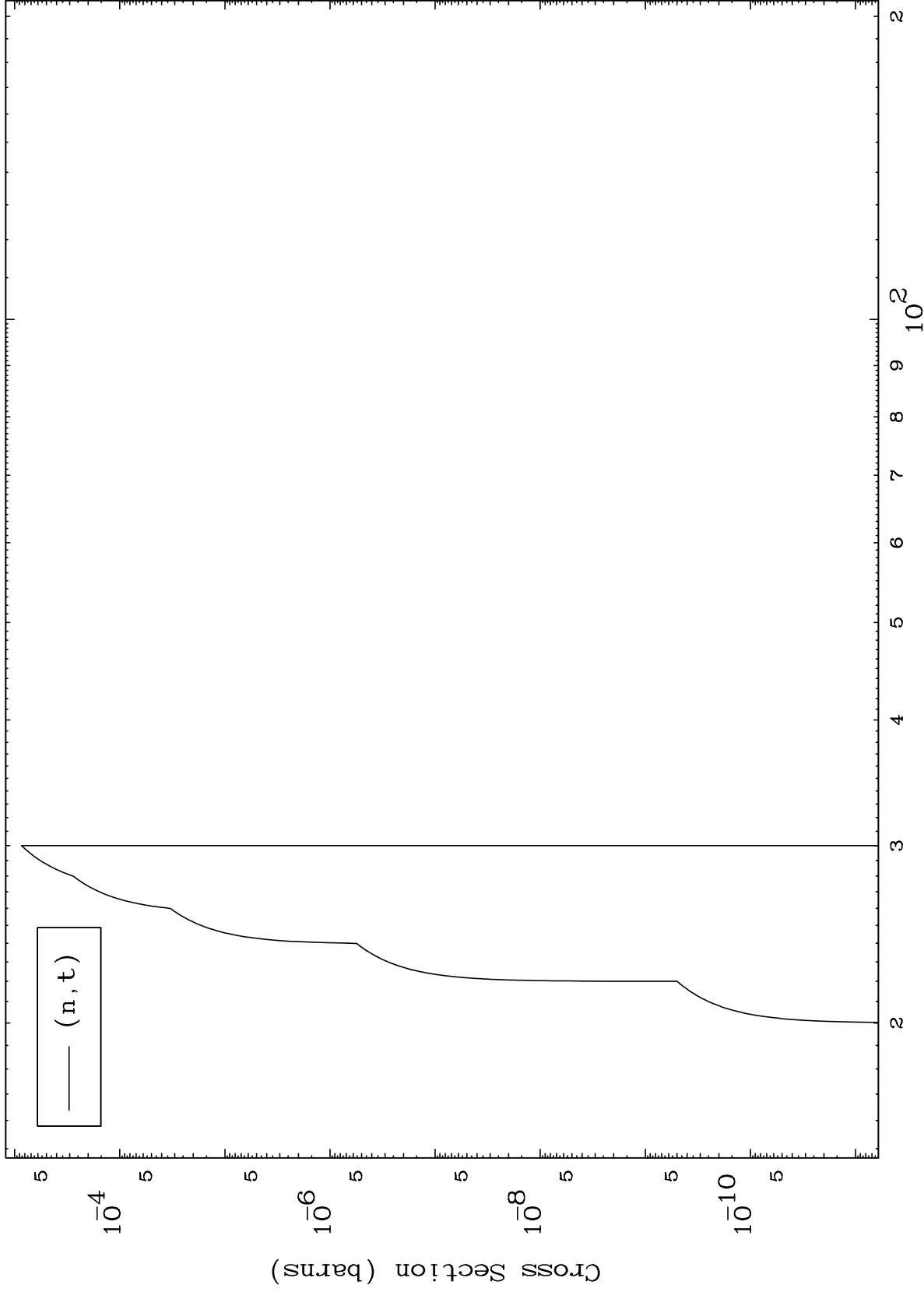
6

Incident Energy (MeV)

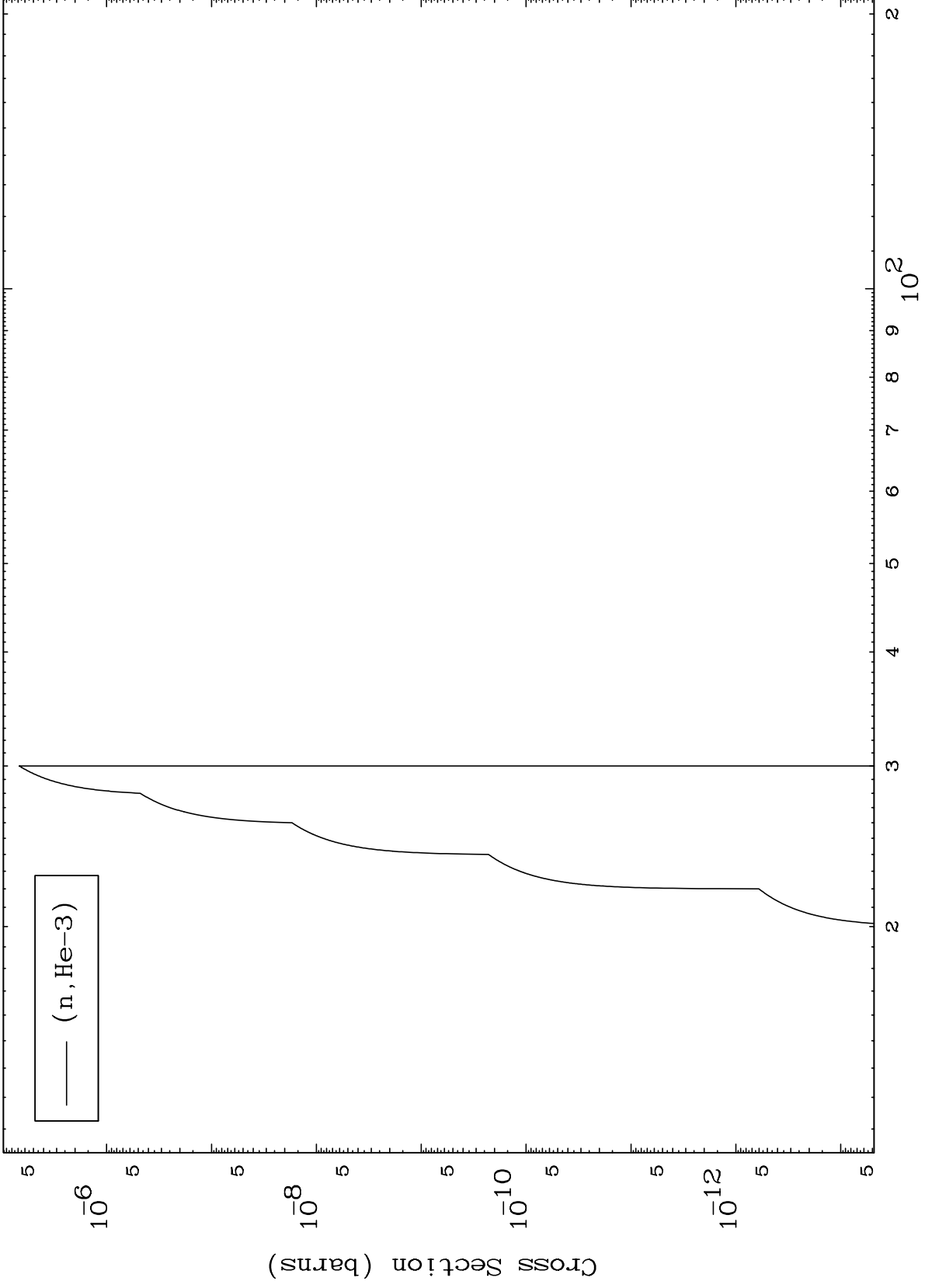
71-Lu-168







( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

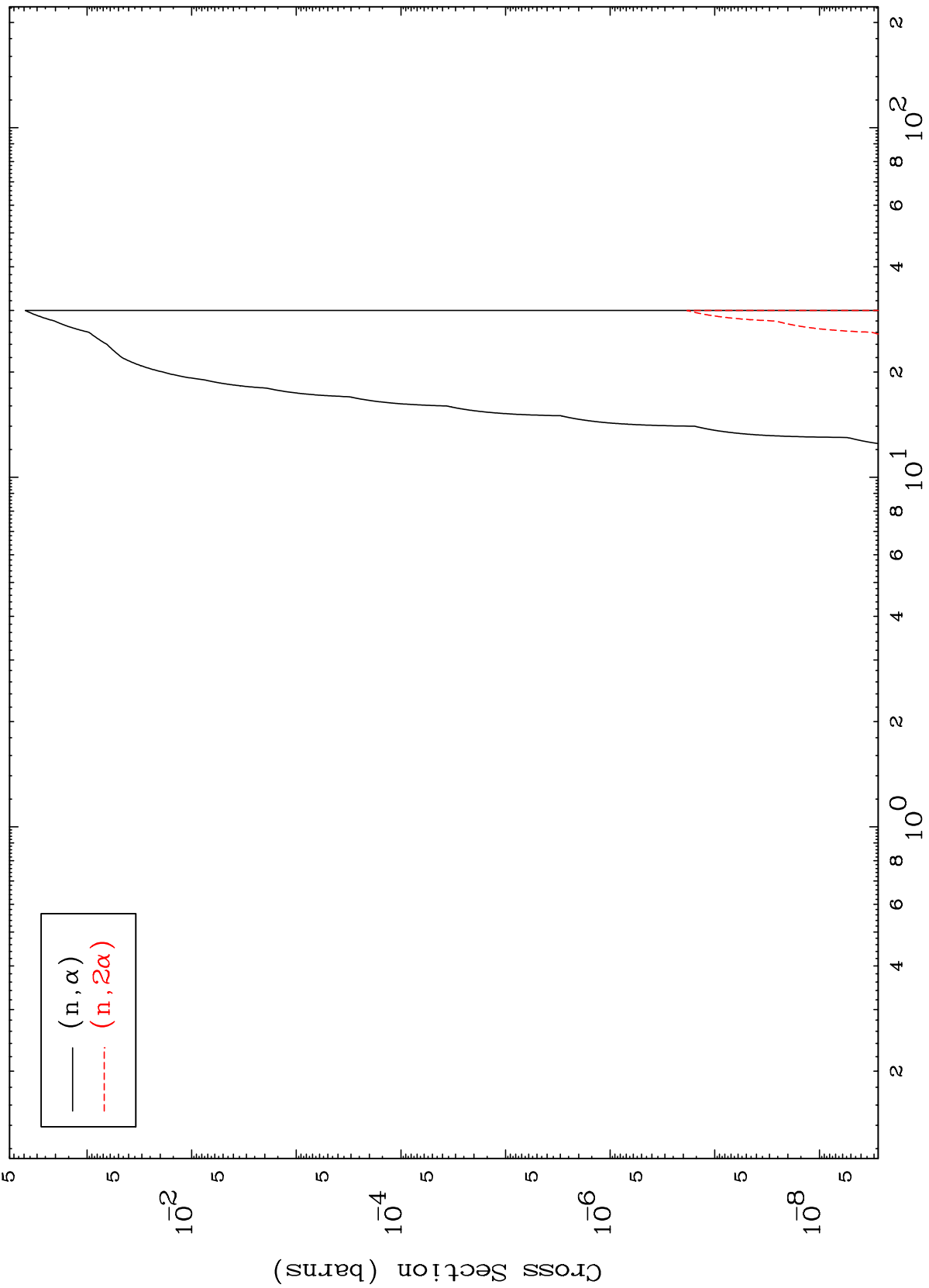


MAT 7104

( $\alpha, \alpha$ ) Levels

71-Lu-168

0 Kelvin Cross Sections

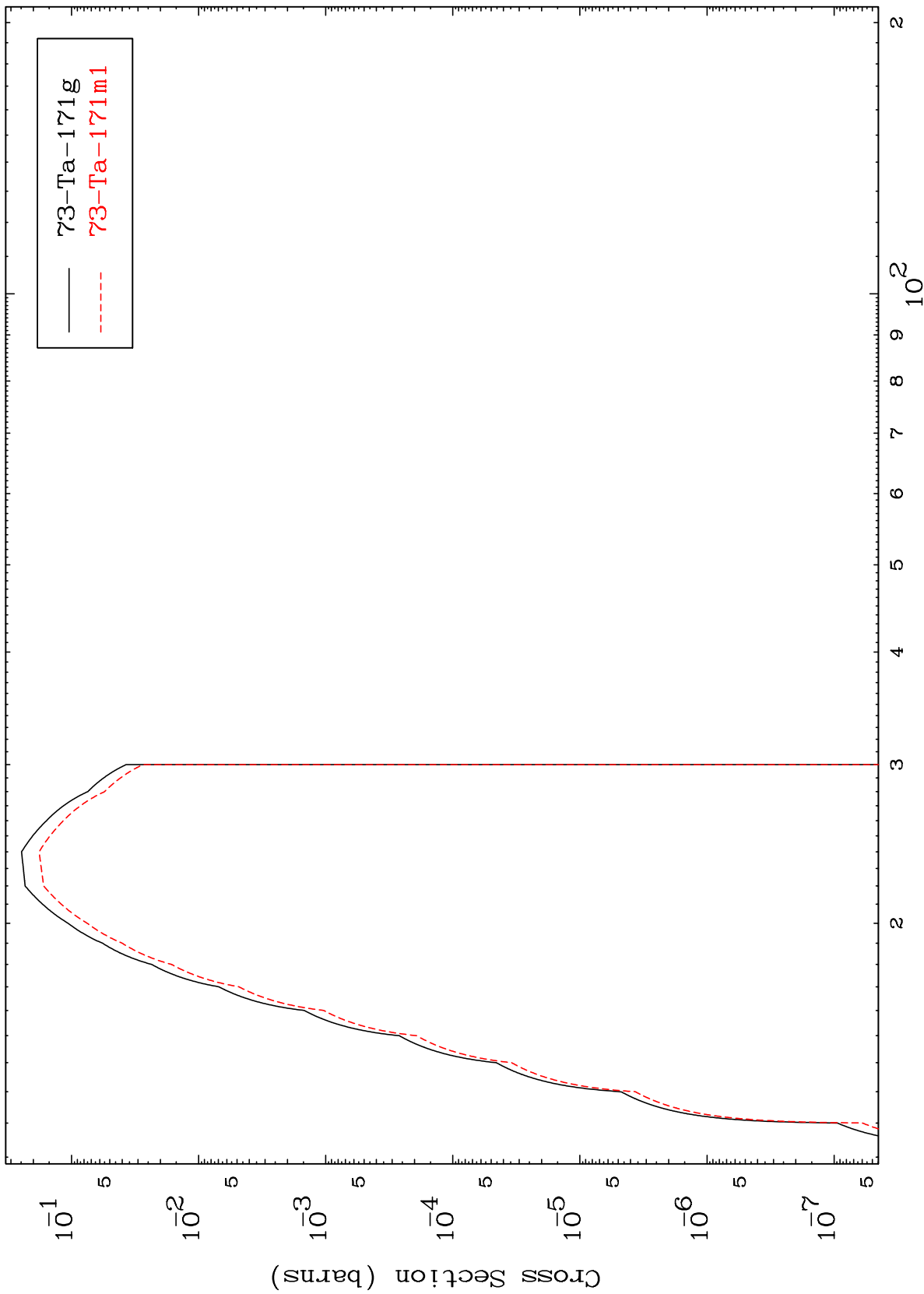


10

Incident Energy (MeV)

71-Lu-168

Inelastic  
Radionuclide Production Cross Section

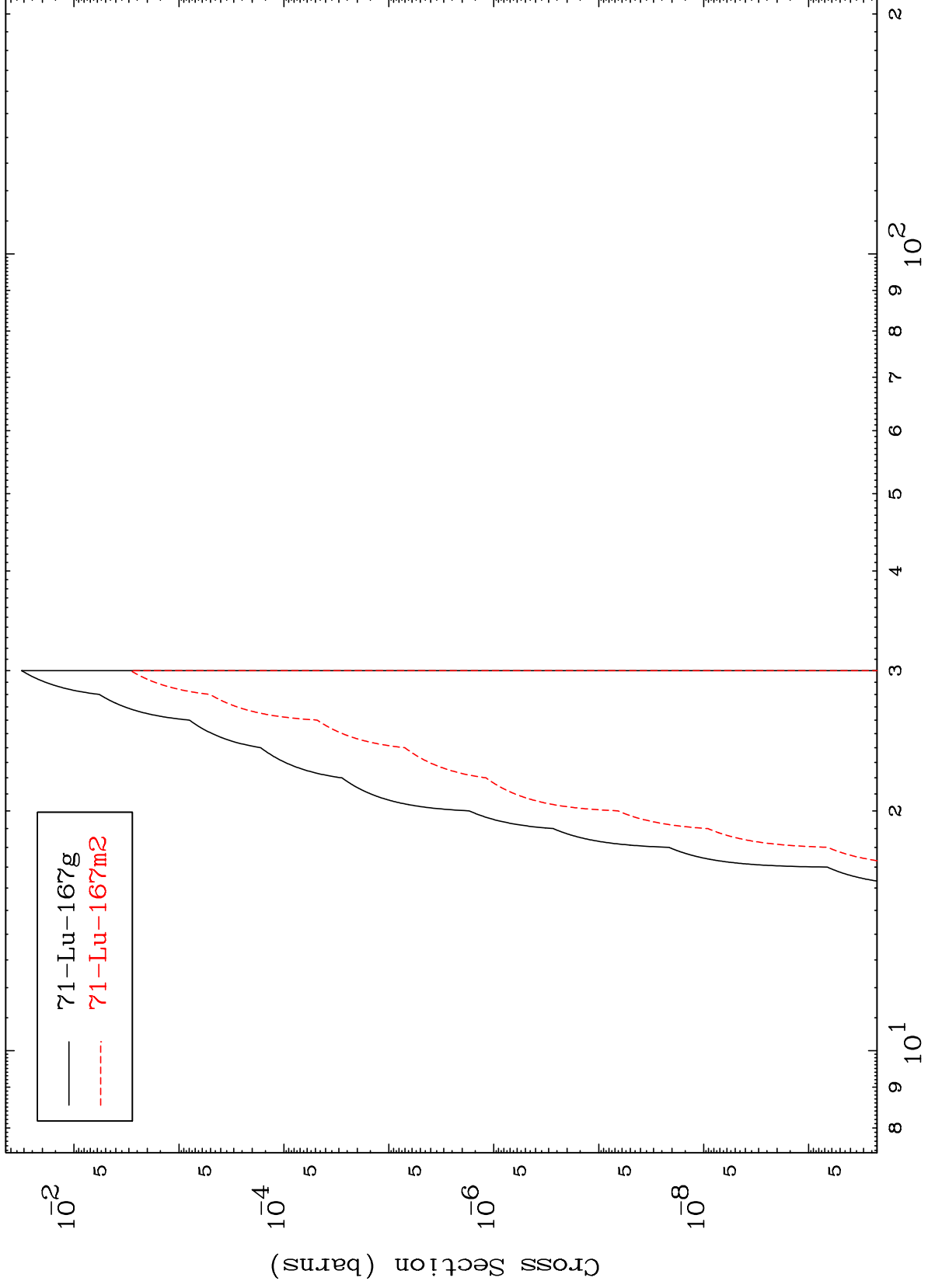


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(n,n')  $\alpha$

71-Lu-168

Radionuclide Production Cross Section



12

Incident Energy (MeV)

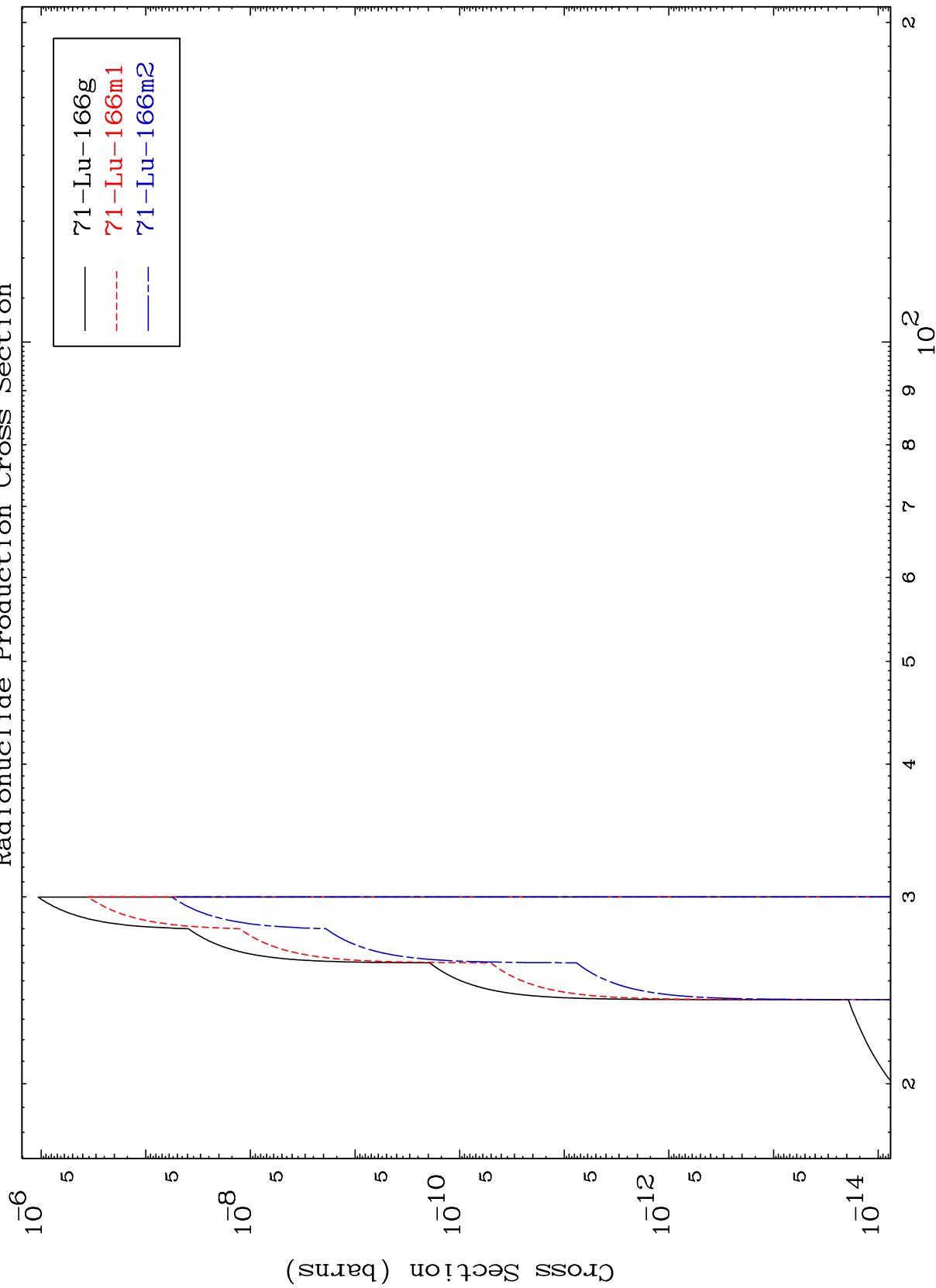
71-Lu-168

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(n,2n)  $\alpha$

71-Lu-168

Radionuclide Production Cross Section



13

Incident Energy (MeV)

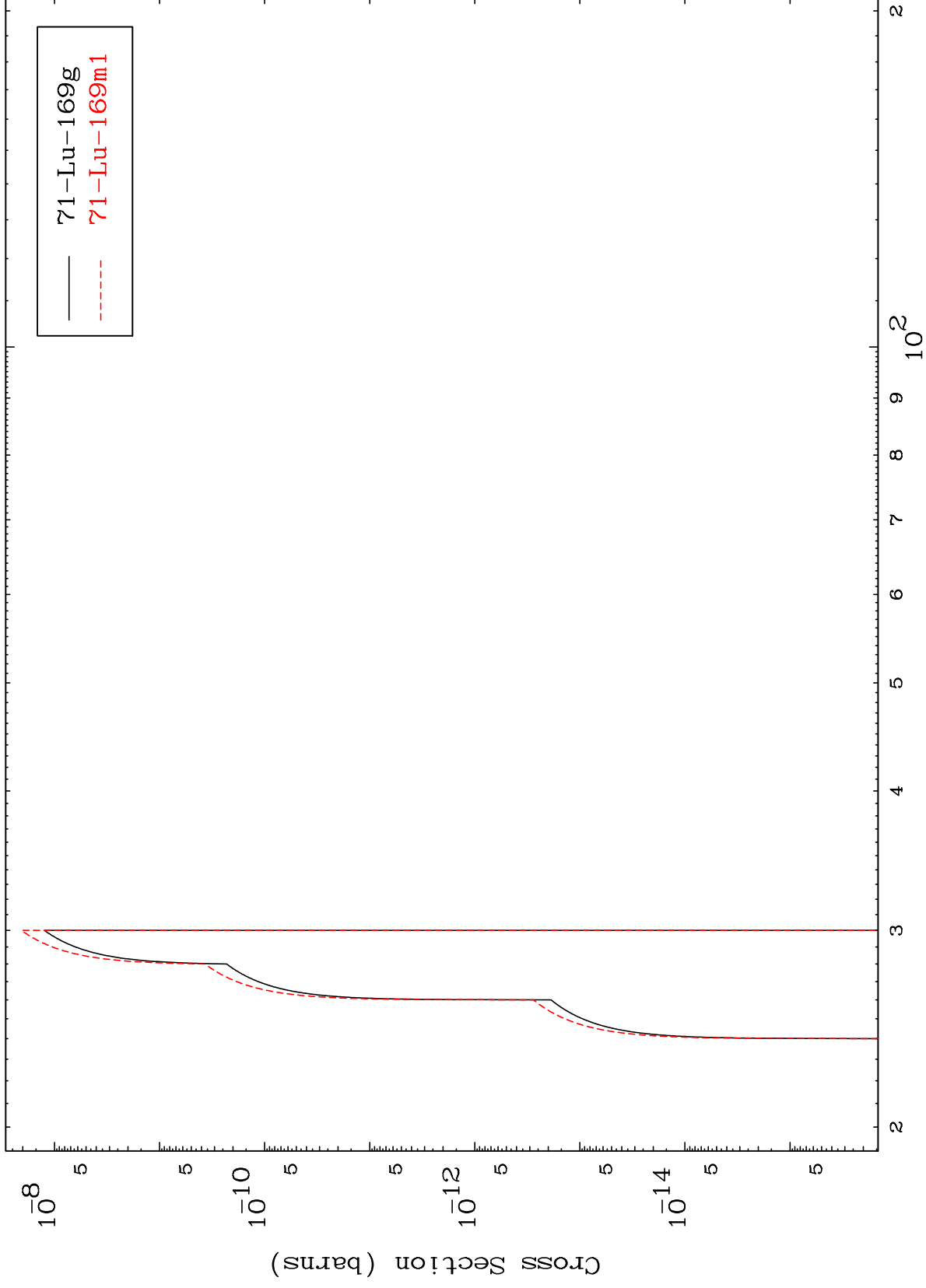
71-Lu-168

MAT 7104

(n,2n) p

<sup>71</sup>Lu-168

Radionuclide Production Cross Section



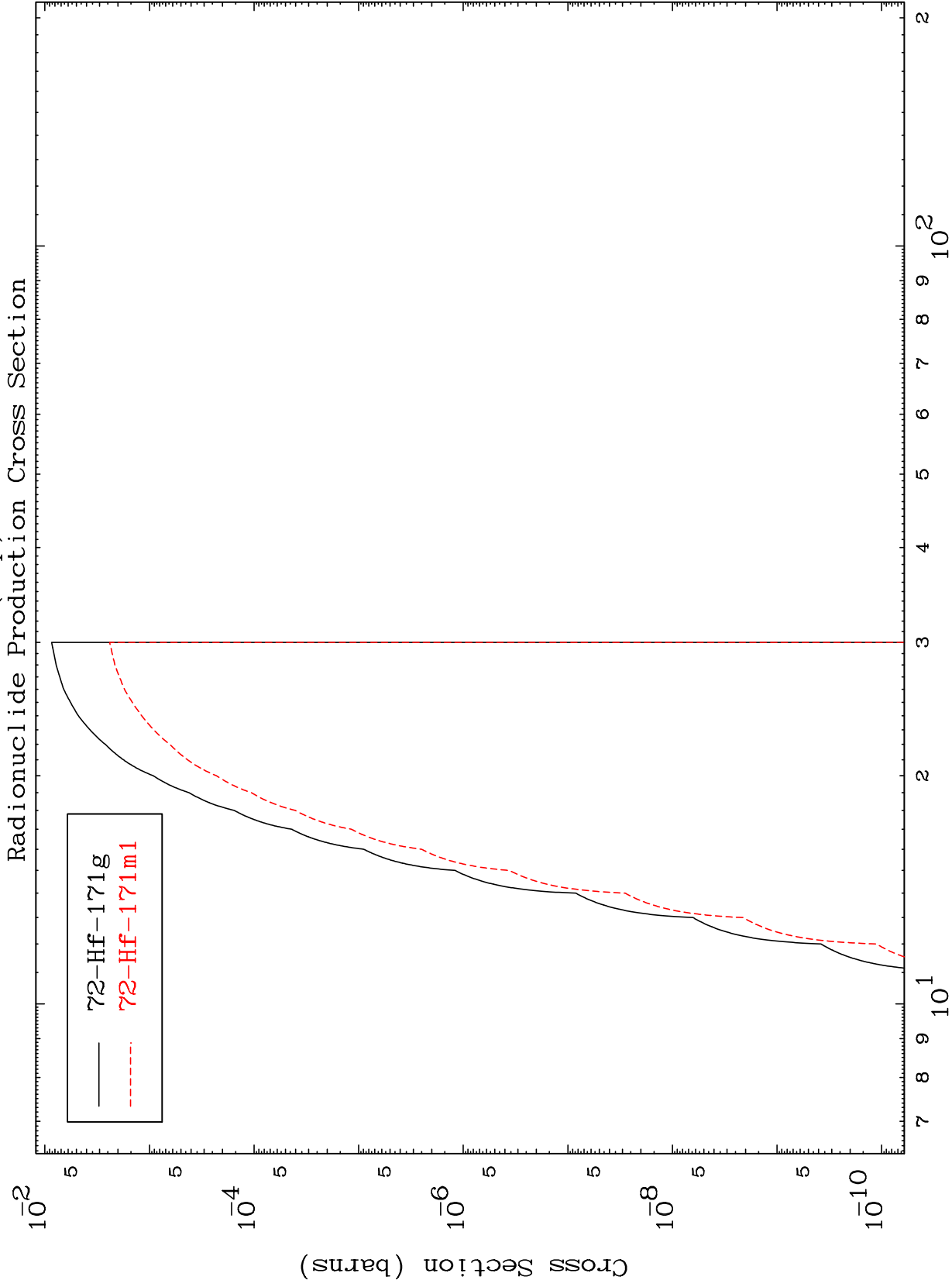
14

Incident Energy (MeV)

<sup>71</sup>Lu-168

MAT 7104

71-Lu-168



15

Incident Energy (MeV)

71-Lu-168

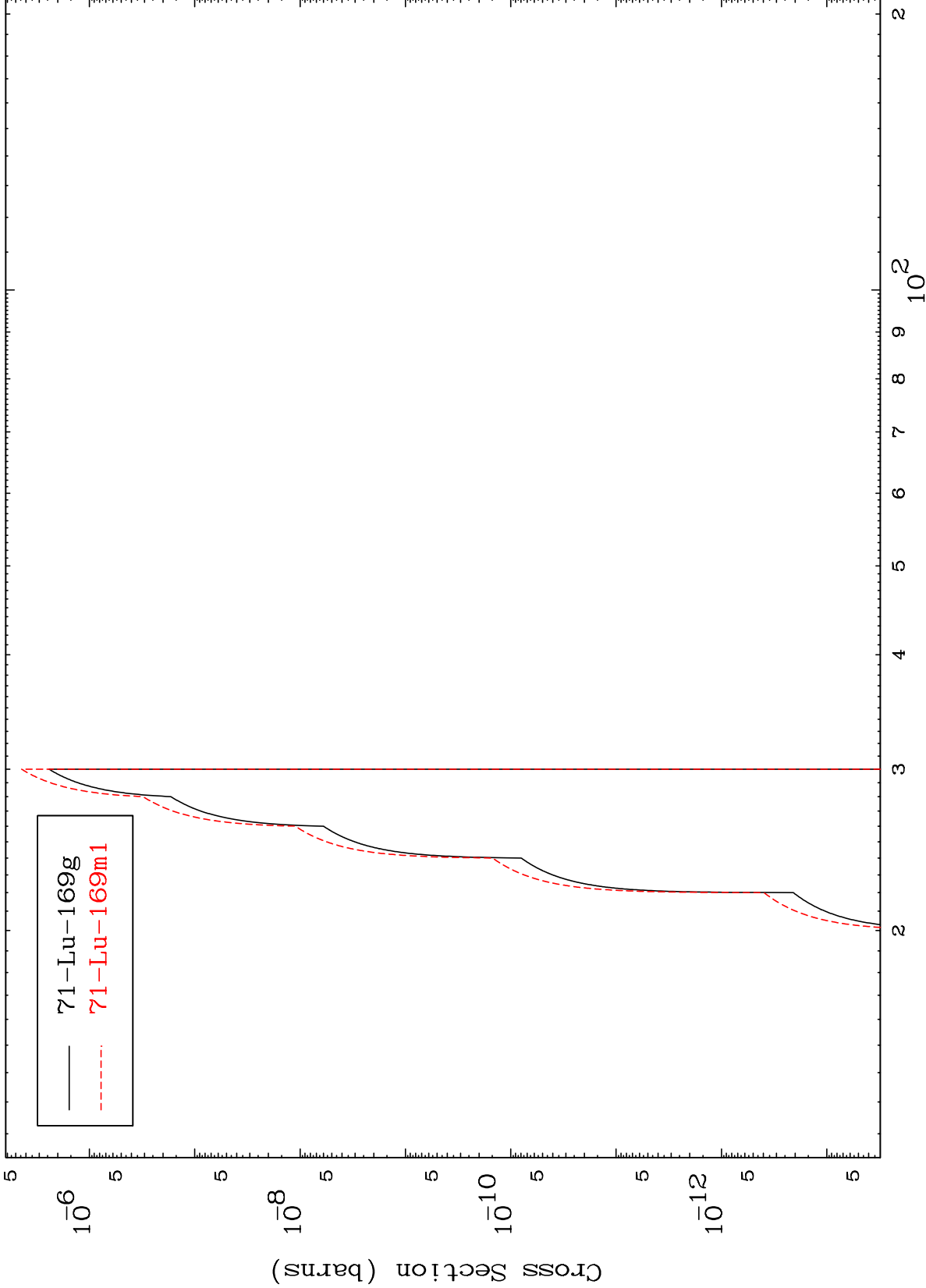


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(n,He-3)

71-Lu-168

Radionuclide Production Cross Section



16

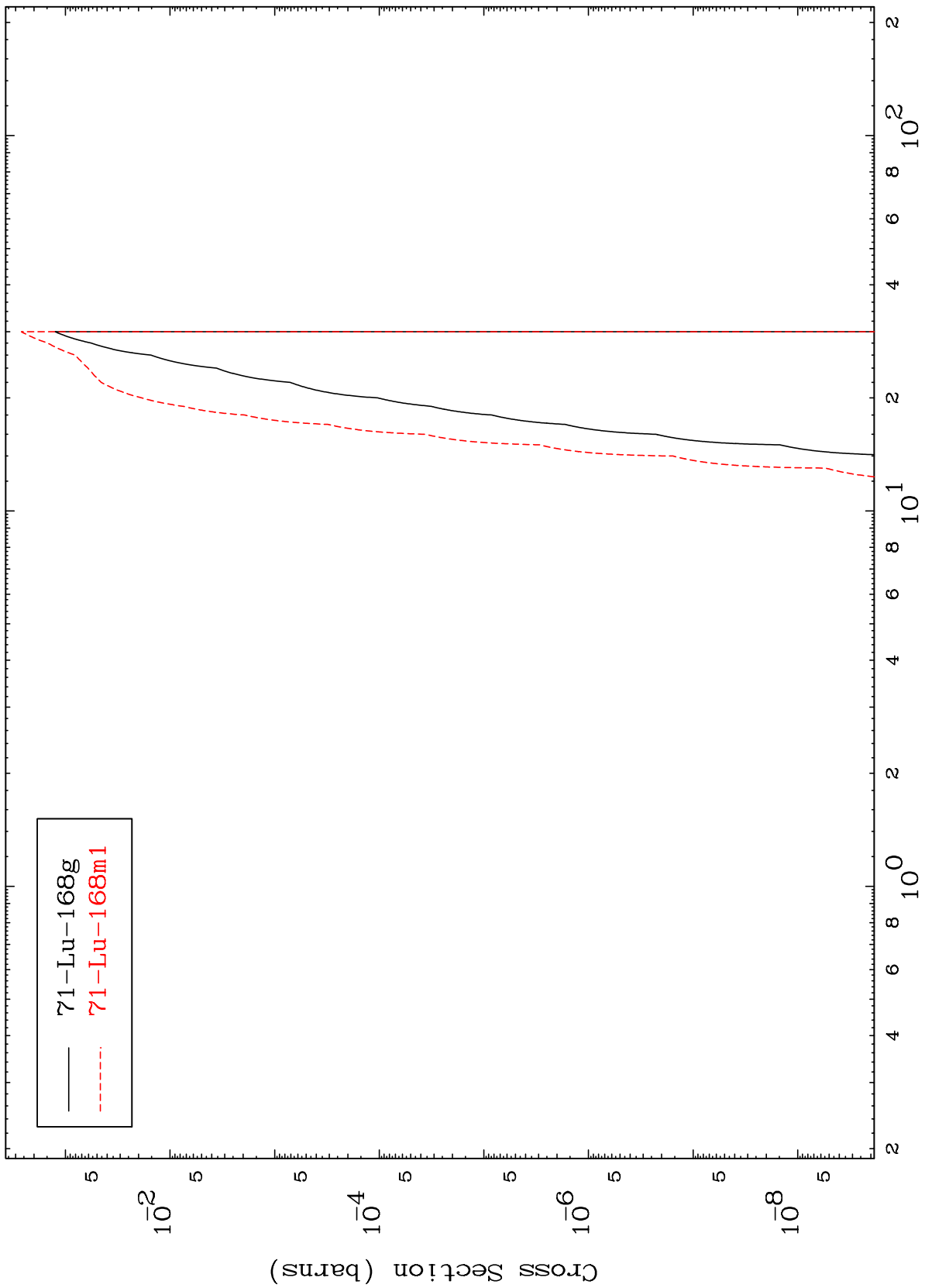
Incident Energy (MeV)

71-Lu-168

MAT 7104

<sup>71</sup>Lu-168

Radionuclide Production Cross Section (n,α)



<sup>71</sup>Lu-168

Incident Energy (MeV)

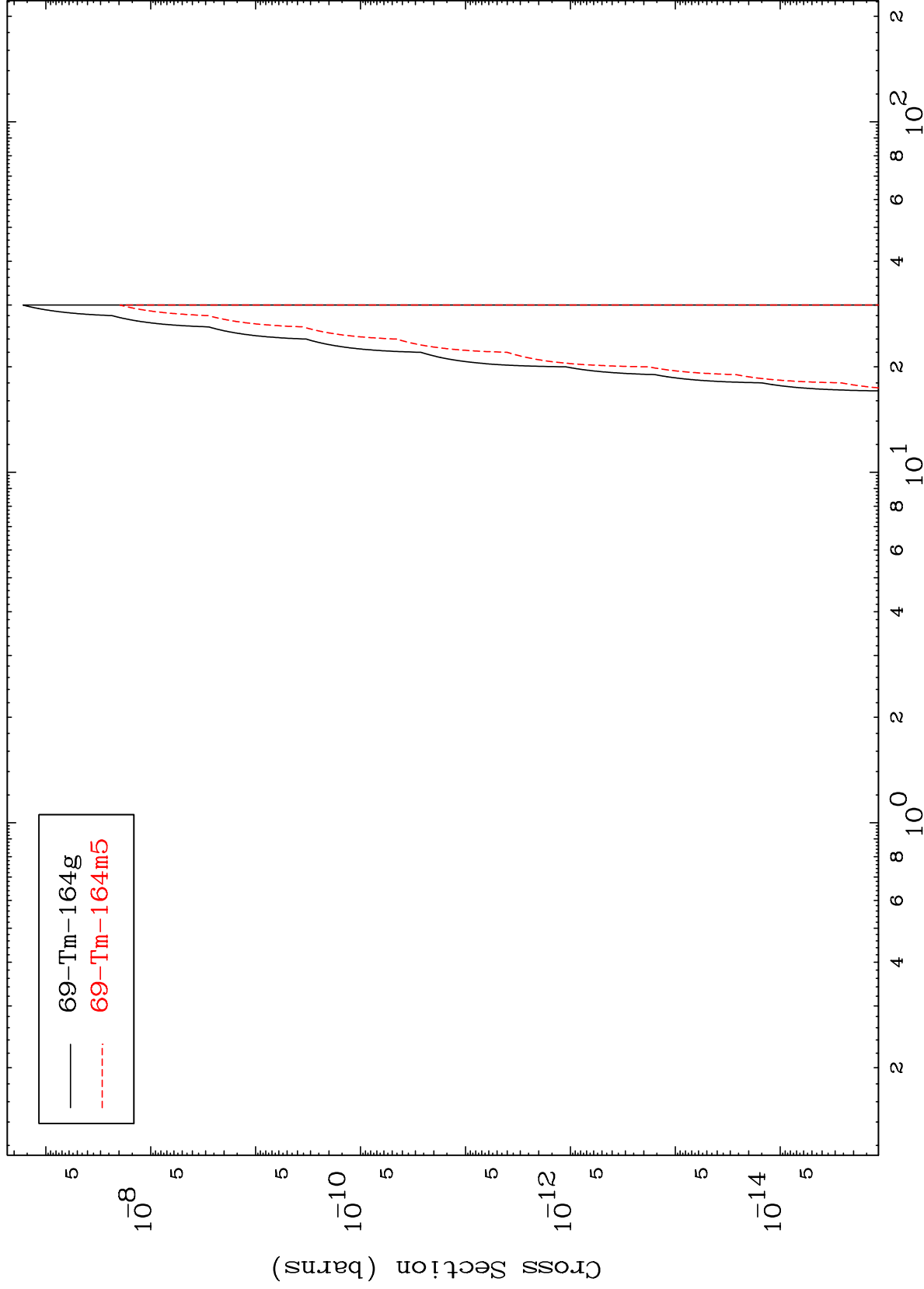
17

MAT 7104

(n,2α)

71-Lu-168

Radionuclide Production Cross Section



18

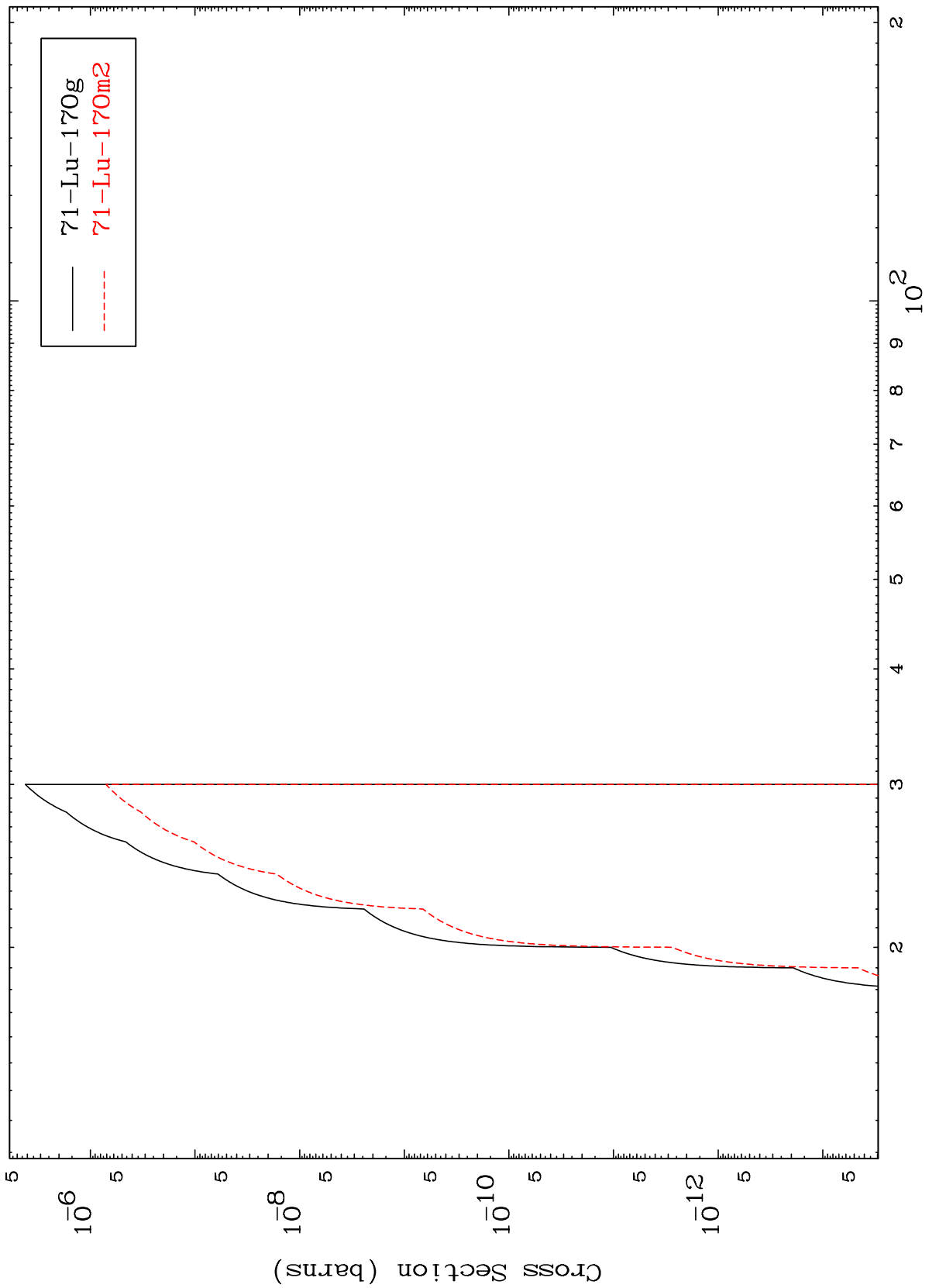
Incident Energy (MeV)

71-Lu-168

MAT 7104

71-Lu-168

(n,2p)  
Radionuclide Production Cross Section



19

Incident Energy (MeV)

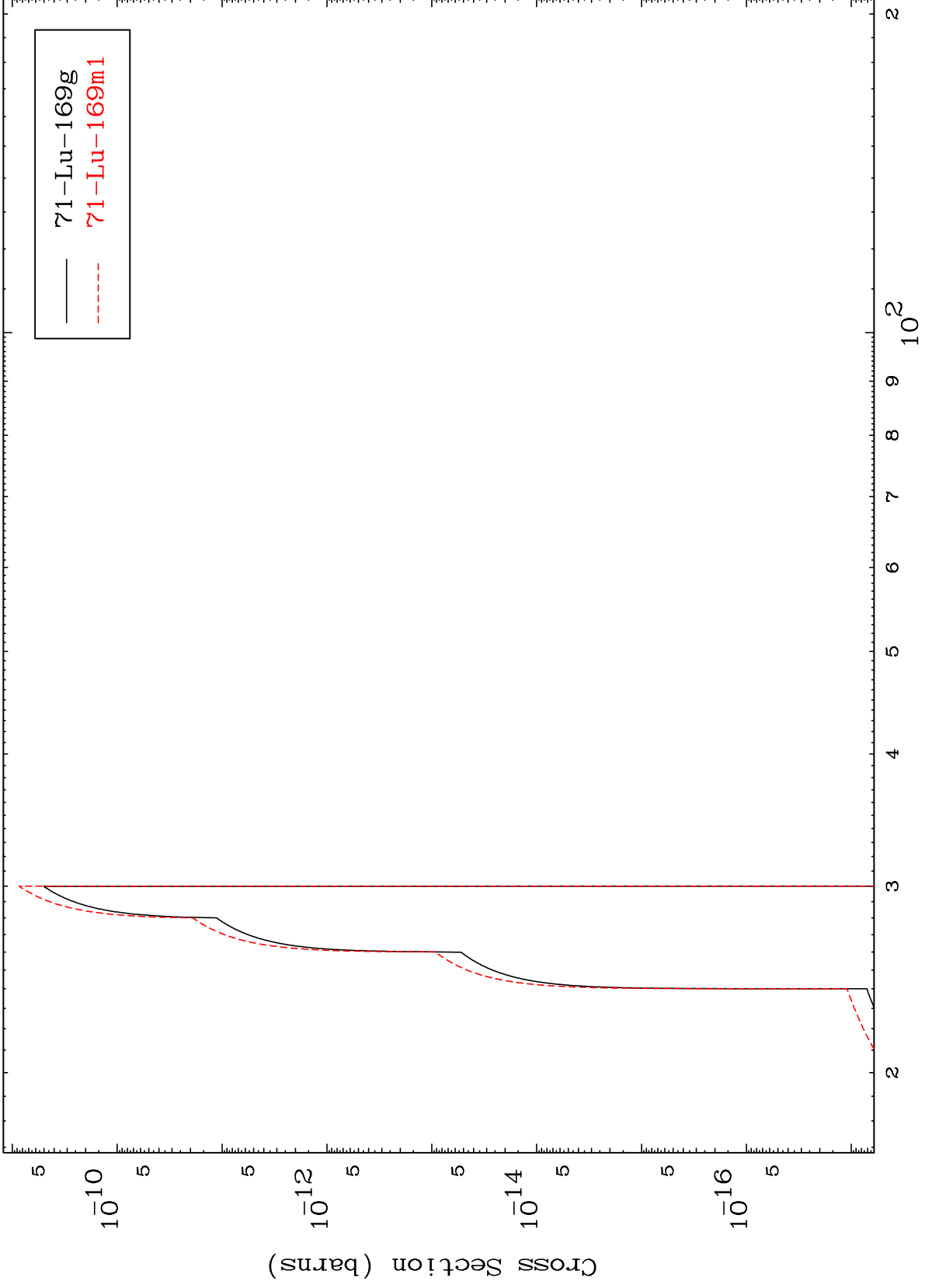
71-Lu-168

MAT 7104

(n,p) d

71-Lu-168

Radionuclide Production Cross Section



20

Incident Energy (MeV)

71-Lu-168