

Program EVALPLOT
(Version 2021-1)

by

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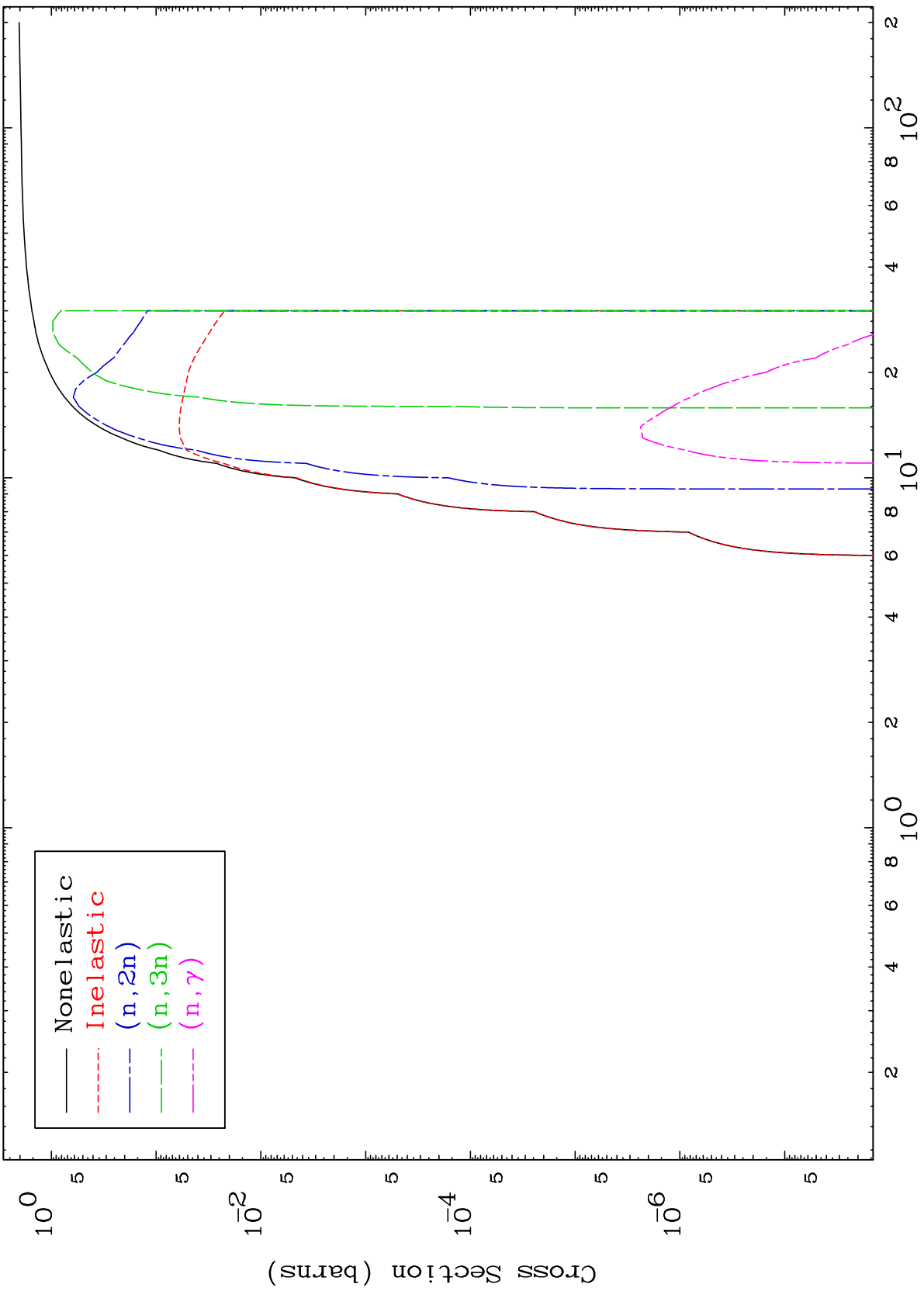
Press Mouse Button to Start

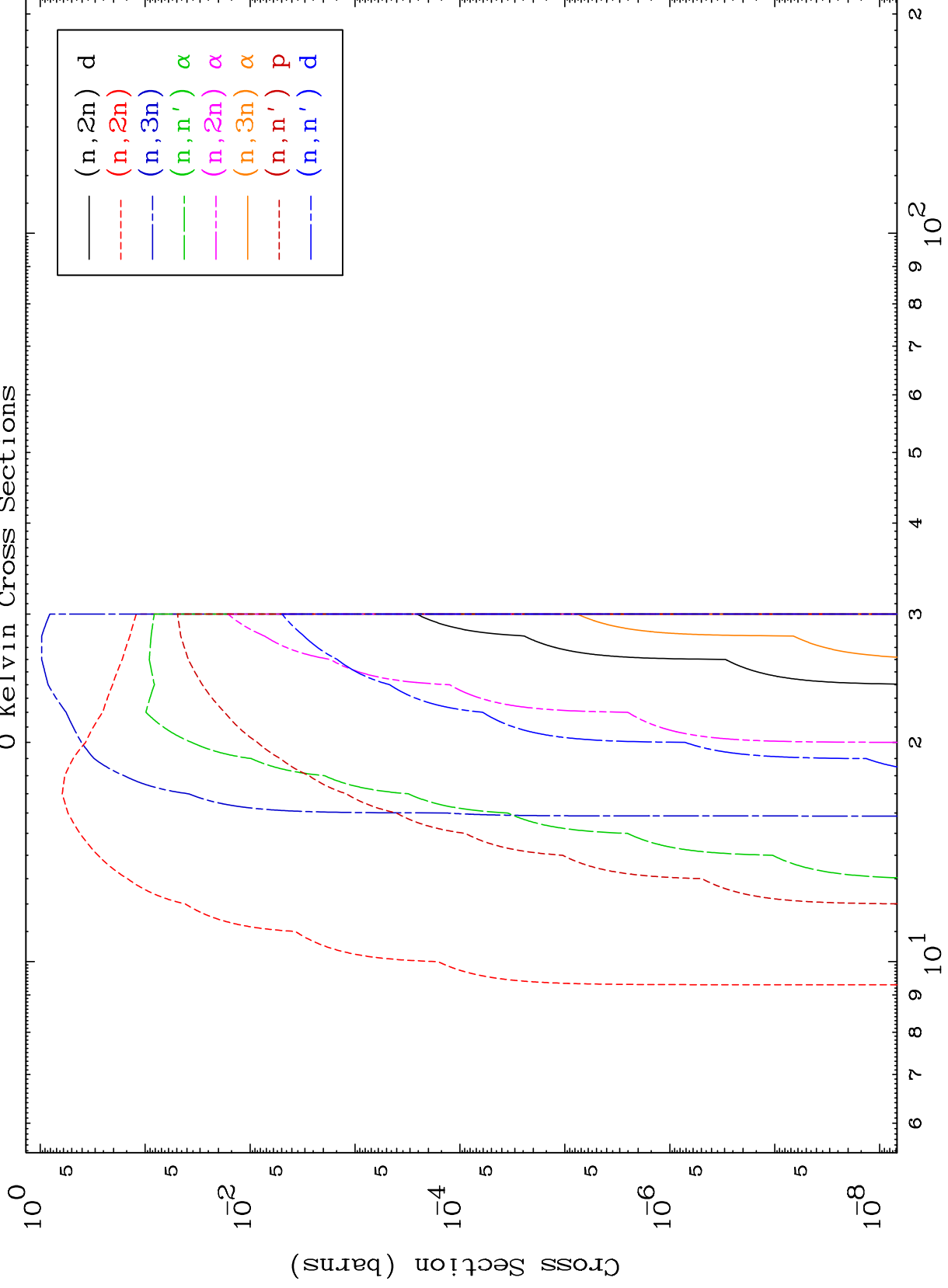
MAT 4146

α Major

41-Nb-100

0 Kelvin Cross Sections

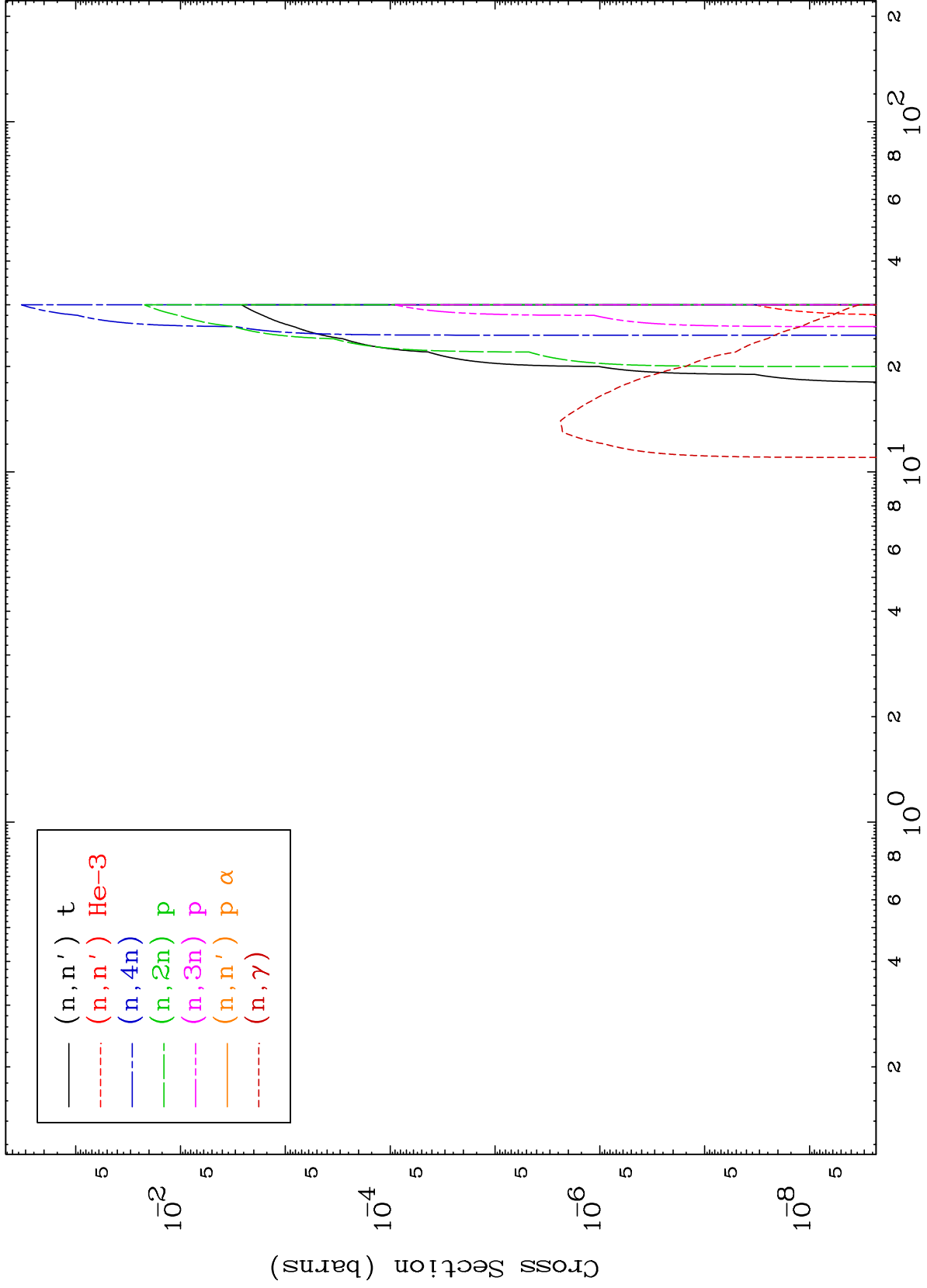


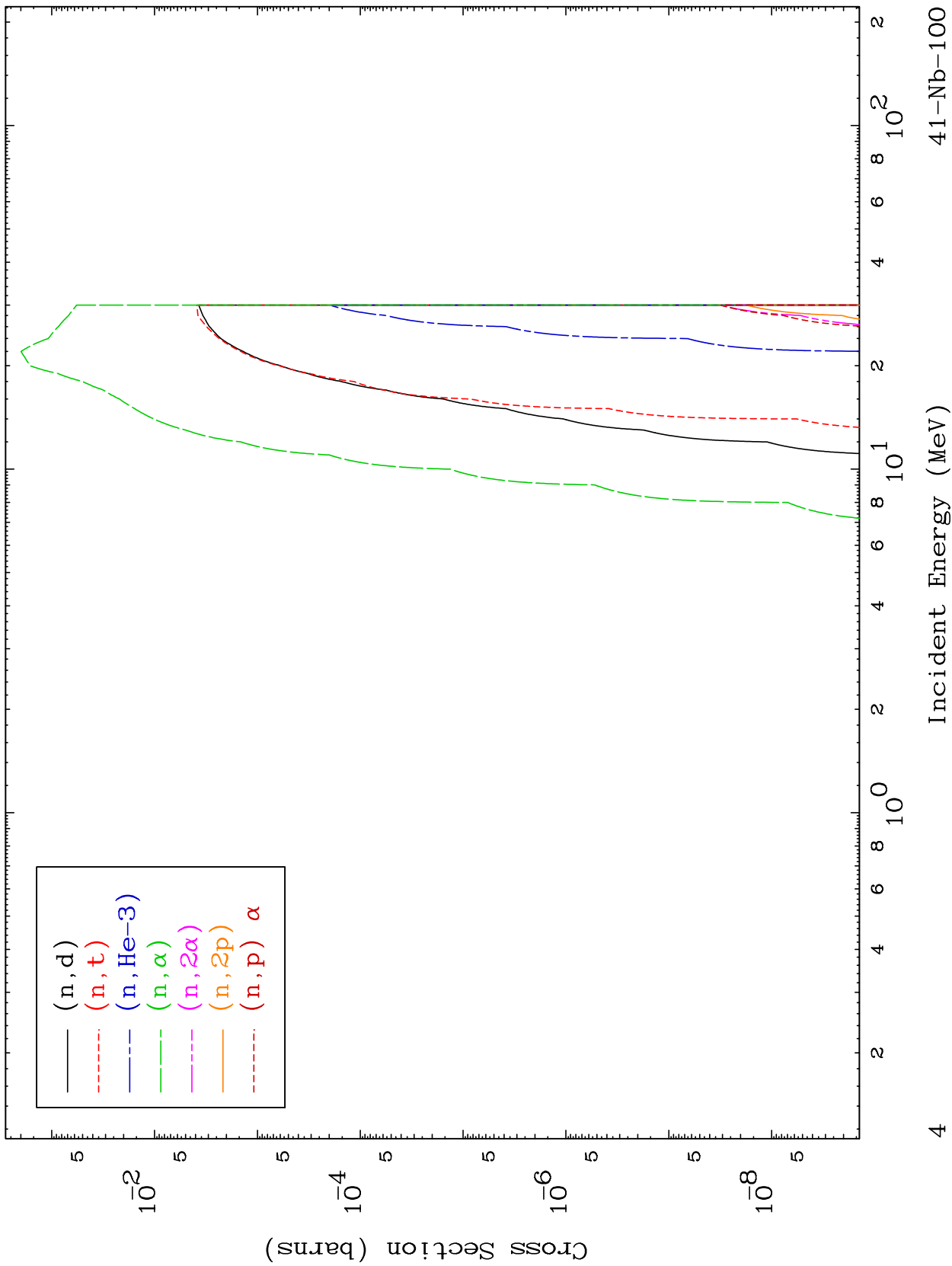


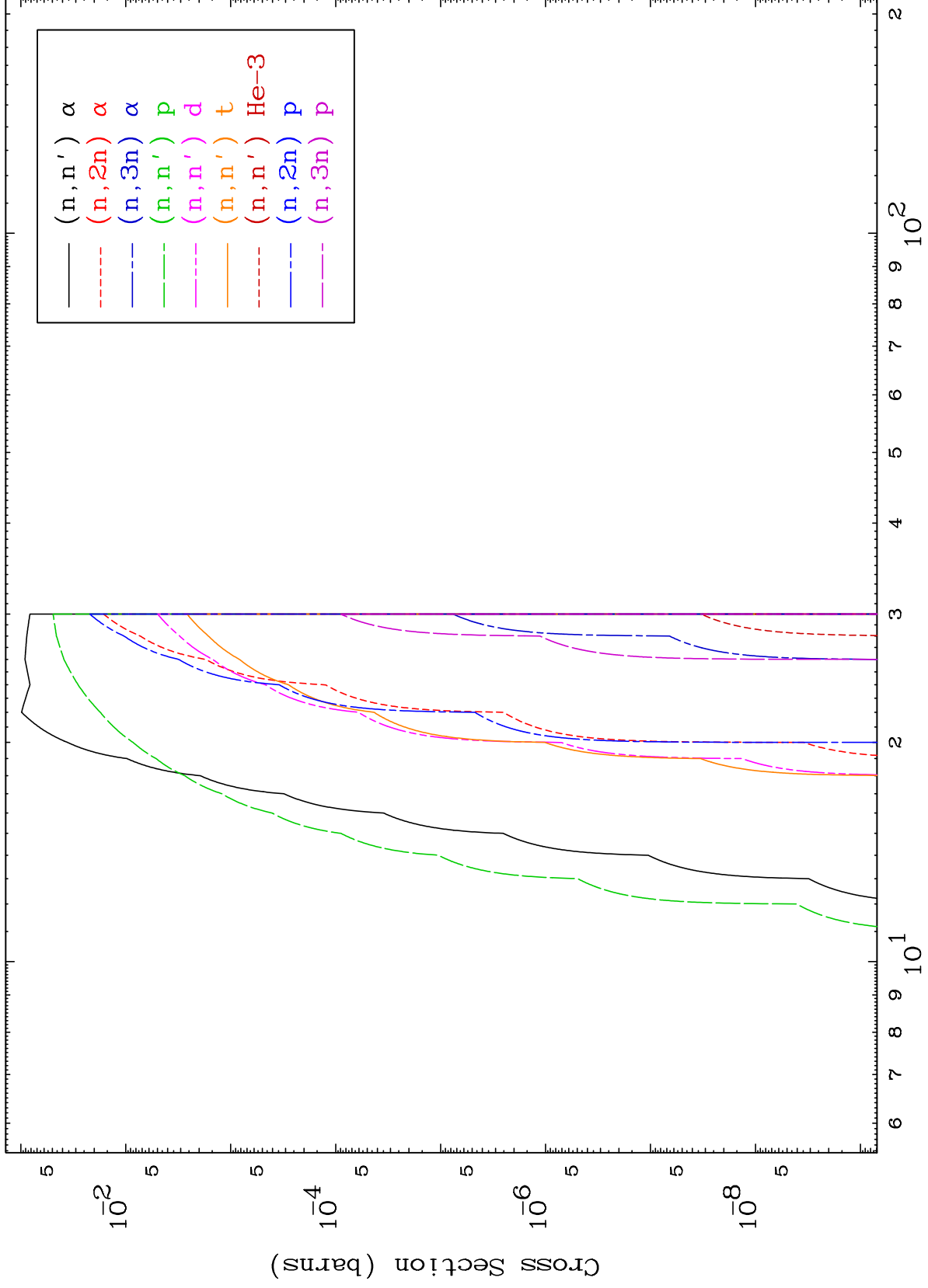
MAT 4146

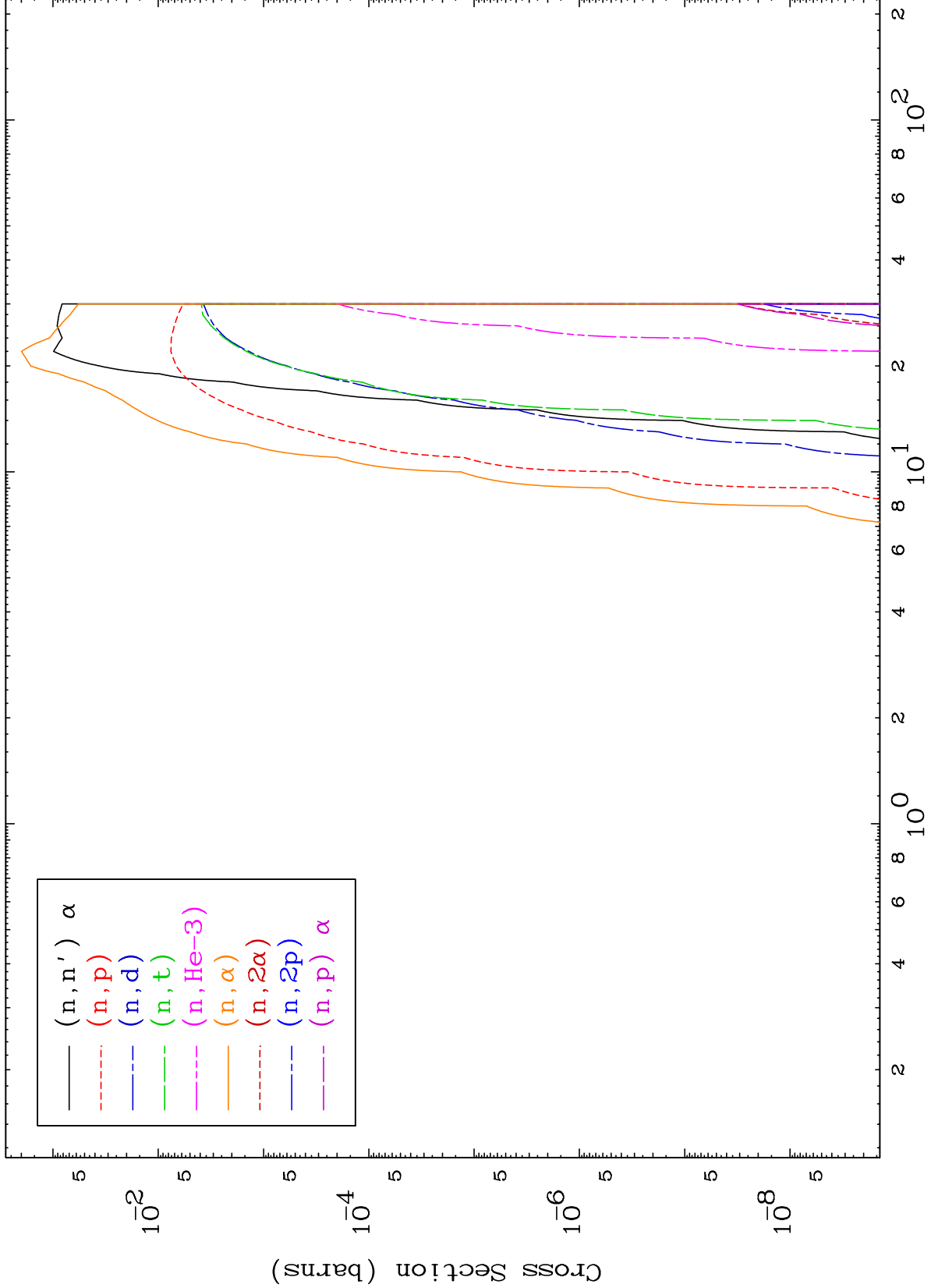
α Neutron Absorption
0 Kelvin Cross Sections

41-Nb-100





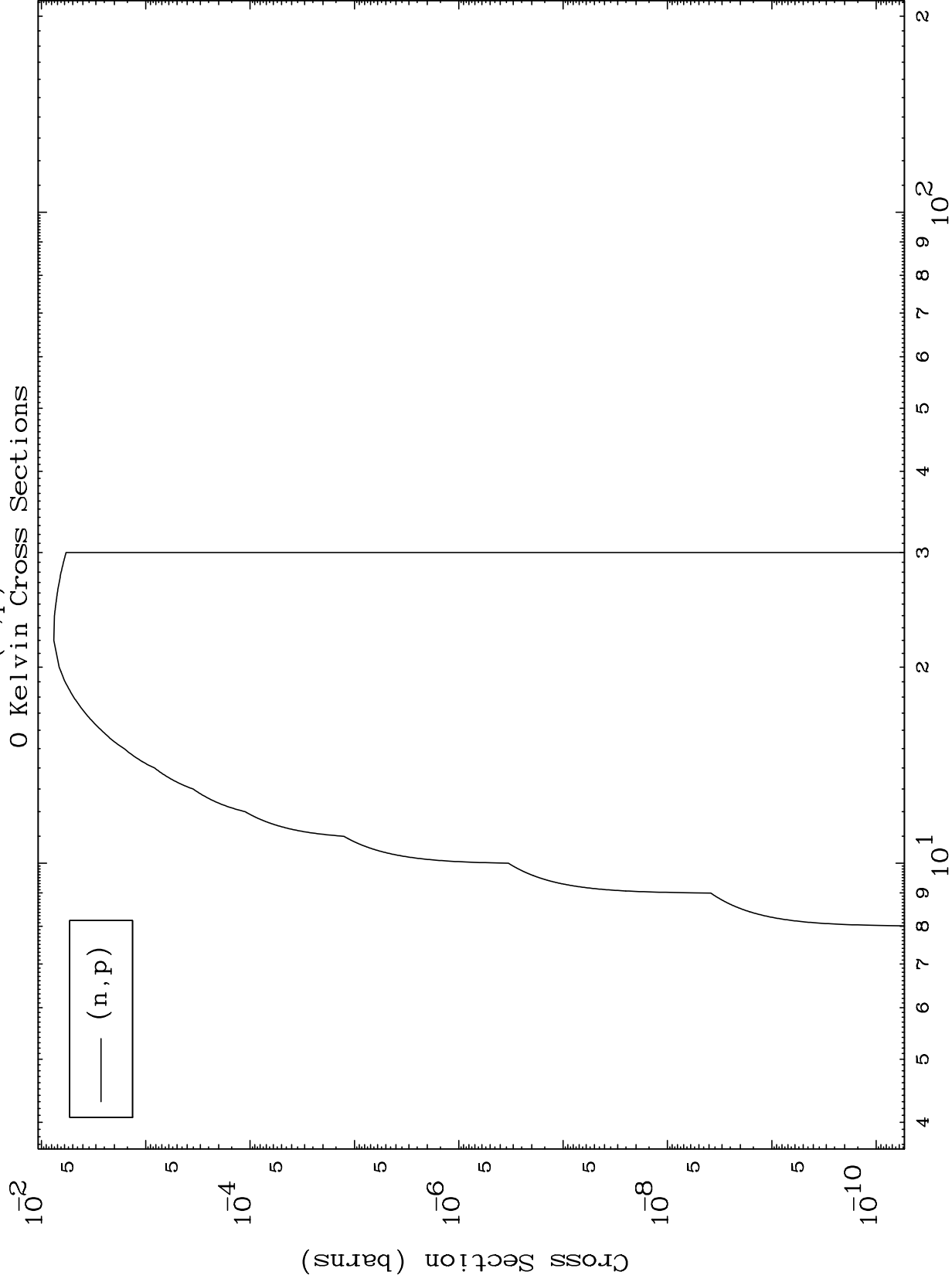




MAT 4146

(α, p) Levels

41-Nb-100



7

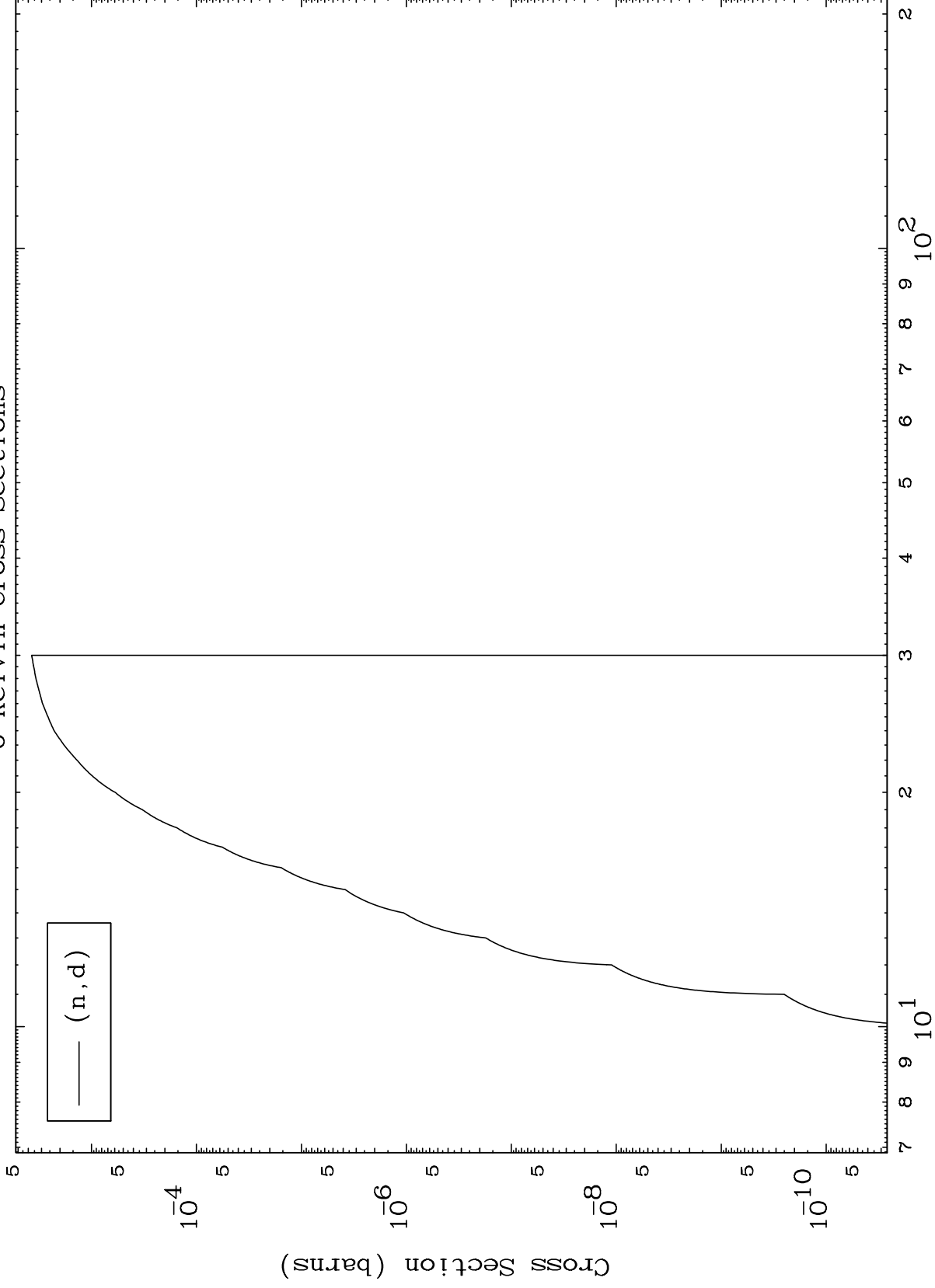
Incident Energy (MeV)

41-Nb-100

MAT 4146

(α, d) Levels
0 Kelvin Cross Sections

41-Nb-100



8

Incident Energy (MeV)

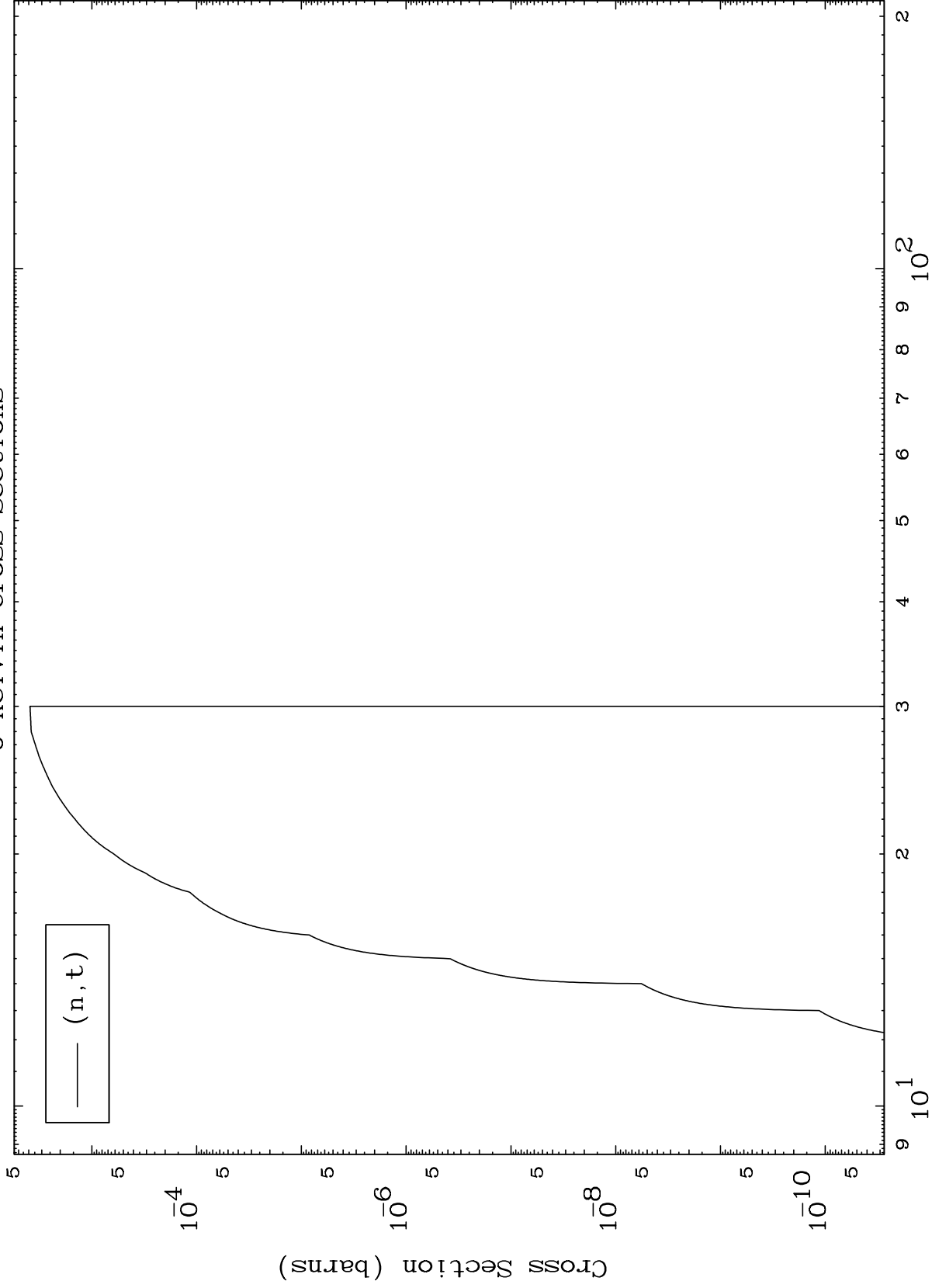
41-Nb-100

MAT 4146

(α, t) Levels

41-Nb-100

0 Kelvin Cross Sections



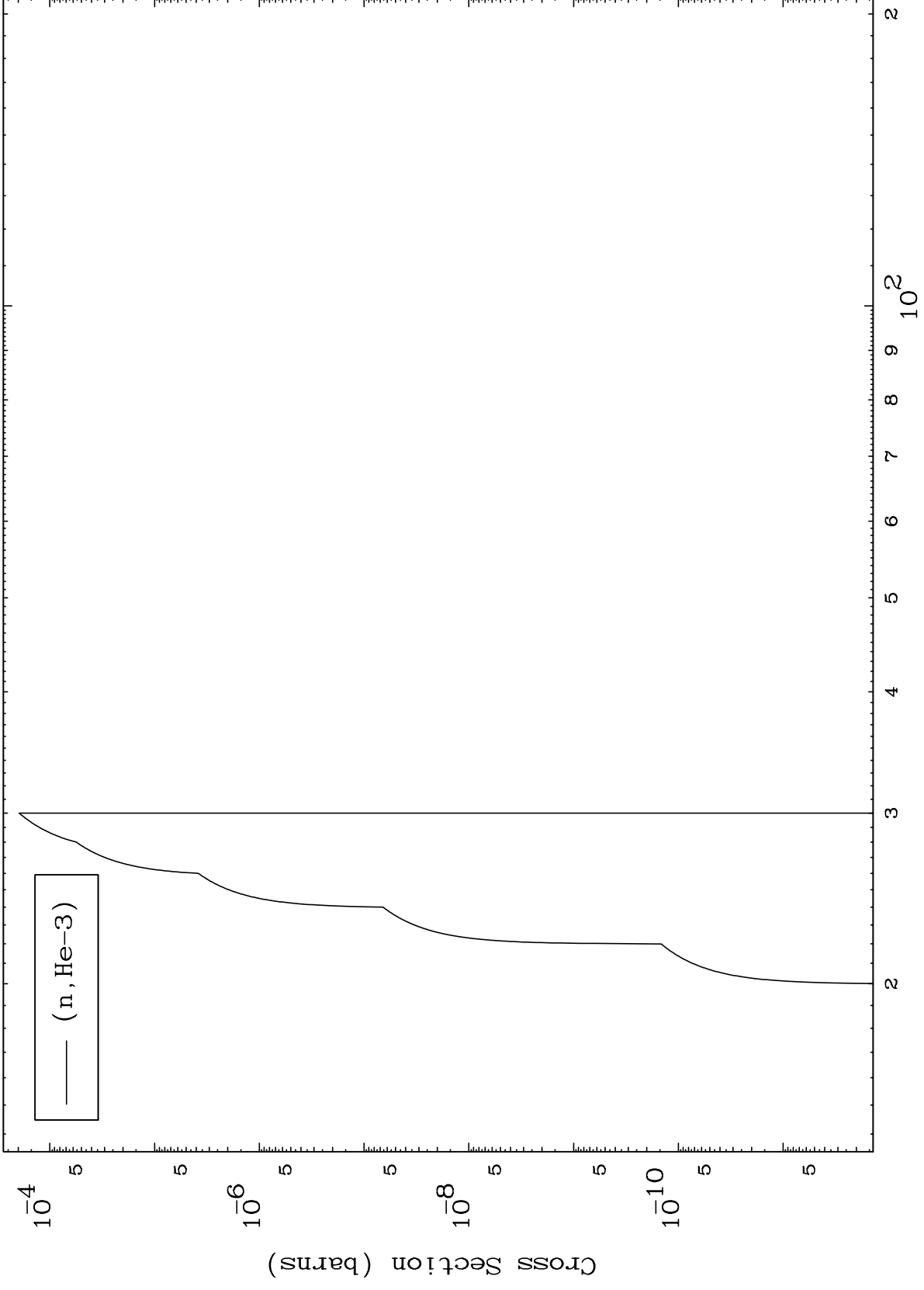
Incident Energy (MeV)

41-Nb-100

MAT 4146

($\alpha, \text{He}3$) Levels
0 Kelvin Cross Sections

41-Nb-100



10

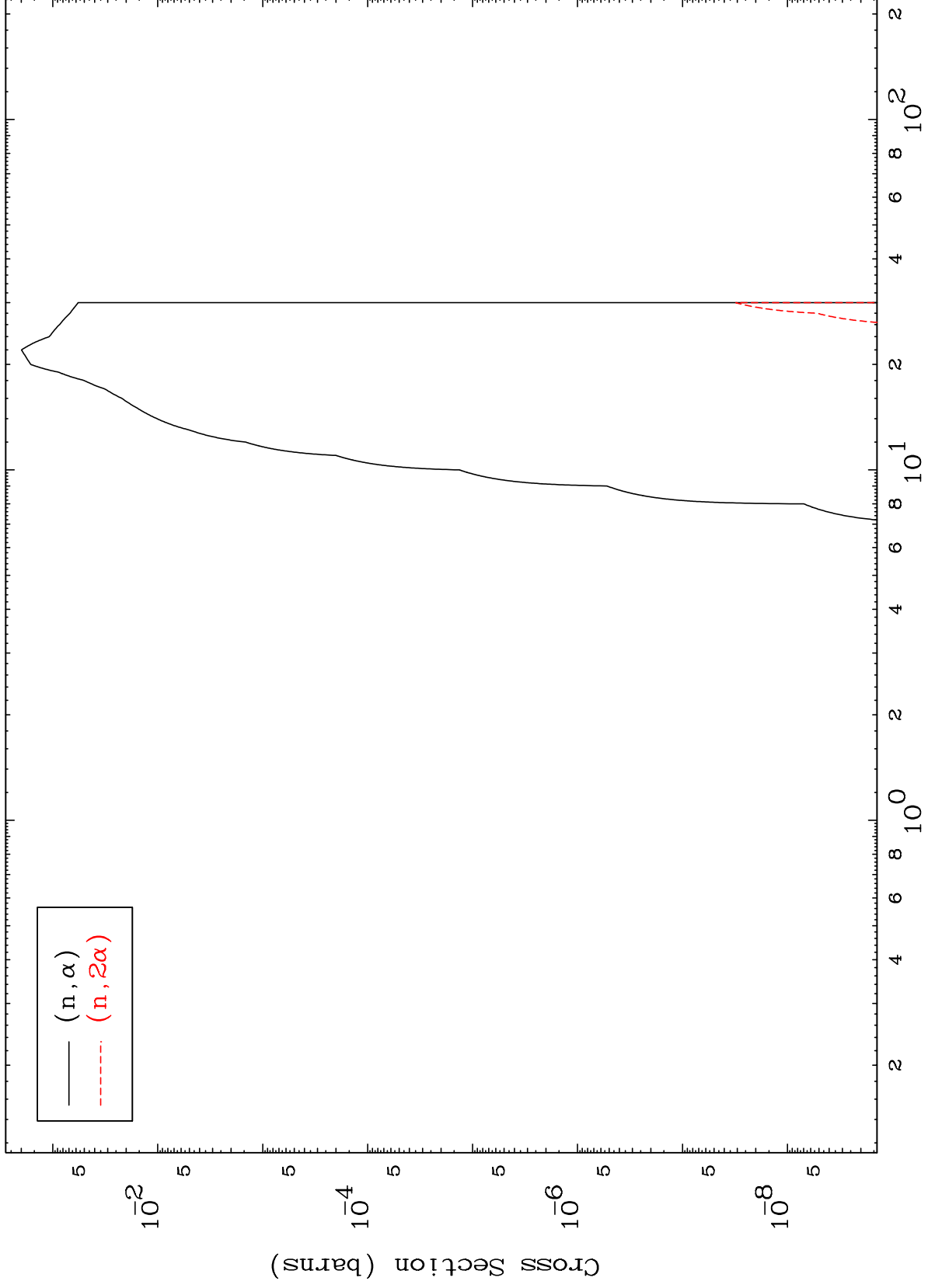
Incident Energy (MeV)

41-Nb-100

MAT 4146

(α, α) Levels
0 Kelvin Cross Sections

41-Nb-100

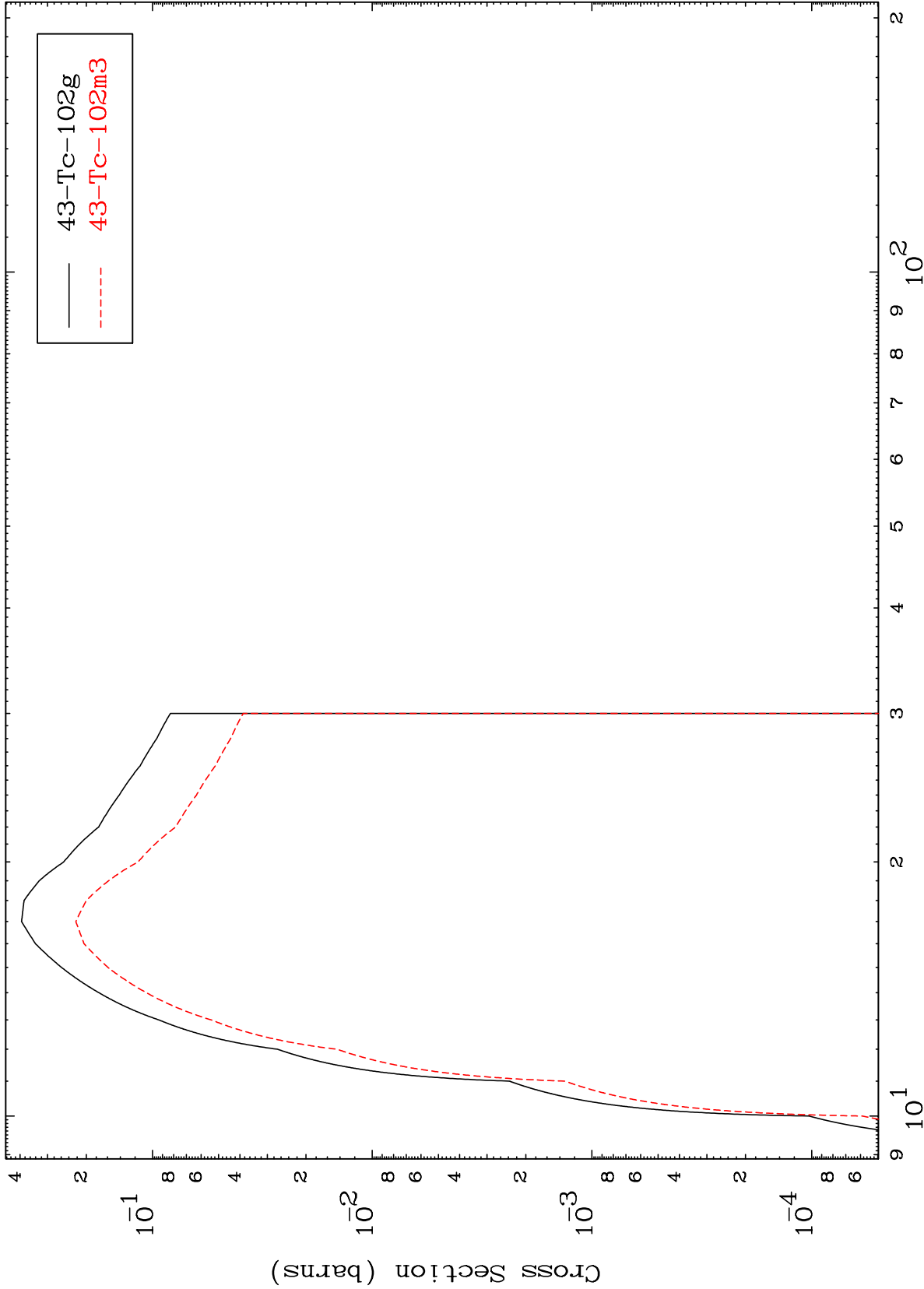


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(n,2n)

41-Nb-100

Radionuclide Production Cross Section



12

Incident Energy (MeV)

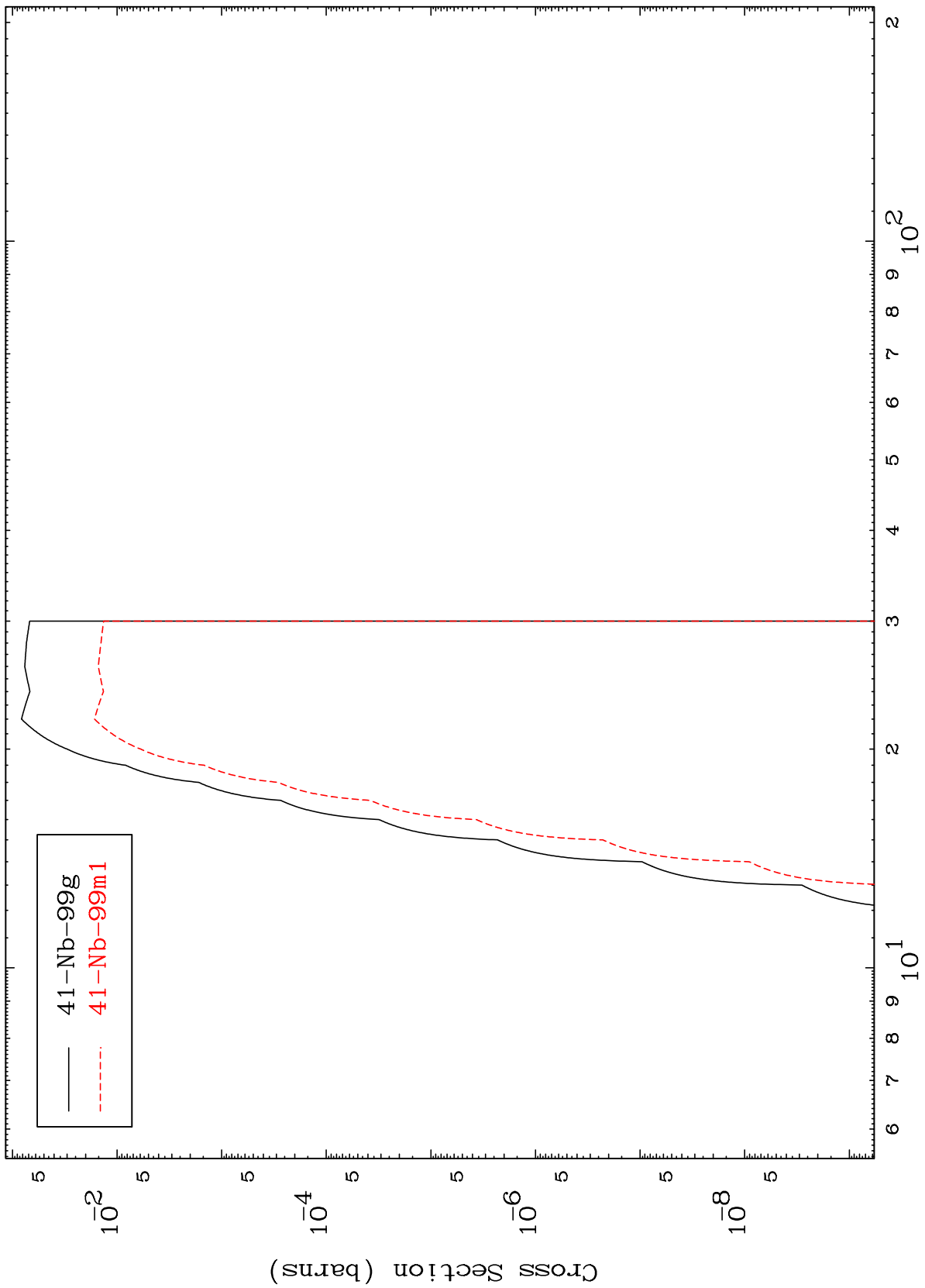
41-Nb-100

MAT 4146

41-Nb-100

(n,n') α

Radionuclide Production Cross Section



13

Incident Energy (MeV)

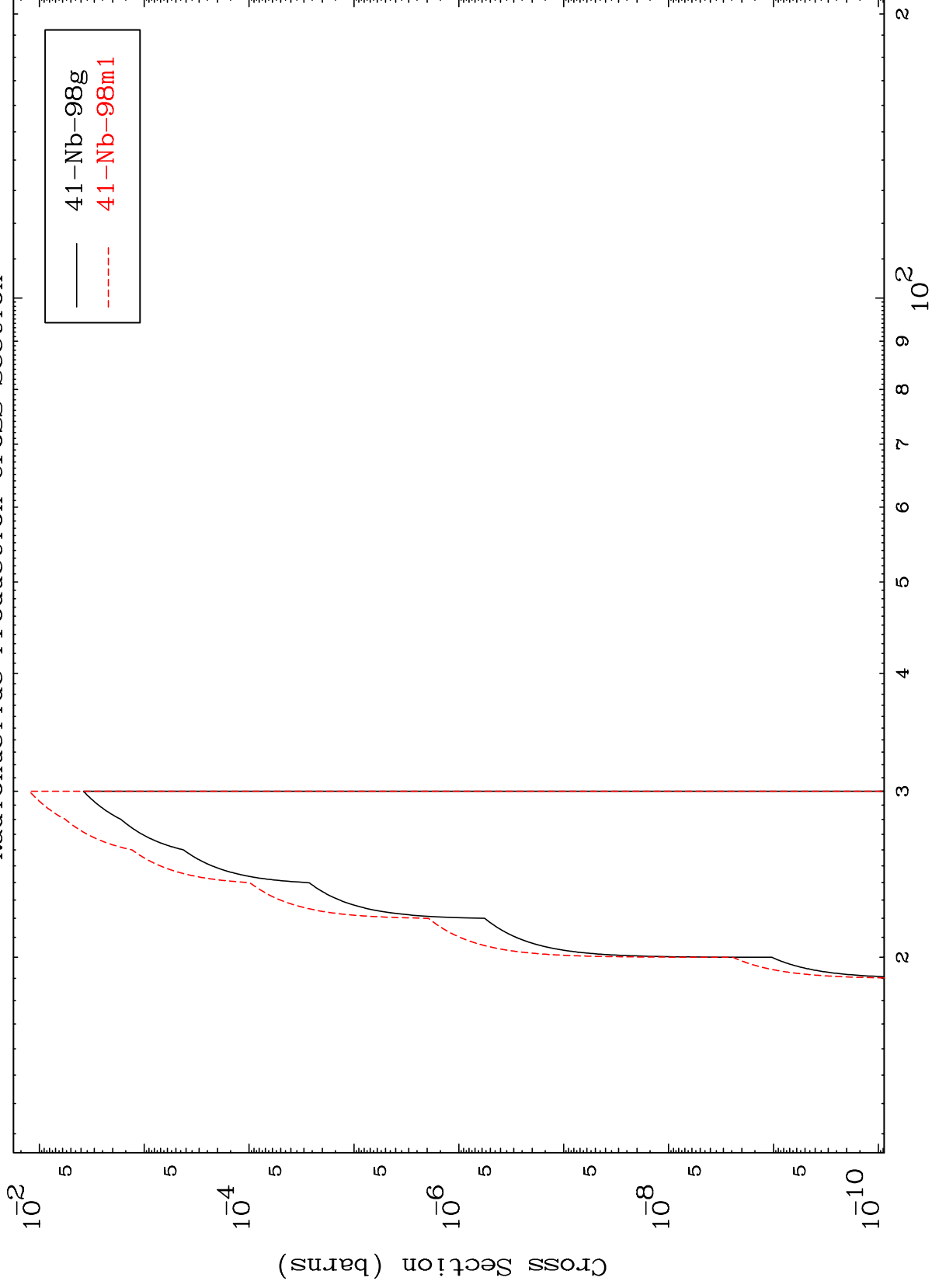
41-Nb-100

MAT 4146

$(n,2n) \alpha$

41-Nb-100

Radionuclide Production Cross Section



14

Incident Energy (MeV)

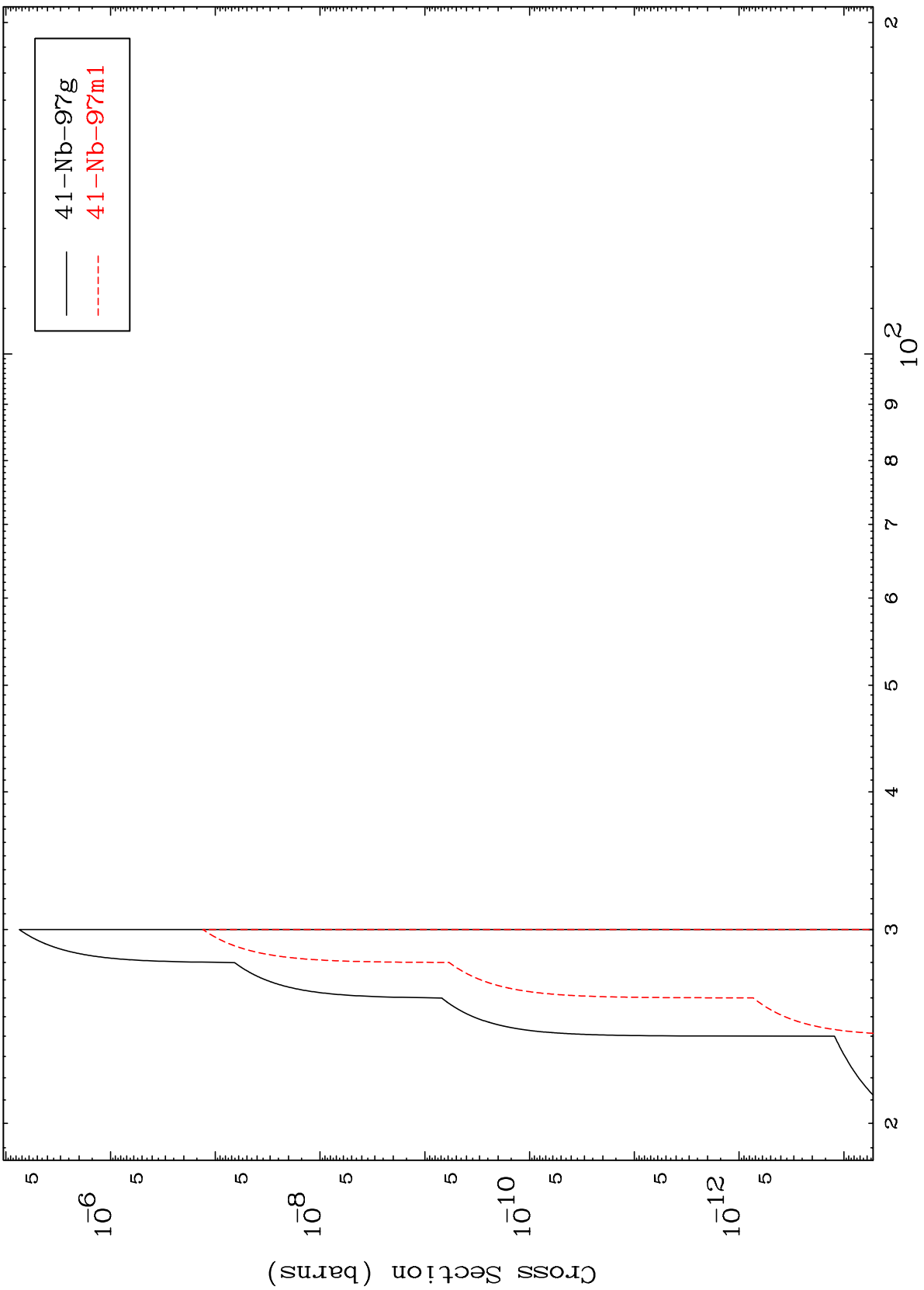
41-Nb-100

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(n,3n) α

41-Nb-100

Radionuclide Production Cross Section



15

Incident Energy (MeV)

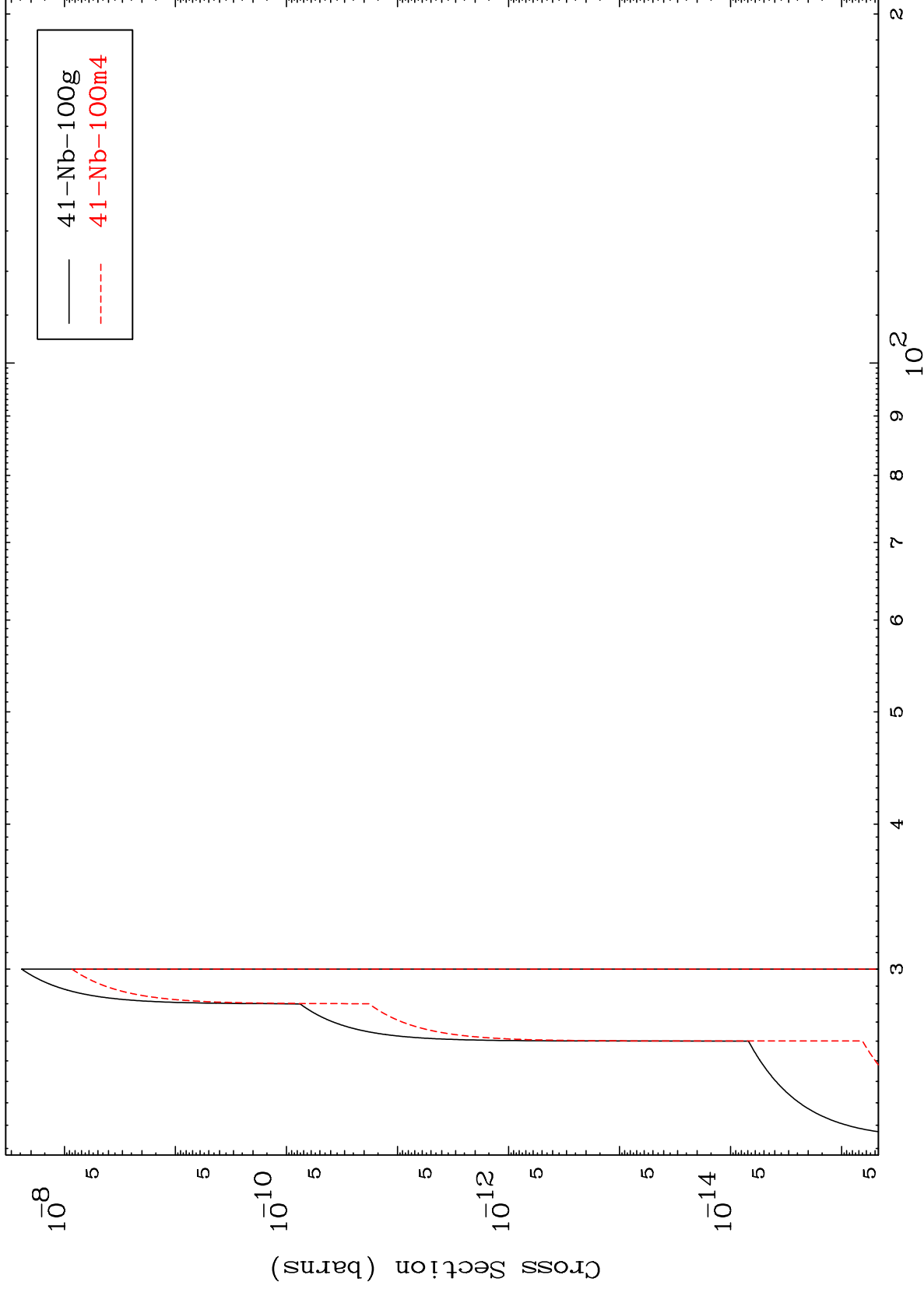
41-Nb-100

MAT 4146

(n,n') He-3

41-Nb-100

Radionuclide Production Cross Section



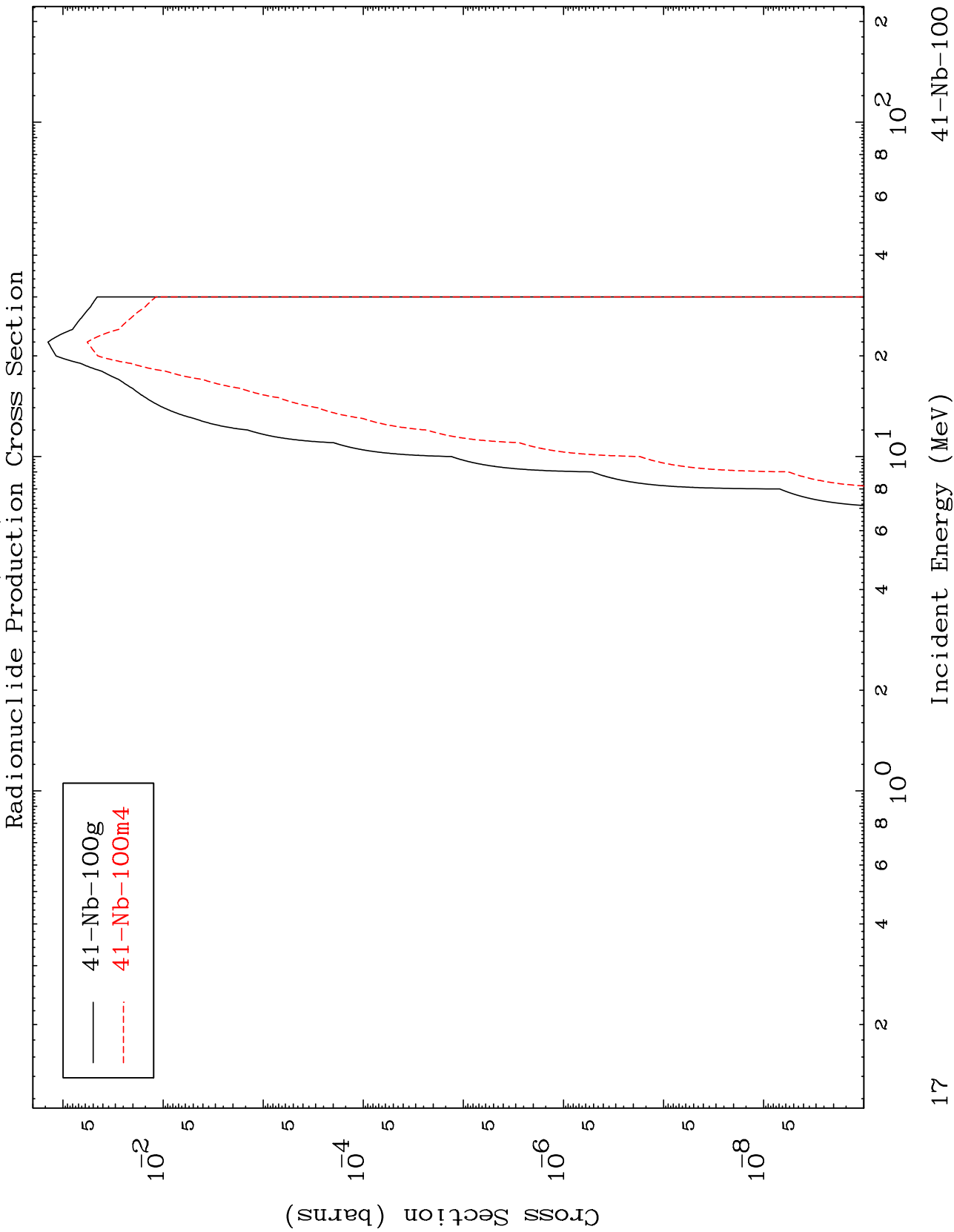
16

Incident Energy (MeV)

41-Nb-100

MAT 4146

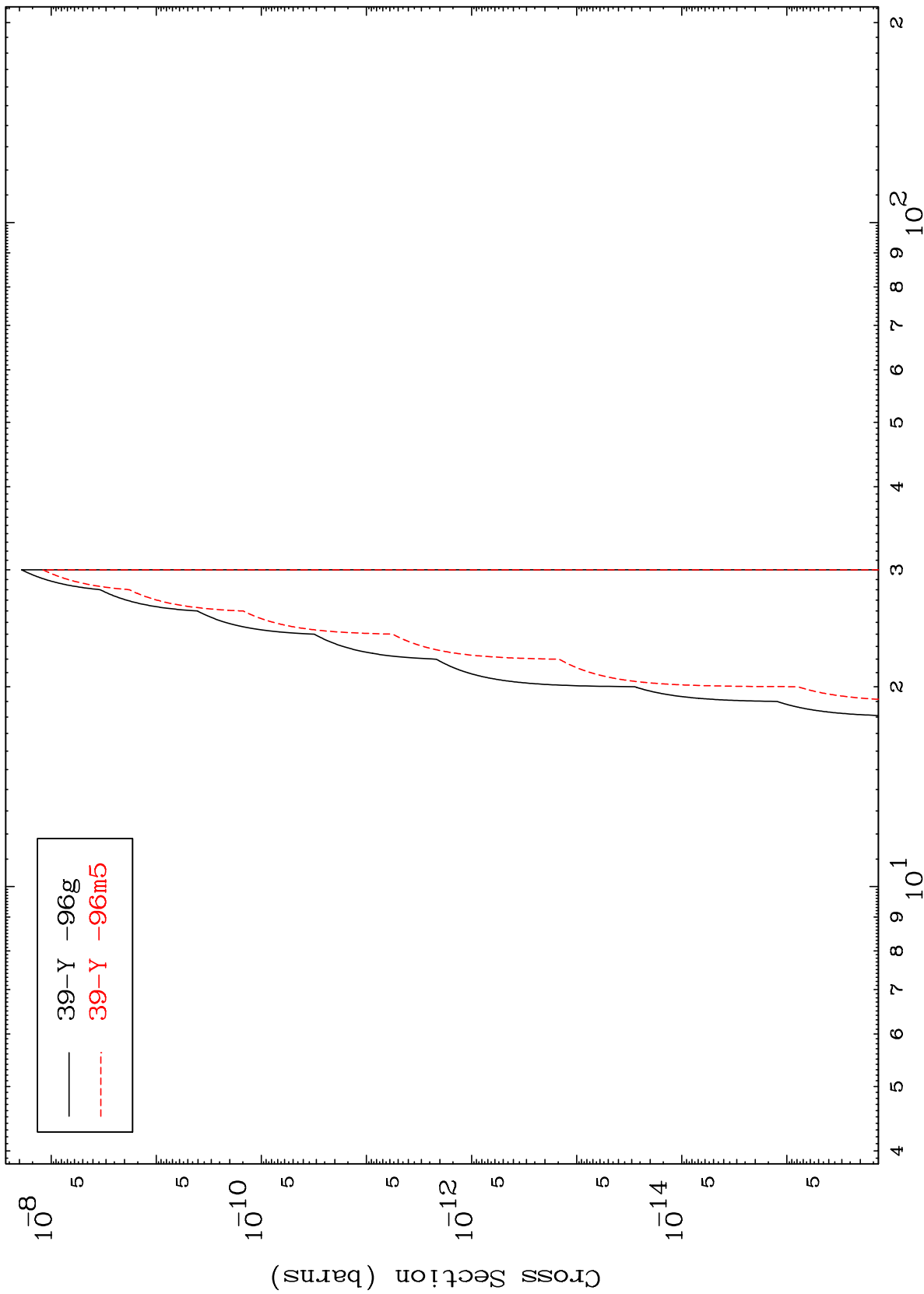
41-Nb-100



MAT 4146

41-Nb-100

Radionuclide Production Cross Section
(n,2α)



— 39-Y -96g
- - - 39-Y -96m5

41-Nb-100

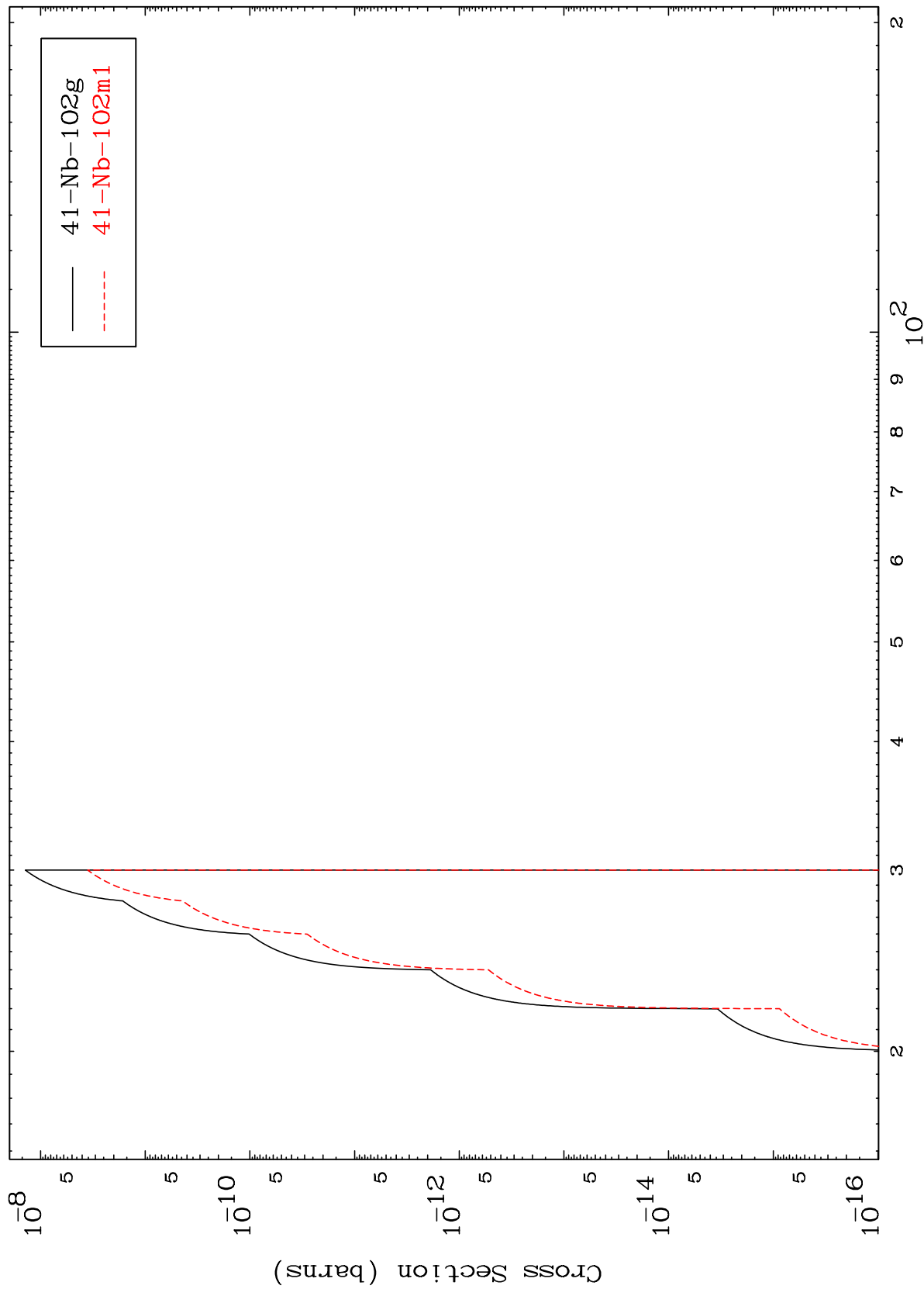
Incident Energy (MeV)

18

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41-Nb-100

(n,2p)
Radionuclide Production Cross Section



19

Incident Energy (MeV)

41-Nb-100