

Program EVALPLOT
(Version 2021-1)

by

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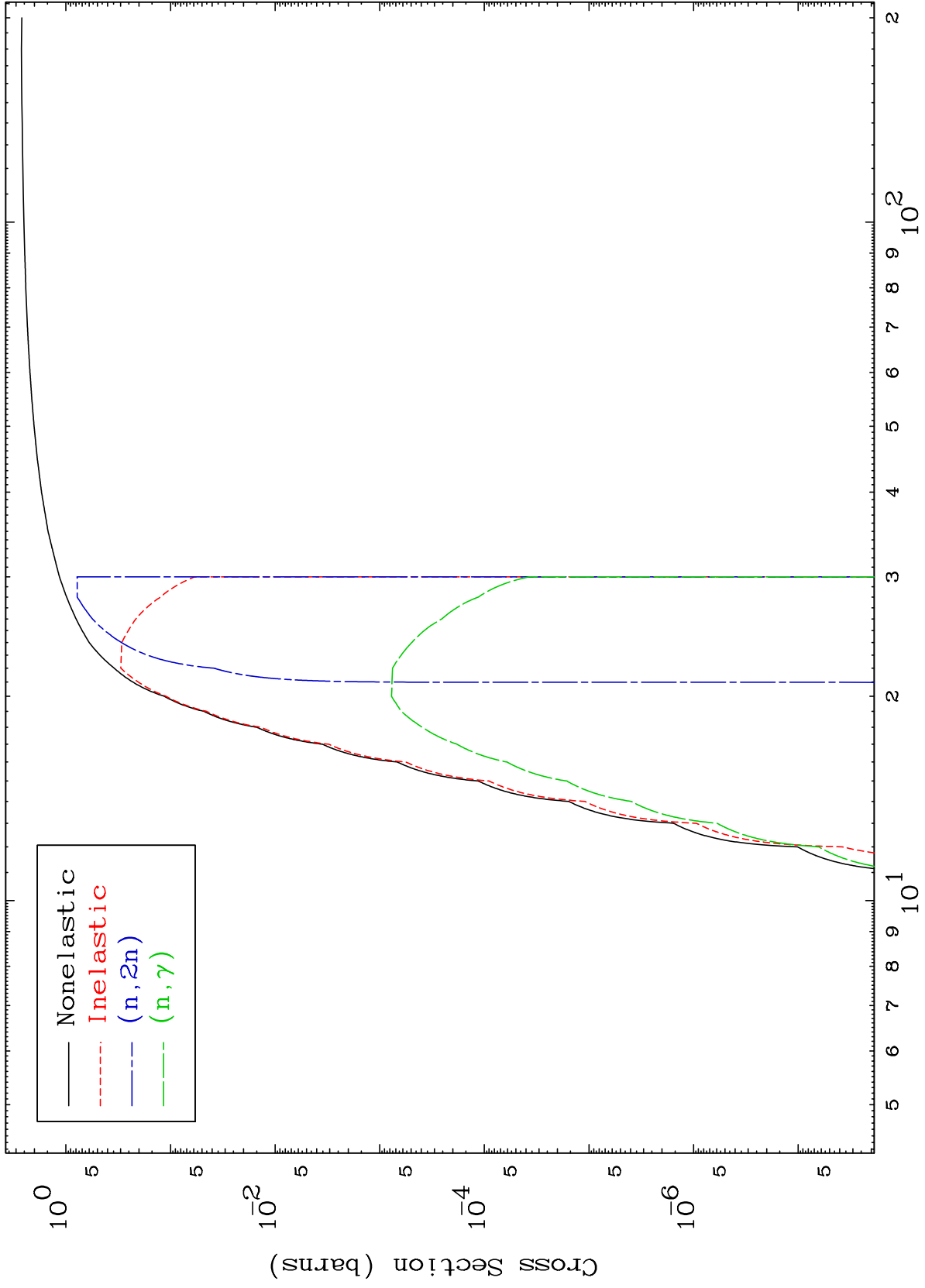
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7623

0 Kelvin Cross Sections

76-0s-183m



1

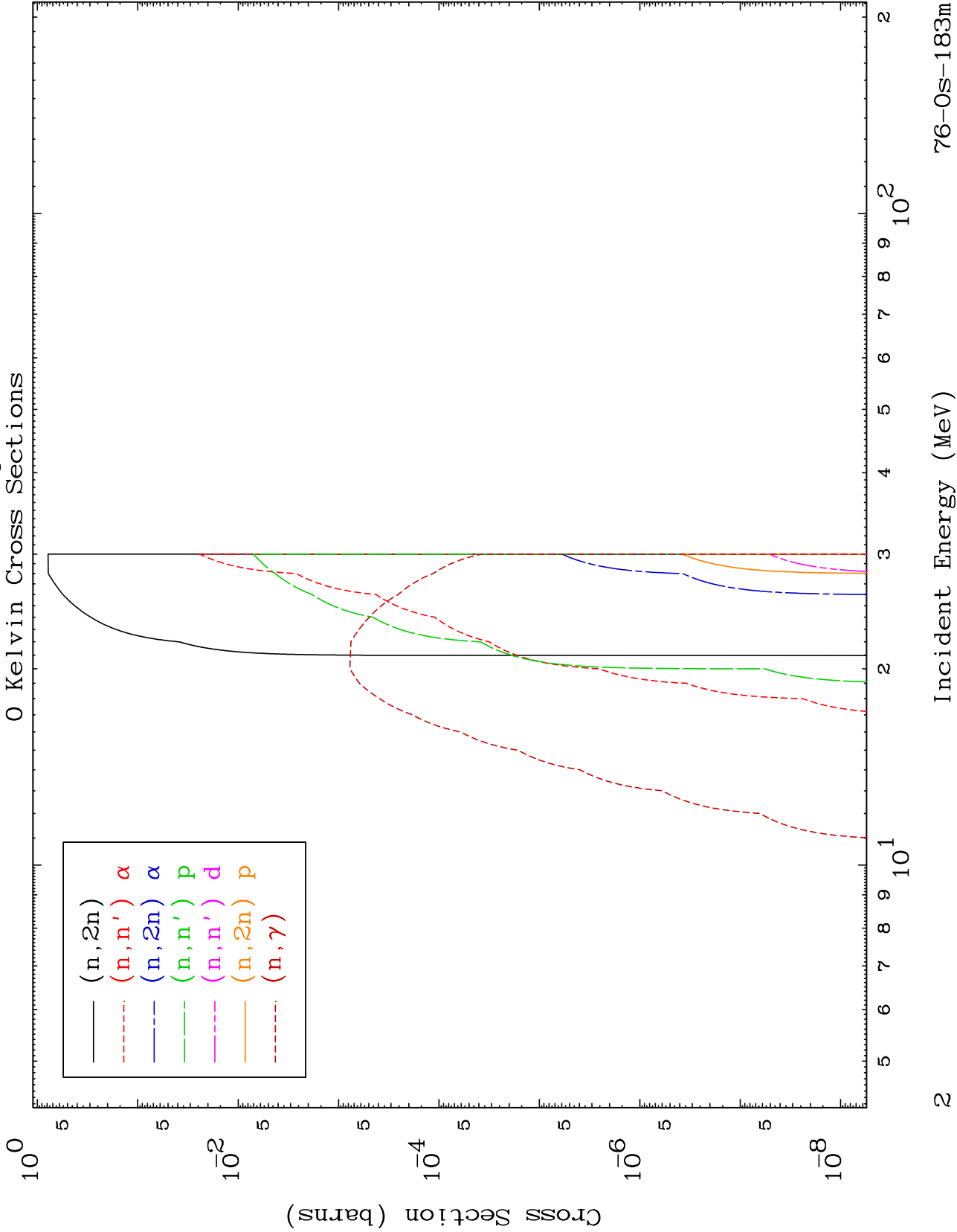
Incident Energy (MeV)

76-0s-183m

MAT 7623

α Neutron Absorption
0 Kelvin Cross Sections

76-0s-183m



76-0s-183m

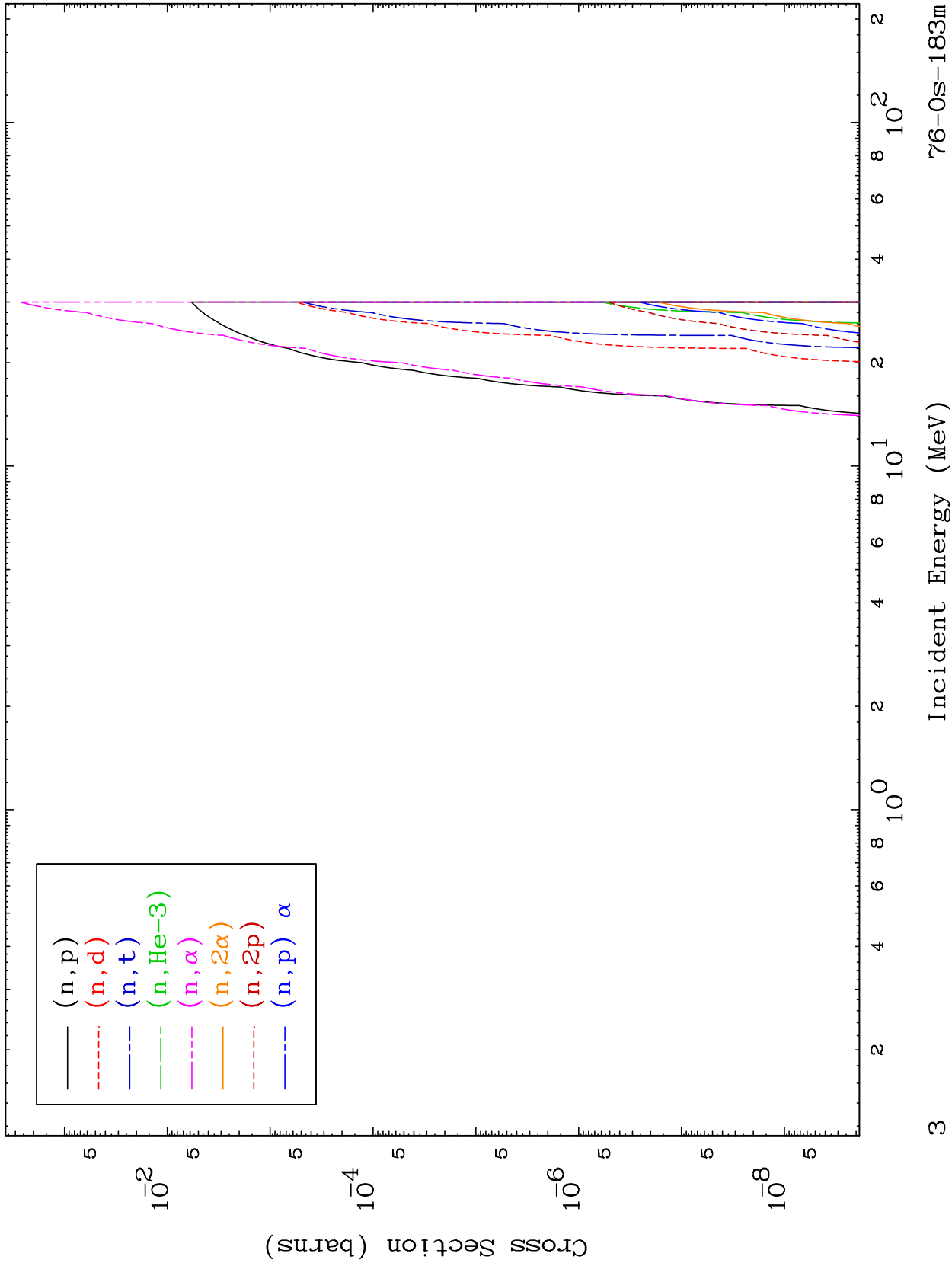
Incident Energy (MeV)

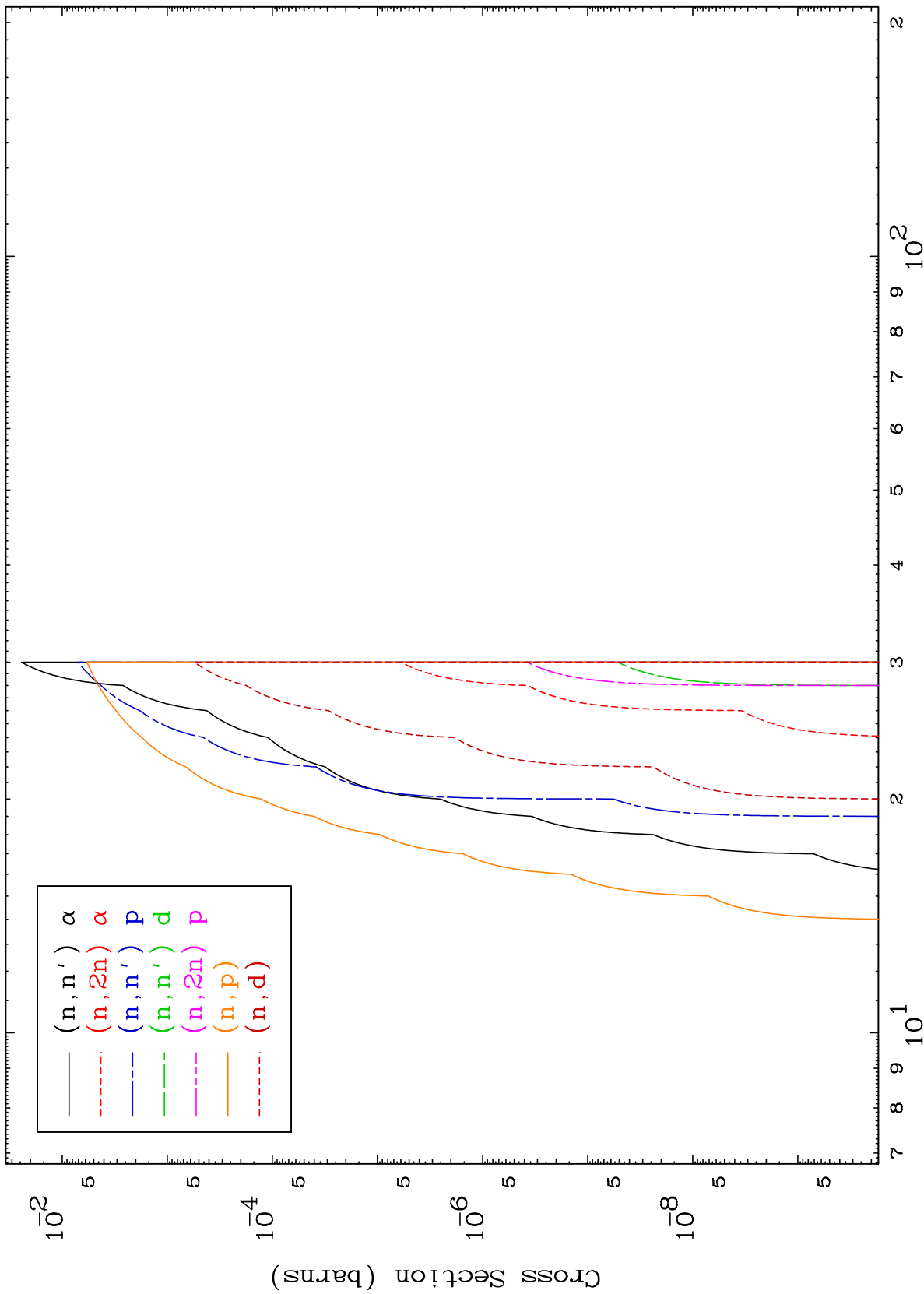
2

MAT 7623

α Neutron Absorption
0 Kelvin Cross Sections

$^{76}\text{Os}-183\text{m}$



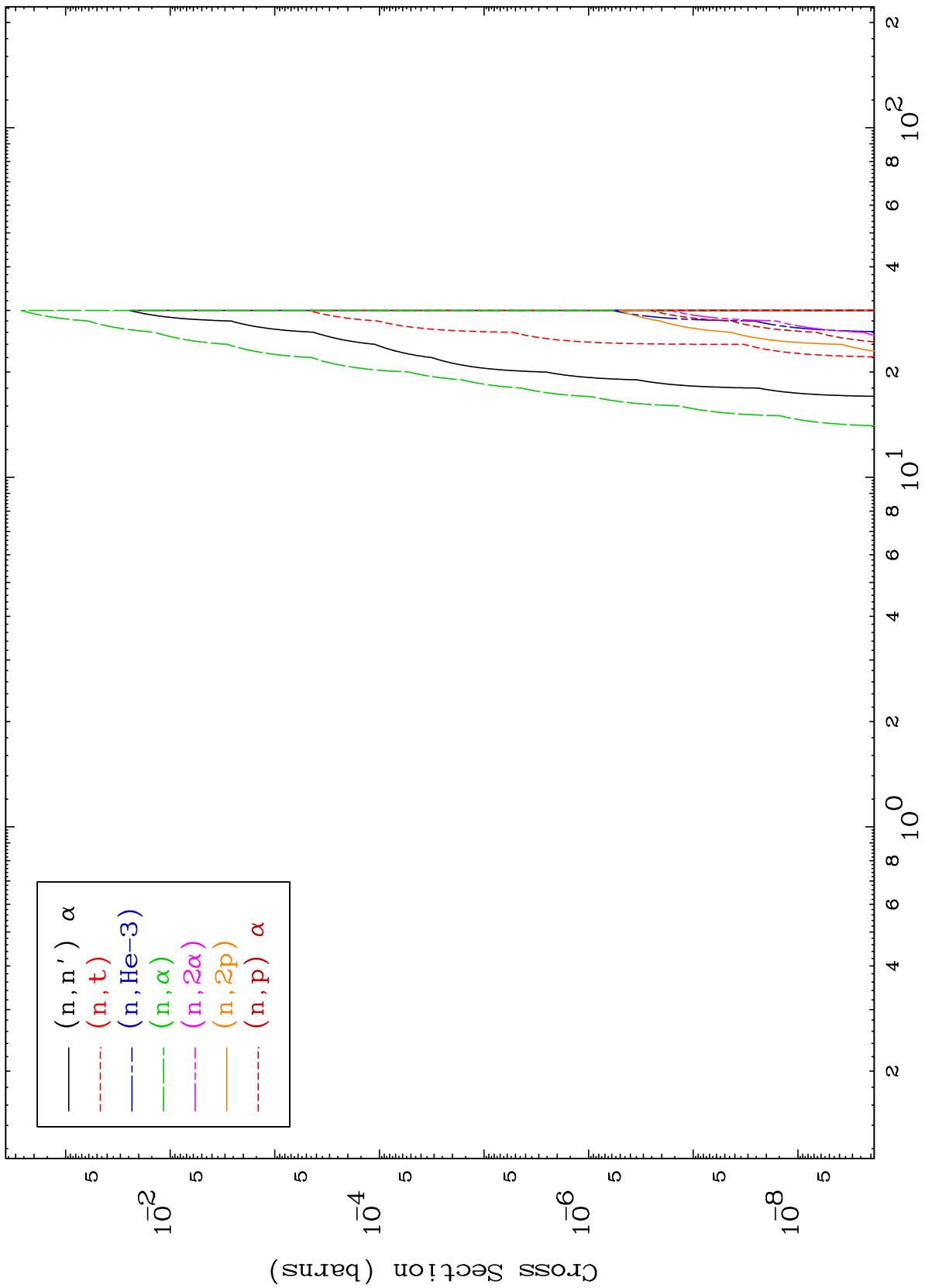


MAT 7623

α Charged Particle

76-0s-183m

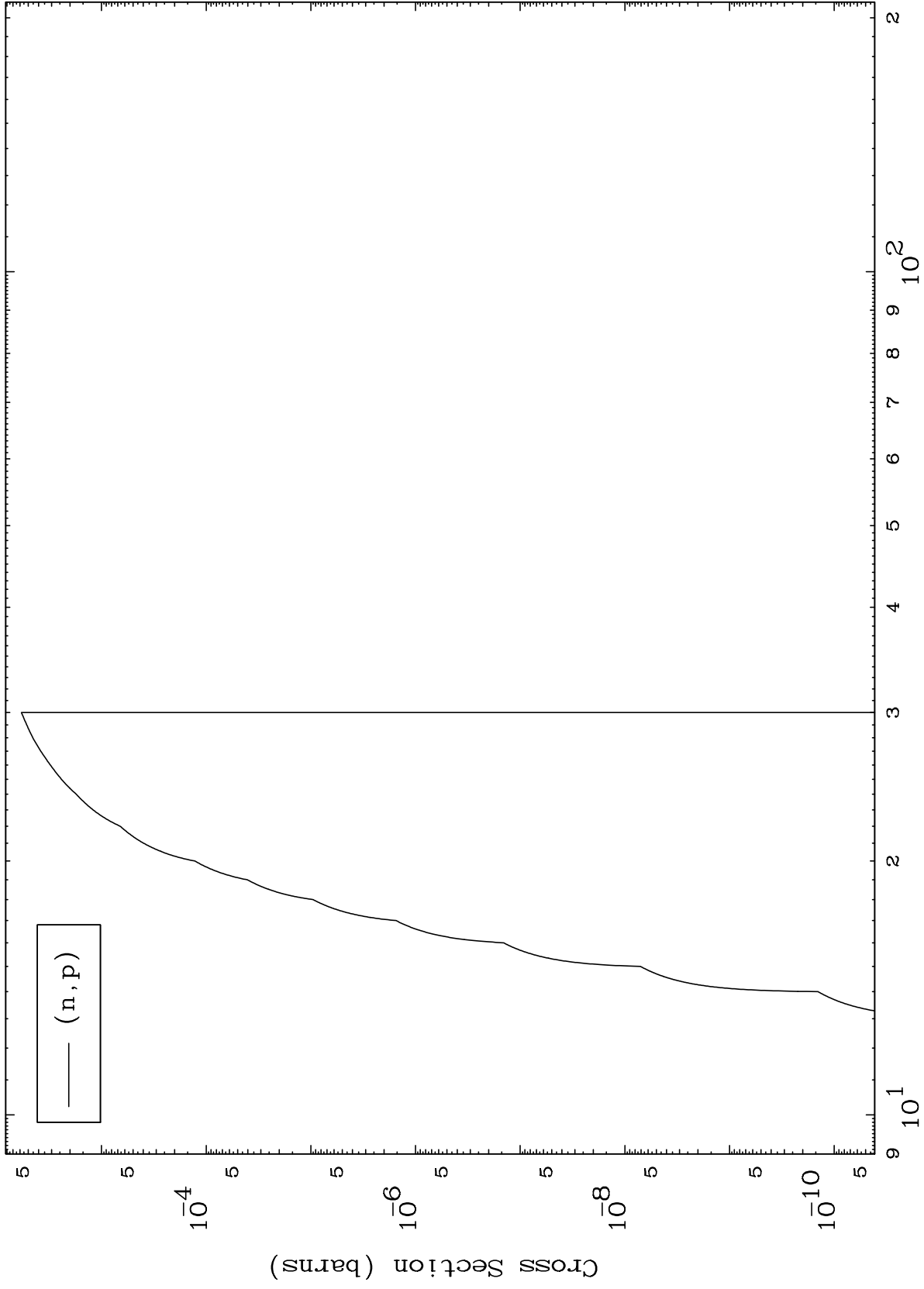
0 Kelvin Cross Sections



MAT 7623

(α, p) Levels
0 Kelvin Cross Sections

76-0s-183m



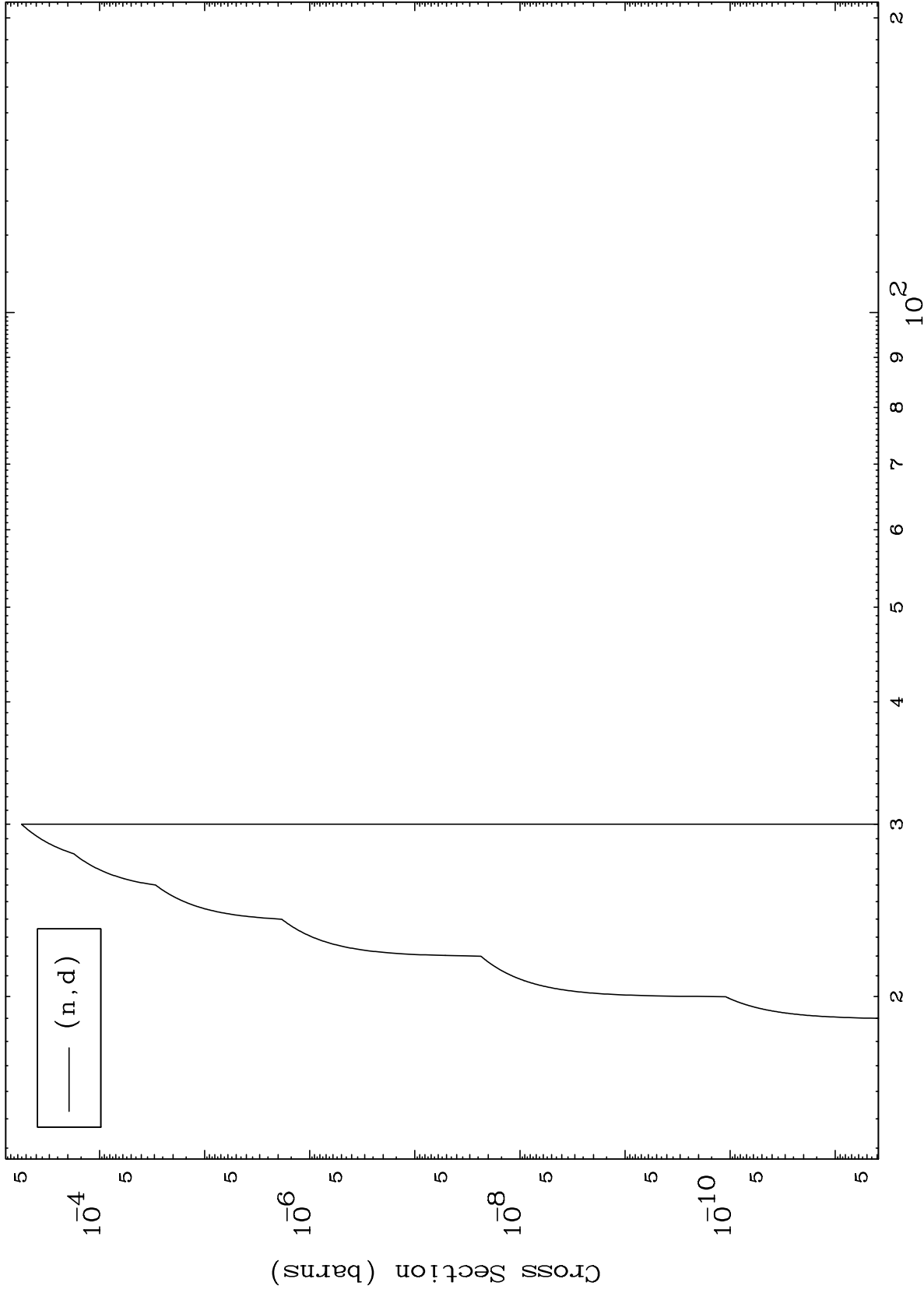
Incident Energy (MeV)

76-0s-183m

MAT 7623

(α, d) Levels
0 Kelvin Cross Sections

76-0s-183m



7

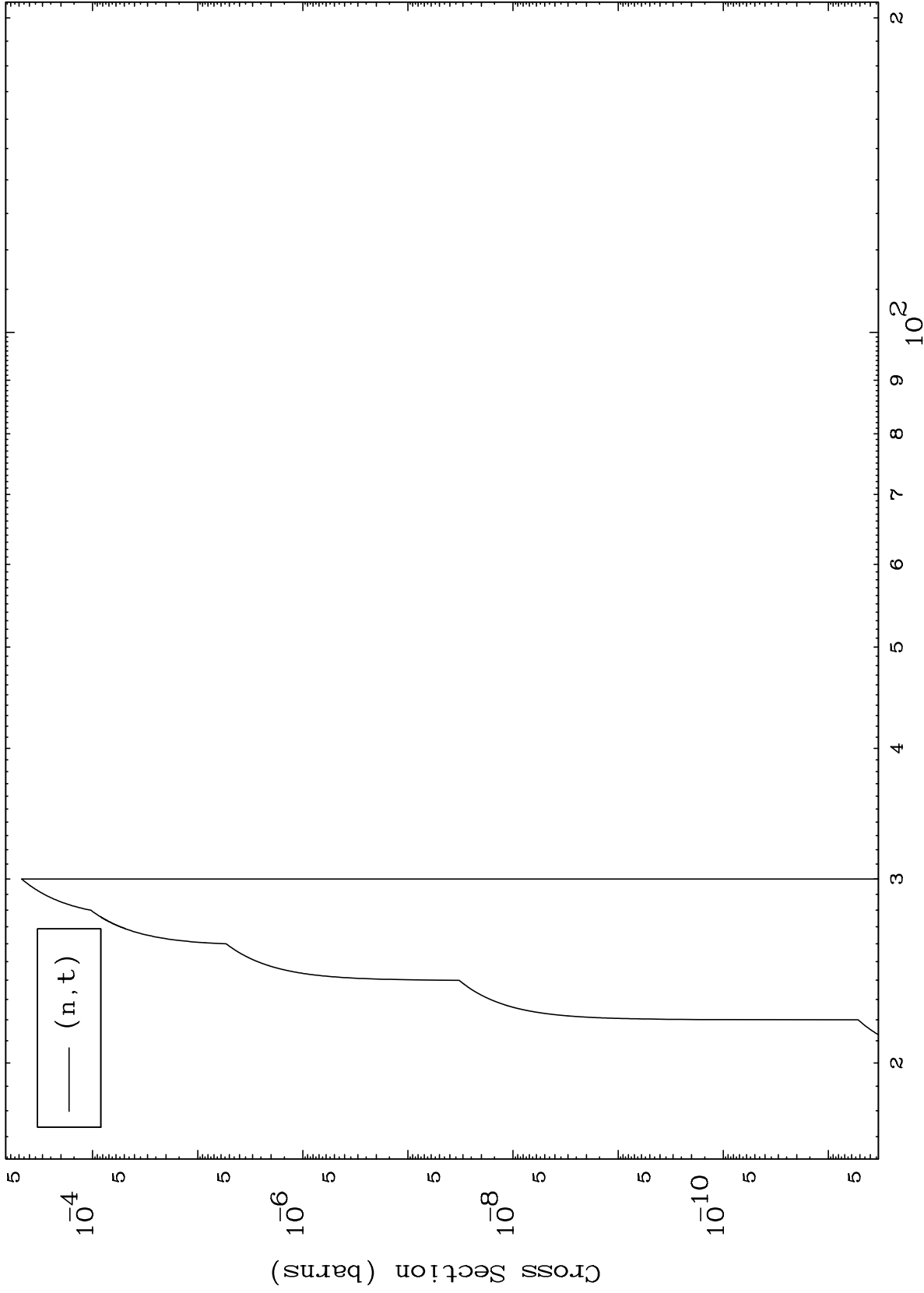
Incident Energy (MeV)

76-0s-183m

MAT 7623

(α, t) Levels
0 Kelvin Cross Sections

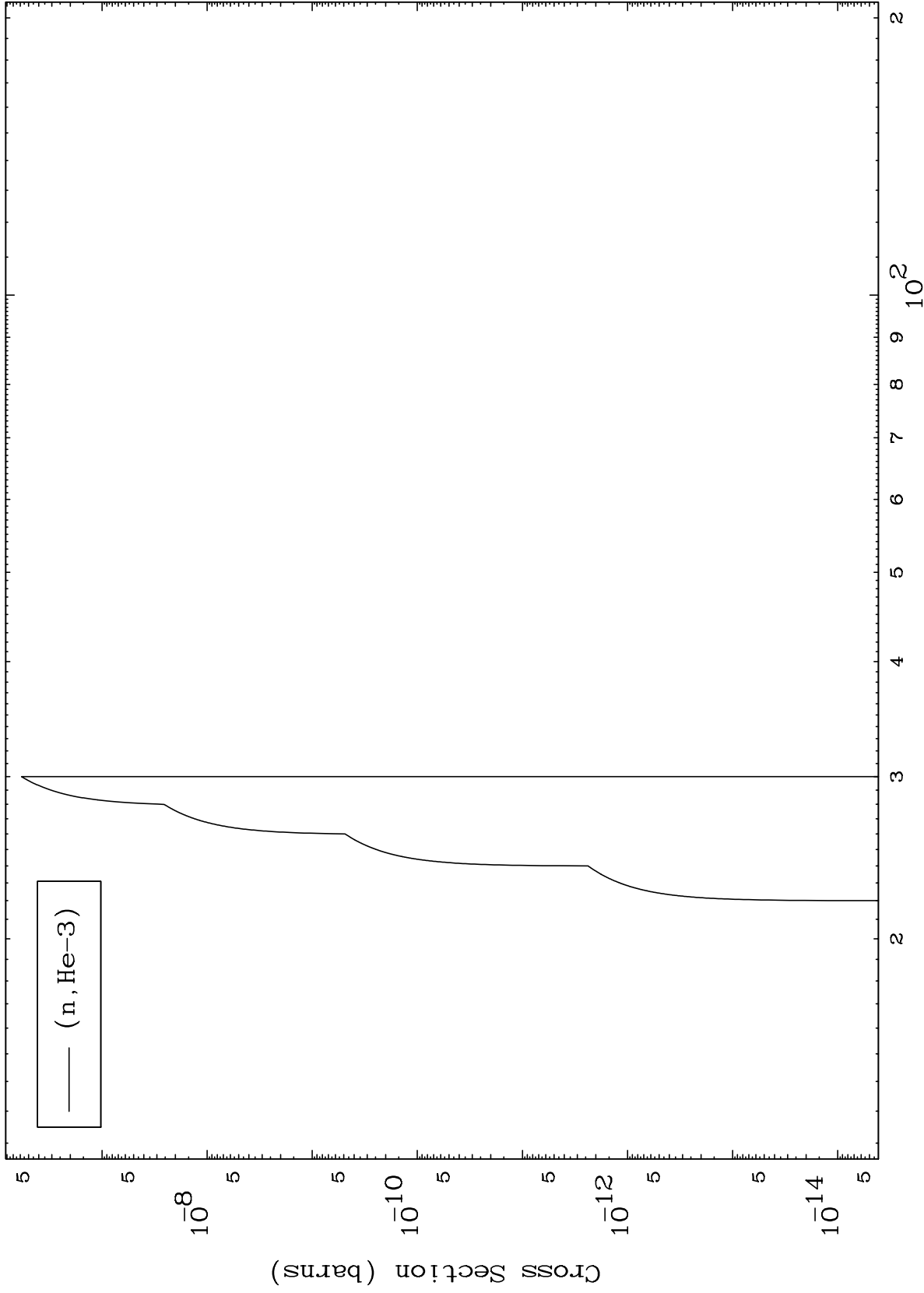
76-0s-183m



8

Incident Energy (MeV)

76-0s-183m

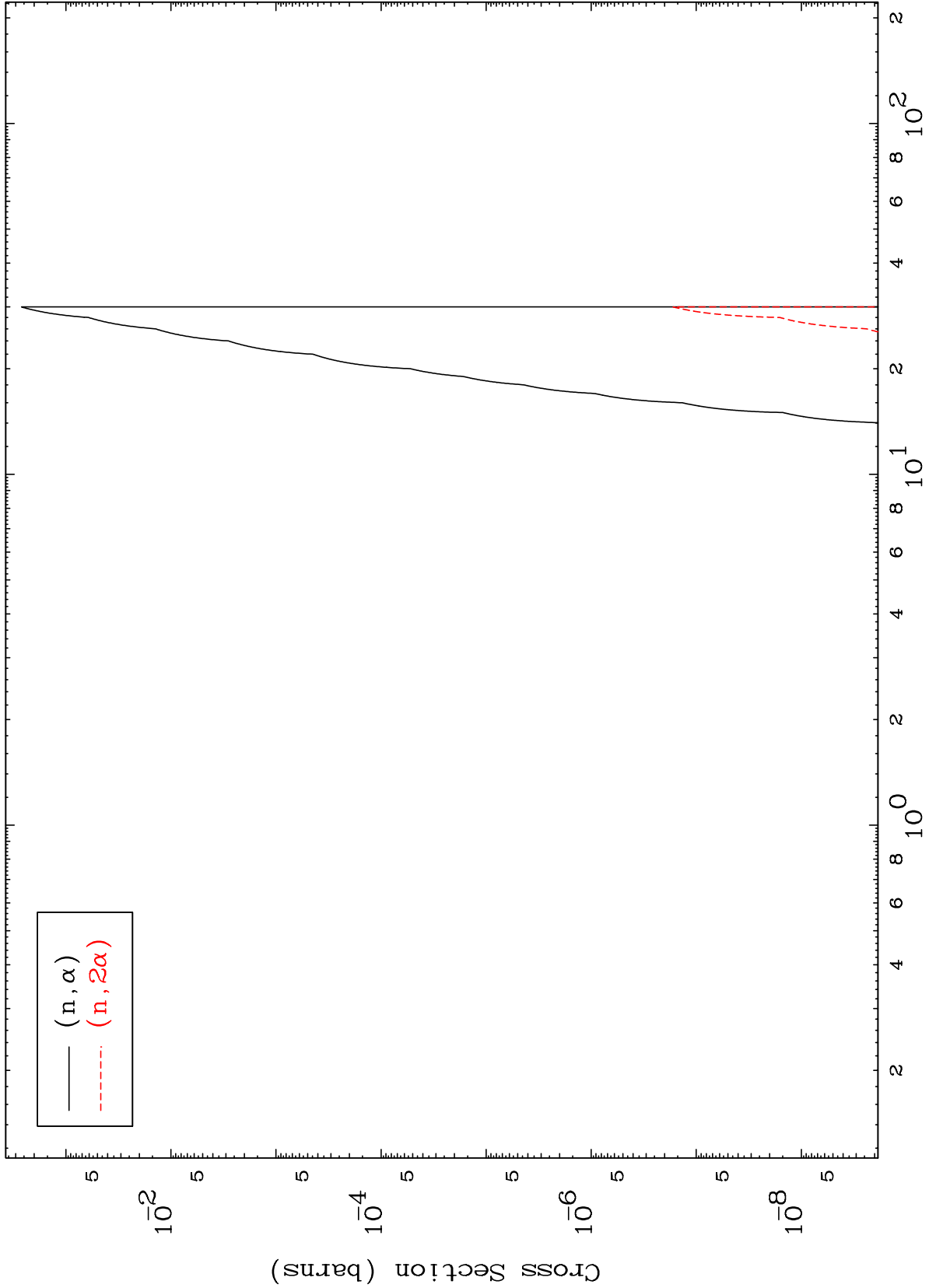


MAT 7623

(α, α) Levels

76-0s-183m

0 Kelvin Cross Sections



— (n, α)
- - - ($n, 2\alpha$)

10

Incident Energy (MeV)

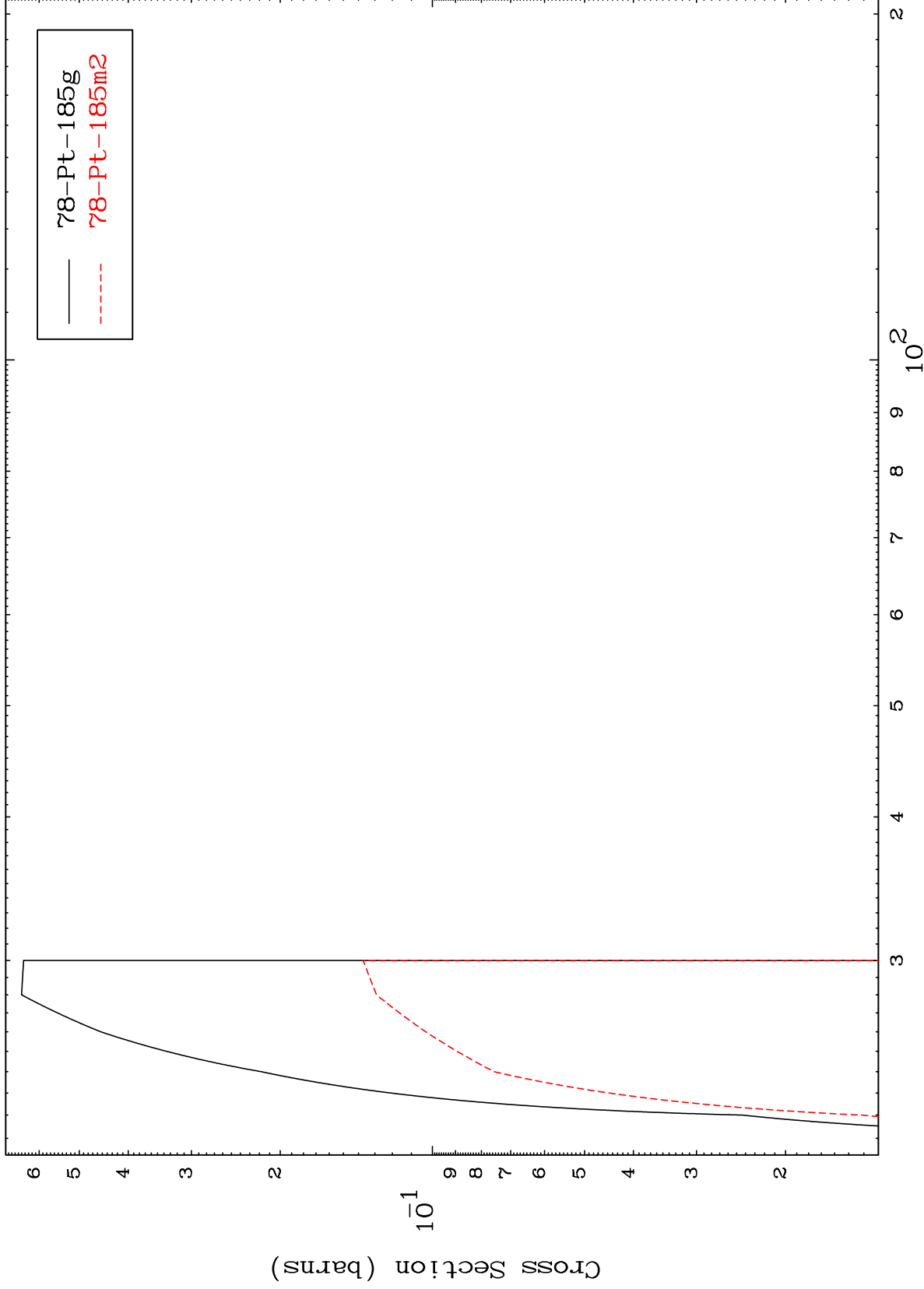
76-0s-183m

MAT 7623

(n,2n)

76-Os-183m

Radionuclide Production Cross Section



11

Incident Energy (MeV)

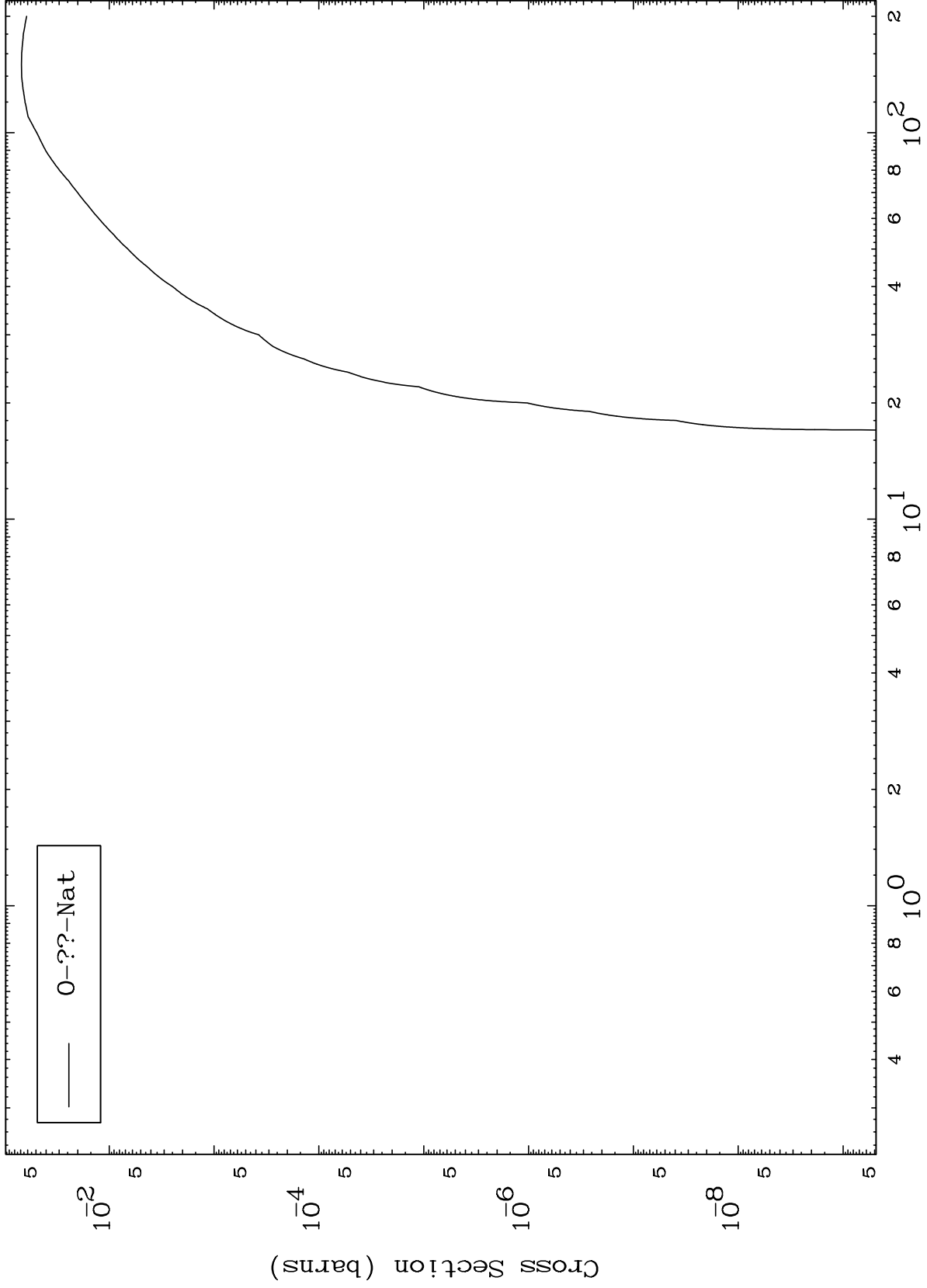
76-Os-183m

MAT 7623

Fission

76-0s-183m

Radionuclide Production Cross Section

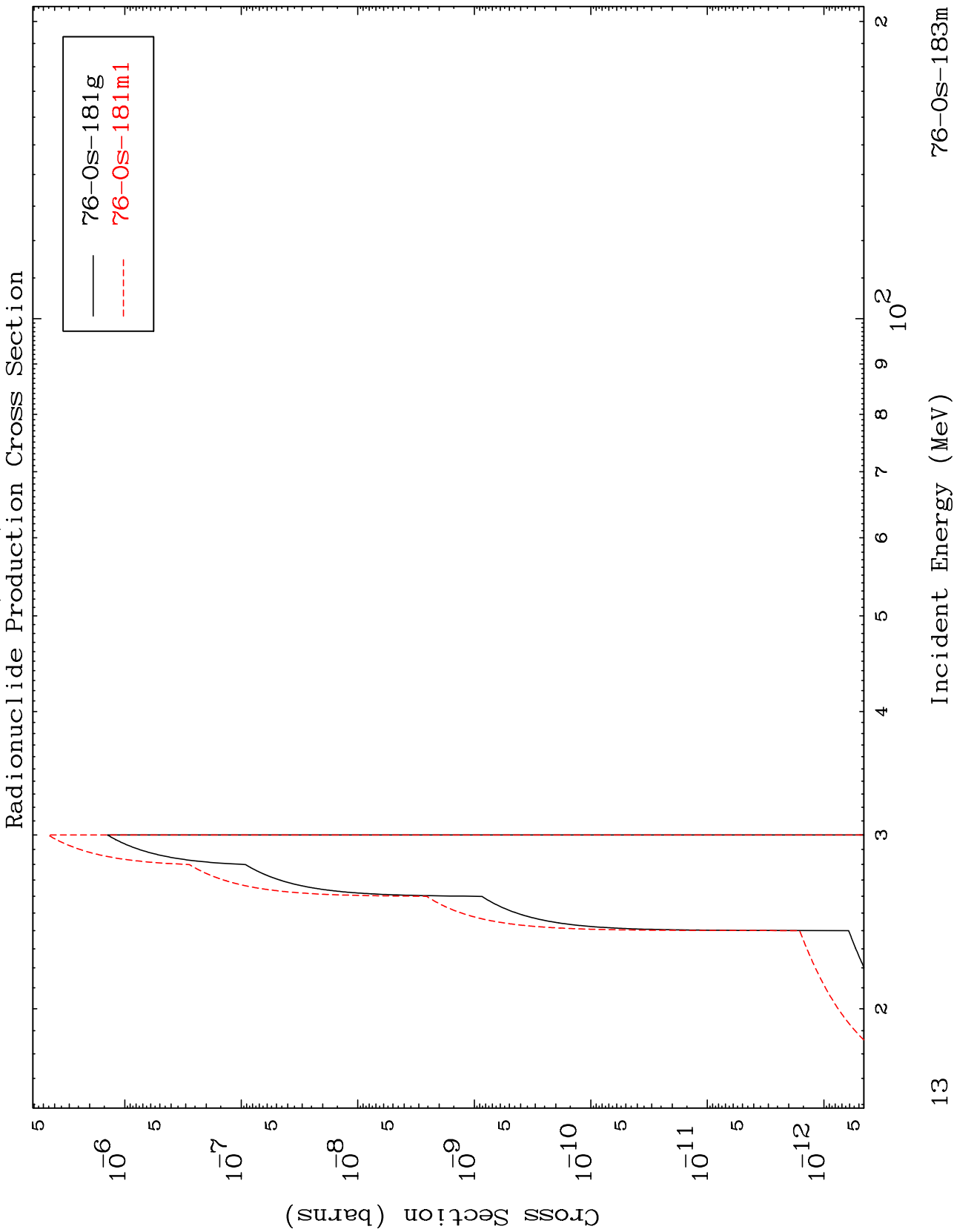


0-??-Nat

MAT 7623

$(n,2n) \alpha$

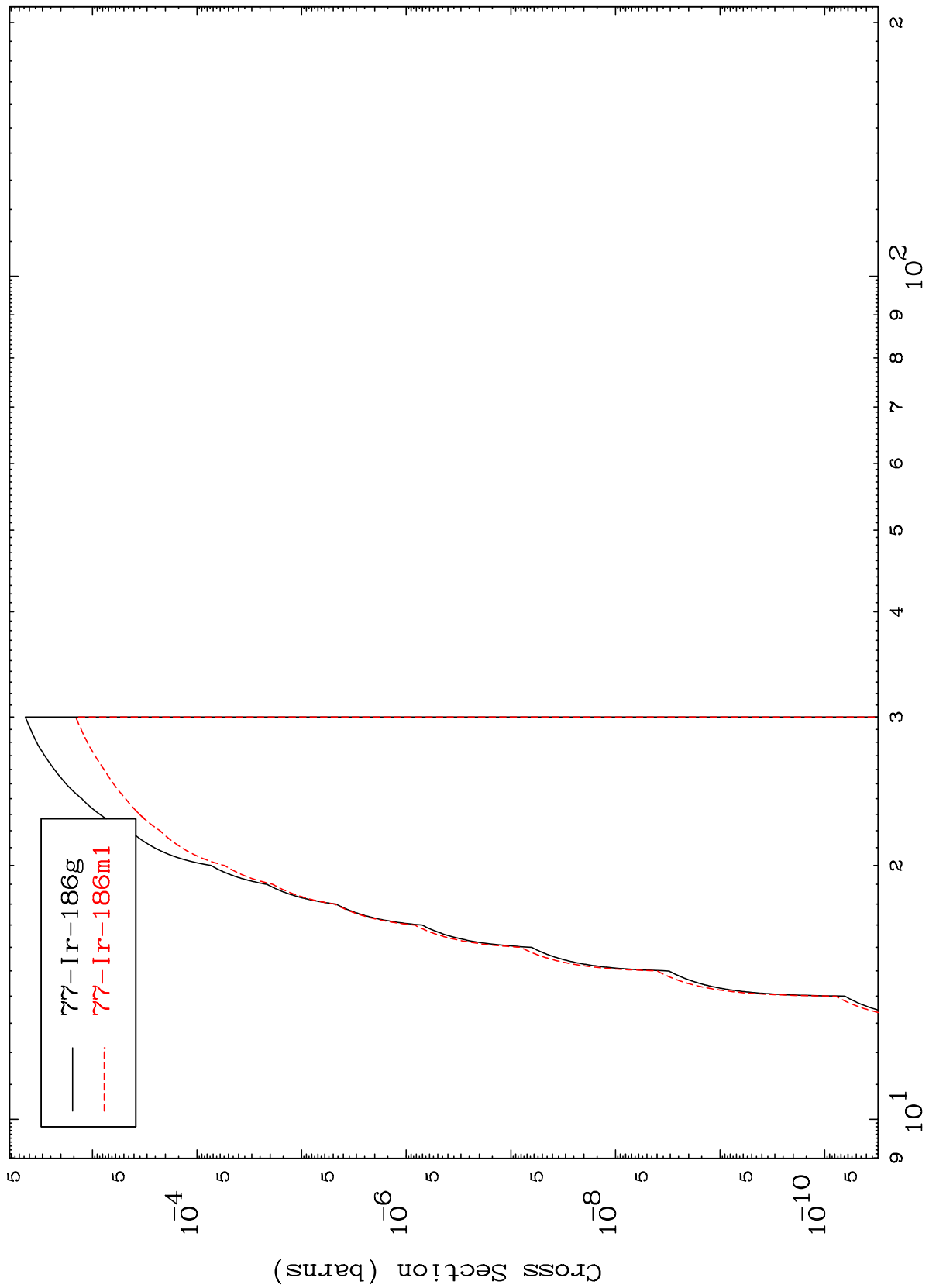
76-0s-183m



MAT 7623

76-0s-183m

(n,p)
Radionuclide Production Cross Section



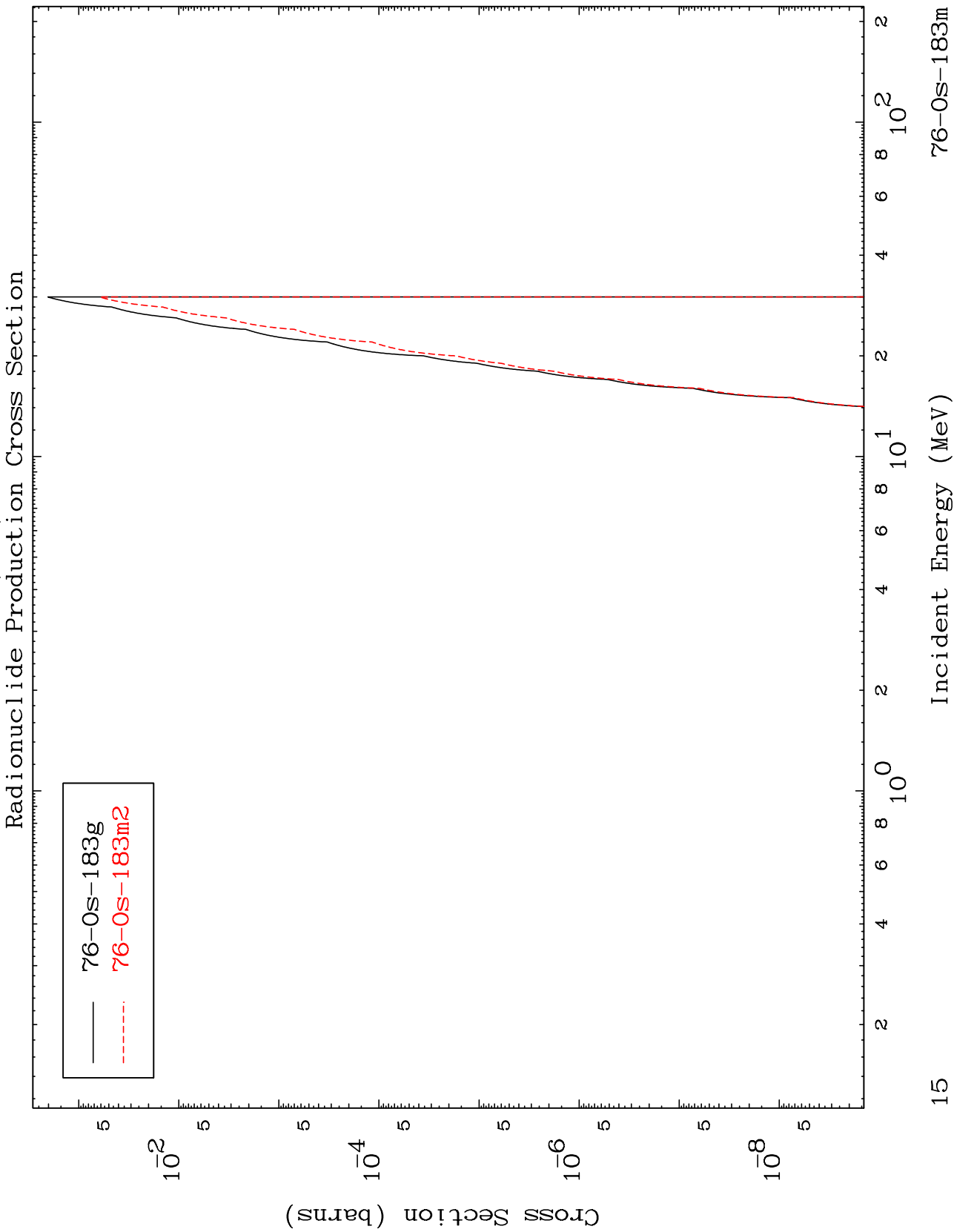
76-0s-183m

Incident Energy (MeV)

14

MAT 7623

76-0s-183m



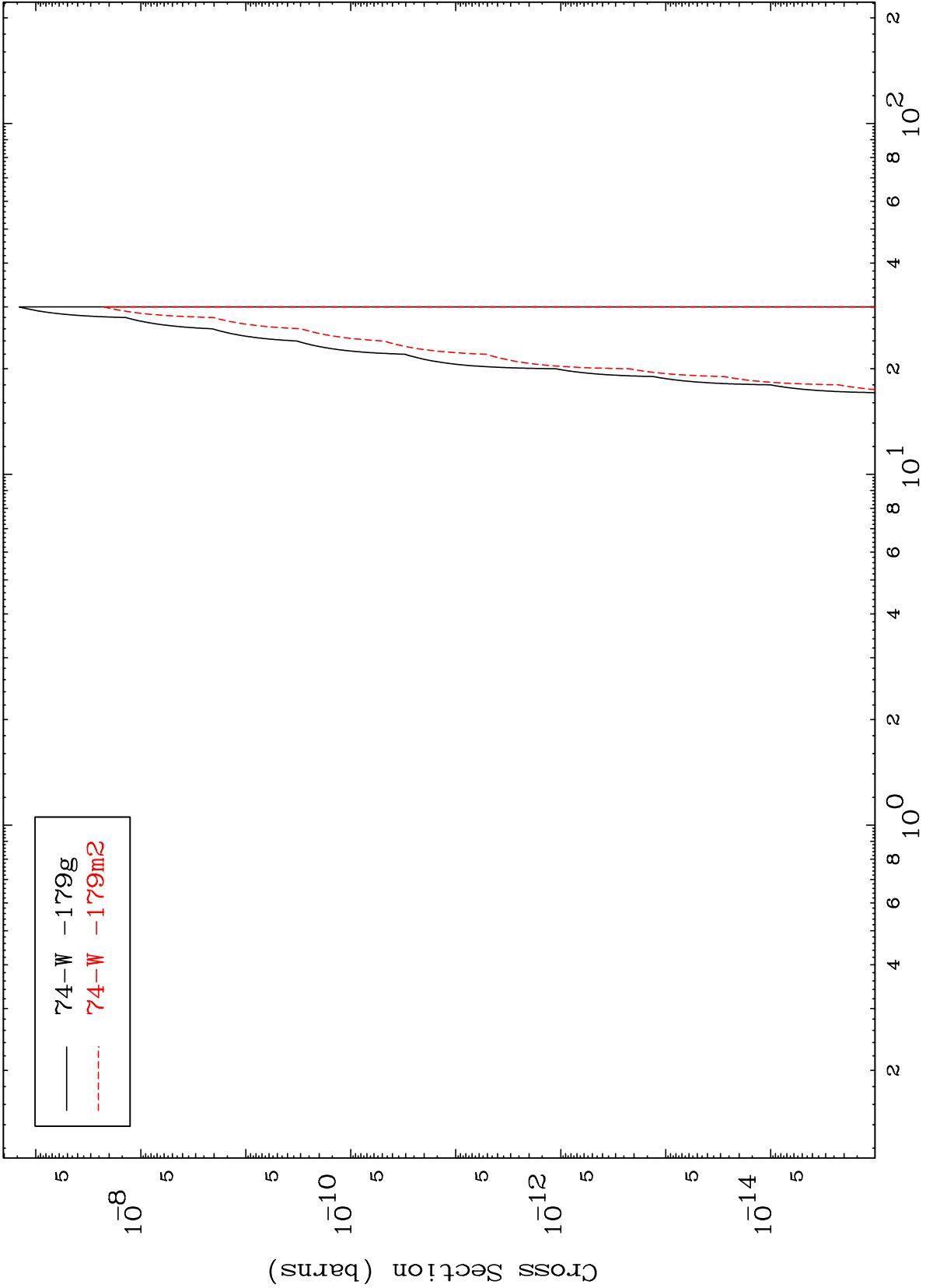
76-0s-183g
76-0s-183m2

MAT 7623

(n,2α)

76-0s-183m

Radionuclide Production Cross Section



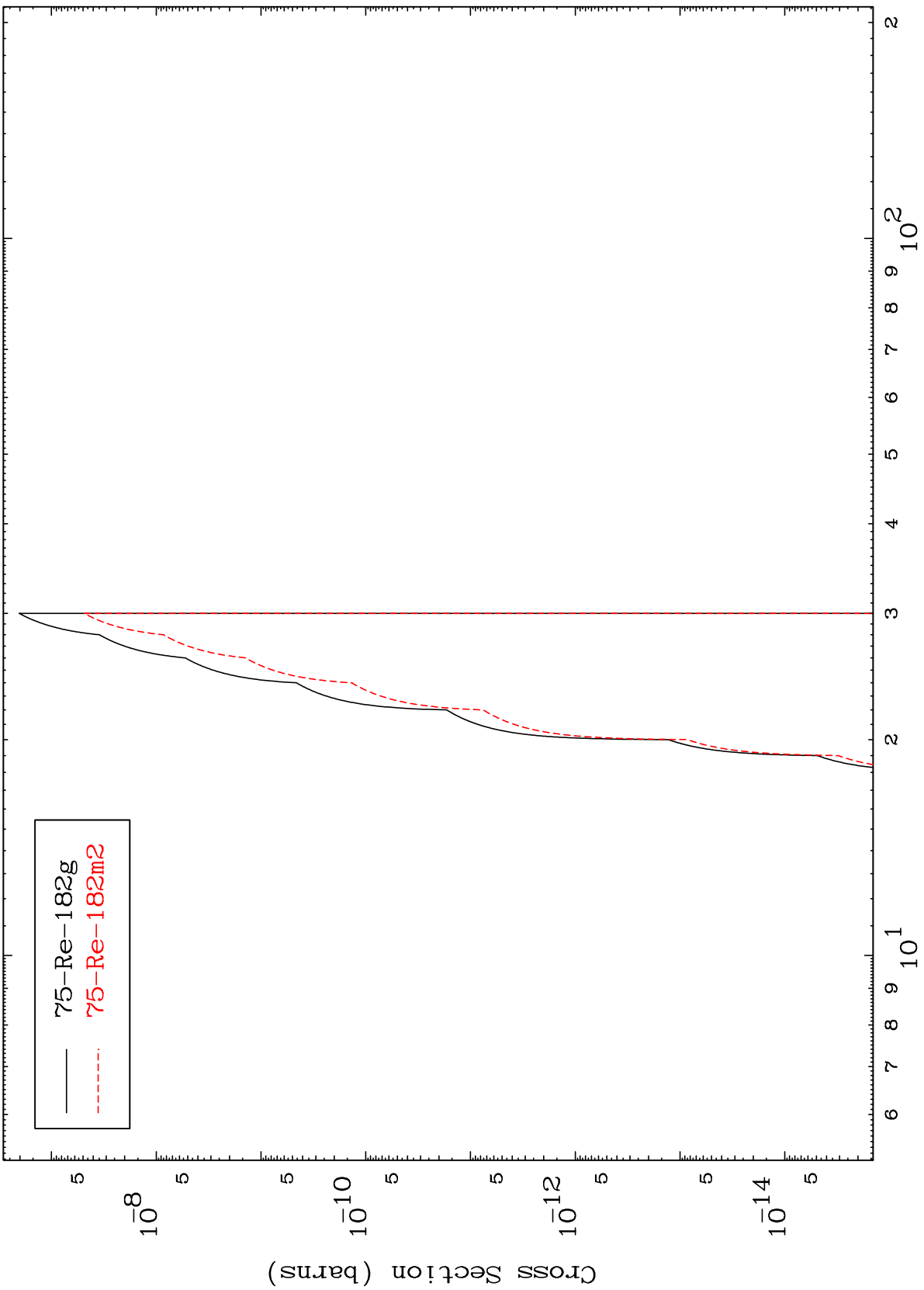
74-W -179g
74-W -179m2

MAT 7623

(n,p) α

⁷⁶Os-183m

Radionuclide Production Cross Section



75-Re-182g
75-Re-182m2

17

Incident Energy (MeV)

⁷⁶Os-183m