

Program EVALPLOT
(Version 2021-1)

by

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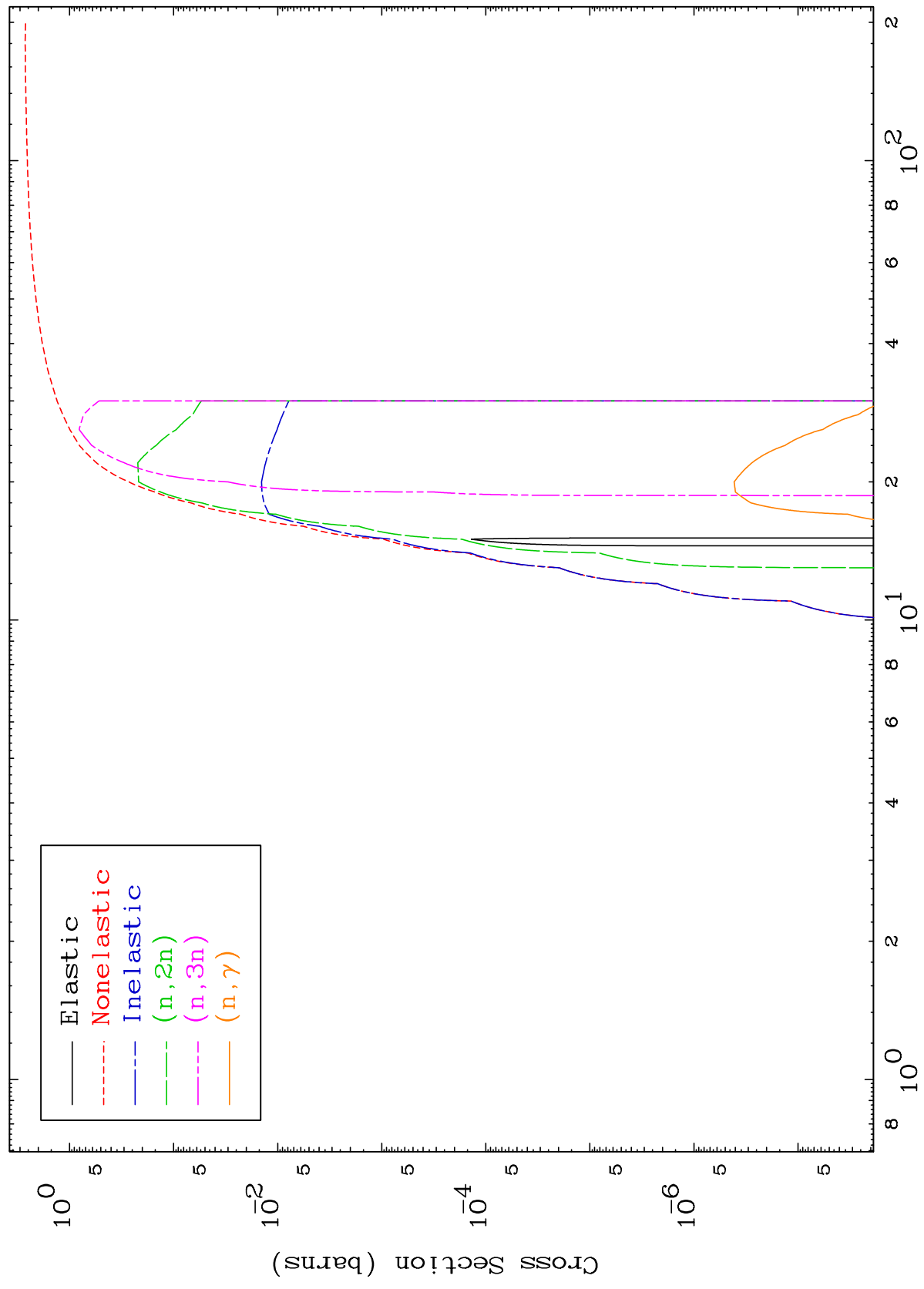
Press Mouse Button to Start

MAT 7146

0 Kelvin

Major Cross Sections

71-Lu-182



1

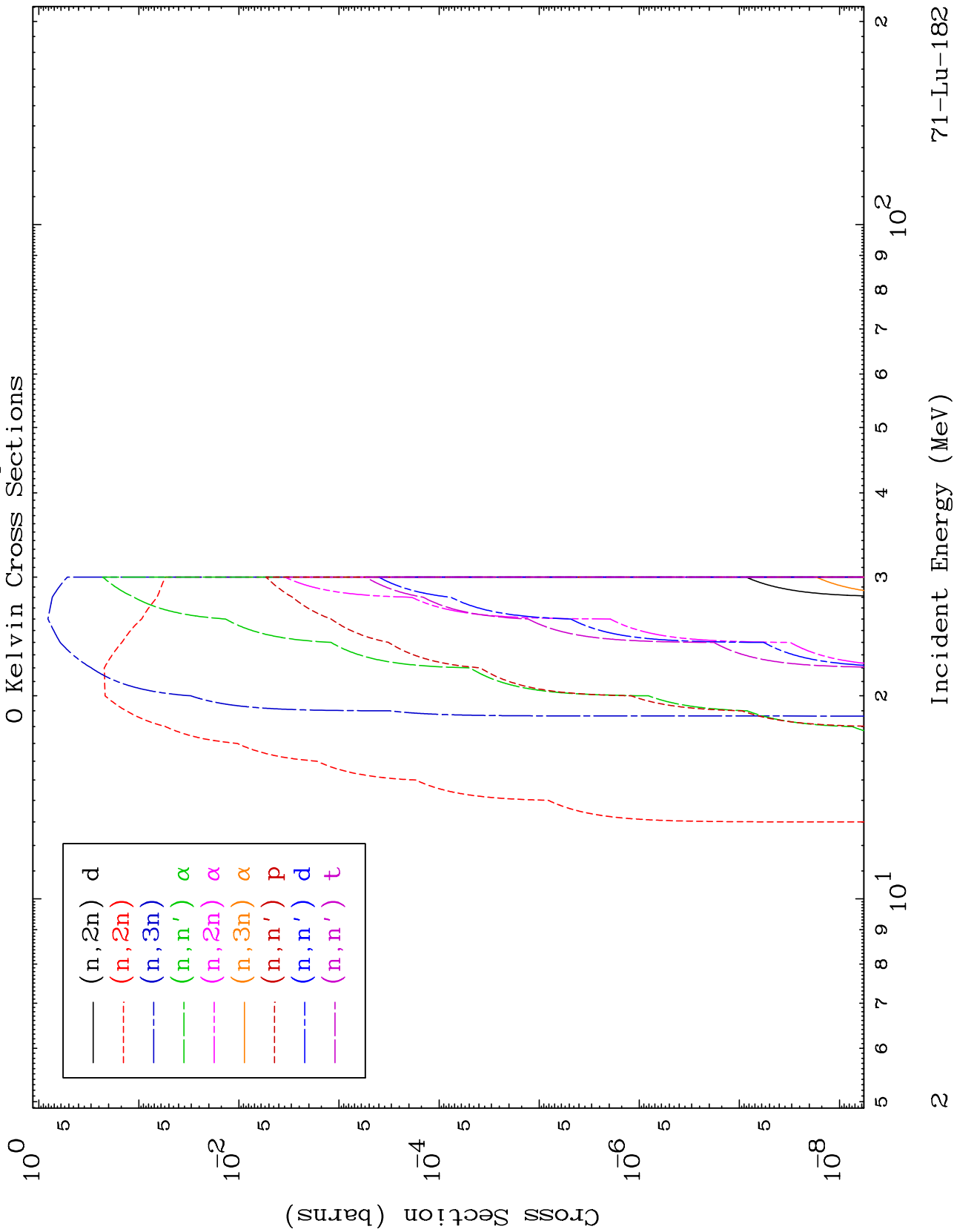
Incident Energy (MeV)

71-Lu-182

MAT 7146

α Neutron Absorption

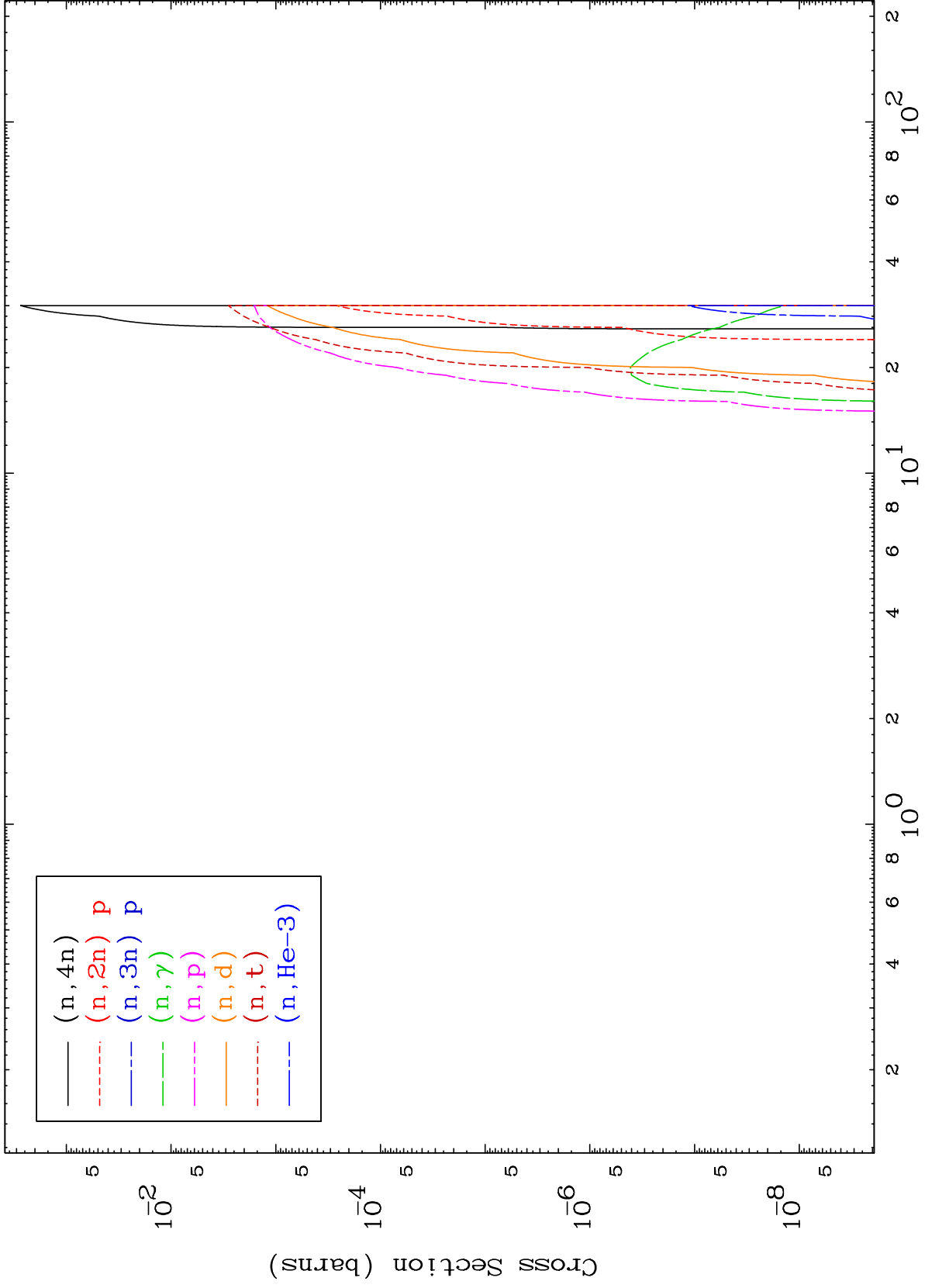
71-Lu-182

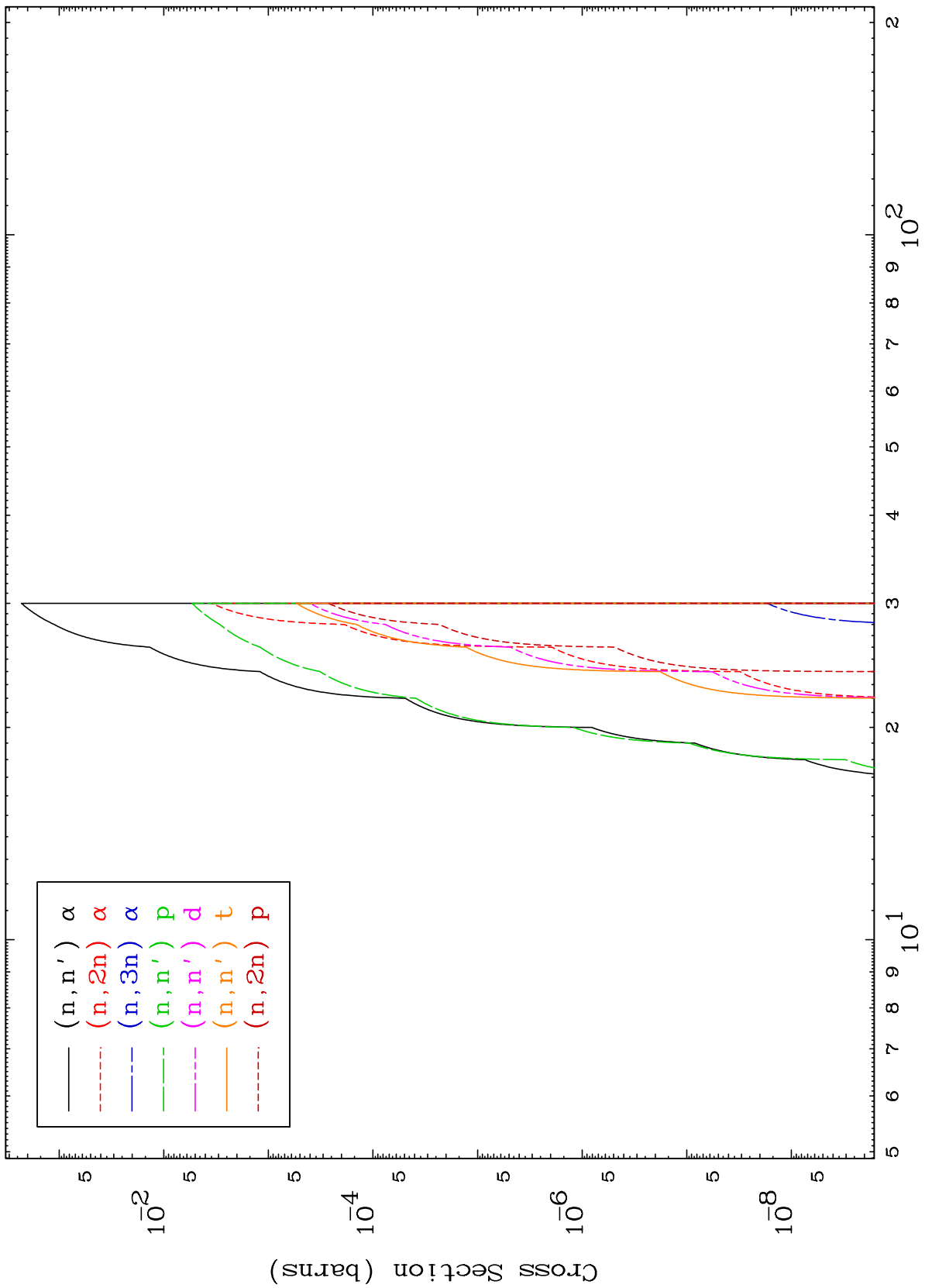


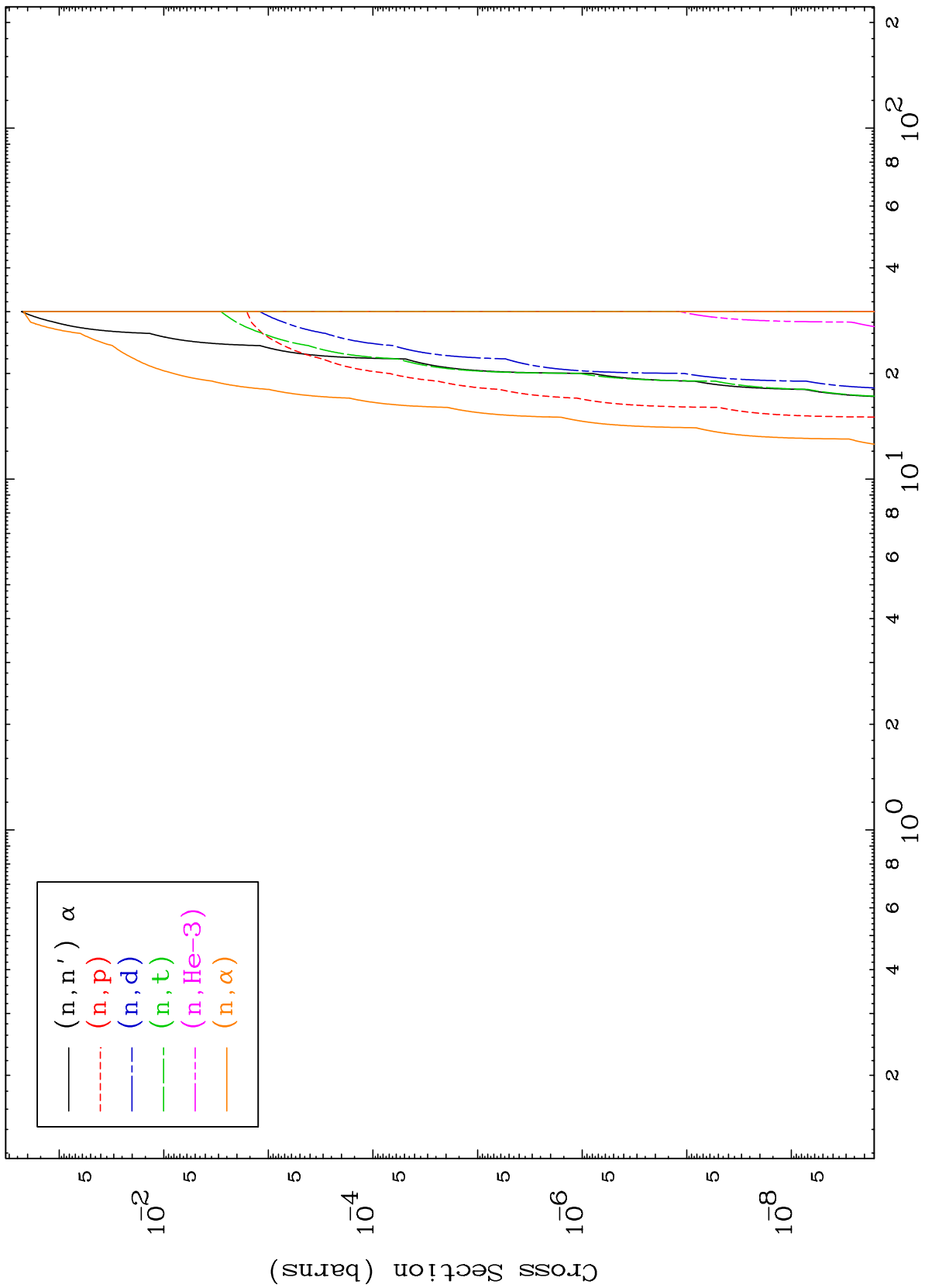
2

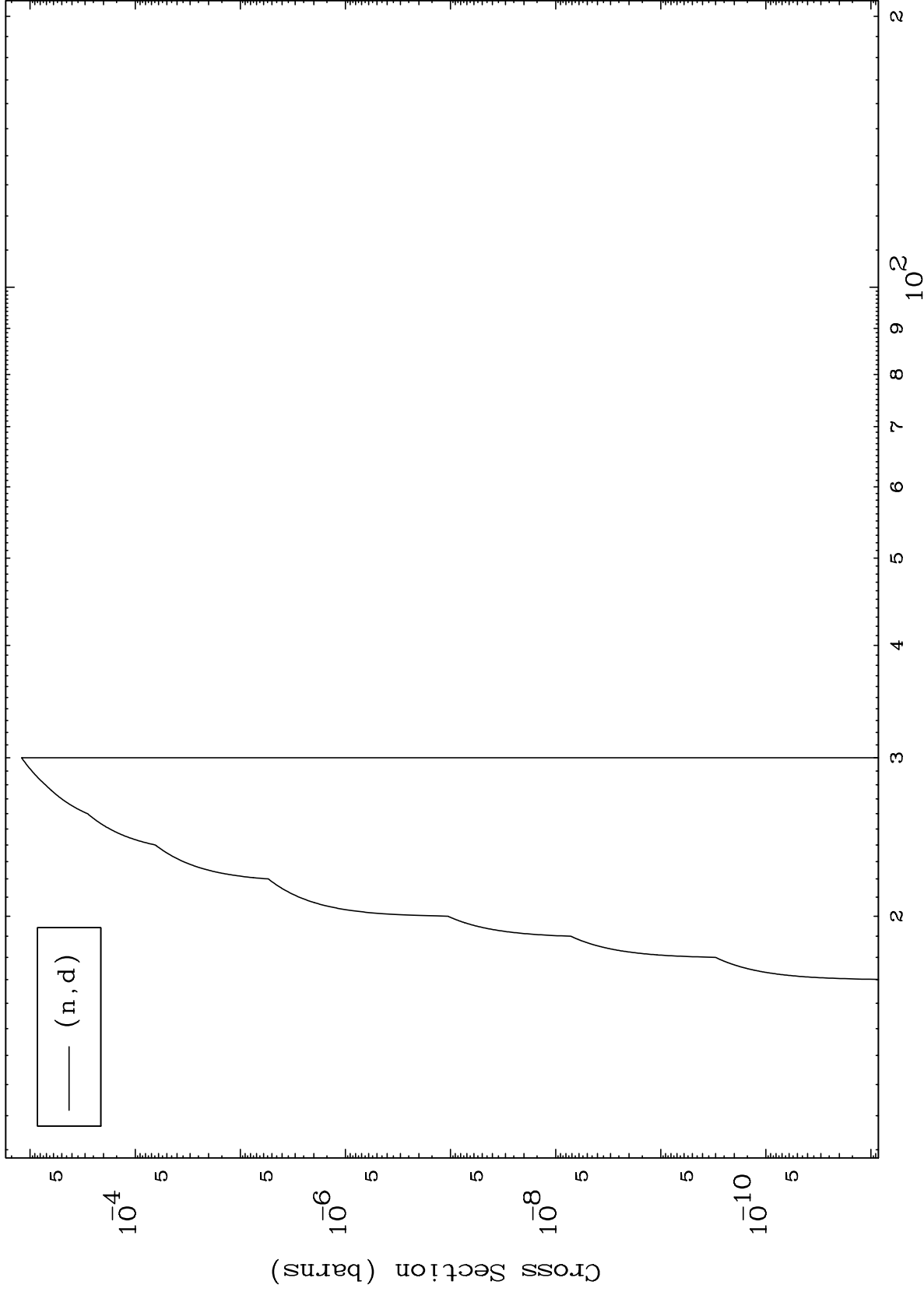
Incident Energy (MeV)

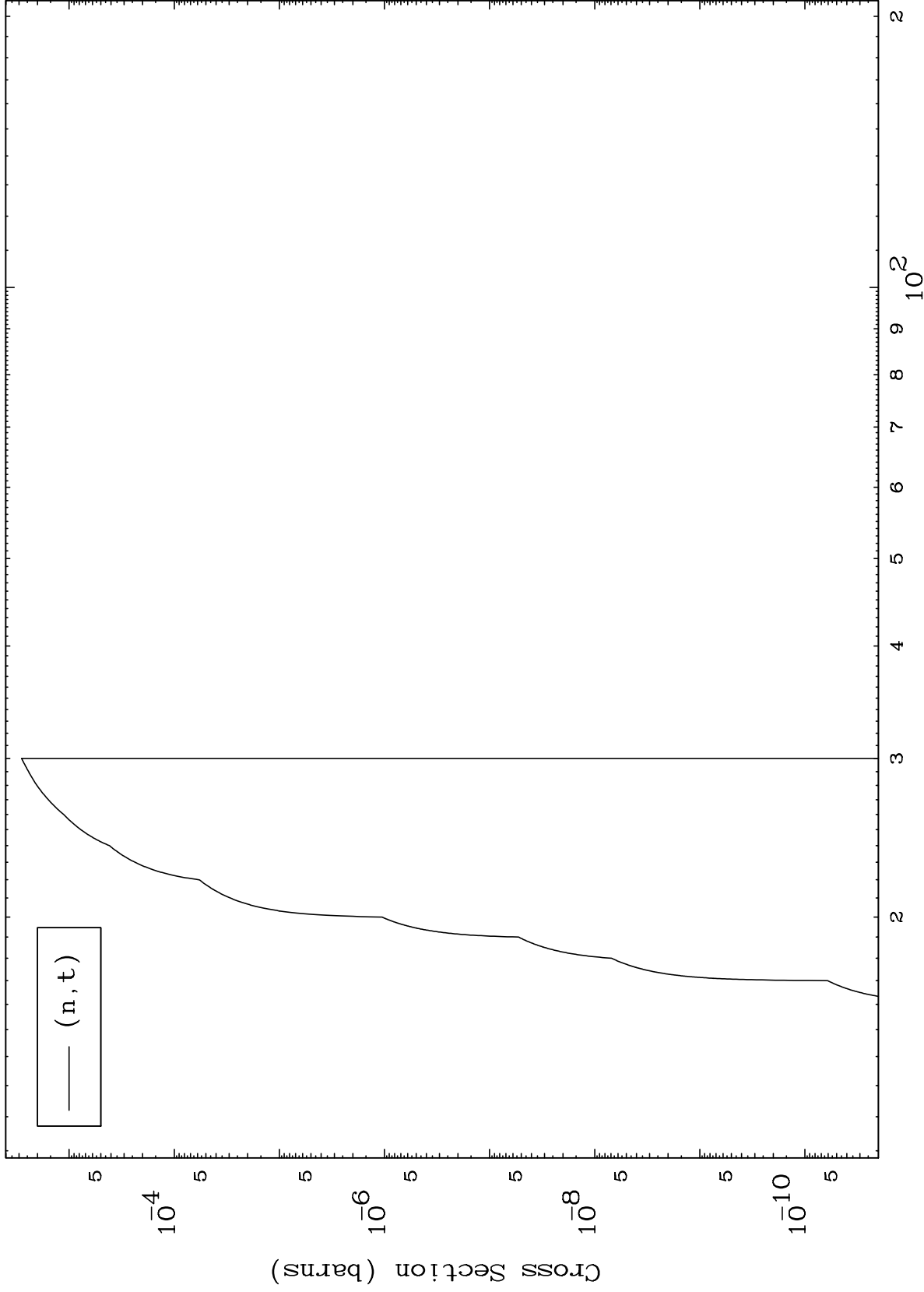
71-Lu-182

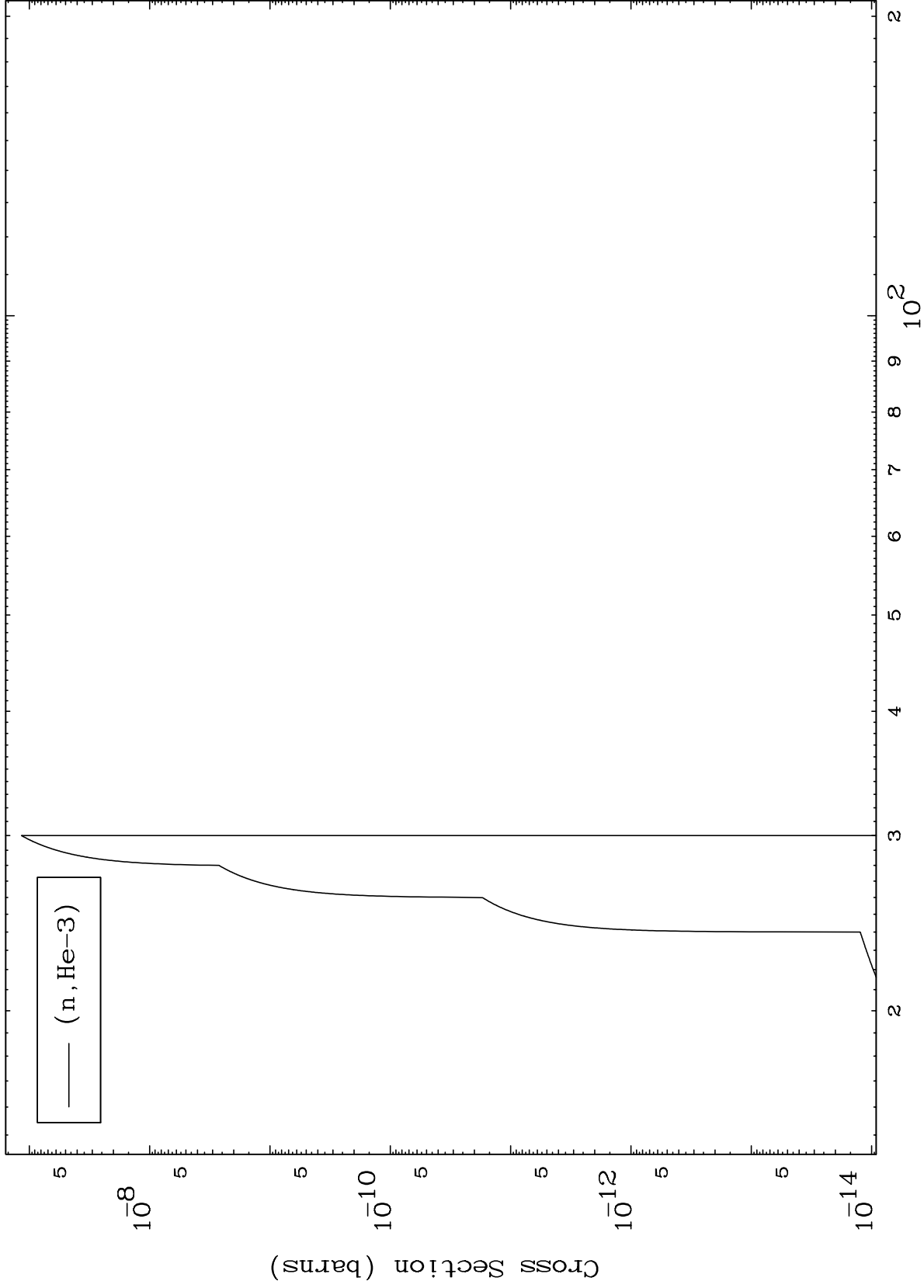










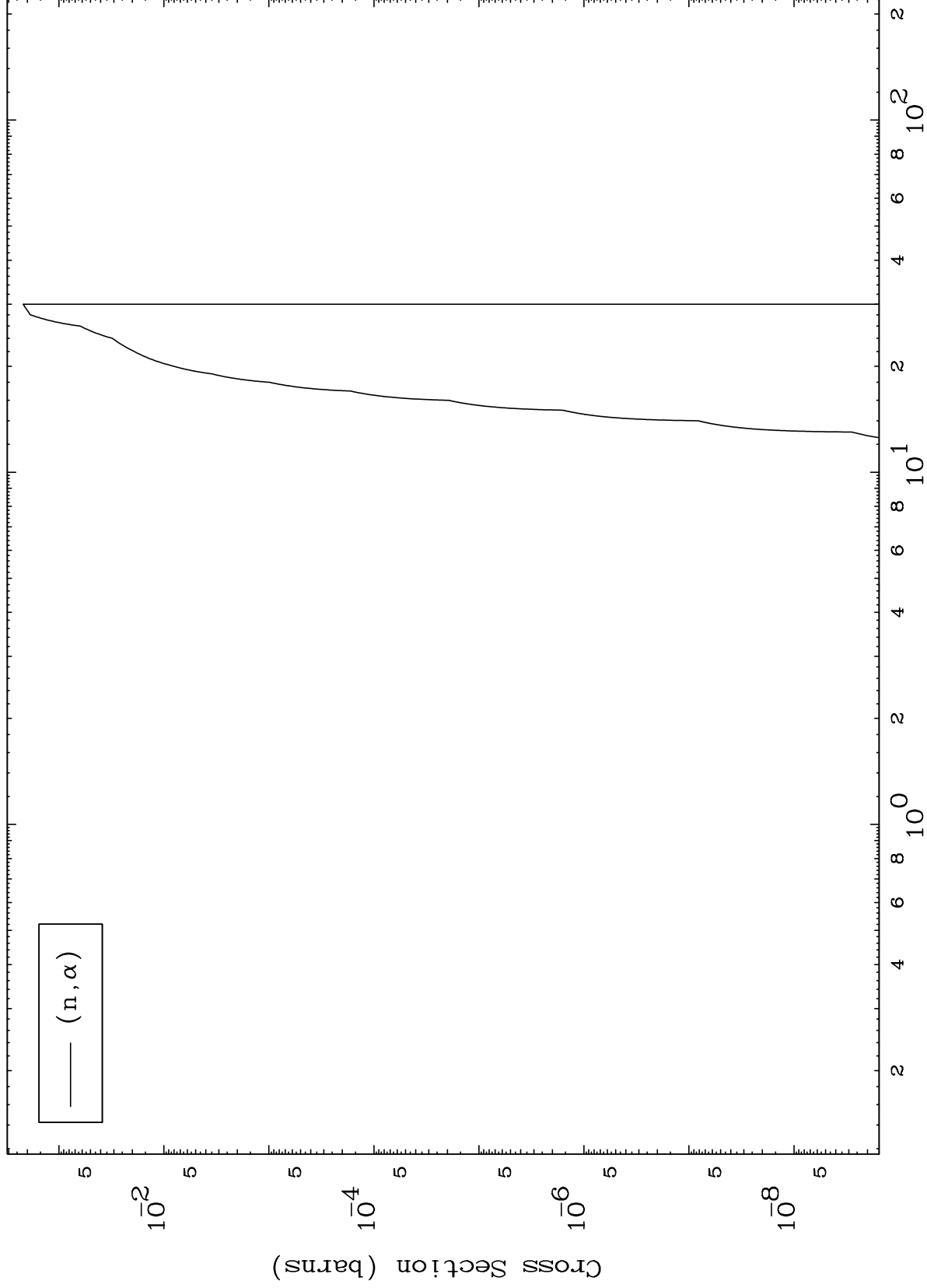


MAT 7146

(α, α) Levels

71-Lu-182

0 Kelvin Cross Sections

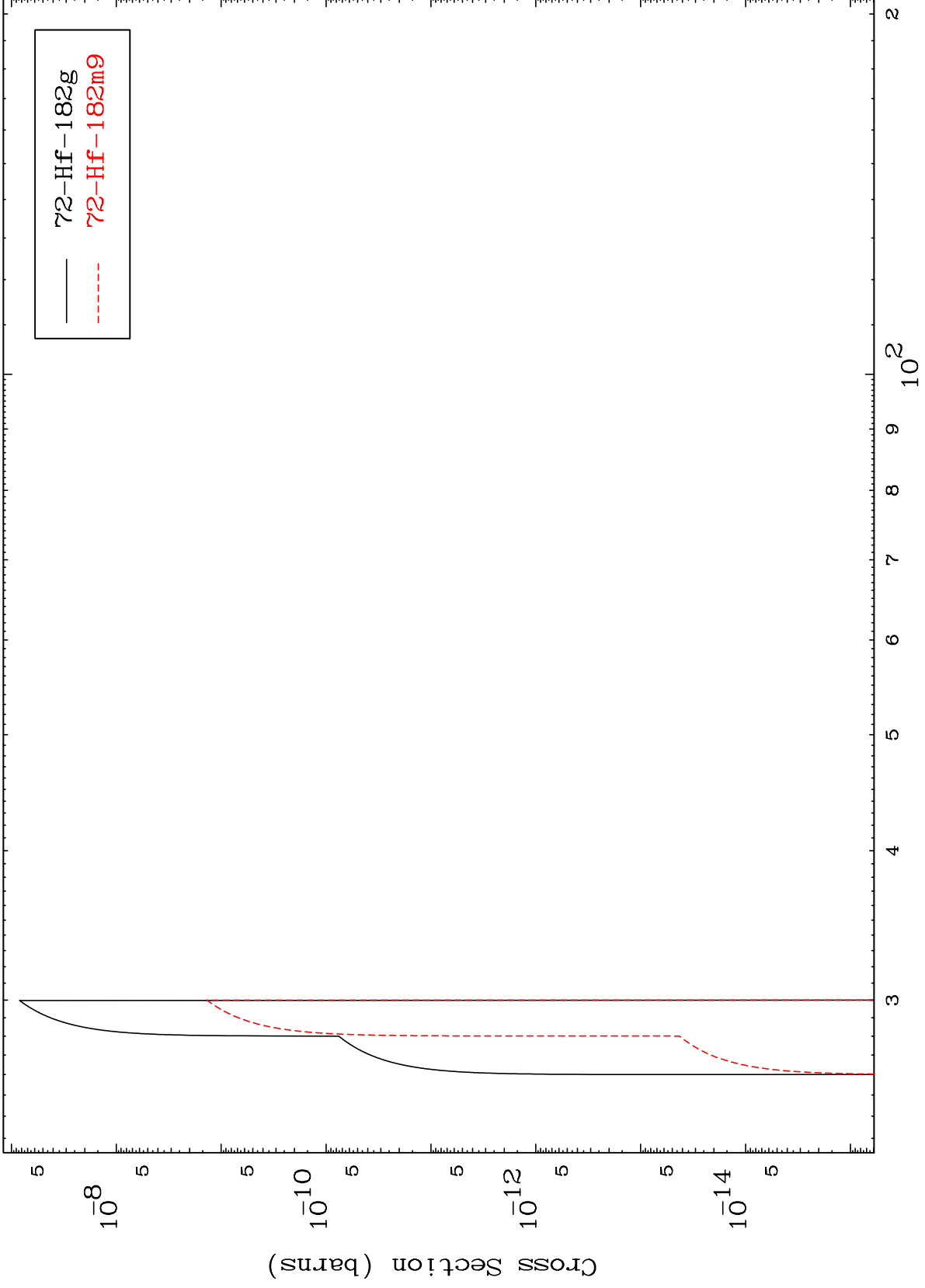


10

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section

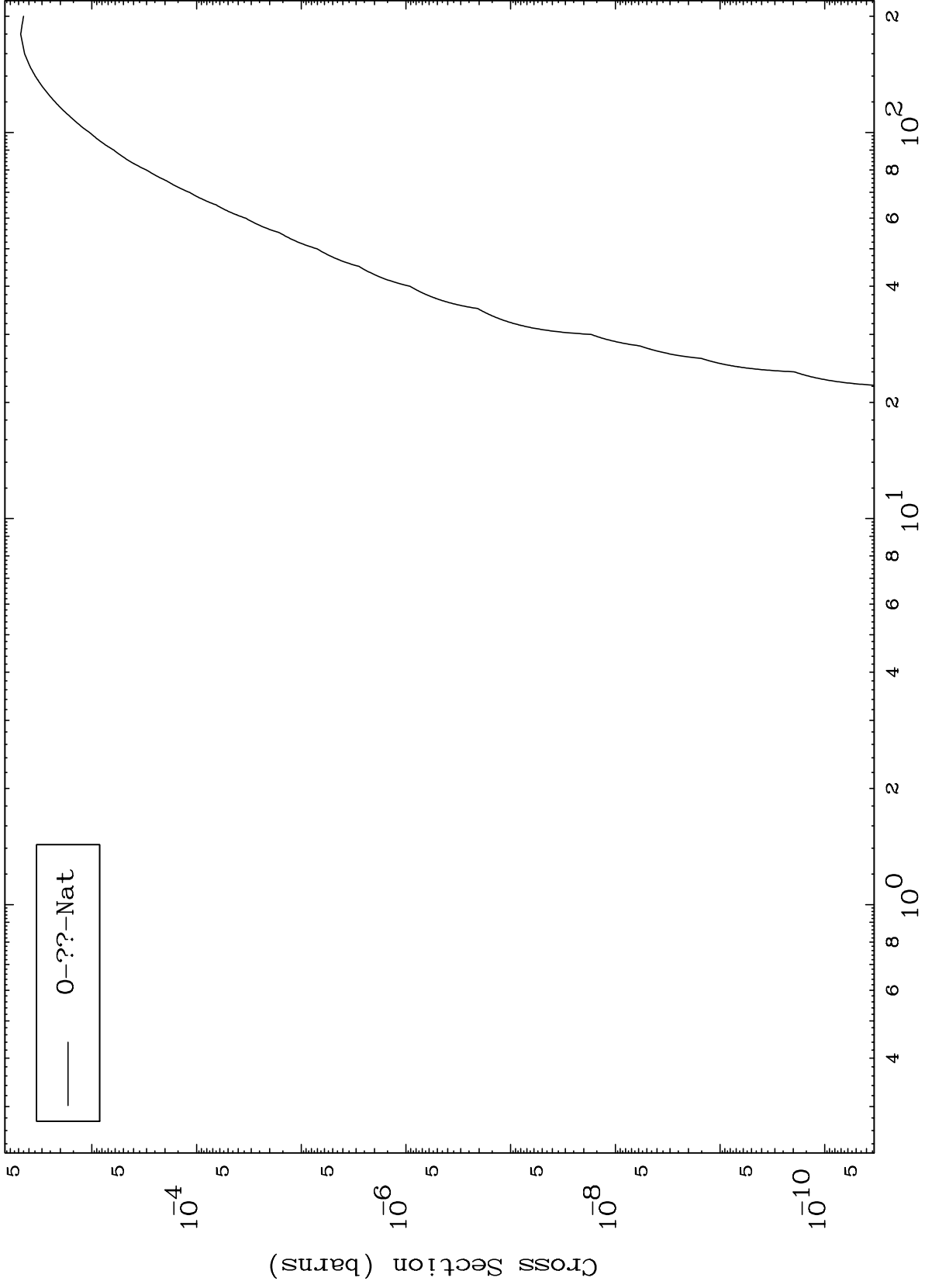


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Fission

71-Lu-182

Radionuclide Production Cross Section

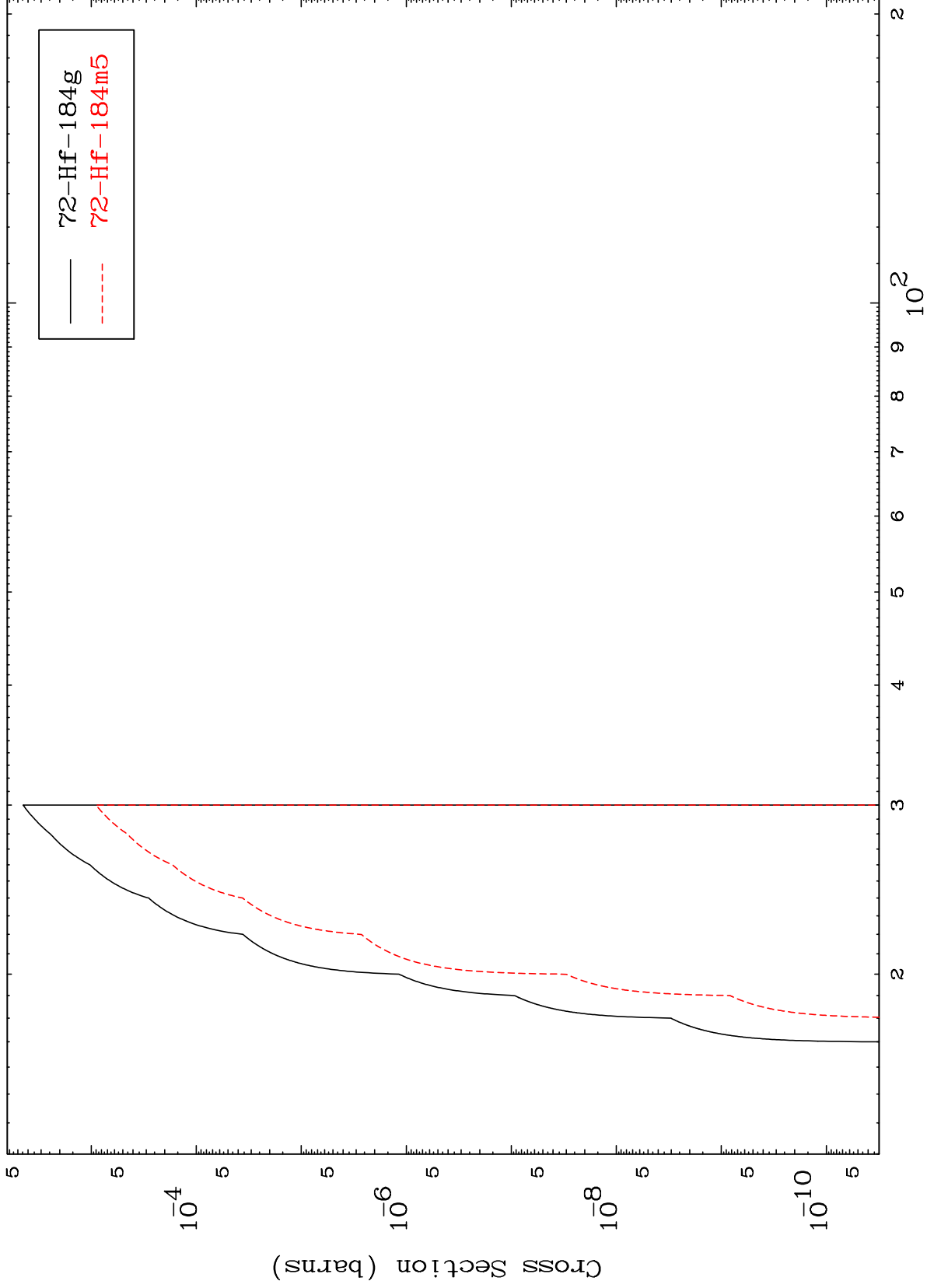


12

Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section

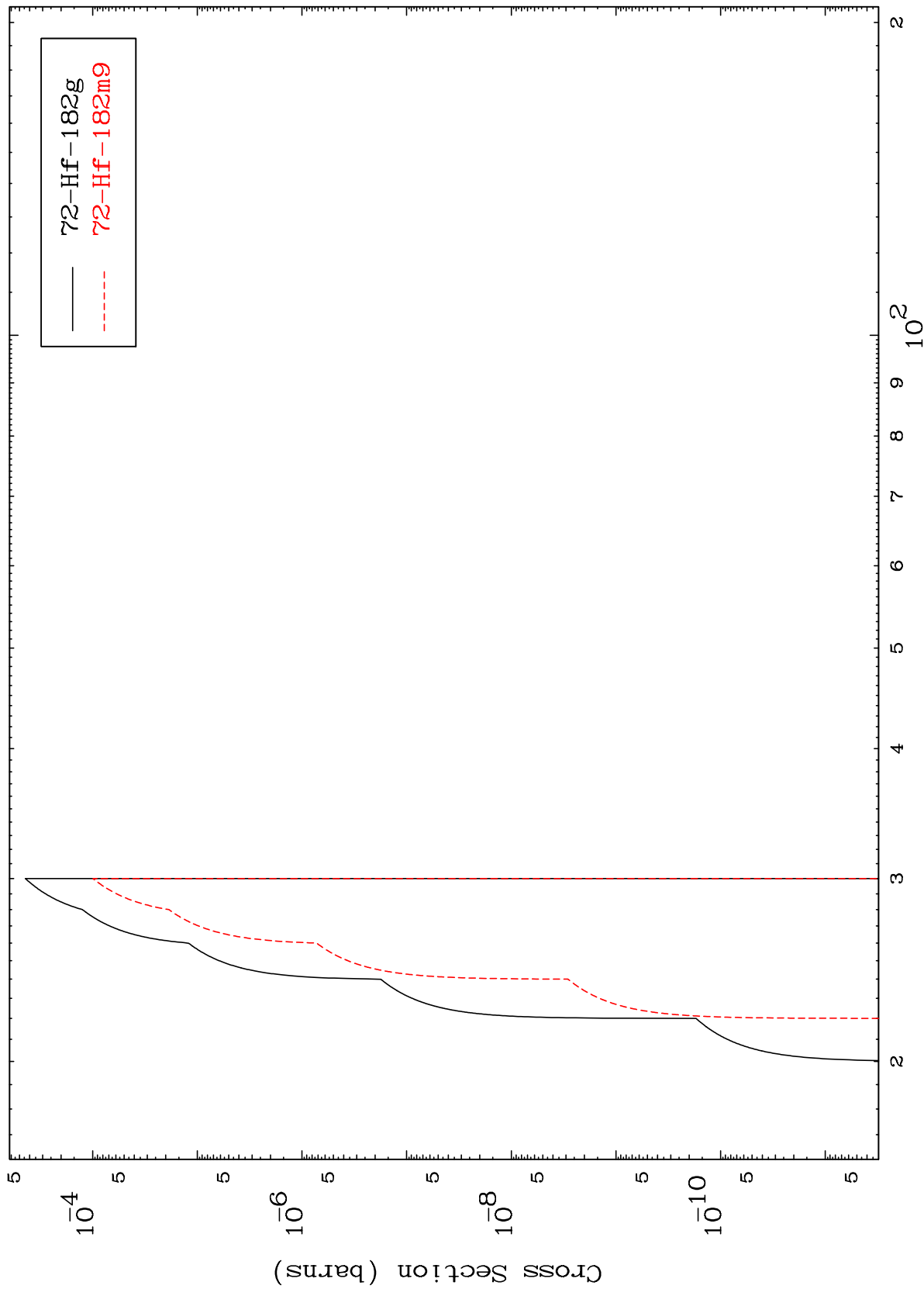


MAT 7146

(n,n') t

71-Lu-182

Radionuclide Production Cross Section

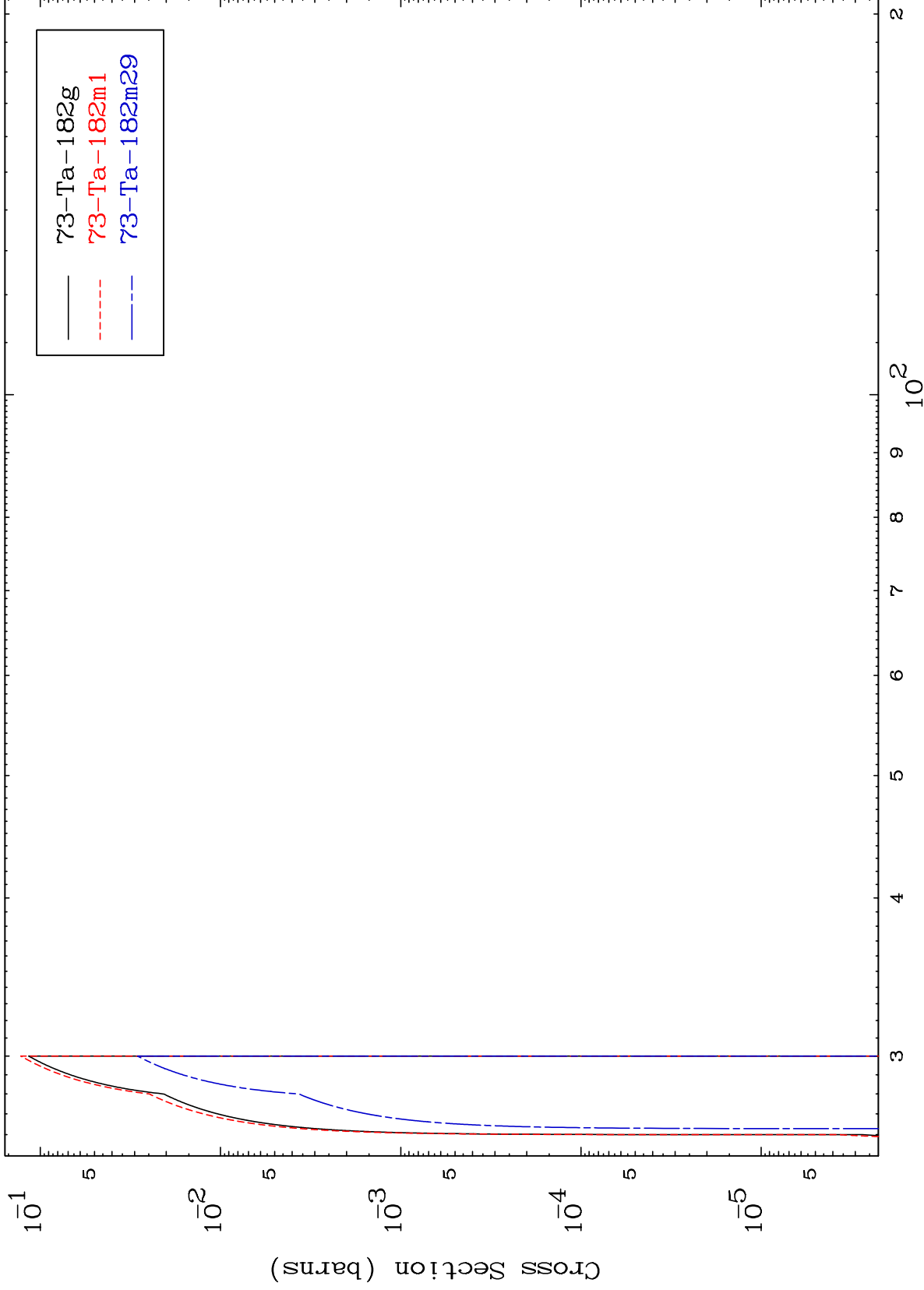


14

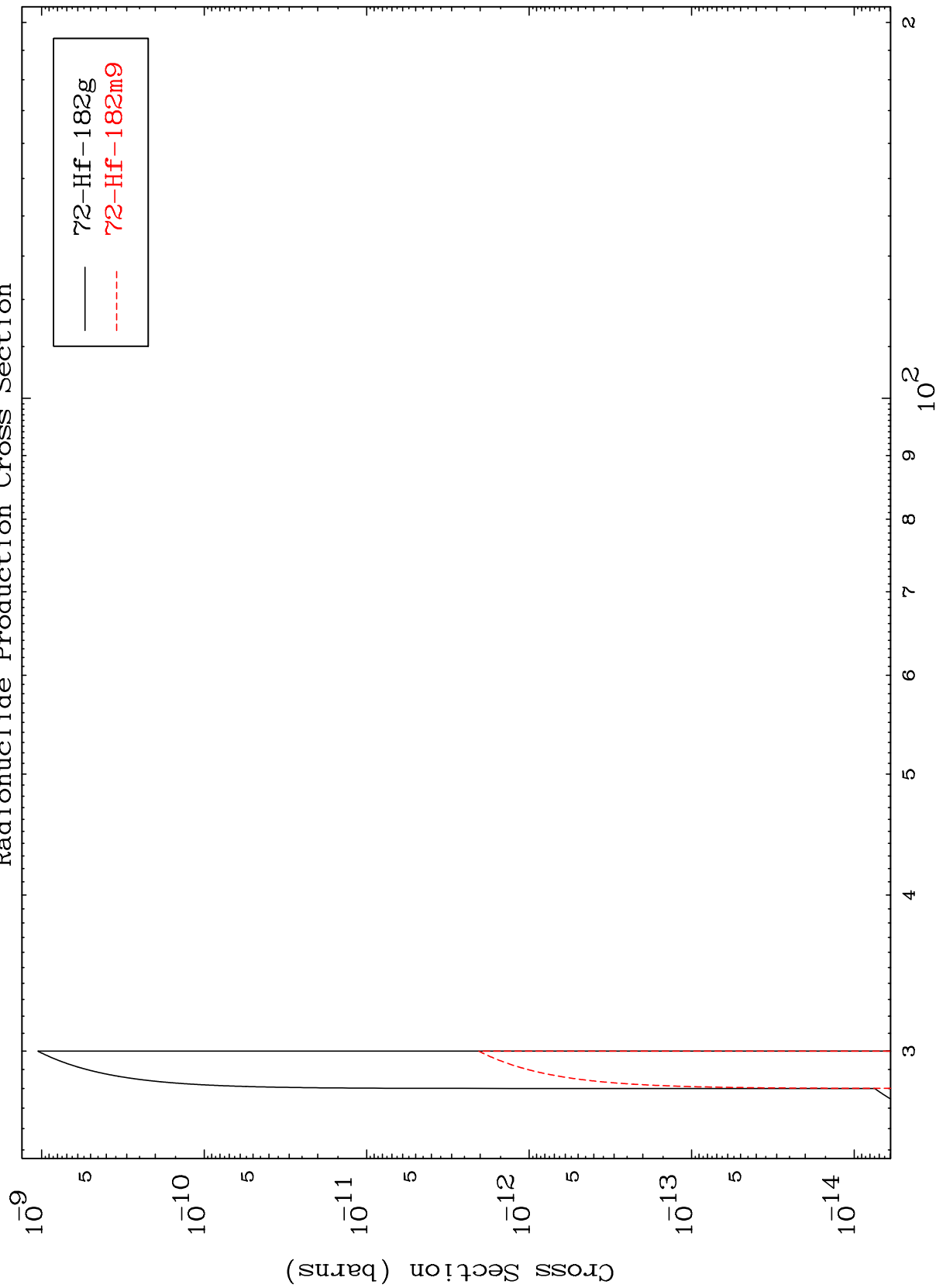
Incident Energy (MeV)

71-Lu-182

Radionuclide Production Cross Section



Radionuclide Production Cross Section



72-Hf-182g
72-Hf-182m9

(n,d)
Radionuclide Production Cross Section

