

Program EVALPLOT  
(Version 2021-1)

by

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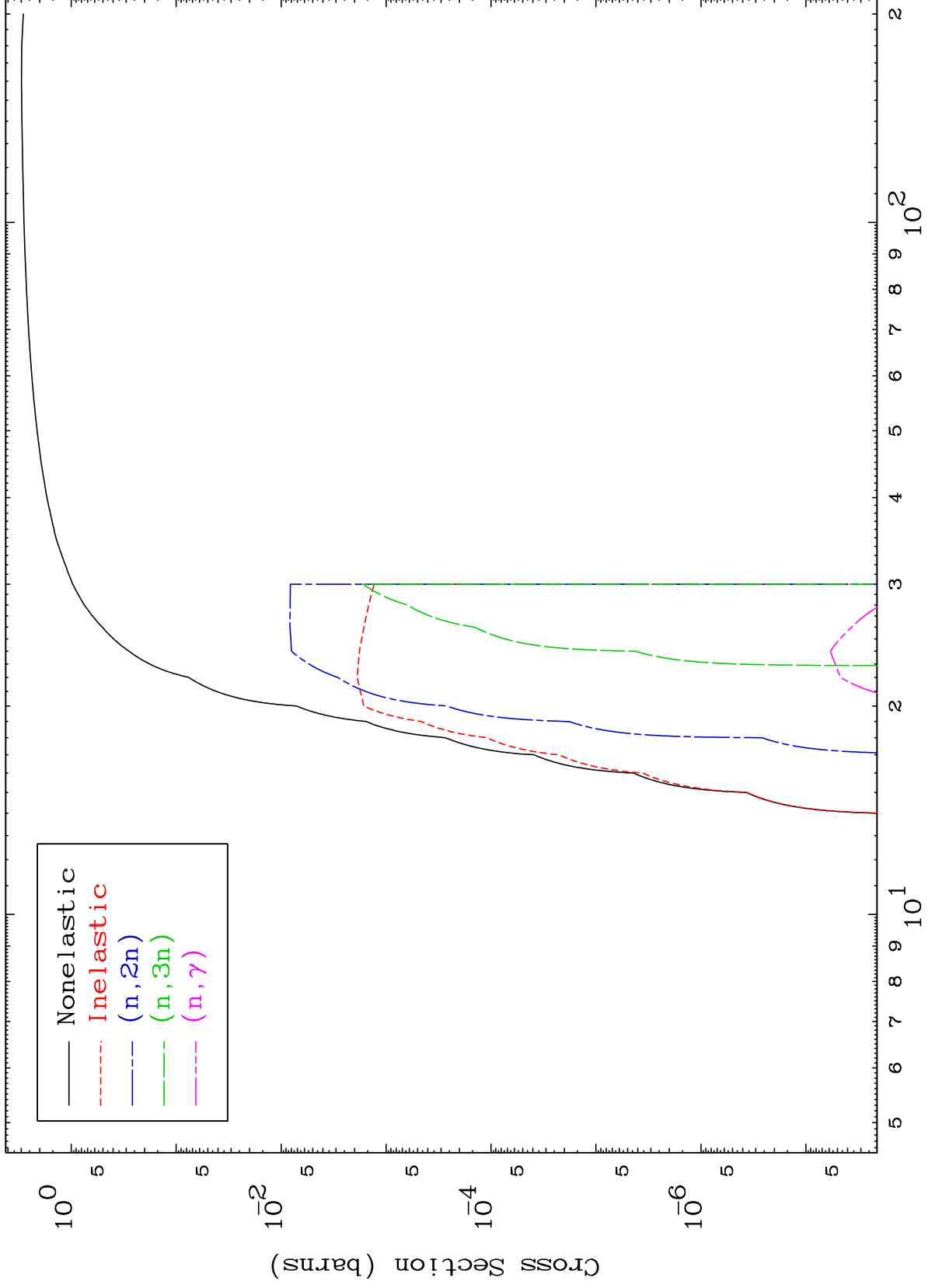
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9140

0 Kelvin Cross Sections

91-Pa-234



1

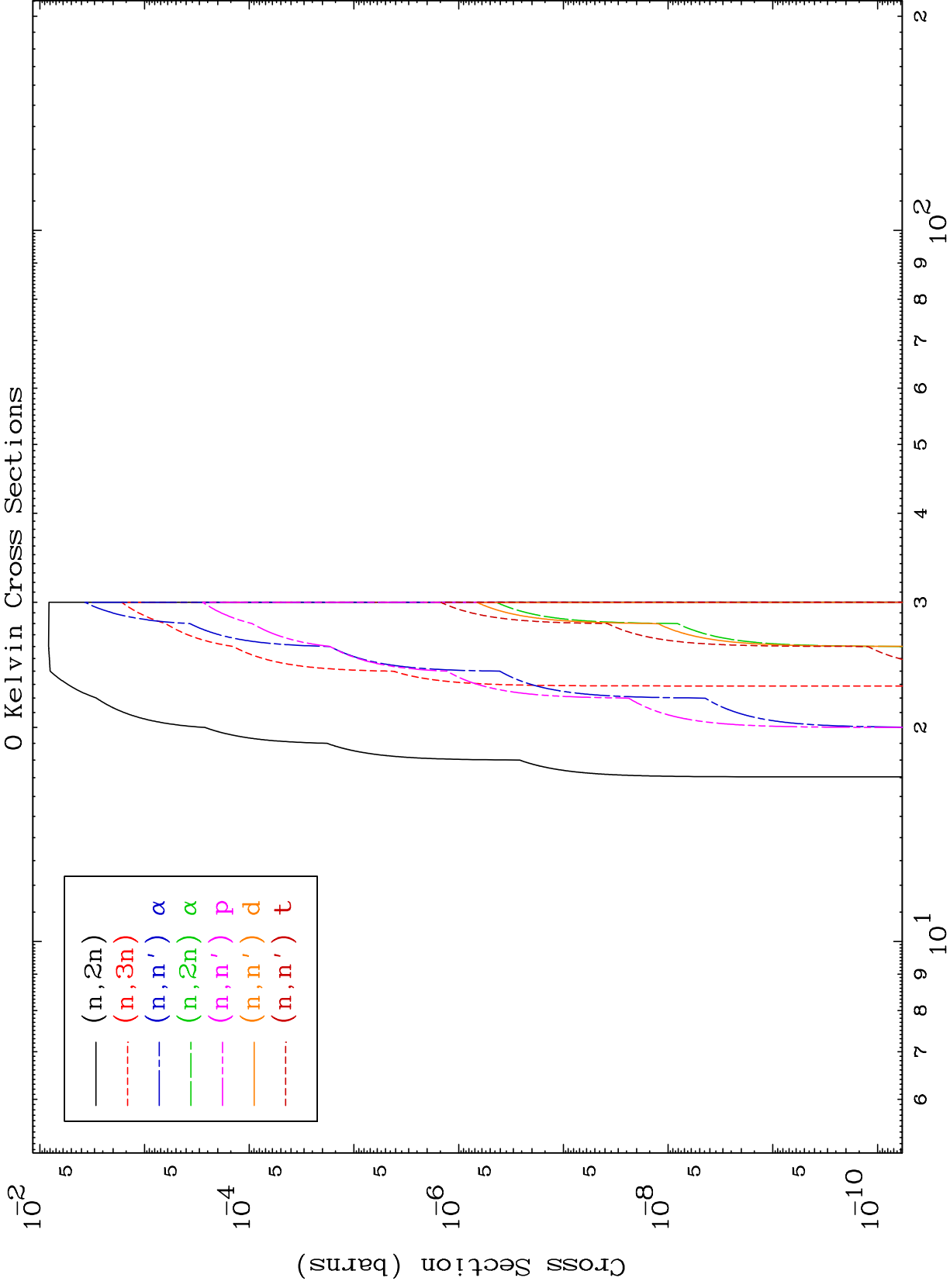
Incident Energy (MeV)

91-Pa-234

MAT 9140

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

91-Pa-234



2

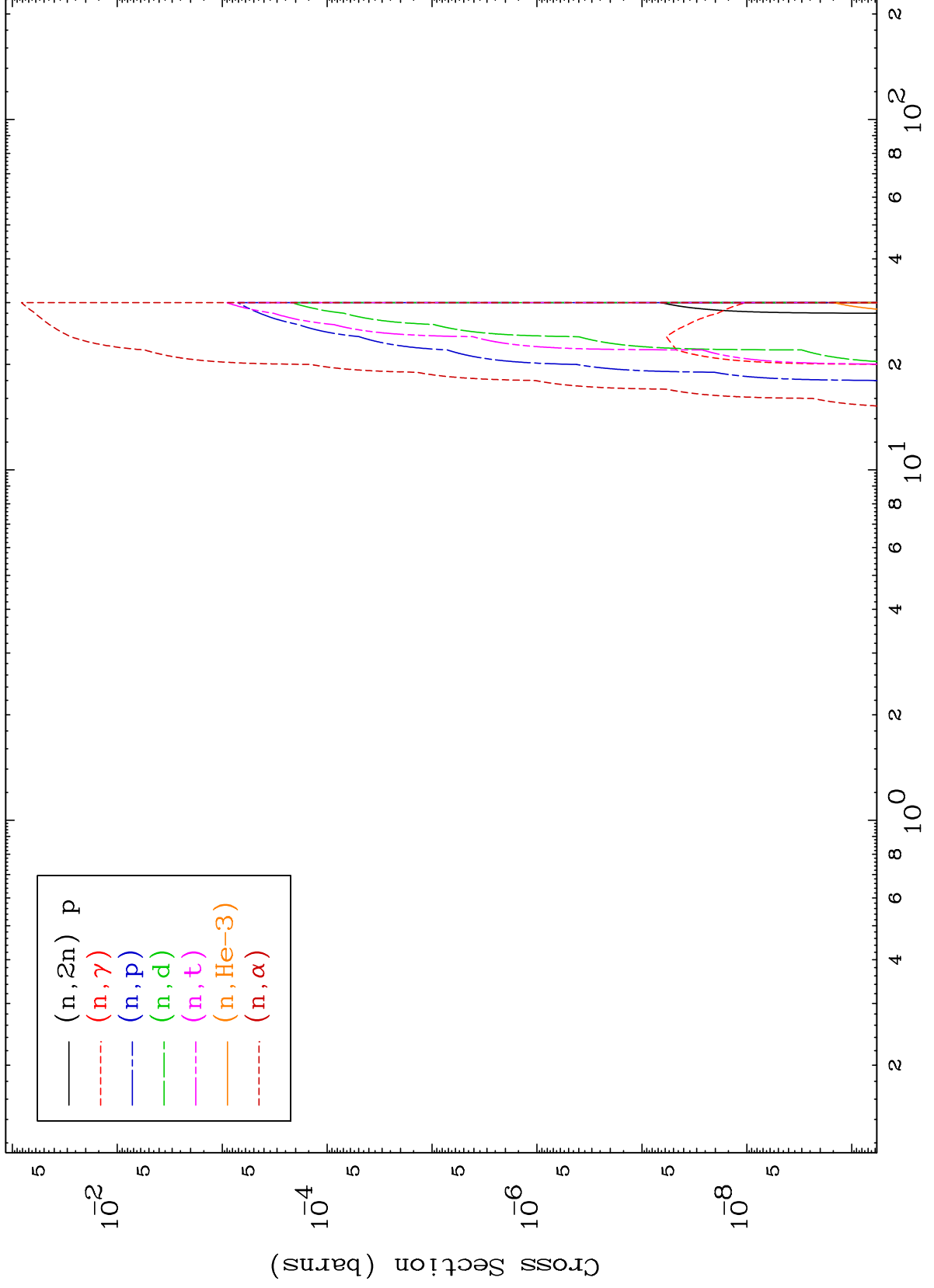
Incident Energy (MeV)

91-Pa-234

MAT 9140

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

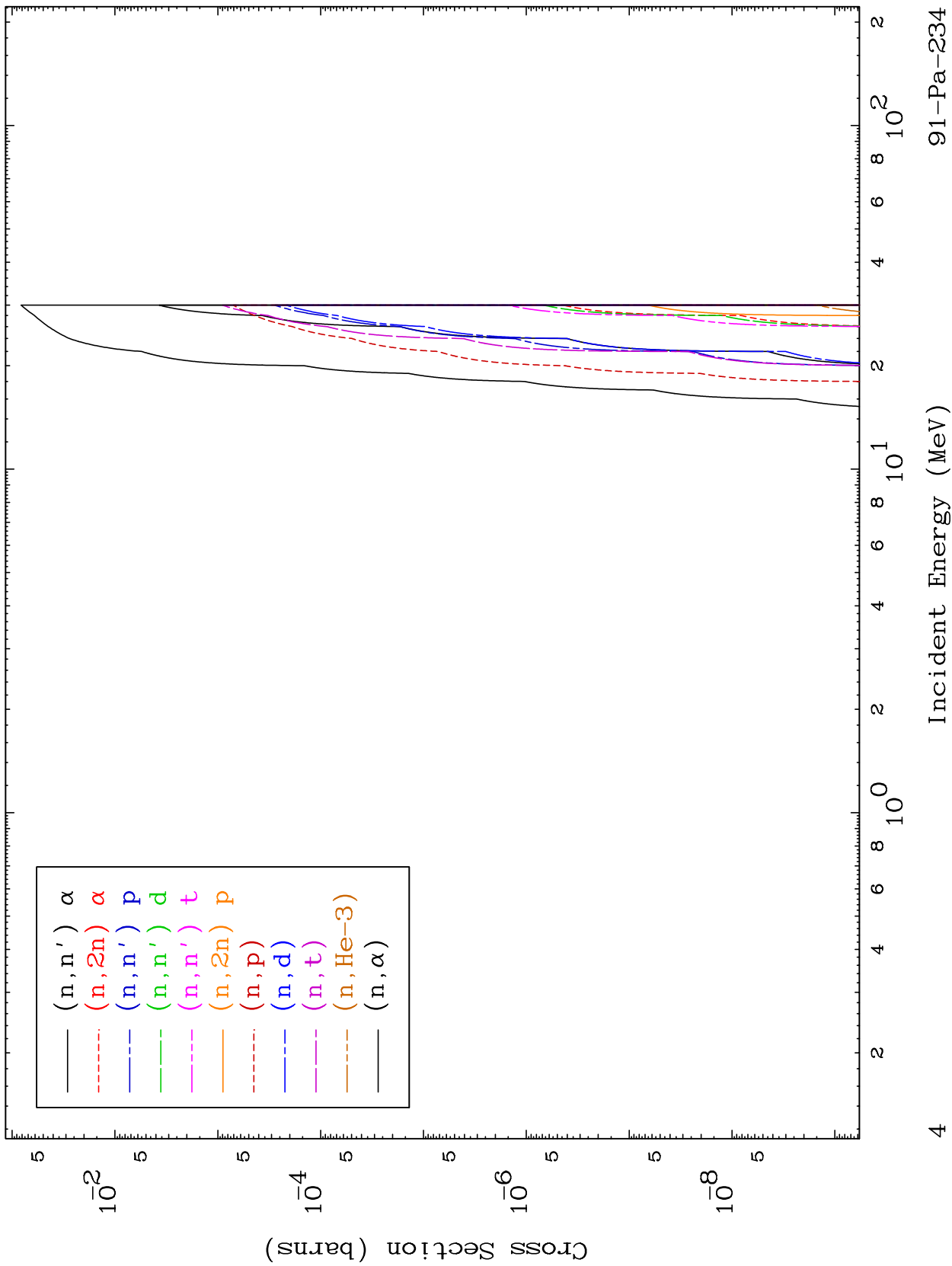
91-Pa-234



MAT 9140

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

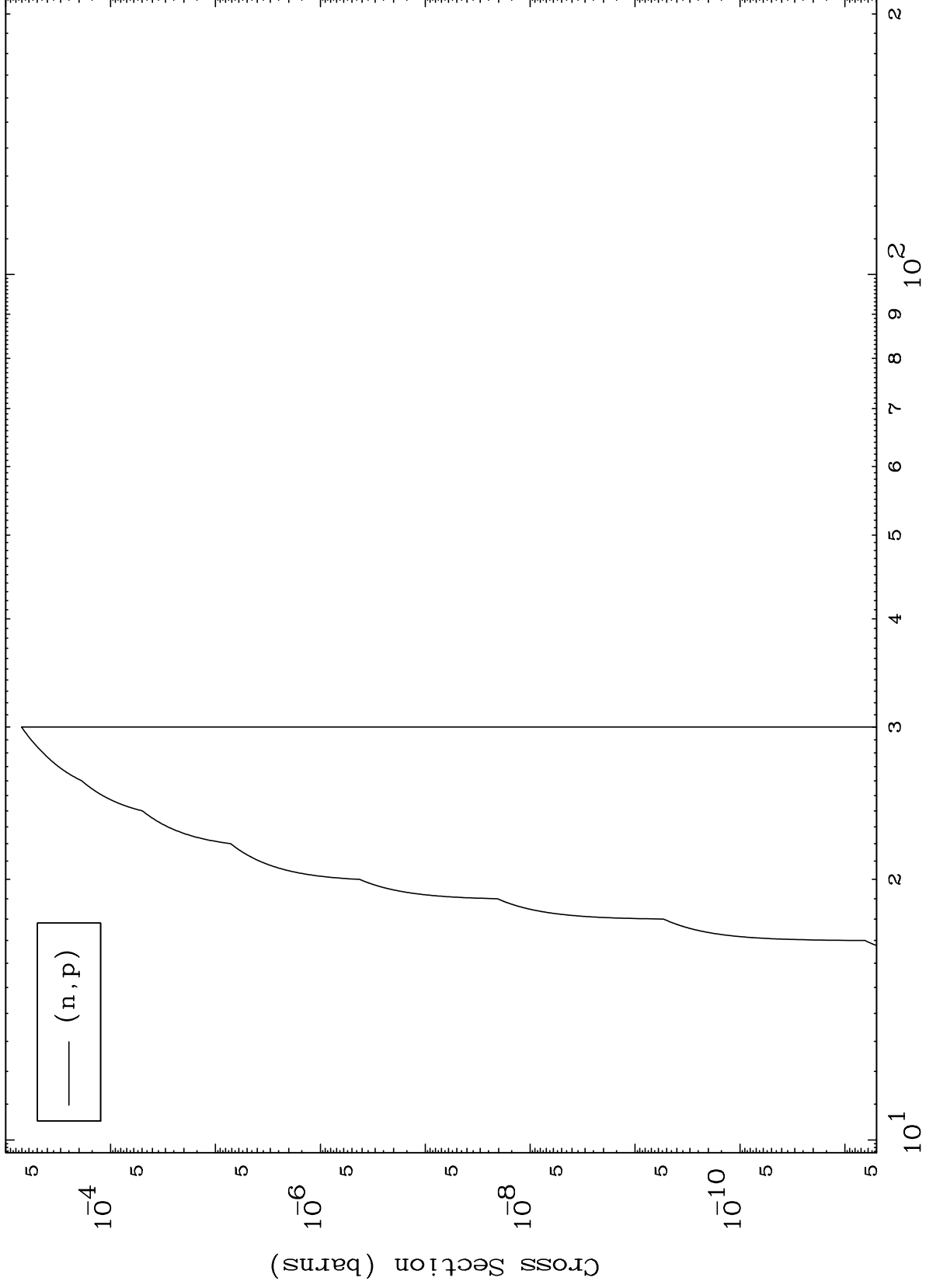
91-Pa-234



MAT 9140

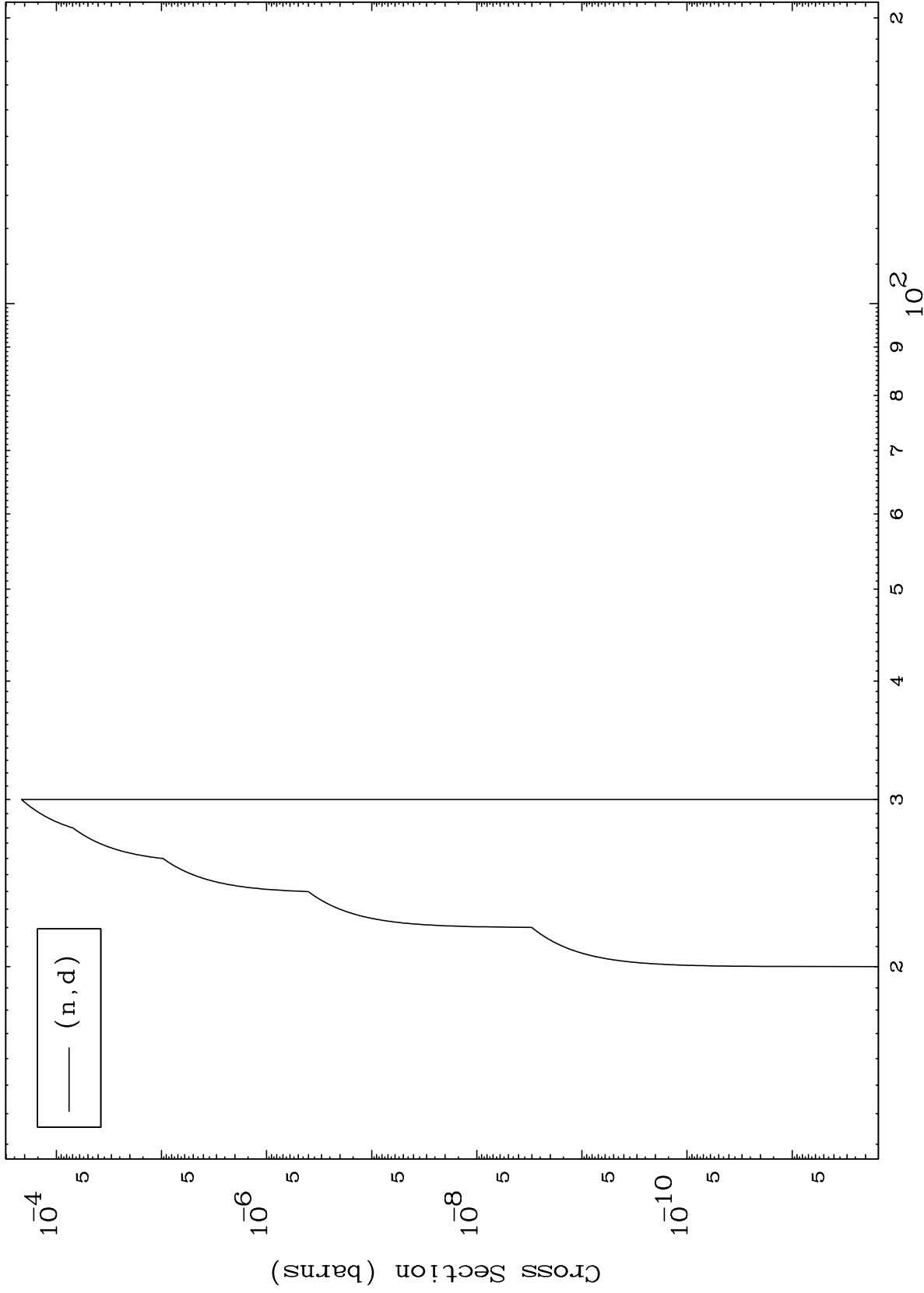
( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

91-Pa-234



Incident Energy (MeV)

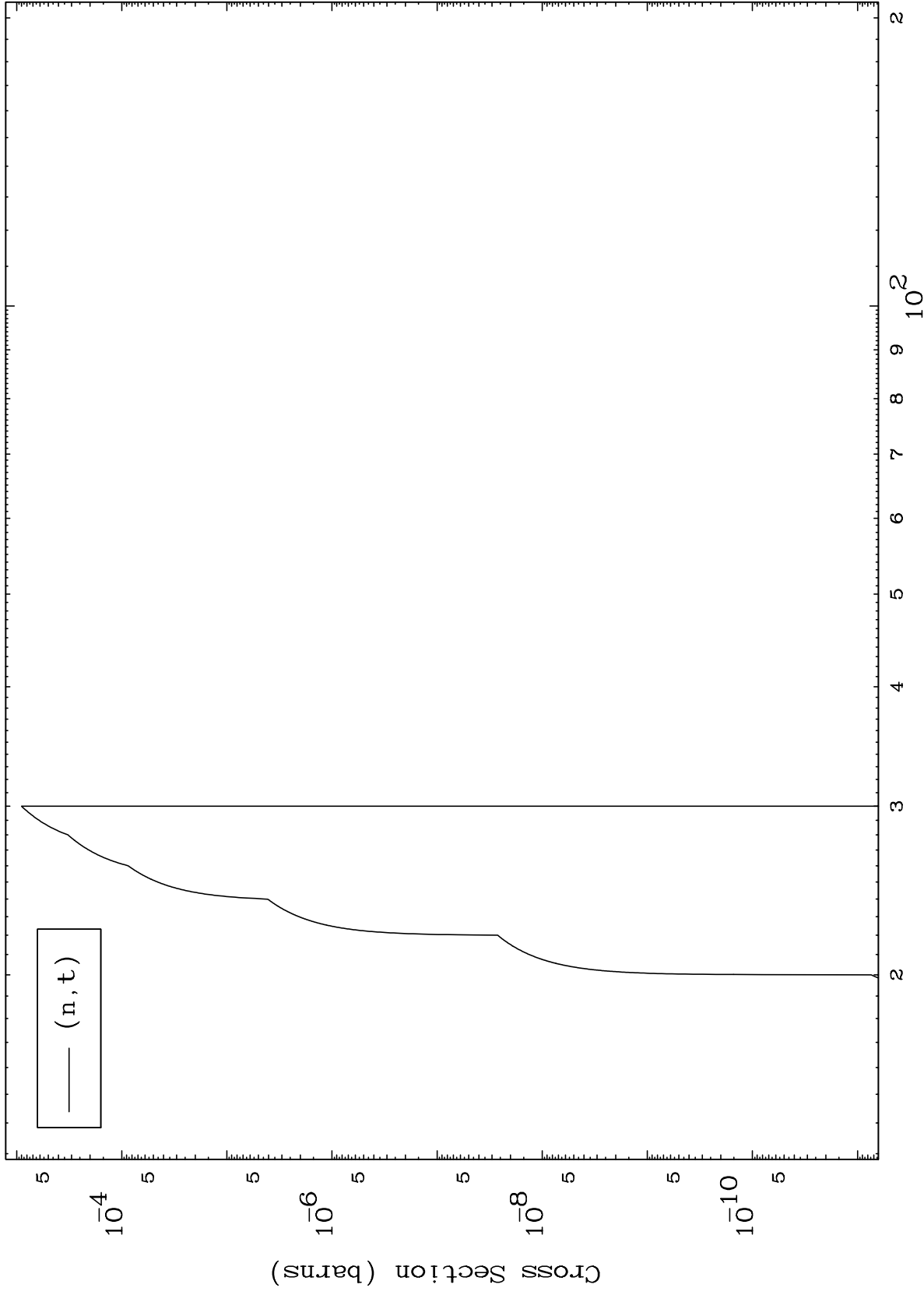
91-Pa-234



MAT 9140

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

91-Pa-234

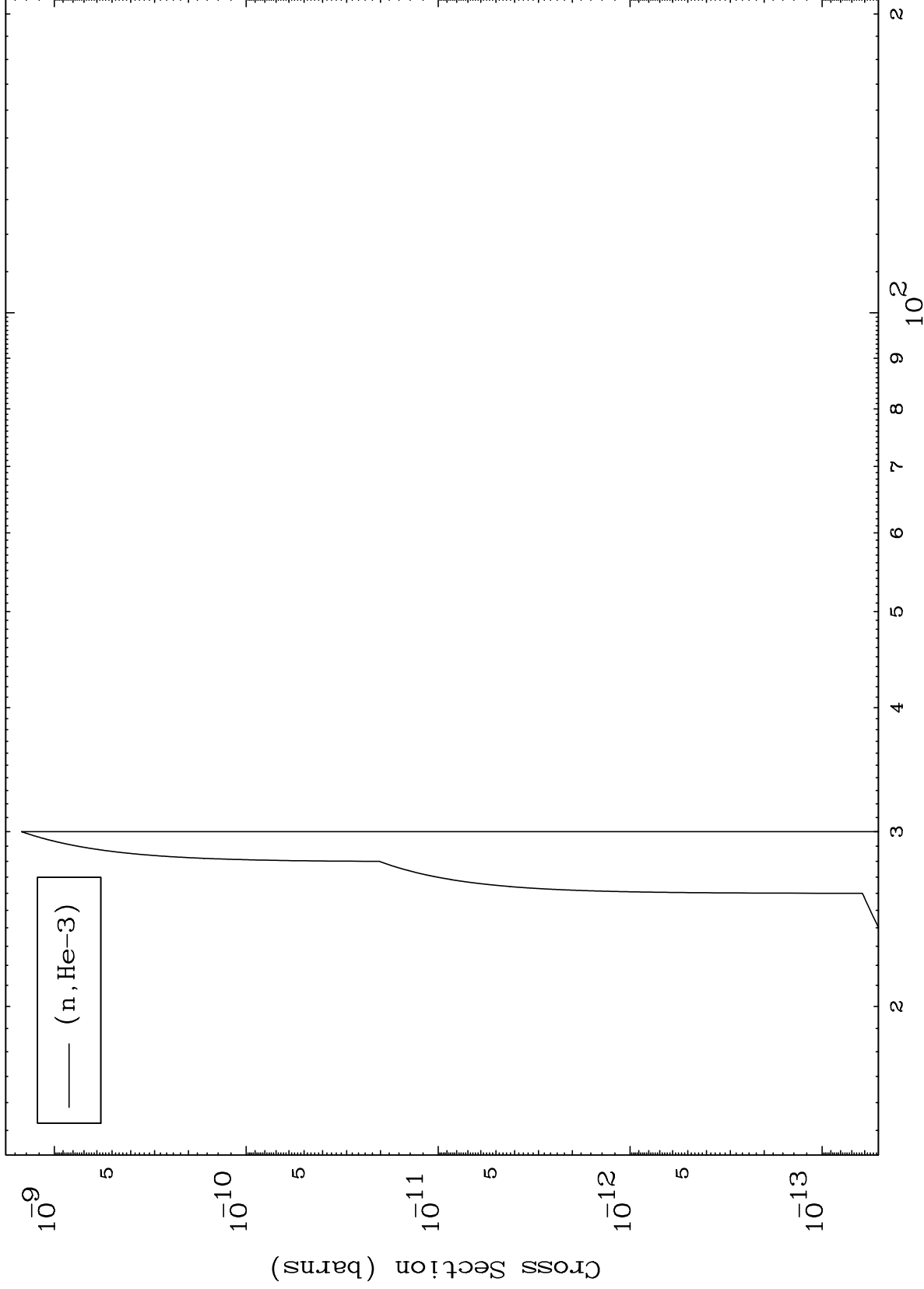


7

Incident Energy (MeV)

91-Pa-234



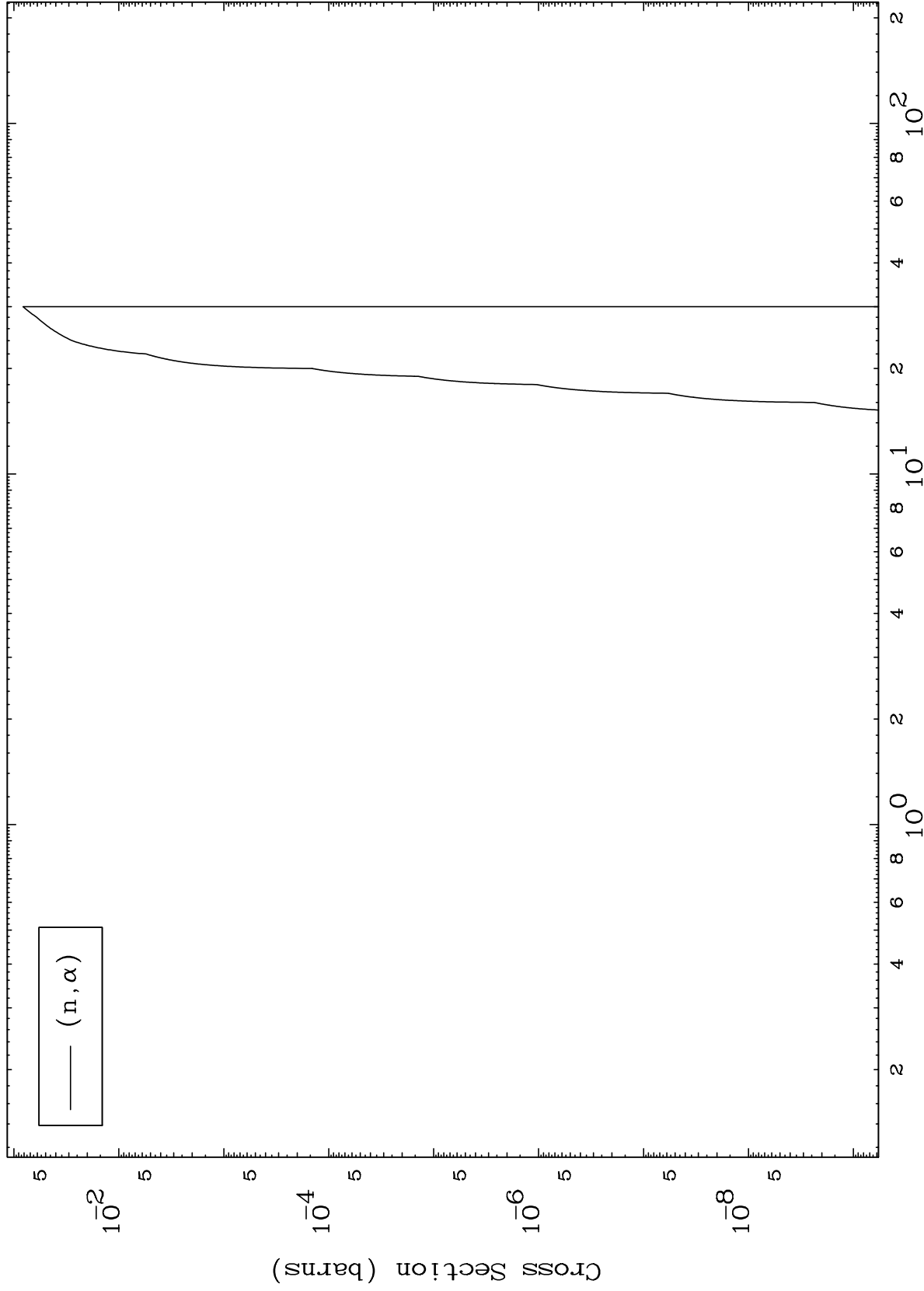


MAT 9140

( $\alpha, \alpha$ ) Levels

91-Pa-234

0 Kelvin Cross Sections

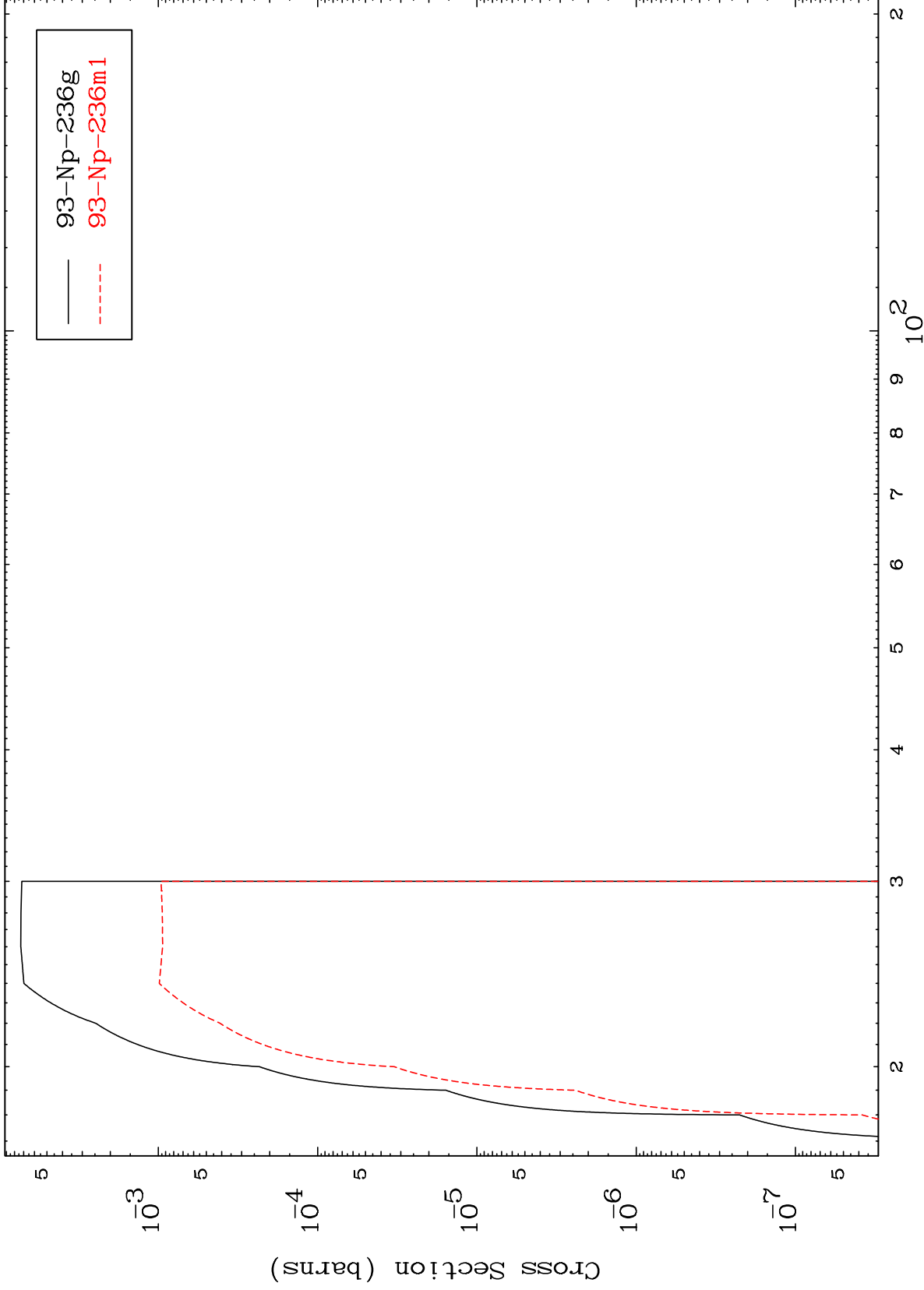


MAT 9140

(n,2n)

91-Pa-234

Radionuclide Production Cross Section



10

Incident Energy (MeV)

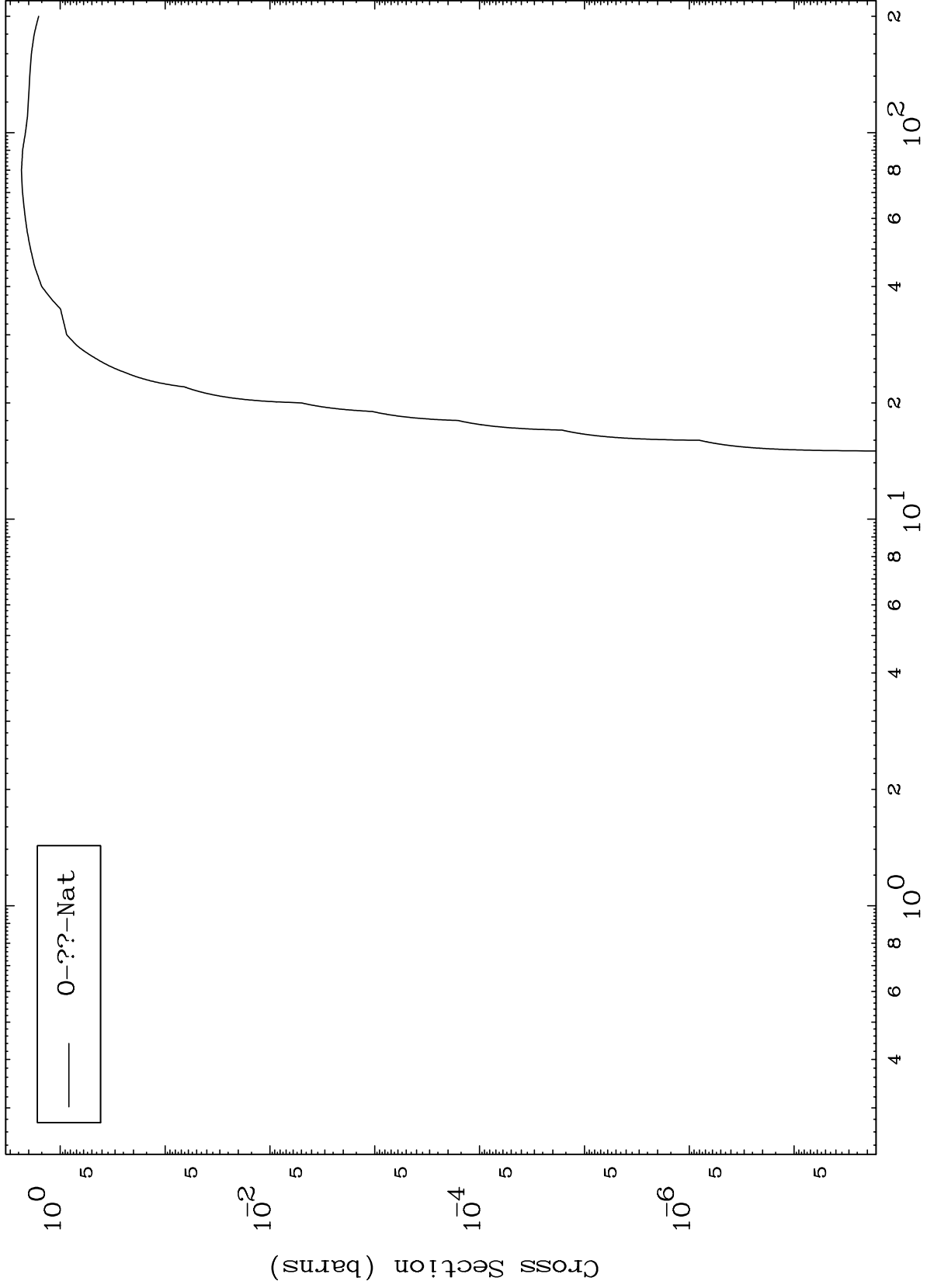
91-Pa-234

MAT 9140

Fission

91-Pa-234

Radionuclide Production Cross Section

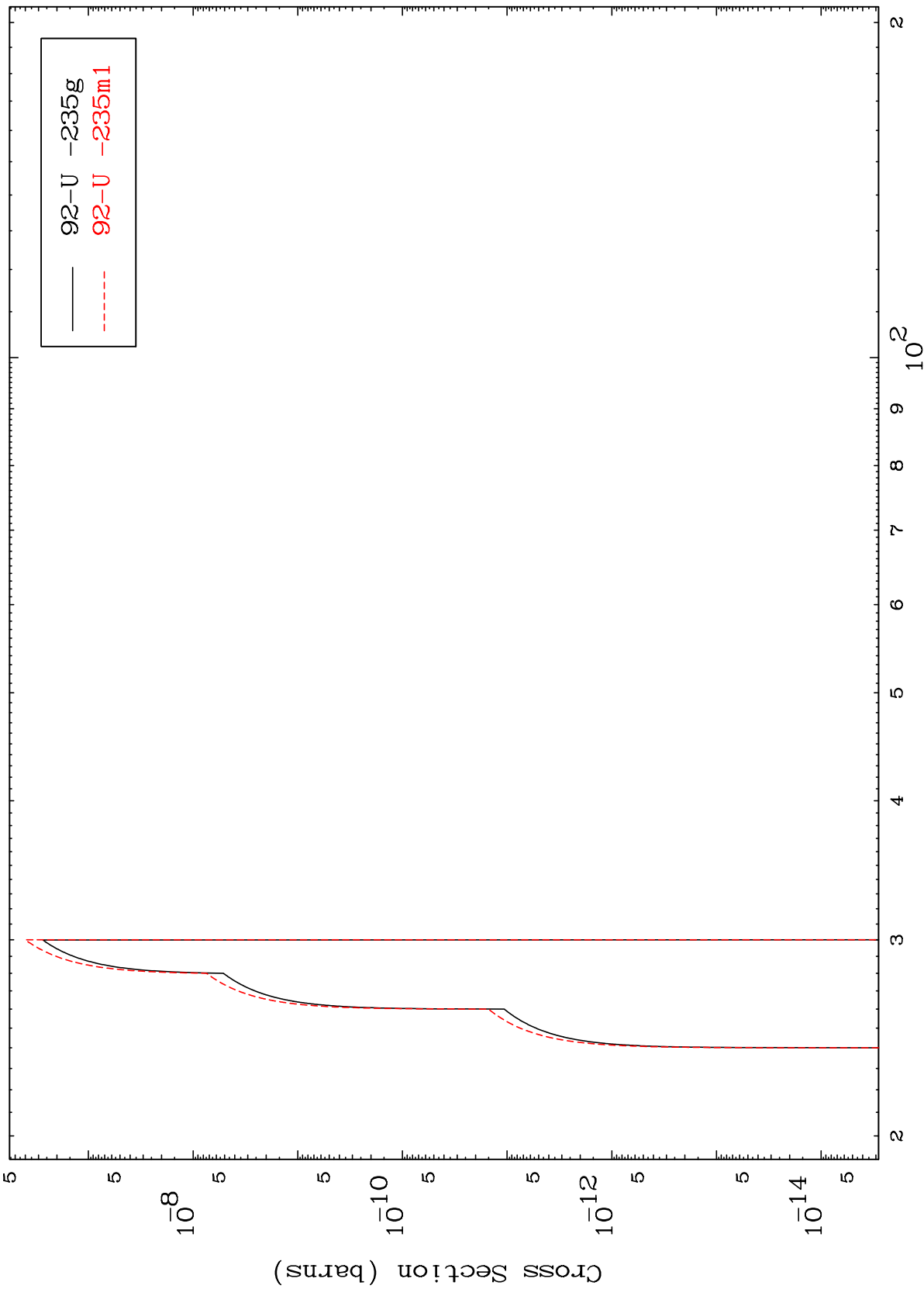


MAT 9140

(n,n') d

91-Pa-234

Radionuclide Production Cross Section



12

Incident Energy (MeV)

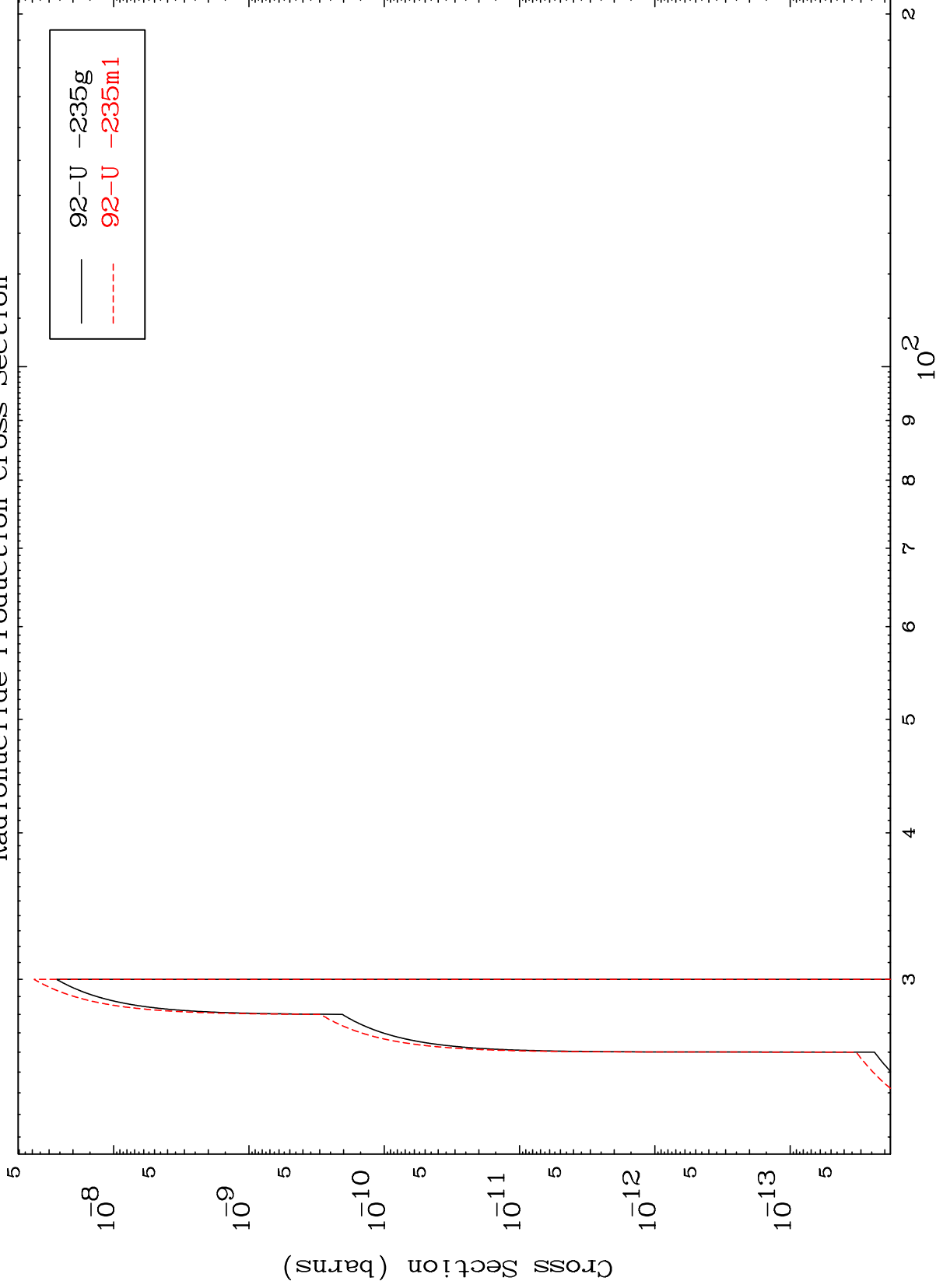
91-Pa-234

MAT 9140

(n,2n) p

91-Pa-234

Radionuclide Production Cross Section



13

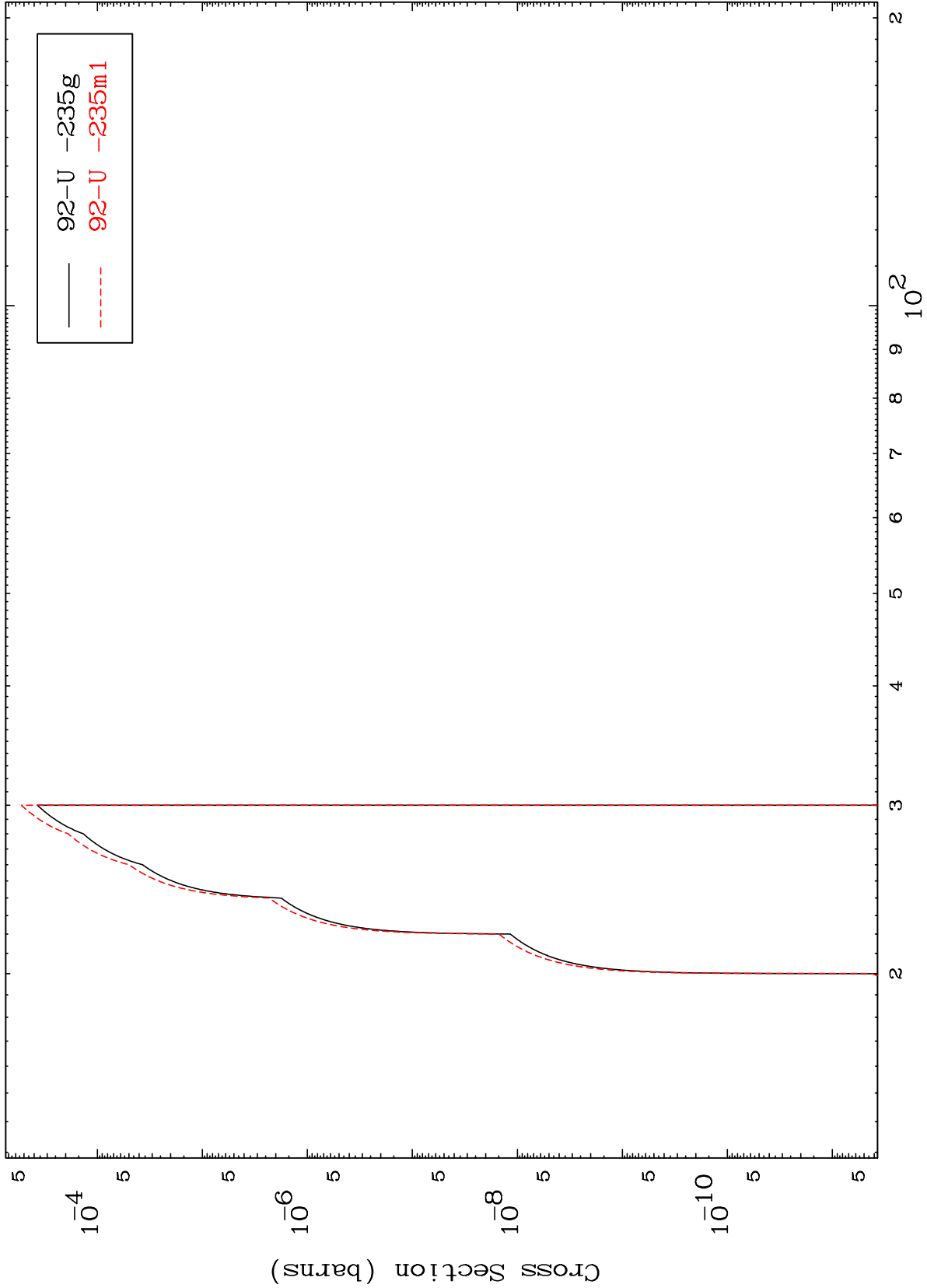
Incident Energy (MeV)

91-Pa-234

MAT 9140

91-Pa-234

(n, t)  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

91-Pa-234

MAT 9140

91-Pa-234

