

Program EVALPLOT
(Version 2021-1)

by

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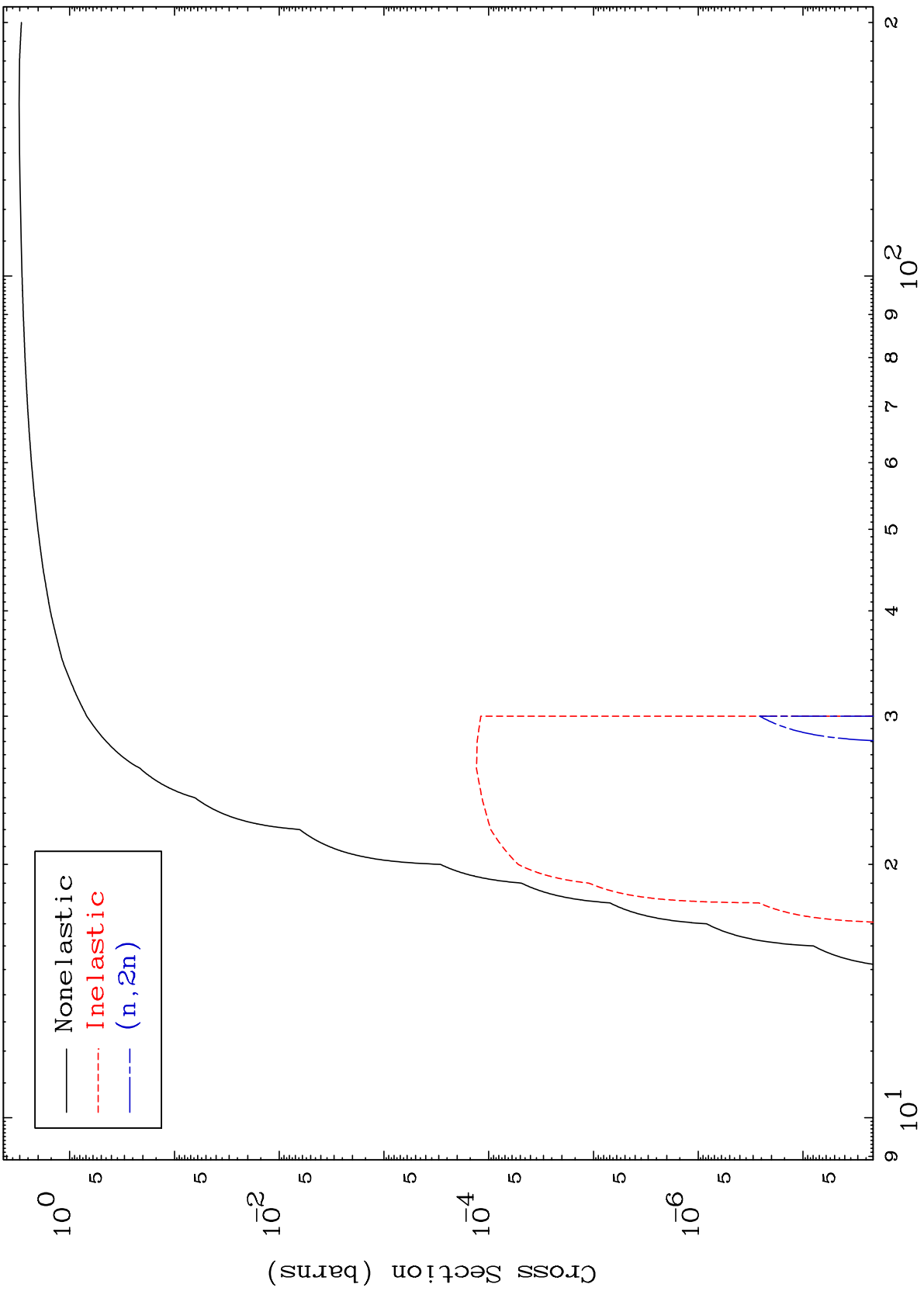
Press Mouse Button to Start

MAT 9949

α Major

101-Md-248

0 Kelvin Cross Sections

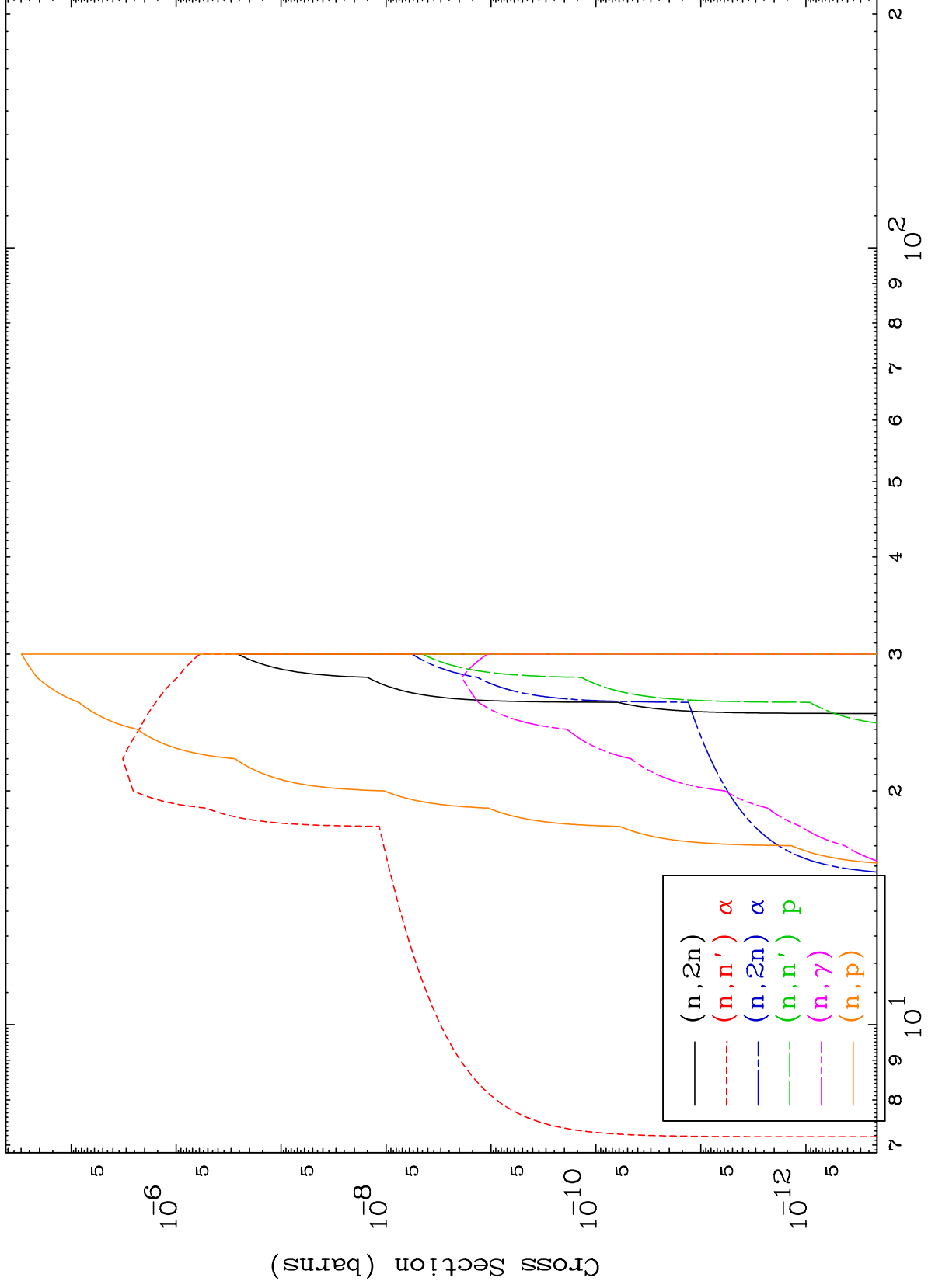


Legend:
— Nonelastic
- - - Inelastic
- . - (n, 2n)

101-Md-248

Incident Energy (MeV)

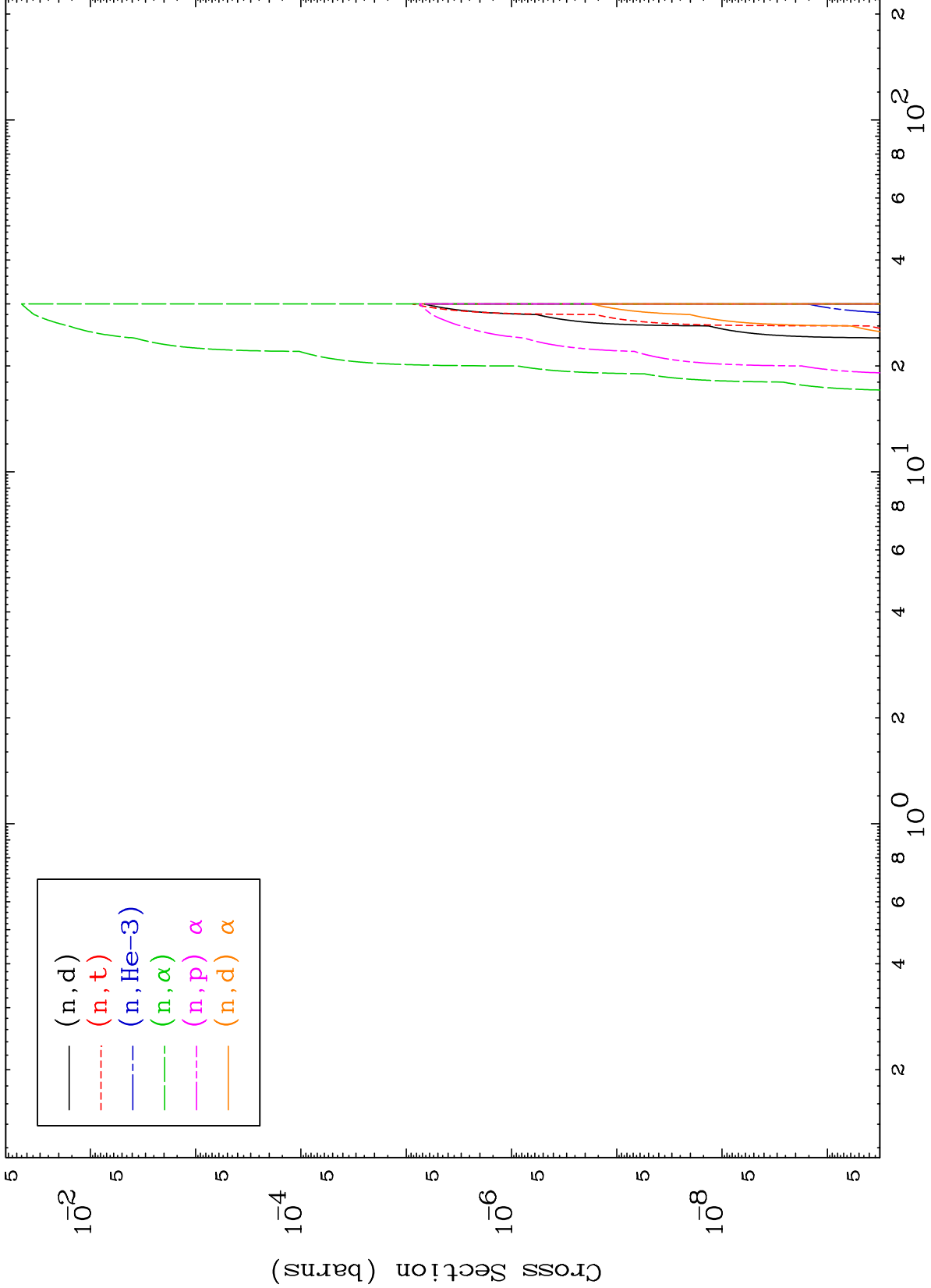
1



MAT 9949

α Neutron Absorption
0 Kelvin Cross Sections

101-Md-248

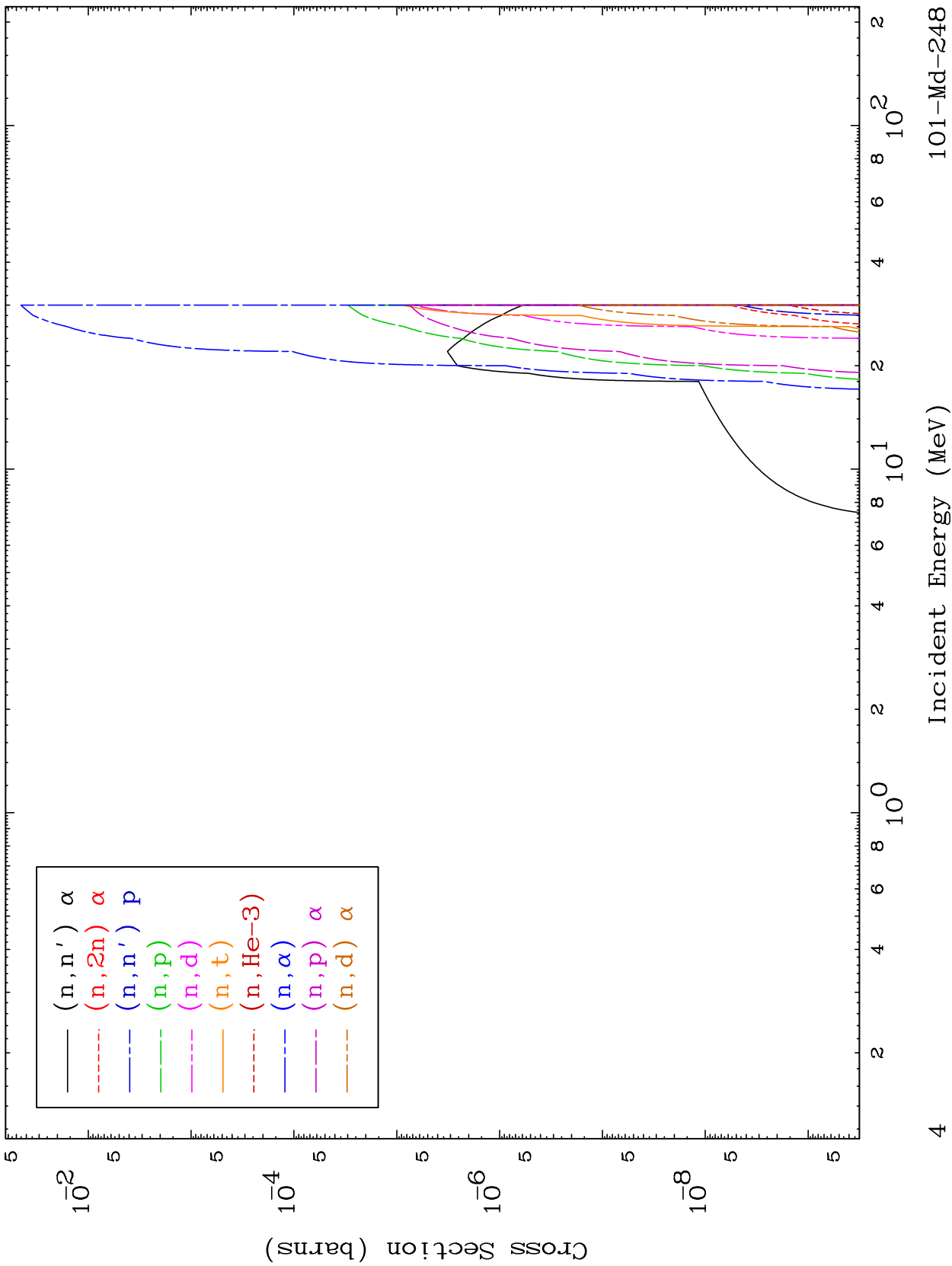


MAT 9949

α Charged Particle

101-Md-248

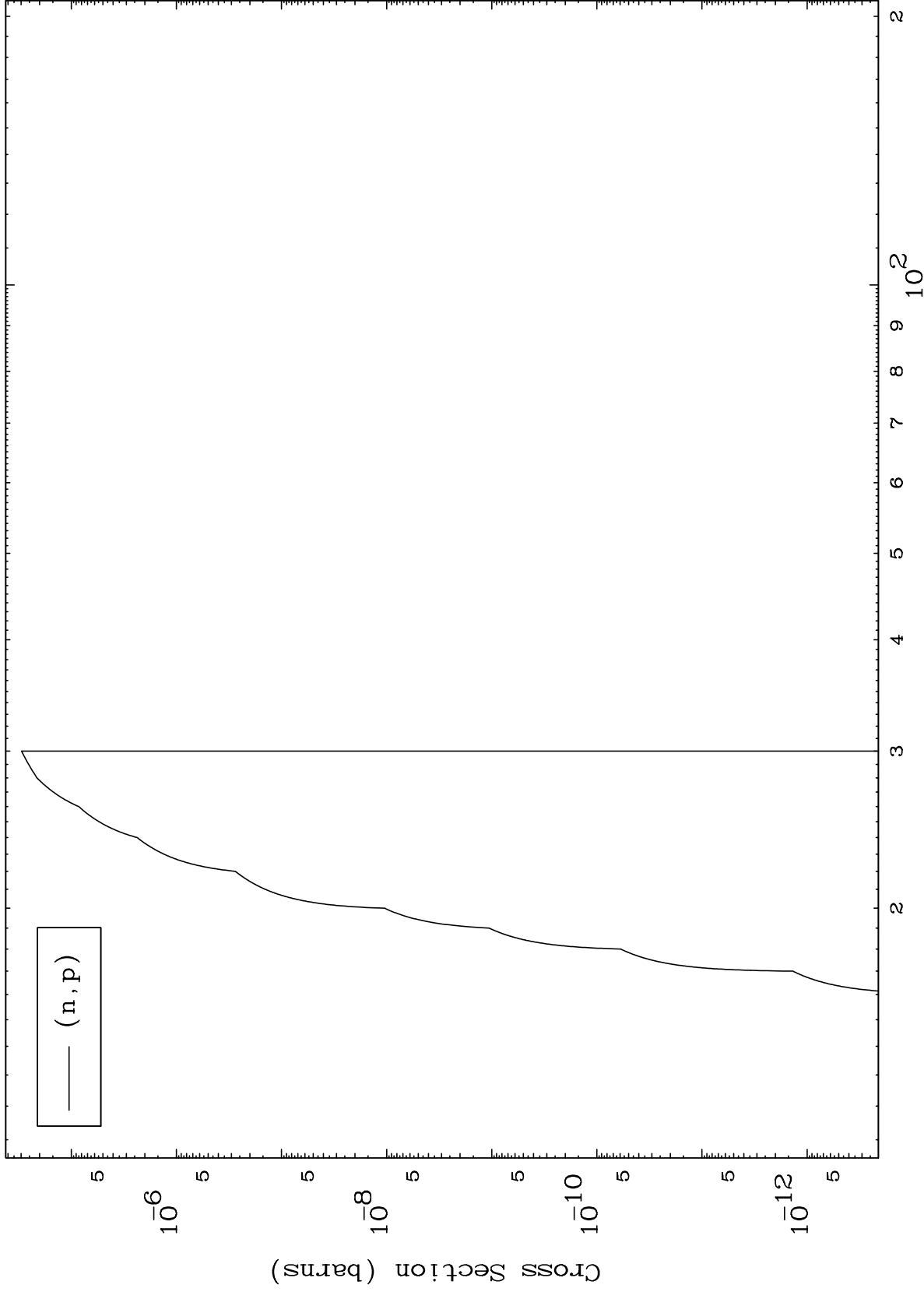
0 Kelvin Cross Sections



MAT 9949

(α, p) Levels
0 Kelvin Cross Sections

101-Md-248



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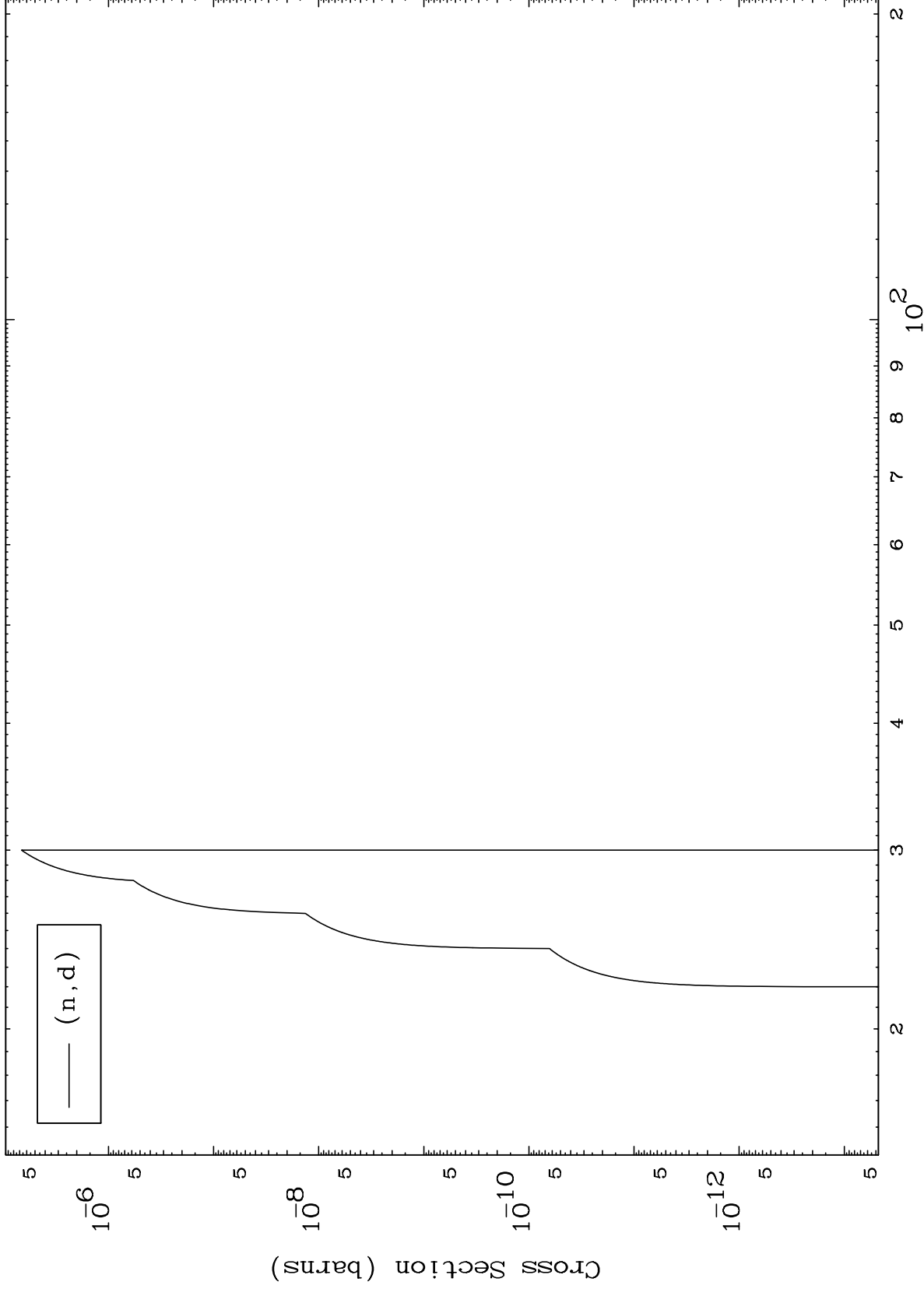
Incident Energy (MeV)

101-Md-248

MAT 9949

(α, d) Levels
0 Kelvin Cross Sections

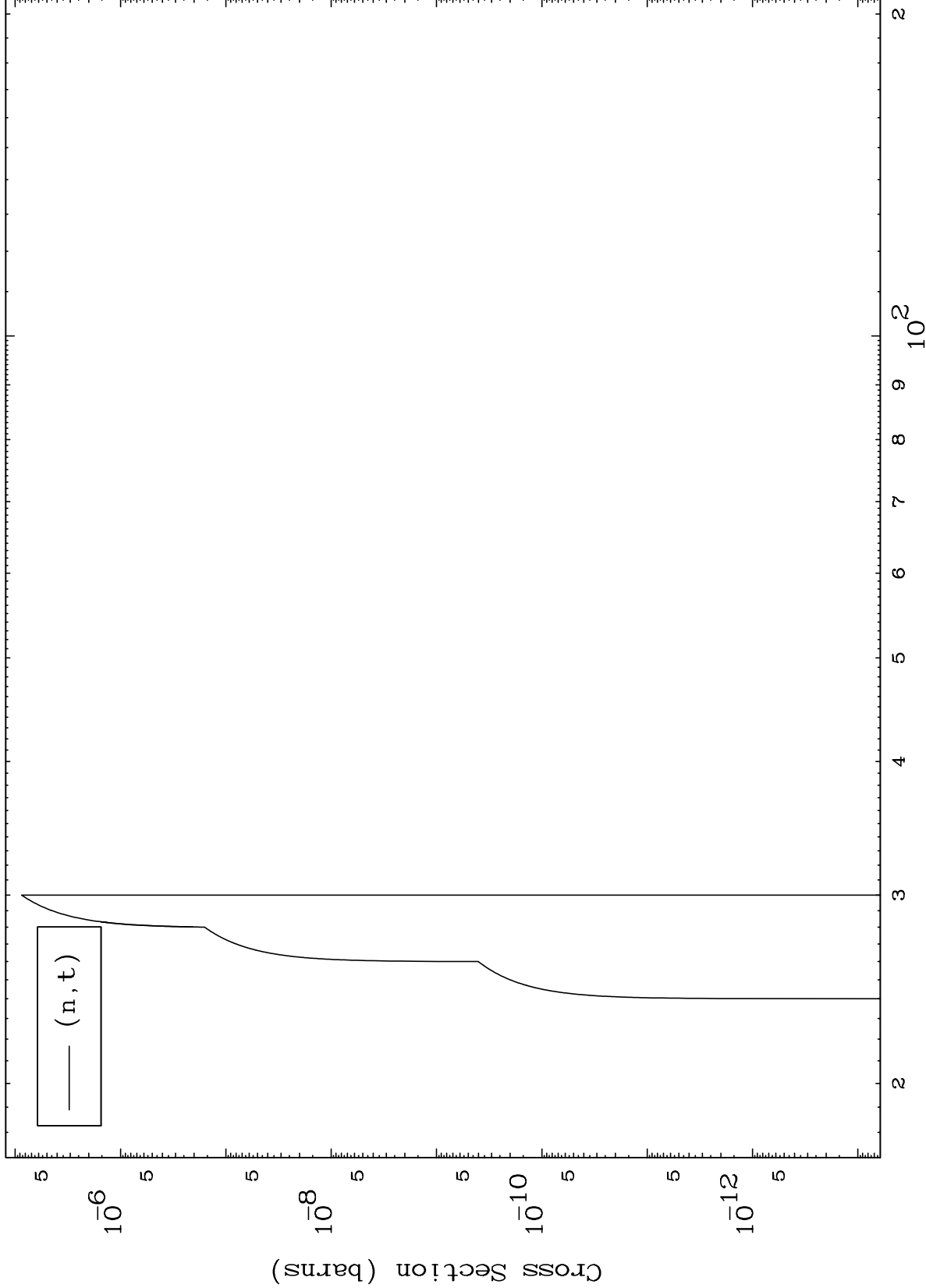
101-Md-248



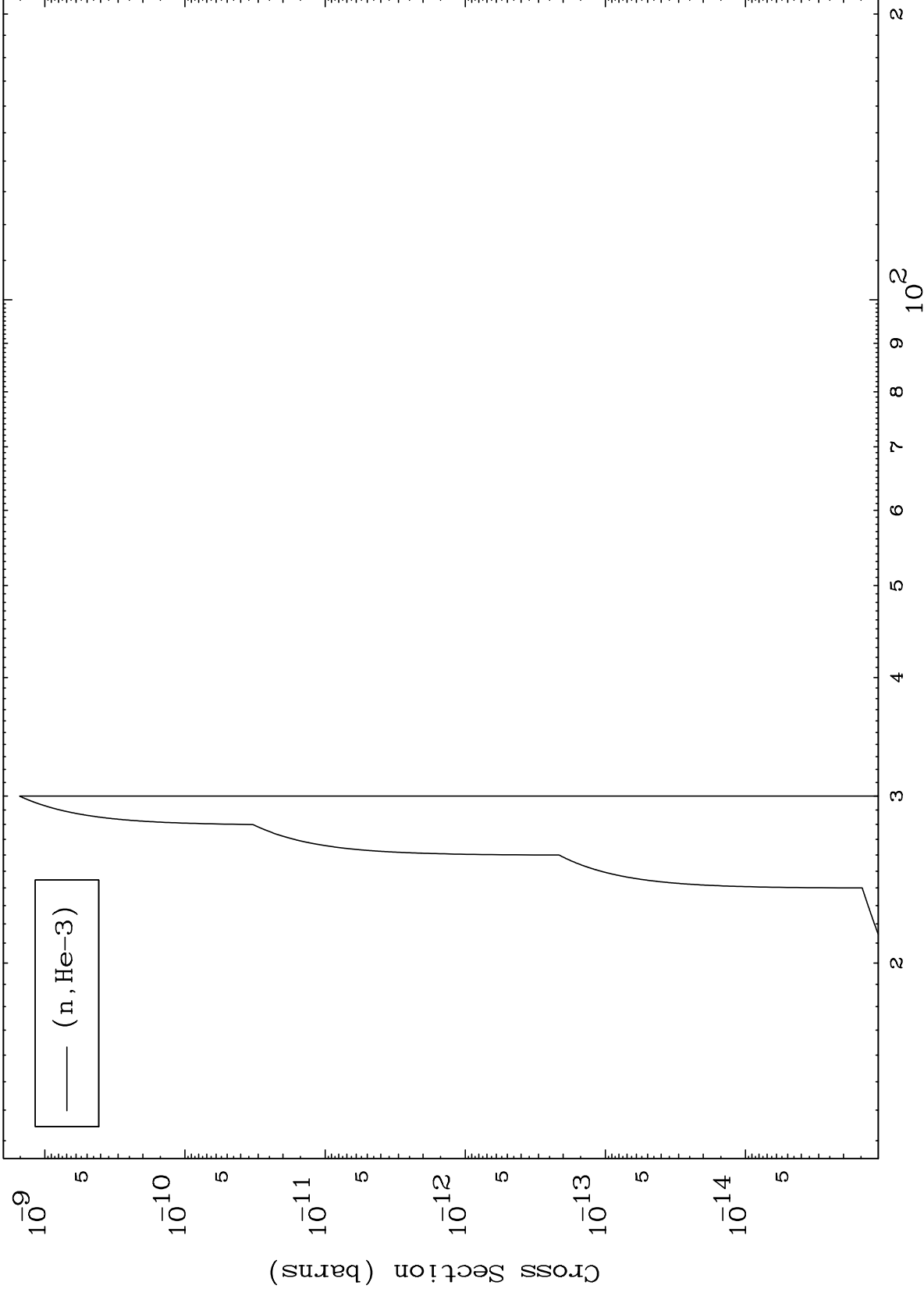
6

Incident Energy (MeV)

101-Md-248



($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

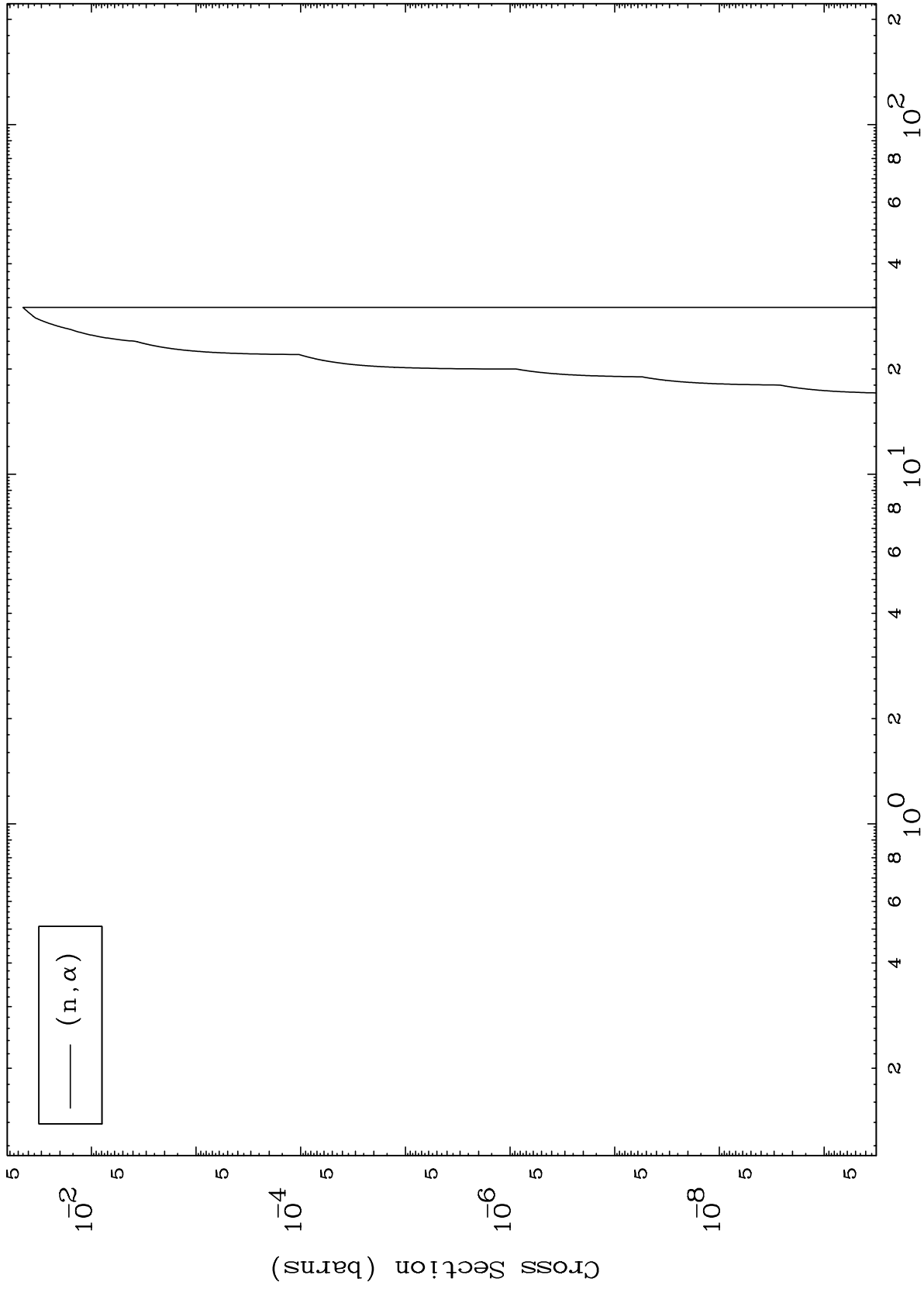


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(α, α) Levels

101-Md-248

0 Kelvin Cross Sections



9

Incident Energy (MeV)

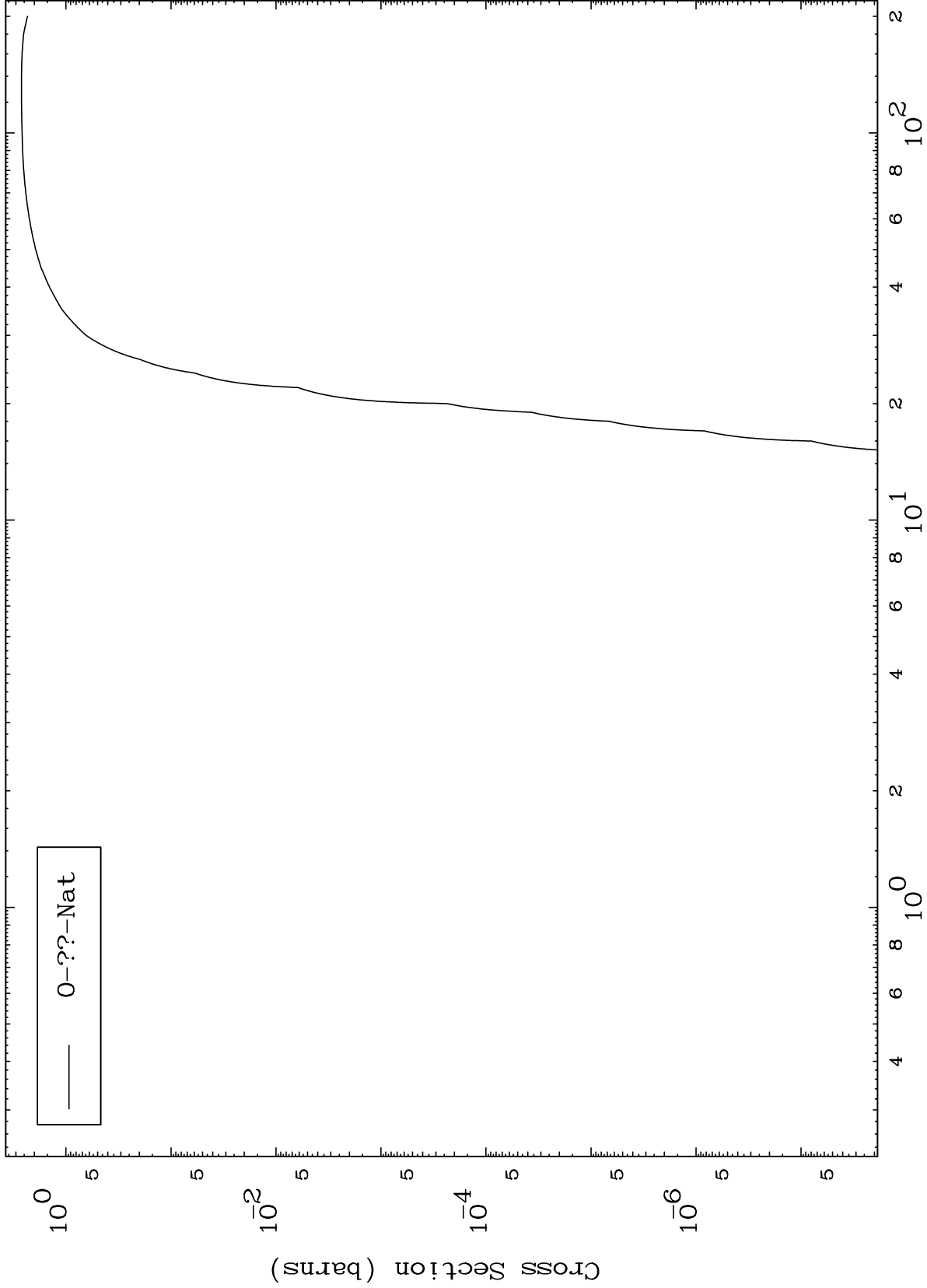
101-Md-248

MAT 9949

Fission

101-Md-248

Radionuclide Production Cross Section

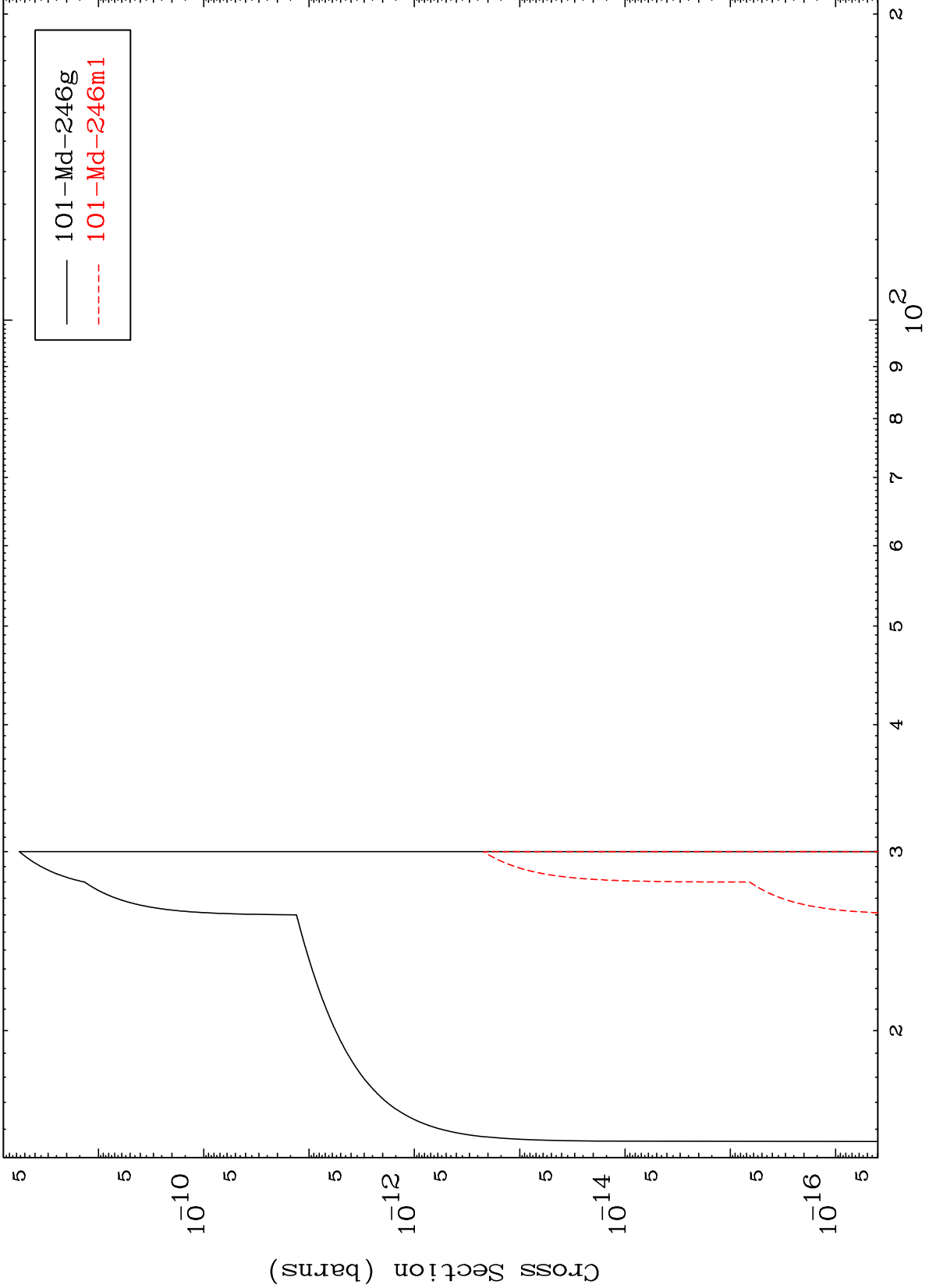


10

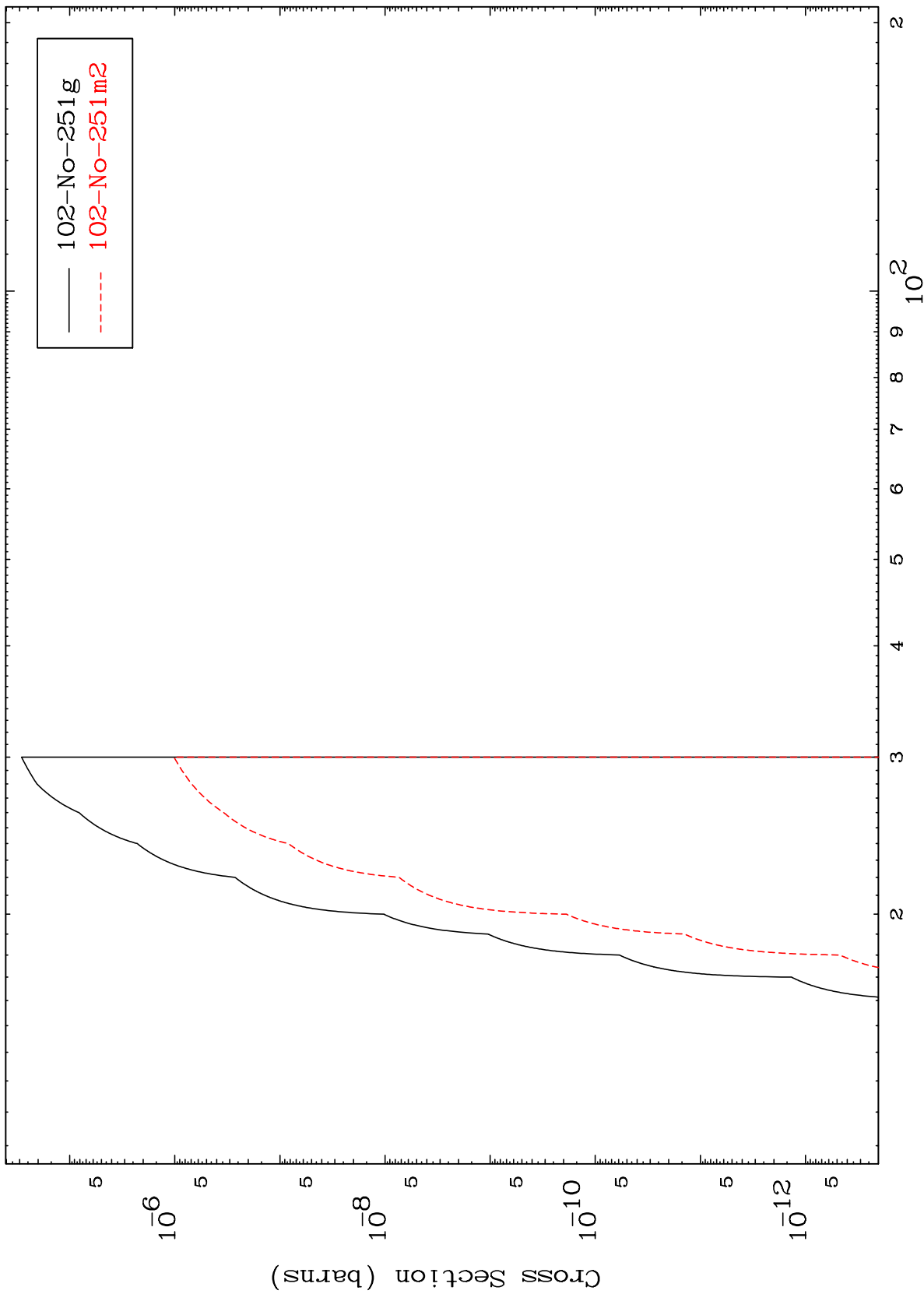
Incident Energy (MeV)

101-Md-248

Radionuclide Production Cross Section



(n,p)
Radionuclide Production Cross Section

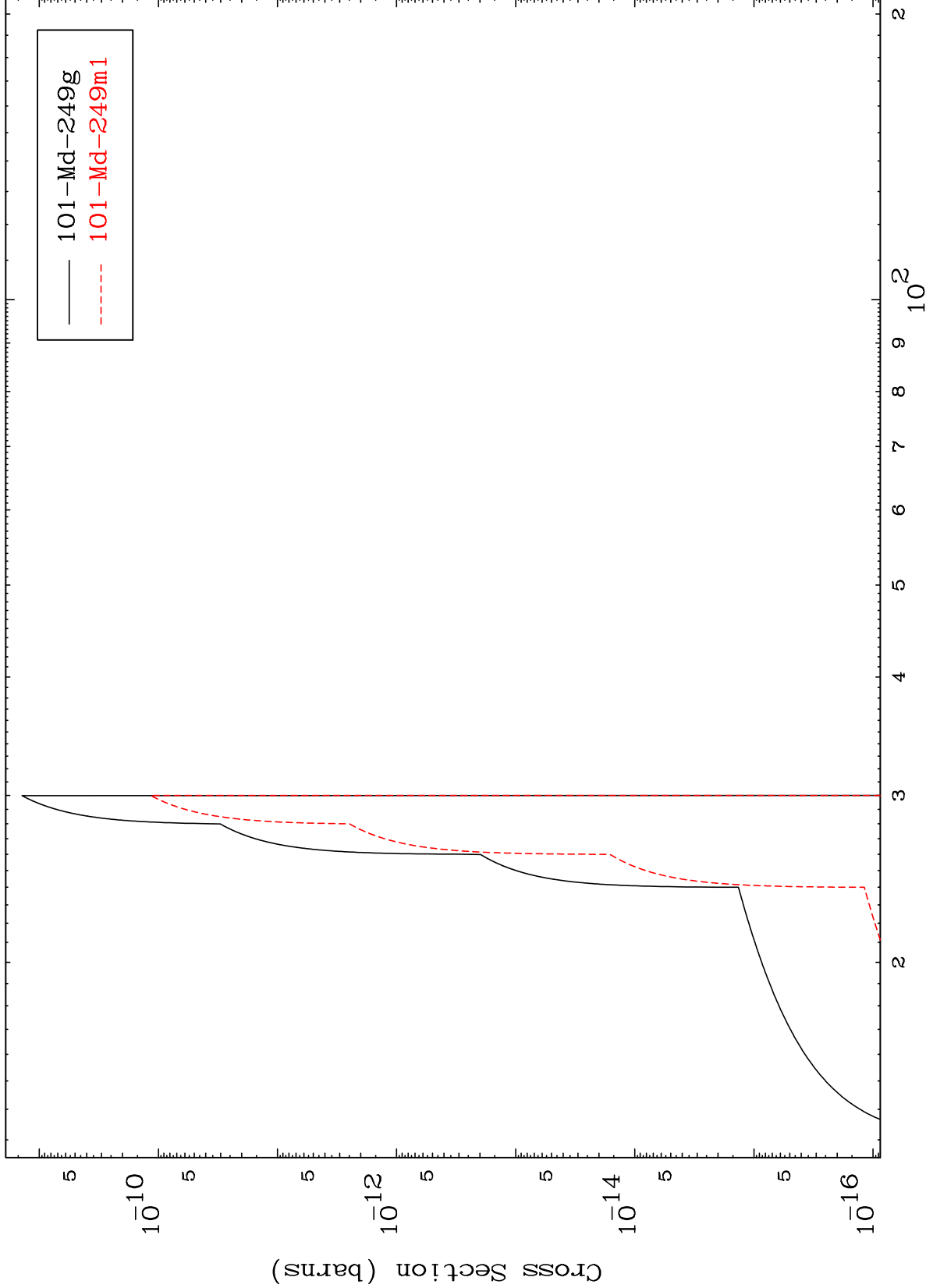


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(n,He-3)

101-Md-248

Radionuclide Production Cross Section



13

Incident Energy (MeV)

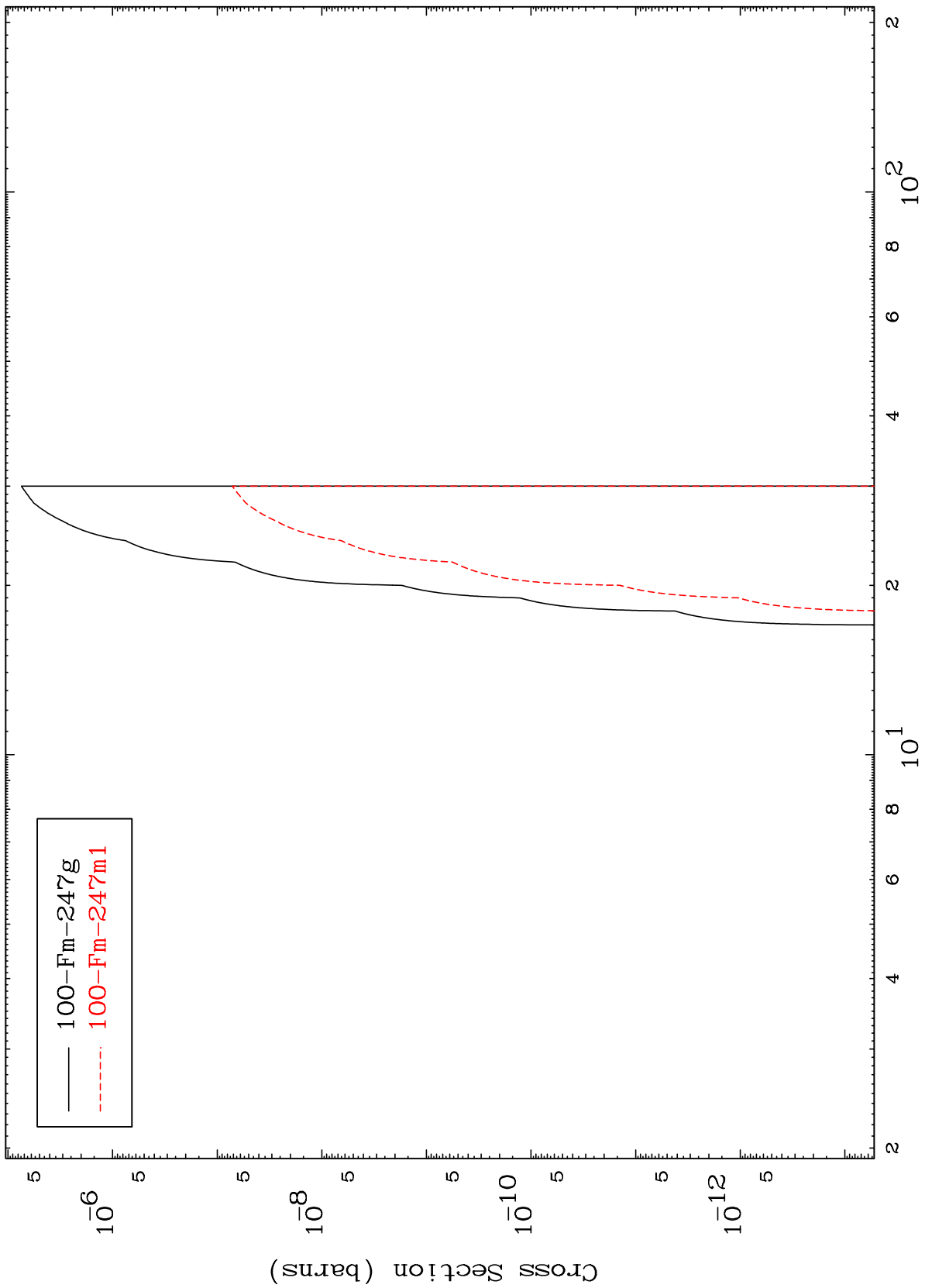
101-Md-248

MAT 9949

(n,p) α

101-Md-248

Radionuclide Production Cross Section



— 100-Fm-247g
- - - 100-Fm-247m1

14

Incident Energy (MeV)

101-Md-248