

Program EVALPLOT  
(Version 2021-1)

by

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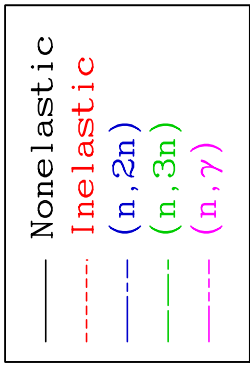
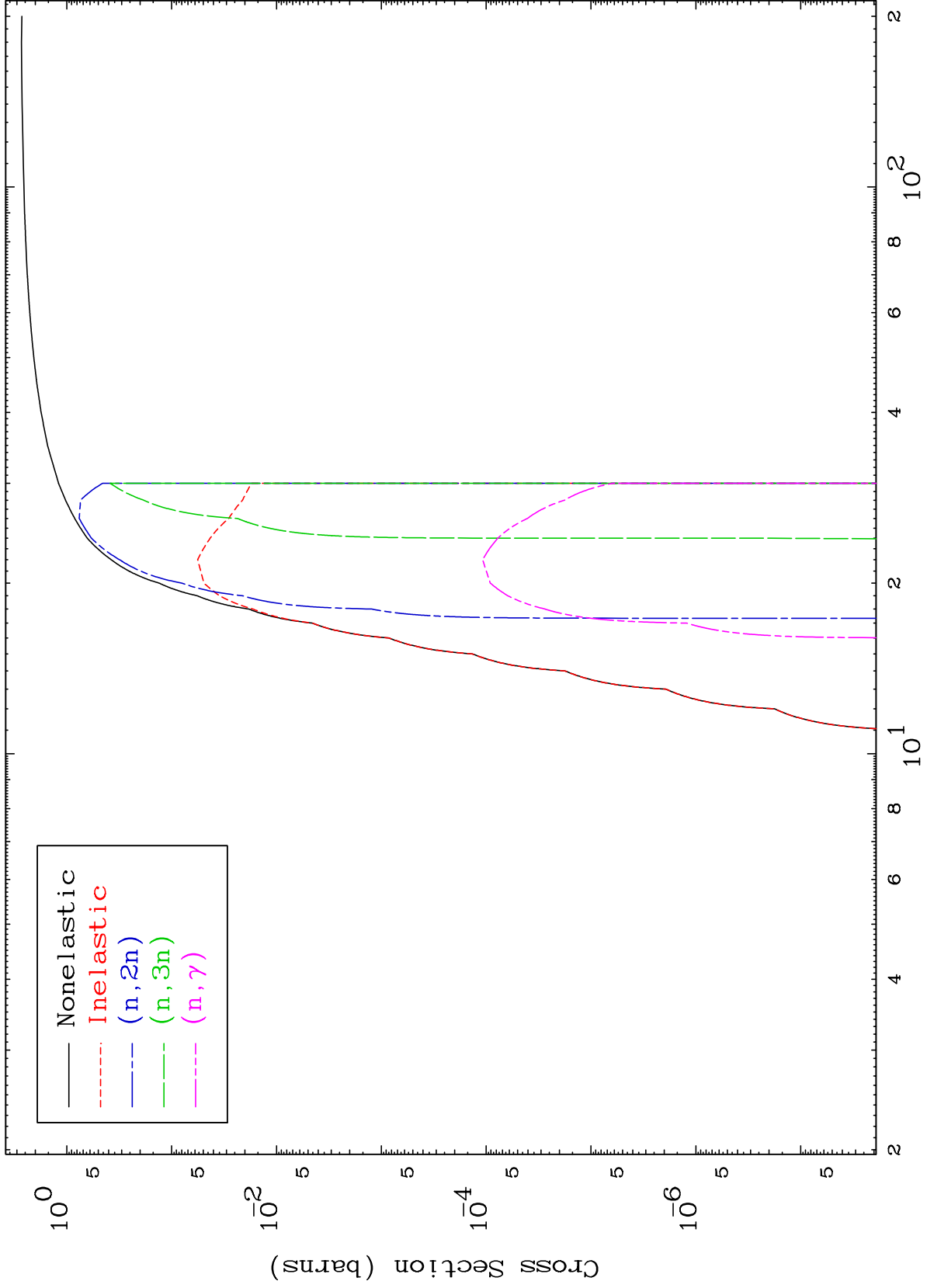
Press Mouse Button to Start

MAT 7641

0 Kelvin

$\alpha$  Major Cross Sections

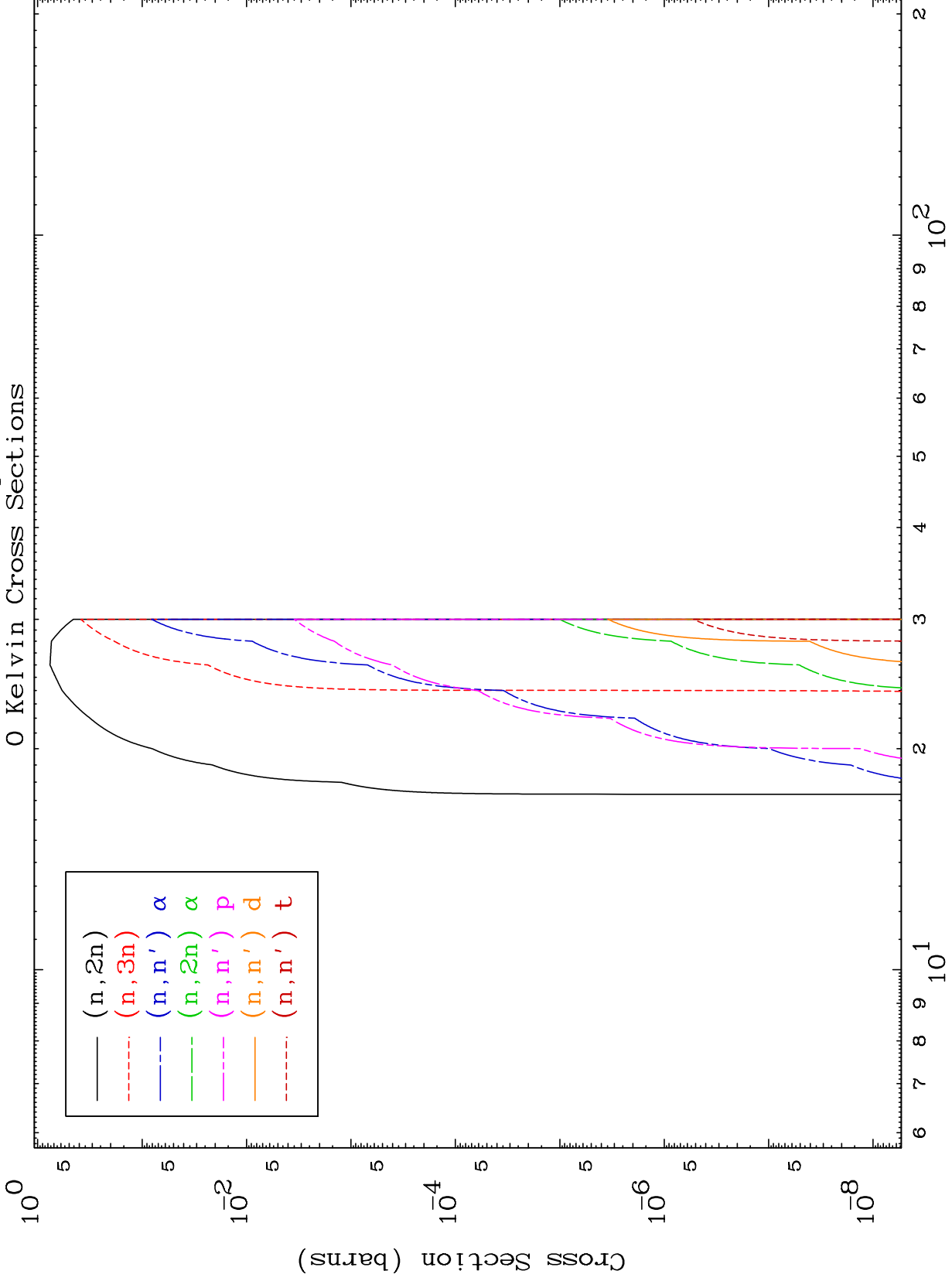
76-0s-189m



MAT 7641

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

76-0s-189m



2

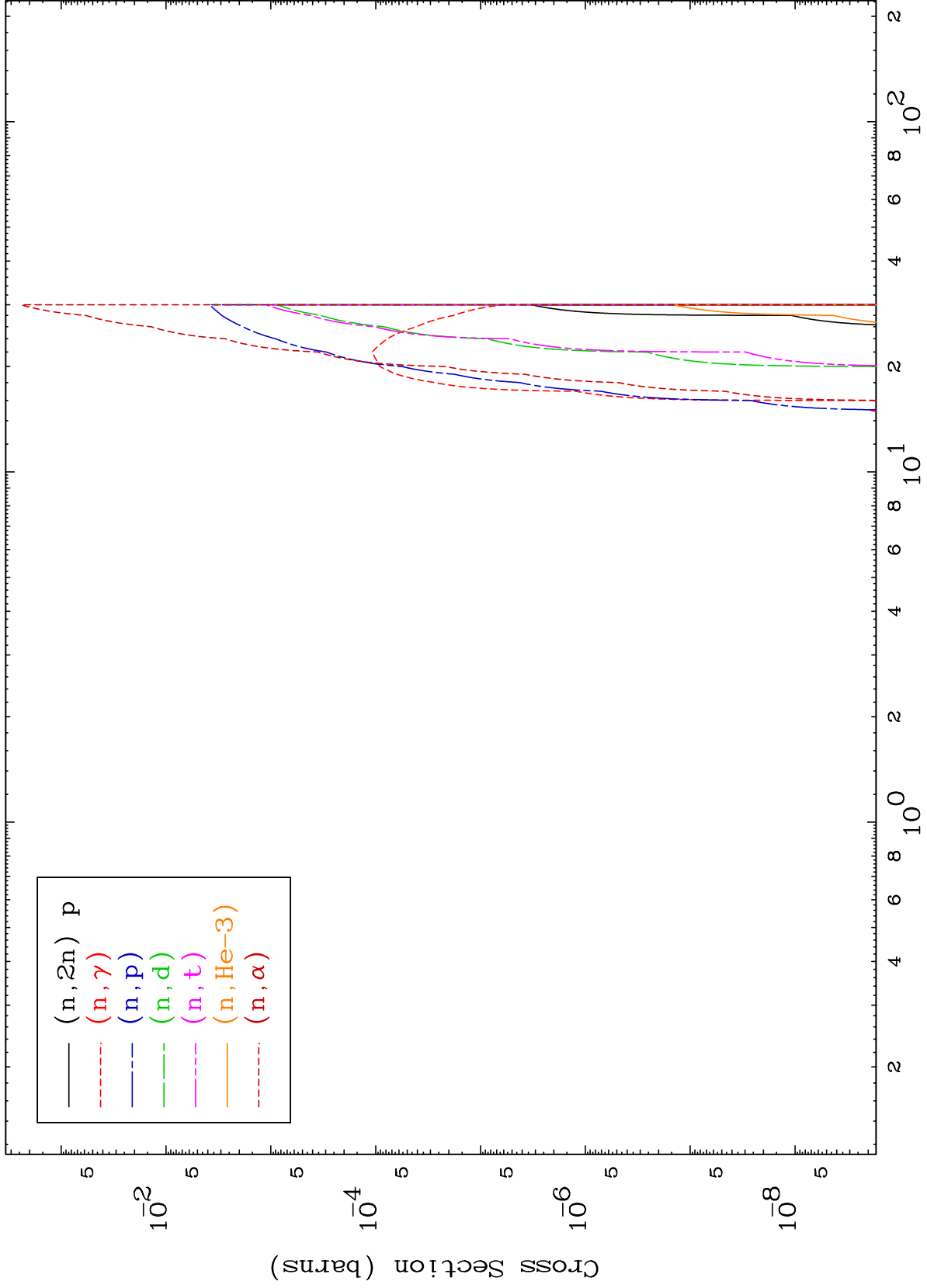
Incident Energy (MeV)

76-0s-189m

MAT 7641

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

76-0s-189m

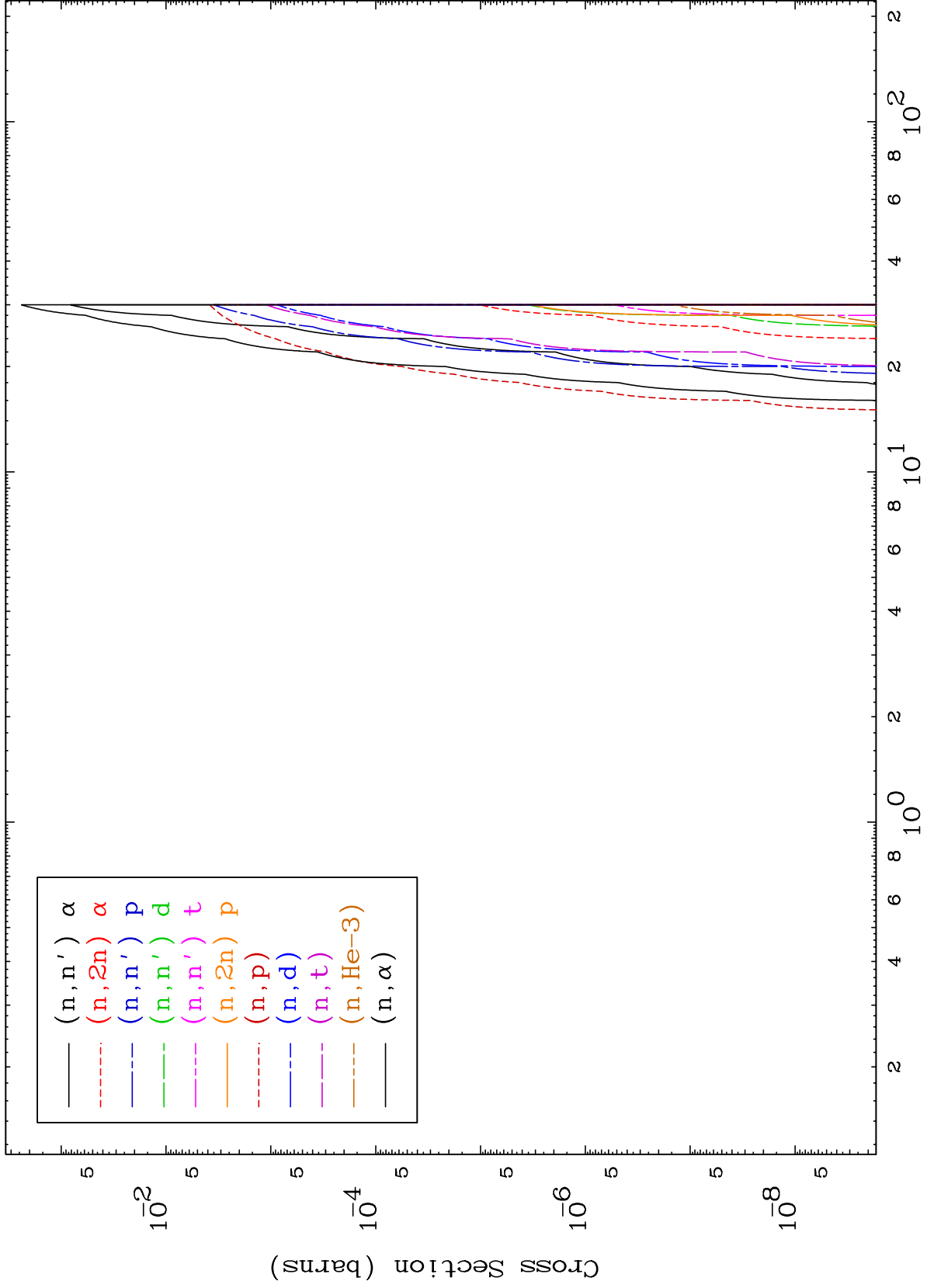


MAT 7641

$\alpha$  Charged Particle

76-0s-189m

0 Kelvin Cross Sections

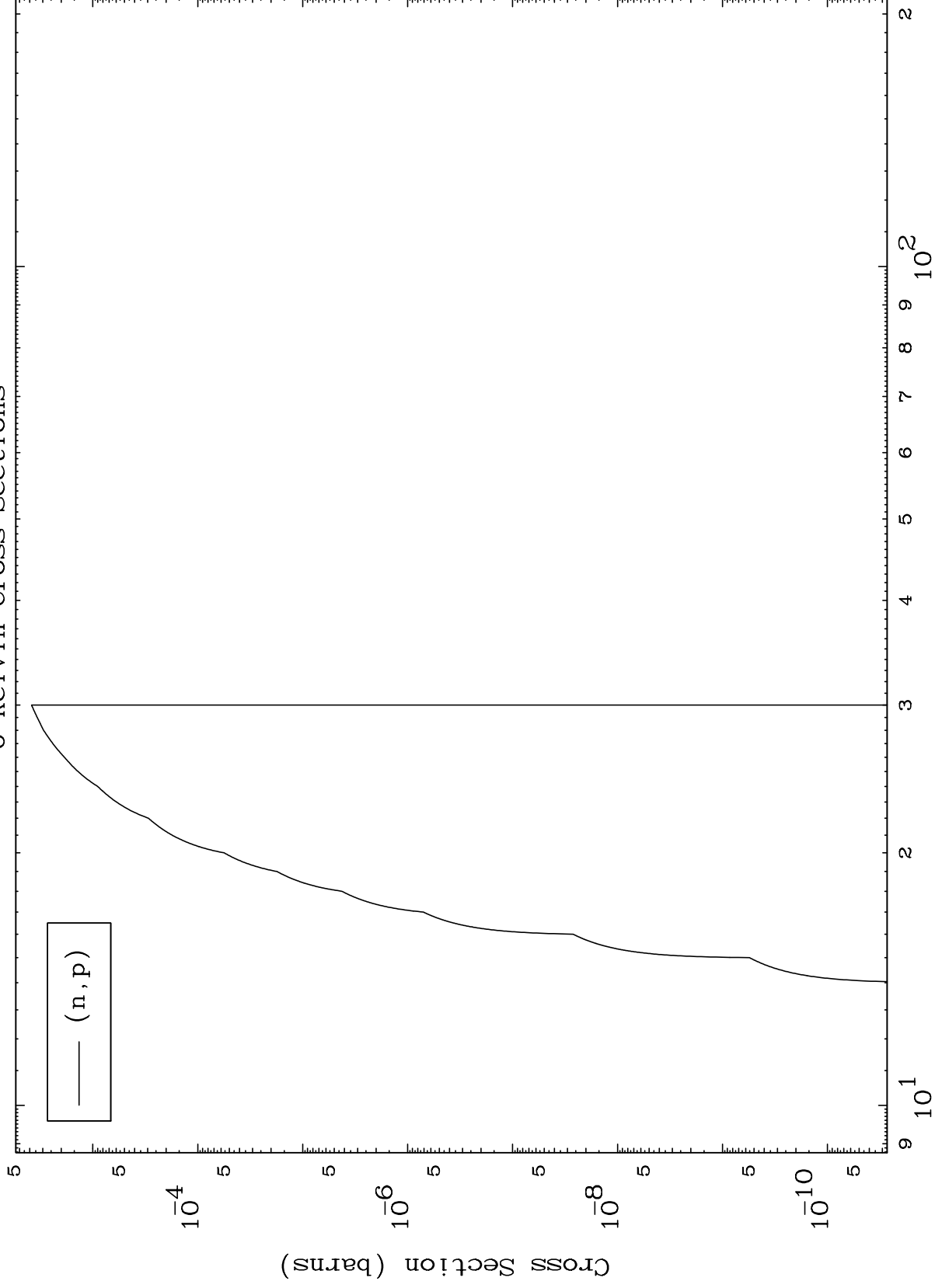


MAT 7641

( $\alpha, p$ ) Levels

76-0s-189m

0 Kelvin Cross Sections



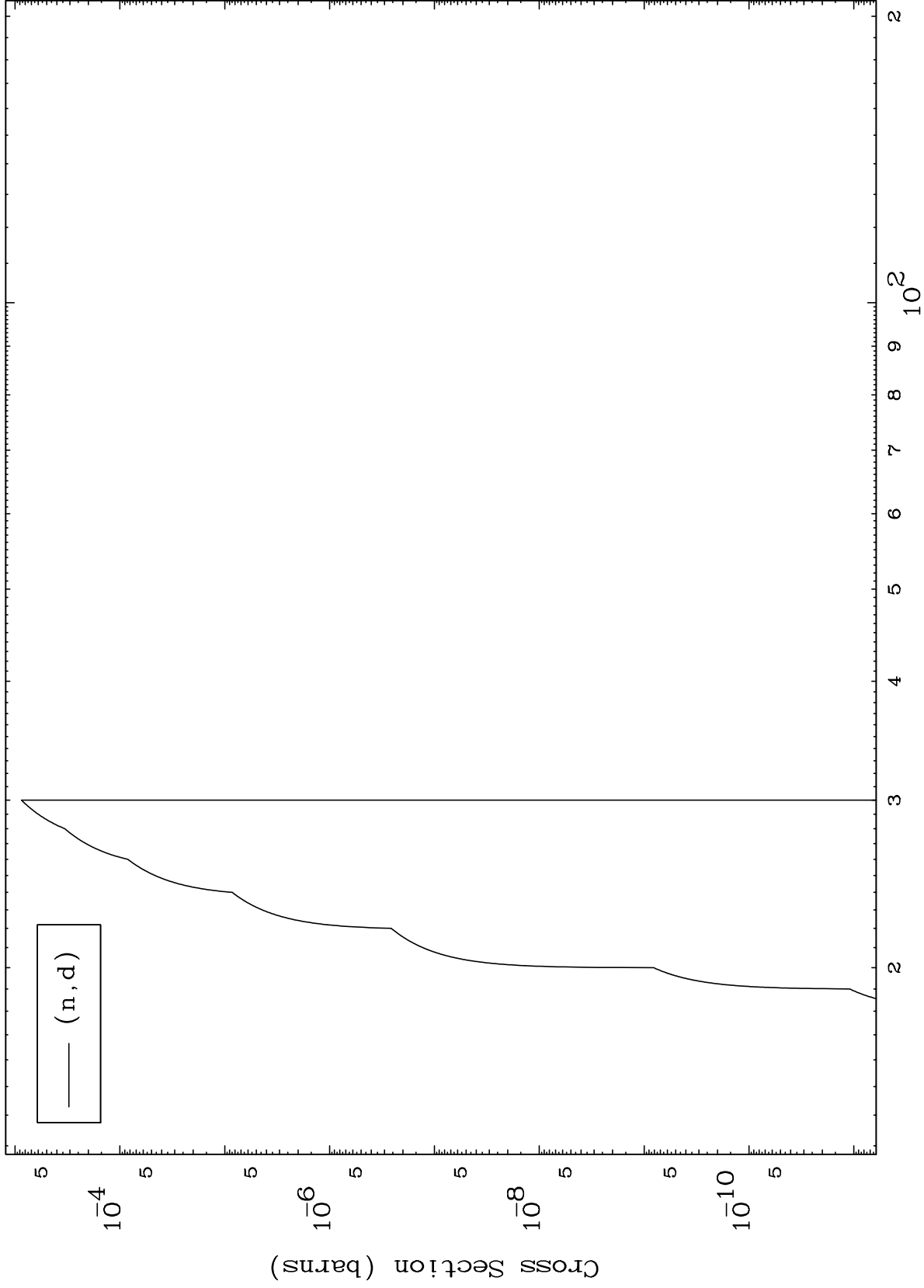
Incident Energy (MeV)

76-0s-189m

MAT 7641

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

76-0s-189m



6

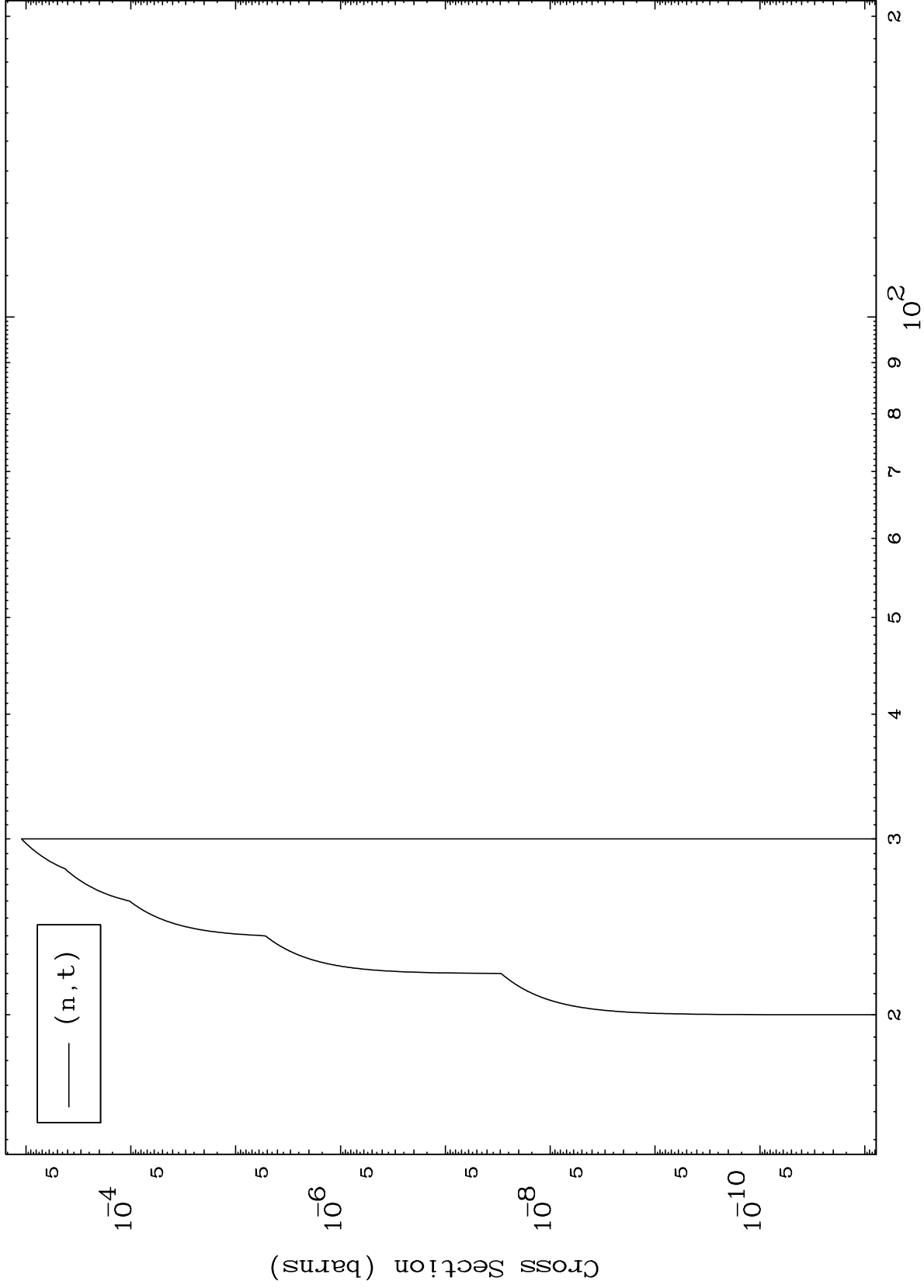
Incident Energy (MeV)

76-0s-189m

MAT 7641

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

76-0s-189m



7

Incident Energy (MeV)

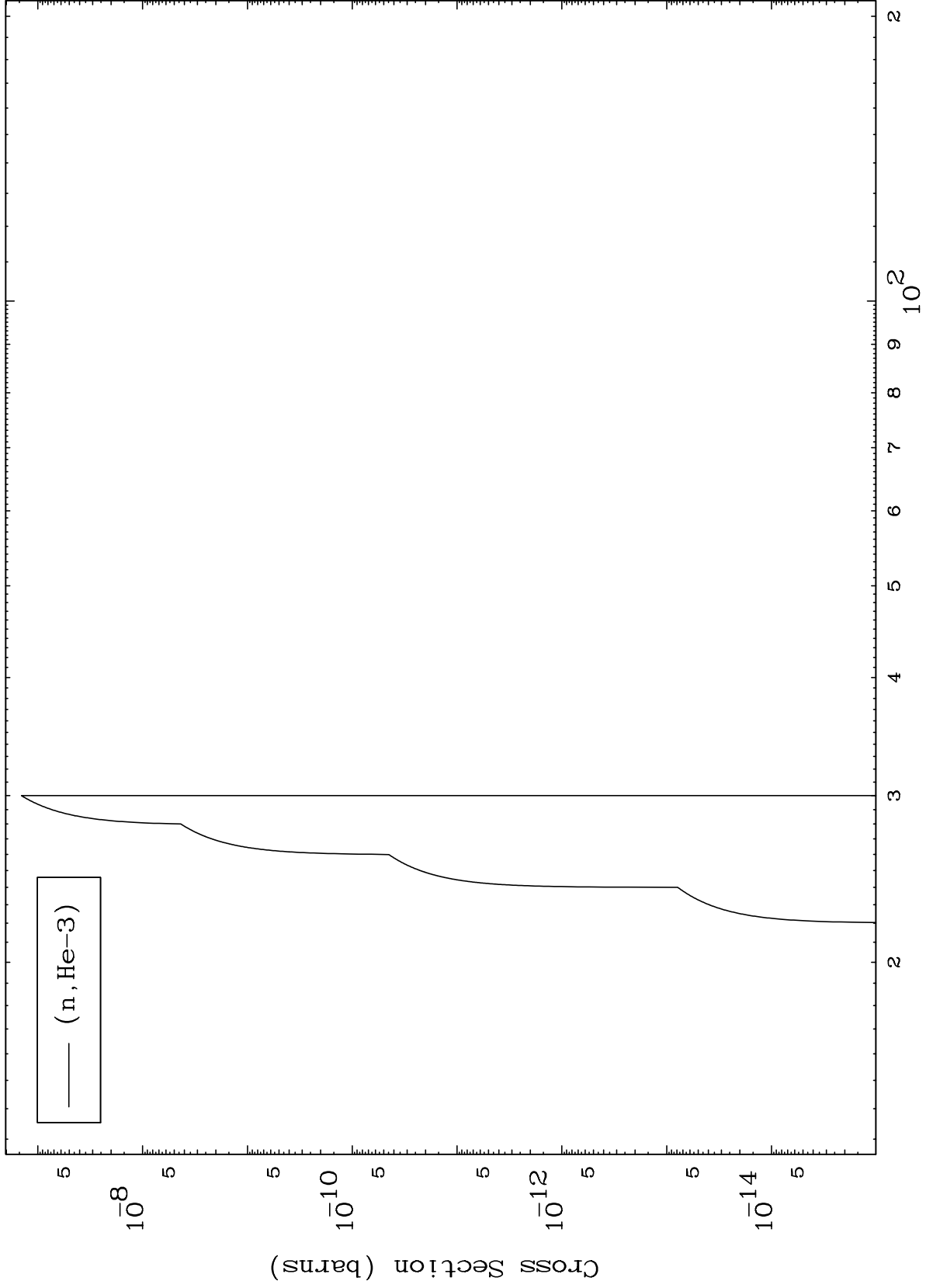
76-0s-189m



MAT 7641

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

76-0s-189m

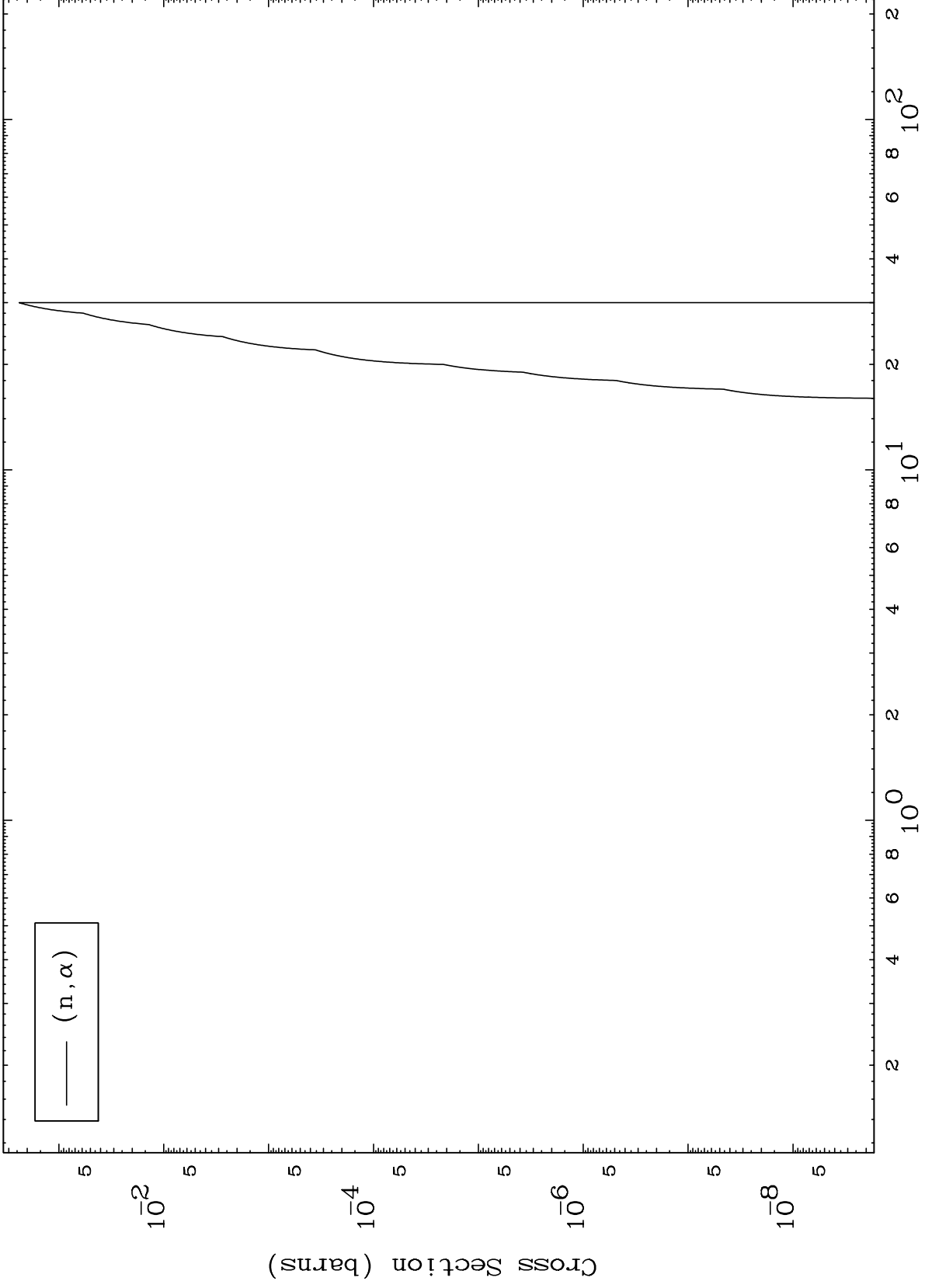


MAT 7641

( $\alpha, \alpha$ ) Levels

76-0s-189m

0 Kelvin Cross Sections

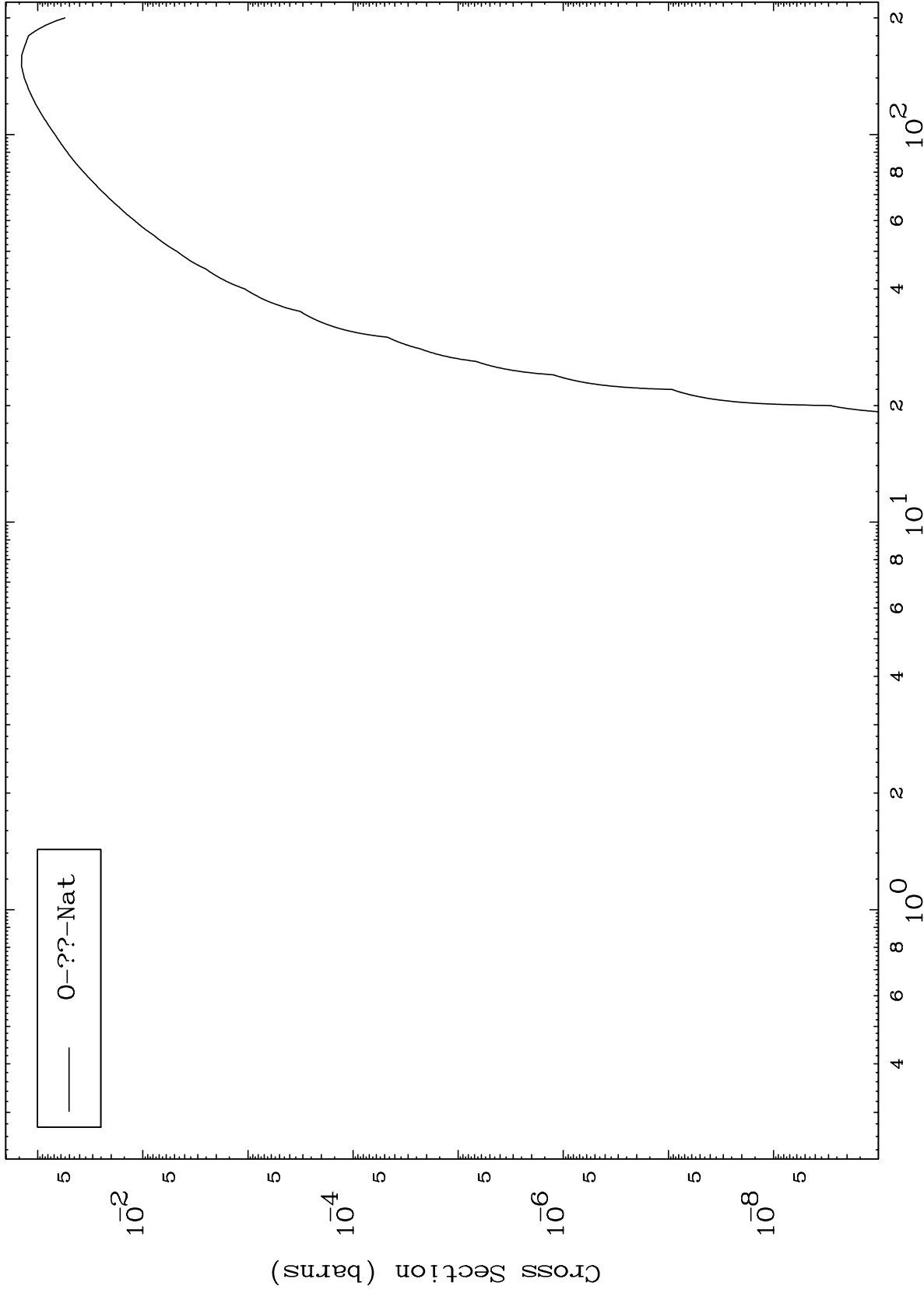


MAT 7641

Fission

<sup>76</sup>Os-189m

Radionuclide Production Cross Section



10

Incident Energy (MeV)

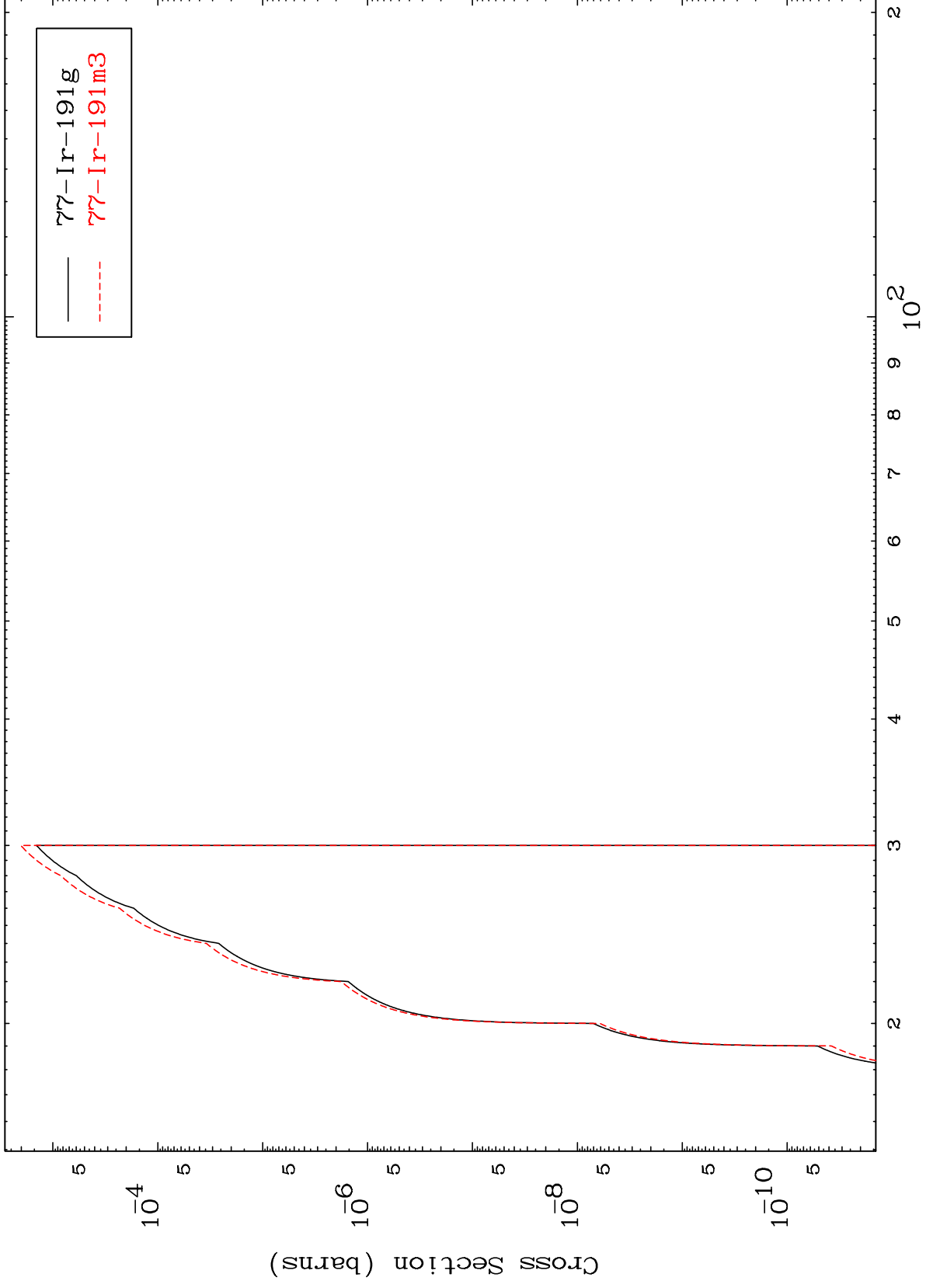
<sup>76</sup>Os-189m

MAT 7641

(n,n') p

76-Os-189m

Radionuclide Production Cross Section



11

Incident Energy (MeV)

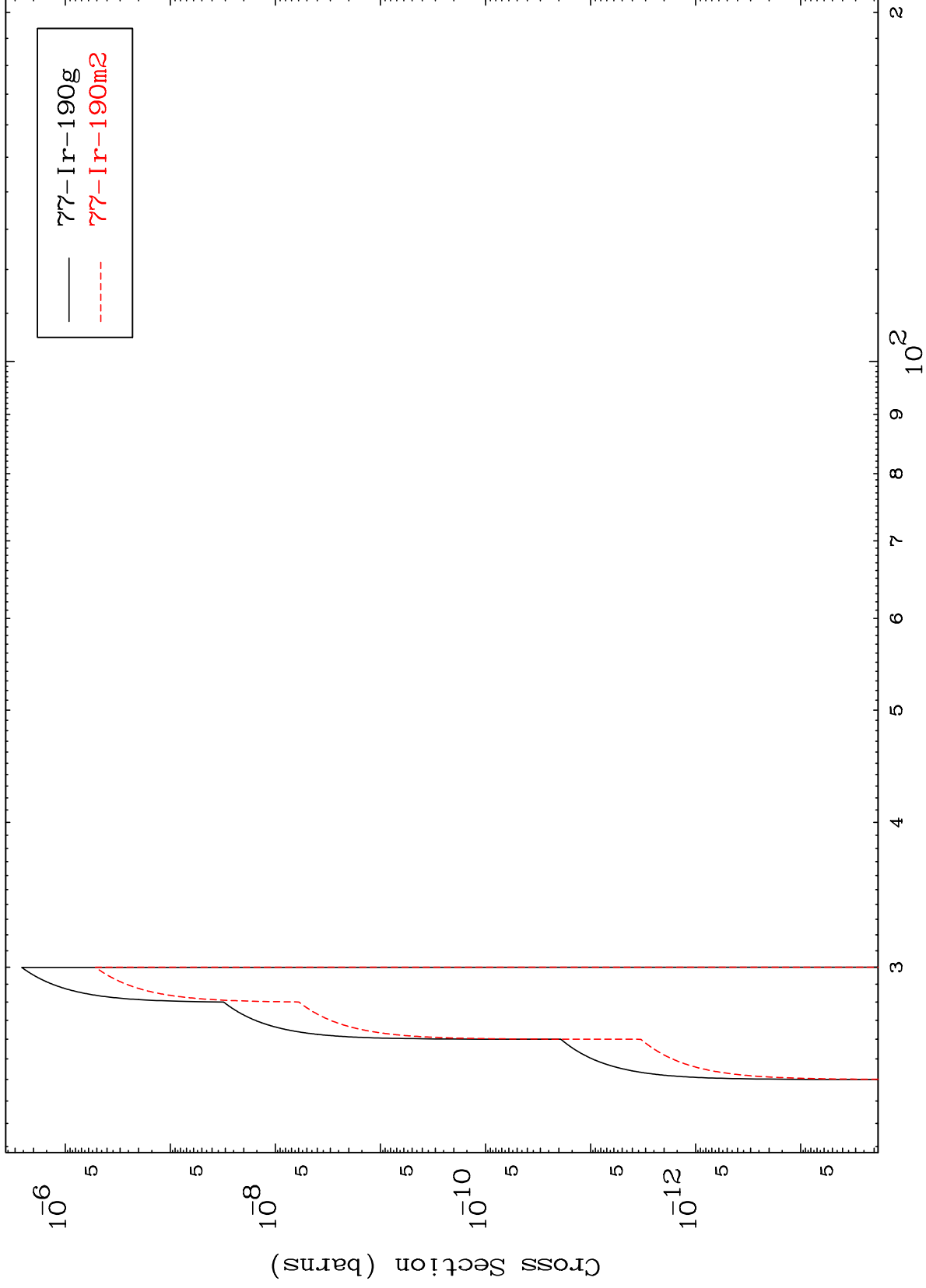
76-Os-189m

MAT 7641

(n,n') d

76-0s-189m

Radionuclide Production Cross Section



12

Incident Energy (MeV)

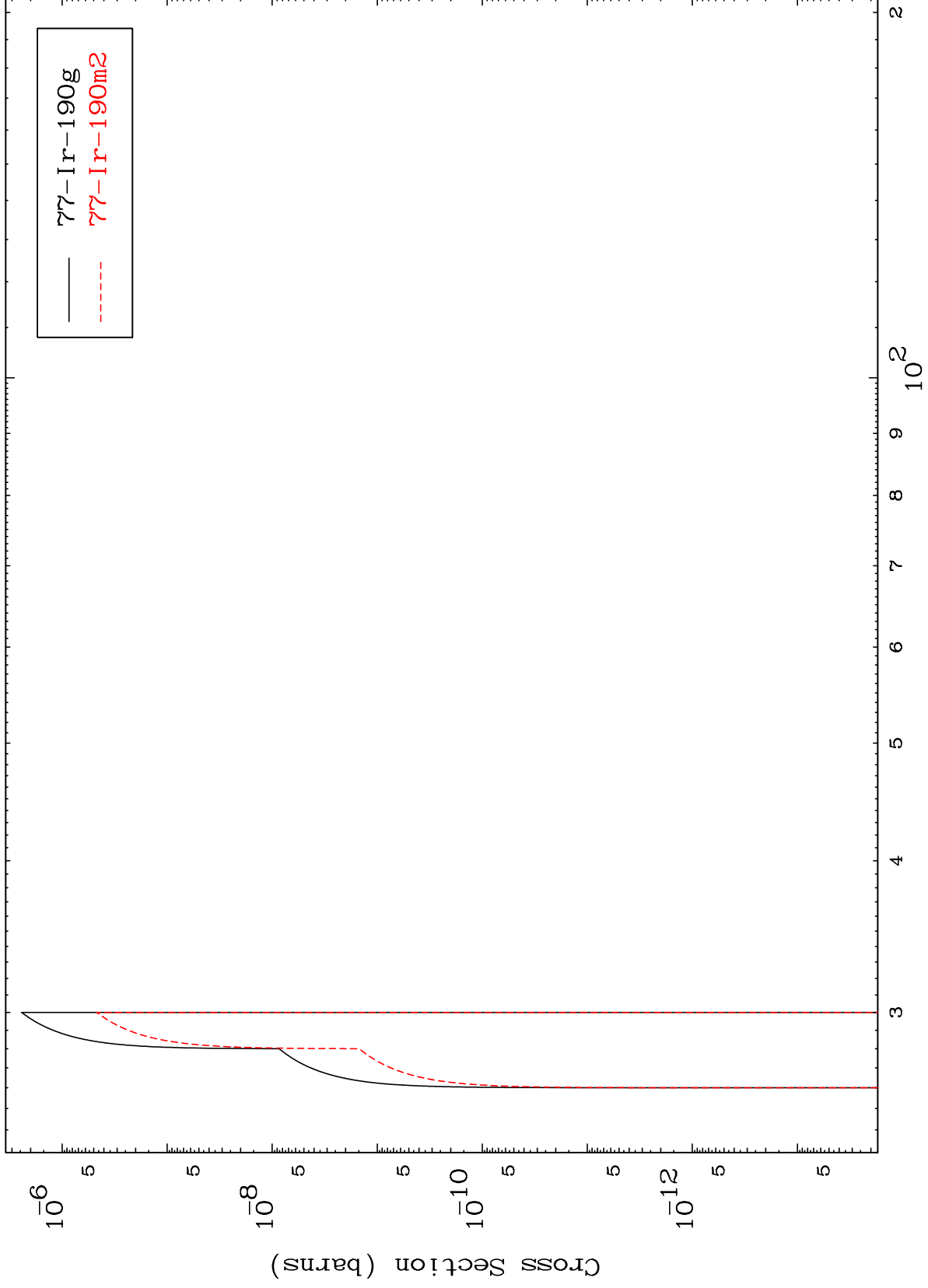
76-0s-189m

MAT 7641

(n,2n) p

76-0s-189m

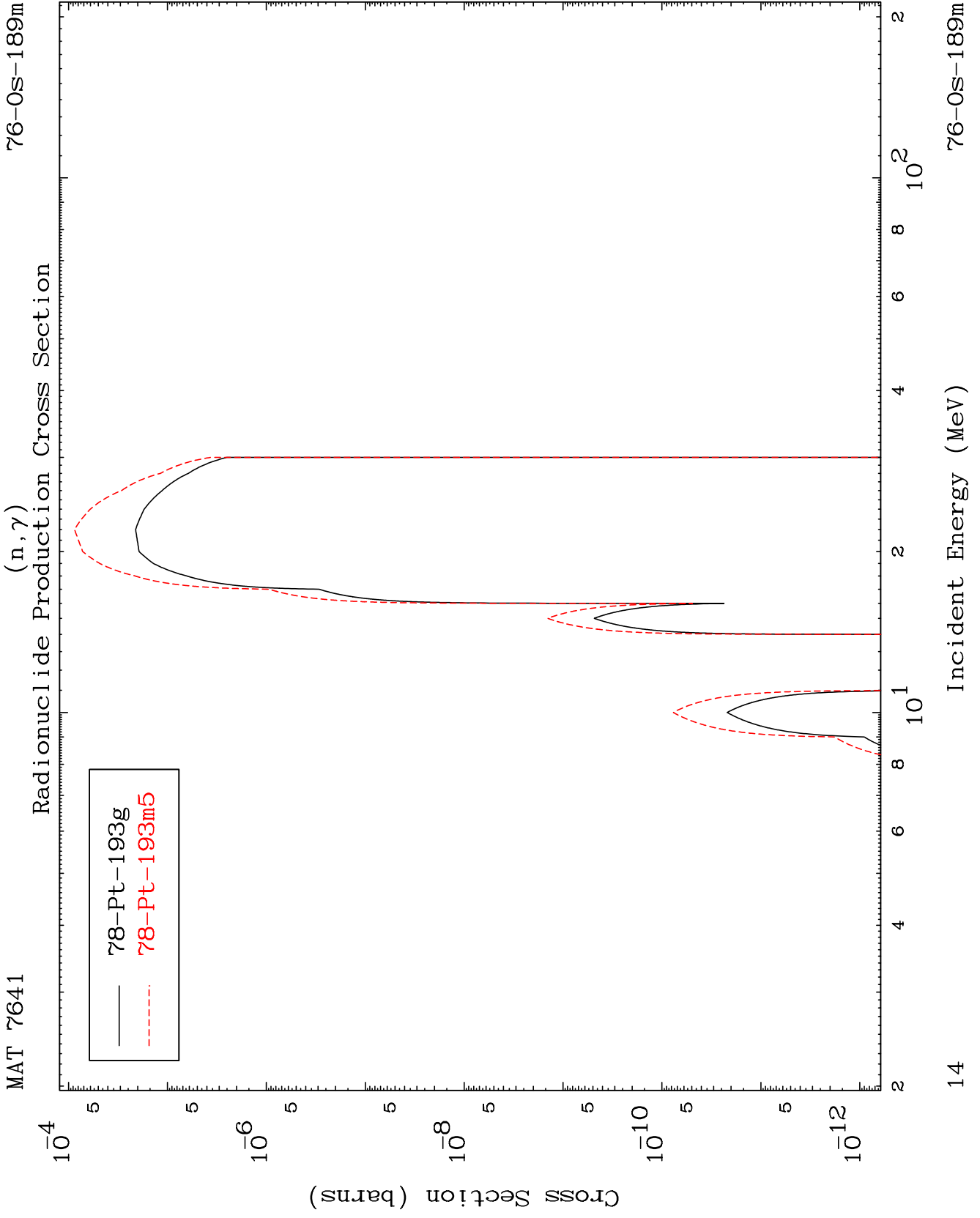
Radionuclide Production Cross Section



13

Incident Energy (MeV)

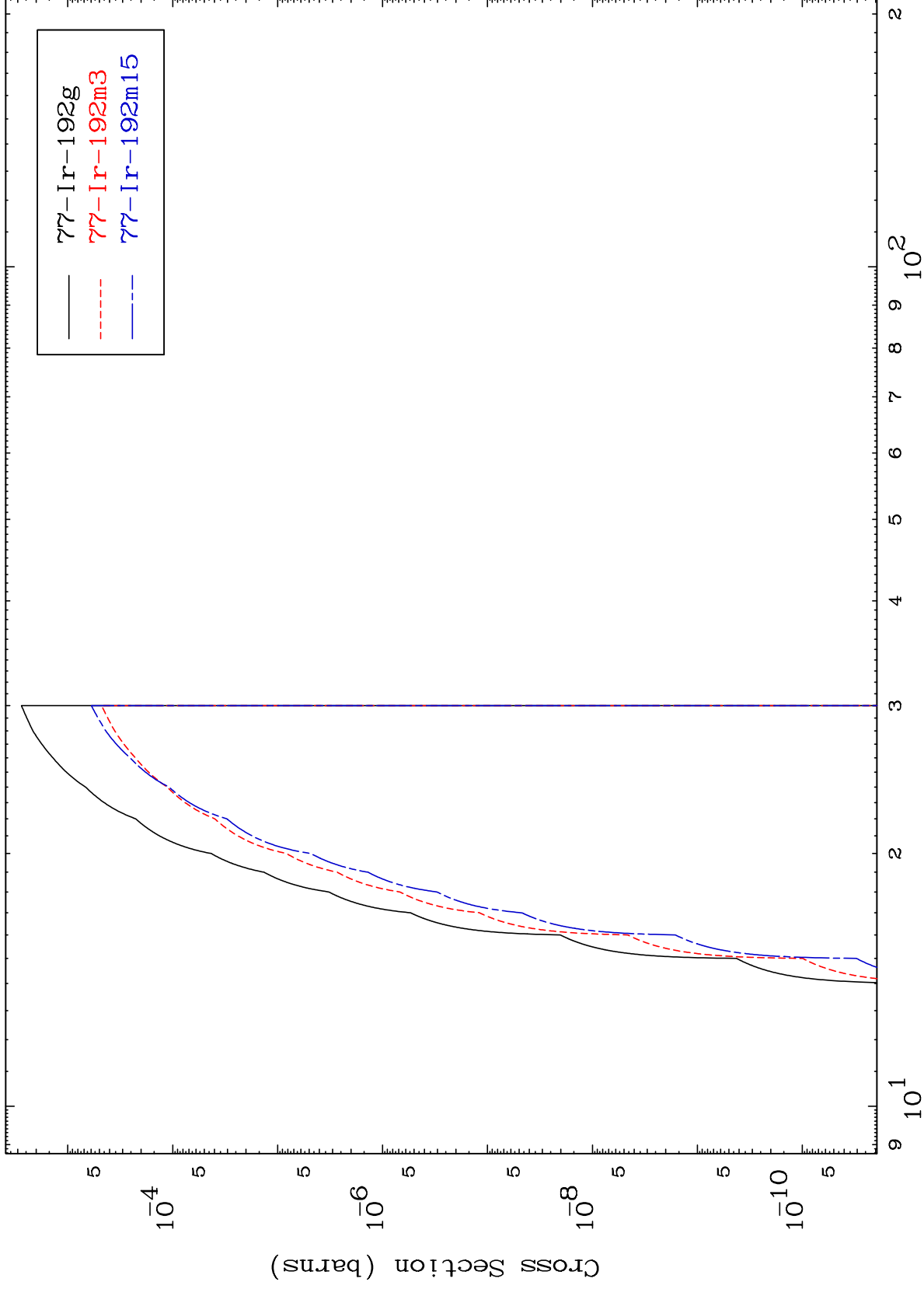
76-0s-189m



MAT 7641

76-0s-189m

(n,p)  
Radionuclide Production Cross Section



76-0s-189m

Incident Energy (MeV)

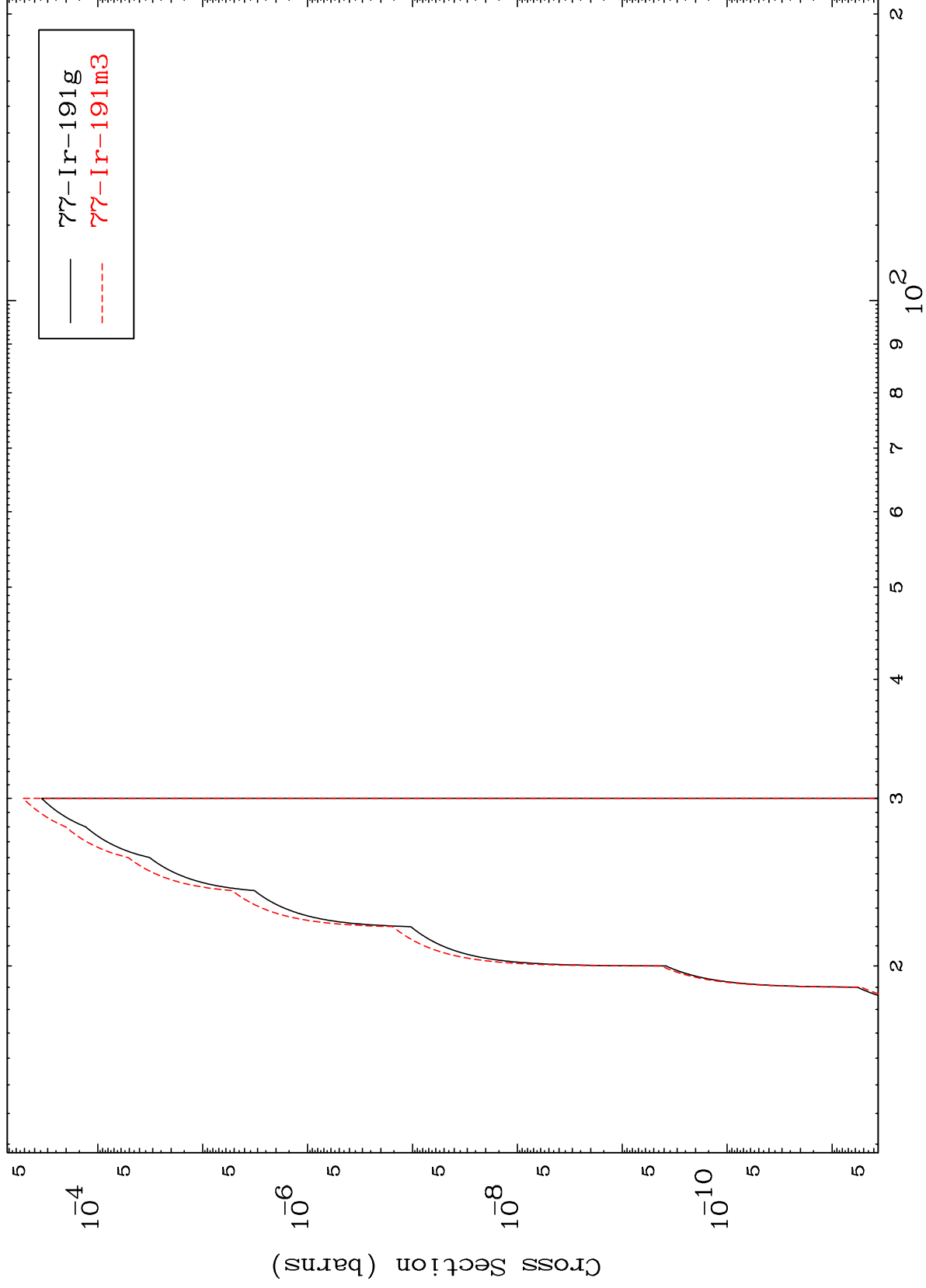
15



MAT 7641

76-0s-189m

(n,d)  
Radionuclide Production Cross Section



16

Incident Energy (MeV)

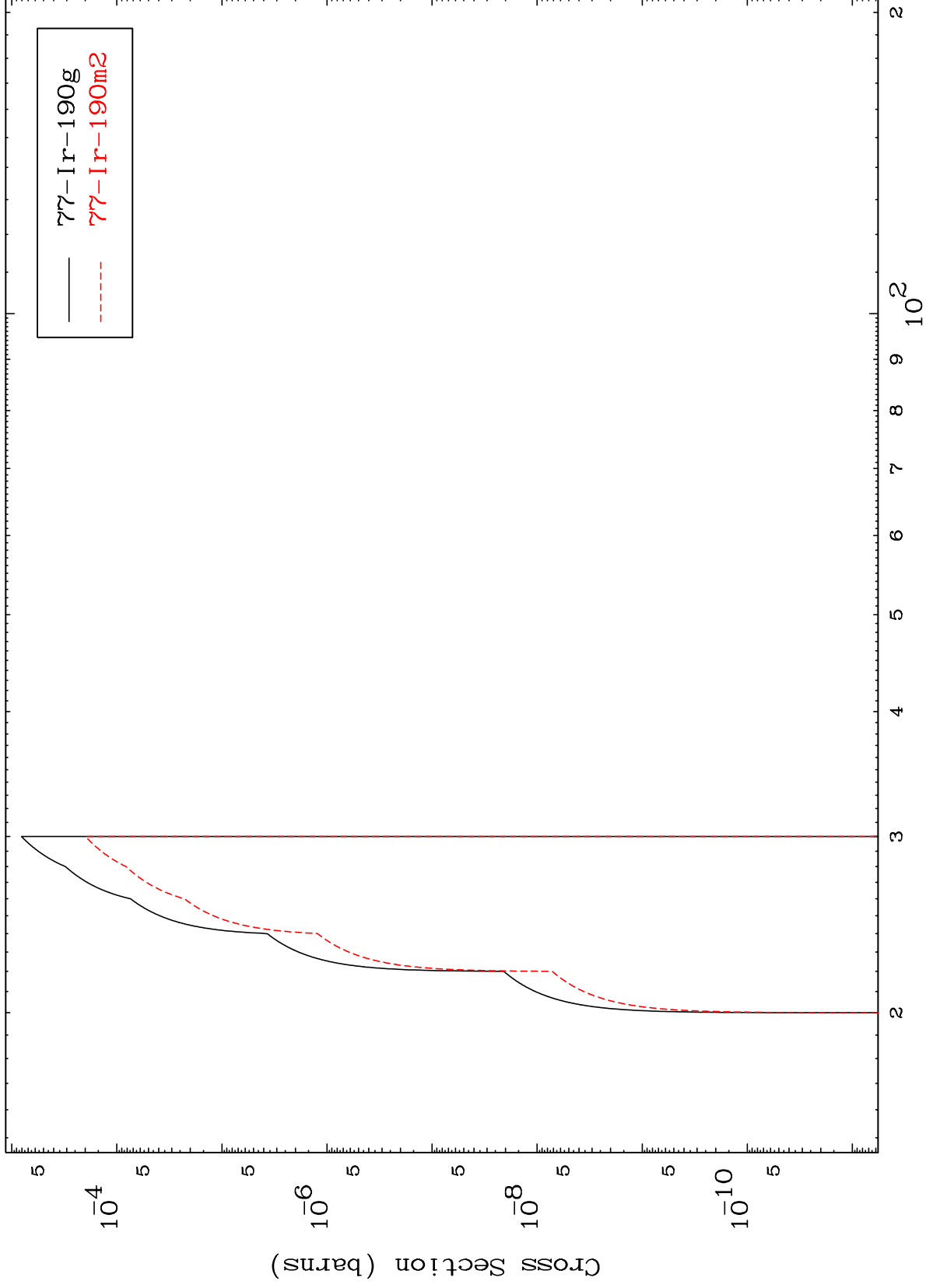
76-0s-189m

MAT 7641

(n, t)

76-0s-189m

Radionuclide Production Cross Section



17

Incident Energy (MeV)

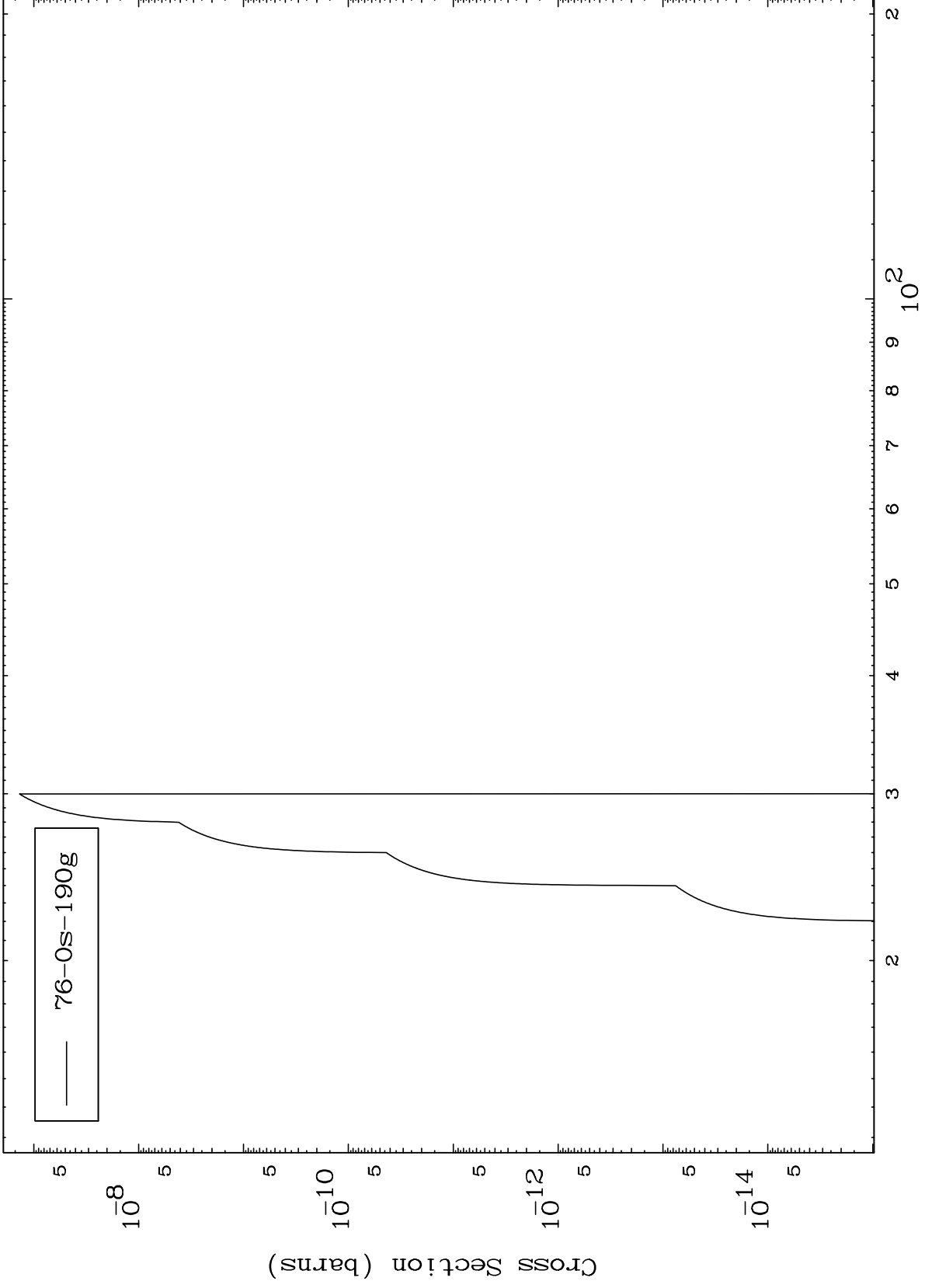
76-0s-189m

MAT 7641

(n,He-3)

76-0s-189m

Radionuclide Production Cross Section



18

Incident Energy (MeV)

76-0s-189m

MAT 7641

76-0s-189m

Radionuclide Production Cross Section  
(n,  $\alpha$ )

