

Program EVALPLOT  
(Version 2021-1)

by

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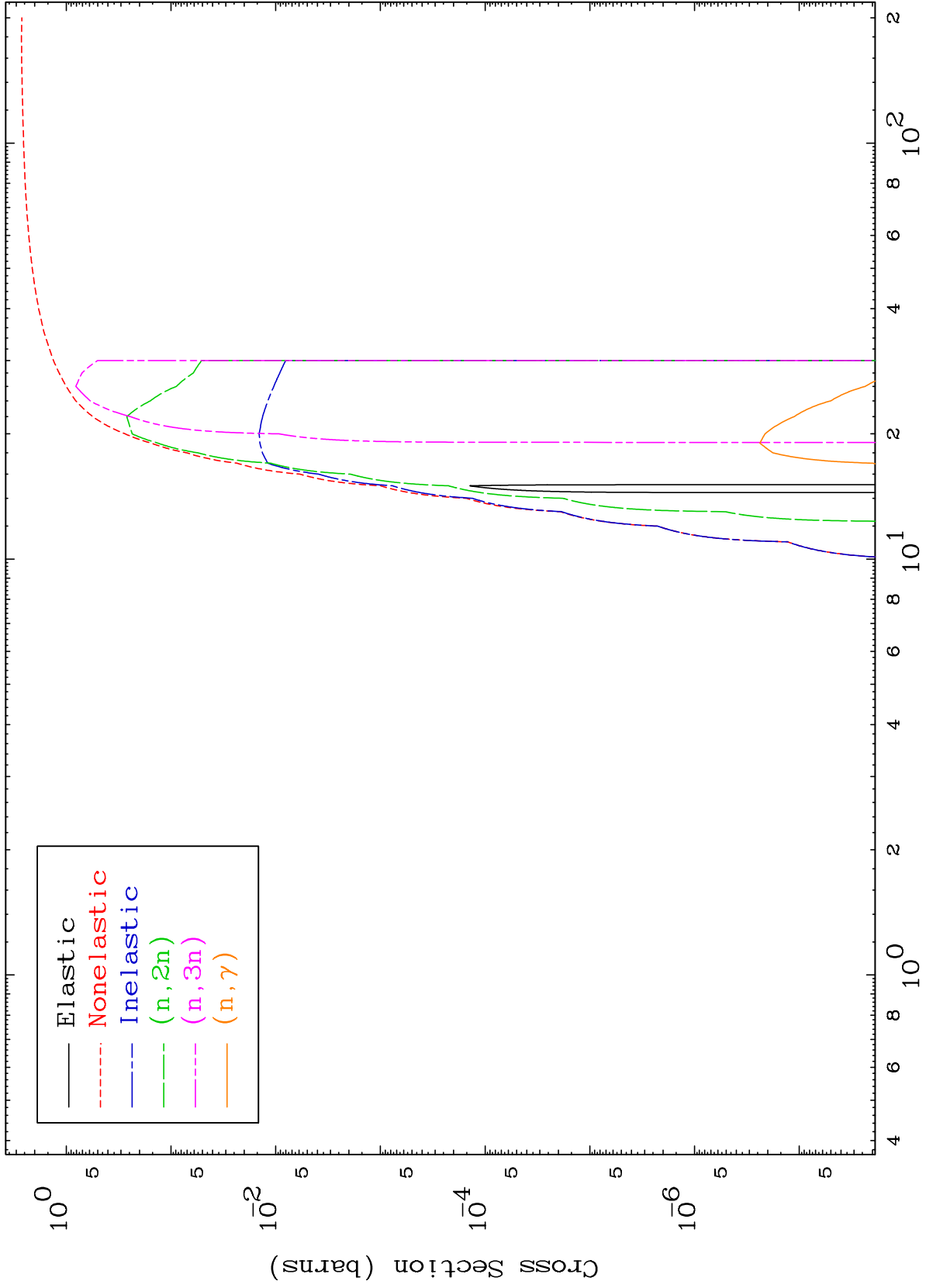
Press Mouse Button to Start

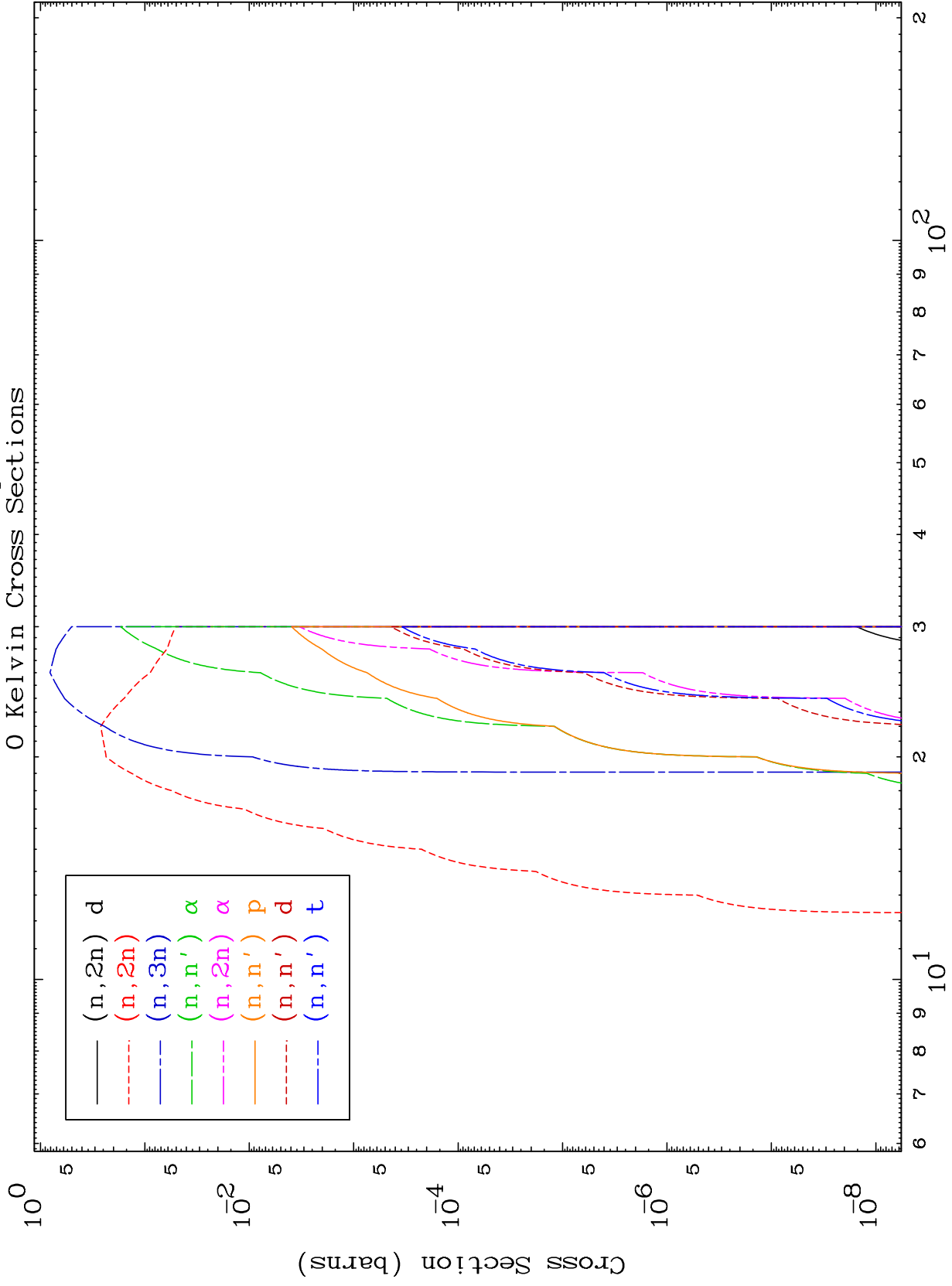
MAT 7149

0 Kelvin

Major Cross Sections

71-Lu-183

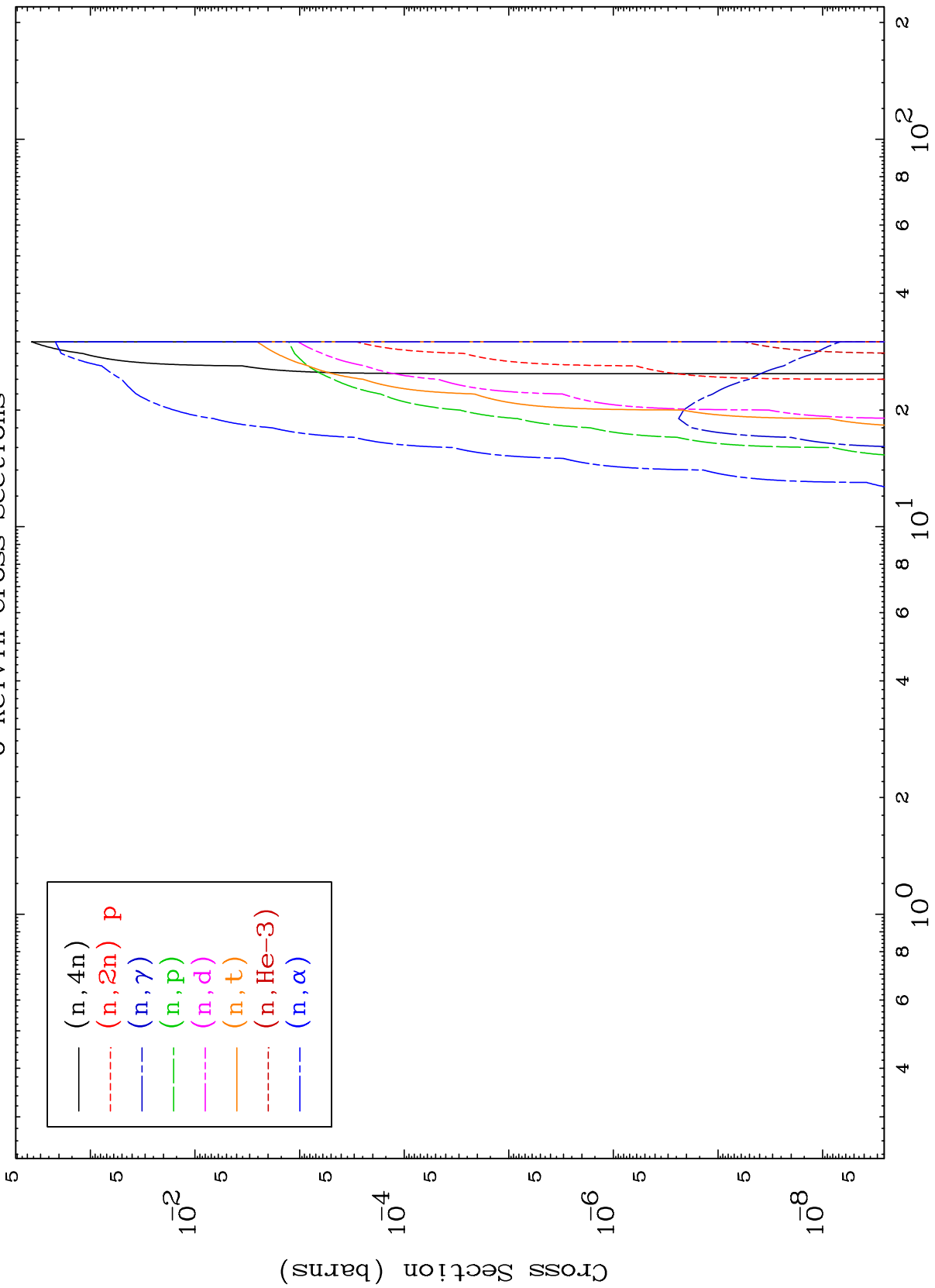




MAT 7149

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

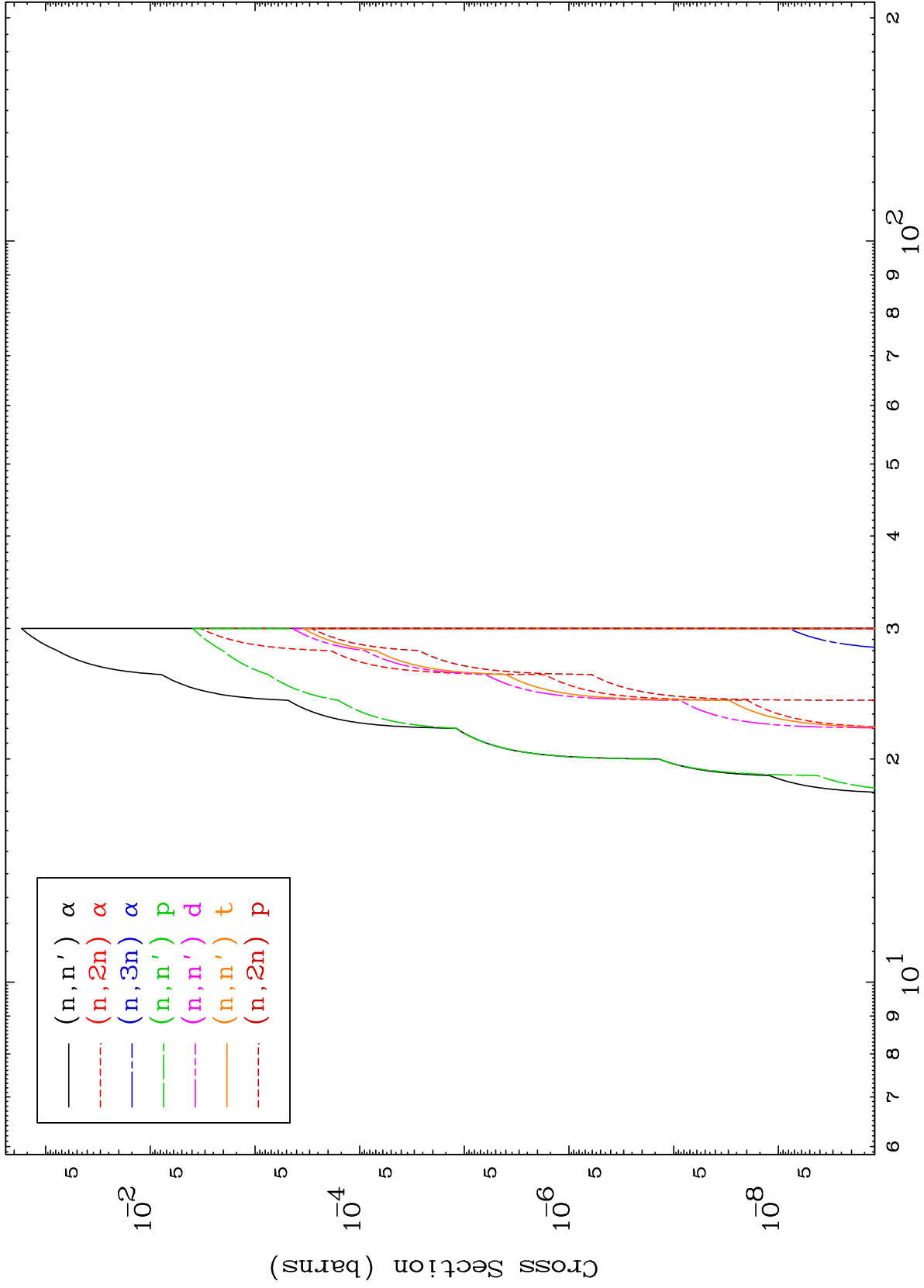
71-Lu-183

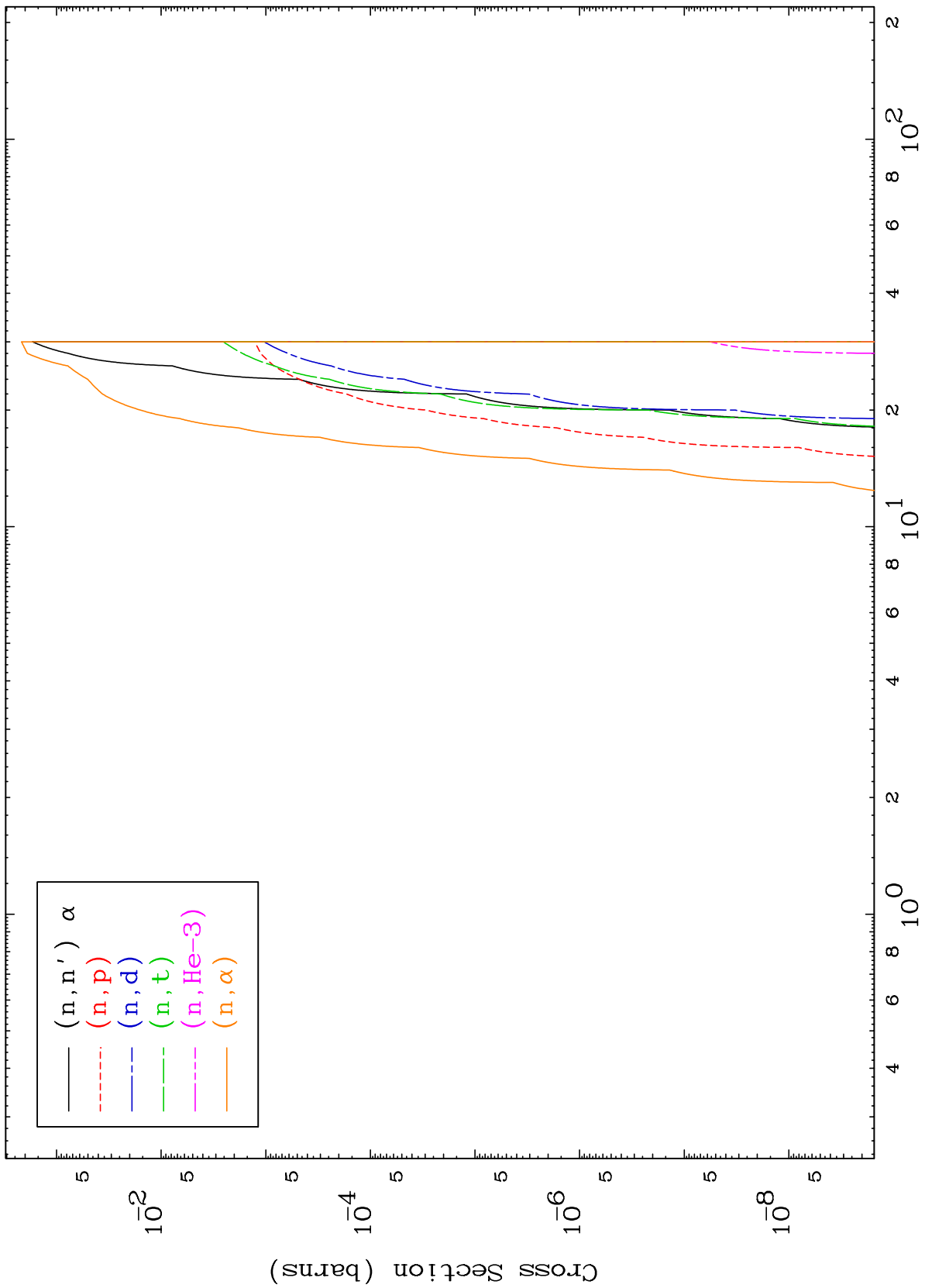


3

Incident Energy (MeV)

71-Lu-183

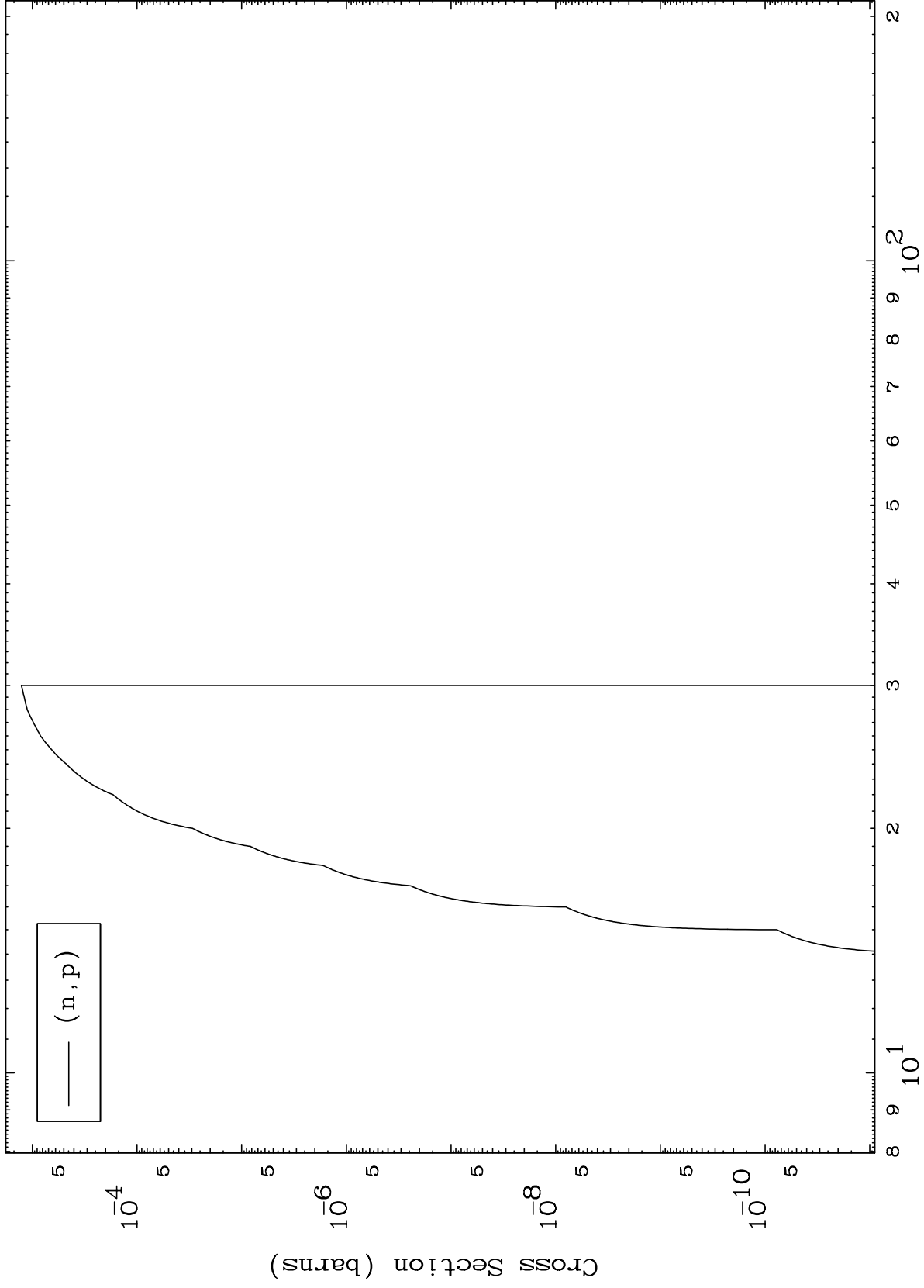




MAT 7149

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

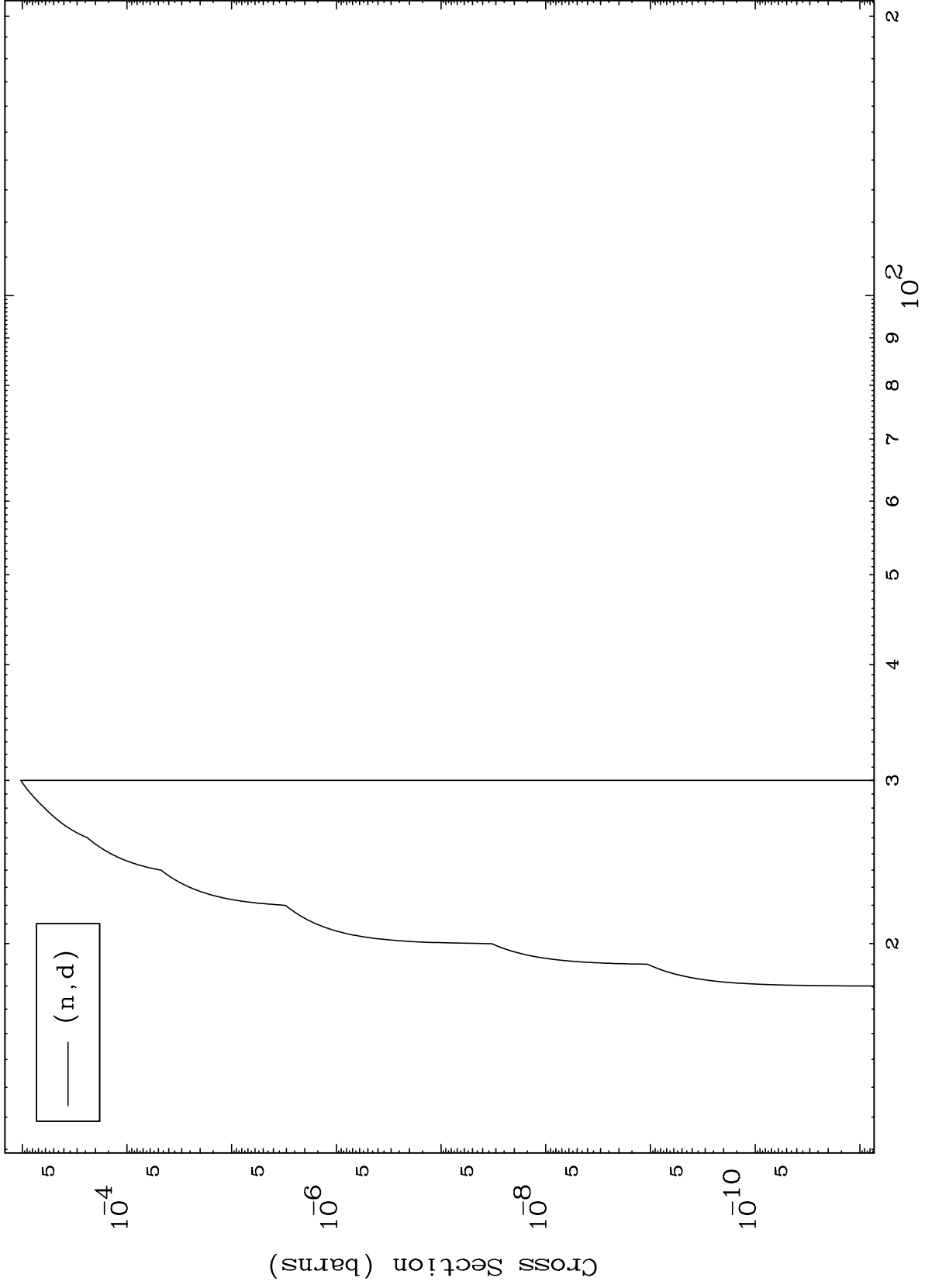
71-Lu-183



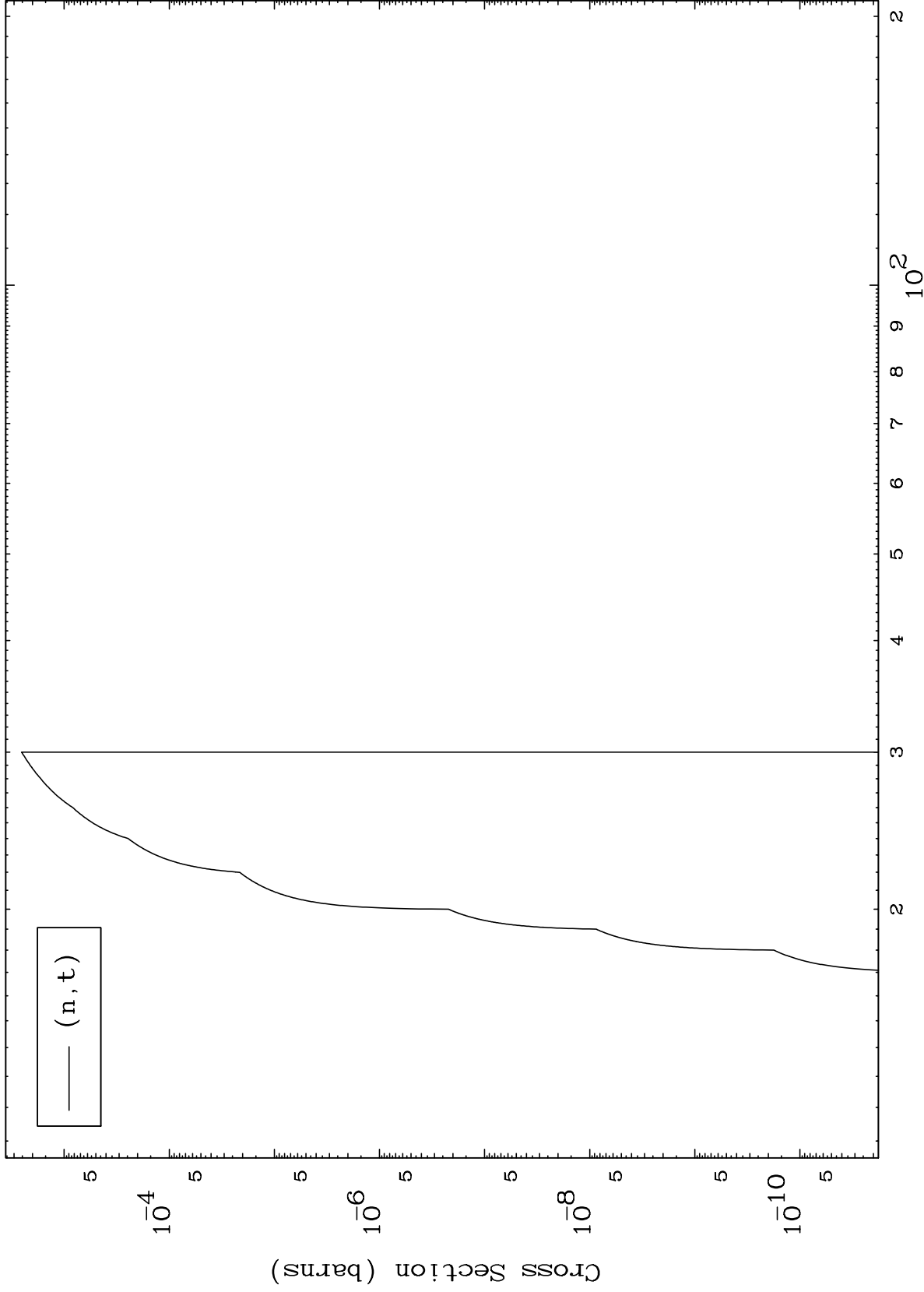
Incident Energy (MeV)

71-Lu-183

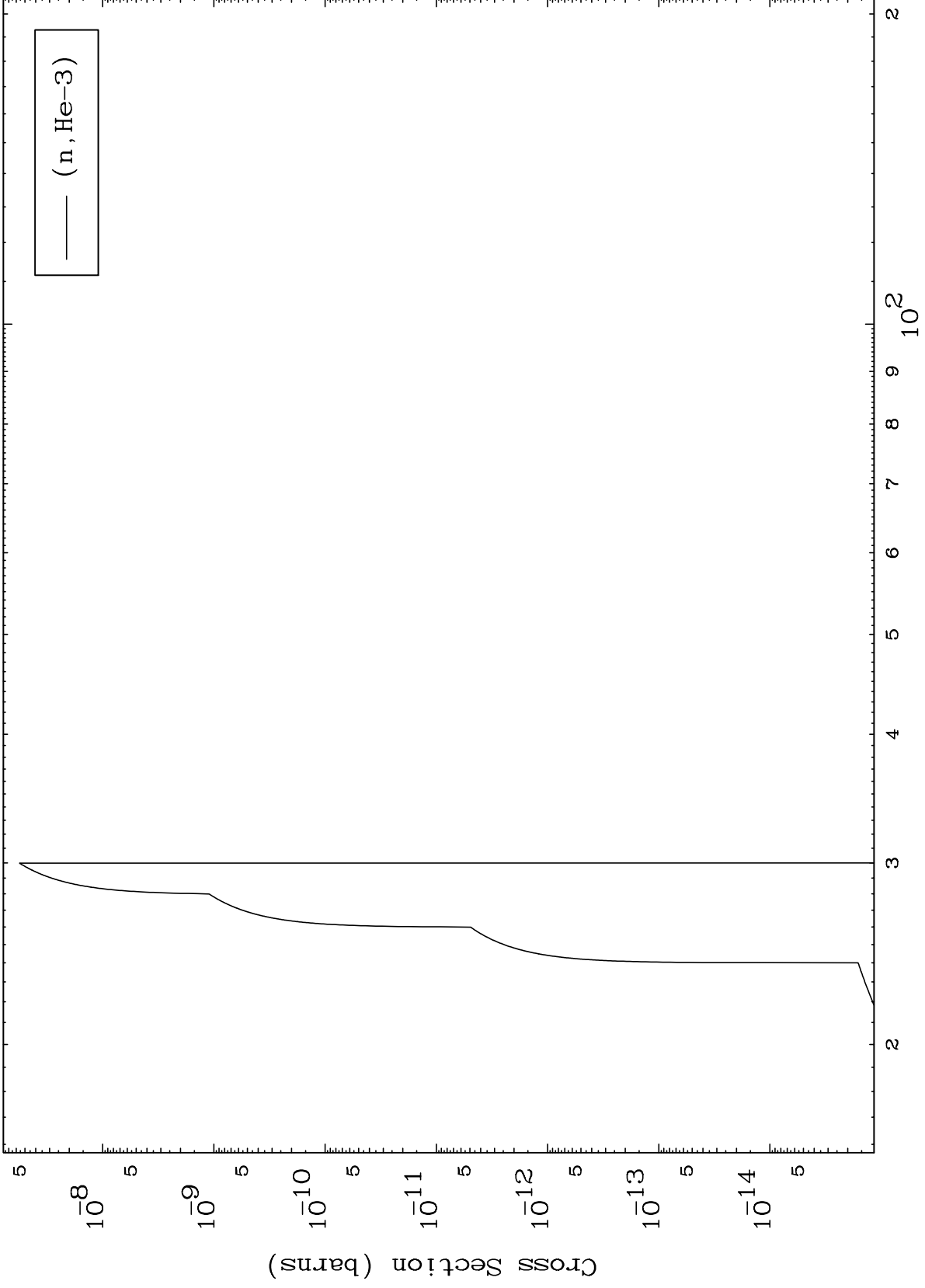
6







( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

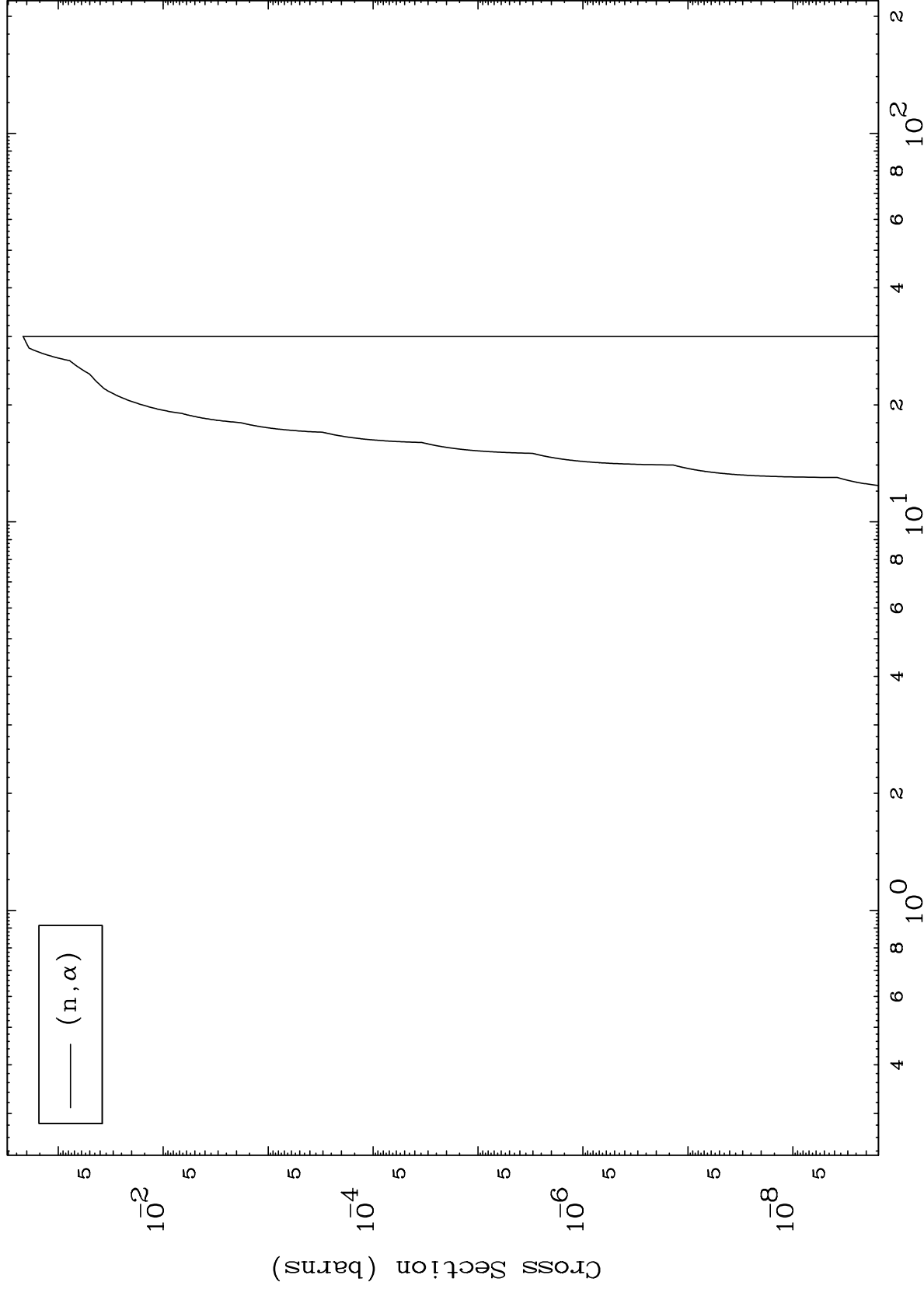


MAT 7149

( $\alpha, \alpha$ ) Levels

71-Lu-183

0 Kelvin Cross Sections



10

Incident Energy (MeV)

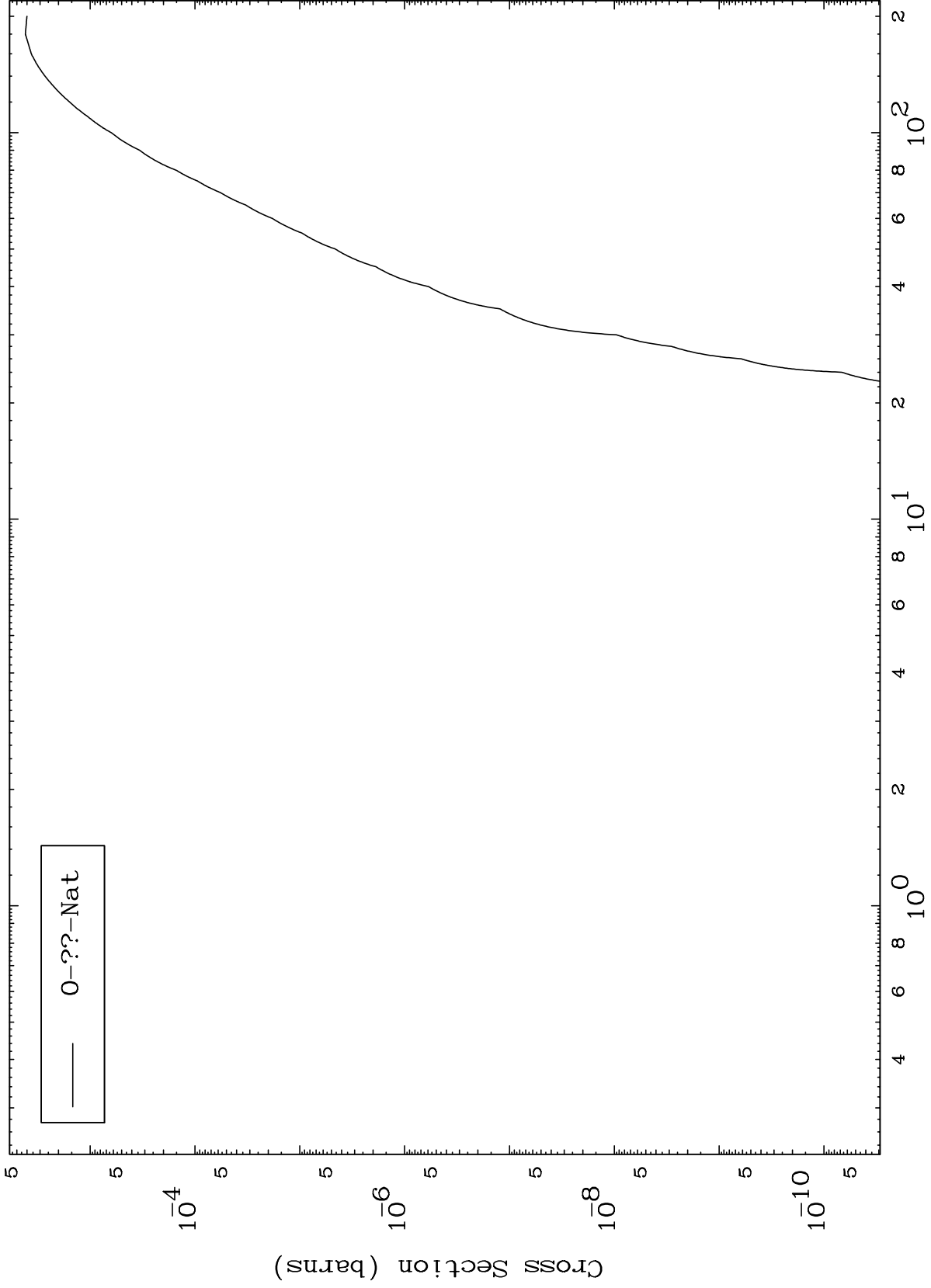
71-Lu-183

MAT 7149

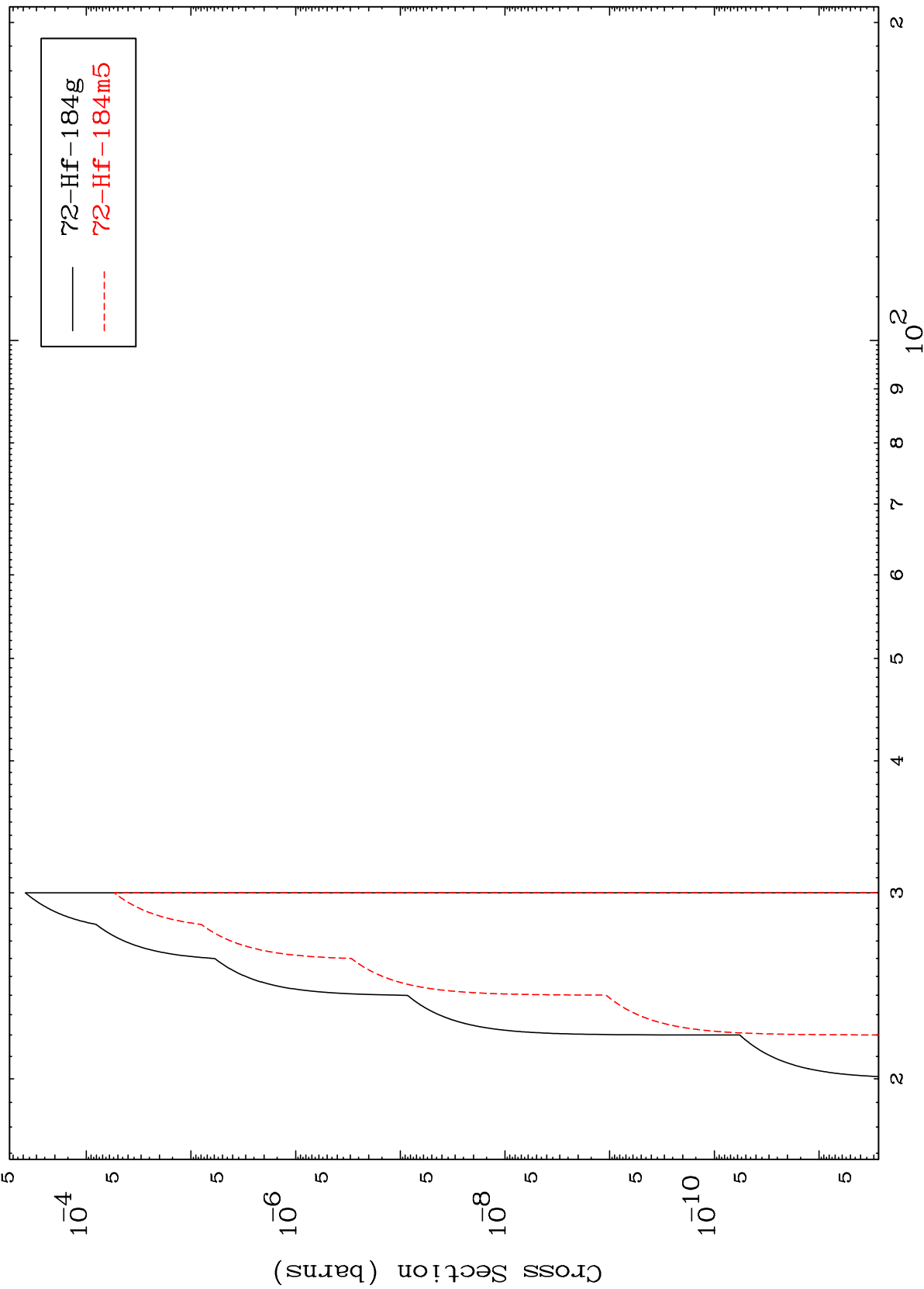
Fission

<sup>71</sup>Lu-183

Radionuclide Production Cross Section



Radionuclide Production Cross Section

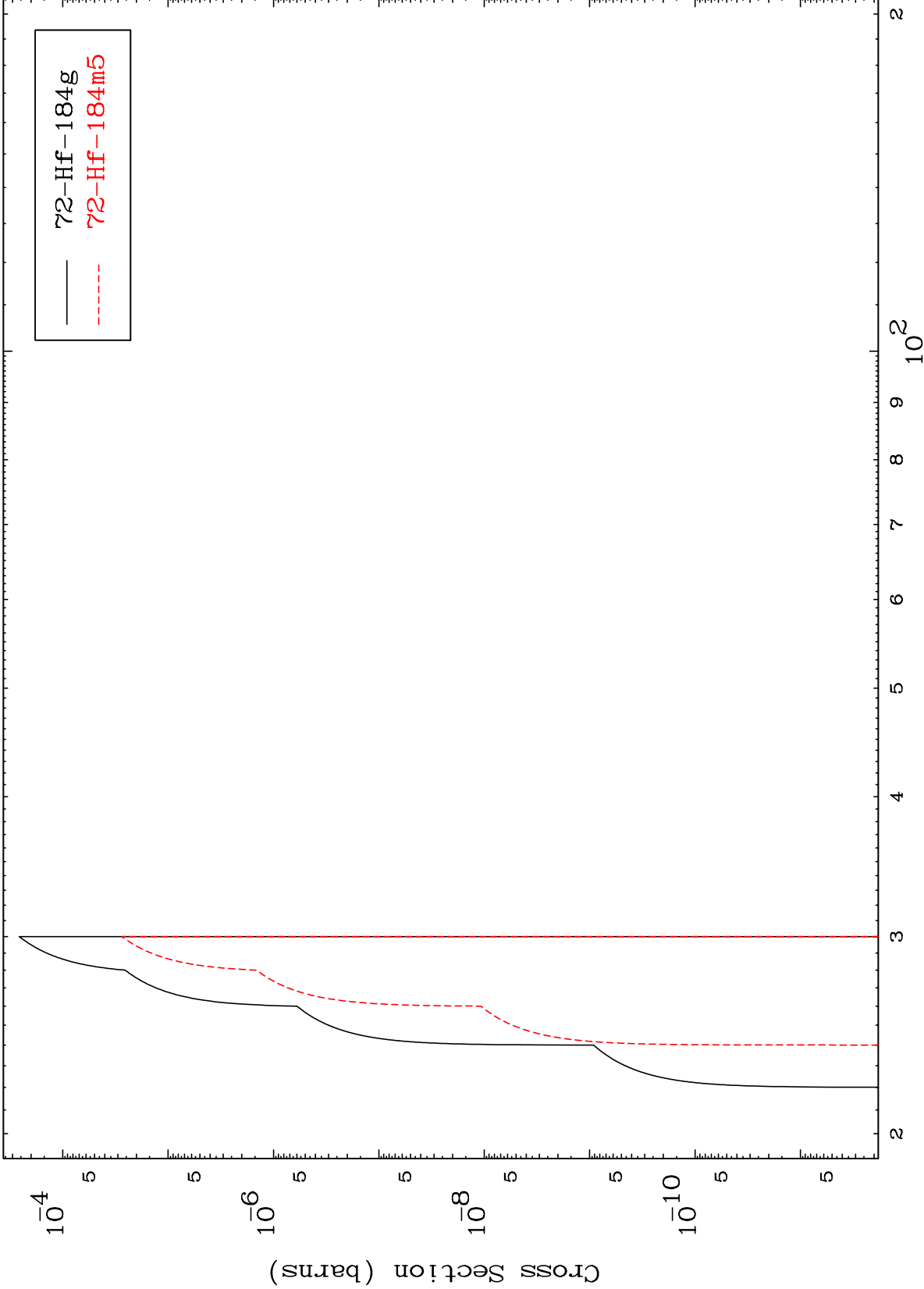


MAT 7149

(n,2n) p

71-Lu-183

Radionuclide Production Cross Section



13

Incident Energy (MeV)

71-Lu-183

(n, t)  
Radionuclide Production Cross Section

