

Program EVALPLOT  
(Version 2021-1)

by

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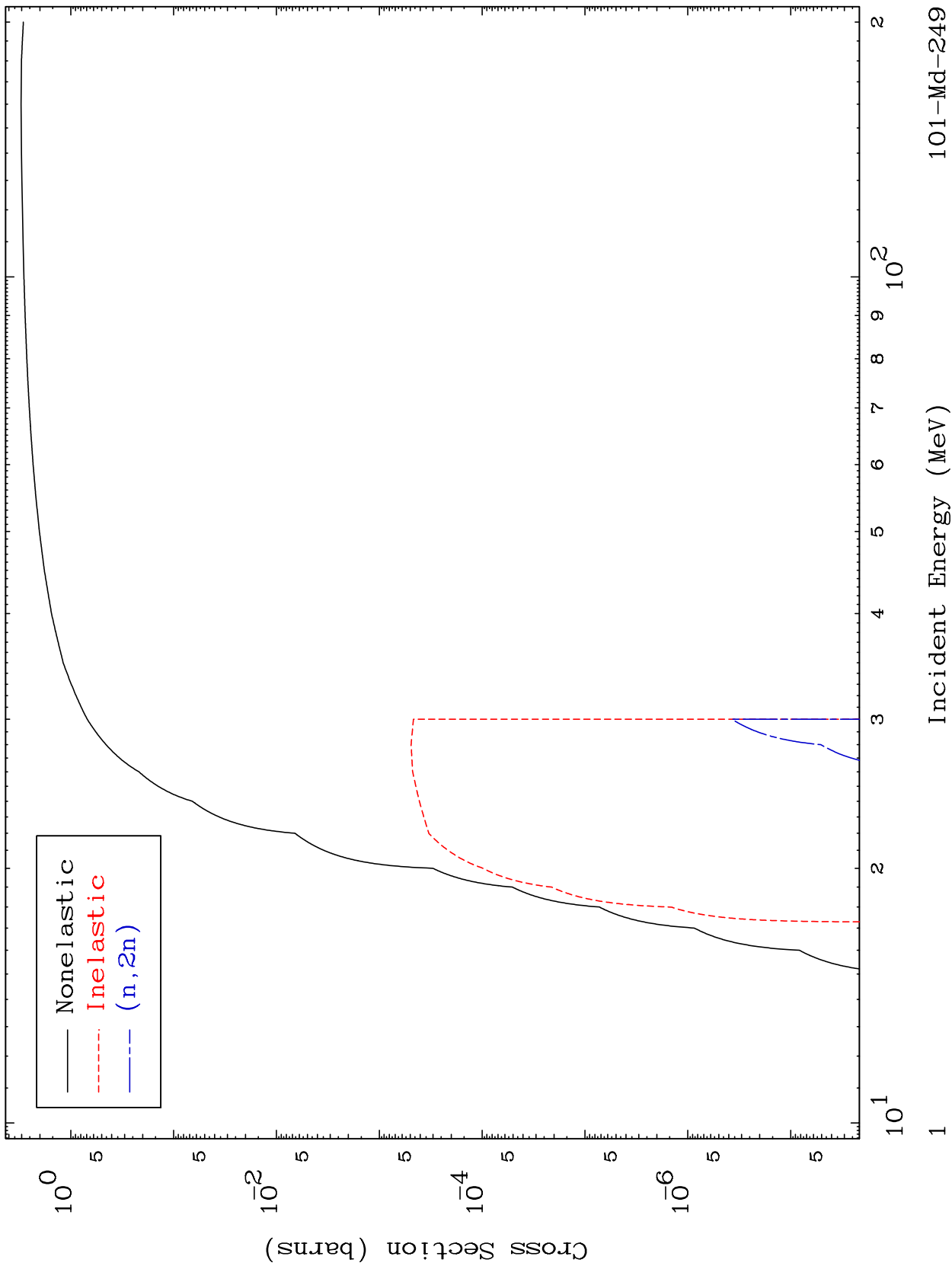
Press Mouse Button to Start

MAT 9950

$\alpha$  Major

101-Md-249

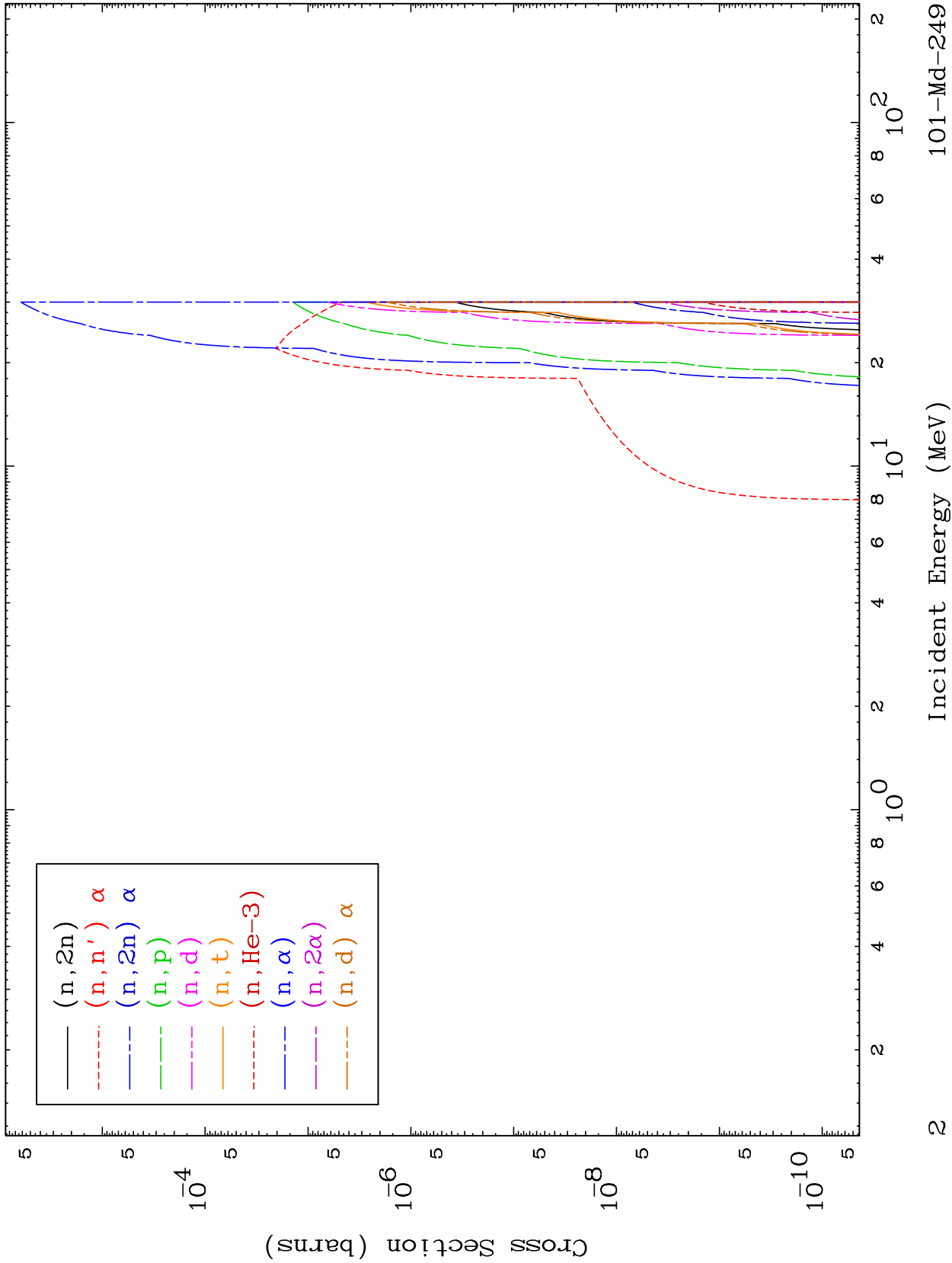
0 Kelvin Cross Sections



MAT 9950

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

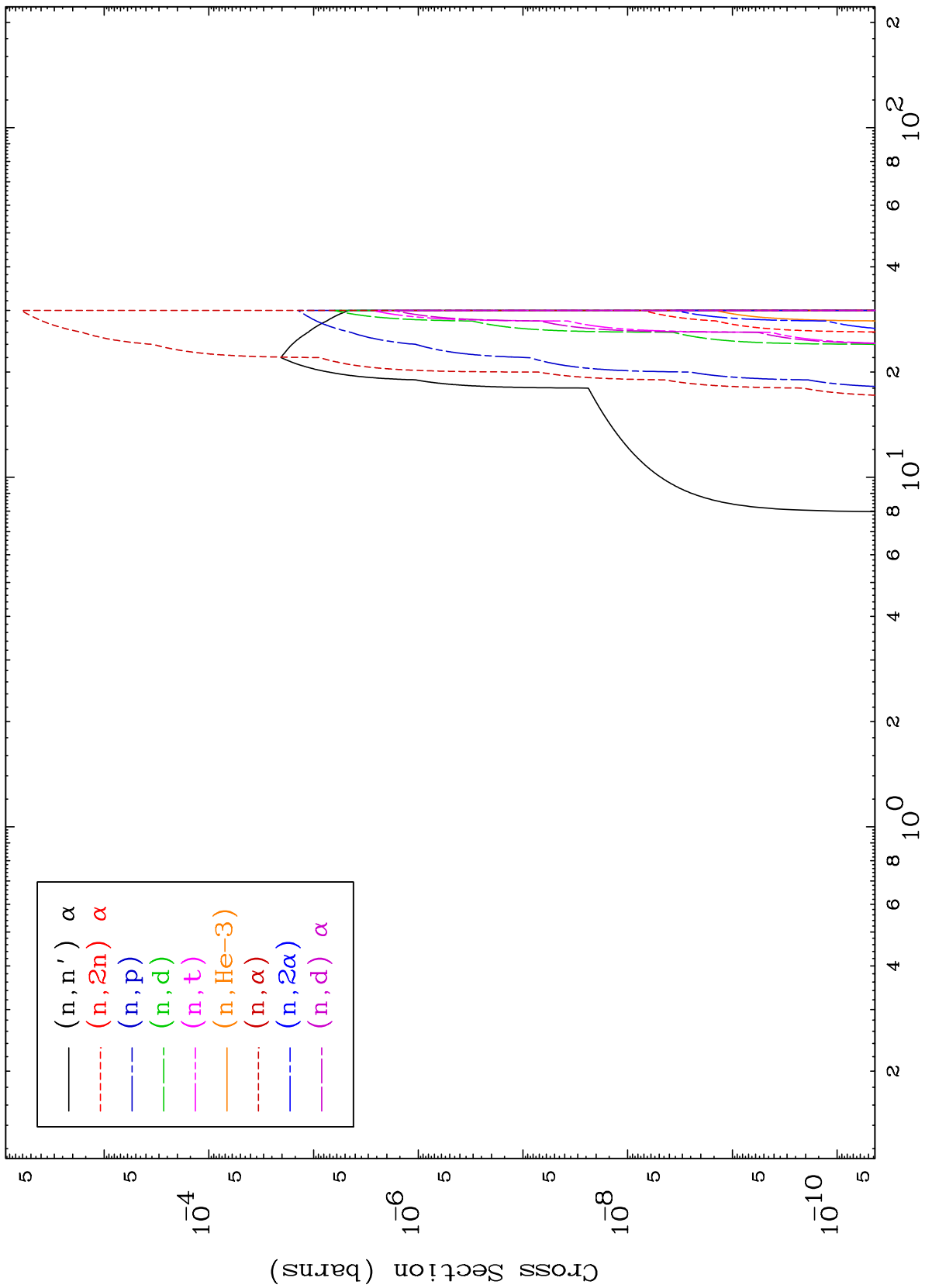
101-Md-249



MAT 9950

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

101-Md-249



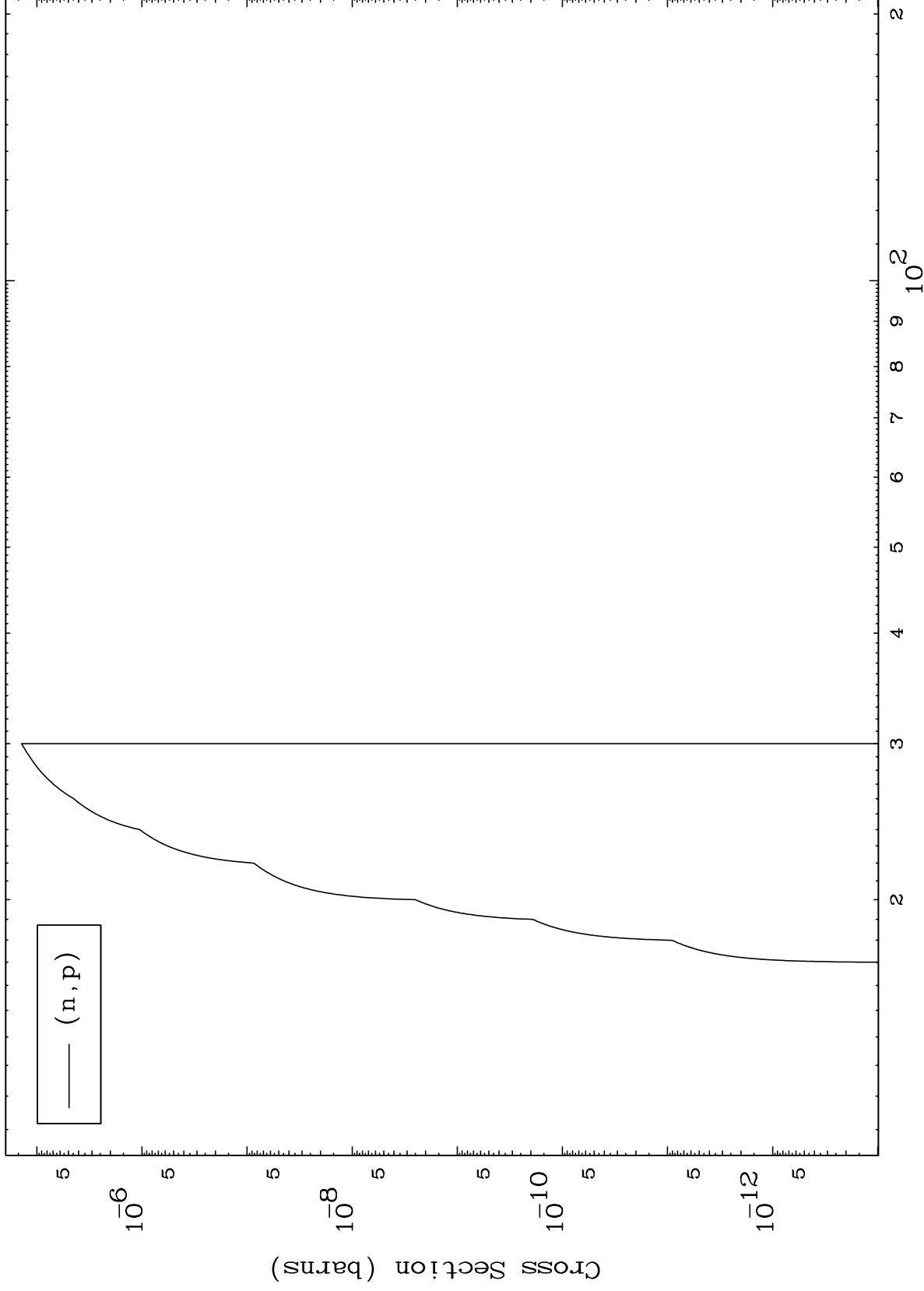
Incident Energy (MeV)

101-Md-249

MAT 9950

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

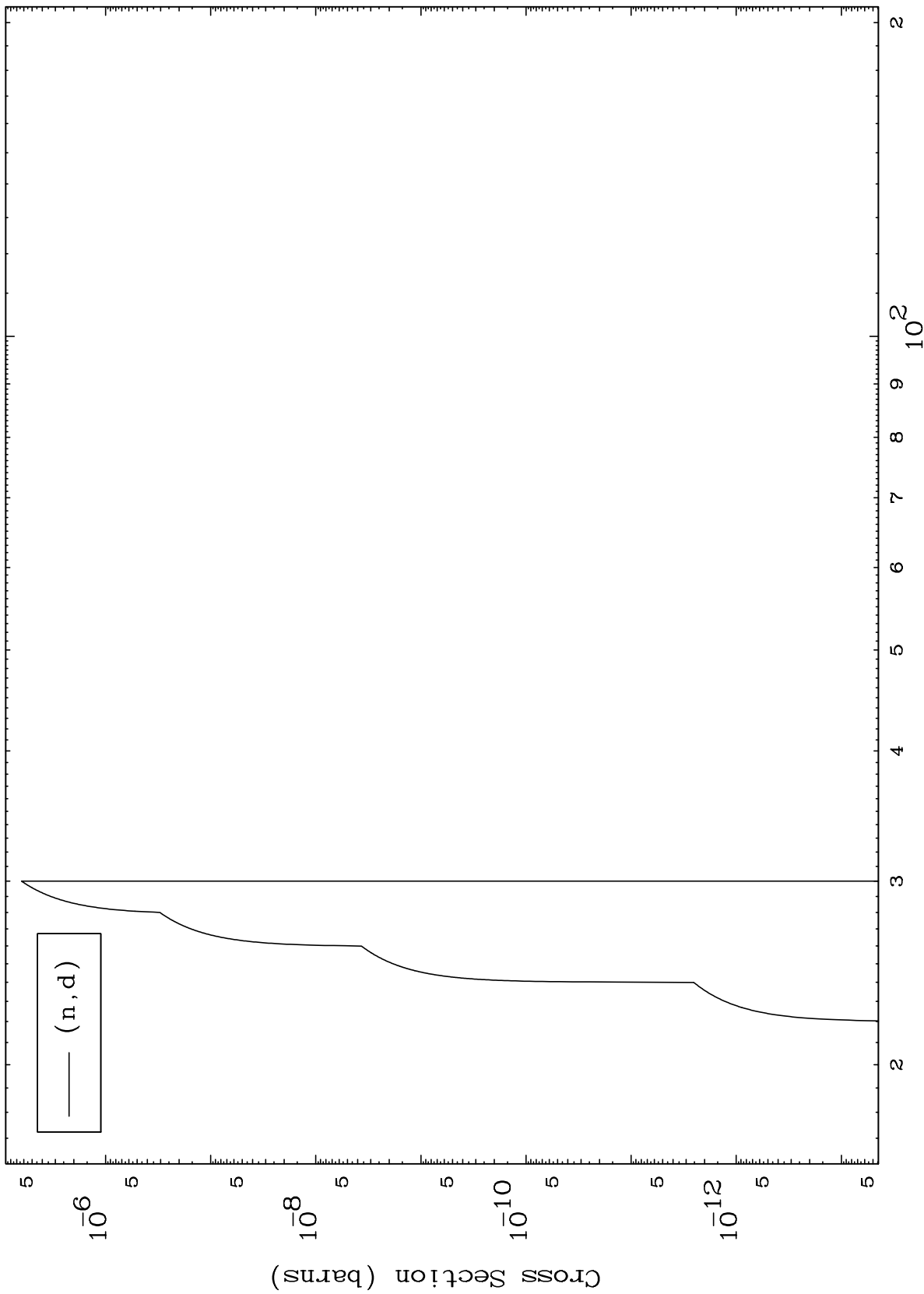
101-Md-249



MAT 9950

101-Md-249

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections



101-Md-249

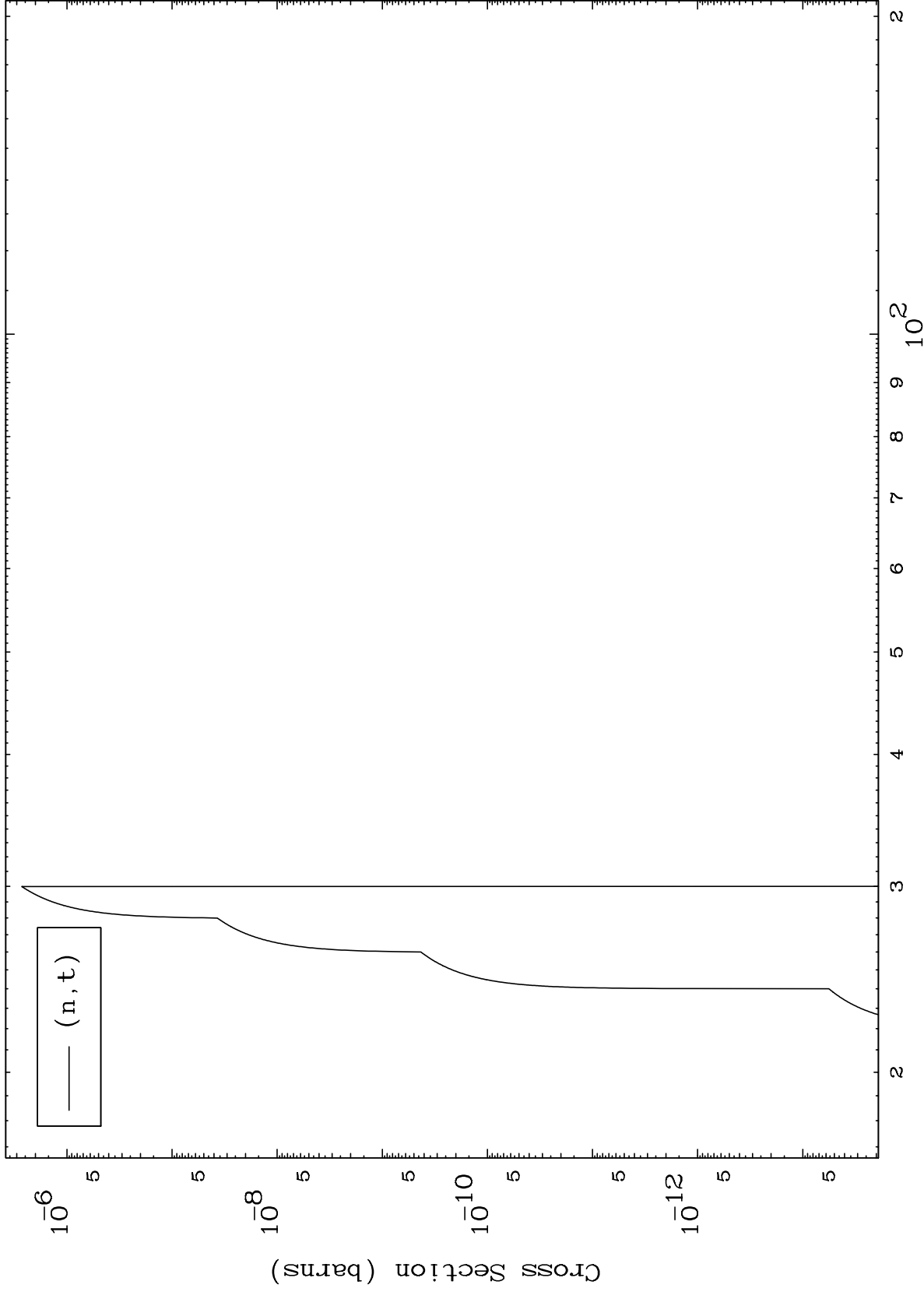
Incident Energy (MeV)

5

MAT 9950

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



6

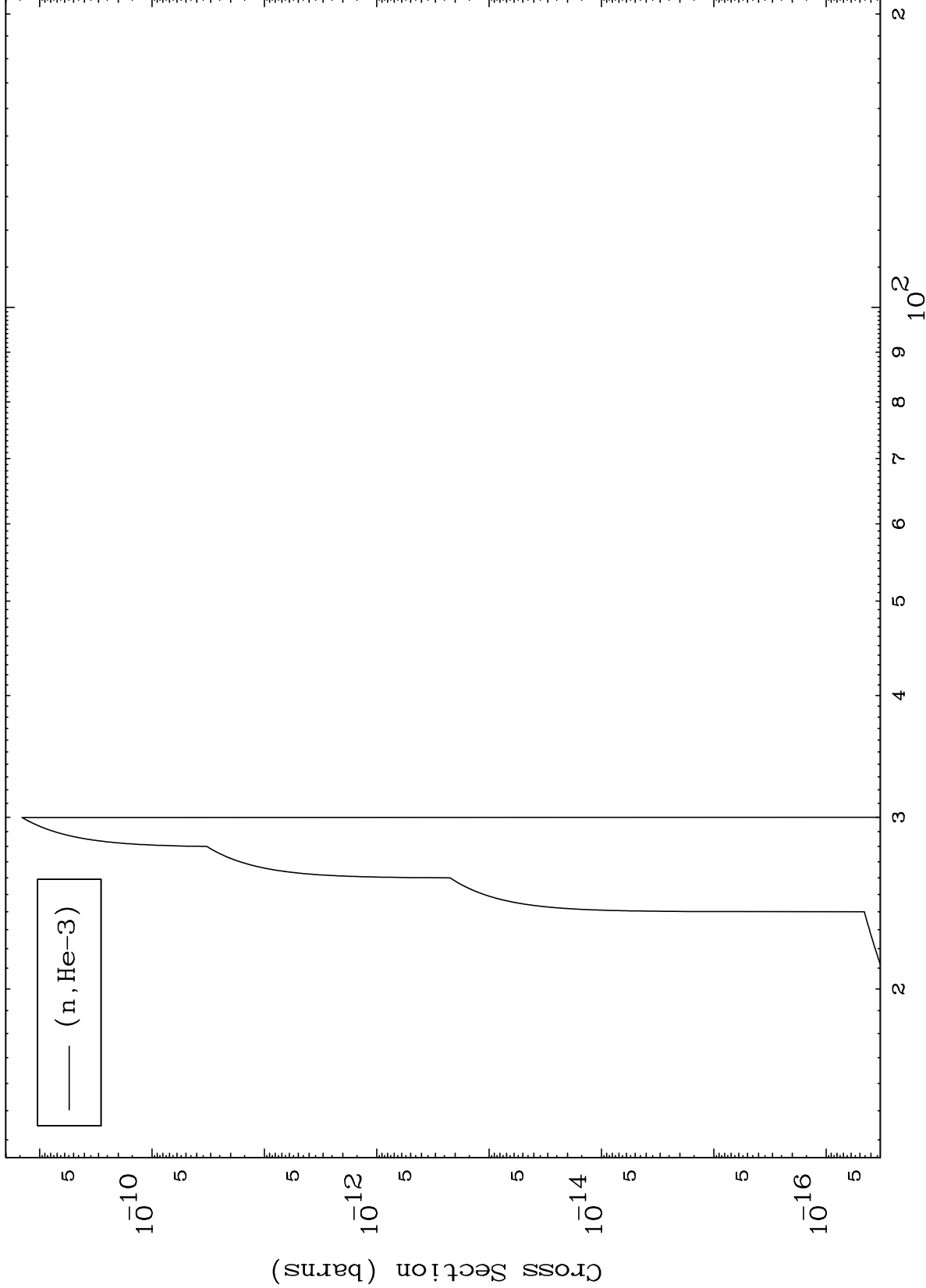
Incident Energy (MeV)

101-Md-249

MAT 9950

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

101-Md-249



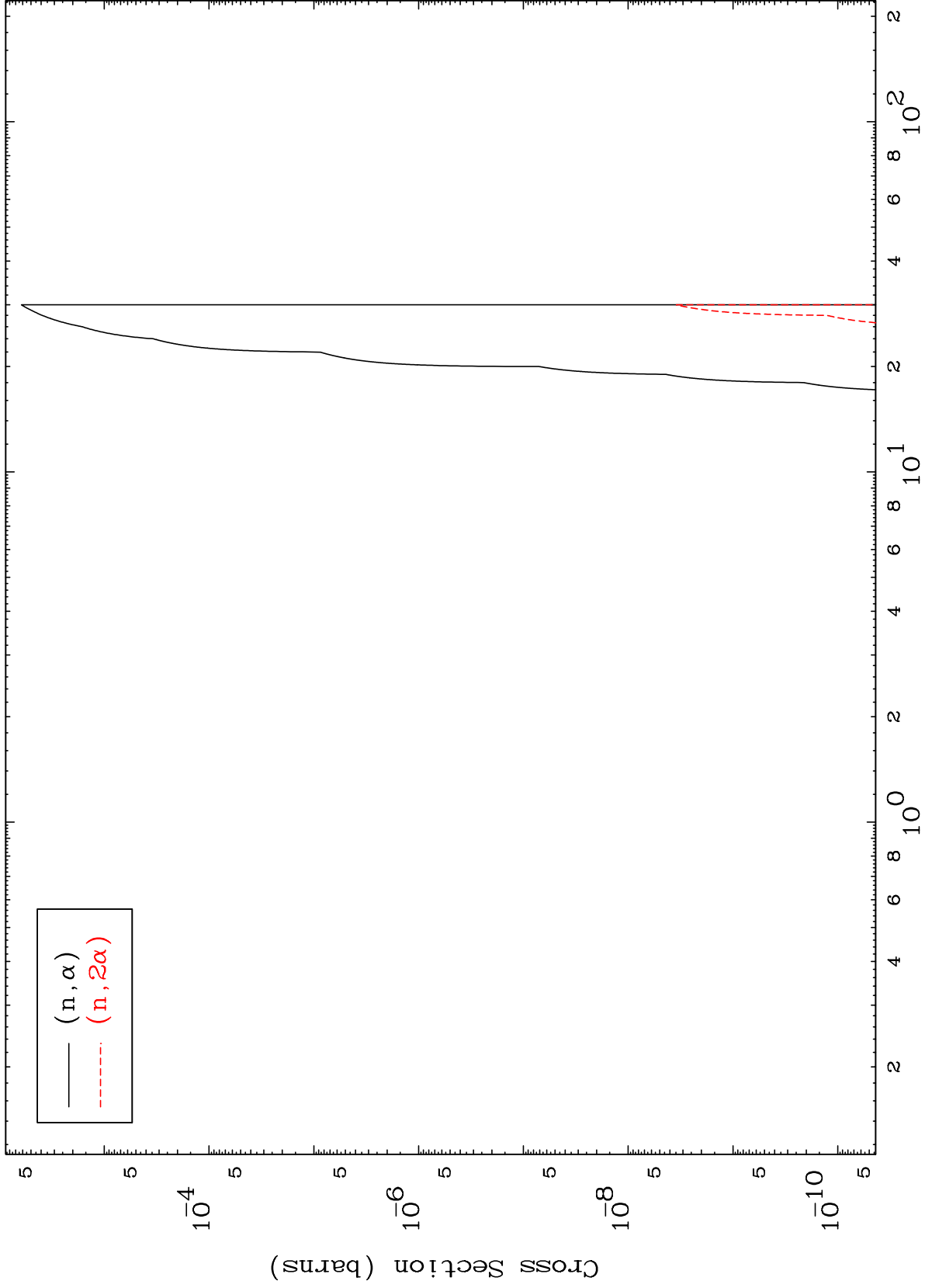


MAT 9950

( $\alpha, \alpha$ ) Levels

101-Md-249

0 Kelvin Cross Sections

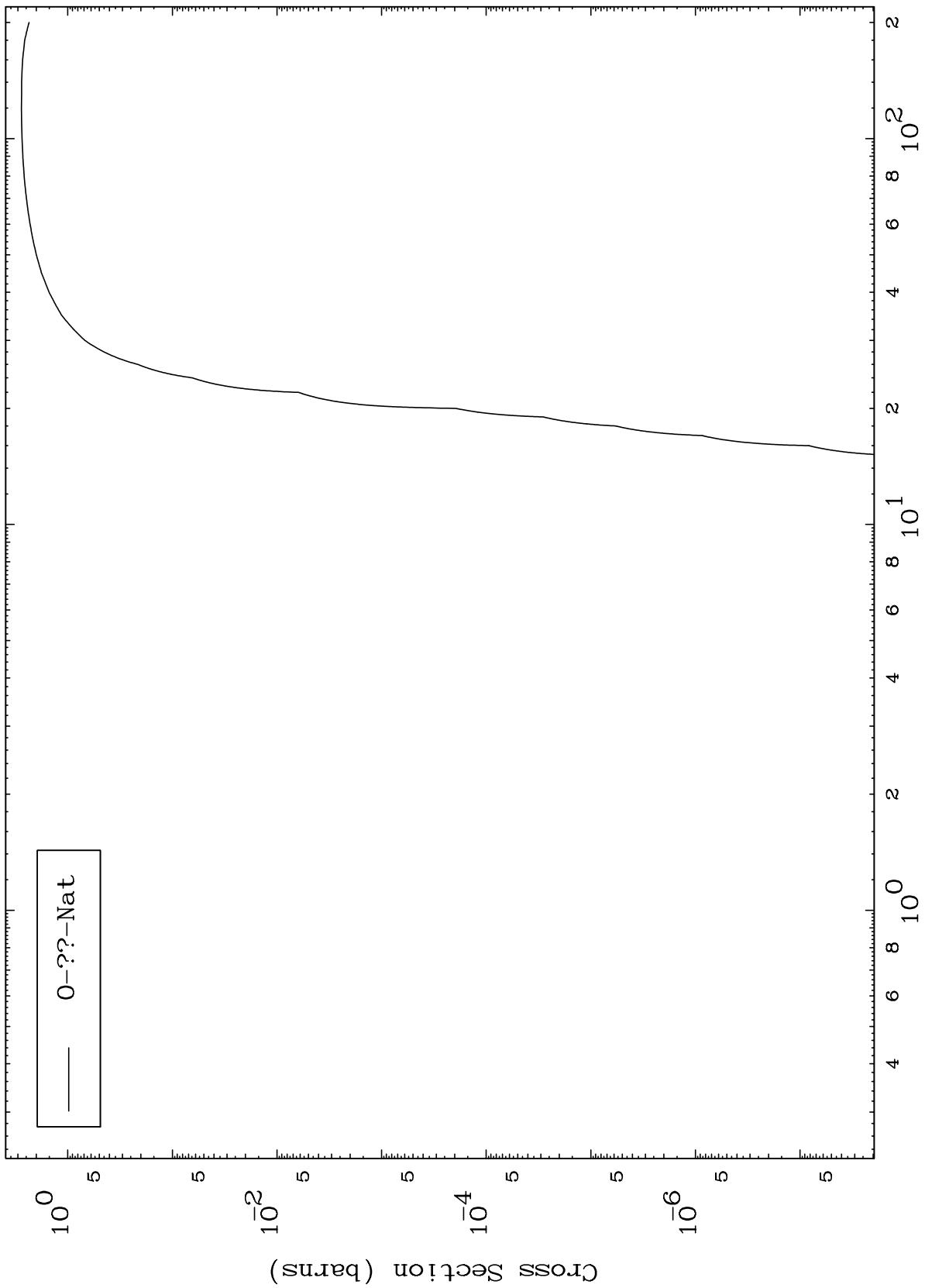


MAT 9950

Fission

101-Md-249

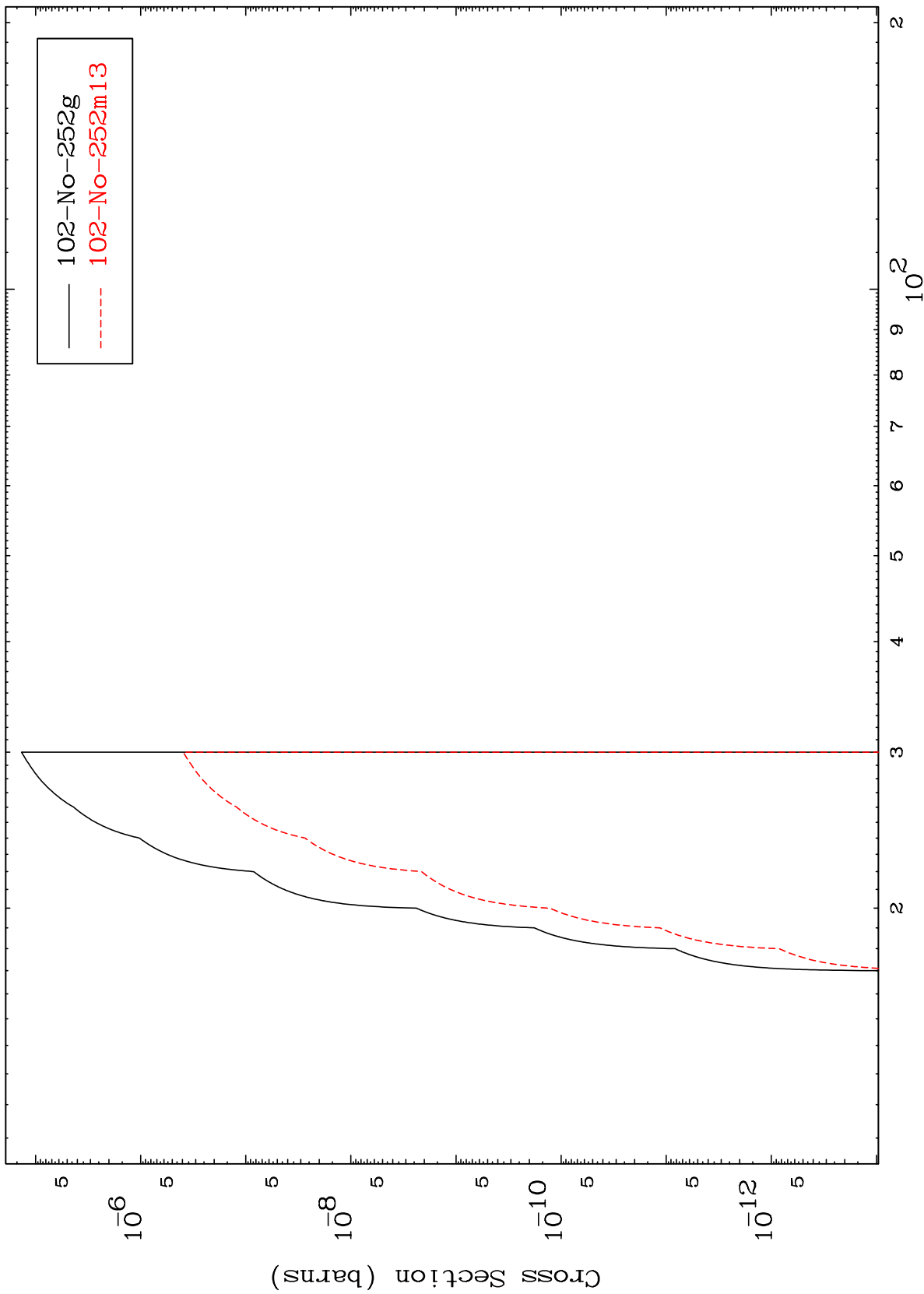
Radionuclide Production Cross Section



MAT 9950

101-Md-249

(n,p)  
Radionuclide Production Cross Section



101-Md-249

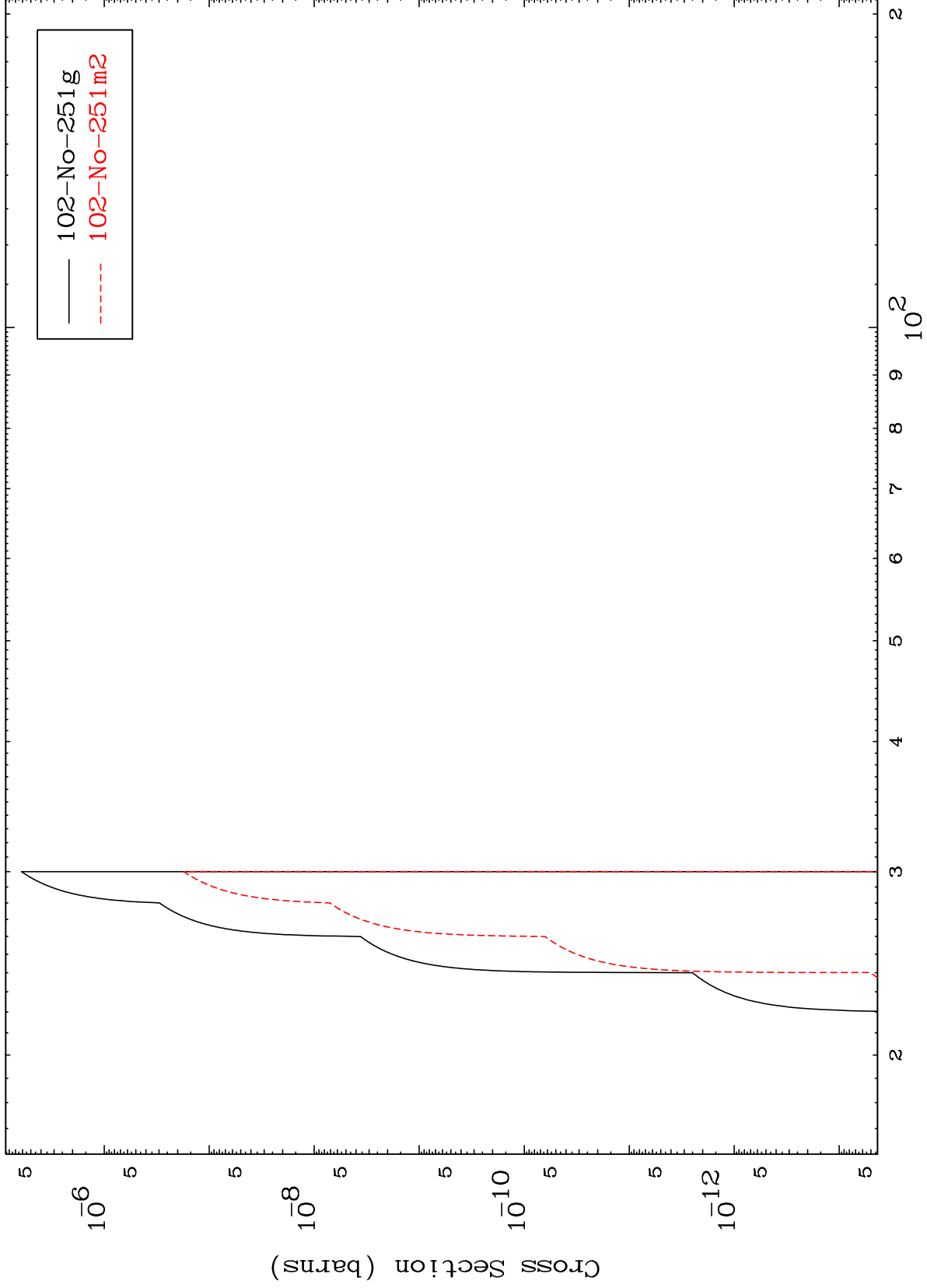
Incident Energy (MeV)

10

MAT 9950

101-Md-249

(n,d)  
Radionuclide Production Cross Section



11

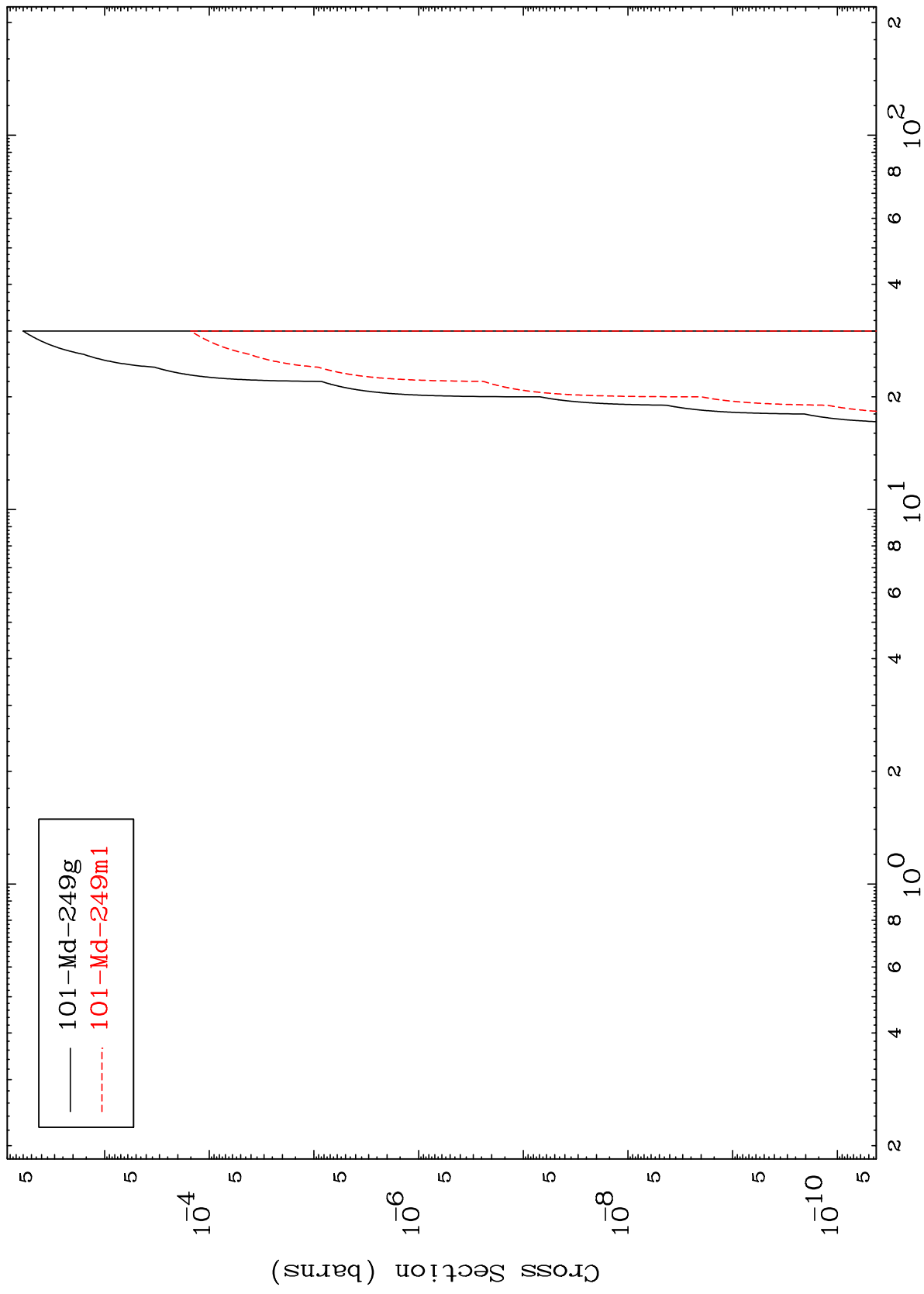
101-Md-249

Incident Energy (MeV)

MAT 9950

101-Md-249

Radionuclide Production Cross Section  
(n,  $\alpha$ )



12

101-Md-249

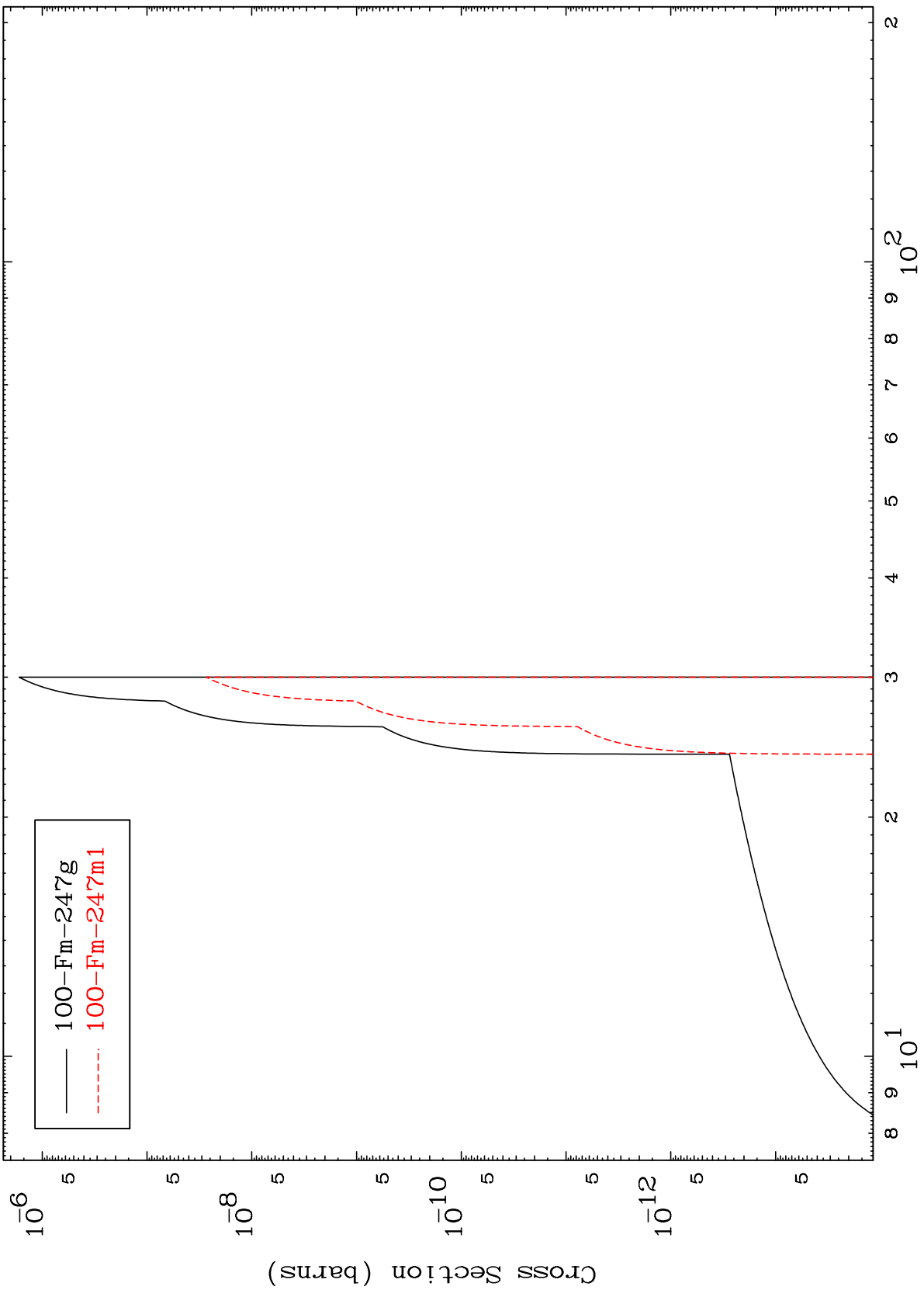
Incident Energy (MeV)

MAT 9950

(n,d)  $\alpha$

101-Md-249

Radionuclide Production Cross Section



13

Incident Energy (MeV)

101-Md-249