

Program EVALPLOT  
(Version 2021-1)

by

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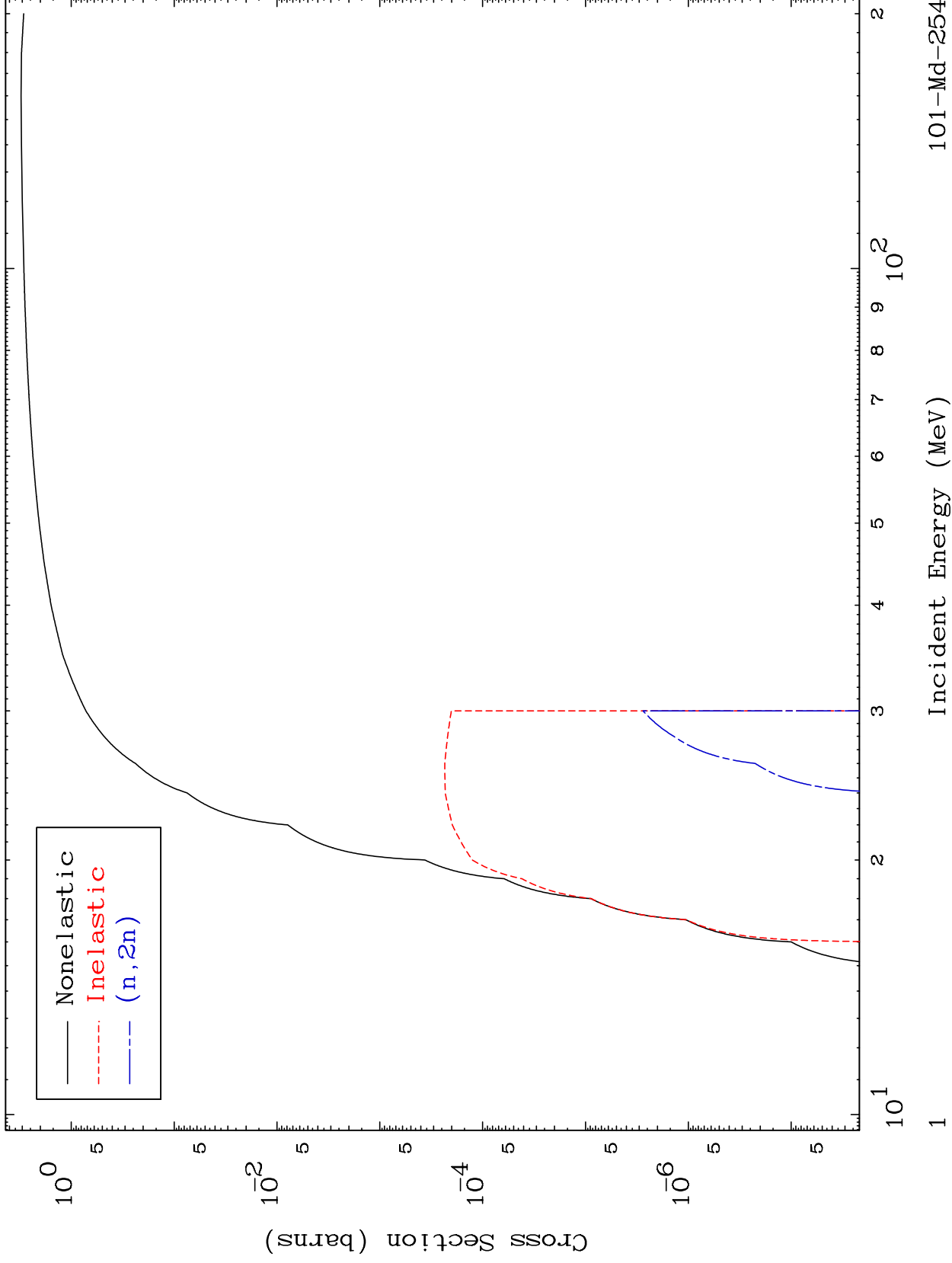
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9955

$\alpha$  Major  
0 Kelvin Cross Sections

101-Md-254



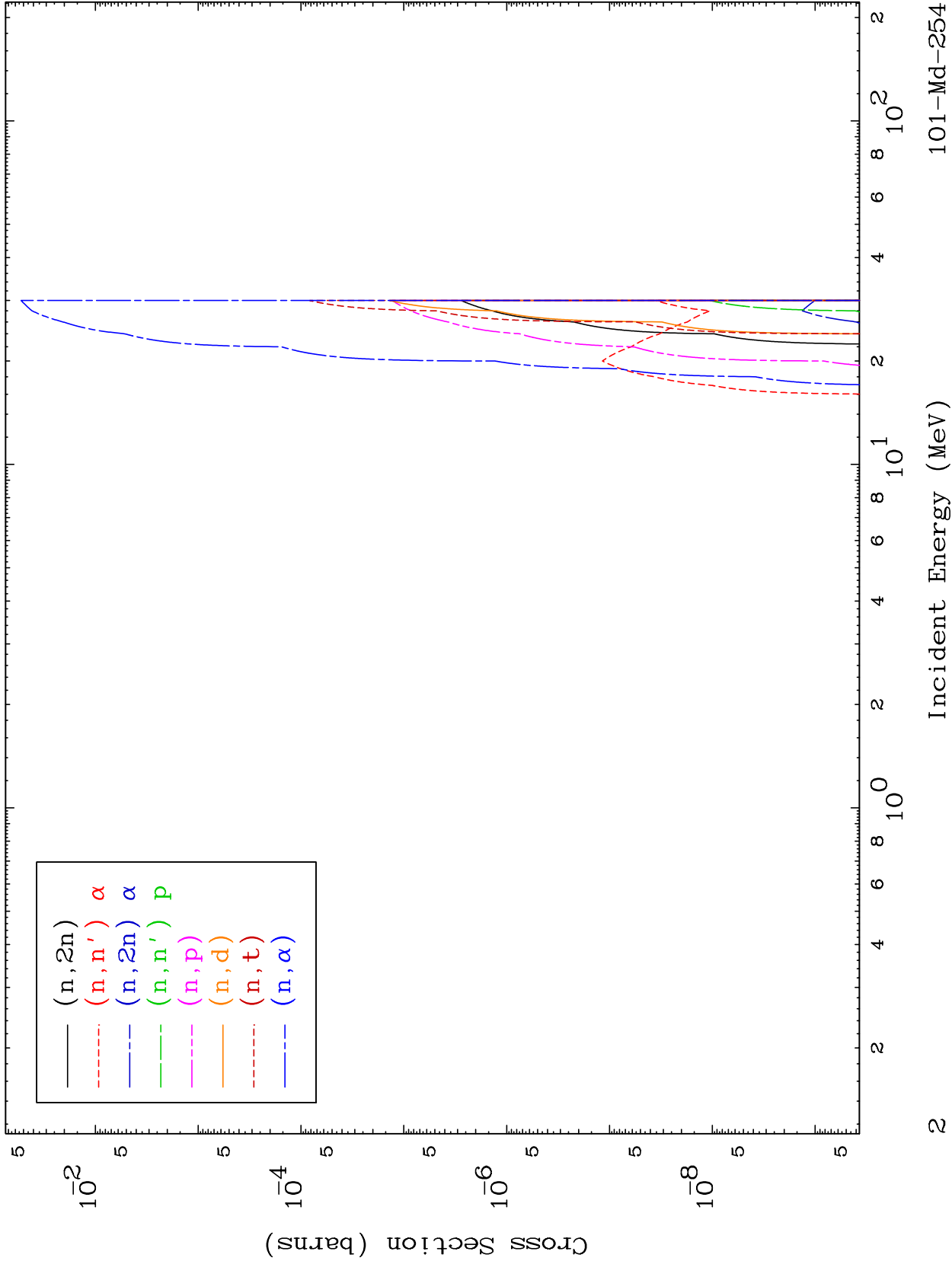
101-Md-254

Incident Energy (MeV)

MAT 9955

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

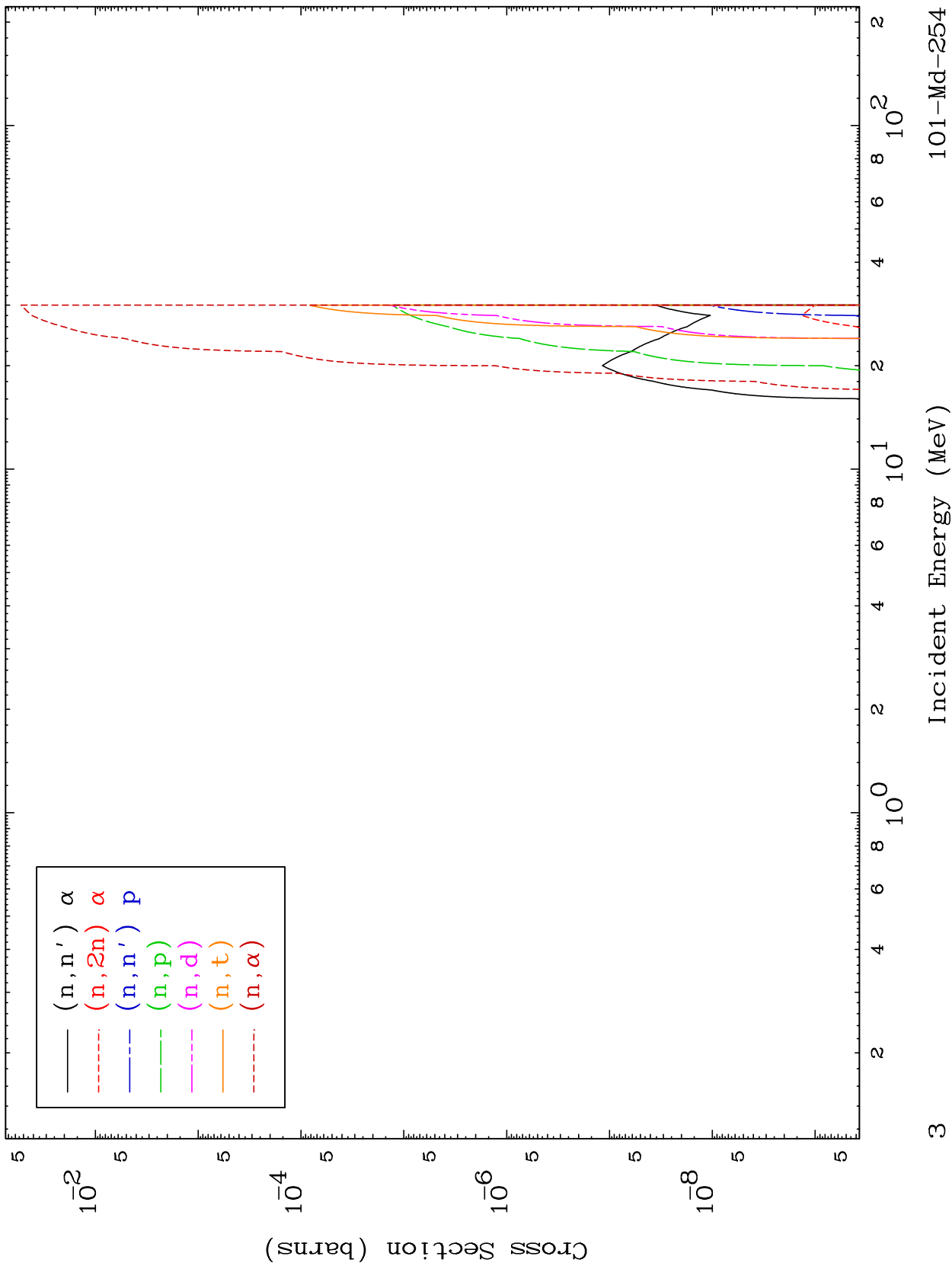
101-Md-254



MAT 9955

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

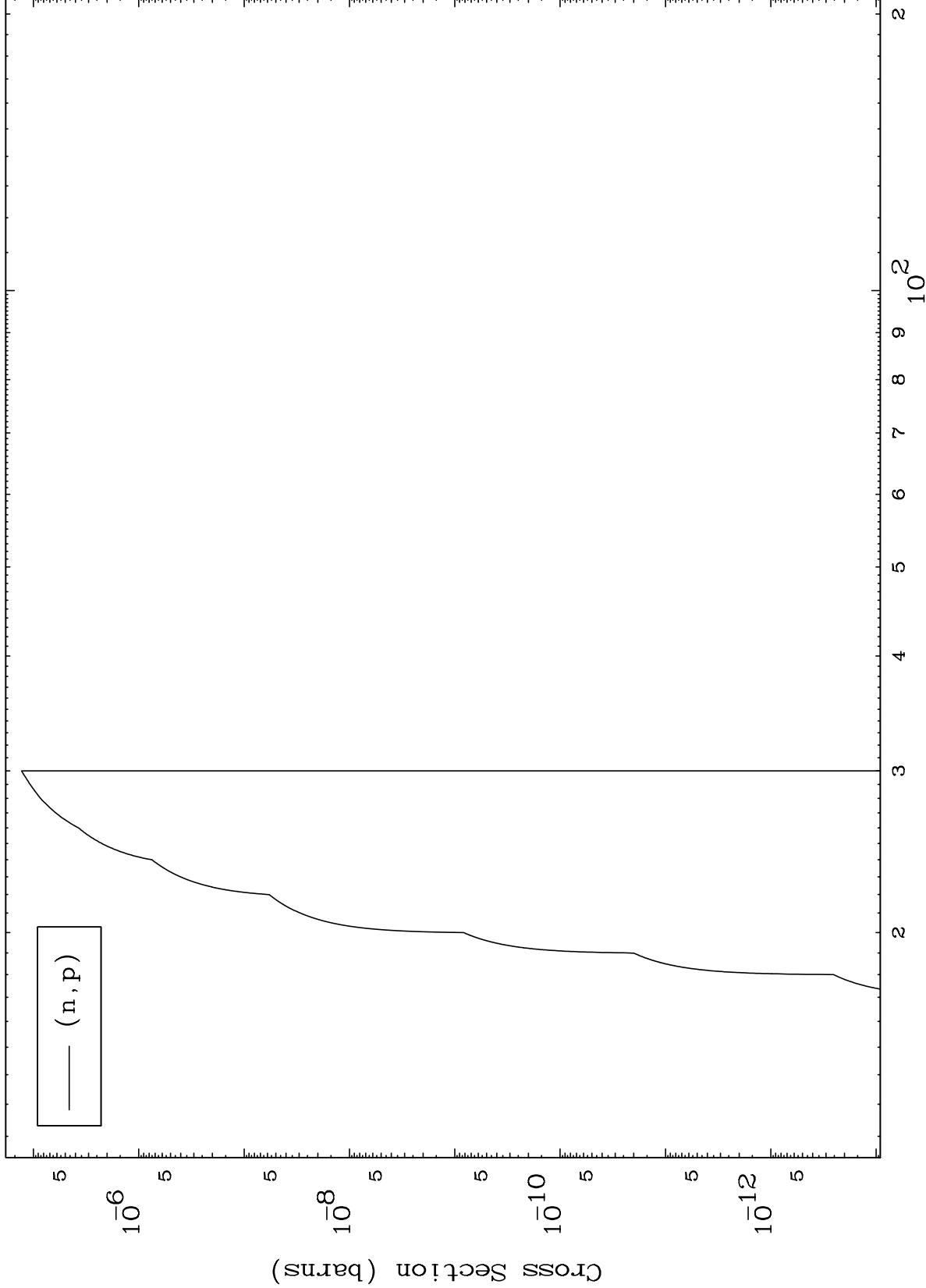
101-Md-254



MAT 9955

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

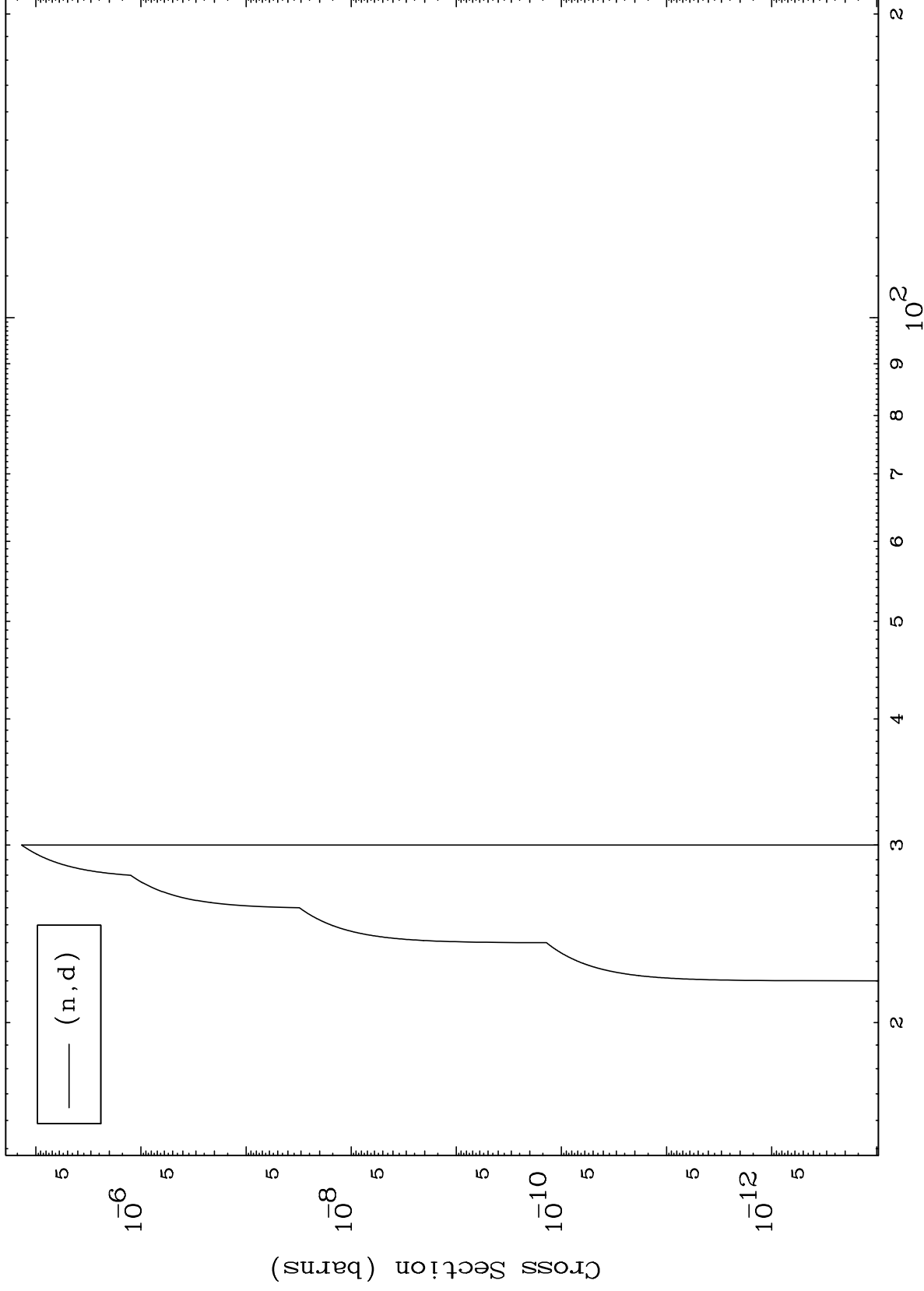
101-Md-254



MAT 9955

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

101-Md-254



5

Incident Energy (MeV)

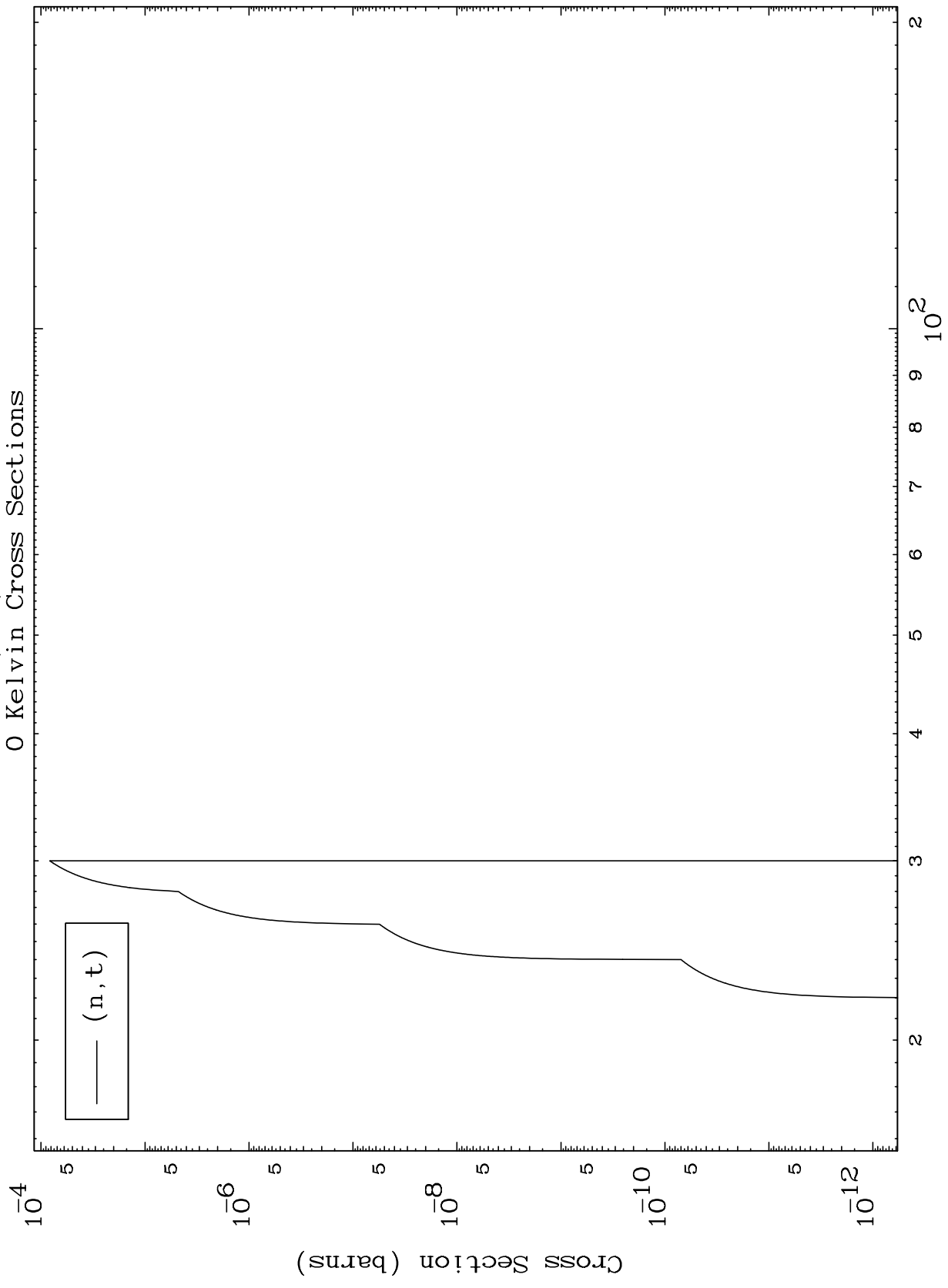
101-Md-254

MAT 9955

( $\alpha, t$ ) Levels

101-Md-254

0 Kelvin Cross Sections



6

Incident Energy (MeV)

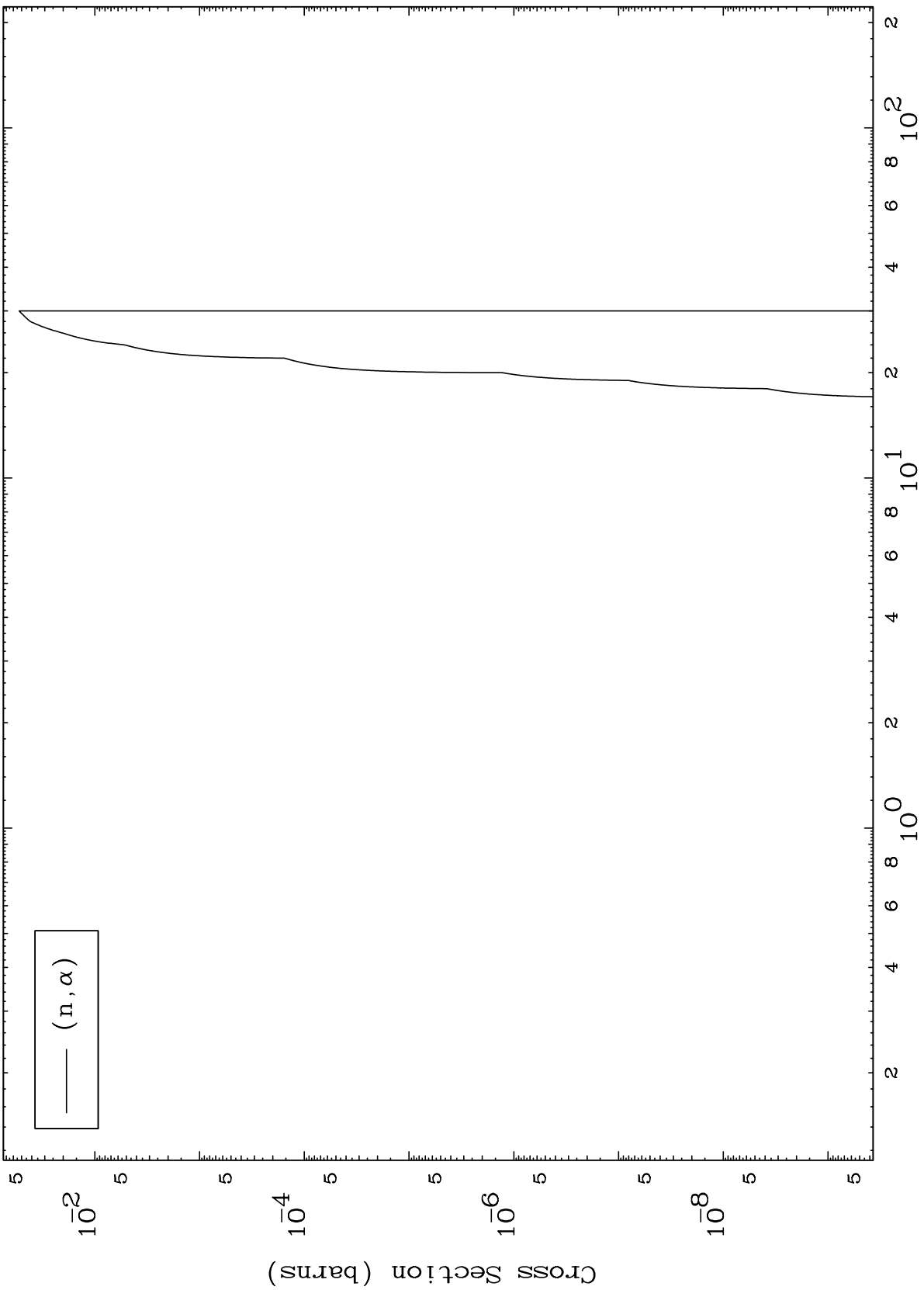
101-Md-254

MAT 9955

( $\alpha, \alpha$ ) Levels

101-Md-254

0 Kelvin Cross Sections



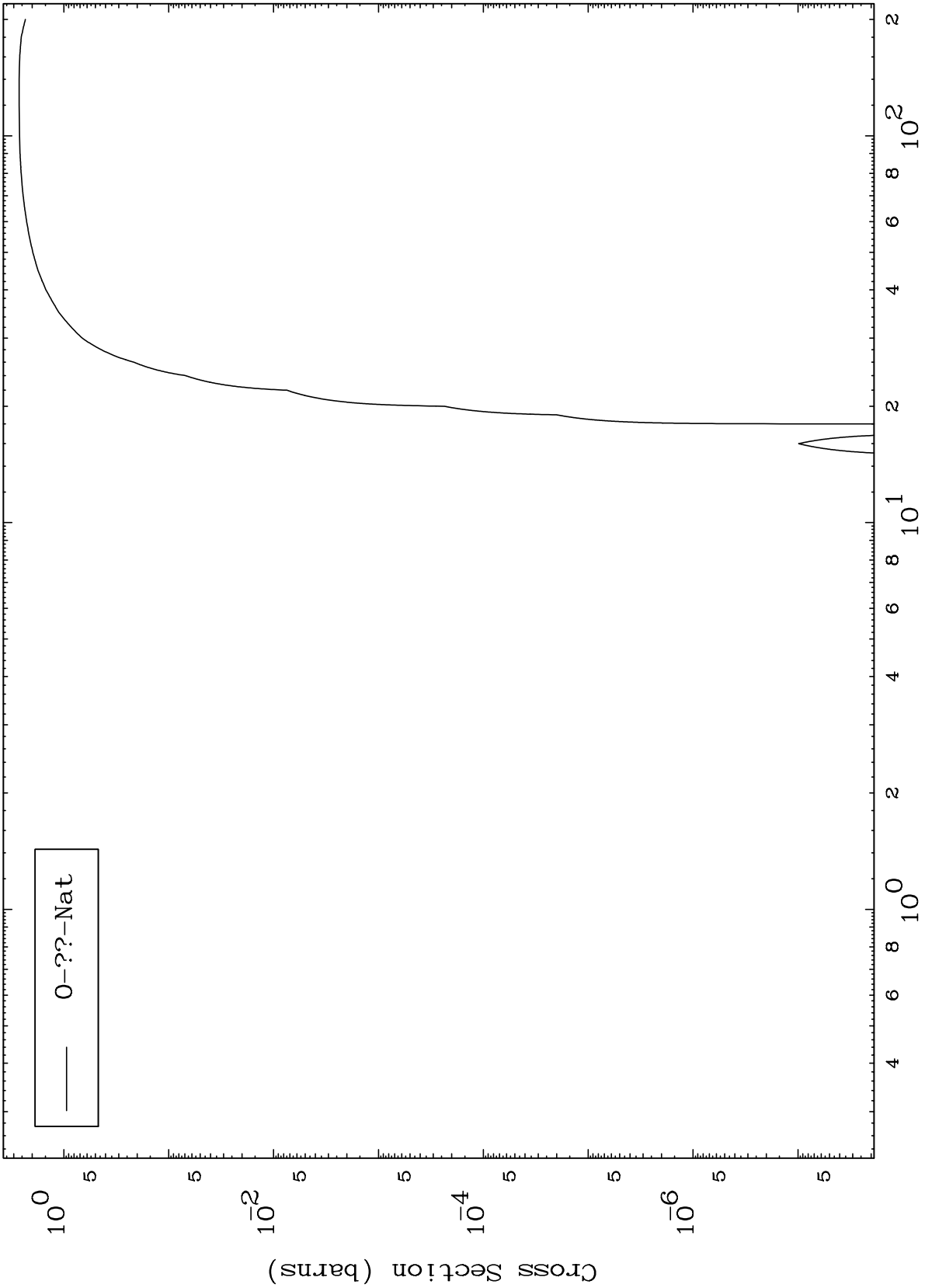
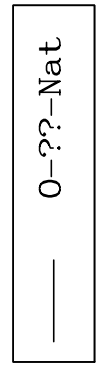


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Fission

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Radionuclide Production Cross Section

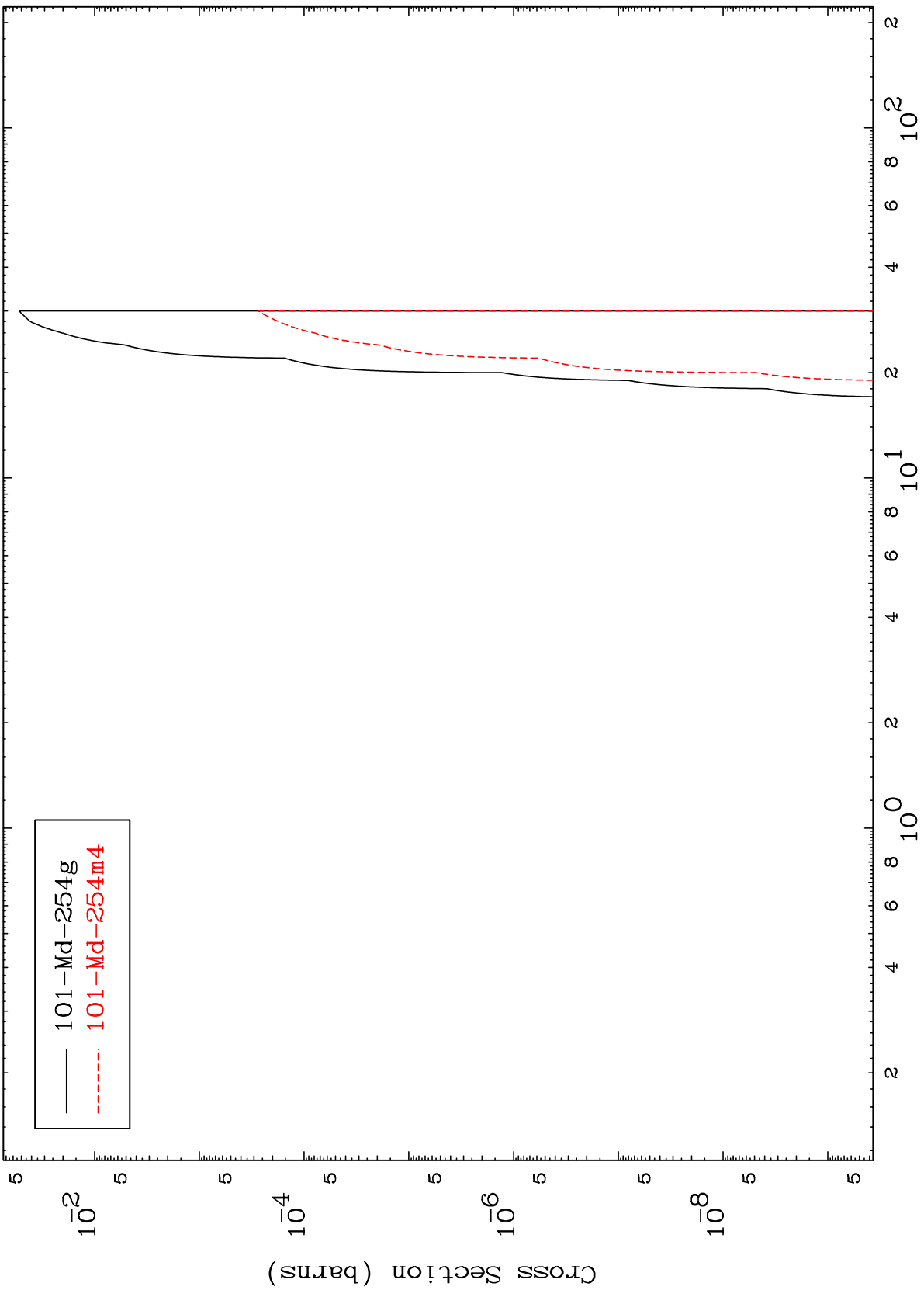


MAT 9955

101-Md-254

(n,  $\alpha$ )

Radionuclide Production Cross Section



101-Md-254

Incident Energy (MeV)