

Program EVALPLOT  
(Version 2021-1)

by

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(Present Contact Information)

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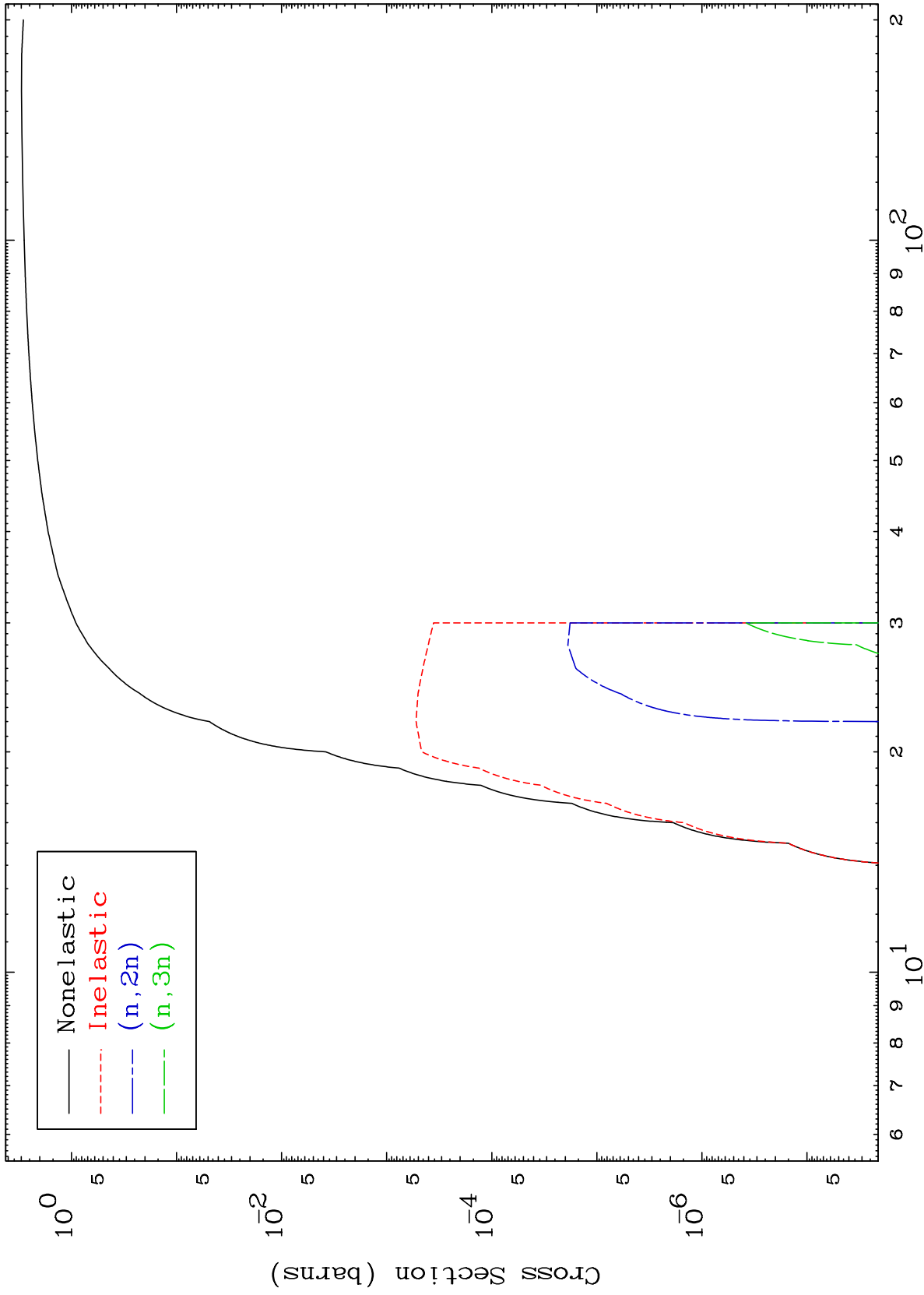
E.Mail:redcullen1@comcast.net  
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9343

0 Kelvin  $\alpha$  Major  
Cross Sections

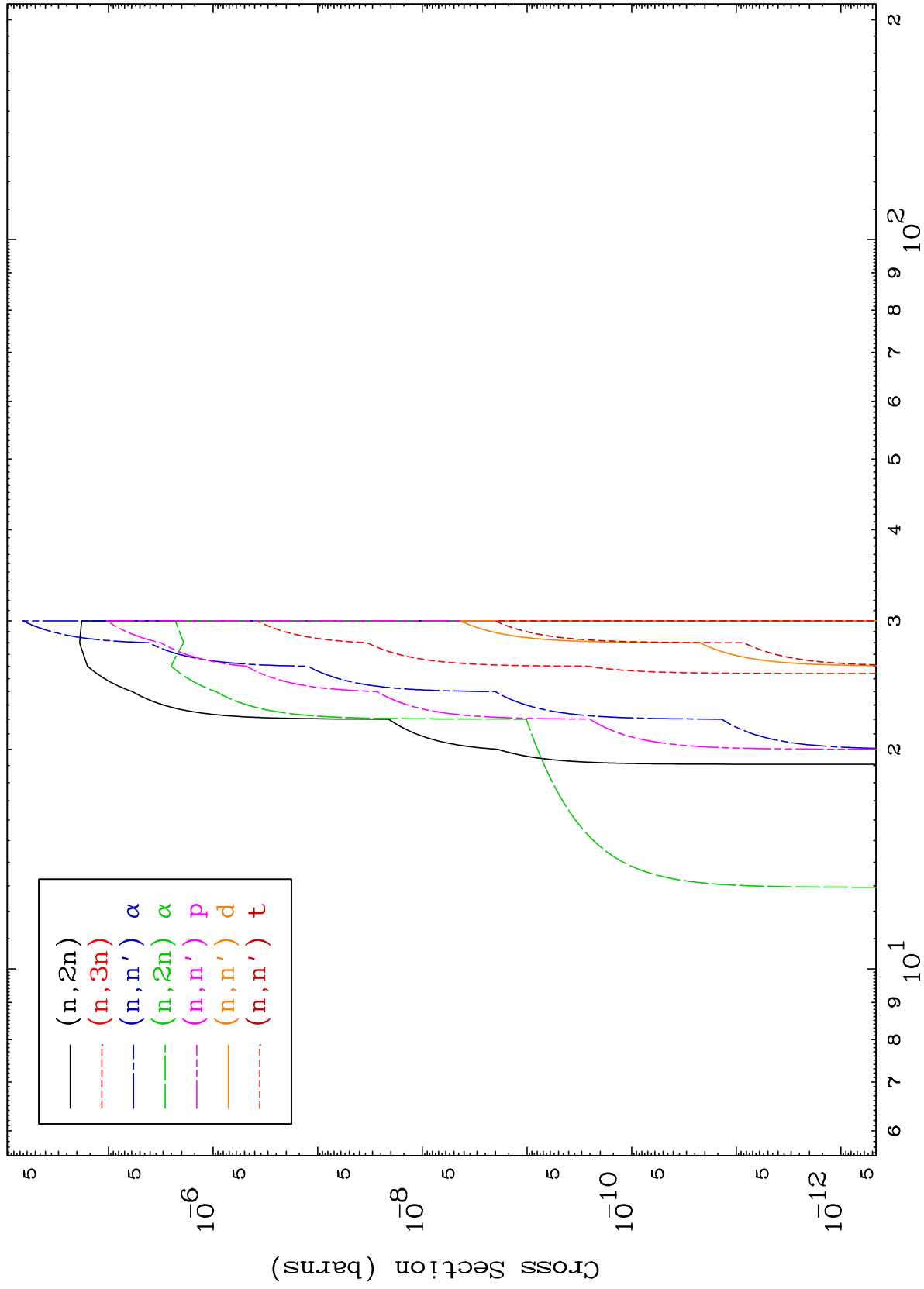
93-Np-236



1

Incident Energy (MeV)

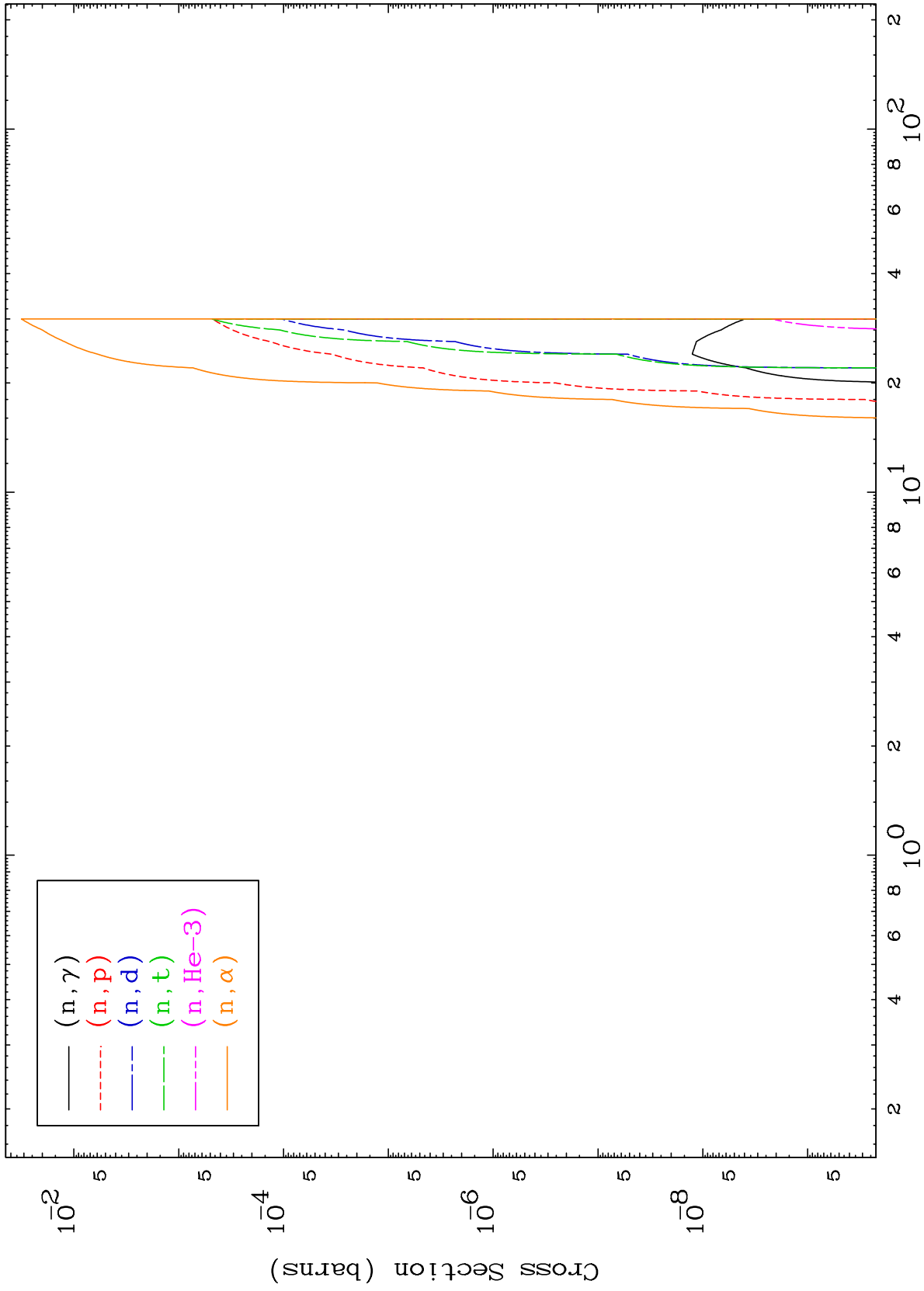
93-Np-236



MAT 9343

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

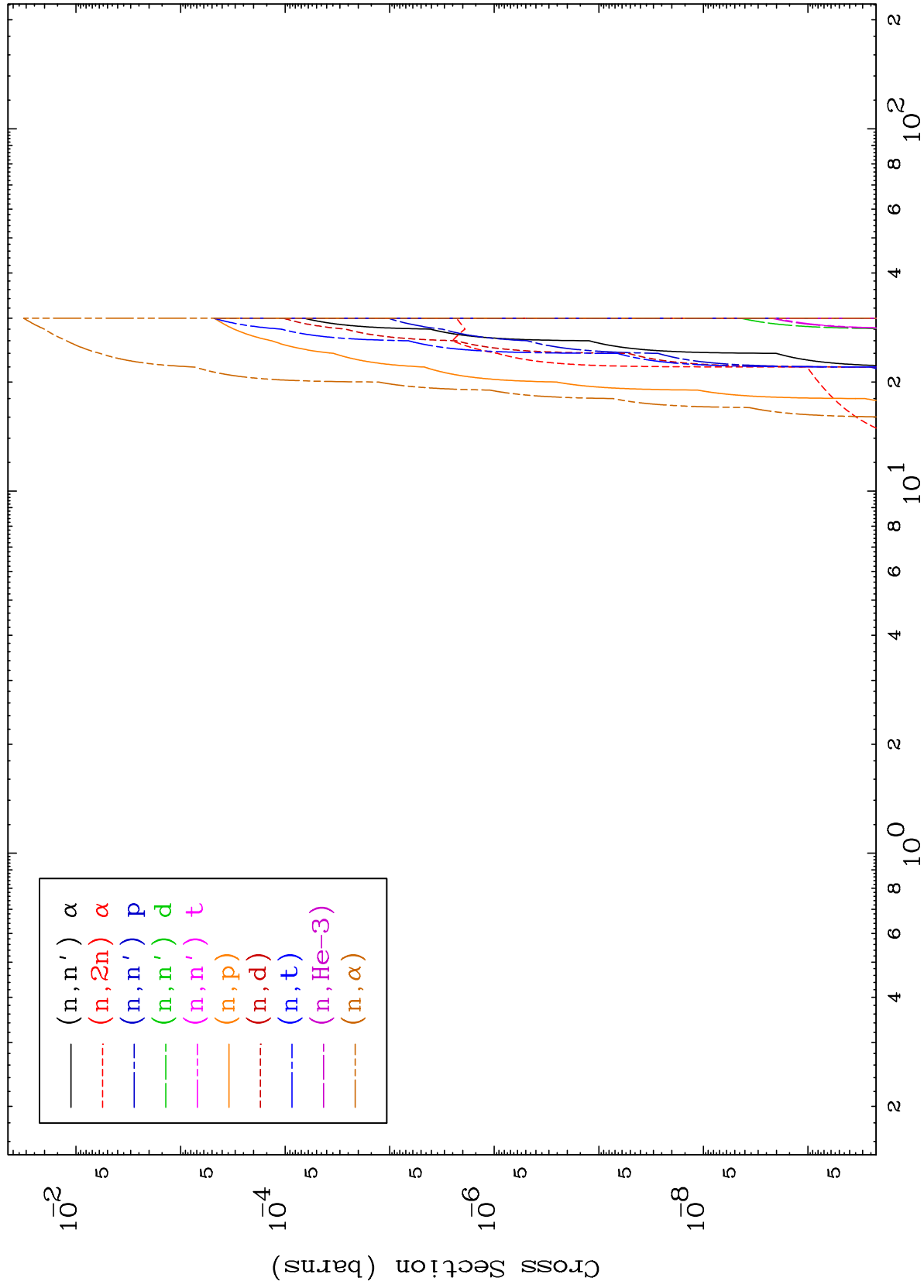
93-Np-236



3

Incident Energy (MeV)

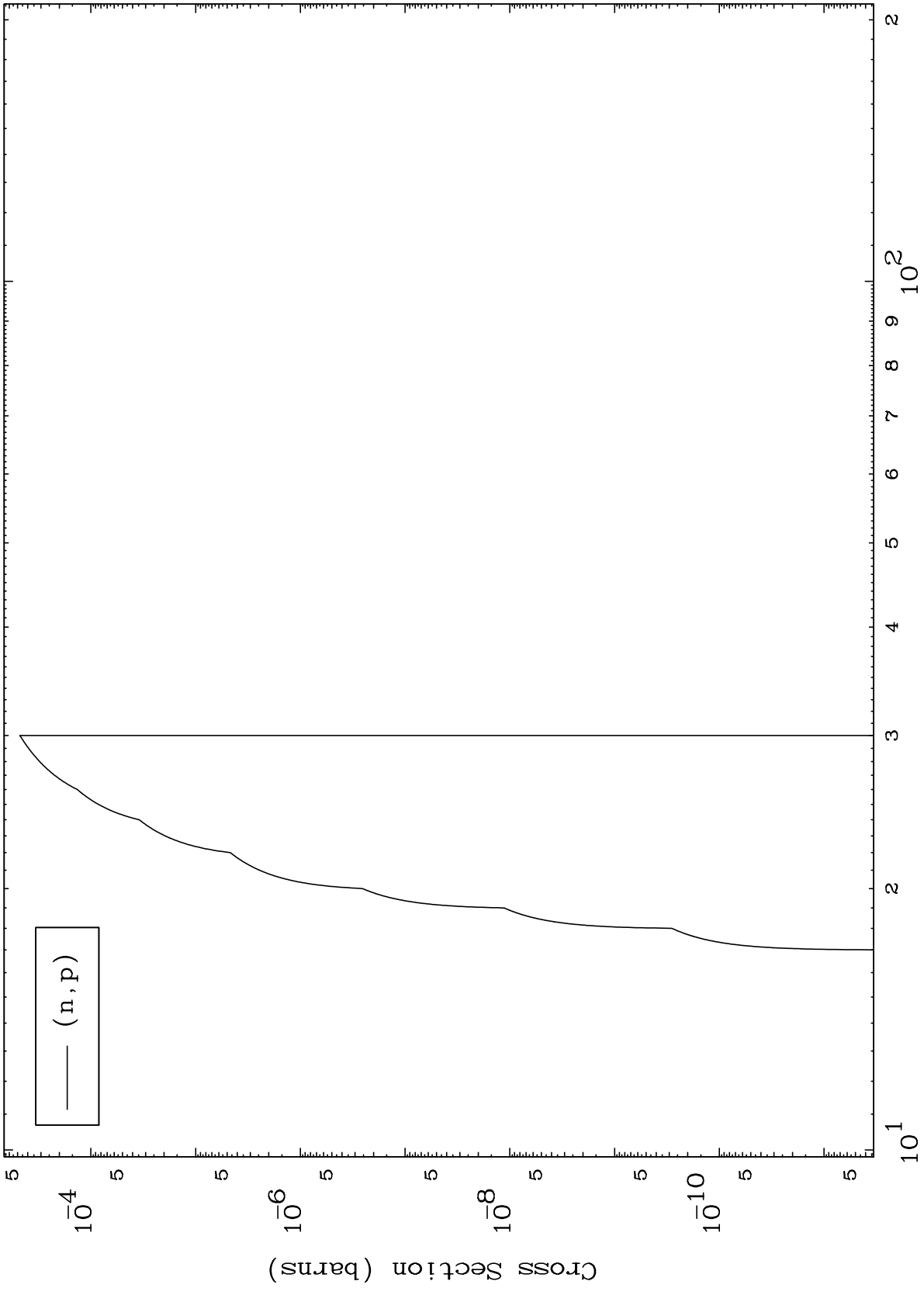
93-Np-236



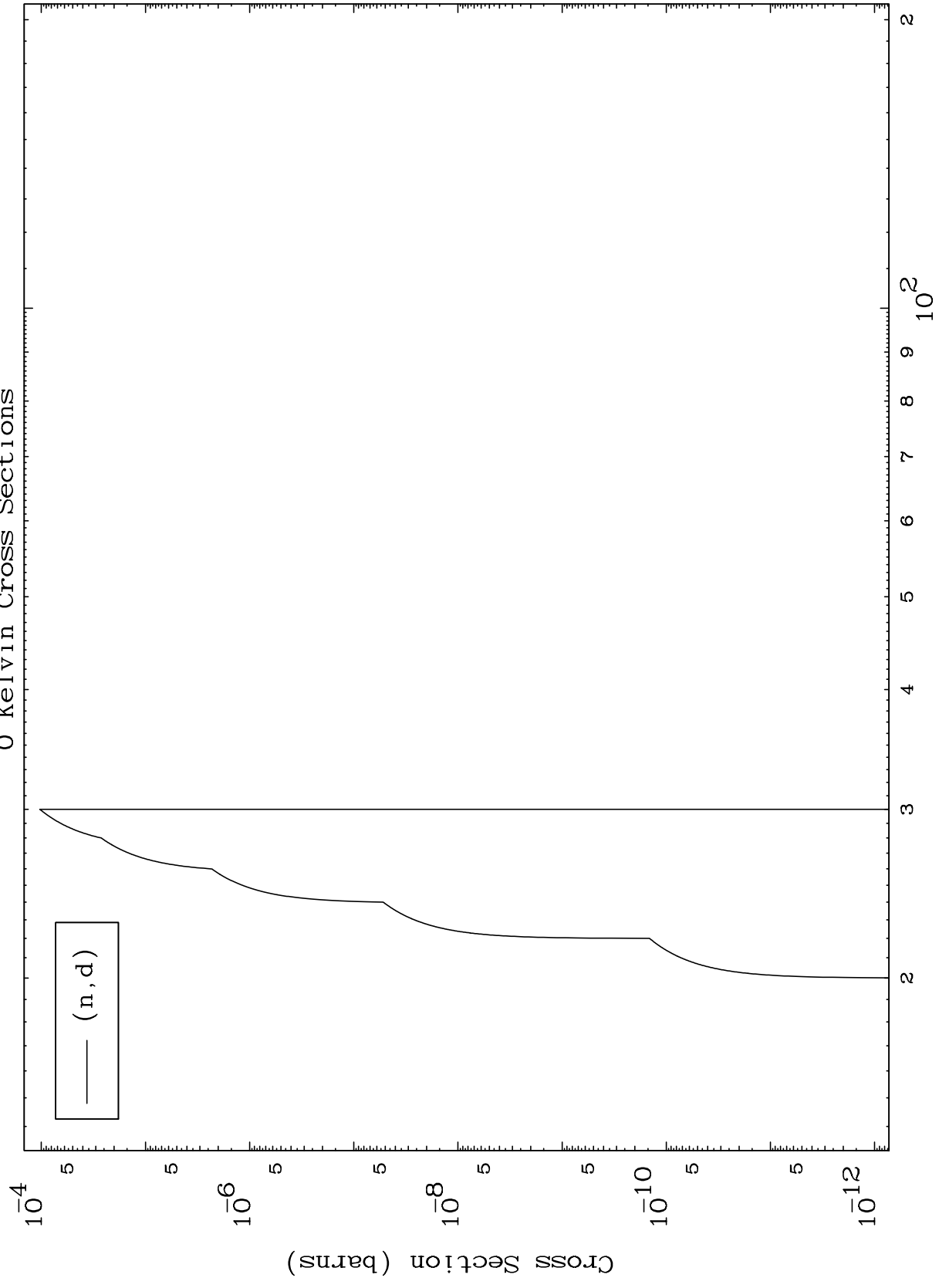
MAT 9343

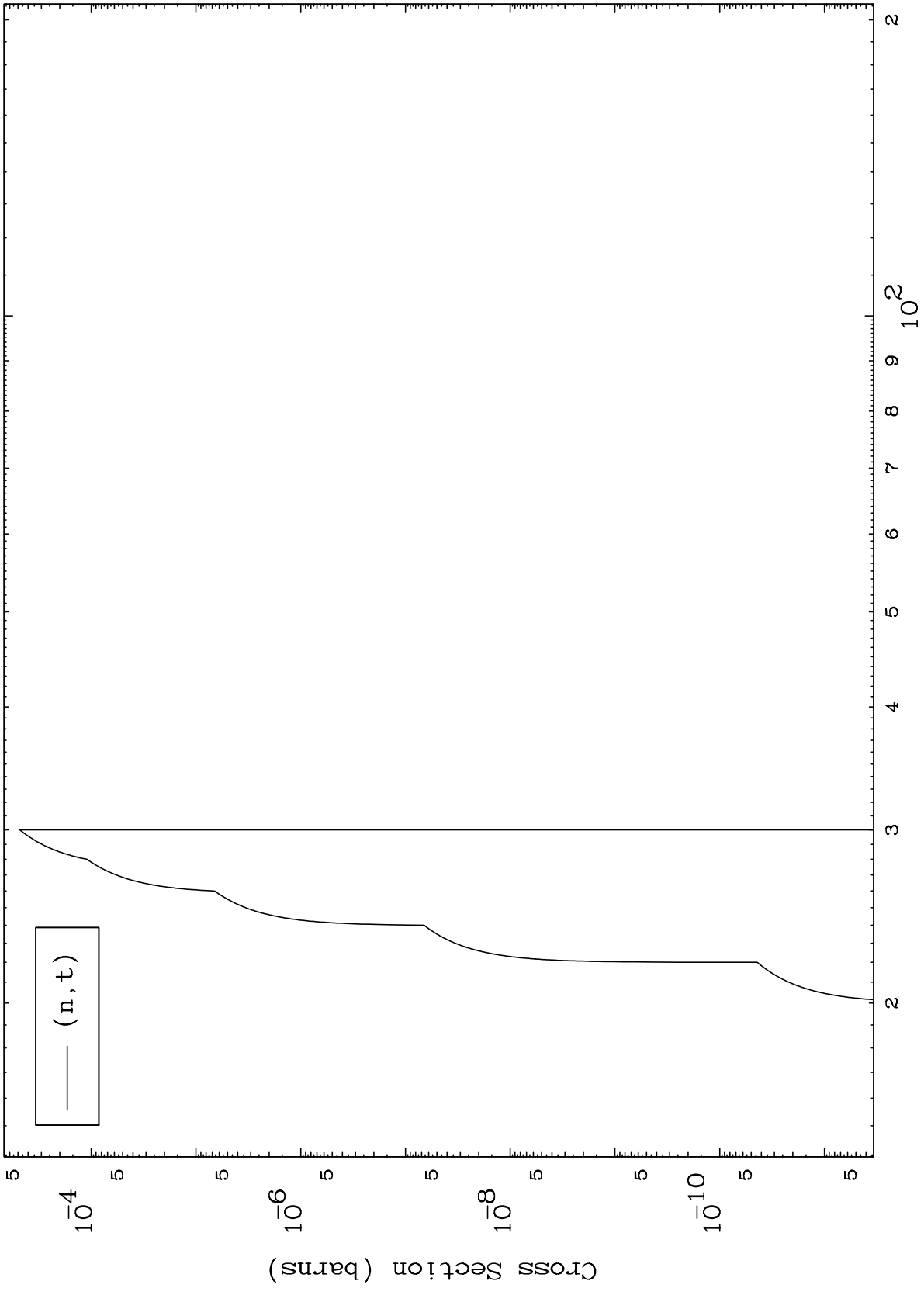
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

93-Np-236

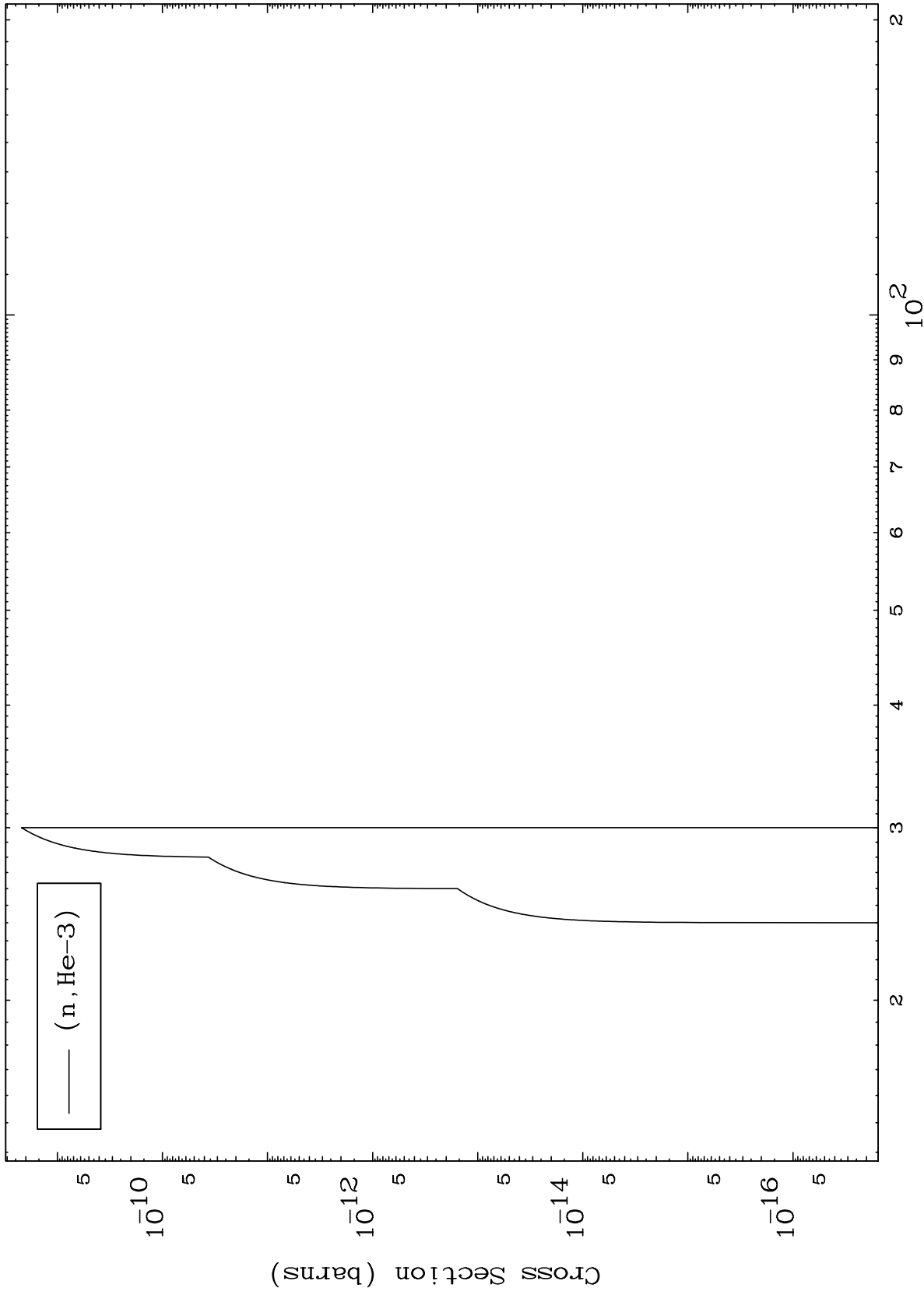


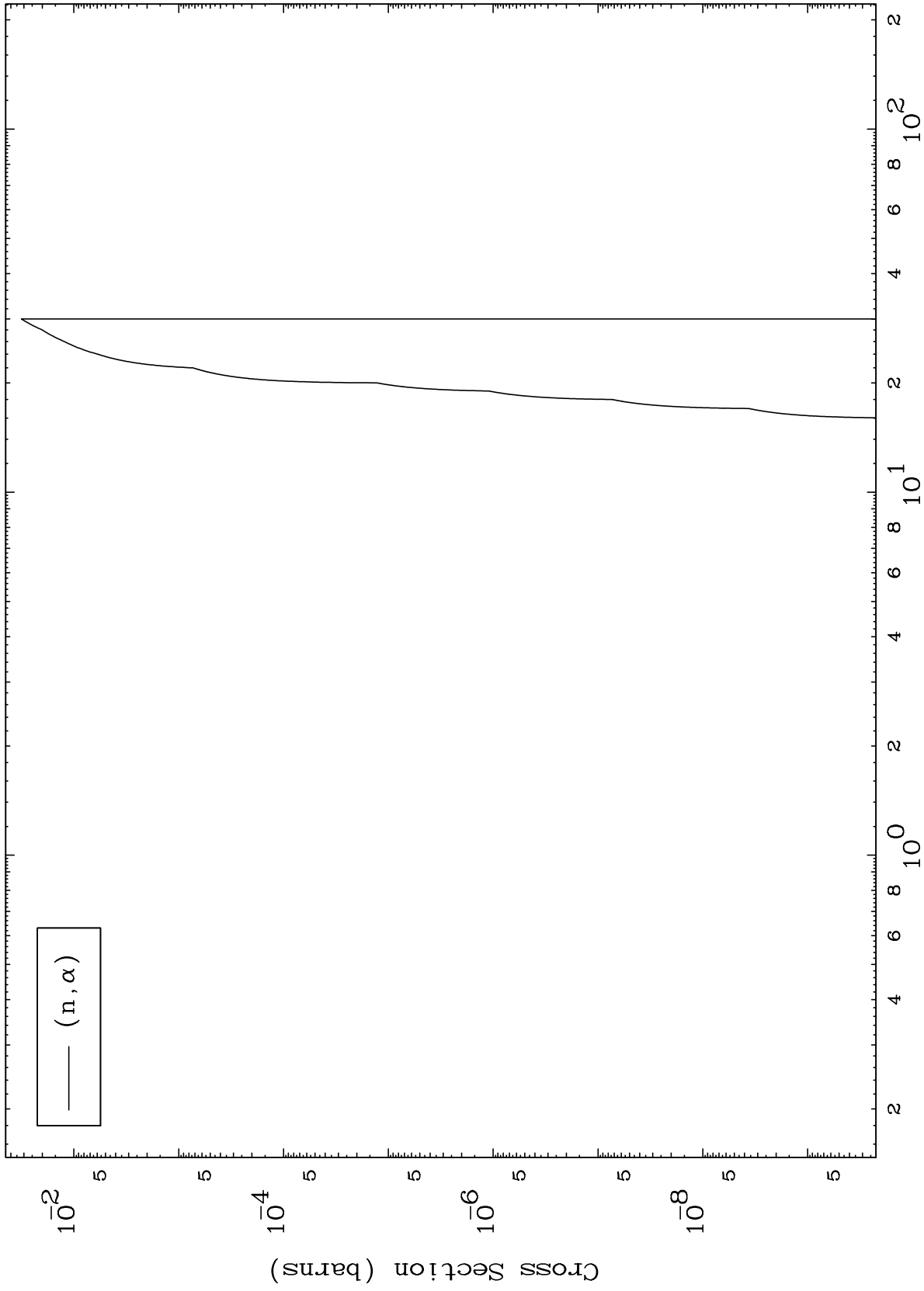
93-Np-236









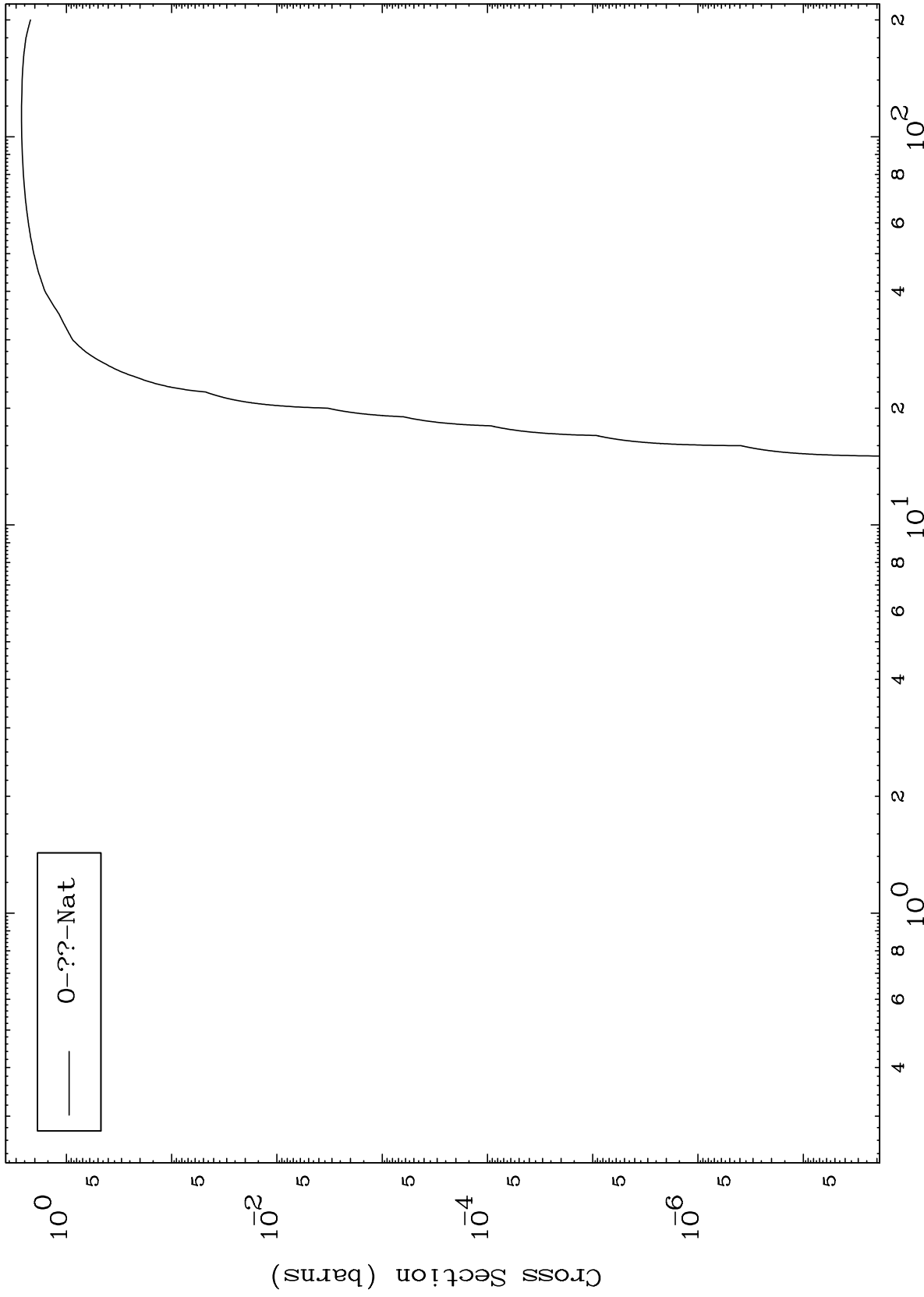


MAT 9343

Fission

93-Np-236

Radionuclide Production Cross Section



10

Incident Energy (MeV)

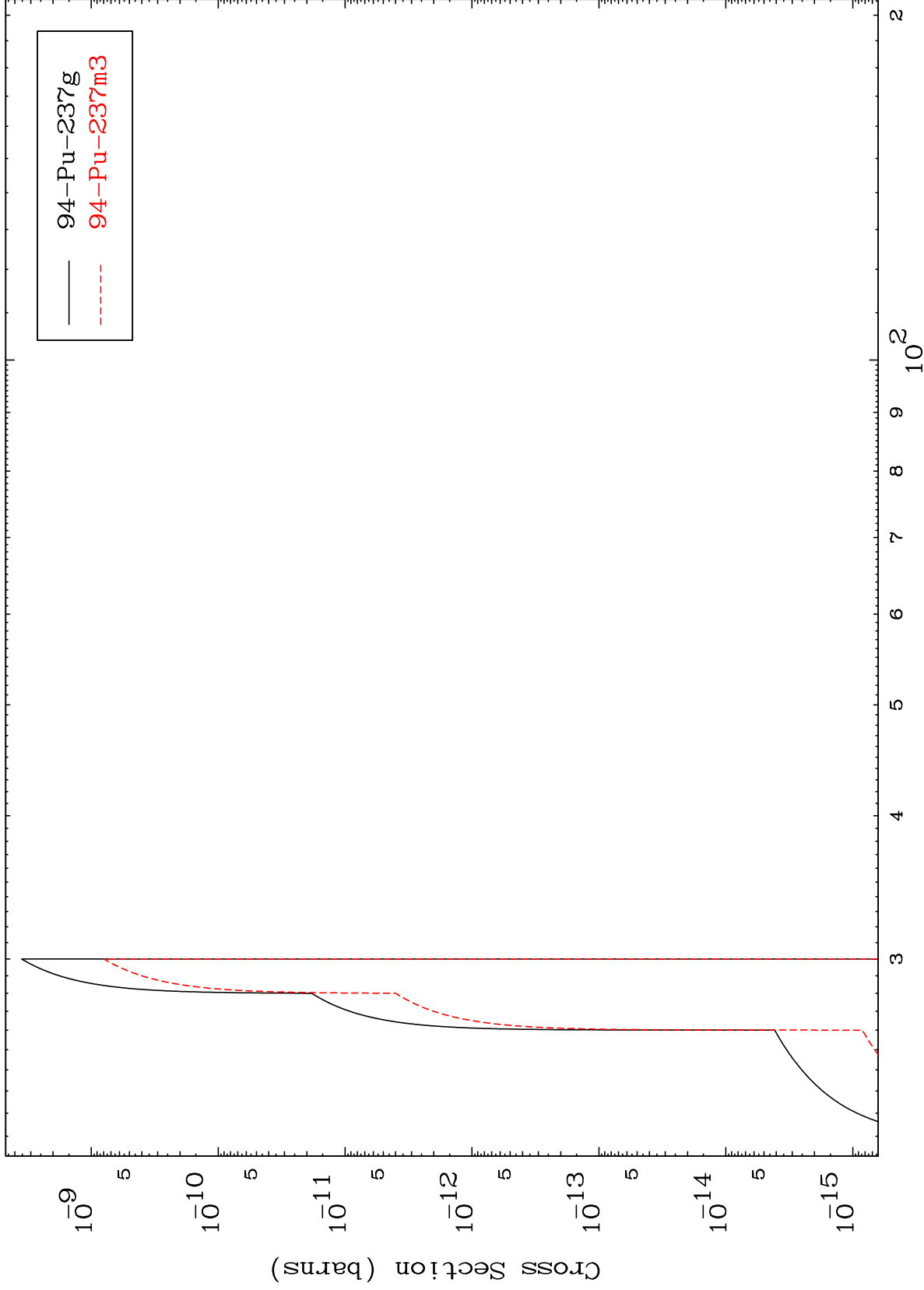
93-Np-236

MAT 9343

(n,n') d

93-Np-236

Radionuclide Production Cross Section



11

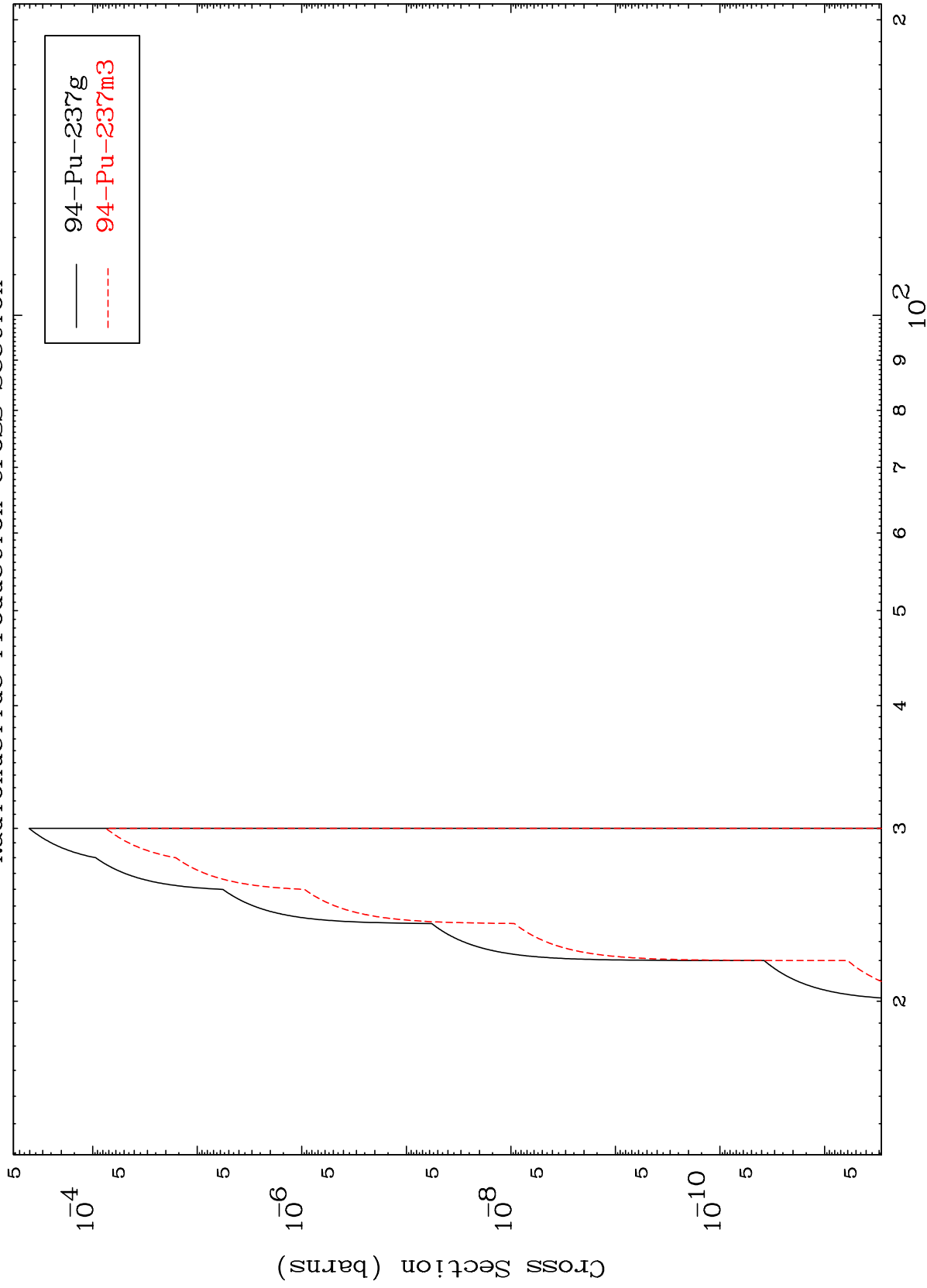
Incident Energy (MeV)

93-Np-236

MAT 9343

93-Np-236

(n, t)  
Radionuclide Production Cross Section



12

Incident Energy (MeV)

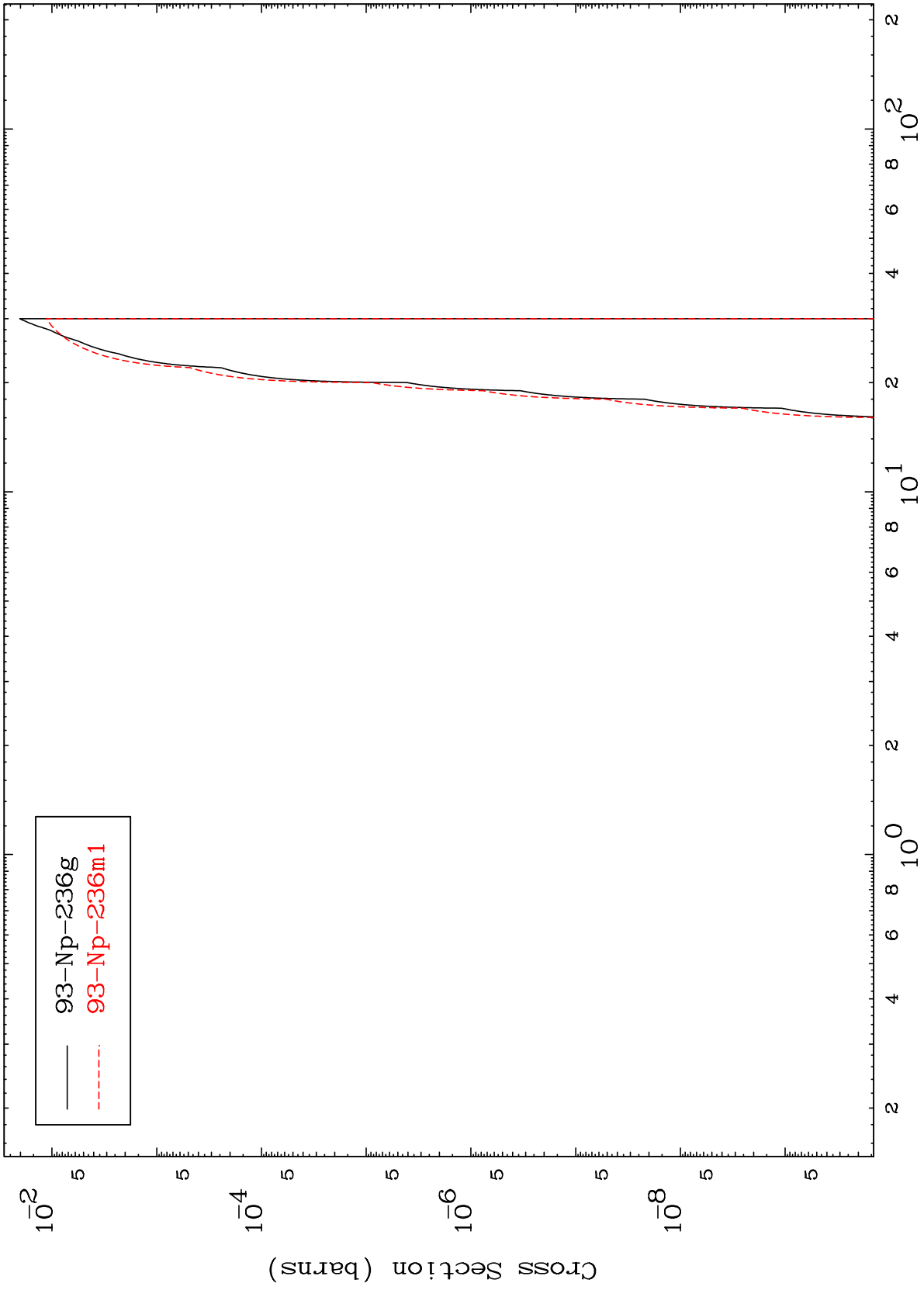
93-Np-236

MAT 9343

(n,  $\alpha$ )

93-Np-236

Radionuclide Production Cross Section



93-Np-236g  
93-Np-236m1