

Program EVALPLOT
(Version 2021-1)

by

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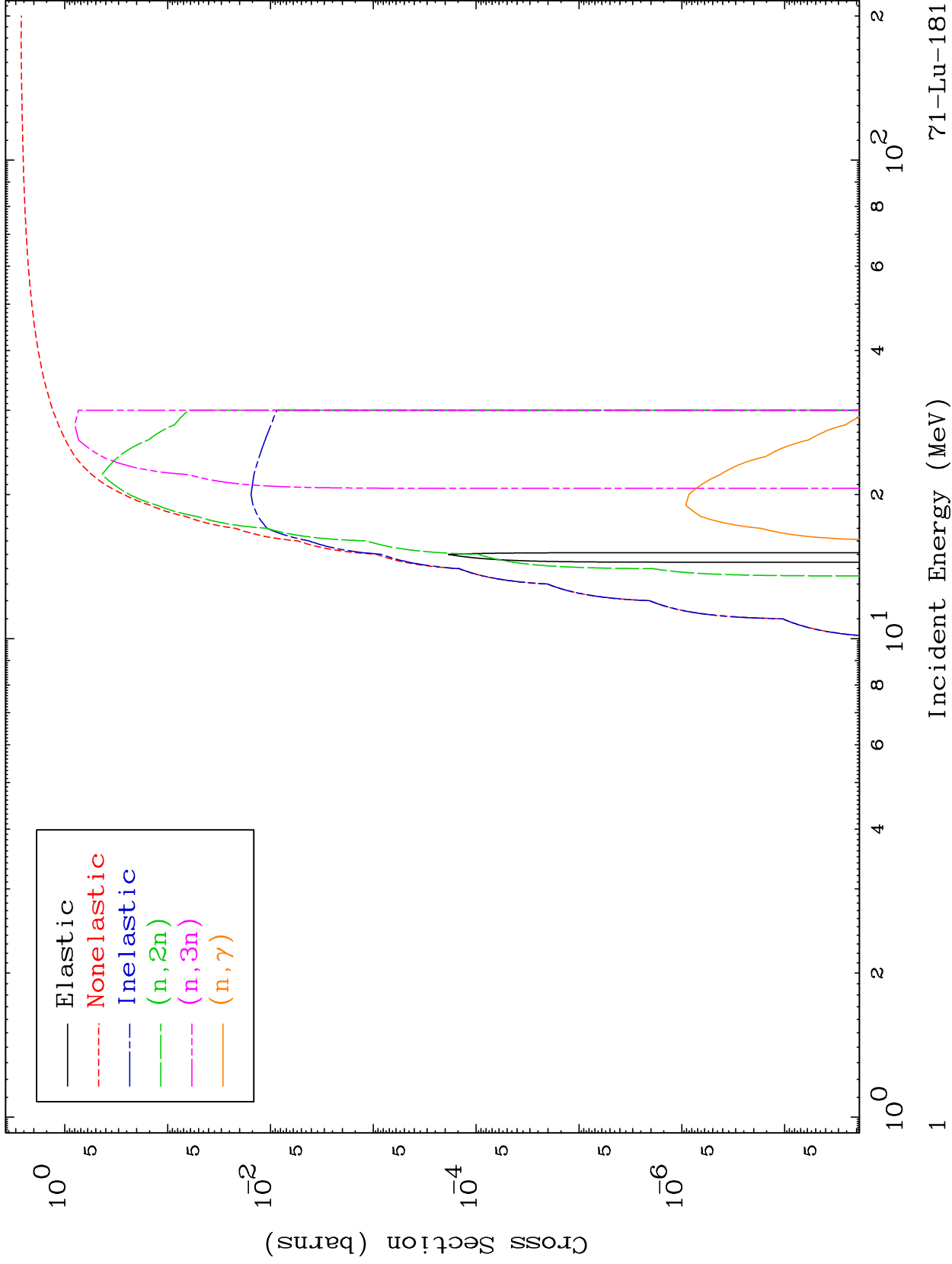
Press Mouse Button to Start

MAT 7143

0 Kelvin

α Major Cross Sections

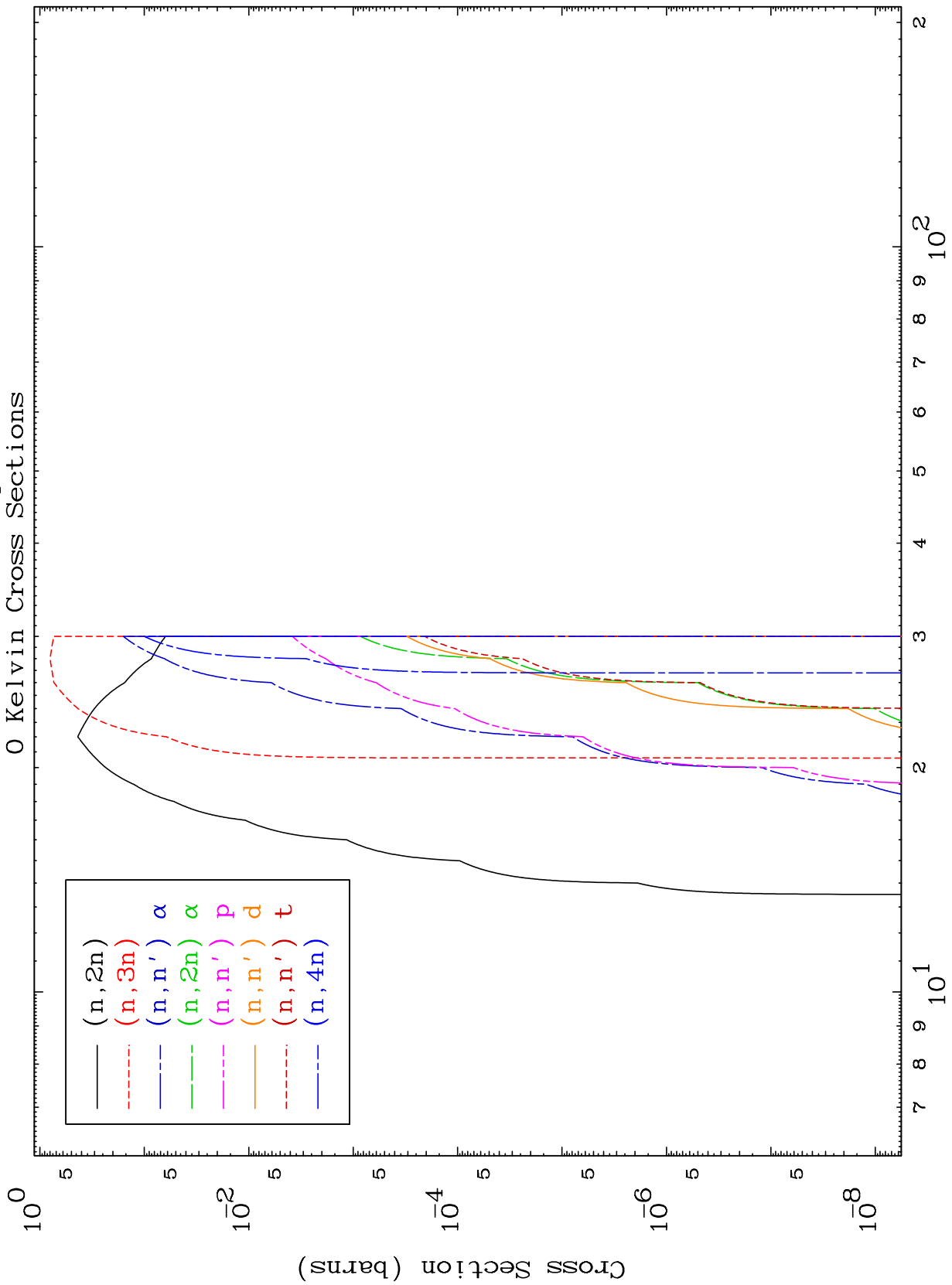
71-Lu-181



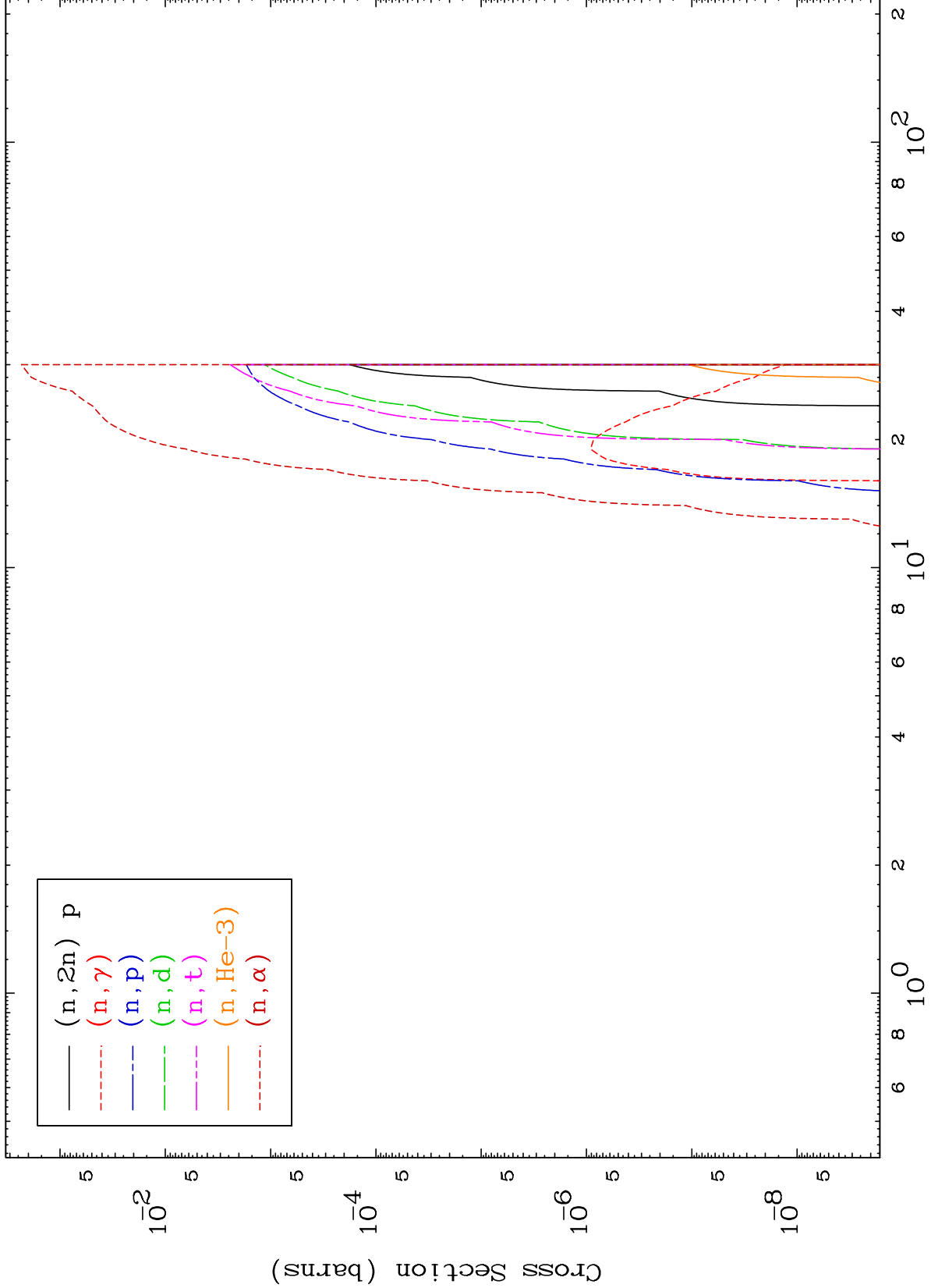
Incident Energy (MeV)

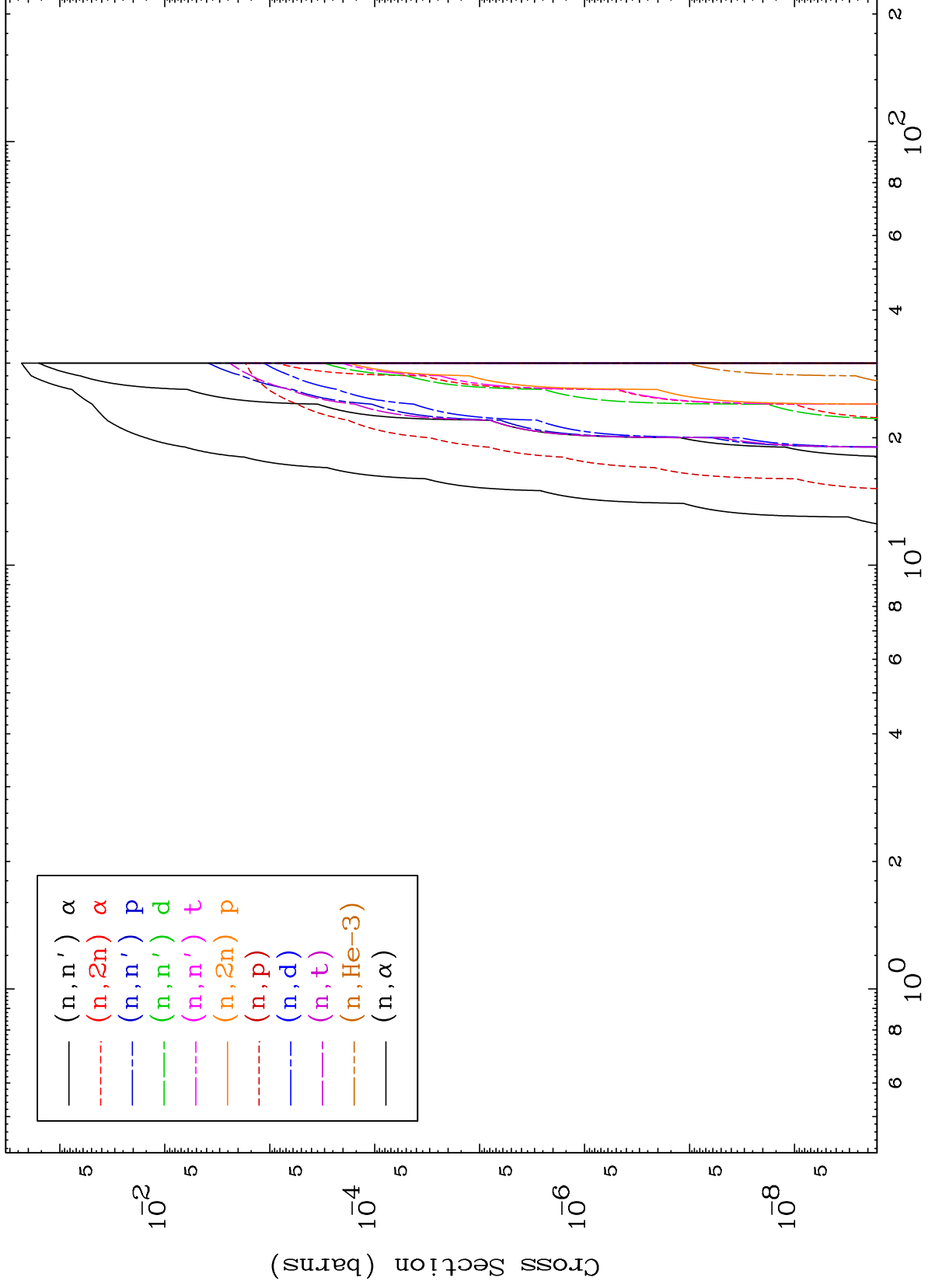
71-Lu-181

α Neutron Absorption
0 Kelvin Cross Sections



0 Kelvin Cross Sections

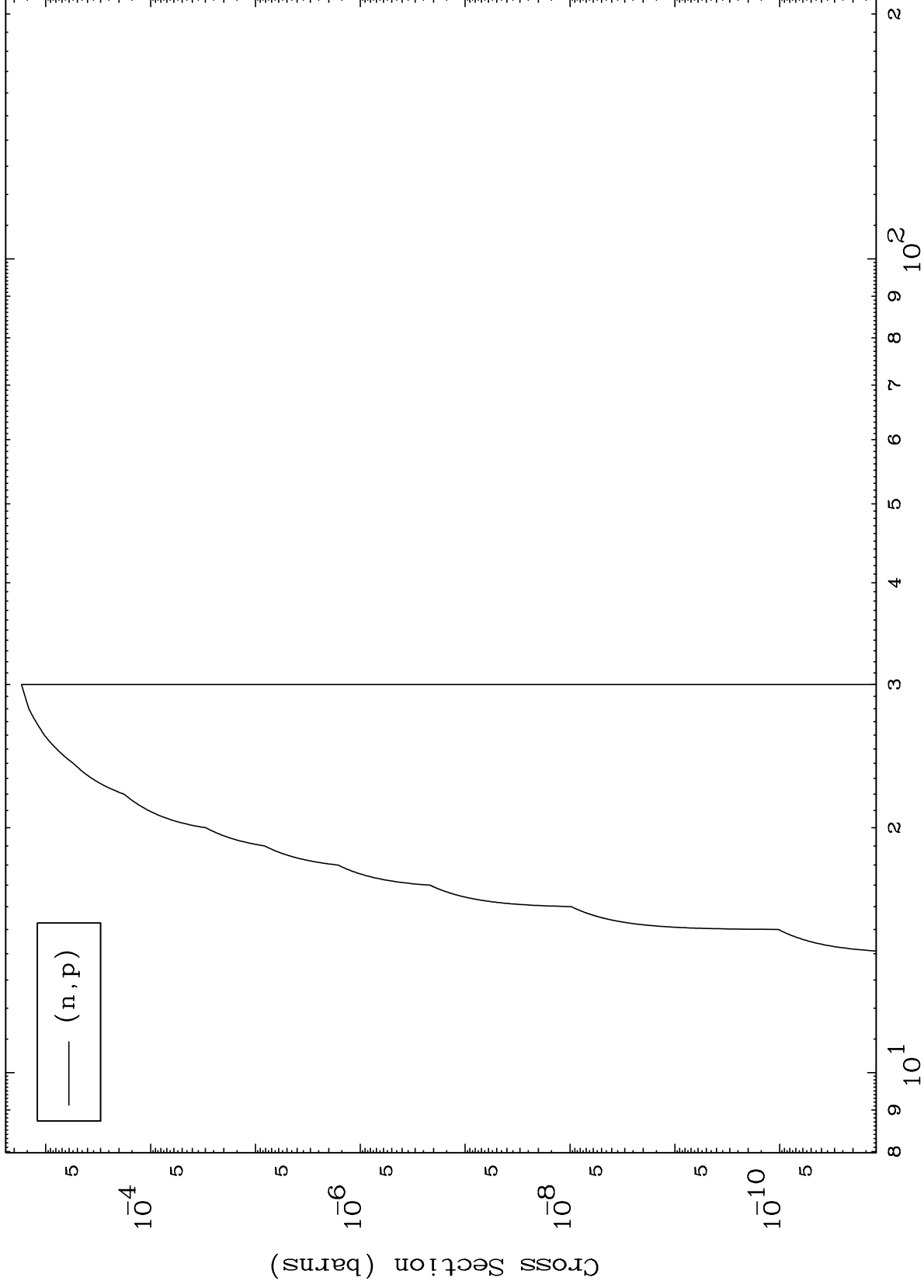




MAT 7143

(α, p) Levels
0 Kelvin Cross Sections

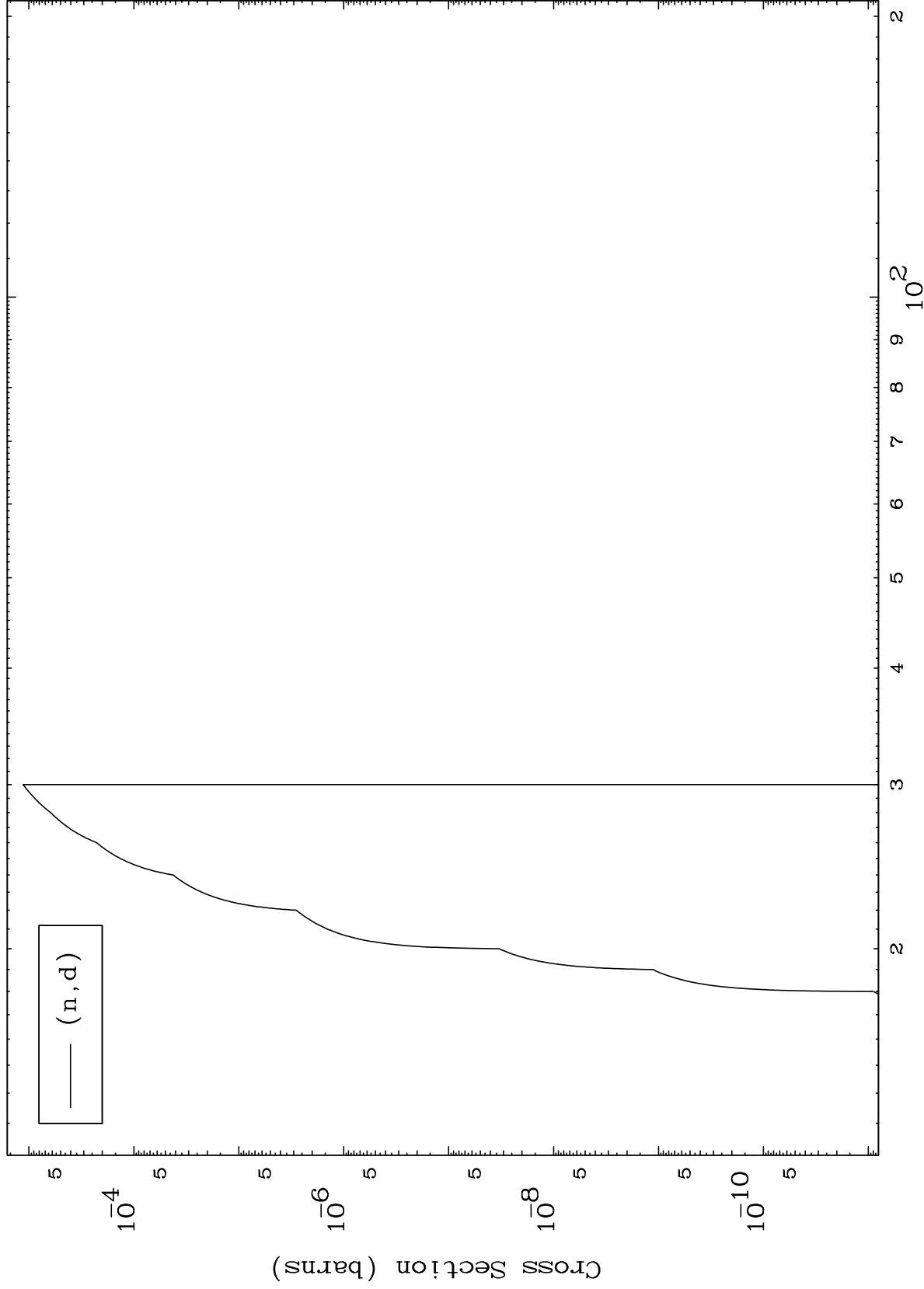
71-Lu-181

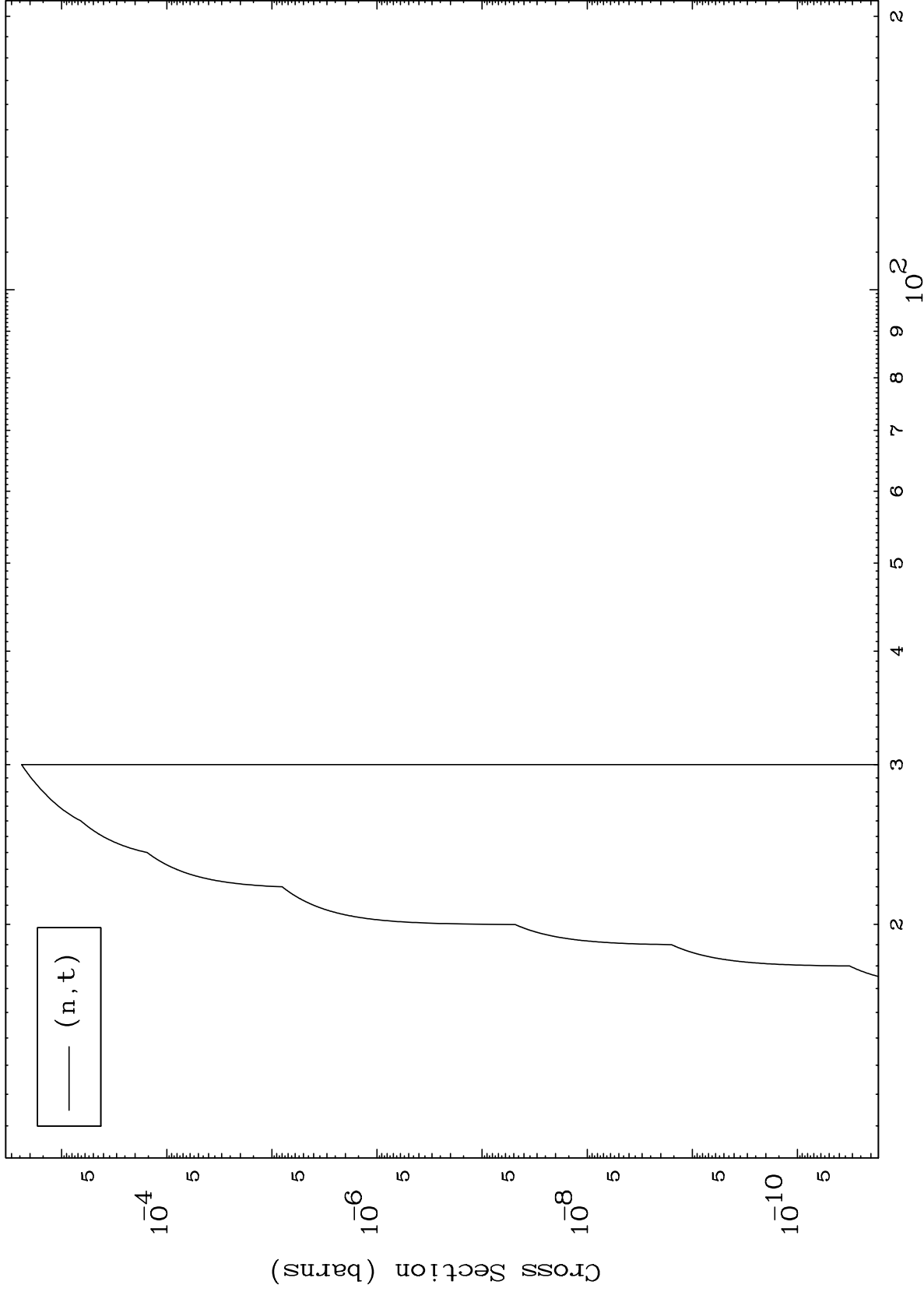


Incident Energy (MeV)

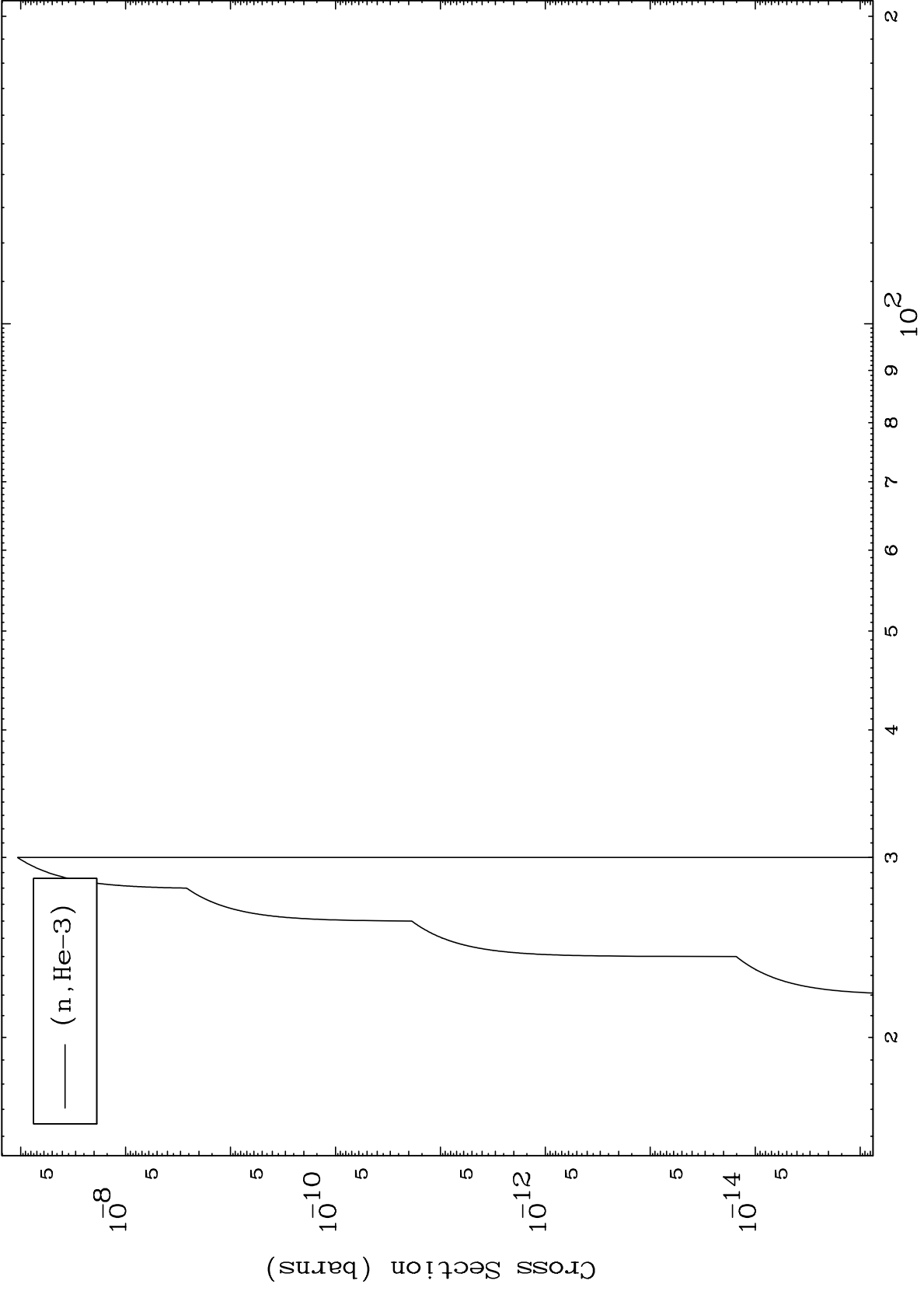
71-Lu-181

5





($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

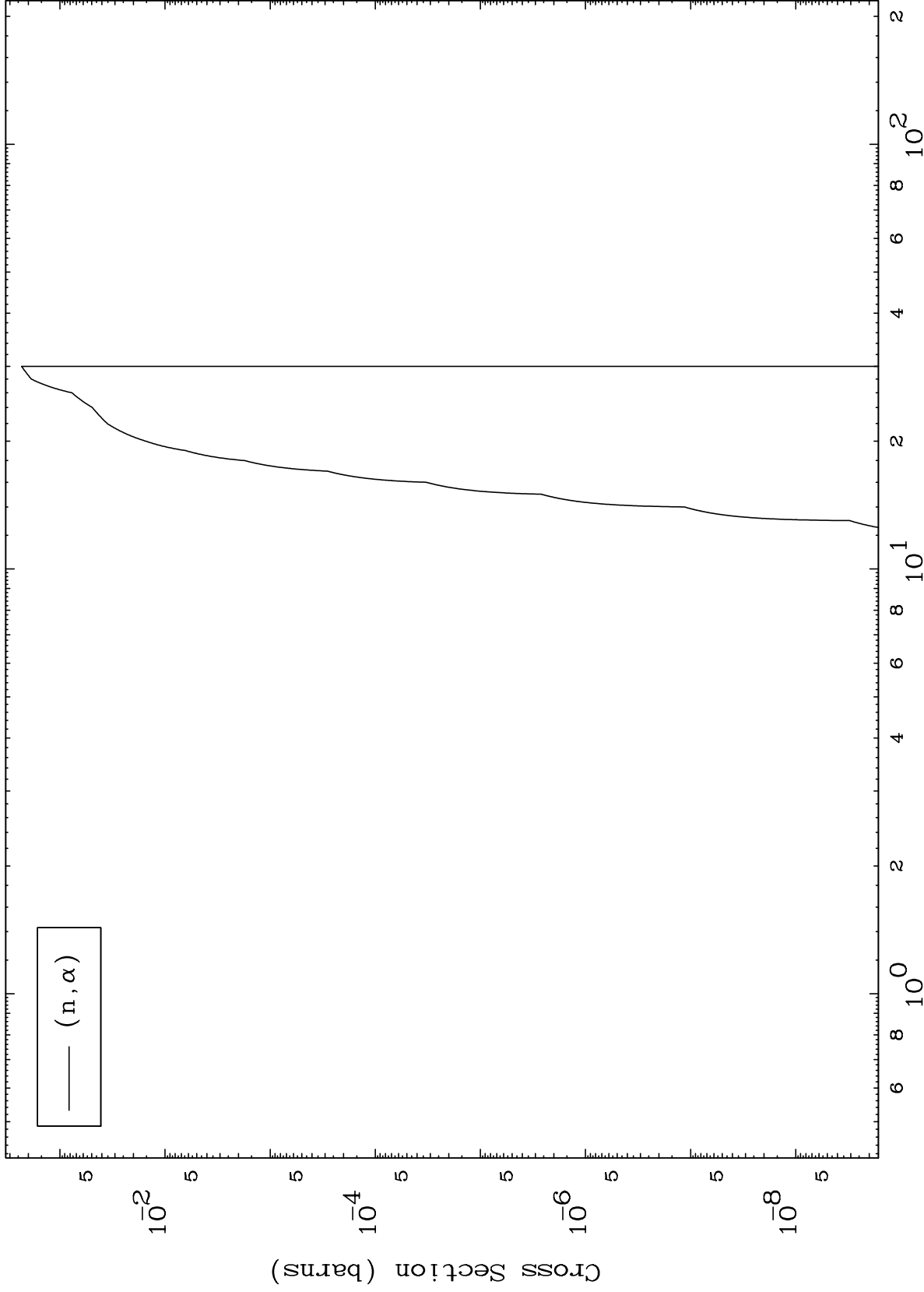


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(α, α) Levels

71-Lu-181

0 Kelvin Cross Sections



9

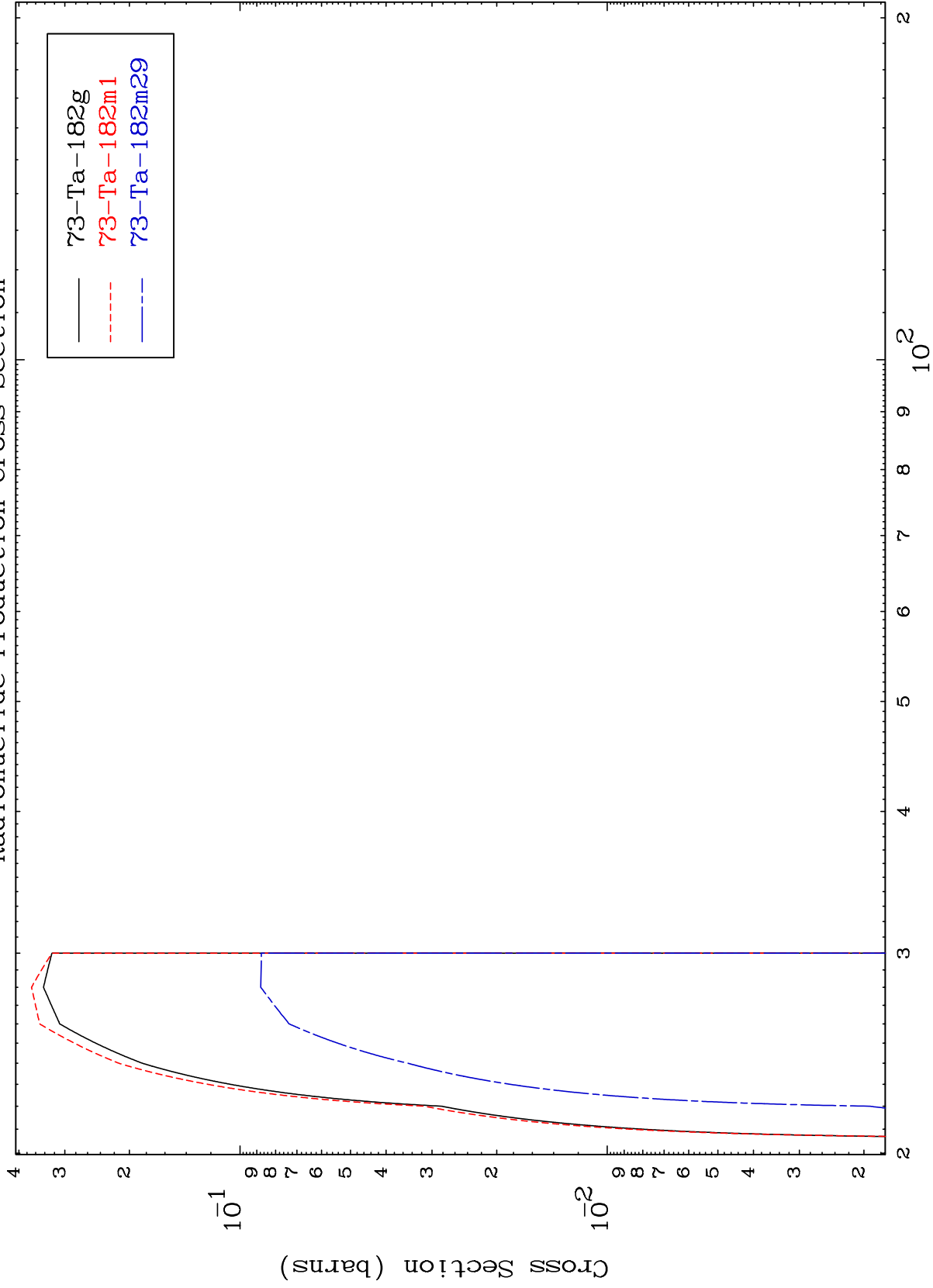
Incident Energy (MeV)

71-Lu-181

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71-Lu-181

(n,3n)
Radionuclide Production Cross Section



71-Lu-181

Incident Energy (MeV)

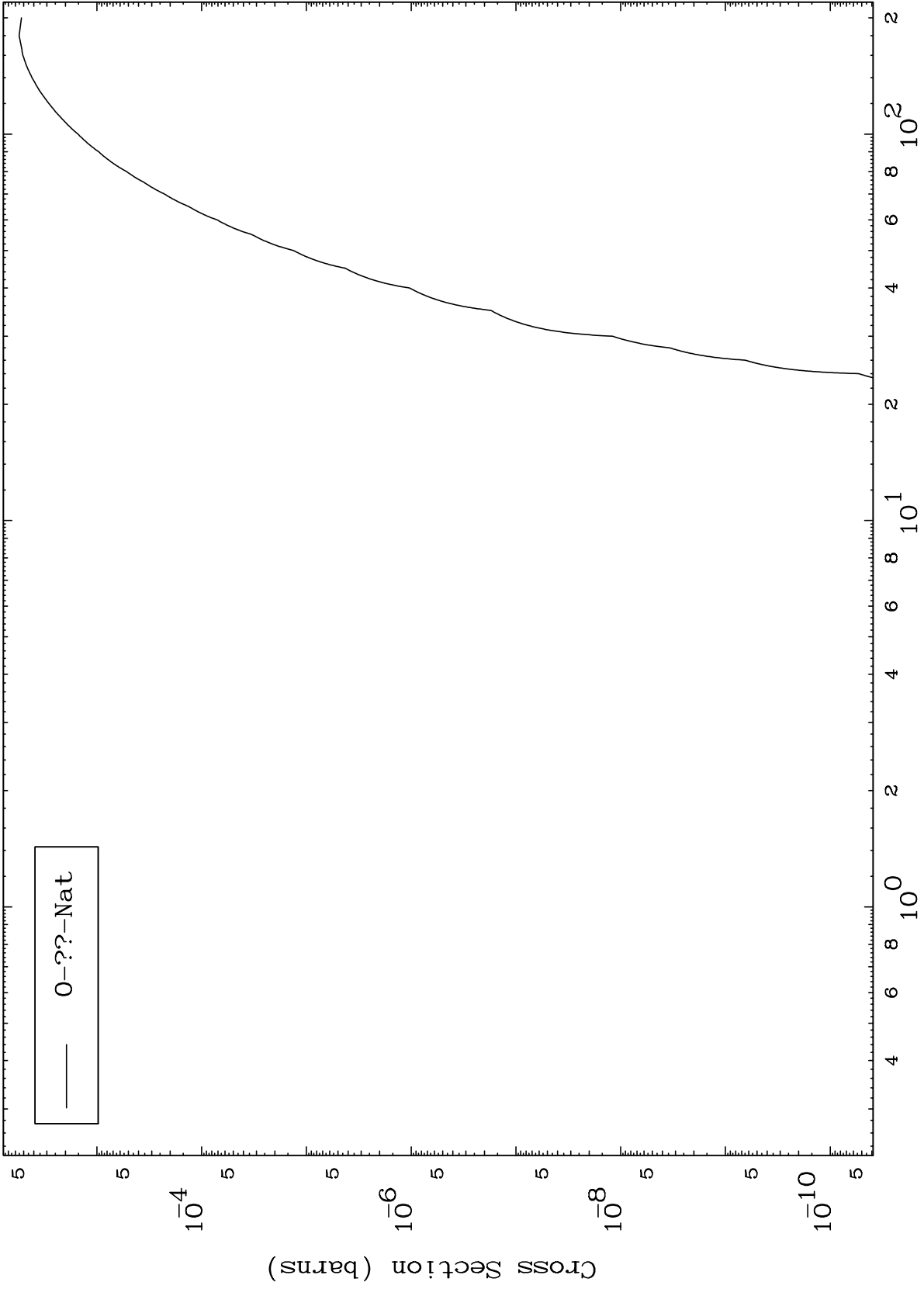
10

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Fission

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Radionuclide Production Cross Section

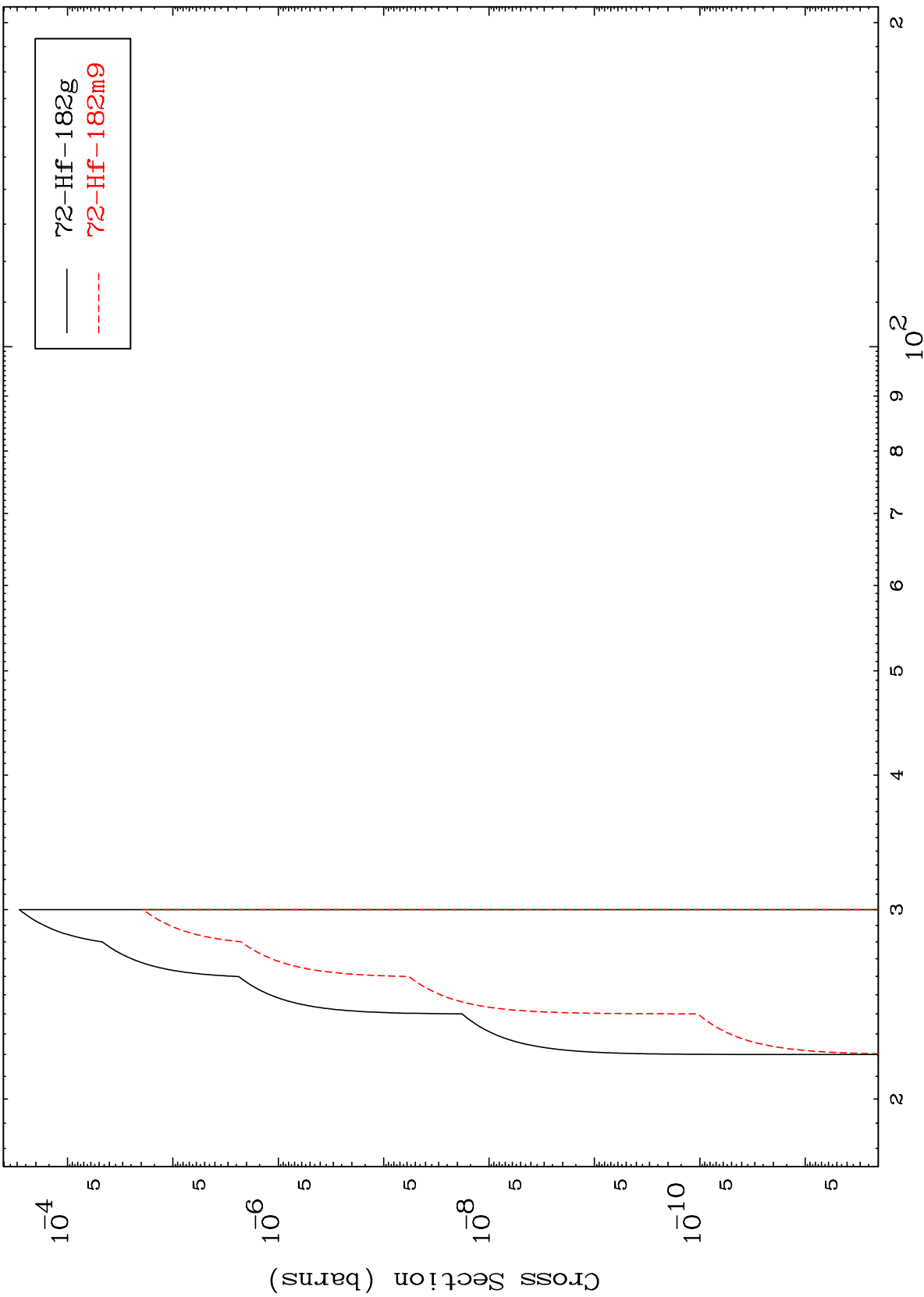


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(n,n') d

71-Lu-181

Radionuclide Production Cross Section



12

Incident Energy (MeV)

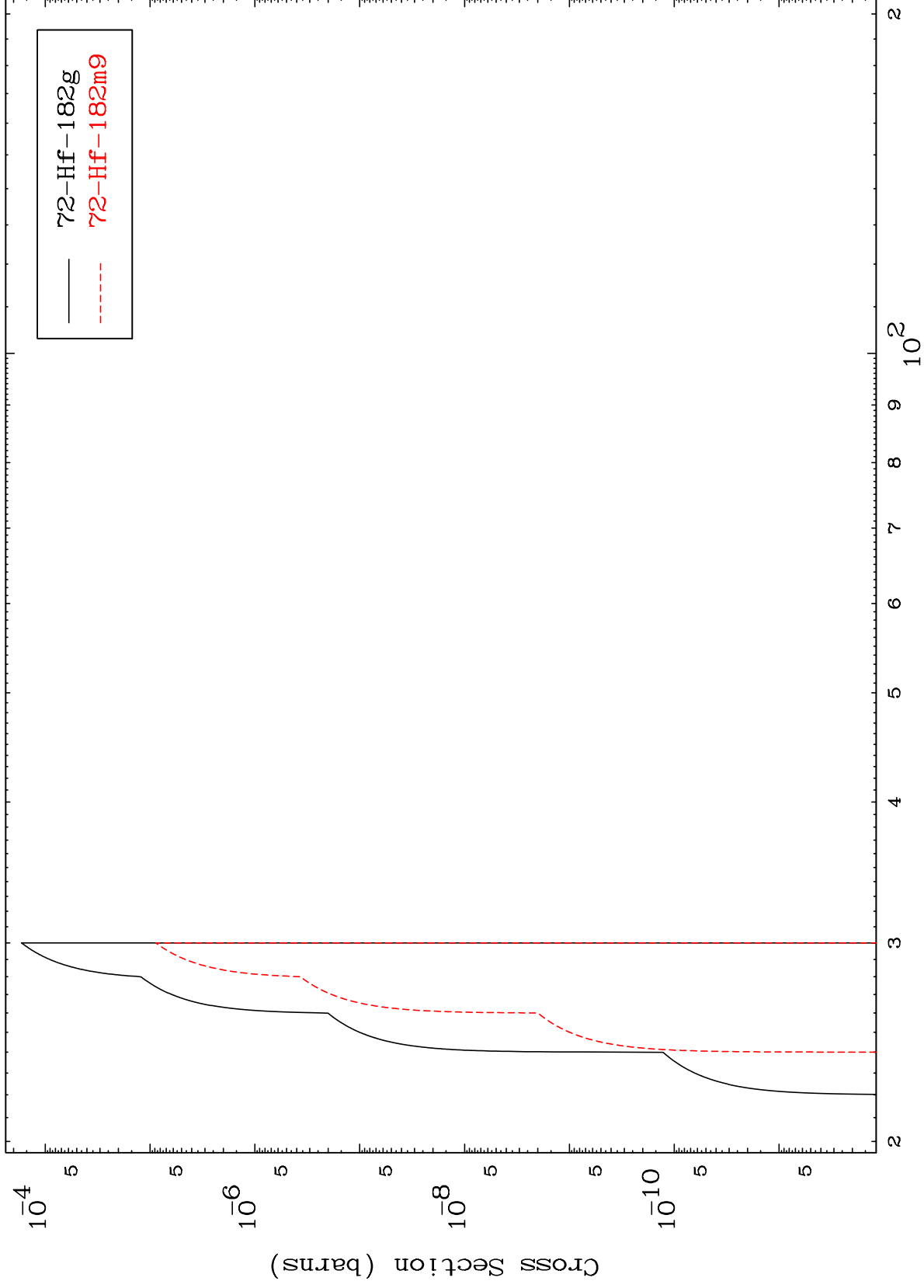
71-Lu-181

MAT 7143

(n,2n) p

71-Lu-181

Radionuclide Production Cross Section



13

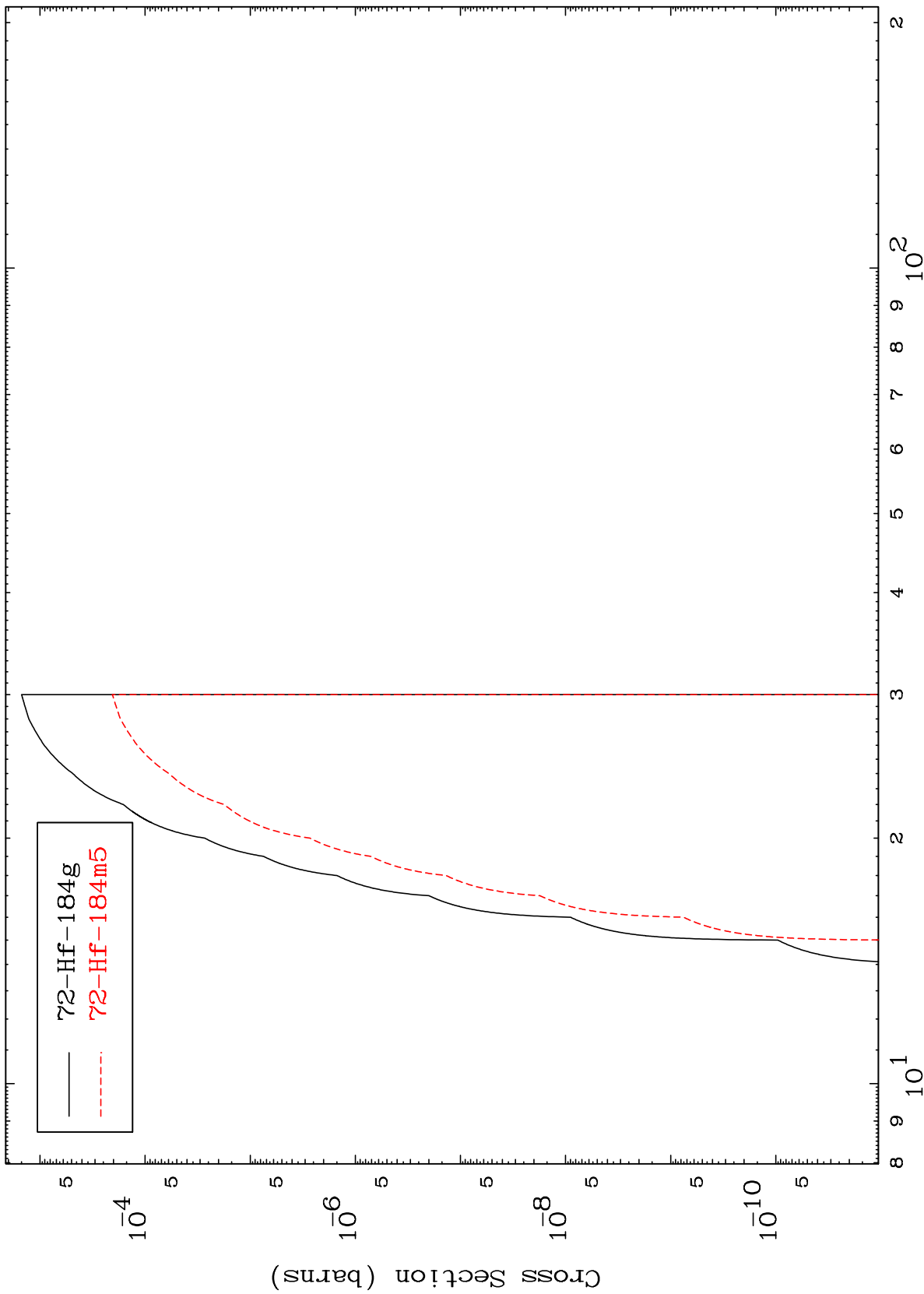
Incident Energy (MeV)

71-Lu-181

MAT 7143

71-Lu-181

(n,p)
Radionuclide Production Cross Section



71-Lu-181

Incident Energy (MeV)

14

(n, t)
Radionuclide Production Cross Section

